Tennessee Valley Authority, Post Office Box 2000, Spring City, Tennessee 37381-2000

Richard T. Purcell Site Vice President, Watts Bar Nuclear Plant

JAN 2 9 1999

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of) Docket No. 50-390 Tennessee Valley Authority)

WATTS BAR NUCLEAR PLANT (WBN) - UNIT 1 - FACILITY OPERATING LICENSE NPF-90 - LICENSEE EVENT REPORT (LER) 50-390/1999001

The enclosed report provides details regarding the entry into Limiting Condition for Operation (LCO) 3.0.3 due to both trains of the electric board room air conditioning unit chillers being out of service for about four hours. Submittal of this report is in accordance with 10 CFR 50.73(a)(2)(i)(B).

If you should have any questions, please contact P. L. Pace at (423) 365-1824.

Sincerely,

R. T. Purcell

Enclosure cc: See page 2

9902050164 990201 PDR ADUCK 05000390 S PDR U.S. Nuclear Regulatory Commission Page 2 JAN 2 9 1999

cc (Enclosure): INPO Records Center Institute of Nuclear Power Operations 700 Galleria Parkway Atlanta, Georgia 30339-5957

NRC Resident Inspector Watts Bar Nuclear Plant 1260 Nuclear Plant Road Spring City, Tennessee 37381

Mr. Robert E. Martin, Senior Project Manager U.S. Nuclear Regulatory Commission One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

U.S. Nuclear Regulatory Commission Region II Atlanta Federal Center 61 Forsyth St., Suite 23T85 Atlanta, Georgia 30303

U.S. NU AR REGULATORY ((6-1998) LICENSEE EVENT REPORT (LER) (See reverse for required number of						OMMISS	ION	Estim collec licens estim Nucle	tion request: ing process a ate to the in ar Regulatory	50.0 hrs: Report nd fed back to ind formation and Re Commission, V	comply ted lesson dustry. Fo scords Mar Vashington	with thi s learned orward co nagement , DC 20	EXPIRES 08/30/2001 s mandatory information are incorporated into the mments regarding burden branch (T-6 F33), U.S. 0555-0001, and to the Management and Budget,					
					se for required aracters for e			t			Washi	ington, DC 20	603. In an infor	mation coll	ection do	es not display a currently sponsor, and a person is		
										·	not re	quired to resp	ond to, the inform	ation collec	tion.			
FACILITY N	AME (1)										DOCK		(2)			PAGE (3)		
Watts	Bar N	luclear	Pla	nt - I	Unit 1							-05	5000390			1 OF 8		
TITLE (4)													·					
LCO 3	3.0.3	Entry D	ue	to Bo	oth Trains of	Electri	ic Bo	ard Ro	om Cl	hillers	Out d	of Servic	е					
	T DAT				LER NUMBER				RT DAT		T		OTHER FACIL	ITIES IN	VOLVE	D (8)		
MONTH	DAY	YEAR	. YE	AR	SEQUENTIAL NUMBER	REVISI NUMB		MONTH	DAY	YEAR	FACIL	ITY NAME			Боск	ET NUMBER		
											FACIL	ITY NAME			роск	ET NUMBER		
01	02	99	19	99	00.1	00	00 02 01 99					05000						
OPERA	TING		ТН	IS REP	PORT IS SUBMIT	TTED PU	RSU	ANT TO	THE RE	QUIREM	ENTS	OF 10 CF	R§: (Check o	one or m	nore) ('	11)		
MODE	E (9)	1		20.2	201(b)			20.2203	(a)(2)(v)		X	50.73(a))(2)(i)		50	50.73(a)(2)(viii)		
POW	ER	Î Î		20.2	203(a)(1)			20.2203	(a)(3)(i)		50.73(a)(2)(ii) 50.73(a)(2)(x).73(a)(2)(x)			
LEVEL	(10)	80		20.2	203(a)(2)(i)			20.2203	(a)(3)(ii)			50.73(a))(2)(iii)		73.71			
				20.2	203(a)(2)(ii)		:	20.2203	(a)(4)		50.73(a)(2)(iv)			го	OTHER			
				20.2	203(a)(2)(iii)			50.36(c)	(1)	50.73(a)(2)(v) Specif			Specify	ecify in Abstract below				
				20.2	203(a)(2)(iv)		1	50.36(c)	(2)			50.73(a)	(2)(vii)		or in Ni	or in NRC Form 366A		
						LICEN	VSEE	CONTA	CT FOR	THIS LE	R (12))	········	<u>_</u>				
NAME											- T	TELEPHONE N	UMBER (Include A	vea Code)				
		Rebec	cca	<u>N. N</u>	lays, Senior	Licensi	ing I	Enginee	er				(423	3)-365-	3855	1		
				COM	PLETE ONE LIN	E FOR E	ACH	COMPO	NENT FA	AILURE I	DESCF	RIBED IN T	HIS REPORT (13)				
CAUSE	s	YSTEM	со	MPONE	NT MANUFACTU	IRER RE		ABLE TO IX		CAU	CAUSE SYSTEM COMPONENT MANUFACTURER		REPORTABLE TO EPIX					
В		VI		CLR	D270		١	(
																·		
			SUP	PLEM	ENTAL REPORT	EXPECT	TED (14)			Ť	EXP	ECTED	MONTH	DA	Y YEAR		
YES (If ye	s, com	plete EXF	PECT	ED SI	JBMISSION DAT	TE).			NO	x			AISSION E (15)			•		

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On January 2, 1999, at approximately 0829 EST, electric board room (EBR) air conditioning unit Train A chiller located in the Control Building tripped and automatically started Train B EBR air conditioning unit. During troubleshooting of Train A EBR air conditioning unit chiller, it was suspected initially that the EBR air conditioning unit tripped due to low water supply pressure to the temperature control valve (TCV) actuator which could increase the valve closing force and trip the chiller on high discharge pressure because of inadequate cooling. The water supply pressure to Train A EBR air conditioning unit TCV pressure was restored to normal. While Train B EBR air conditioning unit exhibited control power circuit and load control instability. At 0937 hours EST, Train B EBR air conditioning unit was stopped to restore Train A EBR air conditioning unit to service. Train A EBR air conditioning unit failed to start. Operations declared both trains inoperable and entered into LCO 3.0.3. Train A EBR air conditioning unit was returned to service at approximately 1350 and LCO 3.0.3 was exited. This condition is being reported in accordance with 10 CFR 50.73 (a)(2)(i)(B).

The cause of this event was determined to be a loss of refrigerant inventory of the Train A EBR air conditioning unit chiller as the result of a leak in a capillary tube connecting to the pressure gauge. The failure occurred due to a combination of aging and high frequency fatigue cycling at a stress riser in the tubing. Corrective actions included returning the air conditioning unit to service within a short period of time.

NRC	FORM	366A	
(4-95)			

.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) **TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET	LER NUMBER (6)	PAGE (3)
· · · · · · · · · · · · · · · · · · ·	05000	YEAR SEQUENTIAL REVISION NUMBER	2 OF 8
Watts Bar Nuclear Plant, Unit 1	05000390	1999 - 001 - 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT CONDITIONS:

Watts Bar Nuclear Plant Unit 1 was in Mode 1 operating at approximately 80 percent reactor power when this event occurred. A down power to 65% of rated thermal power was in progress to repair a minor main condenser tube leak.

DESCRIPTION OF EVENT П.

Α. Event

The Train A and B electric board room (EBR) air conditioning units supply cooling for the computer room and for the safety related Auxiliary Instrumentation Room which contains the reactor trip system (RTS) instrumentation.

On January 2, 1999, at approximately 0829 EST, EBR air conditioning unit Train A (Energy Industry Identification System (EIIS) Code CLR) located in the Control Building (CB) tripped and automatically started Train B EBR air conditioning unit. During troubleshooting of Train A EBR air conditioning unit, it was suspected initially that the EBR air conditioning unit tripped due to low water supply pressure to the temperature control valve (TCV) 0-TCV-67-1052 actuator which could increase the valve closing force and trip the EBR air conditioning unit chiller on high discharge pressure because of inadequate cooling. The water supply pressure to Train A EBR air conditioning unit TCV pressure was restored to normal.

While Train B EBR air conditioning unit was in service, the unit exhibited control power circuit and load control instability. At 0937 hours EST, Train B EBR air conditioning unit was stopped to restore Train A EBR air conditioning unit to service. Train A EBR air conditioning unit failed to start. Both trains of the EBR air conditioning units were subsequently declared inoperable. Operations personnel entered LCO 3.0.3 because there was not a technical specification which covered both trains of this support system being out of service. This action is considered to be conservative, as transient temperature analysis have shown that the abnormal maximum temperature limit of 104°F would not be reached until at least 4 hours under extreme heat conditions following a complete loss of cooling. Actions were initiated to return either of the EBR air conditioning units to operable condition. Since down powering the plant to 65% was already in progress to repair a main condenser tube leak, no further power reduction due to this condition had begun. Troubleshooting identified a refrigerant leak at a capillary tube to suction pressure gauge 0-IPI-31-502A-A. However, prior to any further power reduction, the Train A EBR air conditioning unit chiller capillary tube leak was isolated, additional refrigerant was added, and the unit was started at approximately 1329 hours. The Train A EBR air conditioning unit was declared operable at approximately 1350 hours EST and LCO 3.0.3 was exited.

Problem Evaluation Report (PER) 99-000009-000 was initiated to document this event in the TVA Corrective Action Program.

Inoperable Structures, Components, or Systems that Contributed to the Event Β.

As stated above, Train A EBR air conditioning unit would not start due to low refrigerant suction pressure and Train B air conditioning unit exhibited control power circuit and load control instability while in service.

NRC FORM	366A			U.S. NUCLEAR REGULATOR	COMMISSI
4-95)		LICENSEE EVEN TEXT CON	T REPORT (LEI TINUATION	R)	
	FACILI	TY NAME (1)	DOCKET	LER NUMBER (6)	PAGE (3)
			05000	YEAR SEQUENTIAL REVISION	OF
Watts I	Bar Nuclear Plant,	Unit 1	05000390	1999 - 001 - 00	
		use additional copies of NRC Form 366A	יין (17)	. <u></u>	<u> </u>
				А	
II. DES	CRIPTION OF EV	ENT (continued)			
· C.	Dates and Appr	oximate Times of Major Occurre	ences		
	Time (EST)	Occurre	nces on January	2, 1999	
	0829	Train A EBR air conditioning EBR air conditioning unit.	unit tripped and	automatically started Train B	
	0850	Initially suspected high chille Valve 0-TCV-67-1052 was s trip due to Train A EBR air co 85°F as the supply pressure 65 psig.	uspected as the anditioning unit i	possible cause of the chiller return temperature exceeding	
	0904	Valve was adjusted to achiev	ve 60 to 65 psig	on Train A EBR chiller.	
	0920	Train B EBR air conditioning The slide valve position was lights for low suction pressur temperature trips were flicke the Train A EBR air condition unit from service as a proble control circuit was suspected	cycling betweer re, high discharg ring. Recommen ing unit in servio m with the Train	a 80% and 0%. The trouble ge pressure, and high oil andations were made to place	
	0936	Operations placed handswitc EBR air conditioning unit and start Train A EBR air conditio	hand switch 0-	in Normal to stop Train B HS-31-30A in Pull Standby to	
	0937	the unit had tripped. Howev	low suction pres air conditioning er, the air condi og handswitch O rains of this sup	ssure light. No attempts unit as Operations suspected tioning unit was actually -HS-31-31A to Normal. LCO	
	1125	thermal expansion valves to permit chiller start. Train A l low refrigerant suction press	achieve satisfact EBR air conditior ure. A refrigera	ning unit chiller tripped on	
	1325	Completed installation of refr	igerant to Train	A EBR air conditioning unit	
	1329	Started Train A EBR air cond performance.	itioning unit chi	ller and monitored	
	1350	•	ditioning unit op	erable and exited LCO 3.0.3.	

.....

Þ,

÷.

NRC FC	DRM 36	6A		U.S. NUCLEAR REGULAT	ORY	COMMI	SSION	
(4-95)		LICENSEE EVEN	т рерорт (і ер	N				
Į		TEXT CON		y				
		FACILITY NAME (1)	DOCKET	LER NUMBER (6)		PAGE (3)	
	5		05000	YEAR SEQUENTIAL REVISION NUMBER	4	OF	8	
Wa	tts Ba	r Nuclear Plant, Unit 1	05000390	1999 - 001 - 00				
TEXT (lf more	space is required, use additional copies of NRC Form 366A	/ (17)	₩., ////// · ·///// · ·/////	<u> </u>			
I II.	DES	CRIPTION OF EVENT (continued)		,				
	D.	Other Systems or Secondary Functions Affect	ed					
		As stated above, Train B EBR air conditioning control instability while in service.	unit was exhibit	ing control power circuit an	d loa	d		
	Ε.	Method of Discovery						
		The condition was found during troubleshooting failed to start.	ng Train A EBR a	air conditioning unit after the	e uni	t .		
	F.	Operator Actions						
		Operations personnel entered LCO 3.0.3 when inoperable. Actions were initiated to return th actions to prepare for further load reduction as main condenser tube leak.	e affected equip	ment back to service concu	rrent	: with		
	G.	Automatic and manual safety system response	<u>95</u>					
		There were no automatic or manual safety sys	tem responses a	and none were required.				
111.	CAU	ISE OF EVENT						
	Trair	cause of this event was determined to be a result A EBR air conditioning unit chiller suction present ntory after 5 days of operation.						
	Con	tributing Cause:			-			
Operations and the system engineer suspected that Train B EBR air conditioning unit chiller was tripped before Train A EBR air conditioning unit chiller had attempted to start and failed. This assumption was based on Panel 1-M-9 indicating light for Train B EBR air conditioning unit changing from red to green (on to off) about the same time Train A EBR air conditioning unit handswitch was placed to Pull Standby from Normal. The handswitch alignment was performed per System Operating Instruction (SOI) 31.01, Section 5.11, "Standby Alignment of EBR HVAC." This section is misleading as it does not clearly direct taking the handswitch for the in-service EBR air conditioning unit to Start before placing in Normal. Train B EBR air conditioning the Train B EBR air conditioning unit stopped as a result of placing the Train B EBR air conditionling unit's handswitch from Pull Standby to Normal at the panel.								
				• •				
					•			

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER	(6)		PAGE (3)
·	05000	YEAR	YEAR SEQUENTIAL REVISION NUMBER		5	OF	8
Watts Bar Nuclear Plant, Unit 1	05000390		1999 - 001 -	00			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

III. CAUSE OF EVENT (continued)

Contributing Cause (continued):

There was no functional failure of Train B EBR air conditioning unit chiller, although there were several failed or deficient conditions which adversely affected indications and load controller operation. The temperature load controller load solenoid relay had failed, which caused the unload solenoid to chatter periodically during operation with the load solenoid continuously energized. The temperature load controller had been in service since 1979. The relay failure was due to aging which was aggravated by intermittent grounding at the hot gas solenoid or the unload start solenoid in combination with missing or poorly connected ground leads in the control circuit. The missing ground leads in combination with the intermittent solenoid grounds caused flickering indicating lights during operation and a dim power on light.

IV. ANALYSIS OF EVENT - ASSESSMENT OF SAFETY CONSEQUENCES

The EBR air conditioning units are designed to maintain acceptable temperatures to the Control Building Elevations 692.0 and 708.0. The only safety-related equipment on these elevations is located in the Auxiliary Instrument Room which contains the reactor trip instrumentation. None of these components were impacted by this event and were available to perform their intended function. The potential impact would have been if the air conditioning units had stayed out of service for an extended period of time and if a design basis event had occurred coincident with that condition causing the temperature in the room to approach the abnormal maximum temperature limit of 104°F. The probability of a design basis loss-ofcoolant accident (LOCA) occurring in the four hours that these units were not available is 2.97E-6. Transient temperature analysis performed for 10 CFR 50, Appendix R shutdown have confirmed that the abnormal maximum temperature limit will not be reached before 4 hours when starting at an initial room temperature of 85°F with worst case design basis heat loads. Since the total internally generated heat loads for this condition are the same as a LOCA and the effect of loads transmitted through concrete walls, floors, and ceiling is minimal during the initial phase of any transient, the existing transient analysis is considered to bound any complete loss of cooling event for the first several hours. The temperature in the Auxiliary Instrument Room at the beginning of the event was 77°F. Three hours after the event occurred the temperature had reach approximately 81°F which is well below the abnormal limit. The equipment in the room is operable in environments up to 104°F. Electrical equipment in general is not considered to be thermally stressed for any 100 day period until at least 120°F. Considering this fact, the abnormal temperature at 104°F is conservative. Therefore, considerable heatup would have to occur before any impact on the plant equipment would be manifested. In no time during the three hour and 53 minute event was there any impact or consequences to plant equipment or to the health and safety of the public.

NRC FORM (4-95)	366A
(4-95)	

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER	(6)		PAGE (3)
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	6	OF	8
Watts Bar Nuclear Plant, Unit 1	05000390		1999 - 001 -	00			
TEXT (Kenne and international and international internationa							

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

IV. CORRECTIVE ACTIONS

A. Immediate Corrective Actions

Operators initiated action to return one of the EBR air conditioning unit to service. A refrigeration leak at the suction pressure gauge sense line was identified on Train A air conditioning unit chiller. The leak was isolated and refrigerant was added. Train A EBR air conditioning unit was, subsequently, returned to service at 1350 hours on January 2, 1999.

The Train B EBR air conditioning unit chiller temperature load controller relay was replaced and the ground leads in the control circuits were repaired or replaced. Train B EBR air conditioning unit was declared operable on January 3, 1999.

B. <u>Corrective Actions to Prevent Recurrence</u>

TVA will perform an evaluation of the cause of the failure of Train A EBR air conditioning unit chiller suction pressure sense line tubing and will implement the recommended repairs.

SOI-31.01 will be revised to clarify hand switch operation to support swapping the operating and standby trains following automatic start of the standby train.

Train A EBR air conditioning unit chiller suction pressure gauge broken capillary sense line will be repaired.

Implementation of the above three corrective actions will be completed by July 16, 1999.

C. <u>Other actions Being Tracked Under the Corrective Action Program Which Are Not Considered</u> Regulatory Commitments

A previously issued PER, WBPER980175, similar to this event will be revised to include the January 2, 1999 failure of Train A EBR air conditioning unit chiller suction pressure sense line as a repetitive functional failure. The root cause analysis will be revised and additional corrective actions and monitoring requirements will be added, as necessary.

Proper installation of ground leads in the control circuits will be verified for Train A EBR air conditioning unit chiller and the similar chillers from the same manufacturer which are the Main Control Room chillers Train A and B.

TVA will inspect the EBR and Main Control Room air conditioning unit chiller solenoid valve leads for damaged insulation and potential ground paths at junction box covers and flex conduit connections. Damaged components will be repaired or replaced. Grommets, silicone sealant, or other insulating material will be installed as necessary to protect field wires at interference points.

Recommended repairs will be implemented from the failure evaluation of the Train A EBR air conditioning unit chiller suction, discharge, and oil discharge pressure capillary tubing for Train B EBR air conditioning unit chiller and the Main Control Room Train A and B chillers, if required.

NRC FORM 366A (6-1998)

l	NRC FORM (4-95)	366A
	(4-95)	

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER	(6)		PAGE (3	3)
	05000		YEAR SEQUENTIAL RE NUMBER		7	OF	8
Watts Bar Nuclear Plant, Unit 1	05000390		1999 - 001 -	00			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

VI. ADDITIONAL INFORMATION

A. Failed Components

1. Safety Train Inoperability

The cause of LER 390/1999-001 was due to a refrigerant leak at the suction pressure gauge sense line on Train A EBR air conditioning unit.

2. Component/System Failure Information

a. Method of Discovery of Each Component or System Failure:

The condition was found after a failure to start during troubleshooting.

b. Failure Mode, Mechanism, and Effect of Each Failed Component:

The cause of this event was determined to be a result of a refrigerant leak from the capillary tube to the Train A EBR air conditioning unit chiller suction pressure gauge. The failure occurred after 5 days of operation and did not occur until the chiller lost about 45% of it's refrigerant inventory. This failure mode is being addressed further within the WBN corrective action program.

There was no functional failure of Train B EBR chiller, although there were several failed or deficient conditions which adversely affected indications and load controller operation.

c. Root Cause of Failure:

The failure occurred due to a combination of aging and high frequency fatigue cycling at a stress riser in the tubing. Stress was induced by the mass of the coiled tubing below the stress riser that was subject to vibration at the chiller structural frequency. The failed tubing was supplied by the vendor as part of the chiller package and has been in service for approximately 20 years.

d. For Failed Components With Multiple Functions, List of Systems or Secondary Functions Affected:

This unit's function is to provide cooling to Control Building Elevations 692.0 and 708.0 which include the 24V, 48V, and 250V battery and associated board rooms, battery room exhaust fan room, communication room, secondary alarm station, computer room, auxiliary instrument rooms, and mechanical equipment rooms. No other systems or secondary functions were affected.

NRC FORM 366A (4-95)	<u>.</u>	U.S. NUCLEAR REGULATO	DRY COMMISSION			
LICENSEE EVEN)				
TEXT CON	FINUATION	·				
FACILITY NAME (1)	DOCKET	LER NUMBER (6)	PAGE (3)			
	05000	YEAR SEQUENTIAL REVISION NUMBER	8 OF 8			
Watts Bar Nuclear Plant, Unit 1 05000390 1999 - 001 - 00						
TEXT (If more space is required, use additional copies of NRC Form 366A)	(17)					
VI. ADDITIONAL INFORMATION (continued)						
A. Failed Components (continued)						
e. Manufacturer and Model Number o	f Each Failed Co	omponent:				
Dunham Bush Chiller, Model IPCZX	(230					
B. <u>Previous Similar Events</u>						
 A review of the previous WBN LERs reveals th LER 390/98-003 submitted September 24, 199 Air Temperature Control Systems (CREATCS) I However, the root cause of these LERs were d being an equipment failure. LER 390/1998-005 submitted December 2, 19 units being out of service for approximately 2 I to be an equipment failure as one of the units for a preventative maintenance activity. In addition, a similar failure occurred on Train I The suction pressure gauge sense line broke. EBR chiller, not after the unit had been in oper- a Level A PER, WBPER980175 in TVA's Corre preventable functional failure under the Mainte within a month. The root cause of that failure because the chillers will not start with ambient tripping on low suction pressure during the sta temperature control valve seat leakage cooling inventory was considered a contributing cause actions. Monitoring requirements are no chille minor refrigerant leaks. VII. COMMITMENTS The actions committed to be implemented in re Corrective Actions - Recurrence Control. 	98, involved bot being out of service letermined to be 998, involved the hours. The root threw a belt whi B EBR air conditi This failure was ation for several ective Action Pro- mance Rule as it was described at t or condenser te arting sequence. the condenser to arting PER remain failures due to	th trains of Control Room Em- vice for a short period of time of a procedural nature rathe e 480V board room air condi- t cause of that LER was dete- ile the other unit was out of ioning unit chiller February 1 detected during startup of 1 I days. The failure is docume ogram and was classified as a t was the third failure of this as a design analysis deficien- emperatures below 57°F with Low temperature was caus below 57°F. The loss of refr ains open to monitor correcti- temperature control valve le	nergency ne. er than litioning ermined service 13, 1998. Train B ented as a type lcy hout sed by rigerant ive eakage or			