NRĆ FO (4-95)	RM 366			U.S. I	AR REGU	LATORY CO	OMMISS	ION	ESTI			OMB NO	. 3150- 8	0104	
		LI	CENSEE I (See reverse digits/cha	EVENT RE e for required racters for e	PORT (d number ach block	of			MAN REPO LICEN COMI RECO REGU THE MAN,	DATORY INI DATORY INI DATED LESS NSING PROCE MENTS REGAI DRDS MANA JLATORY COM PAPERWORK AGEMENT AN	FORMATION CC DNS LEARNED ESS AND FED RDING BURDEN E GEMENT BRAN MMISSION, WASI REDUCTION F D BUDGET, WASI	ARE INCI BACK TO STIMATE TO ICH (T-6 HINGTON, D PROJECT ()	REQUES ORPORAT INDUSTF D THE INF F33), DC 20555 3150-010 DC 20603.	5T: 50 ED II RY. ORMA U.S. 5-0001, 4), 0).0 HRS. NTO THE FORWARD TION AND NUCLEAR , AND TO FFICE OF
FACILITY N	NAME (1)								DOCI	KET NUMBER	(2)			PAC	iE (3)
Watts	s Bar N	luclear	Plant - Uni	it 1						05	5000390		1	OF	18
TITLE (4)										<u></u>			M		
INCON	APLET	E SURV	EILLANCE		TION										Í
EVE	NT DAT	E (5)	L	ER NUMBER (6)	REP	ORT DA	re (7)	I	от	HER FACILITI	ES INVOL	VED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACIL	ITY NAME			DOCKET	05000	iER D
04	28	97	97	011	04	10		97	FACIL	ITY NAME			DOCKET	05000	iER)
OPERA	TING	1	THIS REPO	RT IS SUBMIT	TTED PUR	SUANT TO	THE RE	QUIREM	ENTS	6 OF 10 CF	R §: (Check	one or m	ore) (1	1)	
MOD	E (9)	1	20.220	1(b)		20.220)3(a)(2)(/)		X 50.73(a)(2)(i)		50.	73(a)	(2)(viii)
POW	VER		20.220	3(a)(1)		20.220)3(a)(3)(i)		50.73(a)(2)(ii)		50.	73(a)	(2)(x)
LEVEL	. (10)	<u> 100</u>	20.220	3(a)(2)(i)		20.220)3(a)(3)(i	i)		50.73(a)(2)(iii)		73.	71	
the second			20.220	3(a)(2)(iii) 3(a)(2)(iii)	· ·	50.360	03(a)(4) c)(1)			50.730	a)(2)(iv) a)(2)(v)		Specify	in Ab	stract
			20.220	3(a)(2)(iv)		50.36(c)(2)			50.73(a)(2)(vii)		BPINYNF	RC For	m 366A
			<u> </u>	<u></u>	LICENS	EE CONTA	CT FOR	THIS LE	R (12	<u>.</u>)					
NAME									1	TELEPHONE N	UMBER (Include A	Area Code)			
		F	R. A. Stoc	kton, Licen	sing En	gineer	<u> </u>				(423)-	365-18	18		
		<u> </u>	COMPLET	E ONE LINE F	OR EACH	COMPONE	NT FAIL	URE DE	SCRIE	BED IN THIS	S REPORT (1:	3)			
CAUSE	S\			MANUFACTU	RER REPC	RTABLE TO		CAUS	SE	SYSTEM	COMPONENT	MANUFAC	CTURER	REPC TO	NPRDS
В		BE	HS	W120		N									
VEC			SUPPLEMEN	TAL REPORT	EXPECTE	D (14)				EXP	ECTED	MONTH	DAY		YEAR
(If ye	es, com	olete EXP	ECTED SUB	MISSION DA	TE).		XNU			DAT	re (15)				
ABSTRA	CT (Lin	nit to 140	00 spaces, i.	e., approxima	tely 15 si	ngle-space	d typew	ritten lin	es) (1	16)		<u></u> _			
The	purpo	se of t	his LER is	to report fi	indings i	n accord	ance v	vith 10	CFF	R 50.73	associated	with G	eneric	Let	ter
(GL) alac	96-0 [°]	l, "les	ting of Sa	fety Relate	d Logic	Circuits"	' reviev	vs. GL	. 96-	-01 requi	res each lie	censee	to cor	npar	6
Gen	erator	load st	nedding ar	nd sequence	ing, and	actuatio	ne nea	tor th	e En	aineered	Safety Fe	gency L ature Δ	nesei ctuati	on	
Syst	tem (E	SFAS)	against pl	ant surveill	ance te	st proced	lures to	o ensu	e th	at all por	tions of th	e logic	circuit	try	
inclu	uding t	he para	allel logic,	interlocks,	bypass	es and ir	hibit c	ircuits	are a	adequate	ly covered	in the	survei	llanc	;e
proc	edure:	s to ful	fill the Wa	itts Bar Teo	chnical s	Specifica	tion re	quirem	ents	. The fir	st surveilla	nce def	licienc	ies	
iaen findi	inne b	INVOIVE	a unveritie an discovo	a parallel (Circuit p	aths and	were (includ.	ered	on April	28, 1997.	Some	additi	onal	
surv	reilland	e instri	uction def	iciencies h	⊲yni ∠o ave beei	anu nave n attribut	ed to i	nadeo	u in Iate	r une repo technica	rt. The Ca	ause of	ine tive a	ction	ne
con	sist of	comple	eting the r	eviews, ad	dressing	y verifica	tion of	any u	nveri	fied loaid	circuits. i	nformin	a tech	nnica	al
revie	ewers	of the	requireme	nts of GL 9	96-01, a	nd corre	cting a	ny har	dwa	re deficie	ncies foun	d.	0.000		

.

ï

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER	(6)		PAGE (3)
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	2	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. PLANT CONDITIONS:

Watts Bar Nuclear Plant Unit 1 has been operating in various Mode conditions during the course of GL 96-01 review.

II. DESCRIPTION OF EVENT

A. <u>Event</u>

The purpose of this LER is to report findings in accordance with 10 CFR 50.73 associated with Generic Letter (GL) 96-01, "Testing of Safety Related Logic Circuits" reviews. GL 96-01 requires each licensee to compare electrical schematic drawings and logic diagrams for the Reactor Protection System (Energy Industry Identification System (EIIS) code JC/JG), Emergency Diesel Generator (EIIS code EK) load shedding and sequencing, and actuation logic for the Engineered Safety Feature Actuation System (ESFAS) (EIIS code JE) against plant surveillance test procedures to ensure that all portions of the logic circuitry including the parallel logic, interlocks, bypasses and inhibit circuits are adequately covered in the surveillance procedures to fulfill the Technical Specification (TS) requirements. It was established to address industry problems with testing of safety related logic circuits. TVA's letter to NRC dated April 18, 1996, indicated that WBN GL 96-01 reviews would be completed by startup after the first refueling outage currently scheduled to begin in September 1997. As a result of ongoing reviews, the first reportable GL 96-01 findings were identified on April 28, 1997. The findings have been listed in Section II.C by the date of discovery.

B. Inoperable Structures, Components, or Systems that Contributed to the Event

None

C. Dates of Discovery and Reportable Findings

Each finding is listed in a table by the date of discovery.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER (6)				3)
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	3	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

II. DESCRIPTION OF EVENT (continued)

C. Dates of Discovery and Reportable Finding

Item	Date of	Incomplete	
	Discovery	Procedures	Test Deficiency
1	4/28/97	1-SI-99-5	 The surveillance instruction did not verify that the manual handswitches for Phase A Containment Isolation (EIIS code JM), Containment Vent Isolation (EIIS code JM), and Containment Spray (EIIS code BE) functioned properly. The trip actuating device operational test (TADOT) to satisfy SR 3.3.2.8 and SR 3.3.6.6 was incomplete. Specifically, the procedure did not include: (1) Verification that 1-HS-30-63A successfully initiates Containment Isolation Phase A and Containment Vent Isolation to Train A and Train B of the Solid State Protection System (SSPS) (EIIS code IC/1G)
	-		 (2) Verification that 1-HS-30-63B successfully initiates Containment Isolation Phase A and Containment Vent Isolation to Train A and Train B of SSPS. (For items 1 and 2 above, simultaneous action of handswitches as performed in the surveillance instruction did not verify operability of each switch because Containment Isolation Phase A and Containment Vent Isolation manual signals occur through parallel logic circuit paths.)
r			 (3) Verification that 1-HS-30-64A in combination with 1-HS-30-64B successfully initiates a Containment Spray signal to Train A and Train B of SSPS.
			 (4) Verification that 1-HS-30-68A in combination with 1-HS-30-68B successfully initiates a Containment Spray signal to Train A and Train B of SSPS.
			(For items 3 and 4 above, Containment Spray initiation as performed in the surveillance instruction did not verify operability of each switch because Containment Spray and Containment Isolation Phase B signals occur through parallel circuit paths.
			(continued)

NRC FORM 366A			U.S. NUCLEAR	REGULAT	ORY (COMMI	SSION
(4-95) I.I.CENSEE EVENI	REPORT (LE	R)					
TEXT CON		/					
	DOCKET			(6)		PAGE (3)
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	4	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97 -	011	04		<u></u>	
TEXT (If more space is required, use additional copies of NRC Form 366A) (17)						
II. DESCRIPTION OF EVENT (continued)							
	1						
Item 1 (continued)							
Applicable LCOs LCO 3.3.2 The ESFAS instrumentation for eac Table 3.3.2-1, "Engineered Safety operable.	h function in To Feature Actuati	echnic on Ins	al Specificati trumentation	ion (TS) ," shall b	e		·
LCO 3.3.6 The containment vent isolation inst 3.3.6-1, "Containment Vent Isolati	trumentation for on Instrumentat	r each tion," :	function in T shall be oper	FS Table able.			
A continuity problem was encountered during the to into LCO 3.0.3 was required briefly because the m service within 24 hours after discovery (SR 3.0.3) beyond the 24 hours was encountered during rest HS-30-68A was replaced. All four listed handswith operable status.	esting of 1-HS-3 anual handswitc of the missed su oration of the sw ch functions were	0-68A h circu rveillar itch op e verifi	(EIIS code H its were not r nce. A 20 mir erability. Har ed and return	S). Entry estored to nute delay ndswitch 1 ed to)		
						ł	
Item Date of Incomplete			<u> </u>				

	Discovery	Procedures	Test Deficiency
2	5/1/97	1-SI-92-41 1-SI-92-42 1-SI-92-43 1-SI-92-44 1-SI-92-141 1-SI-92-142 1-SI-92-143 1-SI-92-144	There was no verification in surveillance instructions that indicated the Power Range Protection (P-10) interlock (EIIS code IEL) was in the required state for existing unit conditions. Thus, the channel operability test (COT) to satisfy SR 3.3.1.7 was incomplete.

Applicable LCO

LCO 3.3.1 RTS instrumentation for each function in TS Table 3.3.1-1, "Reactor Trip System Instrumentation," shall be operable.

Plant Engineering Data System (PEDS) (EIIS code ID) archive data indicated that the P-10 interlock (EIIS code IEL) was in the required state for existing unit conditions at the time of the channel operational test (COT) for SR 3.3.1.7.

1307 LICENSEE EVENT REPORT (LER) TEXT CONTINUATION TEXT CONTINUATION FACILITY NAME (1) DOCKET LER NUMBER (6) PAGE Varts Bar Nuclear Plant, Unit 1 DOCKET LICENSEE EVENT REPORT (LER) Varts Bar Nuclear Plant, Unit 1 DOCKET LER NUMBER (6) PAGE Watts Bar Nuclear Plant, Unit 1 ODO000090 97 - 011 - 04 PAGE Watts Bar Nuclear Plant, Unit 1 O5000390 97 - 011 - 04 Page EXT (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Incomplete Procedures Test Deficiency 3 5/1/97 1-SI-92-131 1-SI-92-132 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.		6A					U.S. NUCLEA	R REGULAT	ORY	COMMI	SSIO
TEXT CONTINUATION FACILITY NAME (1) DOCKET LER NUMBER (6) PAGE 05000 YEAR SEQUENTIAL NUMBER REVISION SOO 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 - 04 5 OF EXT (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Test Deficiency 3 5/1/97 1-SI-92-131 1-SI-92-132 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.	1-95)		LICE	NSEE EVEN	r report (li	ER)					
FACILITY NAME (1) DOCKET LER NUMBER (6) PAGE 05000 YEAR SEQUENTIAL NUMBER REVISION 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 5 OF EXT (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Procedures Test Deficiency 3 5/1/97 1-SI-92-131 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.				TEXT CON	TINUATION	5					
Watts Bar Nuclear Plant, Unit 1 05000 YEAR SEQUENTIAL NUMBER REVISION 5 OF Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 5 0F EXT (If more space is required, use additional copies of NRC Form 366A) (17) 1 05000390 97 011 - 04 04 II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Incomplete Procedures Test Deficiency - <		FACILI	TY NAME (1)		DOCKET		LER NUMBER	(6)		PAGE	3)
Watts Bar Nuclear Plant, Unit 1 05000390 97 011 04 EXT (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Incomplete Procedures Test Deficiency 3 5/1/97 1-SI-92-131 1-SI-92-132 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.			<u> </u>		05000	YEAR	SEQUENTIAL NUMBER	REVISION	5	OF	18
EXT (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Incomplete Procedures 3 5/1/97 1-SI-92-131 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.	Watts Bar	Nuclear Plant,	Unit 1		05000390	97 -	- 011	04			
II. DESCRIPTION OF EVENT (continued) Item Date of Incomplete Test Deficiency 3 5/1/97 1-SI-92-131 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.	XT (If more :	space is required, us	e additional copies of NI	RC Form 366A)	(17)						
II. DESCRIPTION OF EVENT (continued) Item Date of Discovery Incomplete Procedures Test Deficiency 3 5/1/97 1-SI-92-131 1-SI-92-132 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.								,			
Item Date of Discovery Incomplete Procedures Test Deficiency 3 5/1/97 1-SI-92-131 1-SI-92-132 There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.		CRIPTION OF F	VENT (continued)								
ItemDate of DiscoveryIncomplete ProceduresTest Deficiency35/1/971-SI-92-131 1-SI-92-132There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.											
DiscoveryProceduresTest Deficiency35/1/971-SI-92-131 1-SI-92-132There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.	Iten	n Date of	Incomplete		<u> </u>						
35/1/971-SI-92-131 1-SI-92-132There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.		Discovery	Procedures	Test Def	iciency						
35/1/971-SI-92-131 1-SI-92-132There was no verification in surveillance instructions to indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.											
1-SI-92-132 indicate the Intermediate Range Neutron Flux (P-6) interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.	3	5/1/97	1-SI-92-131	There wa	as no verification	in surv	eillance instru	uctions to			
interlock (EIIS code IEL) was in the required state for existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.			1-SI-92-132	indicate	the Intermediate	Range	Neutron Flux	(P-6)			
existing unit conditions during performance of the COT. Thus, the COT to satisfy SR 3.3.1.8 was incomplete.				interlock	(EIIS code IEL)	was in	the required	state for		Ì	
Thus, the COT to satisfy SR 3.3.1.8 was incomplete.]			· · · · · · · · · · · · · · · · · · ·		_				
				existing u	unit conditions de	uring pe	erformance of	t the COT.	•		
				existing u Thus, the	unit conditions due to satisfy	uring pe SR 3.3	erformance of .1.8 was inco	mplete.	•		
				existing t Thus, the	unit conditions de e COT to satisfy	uring pe SR 3.3	erformance of .1.8 was inco	mplete.			
Applicable LCO (Applicable to items 3, 4, and 5 respectively)		Applicable LCO	(Applicable to iten	existing to Thus, the	unit conditions de e COT to satisfy 5 respectively)	uring pe SR 3.3	erformance o .1.8 was inco	mplete.			
<u>Applicable LCO</u> (Applicable to items 3, 4, and 5 respectively) I CO 3.3.1 BTS (EIIS code JC/JG) instrumentation for each function in TS Table 3.3.1-1,		Applicable LCO	(Applicable to iten	existing to Thus, the ns 3, 4, and to instrumenta	unit conditions du e COT to satisfy 5 respectively) tion for each fu	uring pe SR 3.3	erformance of .1.8 was inco 	3.3.1-1,			
<u>Applicable LCO</u> (Applicable to items 3, 4, and 5 respectively) LCO 3.3.1 RTS (EIIS code JC/JG) instrumentation for each function in TS Table 3.3.1-1, "Reactor Trip System Instrumentation," shall be operable.		Applicable LCO LCO 3.3.1 RTS "Beactor Trip S	(Applicable to iten S (EIIS code JC/JG)	existing t Thus, the ns 3, 4, and 4 instrumenta tion," shall be	unit conditions du e COT to satisfy 5 respectively) tion for each fu e operable.	uring pe SR 3.3 	in TS Table	3.3.1-1,			
<u>Applicable LCO</u> (Applicable to items 3, 4, and 5 respectively) LCO 3.3.1 RTS (EIIS code JC/JG) instrumentation for each function in TS Table 3.3.1-1, "Reactor Trip System Instrumentation," shall be operable.		<u>Applicable LCO</u> LCO 3.3.1 RTS "Reactor Trip S	(Applicable to iten S (EIIS code JC/JG) System Instrumenta	existing the transformed existing the transfor	unit conditions du e COT to satisfy 5 respectively) tion for each fu e operable.	uring pe SR 3.3 nction	in TS Table	3.3.1-1,			
Applicable LCO (Applicable to items 3, 4, and 5 respectively) LCO 3.3.1 RTS (EIIS code JC/JG) instrumentation for each function in TS Table 3.3.1-1, "Reactor Trip System Instrumentation," shall be operable. PEDS (EIIS code ID) computer point archive data indicated the P-6 interlock (EIIS code IEL) was in		Applicable LCO LCO 3.3.1 RTS "Reactor Trip S PEDS (EIIS cod	(Applicable to iten S (EIIS code JC/JG) System Instrumenta	existing the transformed existing the transfor	unit conditions du cOT to satisfy 5 respectively) tion for each fu e operable.	uring pe SR 3.3 nction 6 interl	in TS Table	3.3.1-1, de IEL) wa	s in		

[It	em Date of Discovery	Incomplete Procedures	Test Deficiency
4	5/1/97	1-SI-92-31 1-SI-92-32 1-SI-92-131 1-SI-92-132	There was insufficient testing in surveillance instructions to completely demonstrate Intermediate Range Neutron Flux, and Source Range Neutron Flux in the COT and channel calibration respectively to satisfy SR 3.3.1.8 and 3.3.1.11. Surveillance instructions did not completely verify Source Range Channel I high flux reactor trip, Source Range Channel II high flux reactor trip, Intermediate Range Channel I P-6 interlock, and Intermediate Range Channel II P-6 interlock. Thus, it was not conclusively demonstrated that Train B of the SSPS input relays (EIIS code RLY) for Intermediate Range Neutron Flux, and Source Range Neutron Flux were verified as required by SR 3.3.1.8 and 3.3.1.11.

PEDS (EIIS code ID) computer point archive data from the previous COT performance indicated the correct state change for Source Range, Intermediate Range, and P-6 outputs. Emergency Response Facility Data System (ERFDS) (EIIS code ID) data from April 24, 1997, during the performance of 1-SI-92-131 demonstrated that the SSPS Train B input relay operated.

IRC FORM 36	6A					U.S. NUCLEAR	REGULAT	ORY (COMMI	SSIO
4-95)		LICE	NSEE EVENI	REPORT (LE	ER)					
			TEXT CON	TINUATION						
	FACILI	TY NAME (1)		DOCKET		LER NUMBER	(6)		PAGE (3)
		<u></u>		05000	YEAR	SEQUENTIAL NUMBER	REVISION	6	OF	18
Watts Bar	Nuclear Plant,	Unit 1		05000390	97 -	- 011	04			
Item	Date of	Incomplete Procedures	Test Defi	ciency		•				
5	5/1/97	1-SI-47-28 1-SI-47-30 1-SI-47-32 1-SI-47-34 1-SI-47-73 1-SI-47-74 1-SI-47-75	There wa Trip Acta instructio (EIIS cod Closure (verificati methodo conclusiv	as insufficient t uating Device O ons for Turbine le IT/TA), and T (EIIS code IT/TA on to satisfy SF logy used in the vely demonstrat	esting peratic Trip Lo furbine A) to co R 3.3.1 ese ins te verif	in the Chann onal Test sur- ow Fluid Oil F Trip Turbine ompletely der .10 and 3.3. tructions did ication of the	el Calibra veillance Pressure Stop Va monstrate .1.14. Th not e subject	ition lve ne te inpu	/ est its	·

Archived computer data from PEDS (EIIS code ID) indicates that during the last performance of the subject instructions the plant process computer received the proper signals, thus indicating that Train B SSPS received the proper signals.

Item	Date of Discovery	Incomplete Procedures	Test Deficiency
6	5-9-97	1-SI-99-300-A 1-SI-99-300-B	 The Volume Control Tank (VCT) (EIIS code CB/TK) to Refueling Water Storage Tank (RWST) (EIIS code CA/TK) swapover of the Centrifugal Charging Pump (CCP) (EIIS code CB/P) suction was not conclusively determined to occur via the safety related interlock. There was insufficient testing in surveillance instructions to verify that 1-LCV-62-135-A (EIIS code LCV) provided the close interlock signal to 1-LCV-62-132-A. There was insufficient testing in surveillance instructions to verify that 1-LCV-62-136-B provided the close interlock signal to 1-LCV-62-133-B. Thus, it was not conclusively demonstrated that the VCT to RWST swapover of the CCP suction was completely tested to satisfy SR 3.3.2.7. (continued)

RC FO	RM 366	5A					U.S. NUCLEAR	R REGULAT	ORY	COMMI	SS
-95)			LICENSE T	EE EVENT	REPORT (LE	ER)					
		FACILITY	(NAME (1)		DOCKET		LER NUMBER	(6)		PAGE	3)
					05000	YEAR	SEQUENTIAL NUMBER	REVISION	7	OF	
FORM 366A U.S. NUCLEAR RECL 9' LICENSEE EVENT REPORT (LER) TEXT CONTINUATION FACILITY NAME (1) OCCKET LER NUMBER (8) OS000 VARts Bar Nuclear Plant, Unit 1 OS00030 97 - 0.11 - 0. 1' (If more space is required, use additional copies of NRC Form 366A) (17) II. DESCRIPTION OF EVENT (continued) Item 6 (continued) Applicable LCO LCO 3.3.2 - ESFAS (EIIS code JE) for each function in TS Table 3.3.2-1 shall be opera 1-SL-99-603-A response time test of slave relay K603A was reviewed and it was determined 1-LCV-62-135-A provided the closed interlock for 1-LCV-62-132-A. PEDS data for the March 6, 1997 inadvertent safety injection was reviewed and it was determined that 1-LCV-62-136-B. Initiated the closing of 1-LCV-62-133-B. Since acceptable documentation// has been extracted from 1-SI-99-603-A and also from the March 6, 1997 inadvertent Train E safety injection, the VCT to RWST swapover is considered to be satisfactorily tested. Item Date of Discovery Incomplete Procedures Test Deficiency 7 6/26/97 1-SI-211-3-A 1-SI-211-3-A Verification in situations where the 6 9 KV shutdow verification in situations where the 6 9 KV shutdow verification in situation subsystems is inopera (EIIS code BKR). The length of time that the alter feeder breaker could have been used in the plant i indeterminate. Therefore, LCO 3.8.9. Action A 1 (restore the AC electrical power distribution subsystems is inopera potentially been exceeded. Applicable LCO LCO 3.8.9 - Train A and Train B AC, four channels of vi	04										
хт <i>(If</i> II.	DES	pace is required, use	additional copies of NRC Fo	orm 366A) (17)]	
		tem 6 (continue	d)								
		Applicable LCO LCO 3.3.2 - ESF	AS (EIIS code JE) for	each fund	tion in TS Tabl	e 3.3.2	2-1 shall be o	operable.			
		1-LCV-62-135-A p March 6, 1997 ina 1-LCV-62-136-B i has been extracte safety injection, th	provided the closed int advertent safety injection nitiated the closing of ed from 1-SI-99-603-A ne VCT to RWST swap	erlock for on was rev 1-LCV-62- and also fr pover is co	I-LCV-62-132-A. iewed and it was 133-B. Since ac rom the March 6, nsidered to be sa	PEDS deterr ceptab , 1997 atisfact	S data for the nined that le documenta nadvertent T orily tested.	ition/data rain B			
	Item	Date of Discovery	Incomplete Procedures	Test Defi	ciency						
•	7	6/26/97	1-SI-211-3-A 1-SI-211-3-B 2-SI-211-3-A 2-SI-211-3-B	Test insu voltage a verificatio (EIIS cod feeder br indeterm restore th operable electrical potential	fficiencies have nd loss of voltag on in situations w le EB) is fed from le BKR). The ler eaker could have inate. Therefore he AC electrical p status within 8 h power distribution y been exceede	been ic le relay where th n its alt ngth of e been b, LCO power of hours w on subs d.	lentified with (EIIS code R ne 6.9 KV shu ernate feeder time that the used in the p 3.8.9, Action distribution su then one or m systems is ind	degraded LY-27) lo tdown bo breaker alternate lant is A.1 (to bbsystem hore AC operable)	igic ard to has		
		Applicable LCO LCO 3.8.9 - Trai vital bus electric When a 6.9 KV sl degraded voltage and 0-SI-82-3, 0- A night order/cau	n A and Train B AC, al power distribution hutdown board is fed f functions have been s SI-82-4, 0-SI-82-5, and tion order has been iss	four chan subsyster rom its nor satisfactori d 0-SI-82-6 sued to en	nels of vital DC ns shall be oper mal feeder breal ly tested by 1-SI 3. ter LCO 3.8.9, C	, and f rable. ker, the -211-3 onditio	our channels loss of volta -A, -B, 2-SI-2 n A, if a 6.9 K	ge and 11-3-A, -E V shutdoo	3, wn		

~

.

NRC FO	RM 366	jA A					U.S. NUCLEA	R REGULAT	ORY	сомм	ISSION
(4-95)			LICENS	EE EVENI EXT CONT	REPORT (LE FINUATION	ER)					
		FACILI	TY NAME (1)		DOCKET		LER NUMBER	(6)		PAGE	(3)
					05000	YEAR	SEQUENTIAL NUMBER	REVISION	8	OF	18
Watt	ts Bar	Nuclear Plant, l	Únit 1		05000390	97	011	04			
II.	DESC	Dace is required, us	e additional copies of NRC F VENT (continued)	Form 366A) (17)					1	
		Discovery	Procedures	Test Defic	ciency						
	8	7/22/97 (9/19/97 for the PASF Supply Fan C1 condition)	0-SI-30-7-A 0-SI-30-7-B 1-SI-99-300-A 1-SI-99-300-B 1-SI-99-306-A 1-SI-99-306-B	Test insut instruction breaker tr 1A, 1B, 2, Post Acci In addition of the stat Fan 2B-B deficiencie Requirem	fficiencies identif ns prevented the rips for Auxiliary A, 2B, Fuel Hand dent Sampling F n, these deficien rting of Penetrati in the test sequ es, Technical Sp ent 3.3.2.5 was	fied in t e verific Buildin dling E: acility cies als ion Roc ence. pecifica not cor	he listed surv ation of shun g General Ex khaust Fans / (PASF) Supp so prevented om Elevation Due to these tion Surveilla npletely satis	reillance t trip circu haust Far A and B, a ly Fan C1 verificatio 737 Coole test nce fied.	it and n er		
	L J S P	pplicable LCO CO 3.3.2 - The .3.2-1 shall be Ipon discovery ourveillance and erformed.	e ESFAS instrumentati operable. of this condition, Operat the above described cir	ion for eac tions perso rcuit verifica	h Function in T nnel entered TS ations were subs	echnic SR 3.0 sequen	al Specificati).3 for a missi tly satisfactor	ion Table ed ily			
		·									

.

Item

9

Date of

8/14/97

Discovery

Incomplete

Procedures

0-SI-82-3

0-SI-82-4

0-SI-82-5

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE	EVENT	REPORT	(LER)
	TT A TT I T Y		

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)		
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	9	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97 -	011 -	04			
TEXT (If more space is required, use additional copies of NRC Form 366A) (17)						
IL DESCRIPTION OF EVENT (continued)							

Test Deficiency

load shed tested:

Vent Board 1A2-A)

Vent Board 1A-A)

Vent Board 2A-A)

Vent Board 2A2-A)

Vent Board 1B-B)

Vent Board 2B-B)

Board 2B2-B)

Board 1B2-B)

A review determined that: 1) for the 480V Shutdown boards 1A1-

A, 1A2-A, 1B1-B, and 1B2-B the current limiting reactor bypass

breaker 52T has not been tested to close when a loss of offsite power load shed occurs, and 2) the following loads had not been

480V Shutdown Board 1A2-A, compartment 10D (Alternate Feeder to Reactor

480V Shutdown Board 2A2-A, compartment 10A (Alternate Feeder to Reactor

480V Shutdown Board 1B2-B, compartment 9A (Alternate Feeder to C&A Vent

480V Shutdown Board 1B1-B, compartment 10D (Alternate Feeder to Reactor

480V Shutdown Board 2B2-B, compartment 9A (Alternate Feeder to Reactor

480V Shutdown Board 2B1-B, compartment 9A (Alternate Feeder to C&A Vent

480V Shutdown Board 2A1-A, compartment 10D (Alternate Feeder to C&A

480V C&A Vent Board 1A1-A, compartment 13C (1-MTR-65-77) 480V C&A Vent Board 1B1-B, compartment 13C (1-MTR-65-74) 480V Reactor MOV Board 1A1-A, compartment 17E (power outlets) 480V Reactor MOV Board 1A1-A, compartment 18F2 (power outlets) 480V Reactor MOV Board 2A1-A, compartment 16F1 (0-CHGR-252-1) 480V Reactor MOV Board 2A1-A, compartment 18F1 (0-CHGR-253-A) 480V Reactor MOV Board 1B1-B, compartment 16E (power outlets) 480V Reactor MOV Board 1B1-B, compartment 17E (power outlets) 480V Reactor MOV Board 1B1-B, compartment 17E (power outlets) 480V Shutdown Board 1A1-A, compartment 17A (Spent Fuel Pit Pump C-S) 480V Shutdown Board 1A1-A, compartment 10D (Alternate Feeder to C&A

Applicable LCO	

LCO 3.8.1 - The following AC electrical sources shall be operable:

- a. Two qualified circuits between the offsite transmission network and the onsite Class 1E AC Electrical Power Distribution System; and
- b. Four diesel generators (DGs) capable of supplying the onsite Class 1E AC Electrical Power Distribution System.

Upon discovery of this condition, Operations personnel entered TS SR 3.0.3 for missed surveillances (3.8.1.11 and 3.8.1.19) and the above described circuit verifications were subsequently satisfactorily performed within the 24 hour time limit or the appropriate LCOs were entered if the alternate board feeders were needed.

95)	RM 366A						U.S. NUCLEA	AR REGULAT	ORY C	OMMI	SSIO			
			LICENS	EE EVENI	r Report (Li	ER)								
			Т	EXT CON	TINUATION									
	,	FACILITY	NAME (1)		DOCKET			2 (6)			31			
					05000	YEAR	SEQUENTIAL	REVISION			<u>.</u>			
					05000	 l	NUMBER		10	OF	1			
Watt	s Bar Nuc	lear Plant. Uni	it 1		05000390	97	011	- 04						
XT (//	more space	is required, use a	dditional copies of NRC F	orm 366A) (17)	<u> </u>			L					
н	DESCRIP													
11.	DESCIVIE													
	Item D	ate of	Incomplete	TID	- .									
	D	iscovery	Procedures		riciency									
	10	8/28/97	0-SI-82-3	A review	determined tha	t the sta	art lock out f	eature for						
			0-SI-82-4	black ou	t relays BOX an	d BOY	has not bee	n						
			0-SI-82-5	demonst	trated for the foll	owing p	pieces of equ	uipment:	<u>~</u>					
			0-31-02-0	Pressuri	zer Heater Cont	rol Gro	ups IA-A, In Jo 1D. Conti	o-o, and in ol & Servid	u, ce					
		· .		Air Com	Air Compressors A and B, Auxiliary Building General									
				Supply F	Supply Fans 1A, 2A, 1B, and 2B, Auxiliary Building									
				General Exhaust	Exhaust Fans 1 Fans A and B	A, 2A, 1	1B, and 2B,	Fuel Handi	ing					
				Exhluge	rans rrand b.									
		.		•										
	<u>Appli</u>	cable LCO												
		3.8.1 - The To	wo qualified circuits	cal sources between	snall be operal the offsite tran	ole: smissio	n network	and the						
		01	nsite Class 1E AC E	lectrical Po	ower Distributio	n Syst	em; and							
				(5.0.)										
		b. F	our diesel generator	rs (DGs) ca tribution S	apable of supply	ying the	e onsite Cla	ss 1E AC						
		L			ystem.									
	Upor	discovery of t	his condition, Operat	tions perso	nnel entered TS	SR 3.0	.3 for misse	d						
	subs	enuently satisf	.11 and 3.8.1.19) and actorily performed with the second sec	d the above	e described circu	uit verifi	cations were	9						
	3003	equently satisf	actomy performed w		nour ame imit.									
			······································	<u></u>		<u> </u>								
		· · ·	· · ·											

ľ

ļ

.

NRC FORM 366A (4-95)						U.S. NUCLEAR	R REGULAT	ORY (OMMI	SSION
		LICENS	EE EVENI	REPORT (LE	ER)					
					1			1		
· .	FACILITY	(NAME (1)		DOCKET		LER NUMBER	(6)	PAGE (3)		
	4			05000	YEAR	SEQUENTIAL NUMBER	REVISION	11	OF	18
Watts Bar Nuclear Plant, Unit 1		05000390	97	011	04					
TEXT (If more space	is required, use	additional copies of NRC	Form 366A) (17)						
II. DESCRI	PTION OF EV	ENT (continued)								
Item D	ate of	Incomplete					<u> </u>			
		Procedures	Test De	ficiency						
14	0/22/07	1 SL 00 200 A	Arouiou	dotorminod the	++ 40			_		

9/23/97	1-SI-99-300-A 1-SI-99-300-B 0-SI-82-3 0-SI-82-4 1-TRI-0-3 1-TRI-0-4	A review determined that the 480 VAC loads listed below have not been verified to shed when a safety injection occurs. This load shed by safety injection is actuated by slave relays K609A and K609A via the Thermal Overload (TOL) By pass relays K1 and K9. Slave Relay K609A via TOL bypass relays K1 and K9 on 480V Reactor MOV Board 1A1-A 480V Reactor MOV Board 1A1-A compartment 16B2, 1-MTR-31-303B 480V Reactor MOV Board 1A1-A compartment 17R, Power Outlets 480V Reactor MOV Board 1A1-A compartment 18F2, Power Outlets Slave Relay K609B via TOL Bypass relays K1 and K9 on 480 V Reactor MOV Board 1B1-B 480 V Reactor MOV Board 1B1-B
		480 V Reactor MOV Board 1B1-B compartment 17F2 1-MTR-31-324B 480 V Reactor MOV Board 1B1-B compartment 16E, Power Outlets 480 V Reactor MOV Board 1B1-B compartment 17E, Power Outlets

Applicable LCO

LCO 3.3.2 - The ESFAS Instrumentation for each function in Table 3.3.2-1 shall be Operable:

Upon discovery of this condition, Operations personnel took action to track completion of the required testing prior to entering Mode 4 from the refueling outage. The testing was subsequently completed.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		PAGE (3)				
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	12	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

II. DESCRIPTION OF EVENT (continued)

Item D	ate of iscovery	Incomplete Procedures	Test Deficiency
12	9/24/97	0-SI-82-3 0-SI-82-4	ESF Pumps are started from accident signals by slave relays in the Solid State Protection System. When the ESF pump starts an ESF pump circuit breaker 52STA contact closes in the ESF pump room cooler start circuit to start the cooler. A review identified that the surveillance instructions that perform slave relay testing verify the ESF Pumps start but do not verify the ESF pump room coolers start.

Applicable LCO

LCO 3.3.2 - The ESFAS Instrumentation for each function in Table 3.3.2-1 shall be Operable:

Upon discovery of this condition, Operations personnel took action to track completion of the required testing prior to entering Mode 4 from the refueling outage. The testing was subsequently completed.

FORM 3664						U.S. NUCLEAF	REGULAT	ORY (COMMI	SSIO
5)		LICENS	ee eveni	REPORT (LI	ER)					
<u></u>			TEXT CON		1			I		
•	FACILITY	NAME (1)		DOCKET	YEAR	LER NUMBER	(6) REVISION		PAGE (3)
				05000		NUMBER		13	OF	18
/atts Bar N	Nuclear Plant, Ur	nit 1		05000390	97	011	04			
T (If more sp	ace is required, use a	additional copies of NRC I	Form 366A) (17)						
II. DESC	RIPTION OF EV	ENT (continued)								
Item	Date of	Incomplete		<u> </u>						
	Discovery	Procedures	Test De	ficiency		<u> </u>	<u> </u>			
13	9/26/97	1-SI-99-307-A 1-SI-99-307-B	A review have no swap th 1-FCV-7	v identified that t t verified slave r e ECCS pumps 73-72-A and 1-F	he liste elay K6 to the c CV-63-	d surveillance 47 contact op ontainment s 73-B must be	e instructio peration. ump, valv opened.	tions . To alves		
open these valves two slave relays K647 and K648 must be actuated. Review of the surveillance indicated that contact operation was verified for the K648 relay but no verification was performed for the K647 relay.										
U re n	Ipon discovery of equired testing pr nonth surveillance	this condition, Operation to entering Mode ior to entering Mode e instructions as part	ations perso 4 from the of the refue	onnel took actior refueling outage eling outage veri	n to trac The r fied pro	k completion ecently perfor per contact o	of the rmed 18 peration.			
D.	Other Systems o	r Secondary Function	ns Affected	ffected.					J	
E	Method of Discov	, verv								
<u> </u>	GL 96-01 review									
F.	Operator Actions	, ,								
·	Entry into applica	able TS actions upon	notification	as applicable.						
G.	Automatic and m	anual safety system	responses							
	No automatic or	manual safety system	m response	s have been ass	sociated	l with the sub	iect LER			
		inanial calley bysici					,			

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET	LER NUMBER (6)				PAGE (3)				
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	14	OF	18			
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04						

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

III. CAUSE OF EVENT

The cause of this event has been attributed to inadequate technical reviews similar in nature to the GL 96-01 issues.

IV. ANALYSIS OF EVENT - ASSESSMENT OF SAFETY CONSEQUENCES

There was no safety significance for the circuits identified in the subject LER that were successfully tested or verified and documented as being operable by alternate means.

1-HS-30-68A (EIIS code HS)

There was no decrease in nuclear safety associated with the inoperability of handswitch 1-HS-30-68A. The continuity problem encountered during testing indicated that the circuit involving 1-HS-30-68A in combination with 1-HS-30-68B required to successfully initiate a Containment Spray signal to Train A and Train B of SSPS was not functional. However, manual initiation is not credited in any accident events analyzed in the Safety Analysis Report (SAR). Therefore, the consequence of failure of 1-HS-30-68A does not represent a decrease in nuclear safety.

Redundant manual initiation switches are also provided in the control room (1-HS-30-64A and (1-HS-30-64B). In addition, capability for manual initiation of containment spray is provided at the system level via control room operation of the containment spray header isolation valves and containment spray pumps.

Item 7

This issue is of limited safety significance because the design and construction of the alternate feeder breaker circuits are identical to the normal feeder breaker circuits. Thus, there is no reason to suspect that they would not function properly. Also, LCO 3.8.1, Condition A, places limitations on the amount of time that one offsite circuit can be inoperable, thereby reducing the time of operation on the alternate fed.

The surveillance test insufficiencies in Item 7 were tested during of the first refueling outage. No circuit problems were identified.

Item 8

Since the previously unverified circuit portions were subsequently tested and functioned correctly with no deficiencies identified, there is no safety significance associated with this condition.

Items 9 and 10

Since the previously unverified circuit portions were subsequently tested and functioned correctly with no deficiencies identified (excluding the alternate feeders), there is no safety significance associated with this condition.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET		LER NUMBER	PAGE (3)						
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	15	OF	18			
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04						

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Items 11, 12, and 13

Since the previously unverified circuit portions were subsequently tested and functioned correctly with no deficiencies identified, there is no safety significance associated with this condition.

V. CORRECTIVE ACTIONS

A. Immediate Corrective Actions

Item 1:

A test procedure was written and the logic circuits listed under item 1 were tested. Handswitch 1-HS-30-68A contacts were found unacceptable. Work Order 97007350-01 replaced the handswitch.

Items 8, 9, and 10

Upon discovery of this condition, Operations personnel entered TS SR 3.0.3 for missed surveillances (3.8.1.11 and 3.8.1.19) and the above described circuit verifications were subsequently satisfactorily performed within the 24 hour time limit or the appropriate LCOs were entered if the alternate board feeders were needed.

Items 11, 12, and 13

Upon discovery of this condition, Operations personnel took action to track completion of the required testing prior to entering Mode 4 from the refueling outage. The testing was subsequently completed.

B. Corrective Actions to Prevent Recurrence

In accordance with the schedule provided in TVA's letter to NRC dated April 18, 1996, technical reviews have been performed to compare electrical schematic drawings and logic diagrams for the Reactor Protections System (EIIS code JC/JG), Emergency Diesel Generator (EIIS code EK) load shedding and sequencing, and actuation logic for Engineered Safety Feature Actuation Systems (EIIS code JE) against plant surveillance test procedures to ensure that all portions of the logic circuitry including the parallel logic, interlocks, bypasses, and inhibit circuits are adequately covered in the surveillance procedures to fulfill the TS requirements. The review included relay contacts, control switches, and other relevant electrical components within these systems, utilized in the logic circuits performing a safety function. Upon discovery of questionable items, the conditions have been determined to be valid or invalid.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

	DOCKET		LER NUMBER		PAGE (3)			
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	16	OF	18	
Watte Bar Nuclear Plant Unit 1	05000390	97 -	- 011	04				

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

B Corrective Actions to Prevent Recurrence (continued)

Historical data including PEDS (EIIS code ID), ERFDS (EIIS code ID), and startup test data have been reviewed to determine if questionable logic circuits can be verified by alternate means. Records have been generated to document acceptance of alternate data in lieu of test verification documentation.

In the absence of acceptable verification documentation, the unverified circuit logic paths have been verified via testing for Items 1 through 6.

<u>Item 7</u>

The surveillance test insufficiencies in Item 7 were tested during of the first refueling outage. No circuit problems were identified.

The appropriate Technical Reviewers were informed of the requirements of GL 96-01 prior to restart of the first refueling outage.

Item 8:

Appropriate procedure changes will be made to correct the identified deficiencies.

The appropriate Technical Reviewers were informed of the review requirements as part of the recurrence control actions for Item 7.

Items 9, 10, 11, 12 and 13

Appropriate procedure changes will be made to correct the identified deficiencies.

General Corrective Actions

WBN has completed the GL 96-01 review discovery process. Testing required to establish compliance with the affected Technical Specification requirements will be completed prior to the end of the current Unit 1 cycle 1 refueling outage. Surveillance Instructions which have not been revised prior to the end of the current refueling outage will be placed on administrative hold and updated prior to their next performance.

NRC FORM 3	66A					U.S. NUCLEAR	REGULAT	ORY C	OMMIS	SION	
(4-95)			LICENSEE EVENT	REPORT (LE	(P)						
			TEXT CONT		167.J						
	•		FACILITY NAME (1)	DOCKET							
				05000	YEAR	SEQUENTIAL	REVISION	<u> </u>	<u>AGE (.</u>	5)	
				00000		NUMBER	[17	OF	18	
Watts Ba	r Nu	clea	r Plant, Unit 1	05000390	97	011 -	04				
TEXT (If more	spac	e is r	equired, use additional copies of NRC Form 366A)(17)				**************************************	<u> </u>		
VI. AD	DITI	ON/	AL INFORMATION						-		
Α.	Fa	iled	Components		·						
	1.	<u>Sa</u>	fety Train Inoperability								
	Item 1:										
		An su the an	inoperable condition existed for handswitch ccessfully initiate a Containment Spray sign ese two switches to initiate a Containment S alyzed in the Safety Analysis Report (SAR).	n 1-HS-30-68A ir al to Train A and pray signal is no	n comb I Train It credi	bination with 1 B of SSPS. M ted in any acc	-HS-30-6 Aanual in ident eve	8B to itiatior ent	ı of		
	2. Component/System Failure Information										
		a.	Method of Discovery of Each Component	or System Failur	e:						
			Item 1: Work Order 97007350-01 found dirty and i	intermittent conta	acts or	1-HS-30-68A	(EIIS co	de HS	S).		
	b. Failure Mode, Mechanism, and Effect of Each Failed Component:										
			Item 1: Switch contacts were intermittent.		s.					8	
		C .	Root Cause of Failure:							-	
			Item 1: Dirt was observed on the switch contacts.		,						
		d.	For Failed Components With Multiple Fu Affected:	nctions, List of	Syste	ms or Second	dary Fund	ctions			
			There were no component failures of thi	is nature.							
		e.	Manufacturer and Model Number of Eac	h Failed Compo	nent:						
			<u>Item 1:</u> Westinghouse type W-2								
Β.	<u>Pre</u>	viou	s Similar Event								
:	The sup	e sul pler	pject LER is bounded by the findings of GL s nented by the subject LER until the reviews	96-01 reviews. (are complete.	GL 96-	01 findings wi	ll be .	•			

NRC FORM 366A (4-95) LICENSEI TE	E EVENT REPORT (LE XT CONTINUATION	ER)	U.S. NUCLEA	IR REGULAT	ORY C	COMMI	SSION
FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)		
	05000	YEAR	SEQUENTIAL NUMBER	REVISION	18	OF	18
Watts Bar Nuclear Plant, Unit 1	05000390	97	011	04			
TEXT (If more space is required, use additional copies of NRC For	m 366A) (17)						

VII. COMMITMENTS

The actions committed to be implemented in response to this event are tabulated in Section V, Corrective Actions.