From: Johnny Hughey
To: thomas.loomis@exeloncorp.com
Date: 9/21/2007 10:58:17 AM
Subject: DRAFT RAI For MD6701/6702

Tom:
NRC technical staff has provided the attached draft Request for Additional Information (RAI) related to the Exelon submittal dated September 10, 2007, regarding request for relief from ASME Code requirements for test frequency (TACs MD6701/6702). After reviewing this draft RAI, please contact me at 301-415-3204 to discuss the need for a teleconference concerning the RAI and to discuss a due date for the response.

Thank you,
John Hughey, PM
NRR/DORL/LPL1-2
U.S. Nuclear Regulatory Commission
Mail Stop #: O-8B3
Phone #: 301-415-3204
E-Mail jdh4@nrc.gov

DRAFT

REQUEST FOR ADDITIONAL INFORMATION

REGARDING REQUESTS FOR RELIEF FROM ASME OM CODE

5-YEAR TEST INTERVAL FOR SAFETY RELIEF VALVE/SAFETY VALVES

PEACH BOTTOM ATOMIC POWER STATION, UNITS NO. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

By letter dated September 10, 2007, Exelon Generation Company, LLC, submitted a Relief Requests regarding the ASME OM Code 5-year test interval for four Safety Relief Valve/Safety Valves (SRVs/SVs) currently installed at Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. The Nuclear Regulatory Commission (NRC) staff has reviewed the request for relief the licensee provided in the September 10, 2007, submittal. In order for the NRC staff to complete its evaluation, response to the following request for additional information (RAI) questions is requested.

RAI-01) The relief request submittal does not discuss the impact of storage in a controlled environment on SRV/SV set pressure. Please provide test results for Target Rock Model 67F and Dresser Model 3707RA SRVs/SVs that were maintained in a controlled environment for an extended period of time and then setpoint tested prior to installation. As an alternative, provide the as-found set-pressure test results for SRVs/SVs (Target Rock Model 67F and Dresser Model 3707RA) that were stored in a controlled environment similar to the controlled environment at Peach Bottom Atomic Power Station for an extended period of time, placed in service for approximately 4 years, and then tested.

RAI-02) Please verify that the current Peach Bottom Atomic Power Station licensing basis assumes that the SRVs/SVs open to relieve pressure at the upper limit of 3% of the opening set pressure. Please explain why the relief request submittal uses the ASME OM Code as-found acceptance criteria of ±3% of set pressure in the justification for extending the test interval beyond 5 years instead of the ±1% of set pressure specified in the technical specifications for the SRVs/SVs.

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From: Johnny Hughey

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