

ENCLOSURE

WATTS BAR NUCLEAR PLANT UNIT 1  
PROPOSED TECHNICAL SPECIFICATION CHANGES  
APPENDIX J, OPTION B

9705220163 970515  
PDR ADOCK 05000390  
P PDR

Insert after Section 5.7.2.18, page 5.0-28.

~~5.5 Programs and Manuals~~

5.7.2.19  
5.5.13

Primary Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program, dated September 1995," ~~as modified by the following exceptions:~~

+

The peak calculated containment internal pressure for the design basis loss of coolant accident,  $P_c$ , is ~~+~~ 15.0 psig.

The maximum allowable primary containment leakage rate,  $L_c$ , at  $P_c$ , shall be ~~+~~ 0.25% of primary containment air weight per day.

0.25

Leakage Rate acceptance criteria are:

a. Primary Containment leakage rate acceptance criterion is  $\leq 1.0 L_c$ . During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are ~~+~~ 0.60  $L_c$  for the Type B and Type C tests and  $\leq 0.75 L_c$  for Type A tests;

b. Air lock testing acceptance criteria are:

- 1) Overall air lock leakage rate is  $\leq 10.05 L_{a1}$  when tested at  $\geq P_a$ ,
- 2) For each door, leakage rate is  $\leq 10.01 L_{a1}$  when pressurized to  $[\geq 10 \text{ psig}]$ .

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the ~~Primary Containment Leakage Rate Testing Program~~.

The provisions of SR 3.0.3 are applicable to the ~~Primary Containment Leakage Rate Testing Program~~.

BASES

SURVEILLANCE  
REQUIREMENTS

SR 3.6.3.6 (continued)

Operating experience has shown that these components usually pass this Surveillance when performed at the 18 month Frequency. Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

SR 3.6.3.7

Verifying that each 24 inch containment lower compartment purge valve is blocked to restrict opening to  $\leq 50^\circ$  is required to ensure that the valves can close under DBA conditions within the times assumed in the analyses of References 1 and 2. If a LOCA occurs, the purge valves must close to maintain containment leakage within the values assumed in the accident analysis. At other times when purge valves are required to be capable of closing (e.g., during movement of irradiated fuel assemblies), pressurization concerns are not present, thus the purge valves can be fully open. The 18 month Frequency is appropriate because the blocking devices are typically removed only during a refueling outage.

SR 3.6.3.8

The as-left bypass leakage rate prior to the first startup after performing a required leakage test requires calculation using maximum pathway leakage.

This SR ensures that the combined leakage rate of all shield building bypass leakage paths is less than or equal to the specified leakage rate. This provides assurance that the assumptions in the safety analysis are met. ~~The leakage rate of each bypass leakage path is assumed to be the maximum pathway leakage (leakage through the worse of the two isolation valves); unless the penetration is isolated by use of one closed and de-activated automatic valve, closed manual valve, or blind flange, in this case, the leakage rate of the isolated bypass leakage path is assumed to be the actual pathway leakage through the isolation device. If both isolation valves in the penetration are closed, the actual leakage rate is the lesser leakage rate of the two valves. This method of quantifying maximum pathway leakage is only to be used for this SR (i.e., Appendix J maximum pathway leakage limits are to be quantified in accordance with Appendix J).~~

At all other times, the leakage rate will be calculated using minimum pathway leakage.

If then

(continued)

BASES

---

SURVEILLANCE  
REQUIREMENTS

SR 3.6.3.6 (continued)

Operating experience has shown that these components usually pass this Surveillance when performed at the 18 month Frequency. Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

SR 3.6.3.7

Verifying that each 24 inch containment lower compartment purge valve is blocked to restrict opening to  $\leq 50^\circ$  is required to ensure that the valves can close under DBA conditions within the times assumed in the analyses of References 1 and 2. If a LOCA occurs, the purge valves must close to maintain containment leakage within the values assumed in the accident analysis. At other times when purge valves are required to be capable of closing (e.g., during movement of irradiated fuel assemblies), pressurization concerns are not present, thus the purge valves can be fully open. The 18 month Frequency is appropriate because the blocking devices are typically removed only during a refueling outage.

SR 3.6.3.8

This SR ensures that the combined leakage rate of all Shield Building bypass leakage paths is less than or equal to the specified leakage rate. This provides assurance that the assumptions in the safety analysis are met. The as-left bypass leakage rate prior to the first startup after performing a leakage test, requires a calculation using maximum pathway leakage (leakage through the worse of the two isolation valves). If the penetration is isolated by use of one closed and de-activated automatic valve, closed manual valve, or blind flange, then the leakage rate of the isolated bypass leakage path is assumed to be the actual pathway leakage through the isolation device. If both isolation valves in the penetration are closed, the actual leakage rate is the lesser leakage rate of the two valves. At all other times the leakage rate will be calculated using minimum pathway leakage.

(continued)