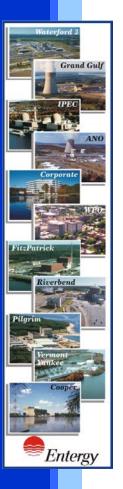
James A. FitzPatrick Nuclear Power Plant

ACRS License Renewal Subcommittee Presentation September 5, 2007



James A. FitzPatrick Personnel in Attendance



Brian Finn John McCann Garry Young

Steve Bono Joe Pechacek James Costedio Alan Cox Rick Plasse Larry Leiter Tom Moskalyk Arturo Smith Site NSA Director Director of Licensing, White Plains Manager, License Renewal

Director of Engineering Manager, Programs & Components Licensing Manager Technical Manager Licensing Lead Technical Lead Structural Lead Class 1 Mechanical Lead

Technical Support Personnel

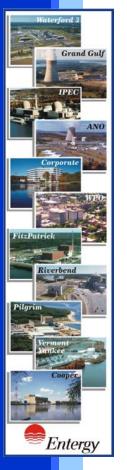
Agenda

- James A. FitzPatrick Site Description
- Current Status

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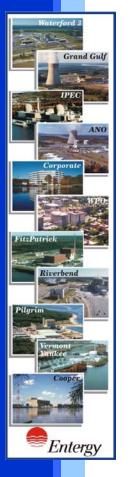
- James A. FitzPatrick Licensing History & Highlights
- License Renewal Project
- Cost-Beneficial SAMAs
- Presentation Topics
 - Drywell and Torus Monitoring
 - Torus Repair
- Questions

JAFNPP Site Description



- General Electric (NSSS & TG), Stone & Webster (AE and Constructor)
- BWR-4, Mark I Containment
- 2536 MWt Thermal Power; ~ 881 MWe
- Once through cooling from Lake Ontario
- Staff Complement: approximately 650

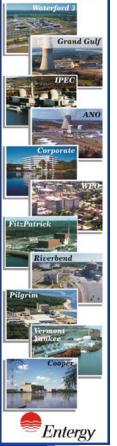
JAFNPP Plant Status



- Startup from RFO 17 November 4, 2006
- Current Plant Status

Next outage Sept 2008

Licensing History



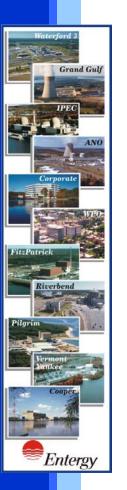
đ	Construction Permit	May 20, 1970
	Operating License	October 17, 1974
A REAL PROPERTY AND INC.	Commercial Operation	July 28, 1975
	Uprated Power License (4%)	December 6, 1996
	License Transfer to Entergy	November 21, 2000
	LR Application Submitted	July 31, 2006
	Operating License Expires	October 17, 2014

Major Improvements

Waterford 3 Grand Gulf	
Corporate WPO:	
FitzPatrick	
Pilgrim Vermont	
Entergy	
8)	

1978-1983	Mark I Containment Modifications
1988	Hydrogen Water Chemistry
1989	Zinc Injection
1990	Power Uprate Equipment Upgrades
1998	ECCS Suction Strainers Replaced
1999	Noble Metals Application
2004	LP Turbine Rotor Replacement
2004	Noble Metals Application 2
2006	HP Turbine Rotor Replacement
2006	Offgas Condenser Replacement
2006	HPCI Discharge Exhaust Sparger Added

Future Improvements



Main Transformer Replacement

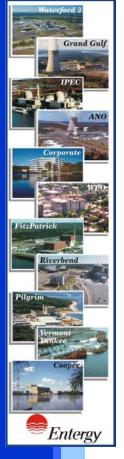
2008

2008

Core Spray Motor Replacement

2008

345KV Breaker Replacement



- LRA Prepared by experienced, multi-discipline Entergy team (utilized corporate and on-site resources)
- Incorporated lessons learned from previous applications
- Peer review conducted
- LRA internal reviews (Safety Review Committees and QA)
- All comments resolved prior to submittal

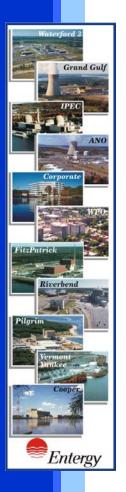
- License Renewal Commitments
 - Refined during audit/inspection process
 - Tracked by Entergy commitment tracking and engineering work tracking systems
- 36 Aging Management Programs
 - 17 Programs in Place w/o Enhancements
 - 9 Programs will be Enhanced
 - 10 New Programs
- GALL Consistancy

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- 10 Consistent
- 20 Consistent with exceptions / enhancements
- 6 Plant Specific

Program Implementation Plan

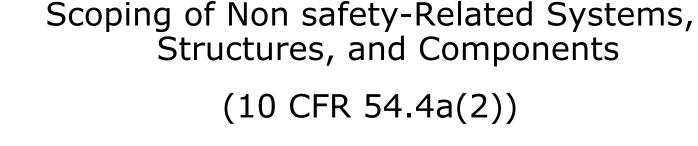
- Develop fleet approach for Entergy plants that have submitted an LRA
- Develop schedule using industry experience



Scoping of Non safety-Related Systems, Structures, and Components

(10 CFR 54.4a(2))

- Utilized site component database, P&IDs, and isometric drawings
- Reviewed safety related cable / piping locations
- Performed walkdowns for a(2) scope verification



- Regional Inspection verified a(2) scoping for in-plant areas and systems
- 10 CFR 54.4a(2) scope changes made in LRA Amendment 11
- Regional Inspection concluded that JAF had implemented an acceptable method of scoping and screening of non-safety related SSCs and that this method resulted in accurate scoping determinations

Draft SER Summary

• Open Items – 2

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- Reactor Vessel Fluence
- Environmentally Assisted Fatigue
- Confirmatory Items None

Reactor Vessel Neutron Fluence

- Current P-T curves valid through 2014 (32EFPY)
- Submit RG 1.190 calculations by September 2007
- Evaluated TLAAs to determine limiting fluence (RG 1.99)
 - Adjusted Reference Temperatures (<200F)
 - Upper Shelf Energy (>50 ft-lb)
 - RPV welds

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- RPV nozzles near beltline
- Axial Weld Failure Probability is limiting at 5x10⁻⁶ per Reactor Year
- ART and USE values will not be challenged at 54 EFPY

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Entergy

Environmentally Assisted Fatigue

- JAF will demonstrate that cumulative usage factors (CUF) of the most fatigue sensitive locations are less than 1.0 throughout the license renewal period by first using Option (1) of commitment #20
- Analysis methods for determination of stresses and fatigue usage will be in accordance with NRC endorsed ASME Boiler and Pressure Vessel Code
- JAF will utilize design transient specifications and information from BWR-4 references to bound operational transients

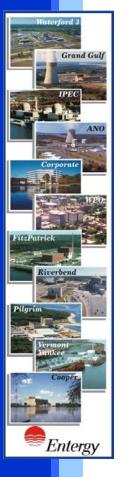
Environmentally Assisted Fatigue

- Environmental effects on fatigue usage will be assessed consistent with the Generic Aging Lessons Learned Report, NUREG-1801, Rev. 1.
- If Option (2) becomes necessary, plant inspection program will be described in terms of the ten elements specified in Branch Technical Position RLSB-1.
- If Option (3) becomes necessary, repair or replacement will be in accordance with plant procedures that meet ASME Section XI requirements.
- Above actions will be incorporated into the Fatigue Monitoring Program.

Entergy

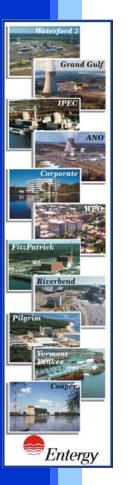
• The Fatigue Montoring Program will manage the effects of EAF in accordance with 10 CFR 54.21(c)(1)(iii).

Cost-Beneficial SAMAs



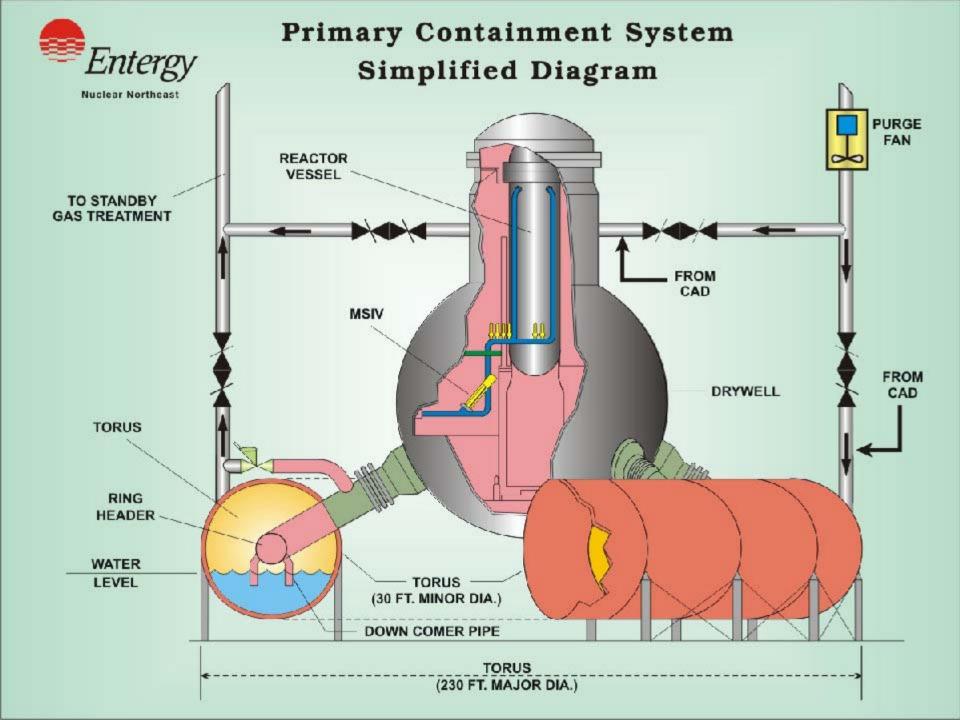
- Six Potentially Cost-Beneficial SAMAs Identified
- No Age-Related SAMAs
- Implementation will be evaluated using the plant cost-benefit analysis process

Presentation Topics

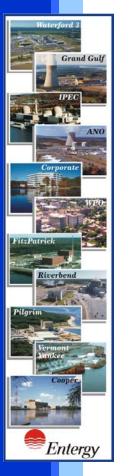


• Drywell and Torus Monitoring

• Torus Repair

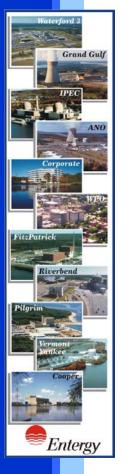


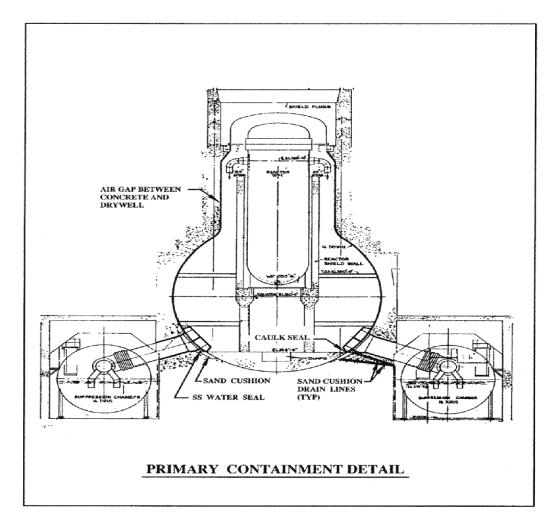
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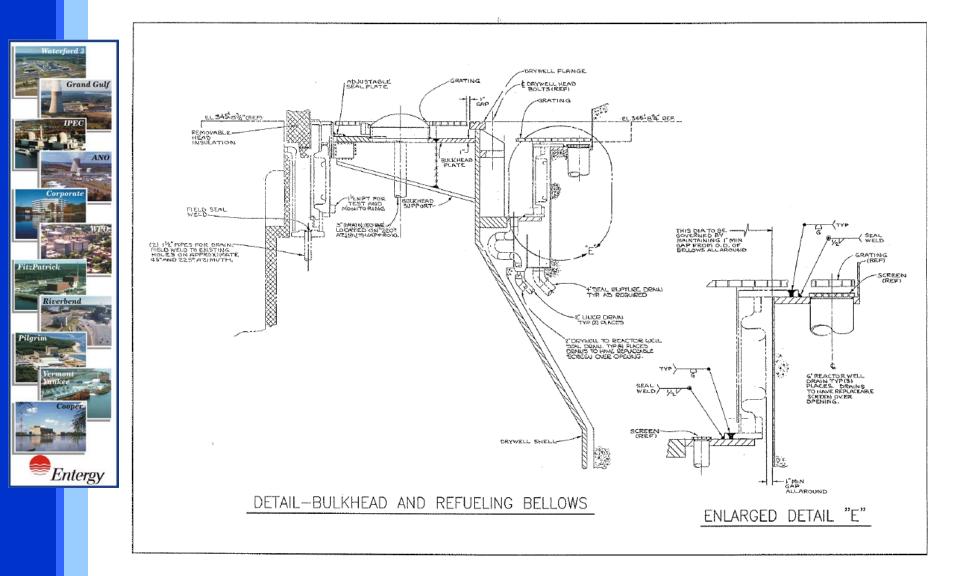
Drywell and Torus Monitoring

James A. FitzPatrick

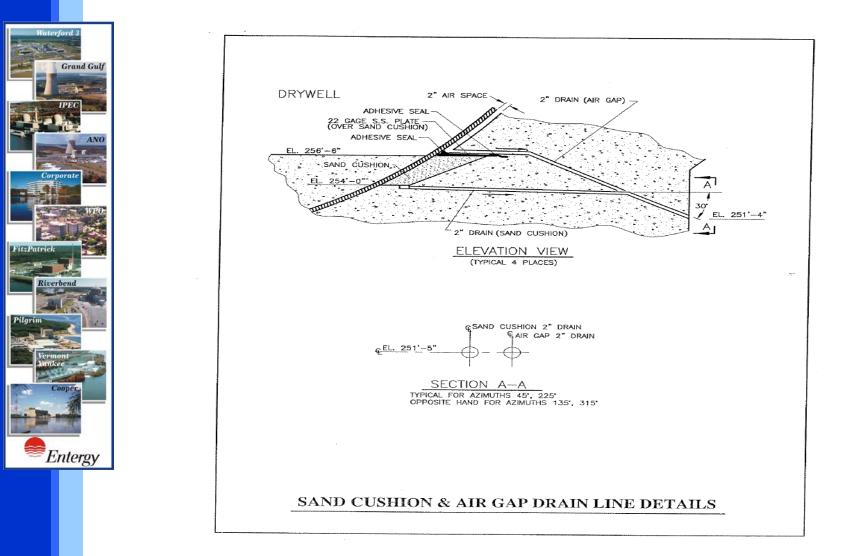




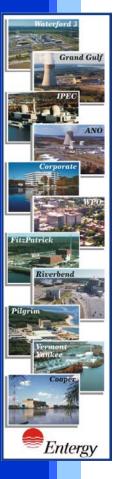
James A. FitzPatrick



James A. FitzPatrick

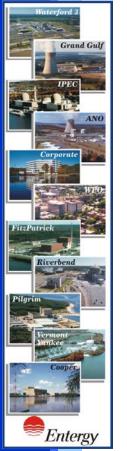


Drywell Monitoring



- Sand Cushion Inspections. No Evidence of Moisture (Boroscopic Inspection).
- Visual Inspection of Interior Drywell Caulk Seal.
- Drywell Interior Coating System (Carbozinc 11 and Dupont Corlar Epoxy) Inspection IAW IWE Program during RFO.

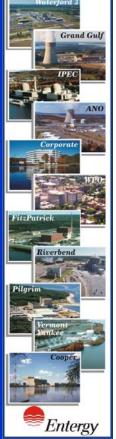
Torus Monitoring



- Torus Interior Shell Inspection 1998 (Installation of ECCS Suction Strainers).
- Torus Interior Coating System (Carbozinc 11) inspected.
- Torus Interior/Exterior Inspected IAW JAF IWE Program.

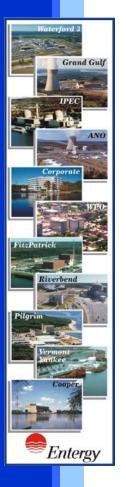


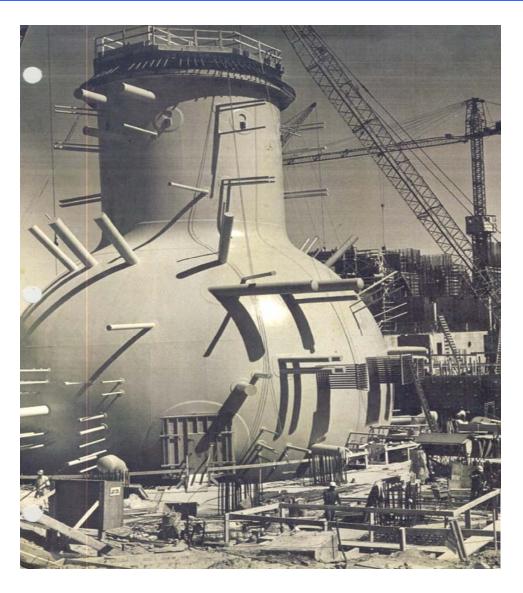




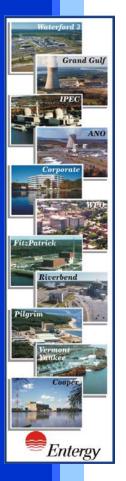




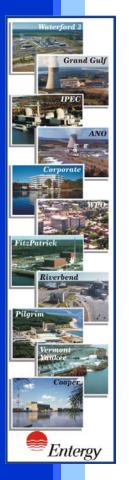




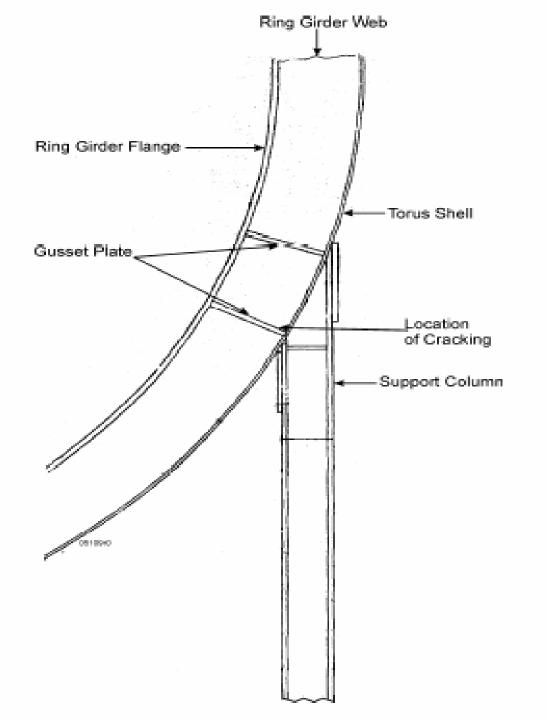
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Torus Repair



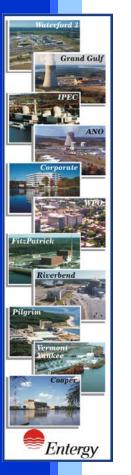
- Torus shell through-wall leak reported in June 2005
- Leak was located in same bay as HPCI Steam Discharge pipe near ring girder gusset plate weld
- ASME Section XI code repair performed in July 2005 by removing the flaw and adding a circular repair plate
- Root cause of flaw was vibration fatigue from HPCI steam condensation oscillation loading



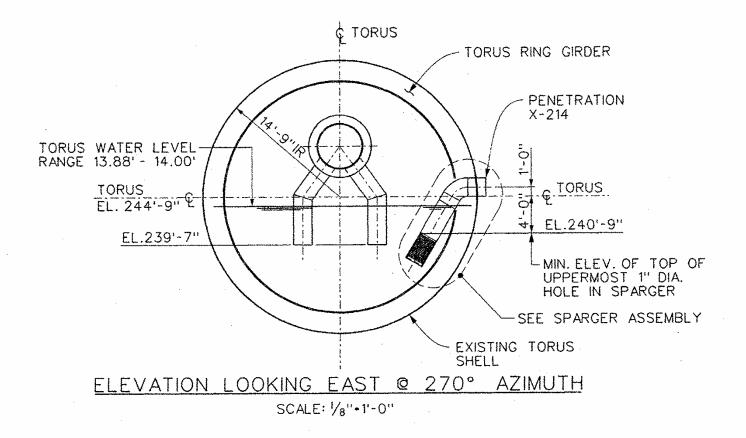
HPCI Open End Discharge

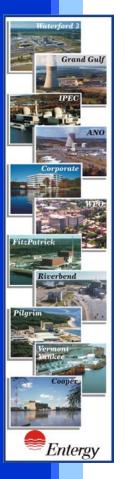
Ring Girder Gusset Plate **Torus Indication 2006**

Torus Leak 2005

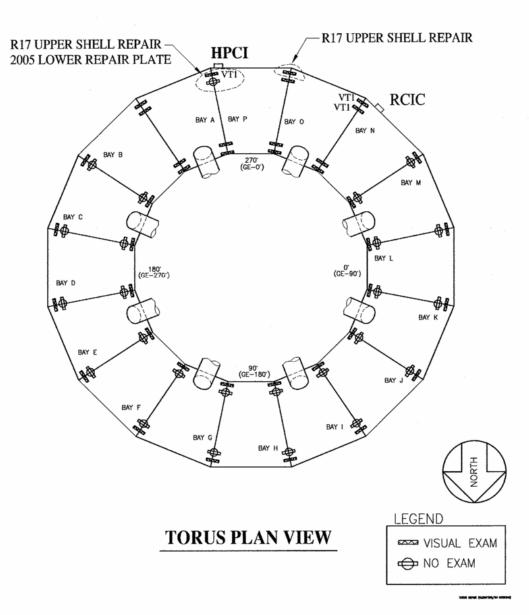


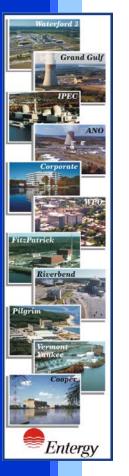
- A HPCI Steam Exhaust Sparger assembly was added during refueling outage October 2006
- The sparger directs steam flow away from the Torus shell
- The sparger significantly reduces loads on the Torus shell from HPCI Steam condensation oscillation





- Extent of condition actions from Root Cause required additional shell exams during refueling outage October 2006
- ASME visual exams of similar ring girder gusset welds performed at HPCI and RCIC steam discharge locations
- General visual exams of similar ring girder gusset welds performed at several locations throughout the Torus
- Exam results reported shell base metal flaws at two additional locations in the HPCI discharge bay





 ASME Section XI code repairs were performed by grinding to remove the flaws and welding to restore configuration

 Review was performed to confirm the HPCI steam discharge loading also caused these flaws

Comments and Questions

