

## **US-APWR**

# 9th Pre-Application Review Meeting Safety Features

# August 23, 2007 Mitsubishi Heavy Industries, Ltd.

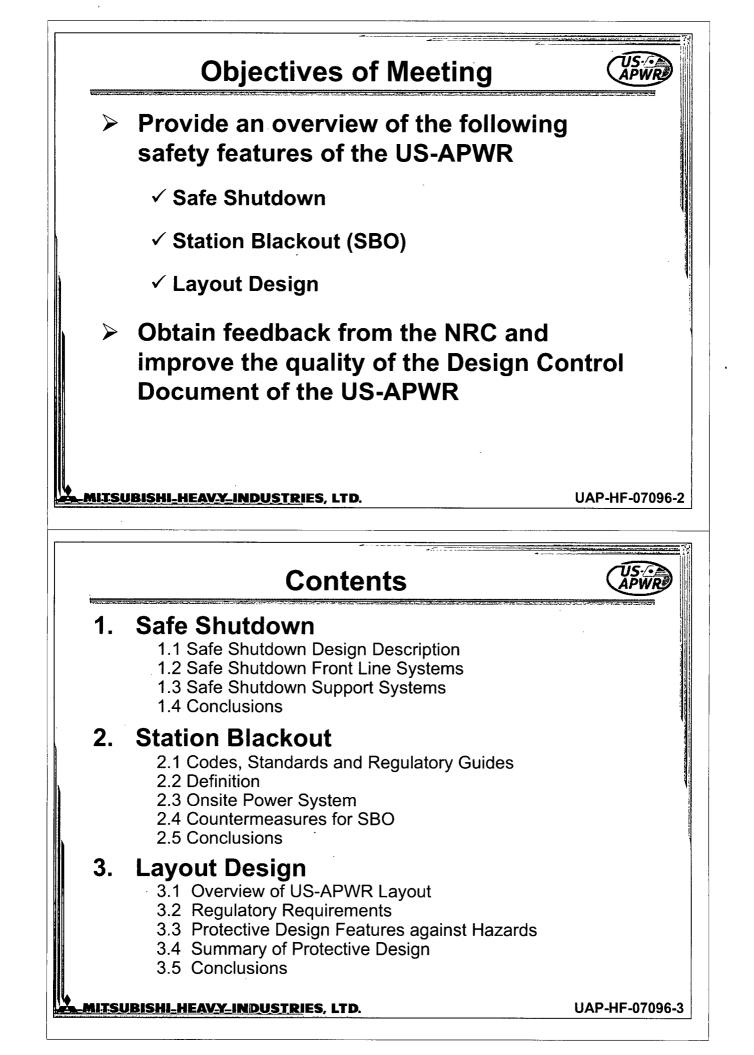
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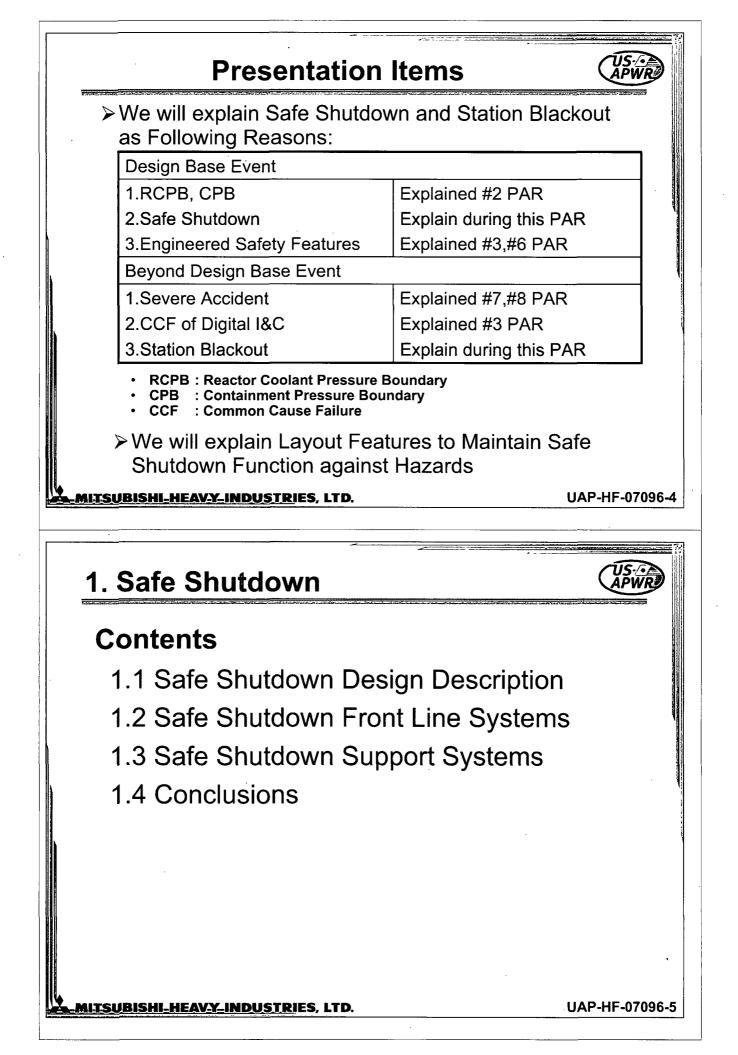
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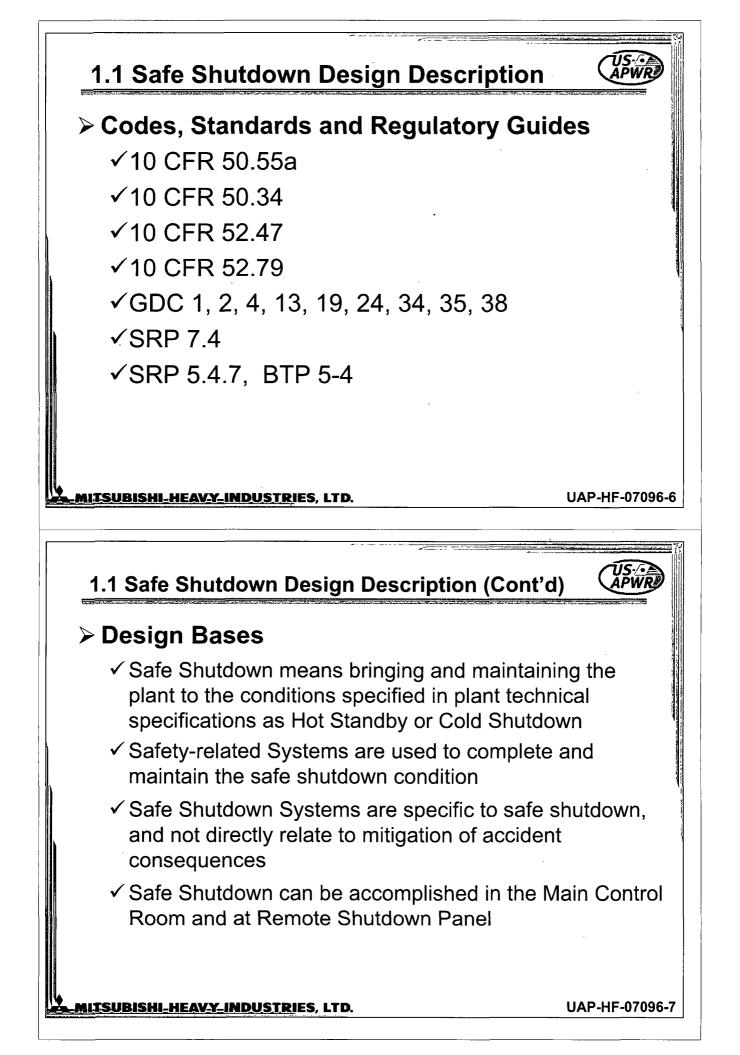
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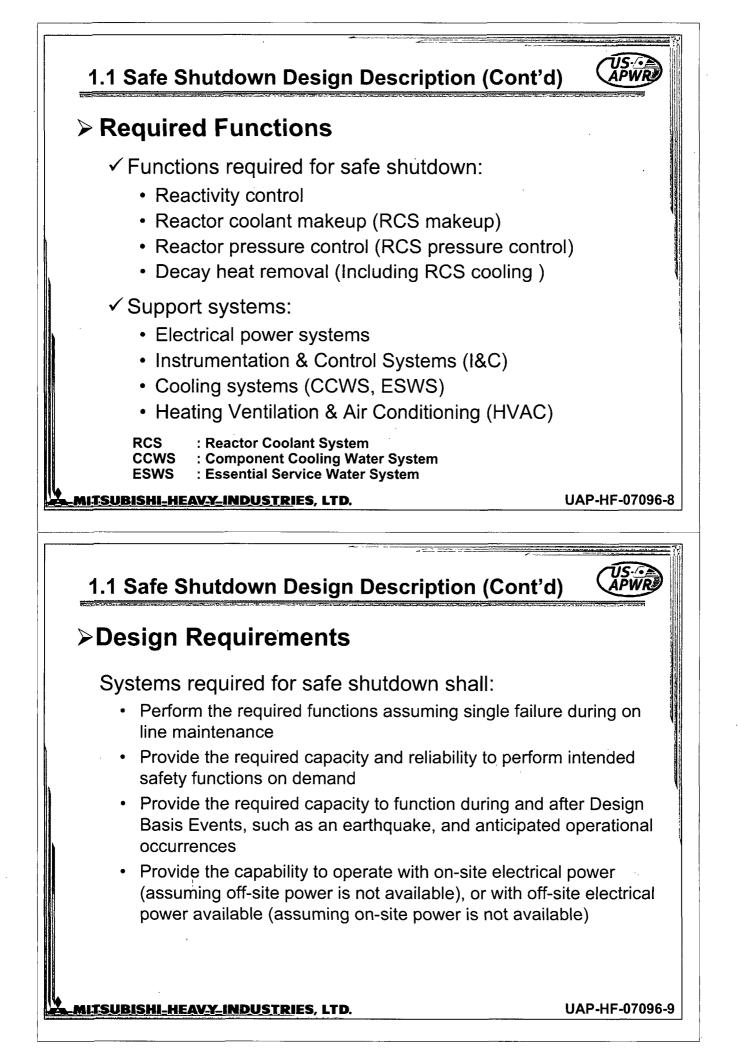
# **Meeting Attendees**

Hiroshi Hamamoto (Responsible for System Design) Deputy Chief Engineer Water Reactor Engineering Department Nuclear Energy Systems Engineering Center Mitsubishi Heavy Industries, LTD. Kaname Shibato (Responsible for General Arrangement Design) Acting Manager Plant Layout Engineering Section Nuclear Energy Systems Engineering Center Mitsubishi Heavy Industries, LTD. Shinji Kawanago (Representative for Electrical Design) Engineering Manager	
Mitsubishi Nuclear Energy Systems, Inc.	
Andrew Johnson - Presenter - (Technical Advisor for Fluid System Design) Mitsubishi Nuclear Energy Systems, Inc.	
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# **1.2 Safe Shutdown Front Line Systems**

### ✓ Reactivity control

Current PWR	US-APWR	Reasons for Selecting System	Compliance to NRC Req.
<ul> <li>Using CVCS</li> <li>Boron injection from BAT using BA Pump, Charging Pump and Charging line</li> <li>Letdown using Reactor Vessel Head letdown</li> </ul>	<ul> <li>Feed &amp; Bleed using SI Pumps and emergency letdown lines</li> <li>Borated water source is RWSP</li> <li>Need to be depressurized below SI pump injection pressure</li> </ul>	•Safe Shutdown function does not rely on CVCS which satisfies US utility requirements	•Meets SRP 7.4 •Same design as SP-90
<ul> <li>RWSP : Refueling</li> <li>SI Pump : Safety</li> </ul>	g Water Storage Pit		
• BAT : Boric Acid	Tank, BA Pump : Boric A		UAP-HF-07096-10 d)
• BAT : Boric Acid	Tank, BA Pump : Boric A USTRIES, LTD. WN Front Line Sys		<u>AIS</u>
• BAT : Boric Acid	Tank, BA Pump : Boric A USTRIES, LTD. WN Front Line Sys trol (Cont'd)		<u>AIS</u>

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G

Charging

Pump

Reactivity Control

F 0

Boric Acid Pump

Boric Acid

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US. APWR

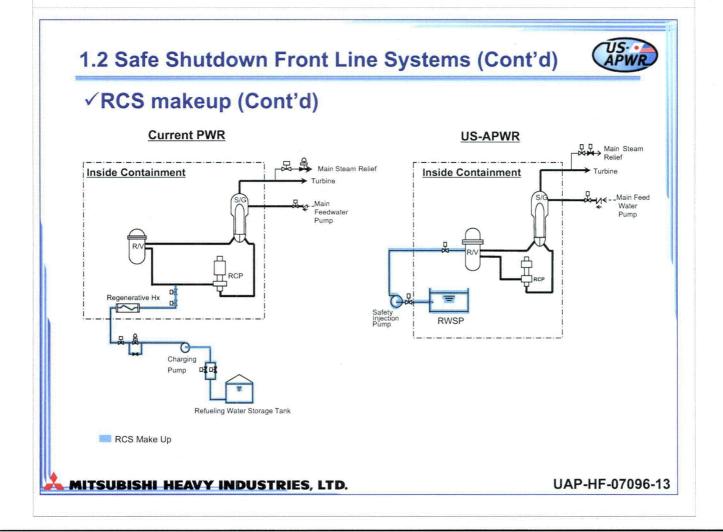
### **1.2 Safe Shutdown Front Line Systems (Cont'd)**



### ✓ RCS makeup

Current PWR	US-APWR	Reasons for Selecting System	Compliance to NRC Req.
<ul> <li>Make up using Charging Pump</li> <li>Borated water source is RWST</li> </ul>	<ul> <li>Make up using SI Pump</li> <li>Borated water source is RWSP</li> <li>Need to be depressurized below SI pump injection pressure</li> </ul>	•Safe Shutdown function does not rely on CVCS which satisfies US utility requirements	<ul> <li>Meets SRP 7.4</li> <li>Same design as SP-90</li> </ul>

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### **1.2 Safe Shutdown Front Line Systems (Cont'd)**

### ✓RCS pressure control

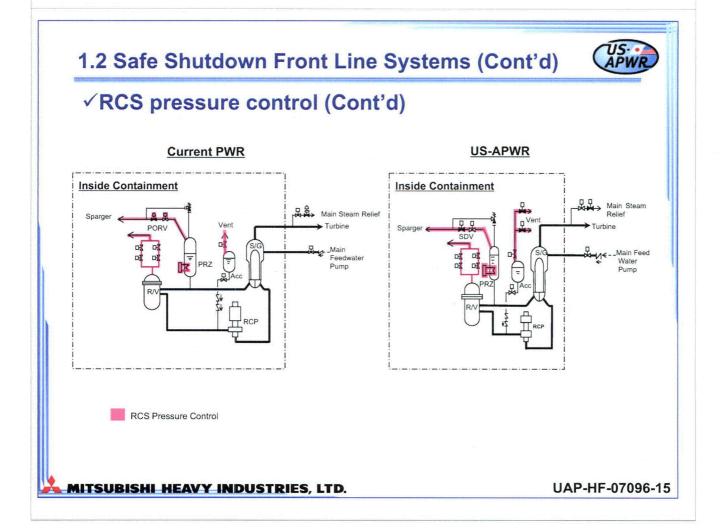
Current PWR	US-APWR	Reasons for Selecting System	Compliance to NRC Req.
<ul> <li>Pressurized by</li> <li>Pressurizer backup heater</li> <li>Depressurized by PORV</li> <li>RV vented by using RV Head Vent Line</li> <li>Accumulator N<sub>2</sub> vented by using Vent Line</li> </ul>	<ul> <li>Pressurized by Pressurizer backup heater</li> <li>Depressurized by SDV</li> <li>RV vented by using RV Head Vent Line</li> <li>Accumulator N<sub>2</sub> vented by using Vent Line</li> </ul>	•US-APWR adopts SDV since IAS is non safety and eliminates automatic actuation which satisfies US utility requirements	•Meets SRP 7.4

PORV : Pressurizer Power Operated Relief Valve (Air Operated Valve)

SDV : Safety Depressurization Valve (Motor Operated Valve)

IAS : Instrument Air System

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### 1.2 Safe Shutdown Front Line Systems (Cont'd)



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Current PWR	US-APWR	Reasons for Selecting System	Compliance to NRC Req.
Decay Heat Removal and Cooling by SG • 3 AFW Pumps (2 M/D, 1 T/D) •AFW supplied by AFW Pumps •AFW source is AFW Tank •Steam relieved by MSRV	Decay Heat Removal and Cooling by SG • 4 EFW Pumps (2 M/D, 2 T/D) •EFW supplied by EFW Pumps •EFW sources are EFW Pits •Steam relieved by MSRV	•4 Train enables to perform required function assuming single failure during OLM (explained at #6 PAR)	<ul> <li>Meets SRP</li> <li>7.4</li> <li>Basically</li> <li>same as</li> <li>Current</li> <li>PWR</li> </ul>
• AFW : Auxiliary Feed	Water		

• EFW : Emergency Feed Water

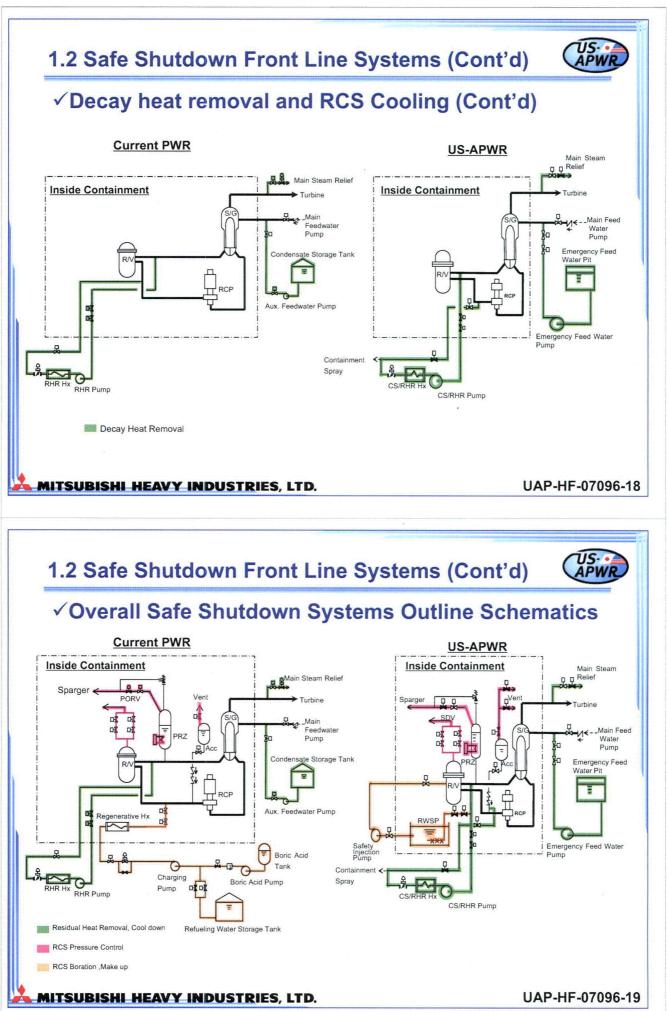
- M/D : Motor Driven, T/D : Turbine Driven
- MSRV : Main Steam Relief Valve
- •OLM : On Line Maintenance

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✓ Decay heat re	emoval and RCS	Cooling (Cont	'd)
Current PWR	US-APWR	Reasons for Selecting System	Compliance to NRC Req.
Cooling by RHRS •2 Trains •RCS cooled using RHR Pump and RHR Hx	Cooling by RHRS •4 Trains •RCS cooled using CS/RHR Pump and CS/RHR Hx	•4 Train enables to perform required function assuming single failure during OLM (explained at #6 PAR)	•Meets SRP 5.4.7, BTP 5-4, SRP 7.4 •Basically same as Current PWR

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# 1.3 Safe Shutdown Support Systems

✓ Support Function and Components

Support System		Remarks
Cooling Systems (CCWS, ESWS)	•ESW Pump •CCW Pump, CCW Hx	Including Ultimate Heat Sink
HVAC Systems	<ul> <li>Main Control Room HVAC System</li> <li>Class 1E Electrical Room HVAC System</li> <li>Emergency Feed Water Pump Area HVAC System</li> <li>Safeguard Component Area HVAC System</li> </ul>	Including Essential Chilled Water System
Electrical Power Systems	Safety Power Systems	

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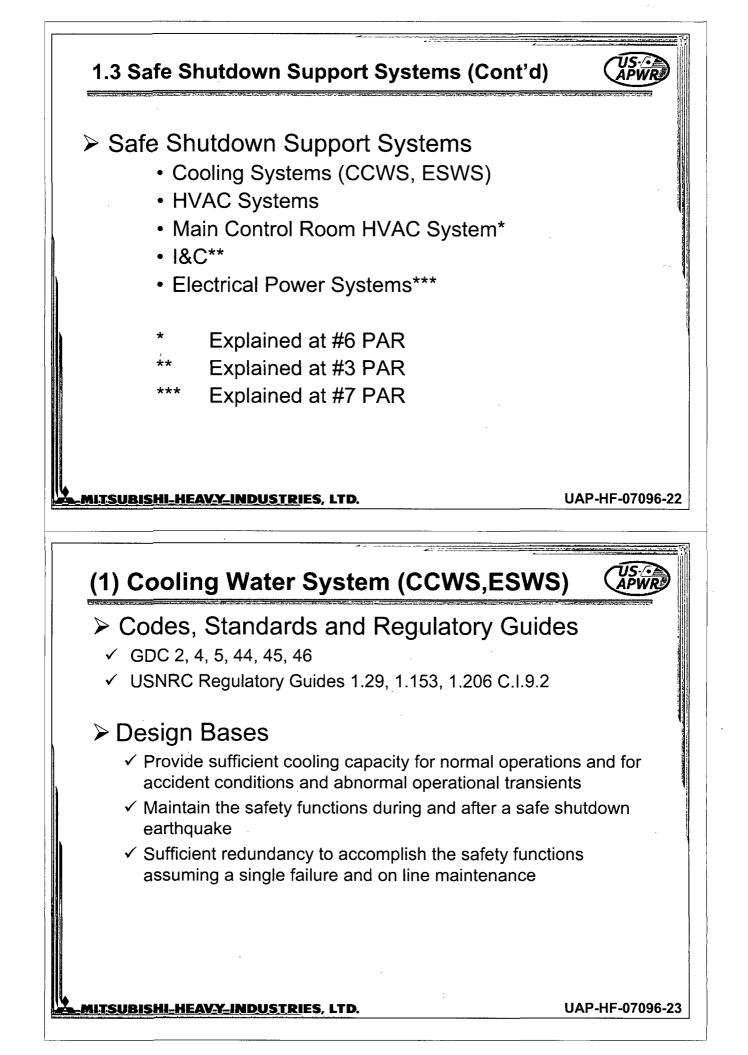
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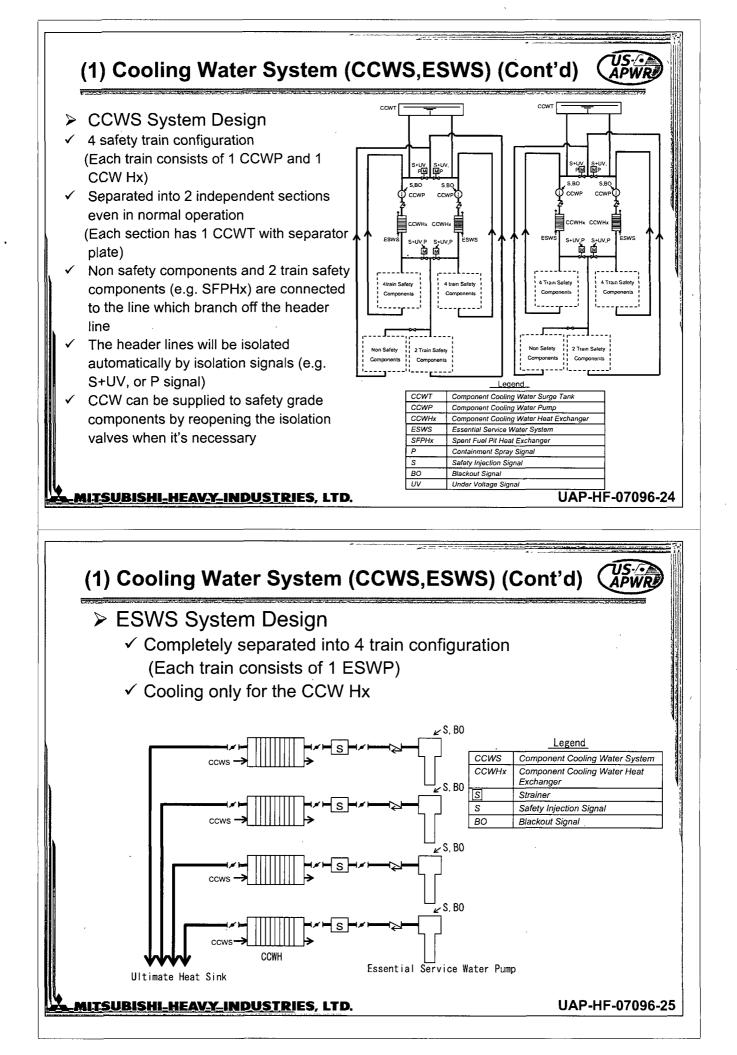
US:02 APWR

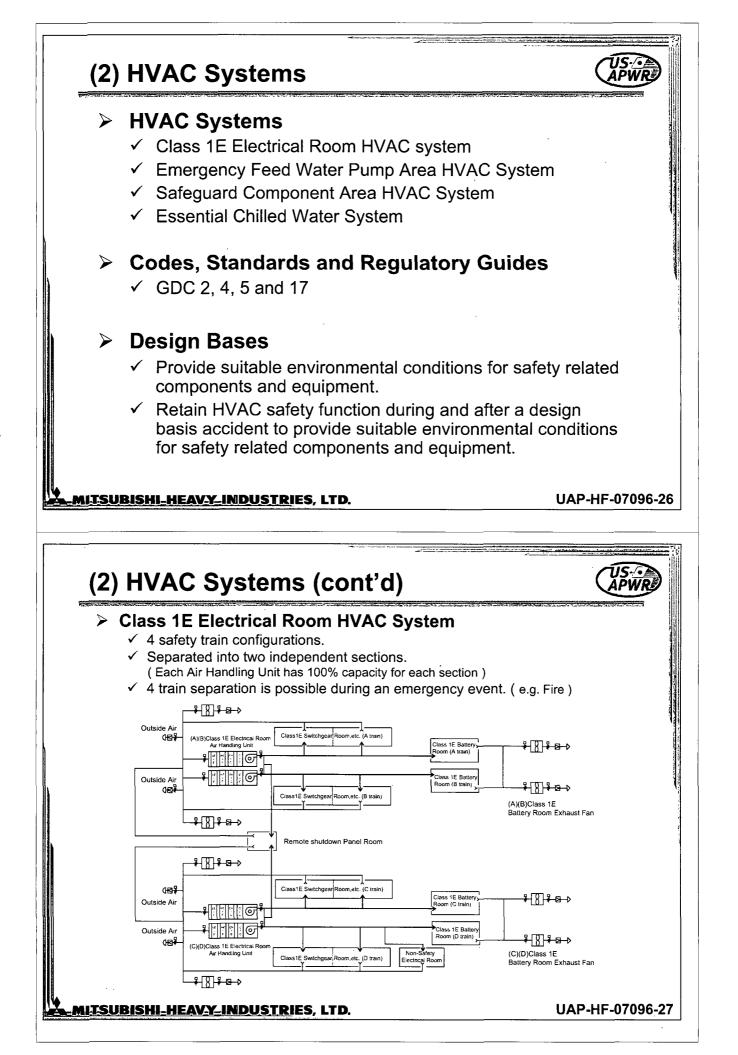
### 1.3 Safe Shutdown Support Systems (Cont'd)

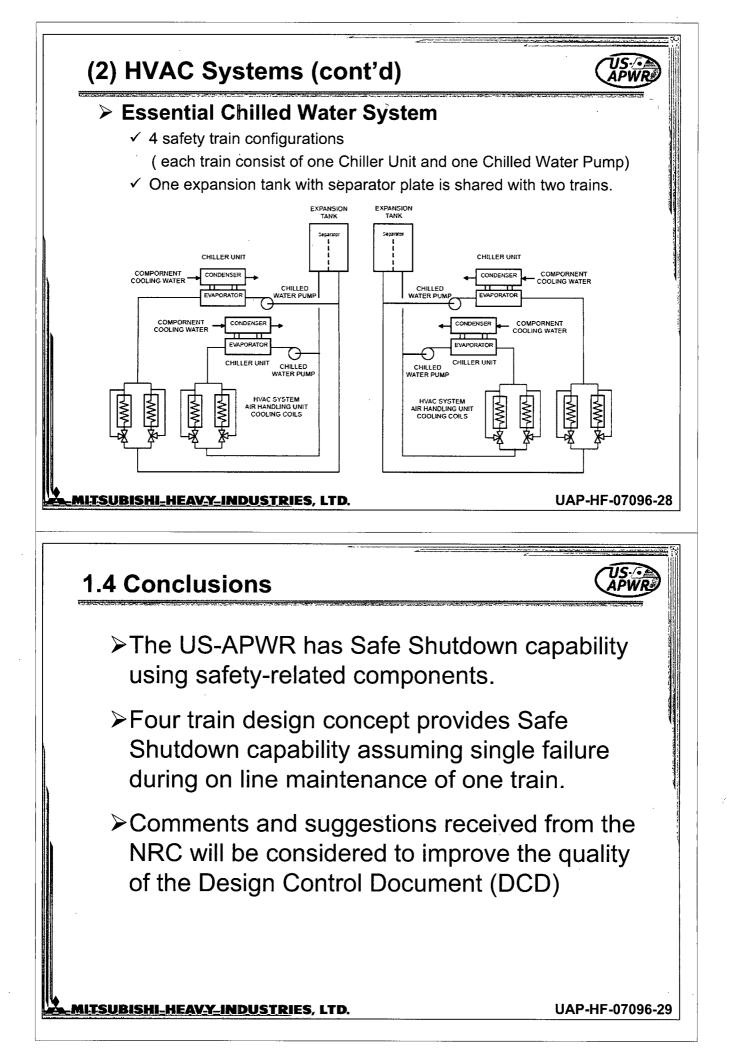
### ✓ Support Function and Components (Cont'd)

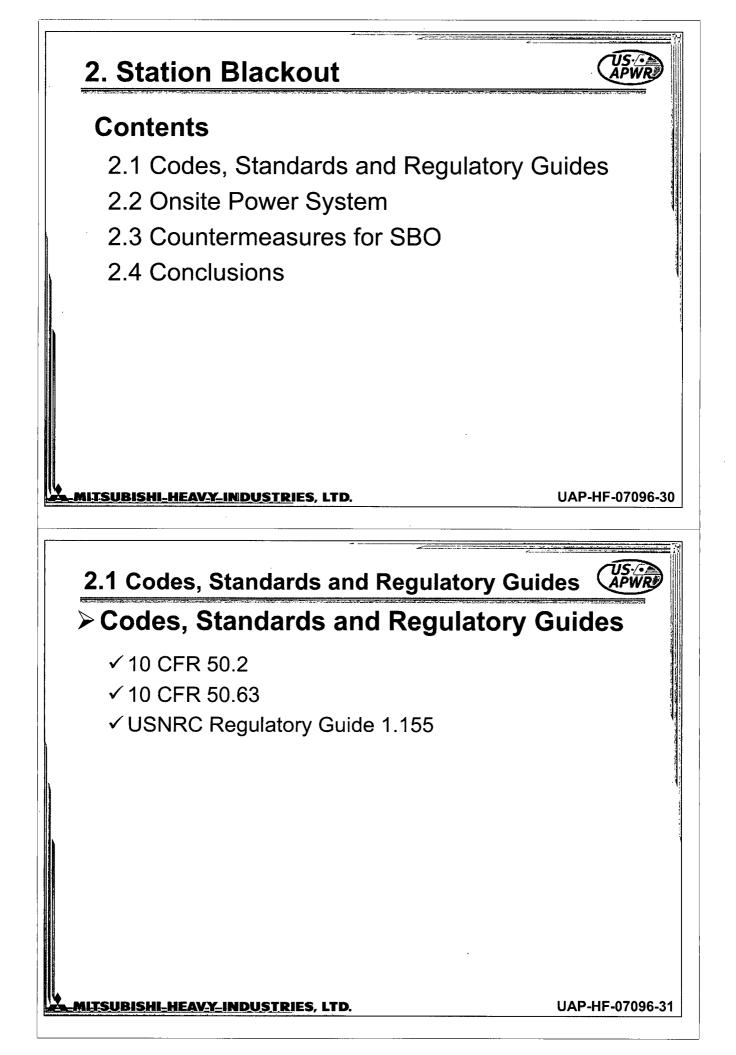
Pressurizer water level	
	Including
Pressurizer pressure	Control
•RCS pressure	System
•RCS temperature	
<ul> <li>Reactor Vessel water level</li> </ul>	
•RHR flow	
<ul> <li>Accumulator tank pressure</li> </ul>	
•SG water level, Main Steam pressure	
<ul> <li>Safety Injection flow rate</li> </ul>	
<ul> <li>RWSP water level</li> </ul>	
•EFWP water level, EFW flow rate	
	<ul> <li>RCS pressure</li> <li>RCS temperature</li> <li>Reactor Vessel water level</li> <li>RHR flow</li> <li>Accumulator tank pressure</li> <li>SG water level, Main Steam pressure</li> <li>Safety Injection flow rate</li> <li>RWSP water level</li> </ul>

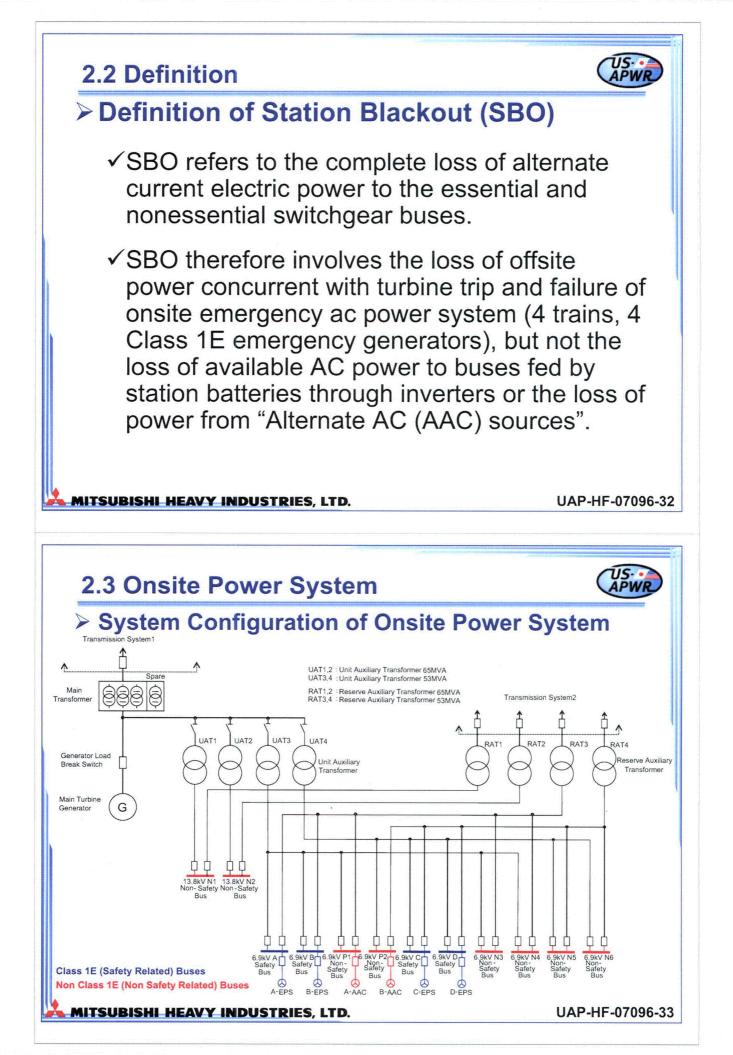










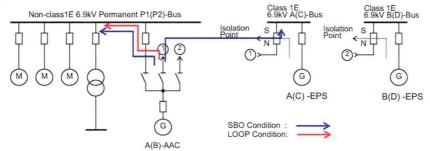


### 2.3 Onsite Power System (Cont'd)



### Design Features of Onsite Power System

- ✓ Class 1E AC electrical power system consists of four (4) separate trains. Each train includes one Class 1E Emergency Power Source (EPS). Class 1E DC power system consists of four (4) separate trains. Each battery can supply the loads for two (2) hours, without any AC power input
- ✓ AACs of a different type (Starting System, Capacity etc.) and are provided to minimize the potential for common mode failure with either the offsite power or the EPS system. The AAC is a non-class 1E gas turbine-generator package connected to a 6.9kV AC "Permanent" bus



When a LOOP occurs, the AAC supplies power to "permanent" bus automatically. Interlocks will prevent the AAC and EPS from being operated in parallel on same bus.

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# 2.3 Onsite Power System (Cont'd)



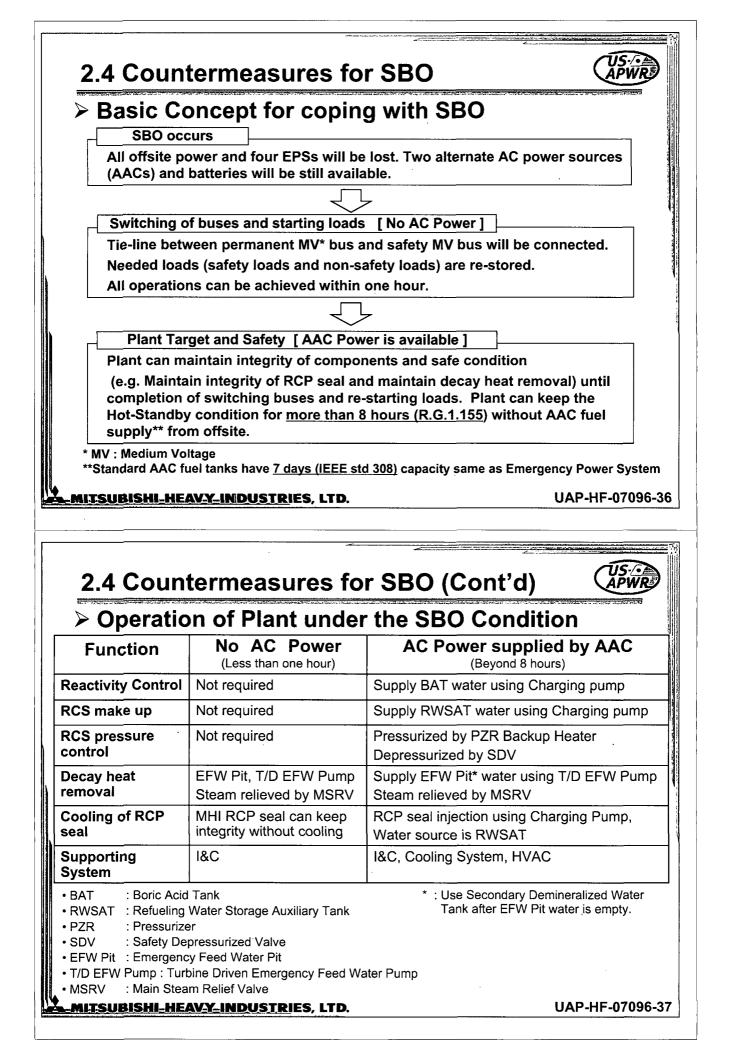
### Operation of AAC under the SBO Condition

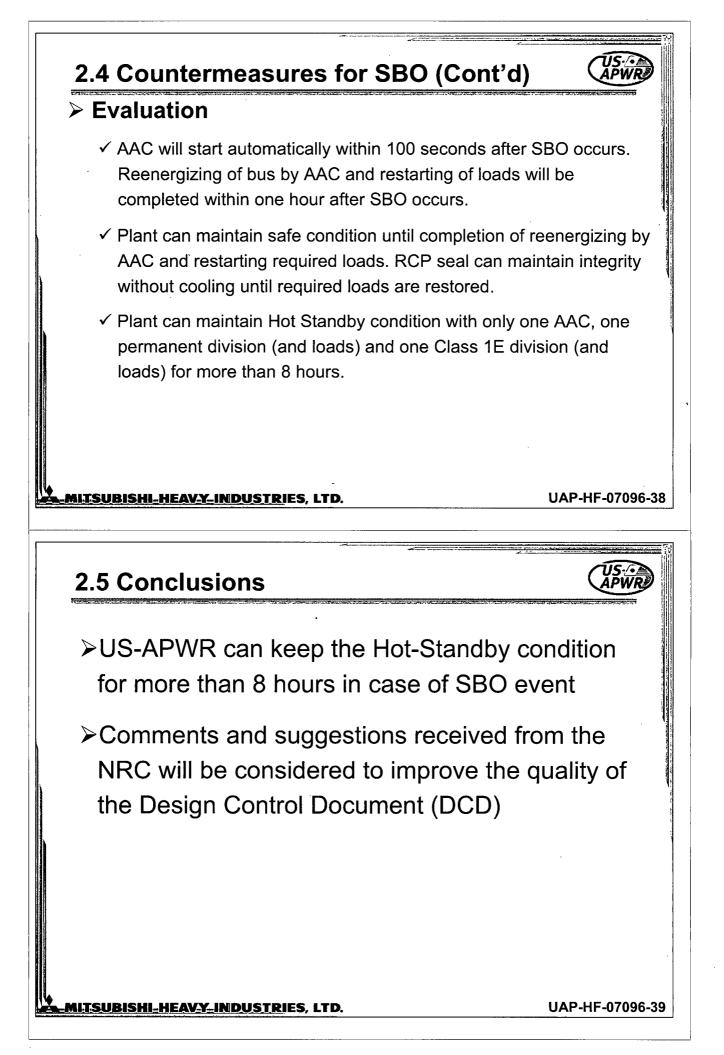
When a SBO occurs, the AAC power source feeds the selected safety related (Class 1E) division in accordance with the following procedure:

- 1) Switching will be done manually to prevent starting loads on any other division.
- 2) Loads that need to be stopped on the "permanent" bus will be tripped manually.
- 3) The required breaker to the back-up division will be closed manually.
- 4) Back-up bus will be energized and loaded as necessary for coping with SBO. The loads will be started manually.

This procedure will be accomplished within one hour.

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# 3. Layout Design

# Contents

3.1 Overview of US-APWR Plant Layout

3.2 Regulatory Requirements

- 3.3 Protective Design Features against Hazards
- 3.4 Summary of Protective Design

3.5 Conclusions

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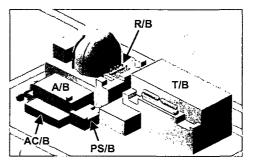
# 3.1 Overview of US-APWR Plant Layout

### Features of US-APWR Plant Layout

- Economical Benefit by Standardized Single Unit

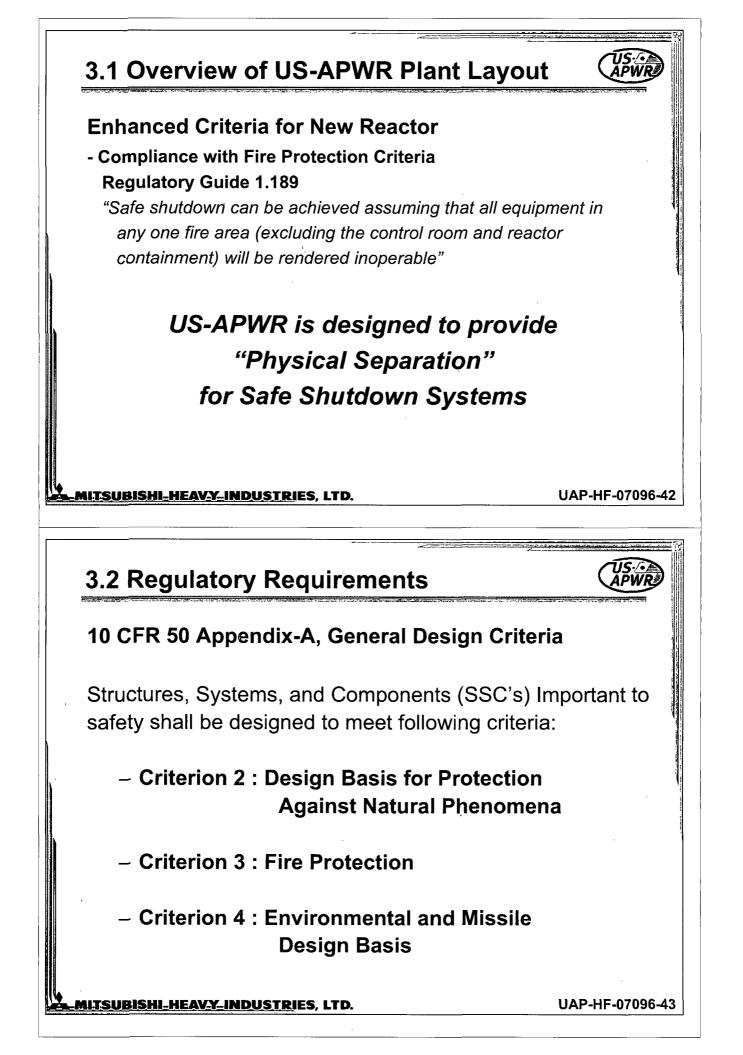
- Shielding to Minimize Radiation Exposure, and Complete Separation of Radioactive Control Area
- Quakeproof Square Footprint & Isolated Base Mat of Seismic Category-I Buildings
- Turbine axis is oriented perpendicular to Safety Buildings
- Protect Safety Systems against Hazards
  - (e.g. Flooding, Missiles, Tornado, Fire)

<u>Major Buildings</u> R/B : Reactor Building PS/B : Power Source Buildings T/B : Turbine Building A/B : Auxiliary Building AC/B : Access Building



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## 3.2 Regulatory Requirements



#### **Regulatory Guidance for Implementing Requirements**

- R.G. 1.59 Design Basis Floods for Nuclear Power Plants (NPPs)
- R.G. 1.76 Design Basis Tornado and Tornado Missiles for NPPs
- R.G. 1.102 Flood Protection for NPPs
- R.G. 1.115 Protection against Low-Trajectory Turbine Missiles
- R.G. 1.117 Tornado Design Classification
- R.G. 1.189 Fire Protection for NPPs

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# 3.3 Protective Design Features against Hazards

# **Protection against Floods**

### **Mitigation of Internal Flood Source**

- Provide watertight compartment for non-safety tank room

#### Layout Safety Components above internal Flood Level - Provide concrete foundation of safety components with proper height

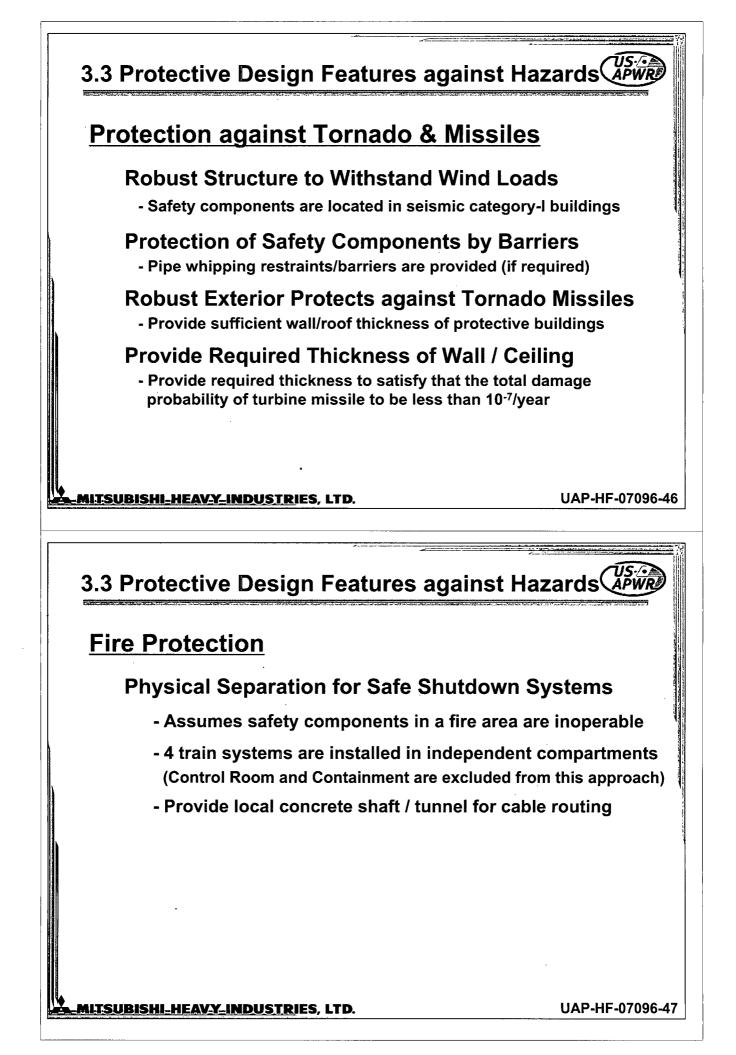
### **Robust Structure to Withstand External Flood Loads**

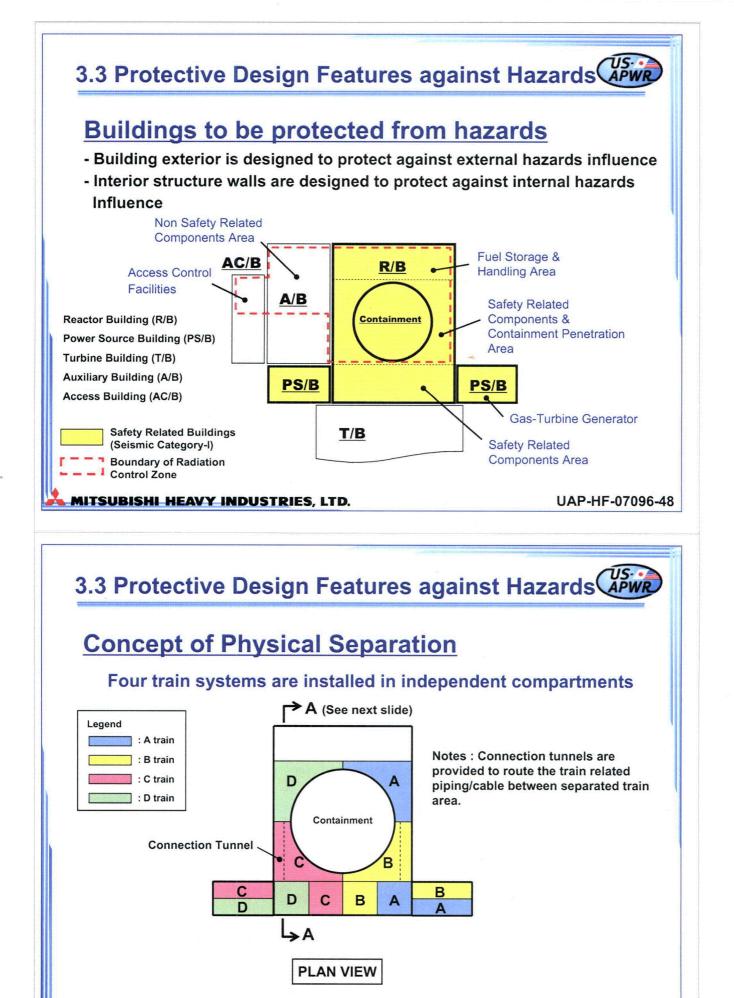
- Safety components are located in safety buildings

### **Prevent Water Immersion against External Flood**

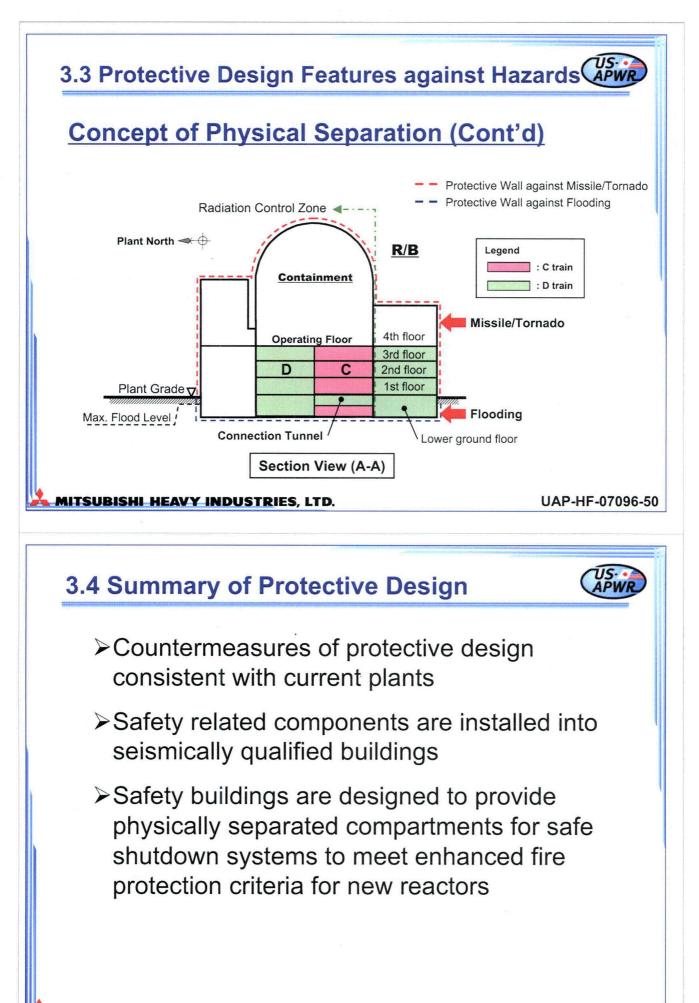
- Exterior openings/penetrations are appropriately sealed

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