

December 21, 2007

Mr. J. Randy Johnson
Vice President - Farley
Joseph M. Farley Nuclear Plant
7388 North State Highway 95
Columbia, AL 36319

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 – REQUEST
FOR ADDITIONAL INFORMATION (RAI) REGARDING RELIEF
REQUEST Q1/2B13-RR-V-2 (TAC NOS. MD6788 AND MD6789)

Dear Mr. Johnson:

By letter dated September 17, 2007, Southern Nuclear Operating Company submitted Relief Request Q1/2B13-RR-V-2. The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is required to complete its evaluation. The staff's RAI is enclosed.

If you have any questions, please do not hesitate to call me.

Sincerely,

/RA by EMarinos for/

Karen Cotton, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure:
RAI

cc w/encl: See next page

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JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2,
REQUEST FOR ADDITIONAL INFORMATION
REGARDING RELIEF REQUEST Q1/2B13-RR-V-2

1. Your justification in the relief request for extending the test interval beyond 5 years indicates that the acceptance criterion for the pressurizer safety valves (PSVs) lift settings is ± 3 percent of set pressure. Joseph M. Farley Nuclear Plant, Units 1 and 2 (Farley), Technical Specification 3.4.10 states that the PSVs are operable when the lift settings are within ± 1 percent of set pressure. Please explain why the acceptance criterion of ± 3 percent of set pressure is relevant justification for extending the test interval beyond 5 years when ± 1 percent of set pressure is in your licensing basis.
2. The American Society of Mechanical Engineers (ASME) is developing a new code case that will extend the test interval for Class 1 pressure relief valves from 60 months to 72 months with an additional 6 month grace period. The justification for extending the test interval in this proposed code case is that the valves will be required to be refurbished when removed for setpoint testing. Although the ASME *Code for Operation and Maintenance of Nuclear Power Plants* does not require that Class I pressure relief valves to be routinely refurbished, routine refurbishment provides reasonable assurance that setpoint drift will be minimized. Please discuss when PSVs N56963-01-0005 and N56963-01-0013 were last disassembled and inspected to verify that parts are free of defects resulting from time-related degradation or service-induced wear, and if degraded and defective parts that did not meet manufacturer specifications were replaced with new parts or reworked to conform to manufacturer specifications.
3. Your relief request submittal states that storage of the PSV is performed in accordance with procedures developed to comply with the requirements of American National Standards Institute (ANSI) Report, ANSI N45.2.2-1972, "Packaging, Shipping, Receiving, Storage, and Handling of Items for Nuclear Power Plants." Subsection 2.7 of ANSI N45.2.2-1972 describes the environmental conditions for items classified as Level A, B, C, and D. Please describe the environmental conditions (Level A, B, C, or D) for the PSVs in storage.
4. The relief request submittal does not discuss the impact of storage of PSVs on PSV set pressure. Please provide setpoint test results for PSVs (same manufacturer and model number as the ones installed at Farley) that were stored in an environment similar to the environment for the Farley PSVs for an extended period of time and setpoint tested prior to installation.
5. Your relief request submittal states that a risk analysis was performed to determine the risk impact due extending the test interval for the PSVs. Your submittal also references Regulatory Guide (RG) 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis." Section 3.3 of RG 1.174 describes the information that should be submitted to the NRC for review. Please provide the information described in Section 3.3 of RG 1.174 regarding the risk analysis discussed in your submittal.

Enclosure

Joseph M. Farley Nuclear Plant, Units 1 & 2

cc:

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