

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

400 Chestnut Street Tower II

April 12, 1985

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Ms. Adensam:

In the Matter of the Application of) Docket Nos. 50-390
Tennessee Valley Authority) 50-391

Recently, an NRC reviewer expressed a concern related to the reactor trip system (RTS) allowable value for reactor coolant pump (RCP) undervoltage which is presently contained in the Watts Bar Nuclear Plant (WBN) draft technical specifications (item 16, page 2-5). The reviewer's concern was that the Final Safety Analysis Report (FSAR) indicated that the safety analysis limit for RCP bus undervoltage was 70 percent of the bus voltage, whereas, the draft technical specification allowable value was 69 percent of bus voltage. This would appear to allow operation with a nonconservative RTS setpoint.

After discussions with Westinghouse, it was determined that the safety analysis limit for RCP undervoltage was actually 68 percent of bus voltage. Accordingly, on April 5, 1985, a telephone conversation was held with the reviewer. During the discussion, the reviewer was informed that the limit used in the safety analysis was 68 percent of bus voltage and that the FSAR was incorrect. The reviewer then requested that TVA formally indicate that the RCP undervoltage safety analysis limit is 68 percent of bus voltage. By letter dated April 10, 1985, Westinghouse has confirmed that the safety analysis limit is 68 percent of the bus voltage.

Enclosed is the corresponding FSAR revision. Please note that this revision will be incorporated into the next FSAR amendment (No. 56).

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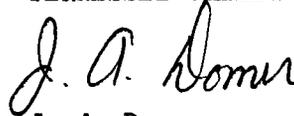
Director Nuclear Reactor Regulation

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If you have any questions concerning this matter, please get in touch with D. B. Ellis at FTS 858-2681.

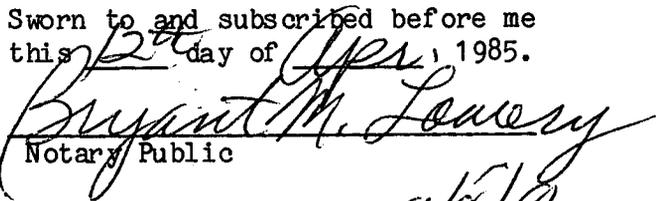
Very truly yours,

TENNESSEE VALLEY AUTHORITY



J. A. Damer
Nuclear Engineer

Sworn to and subscribed before me
this 12th day of April, 1985.



Notary Public

My Commission Expires 4/8/86

Enclosure

cc: U.S. Nuclear Regulatory Commission (Enclosure)
Region II
Attn: Dr. J. Nelson Grace, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

ENCLOSURE

TABLE 15.1-3 (Continued)

TRIP POINTS AND TIME DELAYS TO TRIP
ASSUMED IN ACCIDENT ANALYSES

<u>Trip Function</u>	<u>Limiting Trip Point Assumed In Analysis</u>	<u>Time Delays (Second)</u>
Low Reactor Coolant Flow (from loop flow detectors)	87% loop flow	1.0
Undervoltage Trip	70% 68%	1.5
Turbine Trip	Not applicable	1.0
Low-Low Steam Generator Level	0% of narrow range span between 0 and 20% nominal load, and increasing linearly to 49% of span at 100% of nominal load	2.0
High-High Steam Generator Level, Turbine Trip	89.7% of narrow range level span	2.5
High-High Steam Generator Level, Feedwater Isolation	83% of narrow range level span	11.0

Revised by Amendment