



JAMES R MORRIS
Vice President

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October 12, 2007

U. S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555-0001

Subject: Duke Power Company LLC d/b/a Duke Energy
Carolinas, LLC
Catawba Nuclear Station, Units 1 and 2
Docket Nos. 50-413, 50-414

Response to a Request for Additional Information
(RAI) concerning the Catawba Nuclear Station
Units 1 and 2 Proposal to Revise Commitments to
USNRC Regulatory Guide 1.82, Rev.0 "Sumps For
Emergency Core Cooling and Containment Spray
Systems" and Revising Technical Specification
Surveillance Requirement (SR) 3.5.2.8 and
Associated Bases (TAC Nos. MD5163 and MD5164)

Please find the Duke Energy Carolinas (Duke) response to a
Request for Additional Information (RAI) concerning the license
amendment request (LAR) to Revise Commitments to USNRC
Regulatory Guide 1.82, Rev.0, "Sumps For Emergency Core Cooling
and Containment Spray Systems" and Revising Technical
Specification Surveillance Requirement (SR) 3.5.2.8 and
Associated Bases. This LAR was originally submitted by a Duke
letter to the NRC dated March 29, 2007. The RAI response on
September 7, 2007 left one question unanswered: Question #3 on
Attachment 1 from the Safety Issues Resolution Branch. This
delay was due to the need to obtain vendor calculations to
confirm the response. Attachment 1 of this document contains the
response to RAI Question #3.

There are no regulatory commitments in this letter.

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A001
NER

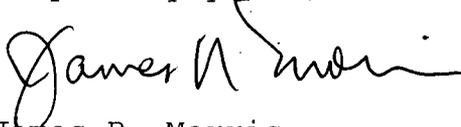
U.S. Nuclear Regulatory Commission

October 12, 2007

Page 2

If any questions arise or additional information is needed,
please contact Tony Jackson at (803) 831-3742.

Very truly yours,

A handwritten signature in cursive script, appearing to read "James R. Morris". The signature is written in dark ink and is positioned above the printed name.

James R. Morris

Attachment

James R. Morris affirms that he is the person who subscribed his name to the foregoing statement, and that all the matters and facts set forth herein are true and correct to the best of his knowledge.

James R. Morris

James R. Morris
Vice President, Catawba Nuclear Station

Subscribed and sworn to me:

10-12-07

Date

Micky Staudt

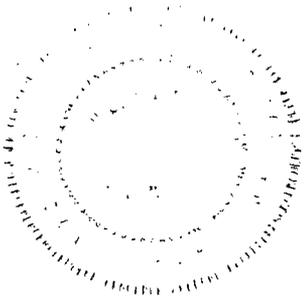
Notary Public

My commission expires:

7-10-2012

Date

SEAL



U.S. Nuclear Regulatory Commission

October 12, 2007

Page 4

xc: w/attachments

W. D. Travers, Region II Administrator
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H. J. Porter, Assistant Director
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U.S. Nuclear Regulatory Commission

October 12, 2007

Page 5

bxc (with attachments):

R. D. Hart

R. L. Gill

A. P. Jackson

K. E. Nicholson

NCMPA-1

NCEMC

PMPA

SREC

Catawba Document Control File: 801.01 - CN04DM

Catawba RGC Date File

ELL-EC050

ATTACHMENT 1

RESPONSE TO THE REQUEST FOR
ADDITIONAL INFORMATION QUESTION #3 FROM THE
SAFETY ISSUES RESOLUTION BRANCH
CATAWBA NUCLEAR STATION, UNITS 1 AND 2

3. As stated in attachment 3, section 2.2 "New ECCS Sump Strainer Assembly Design" of the submittal, the new strainer offers approximately 2000 square feet of surface area versus the original 135 square feet total for the original sump screens. Due to the magnitude of the increase of metallic heat sink mass, the post-LOCA containment temperature and pressure behavior may be subject to change.

Please provide a summarized analysis of the effects on the post-LOCA peak cladding temperature (PCT) due to the change in containment pressure and temperature behavior.

Response:

An evaluation was performed by Westinghouse to determine the impact of the increased containment metal mass and surface area on the Best Estimate Large Break LOCA (BELOCA) analysis for Catawba. The LOTIC2 computer code was used, consistent with the containment backpressure methodology for the Catawba BELOCA analysis of record. During the LOCA reflood phase, where containment pressure can influence the Peak Clad Temperature (PCT), a maximum reduction in containment pressure of 0.01 psig was observed when the additional sump strainer material was taken into account. Furthermore, the current LOTIC2 analysis of record includes some margin in Mass and Energy (M&E) release from the primary vessel during blowdown. From the LOTIC2 results and the M&E margin, Westinghouse has concluded that the additional containment metal mass has a negligible impact on the containment pressure/temperature and therefore a negligible impact on the resulting PCT. Duke agrees with this conclusion.