March 15, 1993

Docket No. 50-390

Tennessee Valley Authority
ATTN: Dr. Mark O. Medford, Vice President
Nuclear Assurance, Licensing and Fuels
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Dr. Medford:

SUBJECT: WATTS BAR UNIT 1 - STAFF QUESTIONS ON THE DRAFT TECHNICAL SPECIFICATIONS (TAC M76742)

We have reviewed the Watts Bar Safety Evaluation Report (SER) against the current draft of the Watts Bar Technical Specifications (TS) and Technical Requirements Manual (TRM), and found some discrepancies. In order for the TS and TRM to fully address the SER statements related to the TS, please respond to the enclosed list of items. Many of these items have been discussed with your staff during the TS meetings we have had since December 1992. Your written response is requested within 30 days of receipt of this letter in order to meet the schedule of the final draft of the Watts Bar TS.

The Proof and Review package consisting of the Watts Bar Proof and Review TS, the TRM , the Core Operating Limits Report (COLR) and Pressure and Temperature Limits Report (PTLR) will be issued for staff and TVA review on March 31, 1993. In order to maintain the review schedule, you should submit 25 copies of the revised TRM as well as any revisions to the August 27, 1992, COLR and PTLR submittals no later than March 22, 1993.

This requirement affects 9 or fewer respondents and, therefore, is not subject to Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original signed by

Peter S. Tam, Senior Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Request for Additional Information

cc with Enclosure:
See next page

OFFICE:	PDII-4/LA	PDII-4/PM	PDII-4/D
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Reviewed by L. Berry

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Tennessee Valley Authority
ATTN: Dr. Mark O. Medford

cc:
Mr. John B. Waters, Chairman
Tennessee Valley Authority
ET 12A
400 West Summit Hill Drive
Knoxville, Tennessee 37902

Mr. D. E. Nunn, Vice President
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Mr. W. J. Museler, Vice President
Watts Bar Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Spring City, Tennessee 37381

Mr. M. J. Burzynski, Manager
Nuclear Licensing and Regulatory Affairs
Tennessee Valley Authority
5B Lookout Place
Chattanooga, Tennessee 37402-2801

Mr. G. L. Pannell, Site Licensing Manager
Watts Bar Nuclear Plant
Tennessee Valley Authority
3B Lookout Place
Spring City, Tennessee 37381

TVA Representative
Tennessee Valley Authority
11921 Rockville Pike
Suite 402
Rockville, Maryland 20852

Mr. Michael H. Mobley, Director
Division of Radiological Health
3rd Floor, L and C Annex
401 Church Street
Nashville, Tennessee 37243-1532

General Counsel
Tennessee Valley Authority
ET 11H
400 West Summit Hill Drive
Knoxville, Tennessee 37902

The Honorable Robert Aikman
County Judge
Rhea County Courthouse
Dayton, Tennessee 37321

The Honorable Johnny Powell
County Judge
Meigs County Courthouse
Route 2
Decatur, Tennessee 37322

Regional Administrator
U.S.N.R.C. Region II
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

Senior Resident Inspector
Watts Bar Nuclear Plant
U.S.N.R.C.
Route 2, Box 700
Spring City, Tennessee 37381

Enclosure

Watts Bar Unit 1

Technical Specification Items Reviewed Against the SER and Supplements

Request for Additional Information

1. Safety Evaluation Report (SER) Section 4.2.1 states the following: "All fuel rods will be internally prepressurized with helium.... The special level of prepressurization will depend on the planned fuel burn up and will be determined before the Technical Specifications are established." The implication of the SER statement is that the fuel prepressurization is a design characteristic for the Watts Bar fuel assemblies and should be specified in Section 4.0, "Design Features" of the Watts Bar TS. The marked-up version of Rev. 0 of the Revised Standard Technical Specifications (RSTS) Section 4.2.1 "Fuel Assemblies," submitted on August 27, 1992, does not reflect the SER statement. Revise Section 4.2.1 to incorporate this design feature.

2. SER Section 4.2.3 states the following: "Although the staff concludes that fuel rod bowing calculations will be performed in an acceptable manner, final resolution of this issue will require that the applicant (1) identify in the basis of the Technical Specifications any plant specific or generic margin (credits) used to offset the reduction in DNBR due to fuel rod bowing and (2) incorporate the residual rod bowing penalty into the Technical Specifications." SER Section 4.4.4.1 provides additional information which could be incorporated into the TS to address this concern. The August 27, 1992, marked-up RSTS does not seem to address this. Either show that the requirement has been incorporated into the August 27, 1992, marked-up RSTS or propose appropriate TS and/or Bases.

3. SER Section 4.4.4.2 requires that appropriate surveillance requirements be included in the Technical Specifications to recognize any rapid crud buildup in the reactor core. Provide the appropriate surveillance requirements.

4. SER Section 5.2.5 describes the various items used to detect identified and unidentified reactor coolant system leakage. For unidentified leakage the staff states that if leakage is alarmed and confirmed in a flow path with no indicators, the TS will require that a water inventory material balance be initiated within 1 hour to determine the extent of the leakage. Provide the appropriate Conditions, Required Actions, and Completion Times in LCO 3.4.1.3, "RCS Operational Leakage."

5. SER Section 8.3.2.5 states that the DC Distribution System batteries have the capacity to supply all connected loads (Class 1E and Non-Class 1E) for a minimum of 2 hours and that the batteries will be tested periodically in accordance with the TS to ensure this capacity. Surveillance Requirement (SR) 3.8.4.12 verifies that the battery has sufficient capacity for the required emergency loads. It is not clear from the Bases discussion of this SR that the emergency loads include

the Non-Class 1E loads connected to the battery. Either modify the SR to include both emergency (Class 1E) and Non-Class 1E loads or define in the Bases that emergency loads includes both Class 1E and Non-Class 1E loads.

6. SER Section 9.5.4 describes the various means of filling the Diesel Generator (DG) fuel oil storage tank. One of these ways involved routing a hose from the delivery vehicle to the DG tank manway openings located in the DG building hallway area. The SER states that this DG tank filling method is acceptable provided that fire watches are stationed in these areas during the tank filling period and that the provision is included in the TS. Since the issuance of the SER, the staff has issued Generic Letter 88-12 which provides guidance to relocate fire protection requirements. Please either provide appropriate TS and Bases, or justification for relocation to the TRM.

7. SSER 7 Section 15.4.2.1 describes the staff analysis of the zero-power uncontrolled rod cluster assembly bank withdrawal event. The analysis assumed that the TS requires two reactor coolant pumps (RCPs) to be OPERABLE and in operation in MODES 3 and 4 when rod withdrawal is possible. The August 27 marked-up RSTS for LCO 3.4.6, "RCS Loops-Mode 4" does not require any RCPs to be running; it only requires two loops consisting of any combination of RCS loops and residual heat removal (RHR) loops be OPERABLE and only one loop shall be in operation. This does not meet the requirements specified in the analysis of the rod withdrawal event. Provide the appropriate changes for LCO 3.4.6 to meet the SSER assumption.

8. In SSER 5, Section 15.3.6 the staff found TVA's response to GL 83-28 with regards to items 3.1.3 and 3.2.3 on post-maintenance testing of the reactor trip breakers in TS to be acceptable. Provide appropriate surveillance requirements for these tests.

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