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- 3) 02049 NGO PRA MANAGER EC08I
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- 5) 02532 MCG NRC INSP MG-ADMIN MAIL RM
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REFERENCE

MCGUIRE NUCLEAR STATION
RECORD RETENTION # 005893

TECHNICAL SPECIFICATIONS (TS)
AND
TECHNICAL SPECIFICATIONS BASES
(TSB)

Page 2 of 3

Date: 10/29/07

Document Transmittal #: DUK073020001

QA CONDITION

☐ Yes ☒ No

OTHER ACKNOWLEDGEMENT REQUIRED ☒ Yes

IF QA OR OTHER ACKNOWLEDGEMENT REQUIRED, PLEASE
ACKNOWLEDGE RECEIPT BY RETURNING THIS FORM TO:

Duke Energy
McGuire
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DOCUMENT NO	QA COND	REV #/ DATE	DISTR CODE	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	TOTAL
TS & TSB MEMORANDUM	NA	--- 10/23/07	MADM-04B	V1	V1	V1	V1	V1	V1	X	V1	V1	V1	V3	V8	V1	V1	V2	48
NPF-9 AMENDMENT 243 (PAGE 3)	NA	- 10/23/07																	
NPF-17 AMENDMENT 224 (PAGE 3)	NA	- 10/23/07																	

REMARKS: PLEASE PLACE IN FRONT OF THE TECHNICAL SPECIFICATION MANUAL
(CHANGE TO FACILITY LICENSE)

RECIPIENT # 00422 PREVIOUSLY COMPLETED

G R PETERSON
VICE PRESIDENT
MCGUIRE NUCLEAR STATION

BY:

B C BEAVER MG01RC BCB/CMK

11/28
1001

October 23, 2007

MEMORANDUM

To: All McGuire Nuclear Station Technical Specification, and Technical Specification Bases (TSB)
Manual Holders

Subject: McGuire Technical Specifications

Technical Specification Manual: This is actually a change to the McGuire Nuclear Station Facility License which should be filed at the front of your Technical Specification Book. This change affects only page 3 of the unit 1 (NPF-9) and the unit 2 (NPF-17) License. See instructions below.

REMOVE

Renewed License No. NPF-9 Page 3 only

Renewed License No. NPF-17 Page 3 only

INSERT

Renewed License No. NPF-9 Page 3 only
Amendment No. 243

Renewed License No. NPF-17 Page 3 only
Amendment No. 224

Revision numbers may skip numbers due to Regulatory Compliance Filing System.

Please call me if you have questions.

Bonnie Beaver

Bonnie Beaver
Regulatory Compliance
875-4180

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3411 megawatts thermal (100%).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 243, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than June 12, 2021, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2; and,
- (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3411 megawatts thermal (100%).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 224, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than March 3, 2023, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.