



830 Power Building

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

Central File
50-390
391

JAN 7 1977

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, NW.
Atlanta, Georgia 30303

Dear Mr. Moseley:

OFFICE OF INSPECTION AND ENFORCEMENT CIRCULAR 76-05 - IE:II:NCM
50-259, -260, -296, -327, -328, -390, -391, -438, -439 - BROWNS
FERRY UNITS 1, 2, AND 3, SEQUOYAH UNITS 1 AND 2, WATTS BAR UNITS 1
AND 2, AND BELLEFONTE UNITS 1 AND 2

This is in response to your October 8, 1976, letter which transmitted
IE Circular 76-05. It has been determined that no Grinnell shock
suppressors or sway braces have been or will be installed at the
Sequoyah, Watts Bar, or Bellefonte Nuclear Plants.

In July 1976, ITT Grinnell Corporation notified TVA that a Grinnell
shock arrestor (Serial No. 4068, Mark No. SS-2) at Browns Ferry Nuclear
Plant contained a control valve assembly that had an unacceptable bleed
rate because of improper factory setting. Since that time, Grinnell
has determined that the notification was in error because this shock
arrestor was manufactured before November 1974. Therefore, no modifi-
cations at Browns Ferry are required. A copy of a Grinnell letter
dated November 26, 1976, which reiterates the above is enclosed.

Very truly yours,

J. E. Gilleland
Assistant Manager of Power

Enclosure

CC (Enclosure):

Dr. E. Volgenau, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

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ENCLOSURE



November 26, 1976

ITT Grinnell Corporation

Executive Offices

*260 West Exchange Street
Providence, Rhode Island 02901
Telephone (401) 831-7000*

Pipe Hanger Division

Tennessee Valley Authority
915 Edney Bldg.
Chattanooga, Tenn. 37401

Attn: Richard Faureau

Subject: ITT Grinnell Hydraulic Shock Suppressors
for Browns Ferry Nuclear Power Plant

Gentlemen:

In our letters of July 12 and July 27 concerning snubber recalibration, we stated that Browns Ferry Nuclear Plant had one snubber, Serial No. 4068 Mark No. SS-2, requiring modification. After reviewing our records, we find this to be in error. The particular snubber mentioned was manufactured prior to Nov. 1974, when our current valve design was incorporated, and therefore requires no modifications.

Very truly yours,

ITT GRINNELL CORP.

A handwritten signature in cursive script, appearing to read 'J. Regan'.

J. REGAN

Hanger Engineer

JR/v

cc: J.D. Firestone - G.E.
R. Brimer - T.V.A.
Dr. E. Volgenau - N.R.C.
M. Plunkett - Prov.
M. Grosso - Prov.

THIS COPY FOR



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N.W. SUITE 818
ATLANTA, GEORGIA 30303

OCT 8 1976

In Reply Refer To:

IE:II:NCM
50-438 50-327
50-439 50-328
50-259 50-390
50-260 50-391
50-296

Tennessee Valley Authority
ATTN: Mr. Godwin Williams, Jr.
Manager of Power
830 Power Building
Chattanooga, Tennessee 37401

Gentlemen:

The enclosed Circular 76-05 is forwarded to you for action. If there are any questions related to your understanding of the actions required, please contact this office.

Sincerely,

Norman C. Moseley
Director

ENCLOSURE:
IE Circular 76-05

HYDRAULIC SHOCK AND SWAY SUPPRESSORS - MAINTENANCE OF BLEED
AND LOCK-UP VELOCITIES ON ITT GRINNELL'S MODEL NOS. - FIG. 200
AND FIG. 201, CATALOG PH-74-R

DESCRIPTION OF CIRCUMSTANCES:

Recent information has become available related to improper lock-up and bleed rates of certain ITT Grinnell shock and sway suppressors (snubbers). The control block or valve block on the snubbers involved is one which contains the mechanisms for a "dual orifice type" snubber valve arrangement. These control blocks or valve blocks can be identified by four locking nuts on the surface of the block. These nuts lock the stem of the valve control into position. Rotation of the stem modifies the orifices which control the lockup and bleed rates. The units in question, at this time, are those snubbers which have control block or valve block serial numbers from B-0001 through B-2000. These snubbers include 1 1/2 inch diameter (3 kip capacity) through 6 inch diameter (72 kip capacity) units.

ITT Grinnell first identified this problem in connection with their own testing program during the last two weeks of August 1975. The origin of the problem was traced by Grinnell to November of 1974 when a design change (known as Revision B) was initiated on Model Nos. - Figure 200 and Figure 201, Catalog PH-74-R snubbers, along with the addition of a Grinnell design specification for lock-up and bleed velocities. These new rates were to be 8 in/min. plus or minus 2 in/min. for lock-up and 4 in/min. plus or minus 1 in/min. for bleed. At that time production testing of completed snubbers included within the above series of serial numbers was completed on the basis of "go" or "no-go" testing and did not specifically determine the lock-up and bleed rates for each snubber.

Grinnell has determined that all snubbers involved should lock under seismic events as designed, but perhaps at a lower velocity. Additionally, the bleed rate has been determined to exist at a value as low as 1/8 in/min. This could result in increased piping stresses depending on the specific design. With the uncertainty of specific lockup and bleed rates, there is the possibility of adverse effects on the piping systems or components the snubbers were designed to protect. It has been stated, however, that to the best of Grinnell's knowledge, there were no lockup and bleed criteria defined by their customers for the plants identified as having utilized snubbers from the B-0001 through B-2000 series.

Even though lock-up and bleed rates on snubbers with adjustable orifices are now determined prior to shipment from the manufacturer, there is always the possibility that subsequent removal or tightening of the locking nuts on the control block or valve block will modify the orifice setting, thus affecting lock-up and bleed rates. At least two models of Grinnell's snubber control valves use some type of seal on the locking washers and in some cases these may have been replaced in the field to change the seal material or correct small leaks. In these cases, if proper procedures were not followed to maintain the stem portion in its original position, the orifice setting could change. The tightening of a lock nut itself could result in a change to the orifice. ITT Grinnell makes the following statement in the printed material furnished with each snubber assembly: "Adjustment of snubber valve requires equipment capable of measuring rate and load. DO NOT ATTEMPT TO RESET ADJUSTMENTS IN THE FIELD."

The recommended corrective action outlined by ITT Grinnell is to replace the entire valve assembly (2 required per snubber) known as the "barrel" in the case of the lock-up control, and the pins or stems (2 required per snubber) used to control the bleed rate. ITT Grinnell has prepared documents which prescribe the replacement procedures. The replacement parts are precalibrated. Therefore, field calibration of the entire snubber unit will not be necessary if the replacement parts are installed in accordance with the prescribed procedures. The tolerances on the precalibrated pins or stems associated with the bleed mechanism will exceed the original value of 4 inches per minute plus or minus 1 inch/min. The new bleed rate will be 4 inches/minute plus or minus 2 inches/minute.

Corrective action could also consist of recalibration of the existing snubbers with adjustments made to bring the operating characteristics of the units back within the tolerances specified by ITT Grinnell. Monitoring of relative rotation of the lock washer and stem could in the future be carried out if position marks were provided on the parts after calibration.

ITT Grinnell has identified the following facilities which have licenses under 10 CFR 50 which received snubbers within the group defined by the bounding serial number.

<u>Plant</u>	<u>Number of Grinnell Snubbers</u> <u>Model Nos.-Fig. 200 & Fig. 201</u>
Farley Nuclear Plant	239
Millstone Point Unit #2	185
North Anna	155
Davis Besse Unit #1	111
D. C. Cook	78
Calvert Cliffs	66
Beaver Valley	53
Diablo Canyon	41
Ginna	8
Kewaunee	2
Peach Bottom	2
Browns Ferry	1

ITT Grinnell has also informed the utilities who own these facilities regarding this matter; however, some of the series may be in use or intended for use at other reactor facilities as a result of utilities loaning or selling units to other utilities or by the fact that sub-contractors or other suppliers might be involved.

ACTION TO BE TAKEN BY LICENSEE:

For all power reactor facilities with an operating license or a construction permit using or intending to use the above described snubbers on safety related systems or components:

1. Determine if any of the subject snubbers are installed or scheduled to be installed in safety-related systems at your facility.
 - a. Provide a list containing each snubber identified and indicate the piping system or component with which it is associated.
 - b. Indicate for each snubber its history of testing and test results as well as its maintenance history.
 - c. For each snubber list the current required lock-up and bleed rates and what rates the snubbers were specified to meet by the original construction/purchase specification. Indicate the organization, by name, that was responsible for the preparation of the construction/purchase specification.
 - d. If replacement parts are utilized instead of recalibration of the snubbers, provide an assessment of the effect of a bleed rate of 2 inches/minute which represents the new lower bound.
2. For snubbers within the above described series that do not have documentation available defining the lock-up and bleed velocities, provide a description of the actions to be taken to substantiate these characteristics. Also, provide the schedule to complete these verifications and modifications, if needed.

3. Review maintenance and modification procedures for the above described series of snubbers where such actions could affect the performance characteristics of the snubbers. Such actions include but are not limited to the disassembly of snubbers, replacement of seals, changing hydraulic fluid and the changing of any valve springs. Modify the procedures, as appropriate, to assure that any future maintenance or modifications will not change the performance characteristics.
4. Provide a report which contains the responses to items 1, 2 and 3 above within 90 days of the date of this Circular. Reports should be submitted to the Director of the NRC Regional Office and a copy should be forwarded to the NRC Office of Inspection and Enforcement, Division of Reactor Inspection Programs, Washington, D. C. 20555.

Approval of NRC requirements for reports concerning possible generic problems has been obtained under 44 U.S.C. 3152 from the U.S. General Accounting Office. (GAO Approval B-180255 (R0062), expires 7/31/77).