

Entergy Nuclear Northeast Indian Point Energy Center 450 Broadway, GSB Buchanan, NY 10511-0249

Robert Walpole Licensing Manager Tel (914) 734-6710

October 5, 2007

Re: Indian Point Unit 3 Docket 50-286

NL-07-121

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

SUBJECT: Response to Second Request for Additional Information Regarding Relief Request 3-43 for Temporary Repair to Service Water Pipe

References: 1. Entergy letter NL-07-118 dated September 27, 2007 regarding Relief Request 3-43 for Temporary Repair to Service Water Pipe.

2. Entergy letter NL-07-120 dated October 3, 2007 regarding Response to Request for Additional Information for Relief Request RR 3-43.

Dear Sir or Madam:

Entergy Nuclear Operations, Inc (Entergy) is providing this response to a request for additional information regarding Relief Request RR 3-43 (Reference 1) as revised by Reference 2. NRC staff discussed this request for additional information with Entergy during a conference call on October 4, 2007. The additional information being provided (Attachments 1 and 2) does not result in a change to the Relief Request 3-43, Revision 1 submitted in Reference 2.

There are no new commitments being made in this submittal. If you have any questions or require additional information, please contact Mr. Robert Walpole, Manager, Licensing at (914) 734-6710.

Sincerely,

Robert Walpole Licensing Manager Indian Point Energy Center

AOU 7 MRR

cc: next page

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cc: Mr. John P. Boska, Senior Project Manager, NRC NRR DORL Mr. Samuel J. Collins, Regional Administrator, NRC Region 1 NRC Resident Inspector, IP3 Mr. Paul D. Tonko, President NYSERDA Mr. Paul Eddy, New York State Dept. of Public Service ATTACHMENT 1 TO NL-07-121

REPLY TO SECOND REQUEST FOR ADDITIONAL INFORMATION REGARDING INDIAN POINT 3 RELIEF REQUEST 3-43 FOR TEMPORARY NON-CODE REPAIR TO SERVICE WATER PIPING

ENTERGY NUCLEAR OPERATIONS, INC INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 DOCKET NO. 50-286

REPLY TO SECOND REQUEST FOR ADDITIONAL INFORMATION REGARDING RELIEF REQUEST 3-43

The following information is being provided based on a conference call between NRC staff and Entergy Nuclear Operations, Inc on October 4, 2007 regarding Relief Request (RR) 3-43 submitted September 27, 2007 (NL-07-118) and as revised October 3, 2007 (NL-07-120). This additional information does not change Revision 1 of RR 3-43 provided in the October 3, 2007 submittal.

NRC Question 1:

In Attachment 3, page 6, Section 6.3, the licensee stated that the minimum required wall thickness was determined by UT readings. In Section 6.4, the licensee stated that the wall thickness used for the Code case N-513-1 evaluation was based on the average value of the thickness around the thinned area of the pipe.

- (a) Discuss why two different approaches are used to in determining the wall thickness.
- (b) Discuss for each approach whether the conservative wall thickness is used.
- (c) Explain Item 2 in Section 6.3.

Entergy Response:

- a) There are two thickness values established for two different purposes in the flaw evaluation provided as Attachment 3 to the Entergy letter dated October 3, 2007.
 - 1. The minimum required wall thickness was calculated based on the USAS B31.1.0 Power Piping Code using the highest value of:
 - i) The stress value based on the pressure / hoop stress,
 - ii) The mechanical stress, or
 - iii) The administrative limit of 20%.

For this situation, the pressure / hoop stress is limiting. This is the approach used for establishing the wall thickness acceptance criterion limit for identifying a flaw.

- 2. The other wall thickness value used in Attachment 3 is developed based on results of field measurements obtained using ultrasonic testing (UT). This wall thickness value is used for evaluating flaw acceptability per Code Case N-513-1.
- b) The wall thickness values for both purposes are conservative. Calculation of minimum required wall thickness incorporates the inherent conservatism from the Power Piping Code. Determination of wall thickness based on UT measurements conservatively used that average thickness in the vicinity of the flaw instead of averaging the larger thickness around the entire pipe.
- c) The method discussed in item 2 of section 6.3 of the calculation describes the method of increasing the calculated pipe stress to account for the reduced section of the metal due to corrosion. There are two methods that can be used; 1) assume the pipe is uniformly thinned around the entire pipe based on the lowest measured value or 2) calculate the section modulus and other properties based on actual UT measurements. For this situation, Entergy used method 2. Code Case N-513-1 requires that periodic examinations of no more than 90-

day intervals be conducted to verify values used in the flaw growth evaluation. Entergy expects to implement a repair before the next examination is due.

NRC Question 2:

In Attachment 3, page 6, Section 6.6, the licensee referenced the 1995 edition of the ASME Code, Section XI, Appendix H in evaluating the flaw(s). However, as stated on page 1 of Attachment 2, the code of record is the 1989 edition. Explain why the 1995 edition of the code is used.

Entergy Response:

The K1c and J1c values needed for the flaw evaluation are not reported in the 1989 Edition of Section XI, so the procedure for flaw evaluation references the 1995 Edition for these values. The current code of record for repair and replacement activities at IP3 is the 2001 Edition through 2003 Addenda, as explained on page 1 of Attachment 2. Entergy has confirmed that coefficient values used in the flaw evaluation are consistent with the code of record.

NRC Question 3:

In Attachment 3, page 7, the licensee used 1.00 for the stress intensification factor (SIF) of the leak location. As the licensee indicated the value of 1.00 is for a pipe configuration. However, the pinholes are located in the part of the elbow, albeit on the straight end/section of the elbow, as shown in the photos that the licensee provided to the NRC. In addition, the pin holes are located close to the weld joining the elbow to the flange. The pinhole location would experience stress concentration because of their proximity to the weld. The purpose of the SIF is to model the piping loads in the analysis to account for the stress concentration at a piping location due to configuration or discontinuity. The staff believes that a SIF for the elbow should be used which is higher than 1.0. Justify why the SIF of 1.0 is acceptable.

Entergy Response:

The SIF of 1.0, actually a value of .75i equal to 1.0, was used to remove the over conservatism caused by using the Elbow SIF over the entire length of the elbow. As the Companion Guide to the ASME BPVC, page 555 states "the SIF at an elbow is maximum at the 45 degrees location." This accounts for the bucking at the out of plane which occurs at the center of the elbow only. Since the leak and thinned area is approximately 3" from the elbow weld, an SIF for a straight pipe was used. In addition, since the flaw is near the weld to the flange an as-welded SIF can be used. For the B31.1 1967 edition of the code the as-welded SIF is 1.3, therefore the .75i would also be 1.0.

NRC Question 4:

In Attachment 3, page 10, the licensee showed the equations for Fm and Fb, the parameters for circumferential flaw, are calculated. However, the equations and associated coefficients do not seem to be consistent with that of Appendix H of the ASME Code Section XI (the 1995 edition). Explain.

Entergy Response:

The equations and coefficients are taken from Appendix I of Code Case N-513-1. The numbers displayed on calculation sheet are truncated to fit the spreadsheet format used for the calculation.

NRC Question 5:

In Attachment 3, page 11, the licensee stated that after weld overlay the SIF needs to be 2.1 for even a straight pipe. Explain why a SIF of 2.1 is used after weld overlay.

Entergy Response:

This is from the requirement of Code Case N-661. The calculation was performed assuming the possible repair was going to be a weld overlay. If an overlay was performed then the code case requires an SIF of 2.1. This is derived from the as-welded SIF requirements contained in the later code versions. The statement is intended to state that stress requirements are met with the repair and an SIF of 2.1.

NRC Question 6:

Section 2(e), 2(f), 2(g), and 2(h) of Code case N-513-1 requires frequent periodic inspection or performing a flaw growth evaluation. Please address the requirements in each of these sections because it is not evident in the submittal that the licensee has met the requirements of N-513-1.

Entergy Response:

Entergy is implementing the requirements of Sections 2(e), 2(f), 2(g), and 2(h) of Code case N-513-1, where applicable for this situation. Evaluation of the flaw has been completed per Section 2(e) and as documented in the calculation IP-CALC-07-00083 provided to NRC (Attachment 3 of Entergy Letter NL-07-120 dated October 3, 2007). The operability evaluation performed for this condition recommended a 30-day time frame for completing a repair. Therefore, Code Case 513-1 requirements in section 2(e) that extend beyond 30 days would not be applicable, unless new data and an updated evaluation support a repair timeframe of longer than 30 days. Daily walkdowns are being performed per Section 2(f). Sections (g) and (h) pertain to performing a repair or replacement. Entergy is requesting NRC approval of Relief Request 3-43, Revision 1 to support our planned repair.

NRC Question 7:

For the flaw characterization and evaluation, identify any deviation from Code Case N-513-1 and other applicable code guidance or requirements including justification.

Entergy Response:

Entergy is following the requirements of Code Case N-513-1 for flaw characterization and evaluation as documented in Entergy Calculation IP-CALC-07-00083.

NRC Question 8:

Discuss and provide a report to show how the corrosion rate of 0.012 inches per year was determined. Discuss and provide the results of local corrosion rate.

Entergy Response:

The corrosion rate value of 0.024 inches per cycle, or 0.012 inches per year is a historical value developed over several years of evaluating service water piping systems and corrosion degradation at IP2 and IP3. There is not a reference document for this value. Local corrosion rates can depend on many factors and Entergy has not developed a standard value to be assumed for local corrosion. The proposed repair does not depend on a precise value for corrosion rate, because Entergy will be monitoring for additional degradation and repair integrity by daily walkdowns and periodic UT examinations as stated in Section E.4 of Relief Request 3-43 and in response to NRC Question 10.

NRC Question 9:

Identify the code requirements for design, fabrication and NDE of the proposed repair (plate and weld). Identify any deviations from such code requirements with justification.

Entergy Response:

The code requirements for design, fabrication, and NDE of the proposed repair are per original design requirements in B31.1 (1967). There have been no deviations from this code.

NRC Question 10:

Perform UT examination on pipe area adjacent to the repair weld (a band of three inches) to monitor the extent of degradation.

Entergy Response:

The scope of the inservice monitoring described in Section E.4 of Relief Request 3-43, Revision 1 will include UT within a 3-inch band around the top and side perimeters of the repair weld to monitor for degradation. The base of the repair weld is adjacent to the bolting flange for this elbow, so that a full 3-inch band is not available. The UT will extend as far as practical beyond the base of the repair.

NRC Question 11:

Please provide the UT report for the measurements recently taken using a ¼-inch grid.

Entergy Response:

UT Report IP3-UT-07-111 using the ¹/₄-inch grid is provided in Attachment 2. This UT exam provides for a more detailed inspection of two regions within the coverage of the UT exam previously performed with a 1-inch grid as documented in UT Report IP3-UT-07-110.

ATTACHMENT 2 TO NL-07-121

REPLY TO SECOND REQUEST FOR ADDITIONAL INFORMATION REGARDING INDIAN POINT 3 RELIEF REQUEST 3-43 FOR TEMPORARY NON-CODE REPAIR TO SERVICE WATER PIPING

Ultrasonic Test Results for Area Downstream of SWN-38

IP3-UT-07-111, September 2007 (1/4" grid)

ENTERGY NUCLEAR OPERATIONS, INC INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 DOCKET NO. 50-286



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Comments: Sketch of sub-mirco grid shown below superimposed over the micro grid established by report IP3-UT-07-110.

.25" X.25" GRIDS

Sketch or Photo:



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Report Date: 9/24/2007