

August 29, 1996

Tennessee Valley Authority
ATTN: Mr. Oliver D. Kingsley, Jr.
President, TVA Nuclear and
Chief Nuclear Officer
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: LARGE LOAD REDUCTION TEST - FINAL SAFETY ANALYSIS REPORT (FSAR)
CHAPTER 14, TABLE 14.2-2 INITIAL TEST PROGRAM - WATTS BAR
DOCKET NO. 50-390

Dear Mr. Kingsley:

Tennessee Valley Authority (TVA) letters of June 7, 1996 and June 10, 1996 summarized the performance of a Large Load Reduction Test (50% reduction from 100% power level) performed on May 12, 1996. These letters also described deviations from the FSAR test objectives that occurred during actual test performance. In addition, the letters provided a proposal to amend the Watts Bar FSAR Chapter 14, Table 14.2-2, sheet 35, Large Load Reduction Test Summary.

The enclosure contains the NRC staff review of your submittal. The staff concluded that the test, as performed, has not demonstrated the response of the plant to be consistent with plant design for the large load reduction transient. TVA must repeat the test as described in Amendment 91 to the FSAR or provide additional justification that the test, as performed, challenged affected systems with the same intensity and complexity as the one-step large load reduction.

A conference call on August 22, 1996, between myself and Bruce Schofield of your staff was held to discuss these matters. In that call, NRC was informed that TVA will perform a second Large Load Reduction Test as described in Amendment 91 to the Watts Bar FSAR and that this test is scheduled to be performed either prior to or after return to power following the September 1996 mid-cycle outage.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10 Code of Federal Regulations, a copy of this letter and its enclosure will be placed in the NRC Public Document Room.

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Should you have any questions concerning this letter, please contact me.

Sincerely,

**Original Signed by
M. S. Lesser**

Mark S. Lesser, Chief
Reactor Project Branch 6
Division Reactor Projects

Docket Nos. 50-390, 50-391
License No. NPF-90 and
Construction Permit No. CPPR-92

Enclosure: Supplemental Safety Evaluation
Initial Test Program Changes

cc w/encl: (See page 3)

TVA

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cc w/encl:

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