

October 31, 2007

Mr. Michael D. Wadley  
Site Vice President  
Prairie Island Nuclear Generating Plant  
Nuclear Management Company, LLC  
1717 Wakonade Drive East  
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 - ISSUANCE  
OF AMENDMENTS RE: (TAC NOS. MD5544 AND MD5545)

Dear Mr. Wadley:

The Commission has issued the enclosed Amendment No. 181 to Facility Operating License No. DPR-42 and Amendment No. 171 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant (PINGP), Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 10, 2007.

The requested changes are a partial adoption of Technical Specification Task Force (TSTF-491), Revision 2, "Removal of Main Steam and Feedwater Valve Isolation Times" which was proposed by the TSTF by letter on May 18, 2006. The proposed changes revise TS 3.7.2 "Main Steam Valves Closure Times" by relocating the isolation valve closure times to a licensee-controlled document identified as a Bases reference. The proposed amendments deviate from TSTF-491 in that the current PINGP TS (3.7.3) and associated surveillance requirements for the main feedwater isolation valves do not include valve closure times, and thus the changes to TS 3.7.3 provided for in TSTF-491 are not applicable to the PINGP TSs and are not adopted.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

*/RA/*

Mahesh L. Chawla, Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 181 to DPR-42
2. Amendment No. 171 to DPR-60
3. Safety Evaluation

cc w/encls: See next page

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NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 181  
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 10, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 181, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Cliff G. Munson, Acting Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License  
and Technical Specifications

Date of Issuance: October 31, 2007

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171  
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 10, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 171, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Cliff G. Munson, Acting Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License  
and Technical Specifications

Date of Issuance: October 31, 2007

ATTACHMENT TO LICENSE AMENDMENT NOS. 181 AND 171

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Facility Operating License No. DPR-42 and DPR-60 with the attached revised pages. The changed areas are identified by a marginal line.

REMOVE

DPR-42, License Page 3  
DPR-60, License Page 3

INSERT

DPR-42, License Page 3  
DPR-60, License Page 3

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE

3.7.2-2

INSERT

3.7.2-2

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NMC to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to transfer byproduct materials from other job sites owned by Northern States Power Company for the purpose of volume reduction and decontamination.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NMC is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 181, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

NMC shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Prairie Island Nuclear Generating Plant Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program," Revision 1, submitted by letters dated October 18, 2006, and January 10, 2007.

- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to transfer byproduct materials from other job sites owned by Northern States Power Company for the purposes of volume reduction and decontamination.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NMC is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 171, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications. |

(3) Physical Protection

NMC shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Prairie Island Nuclear Generating Plant Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program," Revision 1, submitted by letters dated October 18, 2006, and January 10, 2007.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 181 TO FACILITY OPERATING LICENSE NO. DPR-42  
AND AMENDMENT NO. 171 TO FACILITY OPERATING LICENSE NO. DPR-60  
NUCLEAR MANAGEMENT COMPANY, LLC  
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated May 10, 2007, (Agencywide Document and Management System (ADAMS) Accession No. ML071300622), Nuclear Management Company, LLC (NMC), (the licensee), proposed changes to the technical specifications (TS) for Prairie Island Nuclear Generating Plant (PINGP) Units 1 and 2. The requested changes are a partial adoption of Technical Specification Task Force (TSTF) -491, Revision 2, "Removal of Main Steam and Feedwater Valve Isolation Times" which was proposed by the TSTF by letter on May 18, 2006. The proposed changes revise TS 3.7.2 "Main Steam Valves Closure Times" by relocating the isolation valve closure times to a licensee-controlled document identified as a Bases reference. The proposed amendments deviate from TSTF-491 in that the current PINGP TS (3.7.3) and associated surveillance requirements for the main feedwater isolation valves do not include valve closure times, and thus the changes to TS 3.7.3 provided for in TSTF-491 are not applicable to the PINGP TSs and are not adopted. TSTF-491, Revision 2, was announced for availability in the *Federal Register* on December 29, 2006, as part of the consolidated line item improvement process (CLIIP).

2.0 REGULATORY EVALUATION

Section 182(a) of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The TS ensure the operational capability of structures, systems and components that are required to protect the health and safety of the public. The Commission's regulatory requirements related to the content of the TS are contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36. That regulation requires that the TS include items in the following specific categories: (1) safety limits, limiting safety systems settings, and limiting control settings (50.36(c)(1)); (2) limiting conditions for operation (50.36(c)(2)); (3) surveillance requirements (50.36(c)(3)); (4) design features (50.36(c)(4)); and (5) administrative controls (50.36(c)(5)).

In general, there are two classes of changes to TS: (1) Changes needed to reflect modifications to the design basis (TS are derived from the design basis), and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TS over time. This amendment deals with the second class of changes. In determining the acceptability of revising Standard Technical Specifications (STS) 3.7.2, the

Nuclear Regulatory Commission (NRC) staff used the accumulation of generically-approved guidance in NUREG-1431, Revision 3, "Standard Technical Specifications, Westinghouse Plants," dated June 2004. Licensees may revise the TS to adopt current improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards.

The Commission's regulatory requirements regarding codes and standards are specified in 10 CFR 50.55a, which incorporates portions of industry codes and standards, such as the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, into the regulations and the associated requirements for Inservice Testing (IST). Much of the information associated with a licensee's implementation of the testing and surveillance requirements in 10 CFR 50.55a, is contained in their IST Program. The 10 CFR 50.59 criteria provide adequate assurance that prior staff review and approval will be requested by the licensee for changes to the Bases or Licensee-Controlled Document (LCD) requirements with the potential to affect the safe operation of the plant.

The detailed application of this general framework, and additional specialized guidance, are discussed in Section 3.0 in the context of specific proposed changes. Nomenclature specific to the Westinghouse Plants is used in the following technical evaluation.

### 3.0 TECHNICAL EVALUATION

The NRC staff has reviewed the justification for the proposed TSTF as described in the September 13, 2005, submittal. The detailed evaluation below will support the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

#### 3.1 Main Steam Isolation Valves (MSIV)

One MSIV is located in each main steamline outside of the containment. Closing the MSIVs isolates each steam generator from the others and isolates the turbine, steam bypass system, and other auxiliary steam supplies from the steam generator. By isolating the steam flow from the secondary side of the steam generator, the MSIVs prevent overcooling the reactor core following a high-energy line break. By preventing core overcooling the MSIVs protect the reactor core from being damaged. TSTF-491 is proposing to relocate the required closure times for the MSIVs to the LCD, "Technical Requirements Manual," that is referenced in the Bases. Changes to the LCD are subject to the 10 CFR 50.59 process. The 10 CFR 50.59 criteria provide adequate assurance that prior NRC staff review and approval will be requested by the licensee for changes to the Bases or LCD requirements with the potential to affect the safe operation of the plant. Furthermore, the MSIVs are subject to periodic testing and acceptance criteria in accordance with the IST Program. Compliance with the IST Program is required by Section 5.5.7 of the STS and 10 CFR 50.55a. The IST Program includes specific reference value baseline operating times for valves that are not subject to arbitrary changes.

The regulation at 10 CFR 50.36 requires the inclusion of the periodic testing of the MSIVs in the surveillance requirements, not the actual closure time of the valves. TSTF-491 change maintains the periodic testing requirements for MSIVs in accordance with 10 CFR 50.36. Based on the requirements of 10 CFR 50.36, 10 CFR 50.59 and IST Program, the NRC staff concludes that relocating the MSIV closure time to the LCD as referenced in the Bases is acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes the requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (72 FR 39083). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Jason Lising

Date: October 31, 2007

Prairie Island Nuclear Generating Plant,  
Units 1 and 2

cc:

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June 2006