RAS 14088

UNITED STATES
NUCLEAR REGULATORY COMMISSION

DOCKETED
USNRC

September 4, 2007 (5:00 pm)

OFFICE OF THE SECRETARY
RULEMAKINGS AND
ADJUDICATIONS STAFF

In the Matter of	)	
	)	
ENTERGY NUCLEAR VERMONT YANKEE, LLC	)	Docket No. 50-271-LR
and ENTERGY NUCLEAR OPERATIONS, INC.	)	ASLB No. 06-849-03-LR
	)	
Vermont Yankee Nuclear Power Station	)	

NEW ENGLAND COALITION, INC.'S (NEC) MOTION TO FILE A TIMELY NEW OR AMENDED CONTENTION

The New England Coalition, Inc. (NEC) moves, pursuant to 10 C.F.R. § 2.309(f)(2) and the Initial Scheduling Order ¶ 5, for leave to file a timely new or amended contention addressing Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc.'s ("Entergy") recent reanalysis of environmentally assisted metal fatigue. Entergy produced its final reports of this reanalysis, attached hereto as Exhibits A-O to the Sixth Declaration of Dr. Joram Hopenfeld, to NEC on August 3, 2007.[1] *See*, Attachment 1.

NEC's proposed new or amended contention satisfies requirements of 10 C.F.R. § 2.309(f)(2). Entergy's new analysis of environmentally assisted metal fatigue is materially different from the analysis of this phenomenon reported in Entergy's License Renewal Application. Entergy employed different methods, and produced different results. *See,* Attachment 2, Sixth Declaration of Dr. Joram Hopenfeld. Information concerning Entergy's final reanalysis was not available to NEC until Entergy produced

[1] Entergy produced preliminary reports of this reanalysis to NEC on June 7 and June 13, 2007. Per the Board's Order issued during the June 12, 2007 prehearing scheduling conference, NEC filed a Motion to File a Timely New or Amended Contention based on the preliminary reports on July 12, 2007. Both NEC and Entergy have requested that the Board hold this motion in abeyance pending NEC's response to Entergy's final reanalysis reports.

TEMPLATE= SECY-037

SECY-02

its reports on August 3, 2007. *See,* Attachment 1. NEC's filing is timely. Pursuant to 10 C.F.R. §2.306 and the Initial Scheduling Order in this proceeding, the deadline for NEC's new or amended contention addressing Entergy's final reanalysis is September 4, 2007. *See,* Initial Scheduling Order ¶ 5 (a motion for a new or amended contention shall be deemed timely under 10 C.F.R. § 2.309(f)(2)(iii) if it is filed within thirty (30) days of the date when the new and material information on which it is based first becomes available); 10 C.F.R. § 2.306 ("In computing any period of time . . . . the last day of the period so computed is included unless it is a Saturday, Sunday, or legal holiday . . ., in which event the period runs until the end of the next day which is neither a Saturday, Sunday, nor holiday.").

PROPOSED NEW OR AMENDED CONTENTION

NEC incorporates by reference its now pending Contention 2 (metal fatigue) and its Reply to Entergy's Answer to Contention 2. In addition, NEC contends the following.

(1) Statement of Issue of Law or Fact to be Raised, and Brief Explanation of Basis. 10 C.F.R. §§ 2.309(f)(i), 2.309(f)(ii). NEC's now pending Contention 2 (metal fatigue) is that data reported in Entergy's License Renewal Application Table 4.3-3 indicate that critical reactor components may fail due to environmentally assisted metal fatigue during the period of extended operation, and that Entergy has not proposed an adequate aging management plan addressing this issue as required pursuant to 10 C.F.R. § 54.21. NEC understands that Entergy has now completed a final reanalysis of the impact of environmentally assisted metal fatigue, the results of which purportedly indicate that the reactor components listed in License Renewal Application Table 4.3-3 are not in fact subject to fatigue failure. NEC further understands that the reports and

calculations attached hereto as Exhibits A-O to the Sixth Declaration of Dr. Joram Hopenfeld describe the methods and results of this reanalysis (hereinafter termed the "CUFen Reanalysis").

NEC now contends, as explained in detail in the attached Sixth Declaration of Dr. Joram Hopenfeld, that the analytical methods employed in Entergy's CUFen Reanalysis were flawed by numerous uncertainties, unjustified assumptions and insufficient conservatism, and produced unrealistically optimistic results. *See,* Attachment 2, Sixth Declaration of Dr. Joram Hopenfeld. Entergy has not, by this flawed reanalysis, demonstrated that the reactor components assessed will not fail due to metal fatigue during the period of extended operation. *Id.* NEC's Contention 2 concern regarding Entergy's failure to propose an adequate aging management plan consistent with the intent of 10 C.F.R. § 54.21 remains current and valid.

(2) Scope of the Proceeding and Materiality. 10 C.F.R. §§ 2.309(f)(iii), 2.309(f)(iv). This contention addresses Entergy's plan, as stated in the License Renewal Application, to monitor and manage the effects of aging on reactor components that are subject to an aging management review, pursuant to 10 C.F.R. § 54.21(a), and an evaluation of time-limited aging analysis, pursuant to 10 C.F.R. § 54.21(c). These are issues within the scope of this proceeding, and material to findings the NRC must make in this matter. *See,* 10 C.F.R. § 54.4; *Duke Energy Corp. (McGuire Nuclear Station, Units 1 and 2; Catawba Nuclear Station, Units 1, 2 and 3),* 56 NRC 358, 363-64 (2002).

(3) Expert or Factual Support. 10 C.F.R. § 2.309(f)(v). This contention is supported by the attached Declaration of NEC's expert witness, Dr. Joram Hopenfeld.

(4) Genuine Dispute of Material Law or Fact. 10 C.F.R. § 2.309(f)(vi). The attached Sixth Declaration of Dr. Joram Hopenfeld includes ample information to establish a genuine dispute with the Applicant concerning the validity of the CUFen Reanalysis. NEC is required to make only "a minimal showing that the material facts are in dispute, thereby demonstrating that an inquiry in depth is appropriate." *In Gulf State Utilities Co.,* 40 NRC 43, 51 (1994).

NEC HAS CONSULTED OTHER PARTIES

Pursuant to 10 C.F.R. § 2.323(b), NEC has consulted or attempted to consult with all parties to this proceeding concerning this motion. Entergy and the State of Vermont do not object to the filing of this motion, and will respond to its content after reviewing NEC's pleading. The NRC Staff had not responded as of the date and time of this filing. The State of New Hampshire did not take a position.

CONCLUSION

NEC respectfully requests that the Board grant NEC's Motion to File a Timely New or Amended Contention, and admit NEC's new or amended contention for adjudication in this proceeding.

September 4, 2007 New England Coalition

by: Karen Tyler

Andrew Raubvogel
Karen Tyler
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For the firm

Attorneys for NEC

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

Before the Atomic Safety and Licensing Board

In the Matter of	)	
	)	
Entergy Nuclear Vermont Yankee, LLC	)	Docket No. 50-271-LR
and Entergy Nuclear Operations, Inc.	)	ASLBP No. 06-849-03-LR
	)	
(Vermont Yankee Nuclear Power Station)	)	

CERTIFICATE OF SERVICE

I, Michelle Cronin, hereby certify that copies of NEW ENGLAND COALITION, INC.'S (NEC) MOTION TO FILE A TIMELY NEW OR AMENDED CONTENTION in the above-captioned proceeding were served on the persons listed below, by U.S. Mail, first class, postage prepaid; by Fed Ex overnight to Judge Elleman; and, where indicated by an e-mail address below, by electronic mail, on the 4th day of September, 2007.

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ATTACHMENT 1

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August 2, 2007

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In the Matter of
Entergy Nuclear Vermont Yankee, LLC, and Entergy Nuclear Operations, Inc.
(Vermont Yankee Nuclear Power Station)
Docket No. 50-271-LR; ASLBP No. 06-849-03-LR

Re: Structural Integrity Associates Final Fatigue Analysis Reports

Dear Mesdames Baty, Hofmann and Tyler:

On June 7 and 13, 2007, Entergy provided copies of several reports containing preliminary Vermont Yankee site-specific calculations of environmentally assisted fatigue of critical components relevant to New England Coalition's Contention 2. Those calculations have now been finalized and reports describing the calculations and their results are enclosed herewith in a compact disc. Listed in the Attachment to this letter are

the materials being provided. Entergy will supply in the future production numbers for these reports.

Please note that three of the documents included herewith contain proprietary information. They are Calculation No. VY-16Q-303 Rev. 0 and reports SIR-07-130-NPS Rev. 0 (File VY-16Q-401) and SIR-07-132-NPS Rev.0 (File-VY-16Q-404). Unredacted copies of those documents are contained in the compact discs being provided to the New England Coalition and the Department of Public Service. We request that they be treated in accordance with provisions of the Board's Protective Order (January 12, 2007), be protected from disclosure to unauthorized persons, and be made available for review to only those individuals who have executed a Non-Disclosure Agreement. The copies of these documents being provided to the NRC Staff have been redacted to delete the proprietary information.

Sincerely,

Matias F. Travieso-Diaz

Matias F. Travieso-Diaz
Counsel for Entergy

Enclosures (as noted)

ATTACHMENT

Final Structural Integrity Associates Calculations & Reports for VY Being Provided

Structural Integrity Calculation or Report No.	Title	.pdf file name
Calculation File No. VY-16Q-301, Rev. 0	Feedwater Nozzle Stress History Development for Green Functions	VY-16Q-301R0.pdf
Calculation File No. VY-16Q-302, Rev. 0	Fatigue Analysis of Feedwater Nozzle	VY-16Q-302R0.pdf
Calculation File No. VY-16Q-303, Rev. 0 P	Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head	VY-16Q-303R0.pdf
Calculation File No. VY-16Q-304, Rev. 0	Recirculation Outlet Nozzle Finite Element Model	VY-16Q-304R0.pdf
Calculation File No. VY-16Q-305, Rev. 0	Recirculation Outlet Stress History Development for Nozzle Green Function	VY-16Q-305R0.pdf
Calculation File No. VY-16Q-306, Rev. 0	Fatigue Analysis of Recirculation Outlet Nozzle	VY-16Q-306R0.pdf
Calculation File No. VY-16Q-307, Rev. 0	Recirculation Class 1 Piping Fatigue and EAF Analysis	VY-16Q-307R0.pdf
Calculation File No. VY-16Q-308, Rev. 0	Core Spray Nozzle Finite Element Model	VY-16Q-308R0.pdf
Calculation File No. VY-16Q-309, Rev. 0	Core Spray Nozzle Green's Functions	VY-16Q-309R0.pdf
Calculation File No. VY-16Q-310, Rev. 0	Fatigue Analysis of Core Spray Nozzle	VY-16Q-310R0.pdf
Calculation File No. VY-16Q-311, Rev. 0	Feedwater Class 1 Piping Fatigue Analysis	VY-16Q-311R0.pdf

Structural Integrity Calculation or Report No.	**Title**	**.pdf file name**
Report No. SIR-07-130-NPS, Rev. 0 File No. VY-16Q-401	Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Feedwater Nozzles	VY-16Q-401R0.pdf
Report No. SIR-07-141-NPS, Rev .0 File No. VY-16Q-402	Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Reactor Recirculation Outlet Nozzle	VY-16Q-402R0.pdf
Report No. SIR-07-138-NPS, Rev. 0 File No. VY-16Q-403	Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Core Spray Nozzle	VY-16Q-403R0.pdf
Report No. SIR-07-132-NPS, Rev.0 File No. VY-16Q-404	Summary Report of Plant Specific Environmental Fatigue Analyses for the Vermont Yankee Nuclear Power Station	VY-16Q-404R0.pdf

ATTACHMENT 2

UNITED STATES
NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
	)	
ENTERGY NUCLEAR VERMONT YANKEE, LLC	)	Docket No. 50-271-LR
and ENTERGY NUCLEAR OPERATIONS, INC.	)	ASLB No. 06-849-03-LR
	)	
Vermont Yankee Nuclear Power Station	)	

SIXTH DECLARATION OF DR. JORAM HOPENFELD

1. My name is Dr. Joram Hopenfeld. The New England Coalition (NEC) has retained me as an expert witness in proceedings concerning the application of Entergy Nuclear Operations, Inc. ("Entergy") to renew its operating license for the Vermont Yankee Nuclear Power Station ("VYNPS") for twenty years beyond the current expiration date of March 21, 2012.

2. I am a mechanical engineer and hold a doctorate in engineering. I have 45 years of professional experience in the fields of instrumentation, design, project management, and nuclear safety, including 18 years in the employ of the U.S. Nuclear Regulatory Commission (NRC). My curriculum vitae was previously filed in this proceeding as an attachment to my declaration in support of NEC's Petition to Intervene.

REFERENCES

3. I have reviewed the VYNPS License Renewal Application, submitted to the NRC in 2006. I have also reviewed the following Structural Integrity Associates, Inc. (SIA)

Environmental Fatigue Analysis Calculations and Reports, which I understand that Entergy produced to New England Coalition (NEC) on August 3, 2007.

SIA Calculation or Report No.	Title
Calculation File No. VY-16Q-301	Feedwater Nozzle Stress History Development for Green Functions
Calculation File No. VY-16Q-302	Fatigue Analysis of Feedwater Nozzle
Calculation File No. VY-16Q-304	Recirculation Outlet Nozzle Finite Element Model
Calculation File No. VY-16Q-305	Recirculation Outlet Stress History Development for Nozzle Green Function
Calculation File No. VY-16Q-306	Final Analysis of Recirculation Outlet Nozzle
Calculation File No. VY-16Q-307	Recirculation Class 1 Piping Fatigue and EAF Analysis
Calculation File No. VY-16Q-308	Core Spray Nozzle Finite Element Model
Calculation File No. VY-16Q-309	Core Spray Nozzle Green's Functions
Calculation File No. VY-16Q-310	Fatigue Analysis of Core Spray Nozzle
Calculation File No. VY-16Q-311	Feedwater Class 1 Piping Fatigue Analysis
Report No.: SIR-07-141-NPS File No.: VY-16Q-402	Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Reactor Recirculation Outlet Nozzle
Report No.: SIR-07-138-NPS File No.: VY-16Q-403	Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Core Spray Nozzle

Copies of these calculations and reports are attached hereto as Exhibits A, B, D-K, M, and N.

4. I have also reviewed redacted versions of the following one additional SIA calculation, and two additional SIA reports, which I understand that Entergy produced to NEC on August 3, 2007: (1) Calculation No. VY-16Q-303, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head;" (2)

Report No.: SIR-07-130-NPS, File No. VY-16Q-401, "Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Feedwater Nozzles;" and (3) Report No. SIR-07-132-NPS, File No. VY-16Q-404, "Summary Report of Plant-Specific Environmental Fatigue Analyses for the Vermont Yankee Nuclear Power Station." Redacted copies of this calculation and these reports are attached hereto as Exhibits C, L and O.

5. I have also reviewed the following NUREG reports: NUREG/CR-6260, NUREG CR-5999, and NUREG/CR-5754. Finally, I have reviewed the following documents, hereinafter cited in this Declaration as References 1-8:

1. NUREG/CR-6909, "Effect of LWR Coolant Environment on Fatigue Life of Reactor Materials" (Final Report), ANL -06/08 U.S. NRC, Wash., D.C. Feb. 2007.

2. Y.Y. Chen et.al., "Factors Affecting the Crack Growth Rates of Reactor Pressure Vessel Steels Under Simulated Boiling Water Reactor Conditions," Corrosion, Vol. 62, No 5, 2006.

3. Makoto Higuchi, "Revised Proposal of Fatigue Life Correction Factor Fen for Carbon and Low Alloy Steels in LWR Water Environments," Transactions of the ASME, V. 1126 Nov. 2004.

4. M. Itatani, "Fatigue Crack Growth Curves for Austenitic Stainless Steels in BWR Environments," Transaction of the ASME Vol. 123, May 2001.

5. NUREG/CR-6583, "Effect of LWR Coolant Environment on Fatigue-Design Curves of Carbon and Low Alloy Steels," March 1998.

6 NUREG/CR 5704, "Effect of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steel," April 1999.

7. NUREG/CR6936, "Probability of Failure and Uncertainty Estimate for Passive Components- A Literature Survey," May 2007.

8. Advisory Committee on Reactor Safeguards, Subcommittee on Materials, Metallurgy, and Reactor Fuels, December 6, 2007, Pages 22, 25-26.

INTRODUCTION

6. The VYNPS License Renewal Application Table 4.3-3 states the results of an Entergy analysis of the impact of environmentally assisted metal fatigue on certain reactor components during the proposed period of extended operation. This analysis indicates that the environmentally corrected Cumulative Usage Factor (CUFen) of several critical reactor components will exceed unity. Cumulative Usage Factors (CUFs) that exceed unity signal potential fatigue damage. Entergy's License Renewal Application, section 4.3-7, proposes several possible approaches to this problem, including "further refinement of the fatigue analyses to lower the predicted CUFs to less than 1.0."

7. It is my understanding that Entergy has now performed this "further refinement of fatigue analyses," and that the Structural Integrity Associates, Inc. (SIA) Environmental Fatigue Analysis calculations and reports, attached hereto as Exhibits A-O, describe and state the results of Entergy's reanalysis (hereinafter referred to as "Entergy's CUFen Reanalysis").

8. Entergy's CUFen Reanalysis indicates that CUFens for all components assessed are less than unity. These results are summarized in Table 3-10 of the SIA report attached as Exhibit O to this Declaration. However, based on my review of the SIA calculations and reports (Exhibits A-O) and recent publications on the effect of the environment on fatigue (References 1-4), I have concluded that Entergy's results are flawed.

9. As detailed in paragraphs 12-27 of this Declaration, the analytical techniques employed in Entergy's CUFen Reanalysis were not rigorous and resulted in unrealistically low CUFens. I have determined that the CUFen Reanalysis is subject to

numerous uncertainties that are neither quantified nor discussed. To validate the analytical techniques, Entergy should have performed careful error analyses to show the admissible range for each variable, but does not appear to have done so.

10. The global lack of error analysis is especially troubling. For example, the summary results stated in Exhibit O, Table 3-10 report a CUFen of 0.7446 for the RHR Class 1 piping. In the light of the fact that data scatter in fatigue studies often exceeds an order of magnitude, the value of 0.7446 without an error band has little significance and imparts little confidence that fatigue failure will not occur.

11. As detailed in paragraphs 28-32 of this Declaration, I have recalculated CUFens for the same components addressed by Entergy's CUFen Reanalysis, using more reasonable and appropriate CUF and Fen values. My recalculations indicate that CUFens for all components except the RHR return piping will exceed unity.

CRITIQUE OF ENTERGY'S CUFen REANALYSIS (Exhibits A-O)

12. Broadly speaking, the calculation of an environmentally corrected CUF, denoted as CUFen, proceeds in two steps: first, one calculates the fatigue usage factor for each transient CUF, and then one calculates the corresponding environmental fatigue correction factor (Fen). The final CUFen for the life of the plant is obtained by multiplying the CUFs by their respective Fens and summing up all the products. Fen is defined as the ratio of the fatigue life in air at room temperature to that in water at the service temperature.

13. I have determined that both the CUFs and the Fens used in Entergy's CUFen Reanalysis must be recalculated due to numerous uncertainties and unjustified assumptions.

Critique of Entergy's Calculations of Fen

14. I have detected a number of errors in Entergy's calculation of Fen values.

15. Entergy used outdated Argonne National Laboratory (ANL) statistical equations stated in NUREG/CR 6583 and NUREG/CR 5704 ("the NUREG equations"), which were derived more than nine years ago, to calculate the Fen parameters. On December 7, 2006, ANL presented new equations and new data to the ACRS, and in February 2007 ANL published its results in NUREG/CR-6909 (Reference 1). The revised ANL equations are based on a much larger database, and ANL provided a thorough discussion of their limitations. Additional data on the effect of the BWR environment on Fen were also recently published in the papers I have denoted References 2-4.

16. SIA Calculation No. VY-16Q-303 states that per "Section X.M1 of the GALL Report [2], the EAF evaluation **must use** the appropriate F_{en} relationships from NUREG/CR-6583 [4] (for carbon/low alloy steels) and NUREG/CR-5704 [5] (for stainless steels). . . .". Exhibit C at 4 (emphasis added). I strongly disagree with Entergy's contention that it is prohibited from using the latest fatigue data in the calculation of the Fen values. It appears that Entergy incorrectly interpreted the wording of the GALL report. NUREG 1800, which incorporates the GALL report, only provides guidance to the NRC staff on how to review LER applications, and the review is entirely based on the Applicant's information. It would be against the public interest for the NRC

to prescribe the use of outdated data when new data are available and are known to the Applicant. The GALL report did not reference the new fatigue data because they were not available at the time the GALL report was written. The GALL report does not require use of a specific set of equations. It provides only as follows: "Formulae for calculating the environmental life correction factors are contained in NUREG/CR-6583 for carbon and low-alloy steels and in NUREG/CR- 5704 for austenitic stainless steels. As evaluated below, this is an **acceptable option** for managing metal fatigue for the reactor coolant pressure boundary, considering environmental effects." GALL Report NUREG 1801, Sept 2005, Rev 1, Chapter X.M1 (emphasis added). It should also be noted that in spite of Entergy's claim that the ANL equations published in NUREGS/CR 5704 and 6583 must be strictly obeyed, the versions of these equations used in Entergy's CUFen Reanalysis incorporate a modified temperature term as per EPRI-recommended corrections. *See,* Exhibit O, SIR-07-132-NPS, Rev. 0 at 3-1.

17. In calculating the Fen values, Entergy failed to include or consider many significant factors which were not included in the NUREGS 5704 and 6583 equations. These equations were developed in a laboratory environment under controlled conditions. They represent the best fit to the laboratory data, but by no means do they represent the actual conditions of reactor components in service. This was clearly noted by the developer of the NUREG equations, O. Chopra, when he testified before the ACRS as follows: "To apply those [laboratory data] to actual reactor components, we need to adjust these results to account for parameters or variables which we know affect fatigue life but are not included in this data. And these variables are mean stress, surface finish, size, loading history." Reference 8 at 22. After reviewing the ANL experiments, my

conclusions are similar to those of O. Chopra, i.e., great care must be exercised when the ANL data are applied to reactor components. In my opinion, Entergy has applied the ANL equations to VYNPS components in an indiscriminate manner and consequently the results are not conservative and contain unquantified errors. Based on my References 1-4, it is my opinion that Fen values reported in Table 3-10 of Exhibit O must be corrected to account for the following variables:

a. Flow velocity
b. Surface finish
c. Stress ratio
d. Strain rate change
e. Size and geometry
f. Excursions of normal water chemistry
g. Data scatter

18. In addition to the fact that Energy applied the incorrect equations in calculating Fen values, Entergy also did not use the equations properly at low oxygen and low temperatures. If proper corrections are not applied, Entergy's results show that below 50 ppb or below 150 Celsius, Fen is independent of the oxygen, the temperature and the strain rate for carbon and low alloy steels. O. Chopra, the developer of the NUREG equations, stated at the December 6, 2007 ACRS meeting (Reference 8) that the effect of oxygen above 40ppb is not zero, nor is the effect of temperature below 150 Celsius. I believe that Entergy must specify the error band in Fen if it insists on using the NUREG equations.

Critique of Entergy's Calculations of 60-Year CUFs

19. Entergy's CUFen Reanalysis includes the calculation of a "60-Year CUF," defined as "CUF results using updated ASME Code methodology and actual cycles

accumulated to-date and projected to 60 years." Exhibit O, Table 3-10 ("Summary of Environmental Fatigue Calculations for VYNPS"). I have detected a number of uncertainties in Entergy's calculation of these 60-year CUFs. Entergy introduced many new assumptions in the revised analysis that, as discussed below, may not be conservative. Even though some of the data used to calculate the stresses show anomalies, Entergy did not discuss them nor did they assign error bands on the calculated values.

20. Entergy's CUFen Reanalysis does not include information sufficient to quantify the uncertainties in the CUFs. Entergy has provided no discussion of the uncertainties and the major assumptions that were made in the determination of the boundary conditions at the surface of the components during the various thermal transients. Entergy states that the assumptions were "conservative" or "bounding" without further explanation. I am concerned that some of the assumptions Entergy introduced were not conservative. I appreciate that the determination of the fatigue life of reactor components requires many assumptions and approximations, because there is a lack of valid data. Under these conditions, it is imperative that all assumptions be properly identified and justified. Entergy has not provided the information that would allow me to quantify the effects of their key assumptions.

21. To obtain stress histories during plant transients, Entergy used the Green's Function Methodology to avoid the expense of finite element computations for all the transients. The basic heat conduction equation that describes the temperature, and therefore the stress distribution for a given component is non-linear. Green's Function is a simplification that is used to linearize non-linear equations. Such simplifications can

result in errors, depending on the geometry and the variation of the surface temperature with time. Entergy did not specify any error band on the stress results, especially for those transients where the temperature does not vary linearly with time.

22. Using two different Green functions, each with a different heat transfer coefficient, Entergy plotted the stress on several components as a function of time to illustrate the sensitivity of the results to heat transfer. These plots appear to indicate that the thermal stress is not sensitive to the wall heat transfer during the first 100 seconds, yet it has a significant effect later on during the transient. Entergy does not discuss the reason for these results, and they may be a direct consequence of some unspecified assumptions.

23. The calculation of the thermal stress distribution for a given component, using Green's function or a finite element method, depends on the geometry. There is no indication in the SIA calculations and reports provided, Exhibits A-O, that Entergy used pipe wall dimensions from the latest in-service inspection measurements. Carbon steel piping is especially susceptible to flow-accelerated corrosion (corrosion rates as high as 120 mils per year have been detected in BWRs). Piping with difficult routing and geometry may have been installed differently than what was indicated on the original drawing. The SIA calculations and reports do not specifically state that this factor was considered.

24. The heat transfer coefficients Entergy employed in the transient analysis are valid for fully developed flow for straight pipes under steady-state conditions and constant fluid properties. It is not clear why steady-state coefficients should be applicable to transient conditions. In geometries such as elbows and nozzles, higher local heat transfer

coefficients than those used by Entergy would be expected because of the presence of secondary flows. In the temperature range of interest (100 to 500 degrees Celsius), water properties vary considerably. Viscosity, for example, varies by a factor of seven and, therefore, a proper correction to the heat transfer coefficient must be applied to account for variation of fluid properties across the thermal boundary layer.

25. Temperature and flow velocities during plant transients are calculated with a thermal hydraulic computer code. At the EPU proceedings, Entergy stated that it was using the ODYN code for this purpose. Since the ODYN code has not been benchmarked for the EPU flow rates at VYNPS, temperatures and flow velocities calculated using this model are subject to uncertainties. To my knowledge, the NRC has approved the ODYN code only for calculating maximum reactor dome pressure. Entergy's CUFen Reanalysis does not address how uncertainties in velocities and temperature could affect identification of the location of the maximum component stress during the transients.

26. Entergy's apparent assumption that the number of transients that the plant would experience throughout its life varies linearly with time must be challenged. The recent VYNPS 20% power uprate introduced new stresses on already aging components, thereby accelerating the number of unanticipated transients as demonstrated by the recent collapse of the VYNPS cooling tower, plant shutdown due to a steam valve failure, and the continuous development of cracks in the steam dryer. VYNPS experienced two unanticipated transients within 10 days in late August of this year. Based on this experience and the assumption of linearity, one could predict 912 transients during the next 25 years (36.5 transients per year x 25 years). The above extreme case illustrates

that Entergy must consider the actual historical variation in the number of transients with time in determining the predicted number of transients during the extended period of operation. Because the CUFs are sensitive to the number of transients, the assumption of linearity must be viewed as a key assumption in Entergy's analysis. In my opinion, this assumption is not conservative. In my opinion, the number of transients during the extended period should be multiplied by at least 1.2 to account for the probability of an increase in unanticipated failures due to the power uprate.

Other Errors in the calculation of CUFen

27. In calculating the CUFen, it is necessary, as discussed in NUREG/CR-6909, to calculate the partial usage factor for each stress cycle and multiply it by the corresponding Fen; at a minimum, Fen depends on the strain rate, temperature, oxygen and sulfur contents. The actual measured oxygen content in the water at the surface of the component for each transient must be used if one relies on the NUREG 5704 and NUREG 6583 equations. The CUFen is then obtained by summing the individual products for all stress cycles. Entergy's CUFen Reanalysis does not indicate that it followed this procedure. Instead, the Fen was apparently proportioned for two different water chemistries, HWC and NWC. Entergy indicates that it used bounding Fen factors. I do not agree that Entergy used bounding Fen values.

RESULTS OF HOPENFELD CUFen RECALCULATION

28. After carefully considering Energy's approach of applying well-defined but incomplete laboratory data to complex and poorly known reactor conditions, I conclude

that it would be more appropriate and more conservative to use proper bounding Fen values together with the design basis CUF values. As I have stated in paragraphs 17 and 18, the Fen equations must be corrected to account for several important variables; however, in the absence of valid data to do so, one must employ the best available bounding values for Fen. This is exactly what I have done in arriving at the recalculated CUFens presented in Table 1 below.

TABLE 1 - Recalculated Cumulative Usage Factors for Sample Locations at VYNPS

No.	NUREG-6260 Sample Location (License Renewal Application, Table 4.3-3)	CUF (VYNPS License Renewal Application, Table 4.3-3)	Fen (Ref. 1)	**Recalculated CUFen**
1	Vessel shell & bottom head	0.400	17	6.80
2	Core spray safe end	0.182	12	2.18
3	Feed water nozzle	0.750	17	12.75
4	RHR return Piping	0.032	12	0.38
5	RR inlet nozzle	0.610	17	10.37
6	RR piping tee	0.397	12	4.76
7	RR outlet nozzle	0.810	17	13.77
8	Core spray nozzle	0.625	17	10.62
9	Feed water piping	0.427	17	7.26

29. The CUF parameters in Table 1 are the existing design basis CUFs for the VYNPS as reported by Entergy in the VYNPS License Renewal Application, Table 4.3-3. These numbers are larger by a factor of 2 - 100 than the revised "60-Year CUF" values used in Entergy's Reanalysis, as reported in Table 3-10 of the SIA report designated Exhibit O hereto. Due to the many uncertainties in the calculations of the 60-Year CUFs, discussed in paragraphs 19-26 of this Declaration, it is not reasonable or

sufficiently conservative to replace the design basis CUFs with the values used in Entergy's CUFen Reanalysis.

30. The Fen values stated in Table 1 are the bounding values given in the NRC Bibliographic data sheet attached to NUREG/CR 6909. Those bounding values are: Fen ~ 17 for carbon and low-alloy steels and Fen ~ 12 for stainless steel. As I have already stated in paragraphs 14-18 of this Declaration, because of the many uncertainties associated with the statistical equations used in Entergy's CUFen Reanalysis, it is necessary to use the above bounding Fen values.

31. The Fen values in Table 1 are based on the latest ANL data and ANL observations that: "Under certain environmental and loading conditions, fatigue lives in water relative to those in air can be a factor of approximately 12 times lower for austenitic stainless steels, approximately 3 times lower for Ni-Cr-Fe alloys and approximately 17 times lower for carbon and low-alloy steels." *See*, NRC Bibliographic Data Sheet attached to NUREG/CR-6909. Since the ANL data did not include all the variables discussed in paragraph 17 above, Fen values may be changed at some future time when more data become available. Nevertheless, in my opinion these are the best values of Fen available at the present time.

32. Even in the unlikely event that the 60-year CUFs used in Entergy's CUFen Reanalysis are proven to be valid, multiplications of the latter by the respective Fen values in Table 1 show that four components exceed unity. Thus, when the "60-Year CUFs" of components No. 2, No. 3, No. 5 and No. 9 of Table 3-10 (Exhibit O) are multiplied by Fen = 17, the resultant CUFens are 1.316, 1.081, 1.105 and 2.823, respectively.

SUMMARY

33. In conclusion, Entergy's CUFen Reanalysis is flawed, and subject to numerous uncertainties and insufficient conservatism.

34. Since Entergy has not demonstrated that CUFens for the NUREG/CR-6260 selected sample components are less than one, Entergy is required by industry guidelines to calculate CUFen values at additional locations. Accordingly, it is my opinion that:

a. Entergy must calculate the CUFens of additional VYNPS components in accordance with the guidelines of EPRI, "Material Reliability Program: Guidance for Addressing Fatigue Environmental Effects in a License Renewal Applications" MPR-47, Rev 1 Sept 2005, pg. 5-1.

b. Entergy must formulate an appropriate inspection program to manage fatigue at affected locations.

c. Before Entergy is granted a License Extension, the inspection program should be reviewed by at least two competent and qualified engineers for the purpose of ensuring that Vermont Yankee is not operated beyond 2012 in an unanalyzed condition and at increased risk of component failure. Reviewing engineers should be completely independent from those who are responsible for the operation of the VYNPS and other parties to this litigation.

d. The indicated inspection program must be of sufficient rigor and precision as to provide adequate assurance of public health and safety during the proposed period of extended operation.

I declare under penalty of perjury that the foregoing is true and correct.

Executed this 31 day of August, 2007 at Rockville, Maryland.

Joram Hopenfeld

Joram Hopenfeld, PhD

Structural Integrity Associates, Inc.	File No.: VY-16Q-301 EXHIBIT A
CALCULATION PACKAGE	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Feedwater Nozzle Stress History Development for Green Functions

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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%), mid (40%), and low (25%) flow thermal loading of the Vermont Yankee Nuclear Power Station feedwater nozzle.

2.0 FEEDWATER NOZZLE MODEL

An axisymmetric finite element model of the feedwater nozzle was developed in Reference [1] using ANSYS [2]. The geometry used in Reference [1] was utilized in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FE model for Green's Function development. Table 1 listed the material properties at 300°F. The meshed model is shown in Figure 1.

3.0 APPLIED LOADS

Both pressure and thermal loads will be applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the feedwater nozzle and the vessel wall. A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. Since only nodes were modeled, the nodal forces shown in Table 2 are defined by the following equation:

$$F_{element} = \pi(IR)^2 P \cdot \frac{\pi\left(R_o^{\,2} - R_i^{\,2}\right)}{\pi\left(OR^2 - IR^2\right)}$$

where:

P = Pressure = 1,000 psi
IR = Inner Radius = 4.8345 in
OR = Outer Radius = 5.42 in
R_i = Inside Radius of element that node is attached to
R_o = Outside Radius of element that node is attached to
F_{node} = The average of the element forces on either side of the node.
Note: The force on the innermost and outermost nodes is calculated as one half of the force on the element that they are attached to.

The calculated nodal forces were applied as positive values so they would exert tension on the end of the model. The ANSYS input file FWP_VY.INP, in the computer files, contains the feedwater

nozzle geometry as well as the pressure loading. Figures 2, 3, and 4 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

3.2 Thermal Load

Thermal loads are applied to the feedwater nozzle model. The heat transfer coefficients after power uprate were determined from Reference [1]. These values were determined for various regions of the finite element model for 100% (4,590 GPM), and 25% (1,148 GPM) [1]. The annulus leakage flow rate is assumed to be 25 GPM for non-EPU conditions and 31 GPM for EPU conditions. The 25 GPM value is calculated by scaling the 23 GPM [Page 6, 4] value up by approximately 9%. The 23 GPM value is scaled up to provide some conservatism and allow for inaccuracies in the determination of leakage flow. The 31 GPM value is calculated by multiplying the 25 GPM value by 1.25 [Page 6, 4]. Based on this, the annulus leakage flow rate is assumed to be 8 GPM for EPU conditions with 25% flow rate. The temperatures used are based upon a thermal shock from 500°F to 100°F. An additional 40% flow rate (1836 GPM and 13 GPM) was added in this calculation.

3.2.1 *Heat Transfer Coefficients*

Referring to Figure 5, heat transfer coefficients were applied as following:

Region 1

The heat transfer coefficient, h, for 100% flow is 3705 BTU/hr-ft^2-°F at 300°F. [1, Table 5]

The heat transfer coefficient, h, for 40% flow is 1780 BTU/hr-ft^2-°F at 300°F. [Table 4]

The heat transfer coefficient, h, for 25% flow is 1222.2 BTU/hr-ft^2-°F at 300°F. [1, Table 4]

Region 2

Per Reference [1], the heat transfer coefficient for Region 2 (safe end-to-thermal sleeve contact region) should be linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used in Region 3.

Region 3

The heat transfer coefficient, h, for 100% flow is 1489 BTU/hr-ft^2-°F at 300°F. [1, Table 9]

The heat transfer coefficient, h, for 40% flow is 743 BTU/hr-ft^2-°F at 300°F. [1, Table 9]

The heat transfer coefficient, h, for 25% flow is 504 BTU/hr-ft^2-°F at 300°F. [1, Table 9]

Region 4

Per Reference [1], the he heat transfer coefficient for Region 4 (thermal sleeve transition in diameter) should be linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h, for 100% flow is 177.4 BTU/hr-ft^2-°F at 300°F. [1, Table 16]

The heat transfer coefficient, h, for 40% flow is 88.5 BTU/hr-ft^2-°F at 300°F. [1, Table 16]

The heat transfer coefficient, h, for 25% flow is 60 BTU/hr-ft^2-°F at 300°F. [1, Table 16]

Region 6

Per Reference [1], the heat transfer coefficient for Region 6 (nozzle inner blend radius) should be linearly transitioned from the value of the heat transfer coefficient used in Region 5 to the value used in Region 7.

Region 7

Per Reference [1], the heat transfer coefficient for Region 7 (reactor vessel inside wall) is a constant of 864 BTU/hr-ft^2-°F. This value is consistent with the feedwater nozzle work performed in the past for VY and should be used for all reactor conditions.

Region 8

The heat transfer coefficient, h, is 0.2 BTU/hr-ft^2-°F [1].

3.2.2 *Boundary Fluid Temperatures*

For the Green's Functions, a 500°F – 100°F thermal shock is run to determine the stress response to a one-degree change in temperature. The following temperatures are valid when there is water flow. Values between defined points are linearly interpolated. For the 100%, 40%, and 25% flow cases, the thermal shock is run as follows:

Regions 1 to 5
T = 500°F – 100°F
Region 6
Linearly transitioned from the value of the temperature used in Region 5 to the value used in Region 7
Region 7
T = 500°F
Region 8
T = 120°F

4.0 THERMAL AND PRESSURE LOAD RESULTS

The three flow dependent thermal load cases outlined in Section 3.0 were run on the finite element model. Appendix A contains the thermal transient input files FWT_VY_100.INP, FWT_VY_40.INP, and FWT_VY_25.INP for 100%, 40%, and 25% full flow rate, respectively. The three flow dependent input files for the stress runs are also included in Appendix A. The stress filenames are FWS_VY_100.INP, FWS_VY_40.INP, and FWS_VY_25.INP for 100%, 40%, and 25% full flow rate, respectively.

The critical safe end location was chosen as node 192, which has the highest stress intensity due to thermal loading under high flow conditions. As shown in Figures 6 and 7, Node 192 is located on the inside diameter of the nozzle safe end of the model and the maximum stress occurs at 1.4 seconds.

The critical blend radius location was chosen, based upon the highest pressure stress. Conservatively assuming the cladding has cracked, the critical location is selected as node 657 at base metal of the nozzle, as shown in Figures 8 and 9.

The stress intensity for use in the Green's functions are calculated from the component stresses (X, Y, and Z) and compared to the stress intensity reported by ANSYS. As seen in Figure 10, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the safe end. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the safe end. As seen in Figure 11, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the blend radius in very beginning. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the blend radius.

The stress time history for the critical paths was extracted during the stress run for 100% flow rate. This produced two files, HFSE.OUT and HFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files HFSE_Inside.RED and HFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.

The stress time history for the critical paths was extracted during the stress run for 40% flow rate. This produced two files, MFSE.OUT and MFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files MFSE_Inside.RED and MFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.

The stress time history for the critical paths was extracted during the stress run for 25% flow rate. This produced two files, LFSE.OUT and LFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files LFSE_Inside.RED and LFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.

As the models were run with a 400°F step change in temperature, and the Green's Functions are for a 1°F step change in temperature, all data values were divided by 400. The governing Green's Functions for the feedwater nozzle during 100% flow, 40% flow, and 25% flow are shown in Figures 12 to 23. The data for the Green's Functions is included in the files HFBR_M+B-Green.xls, HFBR_T-Green.xls, HFSE_M+B-Green.xls, HFSE_T-Green.xls, MFBR_M+B-Green.xls, MFBR_T-Green.xls, MFSE_M+B-Green.xls, MFSE_T-Green.xls, LFBR_M+B-Green.xls, LFBR_T-Green.xls,LFSE_M+B-Green.xls, and LFSE_T-Green.xls in the project Files. Where HF, MF, and LF corresponded to 100% flow, 40% flow, and 25% flow rate, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

The pressure stress intensities for the path were extracted during the pressure run. The pressure stresses were extracted along the nodal paths as shown in Figures 7 and 9. This produced two files, PSE.OUT and PBLEND.OUT for the safe end and blend radius locations, respectively.

For the pressure loading specified (1,000 psig), the total stress intensity at Node 192 and Node 657 were determined to be 8,891 psi and 28,300 psi, respectively. The membrane plus bending stress intensity at Node 192 and Node 657 were determined to be 8,693 and 27,490 psi, respectively. Table 3 shows the pressure results.

Results were also extracted from the vessel portion of the model to verify the accuracy of the pressure results obtained from the ANSYS model, and to check the results due to the use of the 1.5 multiplier on the vessel radius. These results are contained in the file, PVESS.OUT. Based on earlier work [1], the radius of the finite element model (FEM) was multiplied by a factor of 1.5 to account for the fact that the vessel portion of the two-dimensional (2D) axisymmetric model is a sphere, but the true geometry is the intersection of two cylinders.

The equation for the membrane hoop stress for a sphere is:

$$\sigma = \frac{(pressure) \times (radius)}{2 \times thickness}$$

Considering a vessel base metal radius, R, of 105.90625 inches increased by a factor of 1.5, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is PR/(2t) = 14,608 psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, PVESS.OUT, of 13,410 psi. Thus, considering the peak total pressure stress of 28,300 psi reported above, the stress concentrating effect of the nozzle corner is 28,300/14,608 = 1.94. In other words, the peak nozzle corner stress is 1.94 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \frac{(pressure) \times (radius)}{thickness}$$

Based on the previous dimensions, the calculated stress for a cylinder without the 1.5 factor is 19,477 psi. Increasing this by a factor of 1.94 yields an expected peak nozzle corner stress of 37,785 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (28,300 psi) is lower than the peak nozzle corner stress for a cylindrical geometry because of the use of the 1.5 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.

Based on the foregoing, the ANSYS pressure stresses for the vessel blend radius are increased for use in the subsequent fatigue analysis by 1.33 (2.0/1.5). Thus, the blend radius results presented in Table 3 were obtained by multiplying the ANSYS stresses for the pressure loading by a 1.33X multiplication factor.

5.0 REFERENCES

1. SI Calculation No. VY-10Q-301, Revision 0, "Feedwater Nozzle Finite Element Model and Heat Transfer Coefficients."
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. VY Calculation Change Notice (CCN), CCN Number 1 for Calculation VYC1005 Revision 2, "This CCN Provides a Basis for the Power Uprate Safety Analysis Report being submitted as part of the power uprate project. The 50.59 assessment will be handled by the EPU design change and NRC SER for this submittal." SI File Number VY-05Q-208.
5. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
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7. GE Nuclear Energy Certified Design Specification, "Reactor Vessel – Extended Power Uprate," Revision 1, SI File No. VY-05Q-236.
8. N. P. Cheremisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co, 1984.

Table 1: Material Properties @ 300°F (1)

Material Ident.	Young's Modulus, E x 10^6 (psi)	Instantaneous Coefficient of Thermal Expansion, α x 10^{-6} (in/in-°F)	Density, ρ (lb/in^3) (assumed)	Conductivity, k (BTU/hr-ft-°F)	Diffusivity, d (ft^2/hr)	Specific Heat, c_p (BTU/lbm-°F) (see Note 5)	Poisson's Ratio (assumed)
SA533 Grade B, A508 Class II (see Note 2)	26.7	7.3	0.283	23.4	0.401	0.119	0.3
SS Clad (see Note 3)	27.0	9.8	0.283	9.8	0.160	0.125	0.3
A508 Class I (see Note 4)	28.1	7.3	0.283	32.3	0.561	0.118	0.3
A106 Grade B (see Note 4)	28.3	7.3	0.283	32.3	0.561	0.118	0.3

Notes

1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for density and Poisson's ratio, which are assumed typical values [3].
2. Properties of A508 Class II are used (3/4Ni-1/2Mo-1/3Cr-V).
3. Properties of 18Cr - 8Ni austenitic stainless steel are used.
4. Composition = C-Si.
5. Calculated as $[k/(\rho d)]/12^3$.

Table 2: Nodal Force Calculation for End Cap Load

Node Number	Element Number	Radius (in)	Δ Radius (in)	R_o^2-Ri^2 (in^2)	$F_{element}$ (lb)	F_{node} (lb)
1		5.42				7678.0
	1022		0.1171	1.25565	15356.1	
2		5.3029				15188.4
	1021		0.1171	1.22823	15020.7	
3		5.1858				14853.0
	1020		0.1171	1.20080	14685.3	
4		5.0687				14517.6
	1019		0.1171	1.17338	14349.9	
5		4.9516				14182.2
	1018		0.1171	1.14595	14014.5	
6		4.8345				7007.3

Table 3: Pressure Results

Location	Membrane Plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	8693	8891
Blend Radius	36653	37733

Note: The results for the Blend Radius have been increased by a factor of 1.33 (2.0/1.5) as discussed in Section 4.0.

Table 4: Heat Transfer Coefficients for Region 1 (40% Flow)

Calculation of Heat Transfer Coefficients for Feedwater Nozzle Flow Path

Pipe Inside Diameter, D =	9.669	inches =	0.806	ft				100% rated flow =	4,590 gpm
		=	0.246	m				@ T =	391.8 °F
Flow, % of rated =	40%							Density, ρ =	53.8997 lbm/ft³
Fluid Velocity, V =	8.022	ft/sec =	1,836.0	gpm =	0.793742524	Mlb/hr			
Characteristic Length, L = D =	0.806	ft =	0.246	m					
$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F	
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C	

Note: The above assumption is based on experience with past RPV heat transfer analyses.

		Value at Fluid Temperature, T [8]							Units
Water Property	**Conversion Factor [5]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ (Density)	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β (Volumetric Rate of Expansion)	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g (Gravitational Constant)	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ (Dynamic Viscosity)	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr (Prandtl Number)		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	6.0147E+05	8.7645E+05	1.8859E+06	2.8491E+06	3.7255E+06	4.5248E+06	4.7336E+06	—
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	1.2852E+08	6.6834E+08	1.2721E+10	6.5918E+10	2.0931E+11	5.4429E+11	1.1372E+12	—
Rayleigh Number, Ra	GrPr	8.9710E+08	3.0142E+09	2.4297E+10	8.0420E+10	1.9885E+11	4.6755E+11	1.2168E+12	—
From [5]: ***Inside Surface Forced Convection Heat Transfer Coefficient:***									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	5,132.76	6,119.10	8,626.61	10,107.53	10,960.57	11,236.63	10,678.39	W/m²-°C
		903.95	1,077.66	1,519.26	1,780.07	1,930.31	1,978.92	1,880.61	Btu/hr-ft²-°F
		1.744E-03	2.079E-03	2.931E-03	3.434E-03	3.724E-03	3.817E-03	3.628E-03	Btu/sec-in²-°F
From [5]: ***Inside Surface Natural Convection Heat Transfer Coefficient:***									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [6])		
H_{free} =	$C(GrPr)^n k/L$	232.43	330.57	599.85	815.28	988.69	1,118.54	1,192.73	W/m²-°C
		40.93	58.22	105.64	143.68	174.12	196.99	210.06	Btu/hr-ft²-°F
		7.896E-05	1.123E-04	2.038E-04	2.770E-04	3.359E-04	3.800E-04	4.052E-04	Btu/sec-in²-°F

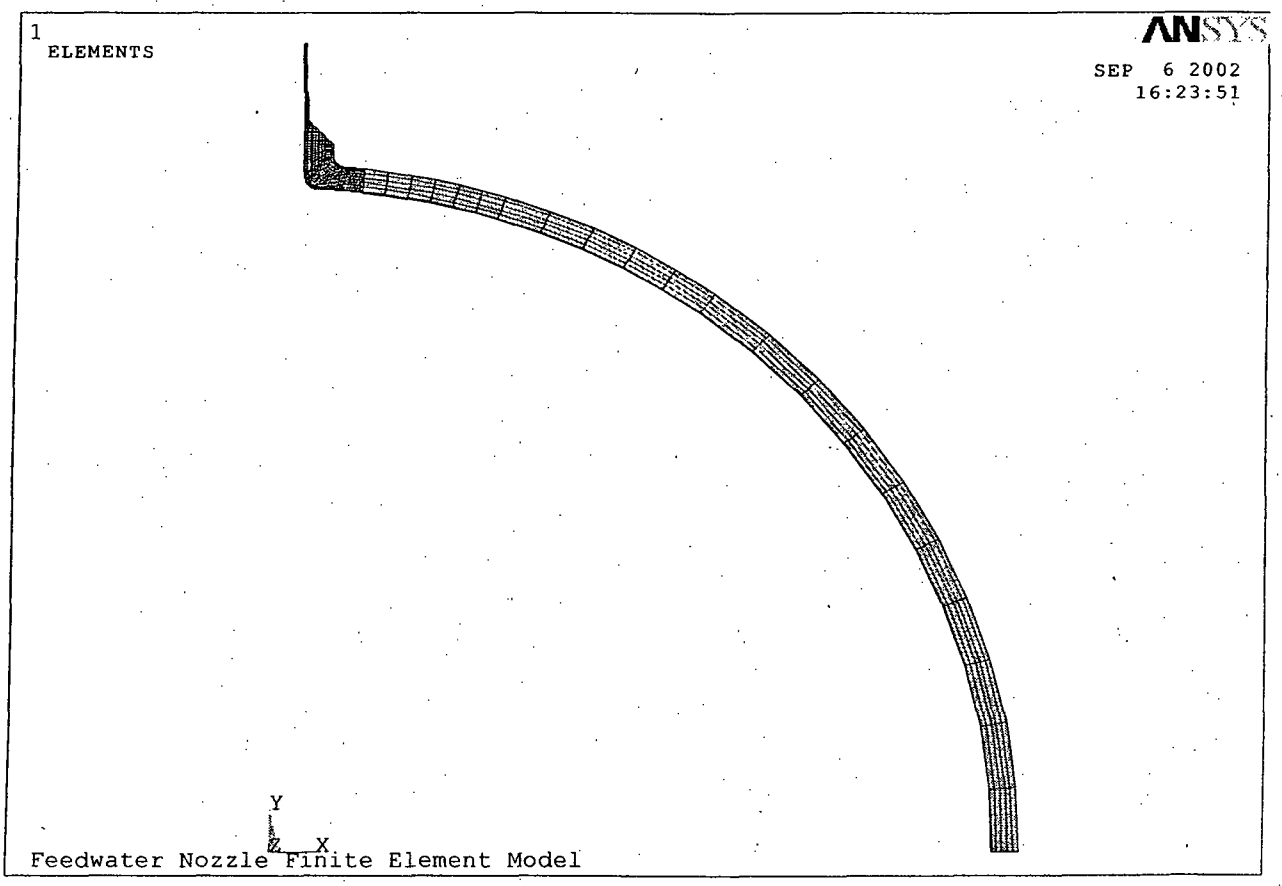


Figure 1: ANSYS Finite Element Model

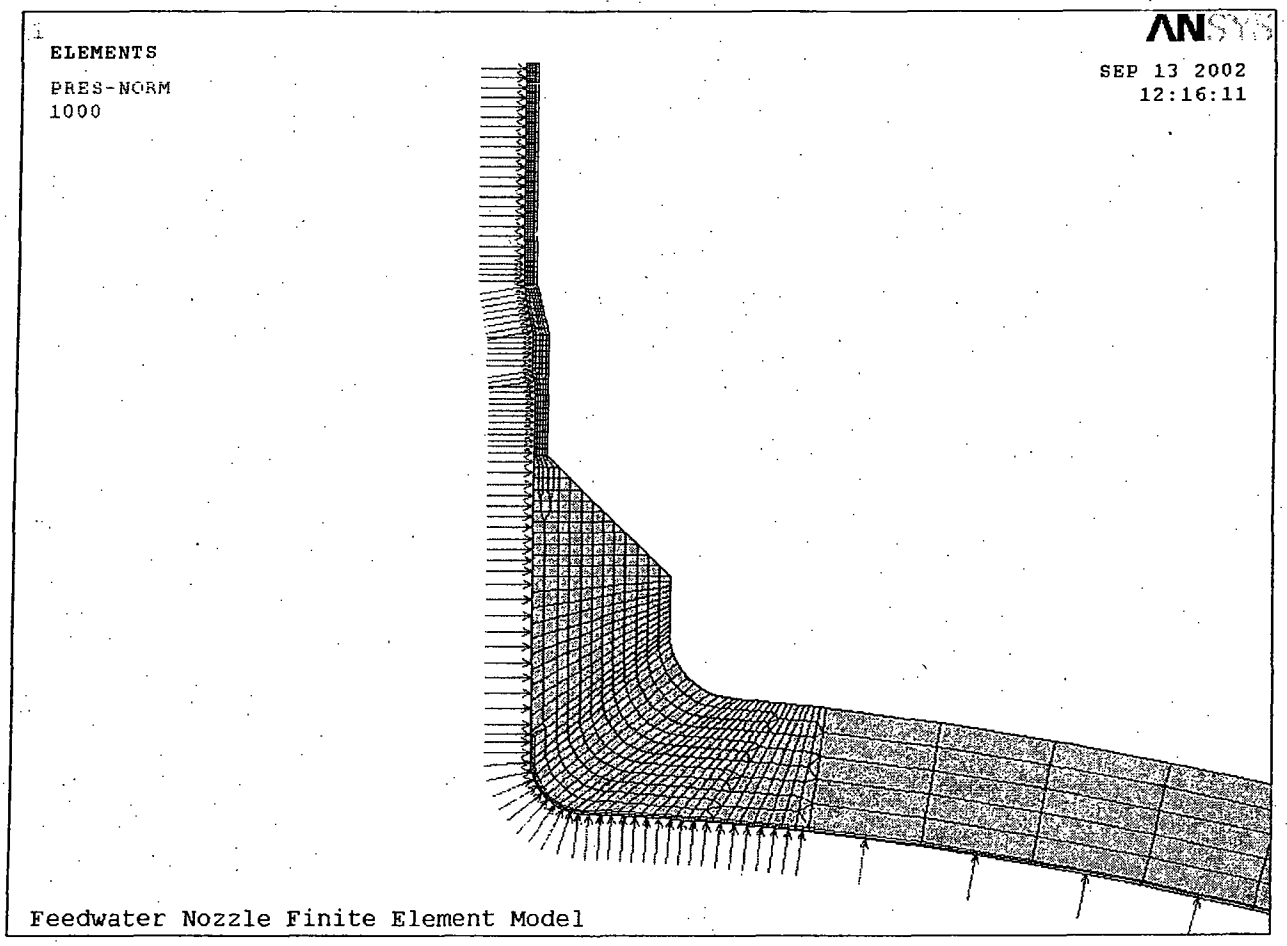


Figure 2: Feedwater Nozzle Internal Pressure Distribution

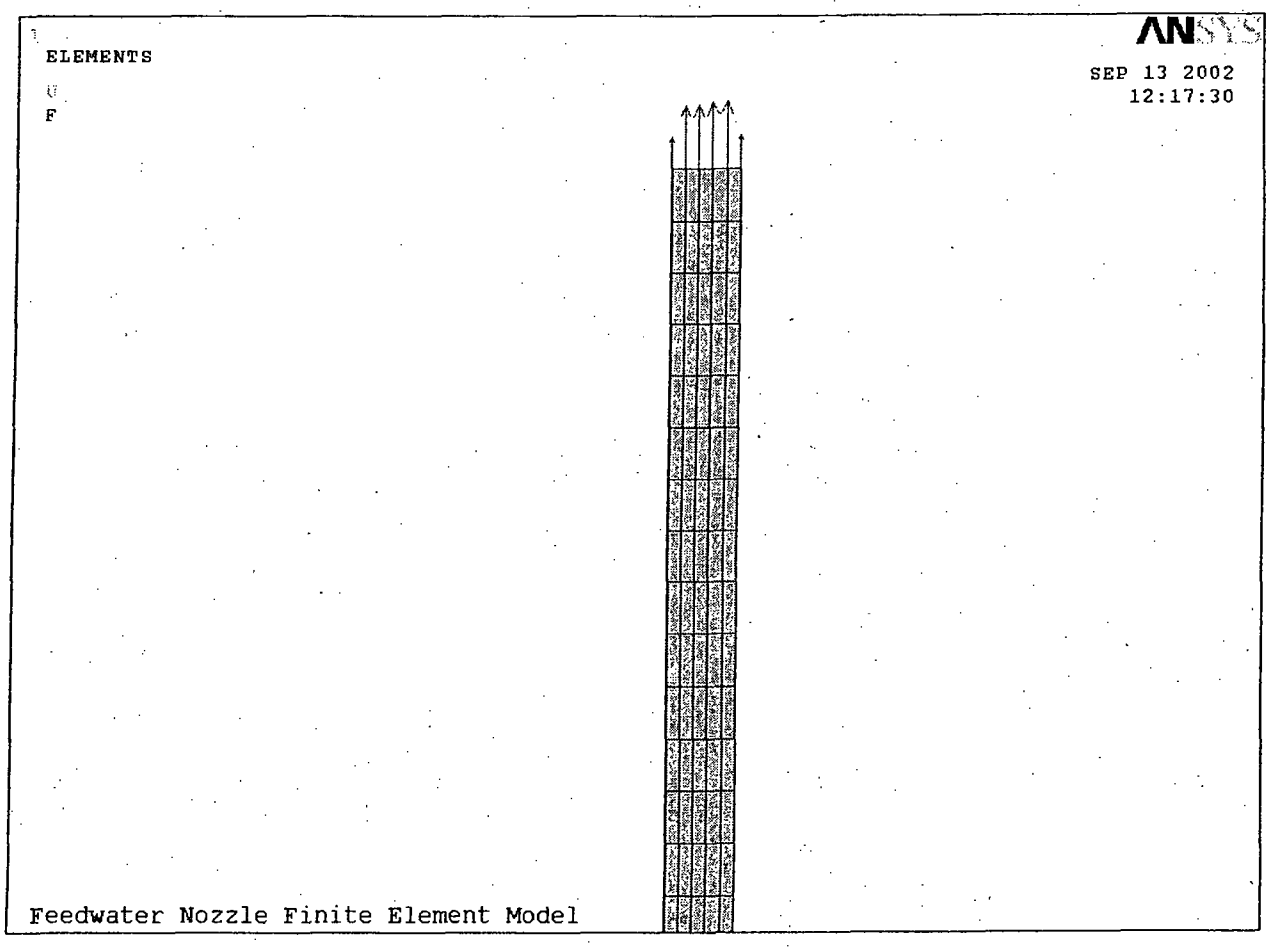


Figure 3: Feedwater Nozzle Pressure Cap Load

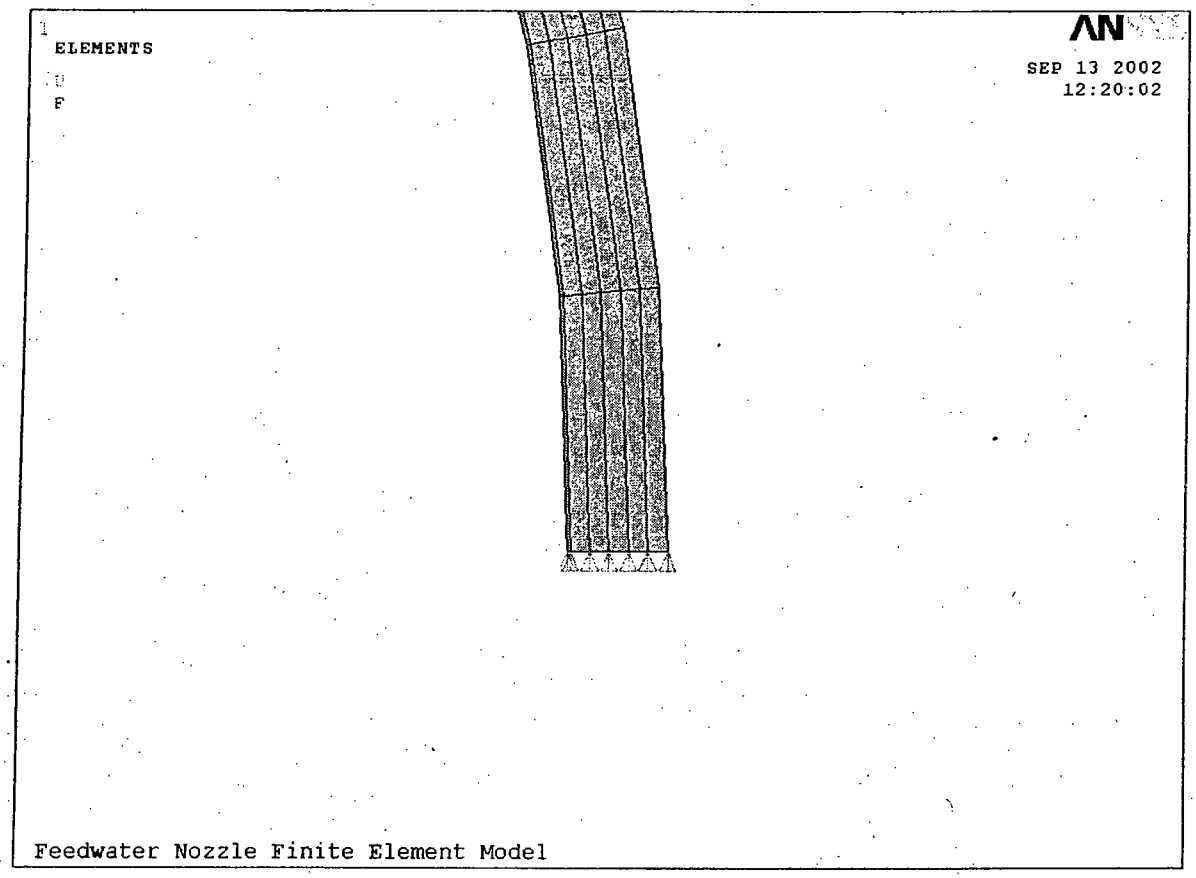
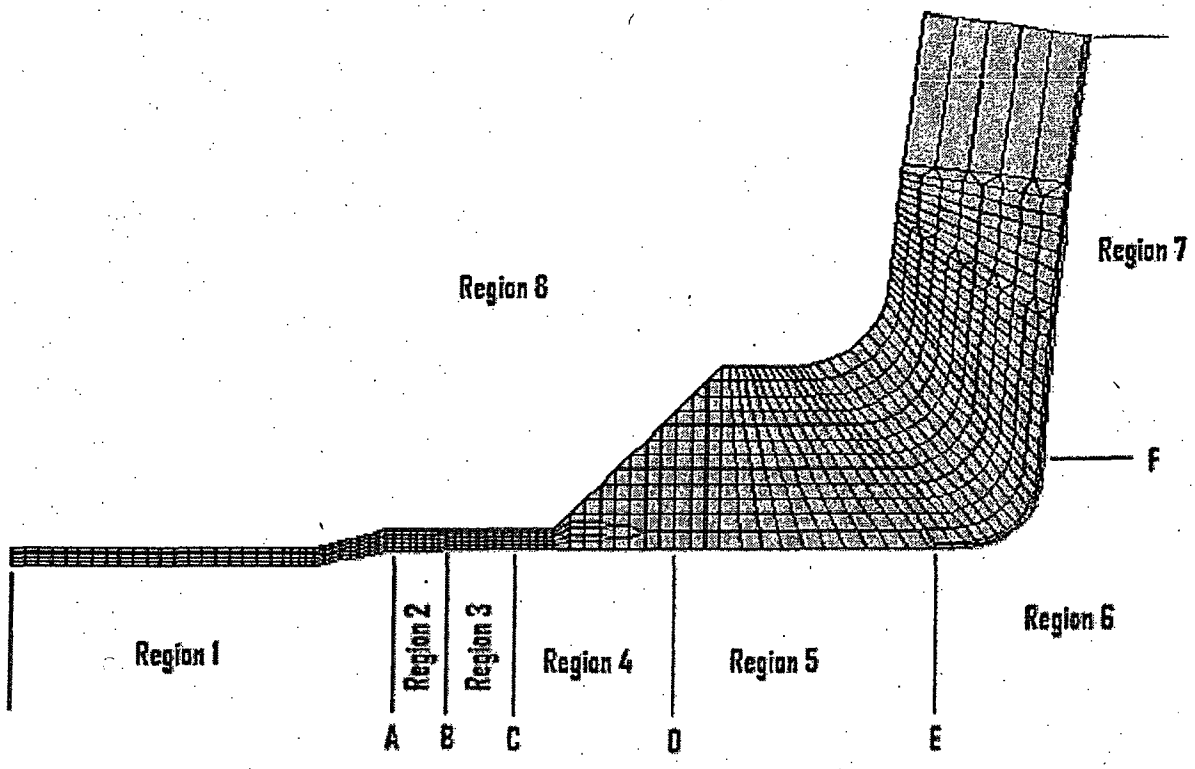


Figure 4: Feedwater Nozzle Vessel Boundary Conditions



Notes: Point A: End of thermal sleeve = Node 204 = 0.25" from feedwater inlet side of thermal sleeve flat.
Point B: Beginning of annulus = Node 252.
Point C: Beginning of thermal sleeve transition = approximately 4.0" from Point A = Node 294.
Point D: End of thermal sleeve transition = approximately 9.5" from Point A = Node 387.
Point E: End of inner blend radius (nozzle side) = Node 553.
Point F: End of inner blend radius (vessel wall side) = Node 779.

Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries [1]

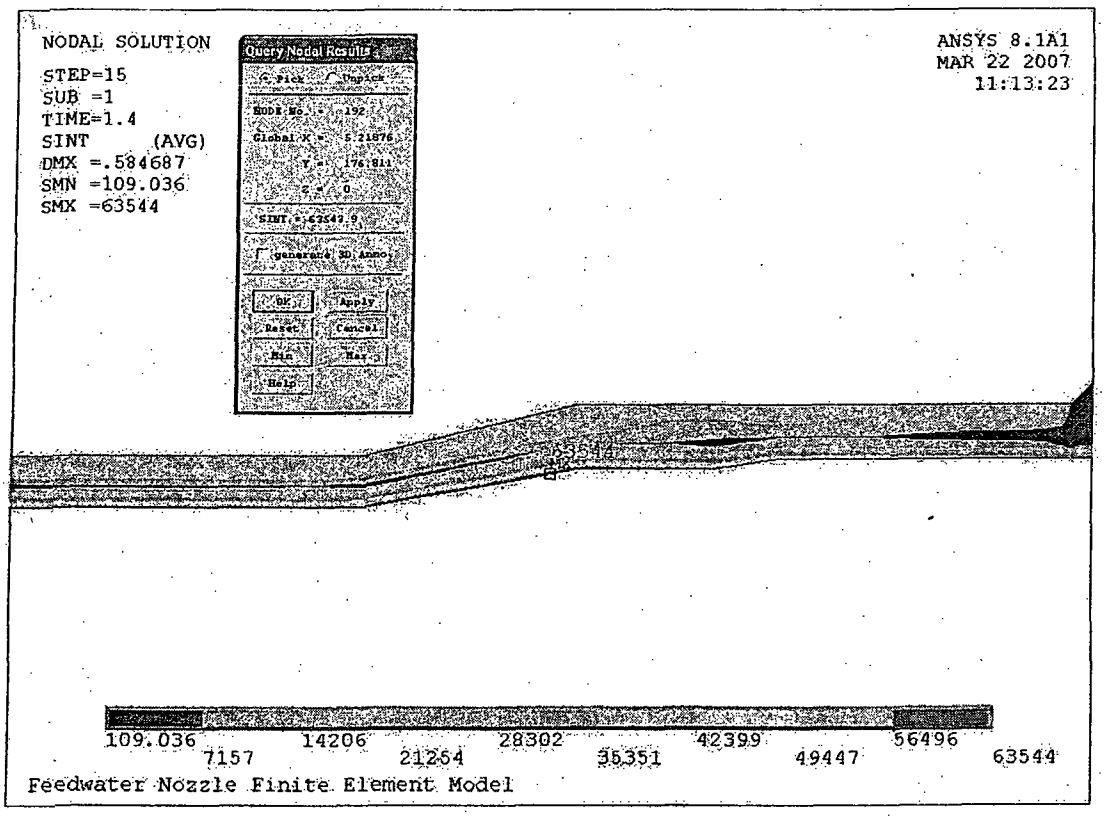


Figure 6: Safe End Critical Thermal Stress Location

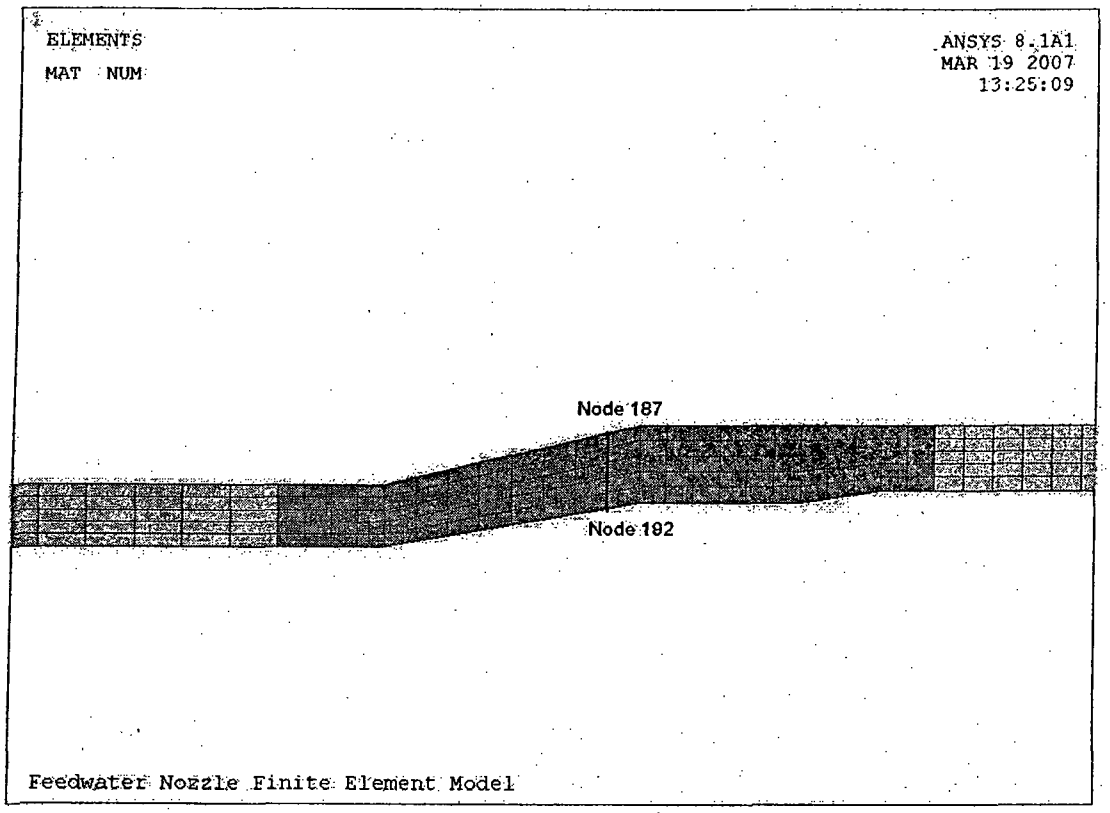


Figure 7: Safe End Limiting Linearized Stress Paths

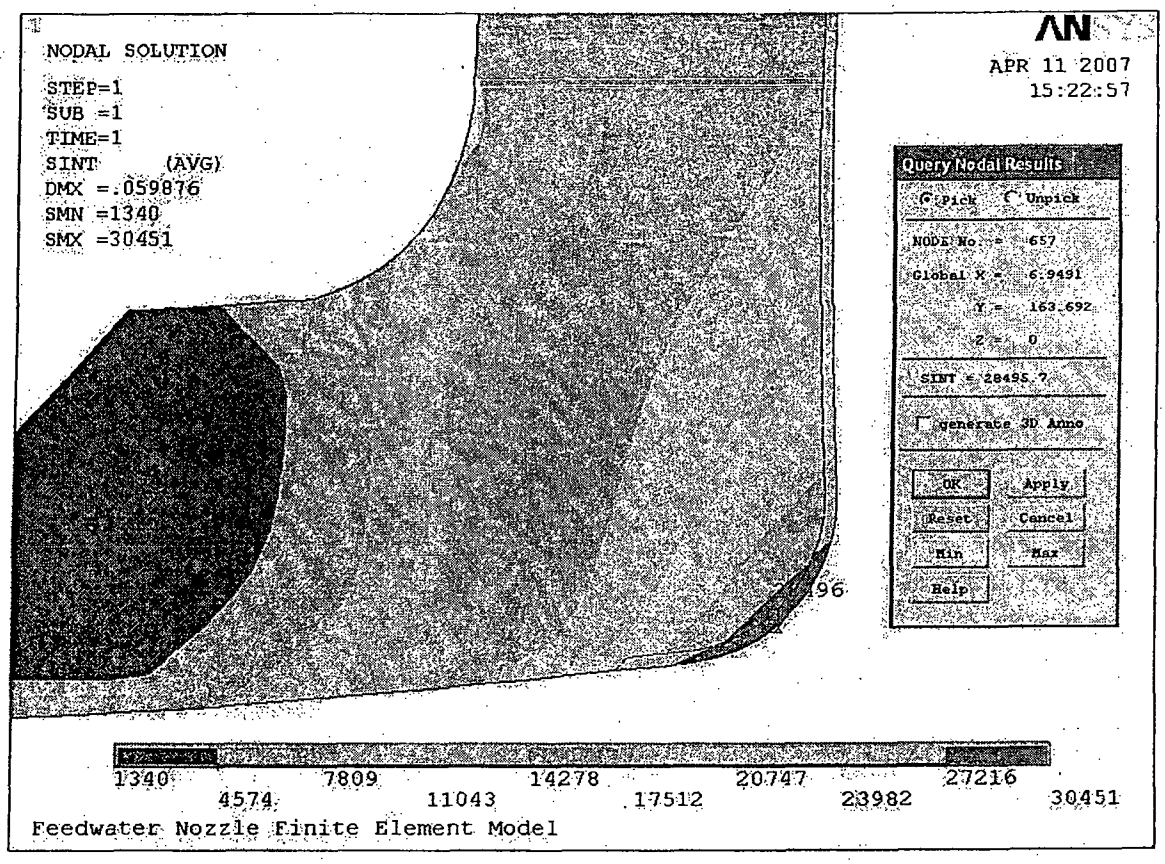


Figure 8: Blend Radius Limiting Pressure Stress Location

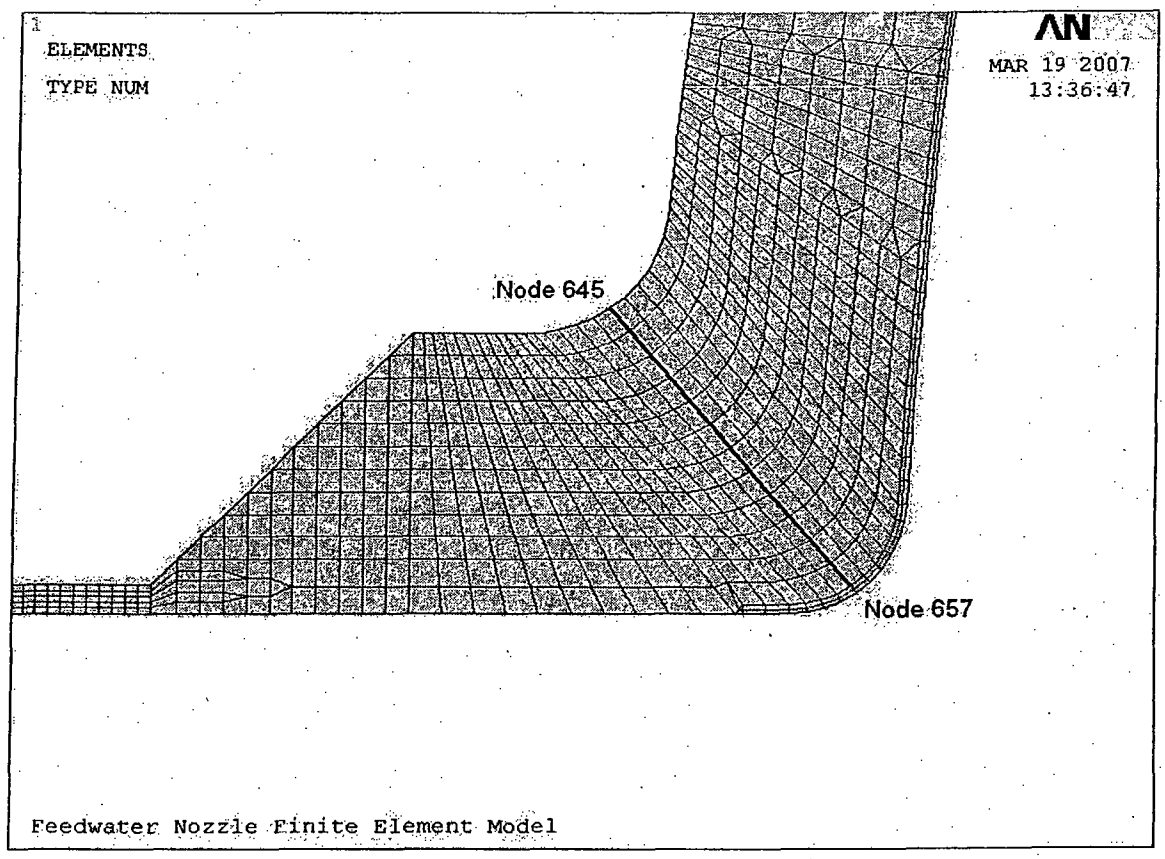


Figure 9: Blend Radius Linearized Stress Path

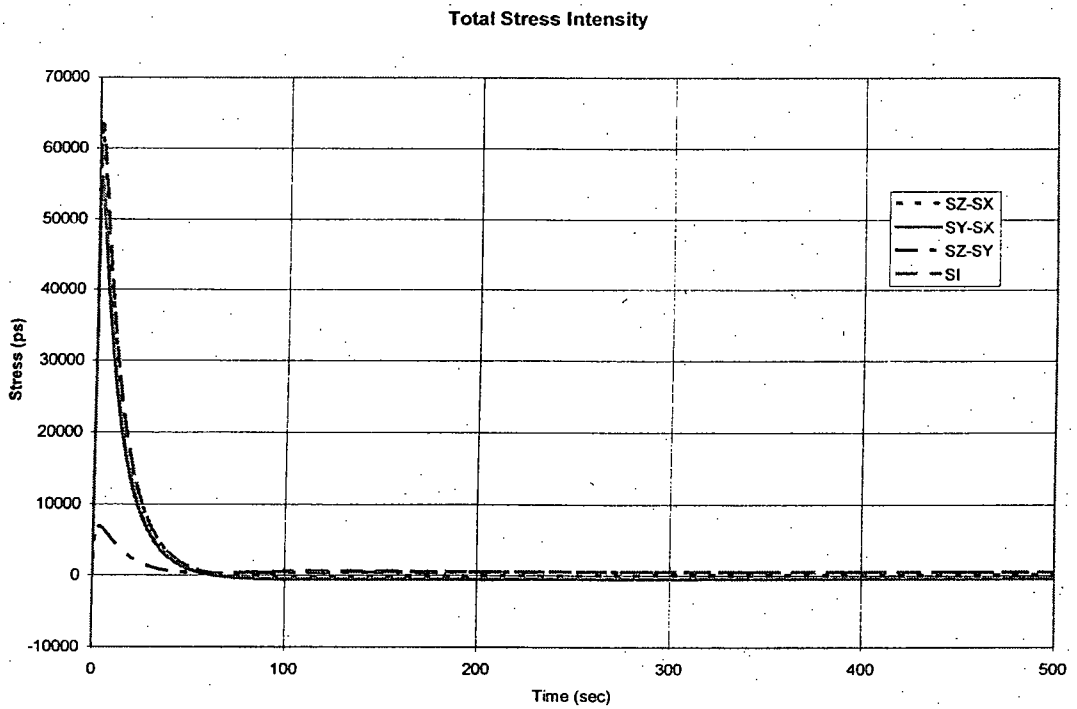


Figure 10: Safe End 100% Flow Total Stress Intensity

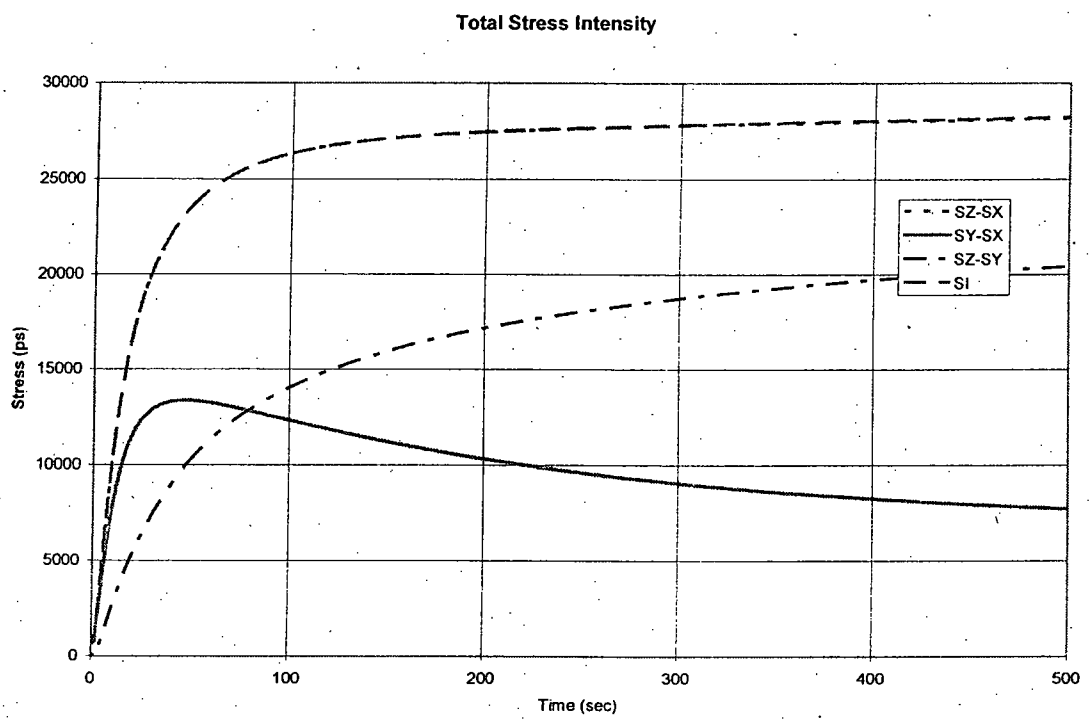


Figure 11: Blend Radius 100% Flow Total Stress Intensity

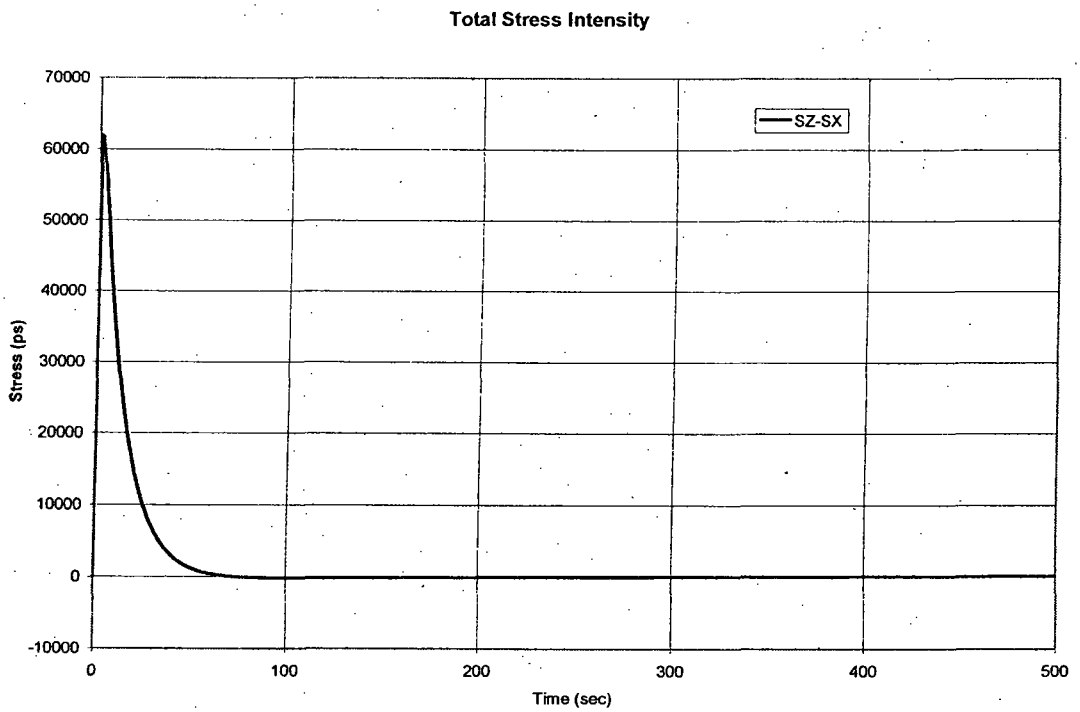


Figure 12: Safe End Total Stress History for 100% Flow

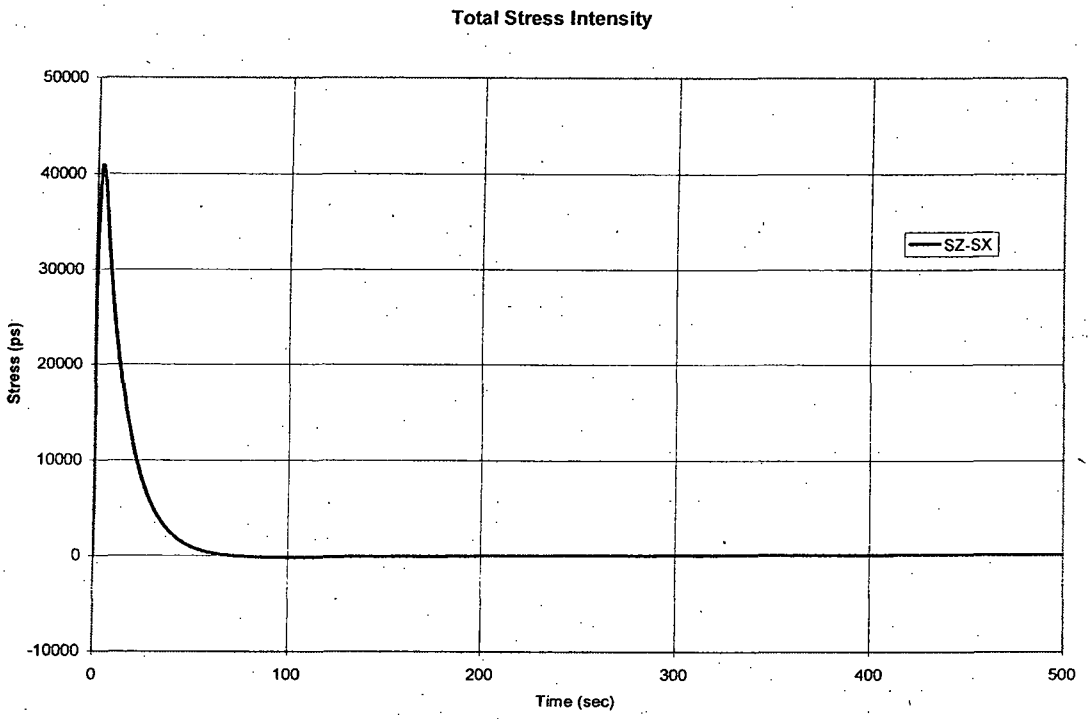


Figure 13: Safe End Membrane Plus Bending Stress History for 100% Flow

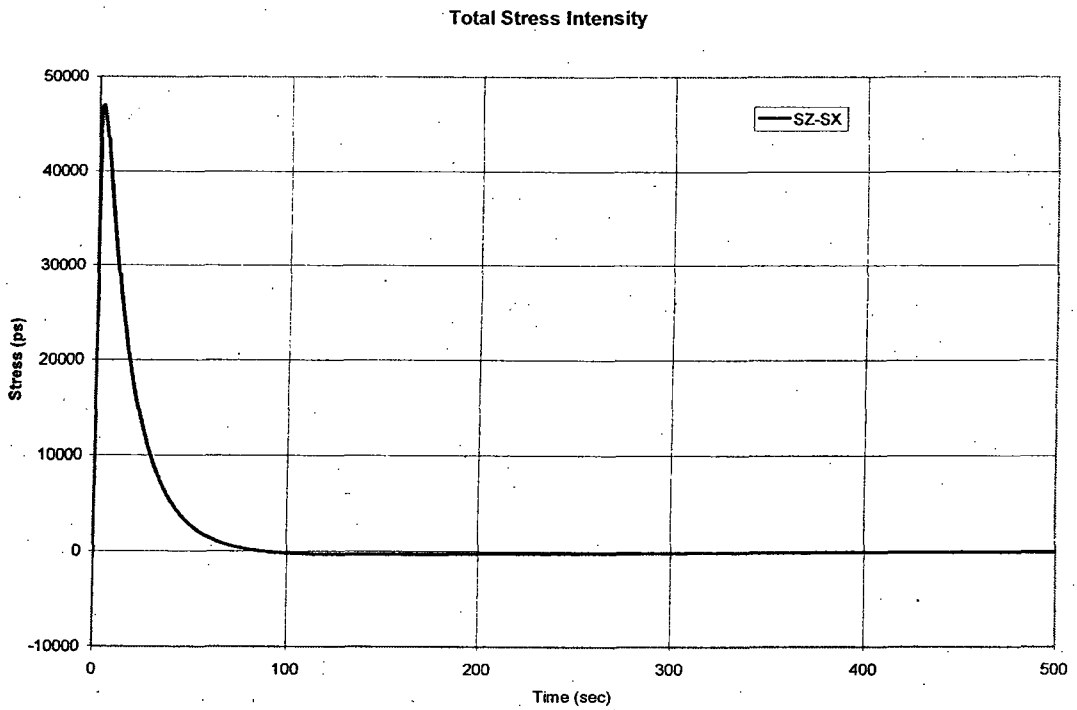


Figure 14: Safe End Total Stress History for 40% Flow

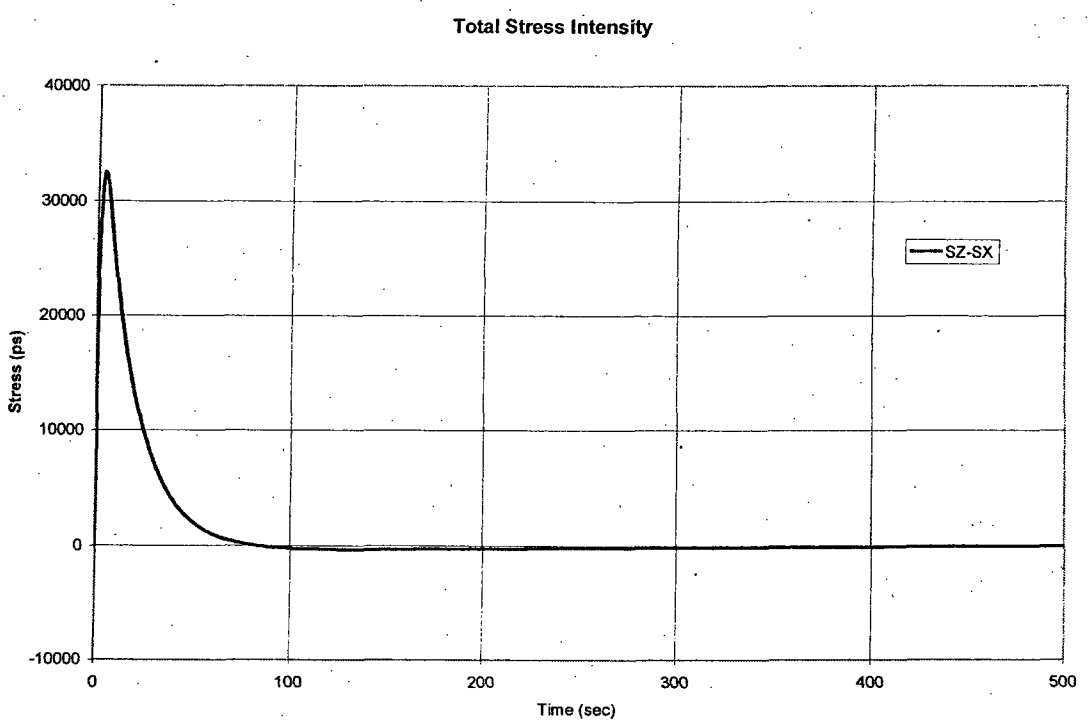


Figure 15: Safe End Membrane Plus Bending Stress History for 40% Flow

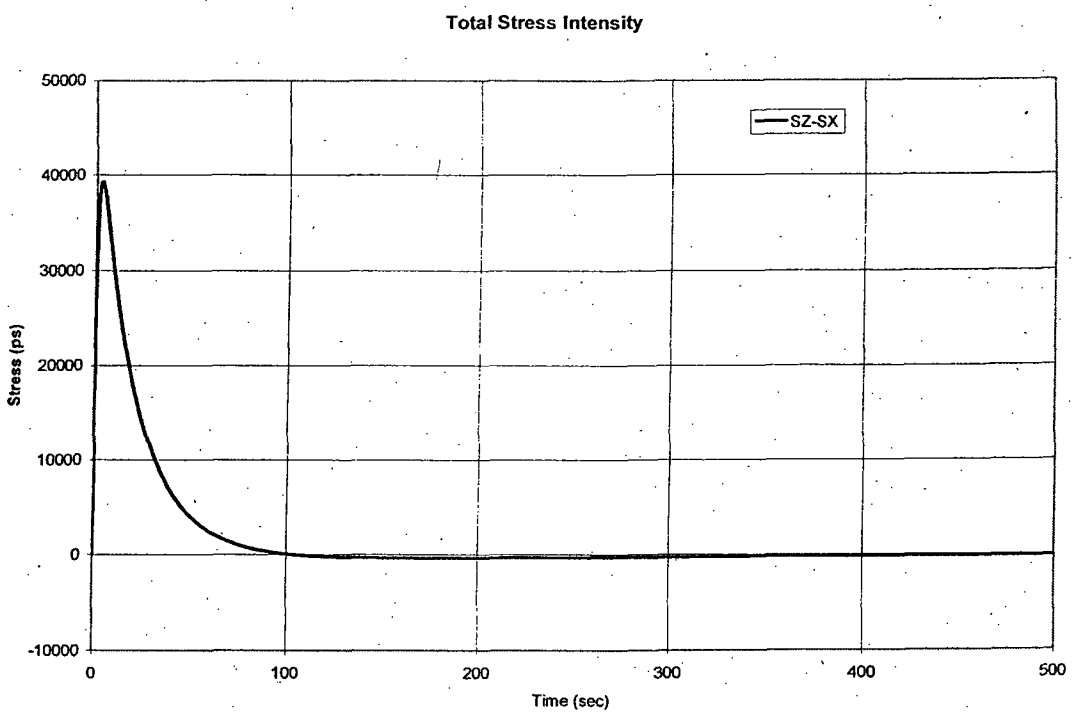


Figure 16: Safe End Total Stress History for 25% Flow

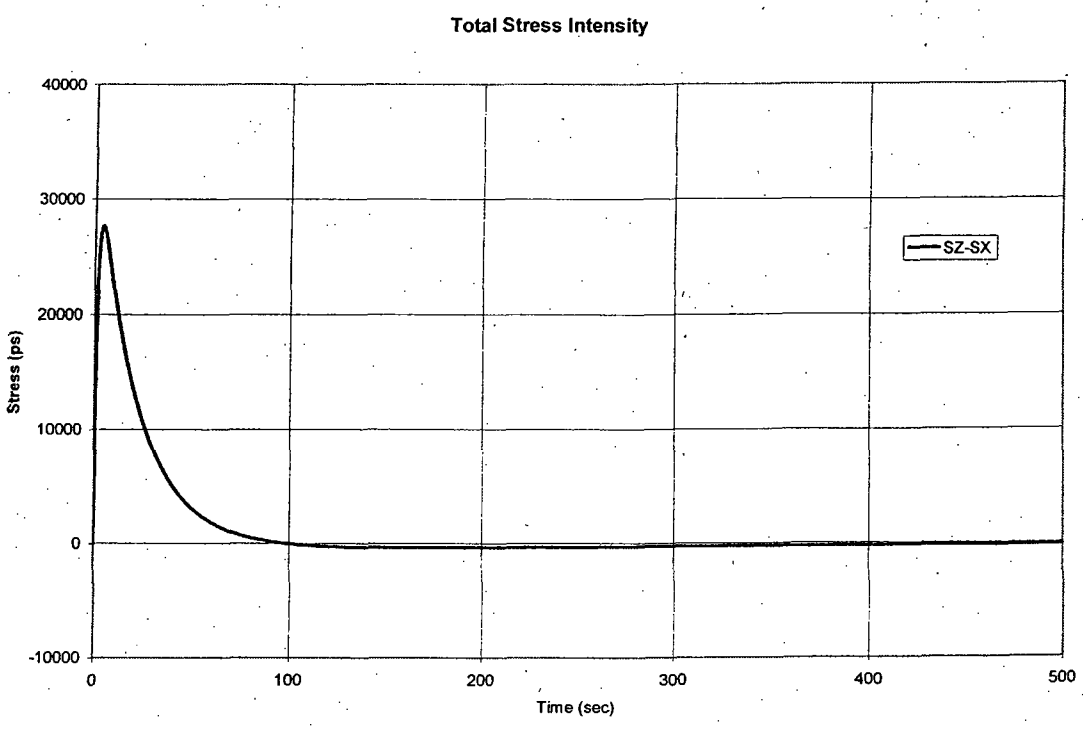


Figure 17: Safe End Membrane Plus Bending Stress History for 25% Flow

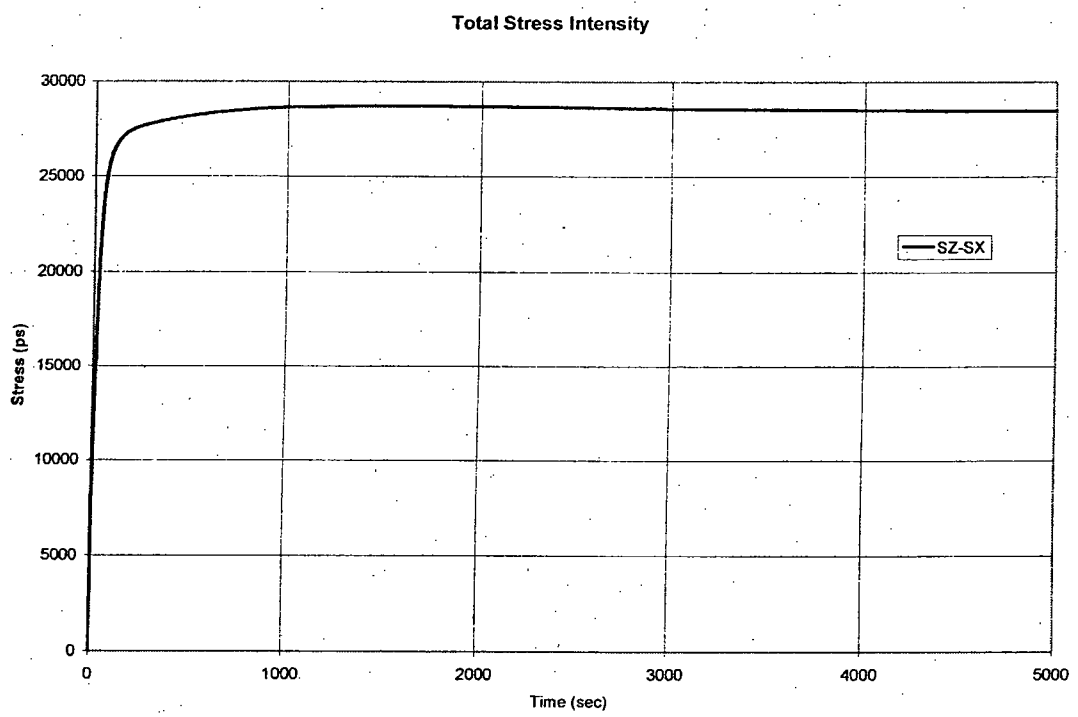


Figure 18: Blend Radius Total Stress History for 100% Flow

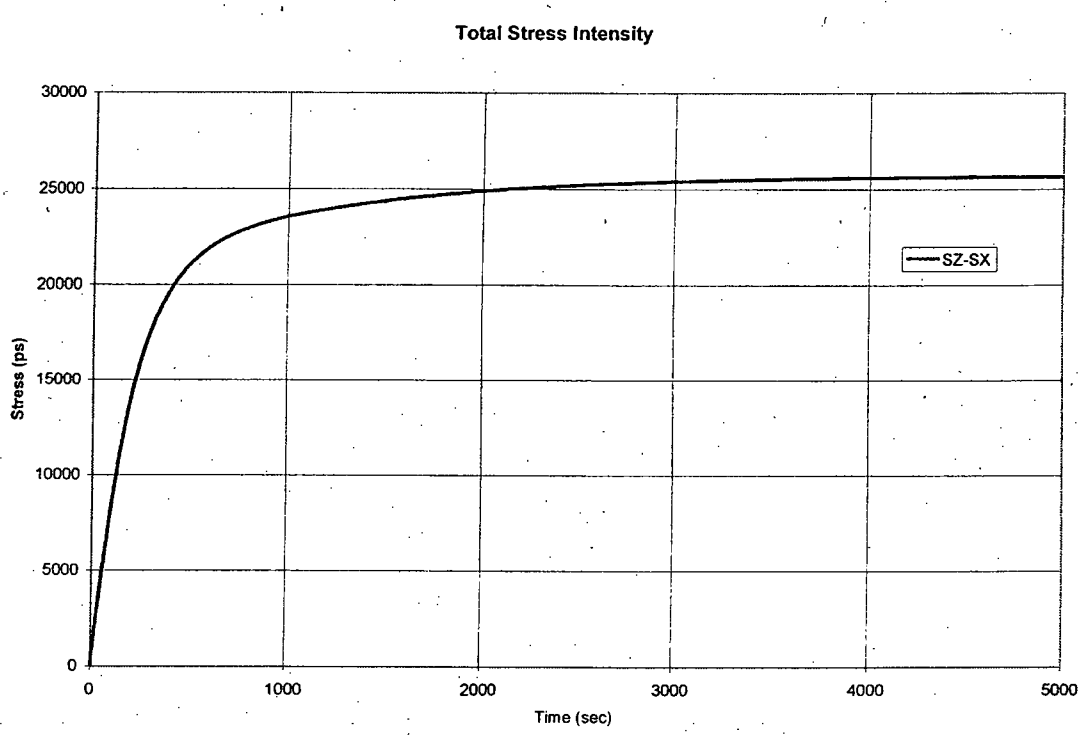


Figure 19: Blend Radius Membrane Plus Bending Stress History for 100% Flow

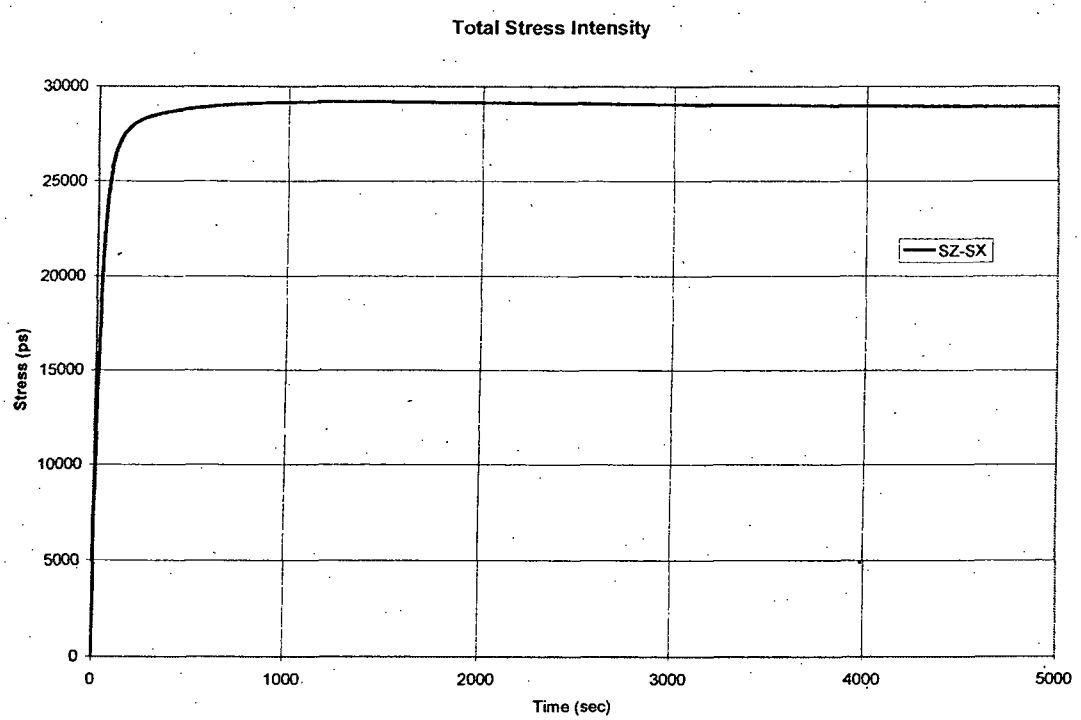


Figure 20: Blend Radius Total Stress History for 40% Flow

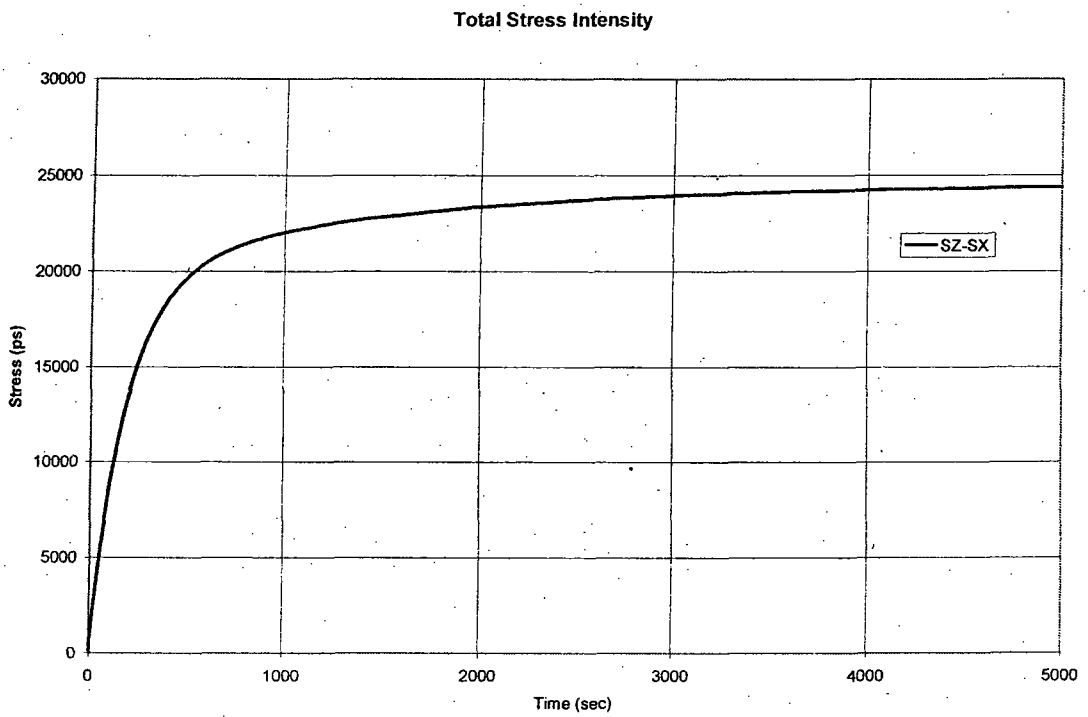


Figure 21: Blend Radius Membrane Plus Bending Stress History for 40% Flow

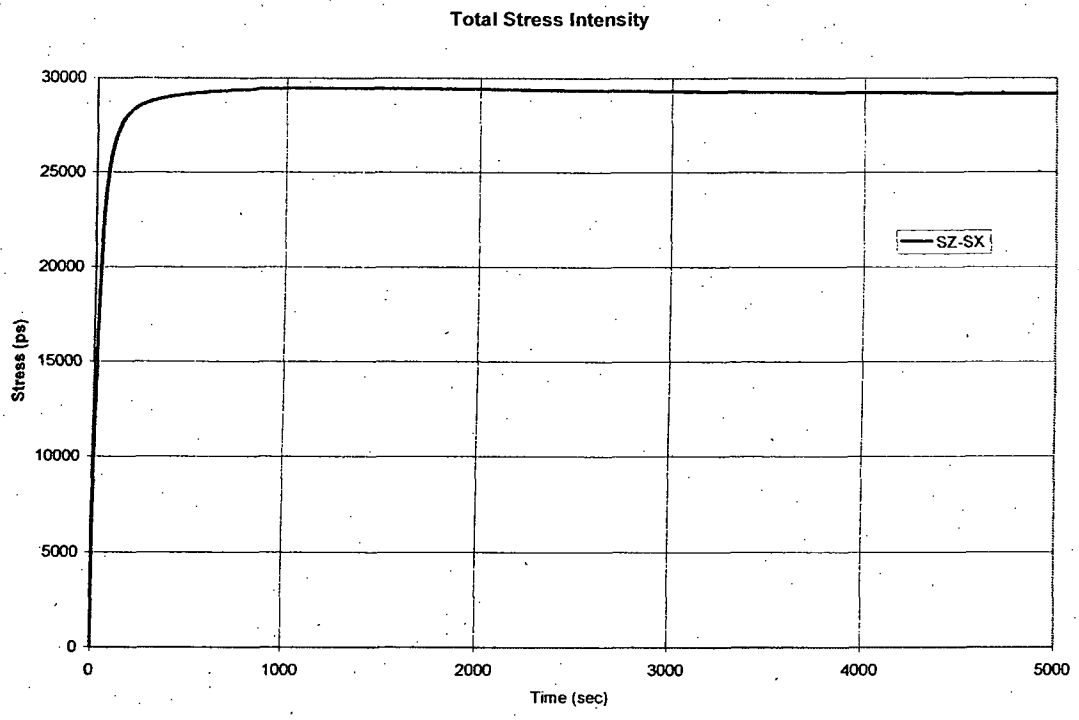


Figure 22: Blend Radius Total Stress History for 25% Flow

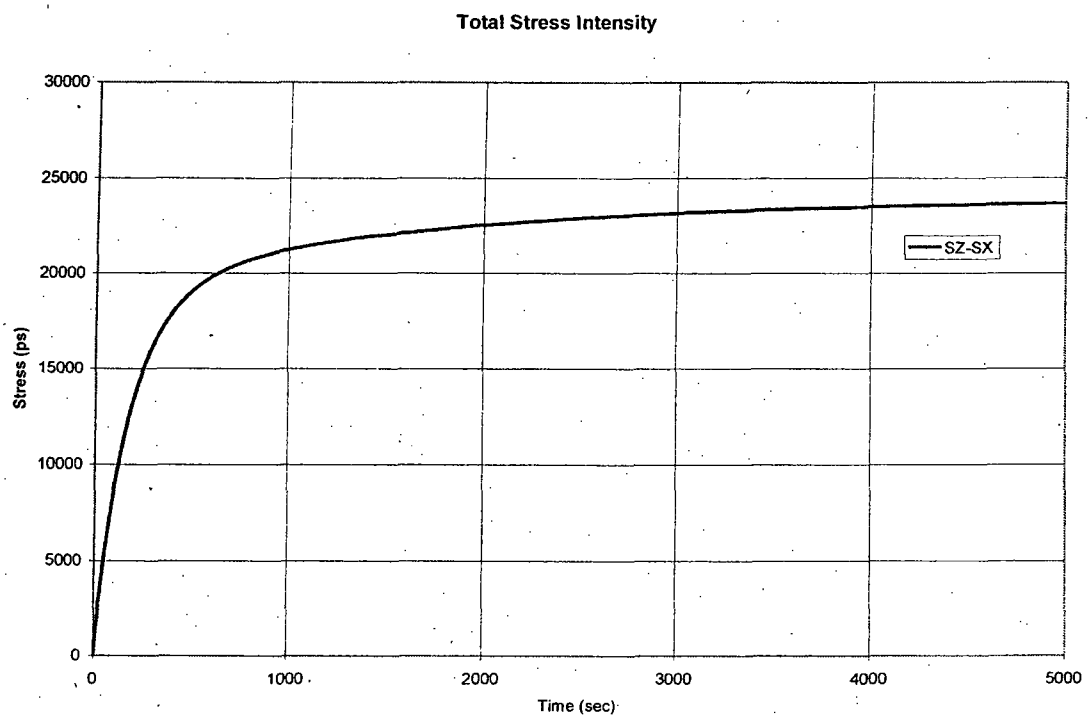


Figure 23: Blend Radius Membrane Plus Bending Stress History for 25% Flow

APPENDIX A

FINITE ELEMENT ANALYSIS FILES

FWP_VY.INP	Input File for Pressure Load	In Computer files
FWT_VY_100.INP	Input File for 100% Flow Thermal Analysis	In Computer files
FWS_VY_100.INP	Input File for 100% Flow Stress Analysis	In Computer files
FWT_VY_40.INP	Input File for 40% Flow Thermal Analysis	In Computer files
FWS_VY_40.INP	Input File for 40% Flow Stress Analysis	In Computer files
FWT_VY_25.INP	Input File for 25% Flow Thermal Analysis	In Computer files
FWS_VY_25.INP	Input File for 25% Flow Stress Analysis	In Computer files
PSE.OUT	Stress Output at Safe End with Pressure Load	In Computer files
PBLEND.OUT	Stress Output at Blend Radius with Pressure Load	In Computer files
PVESS.OUT	Stress Output at Vessel with Pressure Load	In Computer files
#FSE.OUT	Stress Output at Safe End	In Computer files
#FBLEND.OUT	Stress Output at Blend Radius	In Computer files
#FSE_INSIDE.RED	Stress Extracted at Safe End	In Computer files
#FBLEND_INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
#FSE_T-Green.XLS	Green Function with Total Stress at Safe End	In Computer files
#FSE_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Safe End	In Computer files
#FBR_T-Green.XLS	Green Function with Total Stress at Blend Radius	In Computer files
#FBR_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Blend Radius	In Computer files

Where # is H, M, L meaning 100%, 40%, and 25% flow rate, respectively.

Structural Integrity Associates, Inc.	File No.: VY-16Q-302 EXHIBIT B
CALCULATION PACKAGE	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Feedwater Nozzle

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the feedwater nozzle. Two locations will be analyzed for fatigue acceptance: the safe end (SA508 Class 1) and the blend radius (SA508 Class 2). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. A revised cumulative fatigue factor (CUF) will be determined for both locations, the nozzle forging and safe end, respectively. In the end, the environmental fatigue usage factors will be determined for both locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the feedwater nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the feedwater nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [11] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [10]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [12] methodology was originally used to evaluate the feedwater nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original feedwater nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more accurate CUF.

For the feedwater nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's

Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [2]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 2. These five step changes produce the five successive stress responses in the second plot shown in Figure 2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor feedwater nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for thee three feedwater flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility programs will be used to perform the fatigue analysis. The first two calculate stresses in response to transients. The transients analyzed are those described in the thermal cycle diagrams [3] for the feedwater nozzle. These transients are shown in Figures 4 – 20. The temperatures and pressures for these transients have been modified to account for power uprate [4]. The power uprate pressures and temperatures were used for this analysis. The last program calculates fatigue based on the stress output. The three programs are STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [5] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.
The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m, Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.

3.0 ANALYSIS

The fatigue analysis involves the preparing of input files for, and running of three programs verified and described in Reference [5]. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P_V.EXE. The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 12 stress history functions obtained from Reference [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for each of the following flow conditions; 100%, 40%, and 25% flow.
- Green.cfg is configured as described in Reference [5].
- Transnt.inp. These files are created to represent the transients shown on the thermal cycle diagrams and redefined by power uprate. Note that transients 12, 13, and 15 are nearly identical on the thermal cycle diagram [3] and the results from running transient 12 will be used for all three transients. Transient 16, 17 and 18 will not be considered since there is no temperature change. Tables 1 and 2 show the thermal history used to represent each transient. Based upon the thermal cycle diagram for the feedwater nozzle [3], the transients are split into the following groups based upon flow rate:
    - Transients 3, 20, 20A, and 21-23 are run at 25% flow. Although Reference [3] shows 15% flow rate, it is conservative to use 25% flow rate for these transients. Transient 20, Hot Standby, is split up into two parts. The first portion is "Heatup portion" and the second portion is "Feedwater Injection portion" that are defined from Reference [3].
    - Transient 11 is run at 40% flow. Transient 11 starts off and ends at 100% flow.
    - Transients 5, 6, 9, 10, and 19 are run at 100% flow.
    - Transient 4 is run at 100% flow only to obtain the last stress point. The remainder of the stress points for transient 4 is obtained from the 25% flow stress results. The results are pulled from the two flow case results based upon the flow rates defined in the thermal cycle diagram [3].
    - Transients 12, 13, 14 and 15 were run at 100% flow. Heat transfer coefficients were not re-calculated for the 1 minute intervals each of these transients is at 110% flow. The effect of this small flow rate increase for such a relatively short duration should be minor.
    - Transients 1, 2, 24, and 25 are set as no thermal stress due to very small temperature changes (70°F to 100°F) at these transients.

3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

The program P-V.exe is then run to extract the peaks and valleys from the STRESS.OUT file produced by the STRESS.EXE program. The only input required for this program is STRESS.OUT and it outputs all the peaks and valleys to P-V.OUT. Columns 2 through 5 of Tables 4 (for the blend radius) and 5 (for the safe end) show the final peak and valley output. The pressure for column 6 is then filled in using the thermal cycle diagrams. Pressure and piping loads have to be added to the peak and valley points to calculate the final stress values used for fatigue analysis.

3.3 Pressure Load

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [1]. These values are as follows:

Pressure stress for the safe end:

- 8693 psi membrane plus bending stress intensity.
- 8891 psi total linearized stress intensity.

Pressure stress for the blend radius:

- 36653 psi membrane plus bending stress intensity.
- 37733 psi total linearized stress intensity.

These pressure stress values for each location were linearly scaled with pressure. The actual pressure for column 6 of Tables 4 and 5 is obtained from Tables 1 and 2. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4 and 5.

The pressure stress is combined with the thermal and piping loads to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 3.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

For Cut I:

$$(M_x)_1 = M_x - F_y L_1$$
$$(M_y)_1 = M_y + F_x L_1$$

For Cut II:

$$(M_x)_2 = M_x - F_y L_2$$
$$(M_y)_2 = M_y + F_x L_2$$

The total bending moment and shear loads are obtained using the equations below:

For Cut I:

$$M_{xy} = \sqrt{(M_x)_1^{\,2} + (M_y)_1^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_1^{\,2} + (F_y)_1^{\,2}}$$

For Cut II:

$$M_{xy} = \sqrt{(M_x)_2^{\,2} + (M_y)_2^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_2^{\,2} + (F_y)_2^{\,2}}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$N_z = \frac{1}{\pi R_N}\left[\frac{1}{2}F_z + \frac{M_{xy}}{R_N}\right]$$
$$q_N = \frac{1}{\pi R_N}\left[F_{xy} - \frac{M_z}{2R_N}\right]$$

To determine the primary stresses, P_M, due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$
$$(P_M)_\theta = \frac{Pa_N}{t_N}$$
$$(P_M)_R = -P$$
$$\tau_M = \frac{q_N}{t_N}$$
$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\,2}}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\,2}}$$

Because pressure was considered separately in this analysis, the equations used for Cut I are valid for Cut II.

where: L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.
M_{xy} = The maximum bending moment in the xy plane.
F_{yx} = The maximum shear force in the xy plane.
N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
q_N = The shear force per inch of circumference applied to the nozzle.
R_N = The mid-wall nozzle radius.

Since the pressure was considered separately in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{Nz}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{Nz}{2t_N}\right)^2 + (\tau_M)_{z\theta}^{\ 2}}$$

Per Reference [6], the feedwater nozzle piping loads are as follows:

F_x = 3,000 lbs	M_x = 28,000 ft-lb = 336,000 in-lb
F_y = 15,000 lbs	M_y = 13,000 ft-lb = 156,000 in-lb
F_z = 3,200 lbs	M_z = 40,000 ft-lb = 480,000 in-lb

The loads are applied at the connection of the piping and safe end. Therefore, the L_1 is equal to 12.0871 inches and the L_2 is equal to 27.572 inches. The calculations for the safe end and blend radius are shown in Table 3. The first cut location is the same as the Green's Function cross section per [1] at the safe end, and the second cut is from Node 645 (outside) to Node 501 (inside). The maximum stress intensities due to piping loads are 5707.97 psi at the safe end and 265.47 psi at the blend radius, respectively. The piping load sign is set as the same as the thermal stress sign.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F. The scaled piping stress values

are shown in columns 9 and 10 of Tables 4 and 5. Columns 11 and 12 of Tables 4 and 5 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient [3]:

Column 13 in Tables 4 and 5 shows the number of cycles associated with each transient. The number of cycles for 60 years was obtained from Reference [3].

The program FATIGUE.EXE performs the "ASME Code style" peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius	Safe End
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel) [10]	3.0 & 0.2 (carbon steel) [10]
Design Stress Intensity Values, S_m	26700 psi [8] @ 600°F	17800 psi [8] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [10]	30.0×10^6 psi [10]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi	28.1×10^6 psi
The Geometric Stress Concentration Factor K_t	1.0	1.34 [7, page 35 of S4]

The results of the fatigue analyses are presented in Tables 6 and 7 for the blend radius and safe end for 60 years, respectively.

The results described are contained in EXCEL files *BRresults.xls* and *SEresults.xls*, which are contained in the computer files.

4.0 FATIGUE USAGE RESULTS

The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years.

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

In the response to NRC request for additional information (RAI) 4.3-H-02, VYNPS states that they have conservatively assumed that fatigue cracks may be present in the clad. VYNPS manages this cracking by performing periodic inspections that were implemented in response to Generic Letters 80-095 and

81-11, and NUREG-0619. The inspection frequency is based on the calculated fatigue crack growth of a postulated flaw in the nozzle inner blend radius. The VYNPS fatigue crack growth calculation uses methods in compliance with GE BWR Owners Group Topical Report "Alternate BWR Feedwater Nozzle Inspection Requirements", GE-NE-523-A71-0594, Revision 1, August 1999 and the associated NRC Final Safety Evaluation (TAC No. MA6787) dated March 10, 2000. The NRC has reviewed and approved this approach to handling FW nozzle inner blend radius cracking (Letter D.H. Dorman (USNRC) to D.A. Reid (VYNPC), Subject: Evaluation of Request for Relief from NUREG-0619 for VYNPS dated 2/6/95, (TAC No. M88803)).

The analysis performed for the feedwater nozzle calculated fatigue in the blend radius base metal, not the clad. This is consistent with the VYNPS position stated in the response to RAI 4.3-H-02, and is also consistent with ASME Code methodology since cladding is structurally neglected in fatigue analyses, per ASME Code, Section III, NB-3122.3.

Per Reference [9], the dissolved Oxygen (DO) calculation shows the overall HWC availability is 47%. This means the time ratio under NWC (pre-HWC) is 53%.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are all 1.74 from Table 3 of Reference [9]. It results in an EAF adjusted CUF of 1.74 x 0.1471 = 0.2560 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 1.74.

For the blend radius location, the environmental fatigue factors for post-HWC and pre-HWC are 11.14 and 8.82 from Table 4 of Reference [9]. These results in an EAF adjusted CUF of (11.14 x 53% + 8.82 x 47%) x 0.0636 = 0.6392 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 10.0496.

6.0 REFERENCES

1. SI Calculation No. VY-16Q-301, Revision 0, "Feedwater Nozzle Stress History Development for Green Functions."
2. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
3. Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
4. GE Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," SI File No. VY-05Q-236.
5. Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
6. GE Drawing No. 919D294, Revision 11, Sht. No. 7, "Reactor Vessel," SI File No. VY-05Q-241.
7. Chicago Bridge & Iron Company Contractor 9-6201, Revision 2, "Section S4, Stress Analysis Feedwater Nozzle Vermont Yankee Reactor Vessel," SI File No. VY-05Q-238.

8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
9. SI Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell Bottom Head."
10. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III Subsection NB, 1998 Edition, 2000 Addenda.
11. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
12. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, Article 4, 1965 Edition with Winter 1966 Addenda.

Table 1: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0
123 Cycles	10	70	10	0
2. Design	0	70		0
HYD Test	1080	100	1080	0
120 Cycles	1680	100	600	1100
	5280	100	3600	1100
	5880	100	600	50
	10880	100	5000	50
3. Startup	0	100		50
300 Cycles	16164	549	16164	1010
LF_25	21164	549	5000	1010
4. Turbine Roll	0	549		1010
and Increased to	1	100	1	1010
Rated	1801	100	1800	1010
Power	1802	260	1	1010
300 cycles	3602	392	1800	1010
LF_25, HF_100	8602	392	5000	1010
5. Daily	0	392		1010
Reduction	900	310	900	1010
75% Power	2700	310	1800	1010
10,000 Cycles	3600	392	900	1010
HF_100	8600	392	5000	1010
6. Weekly Reduct	0	392		1010
50% Power	1800	280	1800	1010
2,000 Cycles	3600	280	1800	1010
HF_100	5400	392	1800	1010
	10400	392	5000	1010
9. Turbine	0	392		1010
Trip at 25%	1800	265	1800	1010
Power	1980	265	180	1010
10 Cycles	2340	90	360	1010
HF_100	2520	90	180	1010
	3420	265	900	1010
	3600	265	180	1010
	5400	392	1800	1010
	10400	392	5000	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
10. FW Heater	0	392		1010
Bypass	90	265	90	1010
70 Cycles	1890	265	1800	1010
HF_100	2070	392	180	1010
	7070	392	5000	1010
11. Loss of	0	392		1010
FW Pumps	1	565	1	1010
10 Cycles	3.5	565	2.5	1190
MF_40, HF_100	4.5	50	1	1185
	13.5	50	9	1135
	184.5	50	171	1135
	1564.5	440	1380	1135
	1565.5	565	1	1135
	2165.5	565	600	1135
	2166.5	50	1	1135
	2346.5	50	180	885
	5406.5	440	3060	1055
	5407.5	565	1	1055
	6727.5	565	1320	1135
	6728.5	50	1	1135
	7148.5	50	420	675
	7448.5	300	300	675
	11048.5	400	3600	232
	16411.5	549	5363	885
	16412.5	549	1	1010
	18212.5	549	1800	1010
	18213.5	100	1	1010
	20013.5	100	1800	1010
	20014.5	260	1	1010
	21814.5	392	1800	1010
	26814.5	392	5000	1010
12. Turbine	0	392		1010
Generator Trip	10	392	10	1135/1375[2]
60 Cycles	15	392	5	1135/1375[2]
13. Reactor	30	392	15	940
Overpressure	90	275	60	940
1 Cycles	990	100	900	940
15. Other	2790	100	1800	940
SCRAMs	2791	260	1	940
228 cycles	3210	291	419	1010
HF_100	4591	392	1381	1010
	9591	392	5000	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
14. SRV	0	392		1010
Blowdown	60	275	60	885
1 Cycles	960	100	900	50
HF_100	5960	100	5000	50
19. Reduction to 0%	0	392		1010
Power 300 Cycles	1800	265	1800	1010
HF_100	6800	265	5000	1010
20. Hot Standby	0	265		1010
(Heatup Portion)	1	440	1	1010
300 Cycles	3925	549	3924	1010
LF_25	8925	549	5000	1010
20A. Hot Standby	0	549		1010
(FW Injection Portion)	1	100	1	1010
300 Cycles	181	100	180	1010
LF_25	241	290	60	1010
	451	549	210	1010
	5451	549	5000	1010
21-23. Shutdown	0	549		1010
300 Cycles	6264	375	6264	50
LF_25	6864	330	600	50
	15144	100	8280	50
	20144	100	5000	50
24. Hydrostatic	0	100		50
Test	600	100	600	1563
1 Cycles	1200	100	600	1563
	1800	100	600	50
	2400	100	600	50
25. Unbolt	0	100		0
123 Cycles	1080	70	1080	0
	6080	70	5000	0

Note: 1. The indicated time or pressure was assumed.
2. 1375 psi is for Transient 13 only.

Table 2: Safe End Transient

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0
123 Cycles	10	70	10	0
2. Design	0	70		0
HYD Test	1080	100	1080	0
120 Cycles	1680	100	600	1100
	5280	100	3600	1100
	5880	100	600	50
	6380	100	500	50
3. Startup	0	100		50
300 Cycles	16164	549	16164	1010
LF_25	16664	549	500	1010
4. Turbine Roll	0	549		1010
and Increased to	1	100	1	1010
Rated	1801	100	1800	1010
Power	1802	260	1	1010
300 cycles	3602	392	1800	1010
LF_25, HF_100	4102	392	500	1010
5. Daily	0	392		1010
Reduction	900	310	900	1010
75% Power	2700	310	1800	1010
10,000 Cycles	3600	392	900	1010
HF_100	4100	392	500	1010
6. Weekly Reduct	0	392		1010
50% Power	1800	280	1800	1010
2,000 Cycles	3600	280	1800	1010
HF_100	5400	392	1800	1010
	5900	392	500	1010
9. Turbine	0	392		1010
Trip at 25%	1800	265	1800	1010
Power	1980	265	180	1010
10 Cycles	2340	90	360	1010
HF_100	2520	90	180	1010
	3420	265	900	1010
	3600	265	180	1010
	5400	392	1800	1010
	5900	392	500	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
10. FW Heater	0	392		1010
Bypass	90	265	90	1010
70 Cycles	1890	265	1800	1010
HF_100	2070	392	180	1010
	2570	392	500	1010
11. Loss of	0	392		1010
FW Pumps	1	565	1	1010
10 Cycles	3.5	565	2.5	1190
MF_40, HF_100	4.5	50	1	1185
	13.5	50	9	1135
	184.5	50	171	1135
	1564.5	440	1380	1135
	1565.5	565	1	1135
	2165.5	565	600	1135
	2166.5	50	1	1135
	2346.5	50	180	885
	5406.5	440	3060	1055
	5407.5	565	1	1055
	6727.5	565	1320	1135
	6728.5	50	1	1135
	7148.5	50	420	675
	7448.5	300	300	675
	11048.5	400	3600	232
	16411.5	549	5363	885
	16412.5	549	1	1010
	18212.5	549	1800	1010
	18213.5	100	1	1010
	20013.5	100	1800	1010
	20014.5	260	1	1010
	21814.5	392	1800	1010
	22314.5	392	500	1010
12. Turbine	0	392		1010
Generator Trip	10	392	10	1135/1375[2]
60 Cycles	15	392	5	1135/1375[2]
13. Reactor	30	392	15	940
Overpressure	90	275	60	940
1 Cycles	990	100	900	940
15. Other	2790	100	1800	940
SCRAMs	2791	260	1	940
228 cycles	3210	291	419	1010
HF_100	4591	392	1381	1010
	5091	392	500	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
14. SRV	0	392		1010
Blowdown	60	275	60	885
1 Cycles	960	100	900	50
HF_100	1460	100	500	50
19. Reduction to 0%	0	392		1010
Power 300 Cycles	1800	265	1800	1010
HF_100	2300	265	500	1010
20. Hot Standby	0	265		1010
(Heatup Portion)	1	440	1	1010
300 Cycles	3925	549	3924	1010
LF_25	4425	549	500	1010
20A. Hot Standby	0	549		1010
(FW Injection Portion)	1	100	1	1010
300 Cycles	181	100	180	1010
LF_25	241	290	60	1010
	451	549	210	1010
	951	549	500	1010
21-23. Shutdown	0	549		1010
300 Cycles	6264	375	6264	50
LF_25	6864	330	600	50
	15144	100	8280	50
	15644	100	500	50
24. Hydrostatic	0	100		50
Test	600	100	600	1563
1 Cycles	1200	100	600	1563
	1800	100	600	50
	2400	100	600	50
25. Unbolt	0	100		0
123 Cycles	1080	70	1080	0
	1580	70	500	0

Note: 1. These transients are the same as in Table 1 with the exception of the 500 second steady state time increment that is used. The transients in Table 1 are plotted using a 5000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.
2. The indicated time or pressure was assumed.
3. 1375 psi is for Transient 13 only.

Table 3: Maximum Piping Stress Intensity Calculations

Safe End External Piping Loads		
Parameters		
F_x =	3.00	kips
F_y =	15.00	kips
F_Z =	3.20	kips
M_x =	336.00	in-kips
M_y =	156.00	in-kips
M_Z =	480.00	in-kips
OD=	11.86	in
ID=	10.409	in
R_N =	5.57	in
L =	12.09	in
t_N =	0.72	in
$(M_x)_1$ =	154.69	in-kips
$(M_y)_1$ =	192.26	in-kips
M_{xy} =	246.77	in-kips
F_{xy} =	15.30	kips
N_Z =	2.63	kips/in
q_N =	-1.59	kips/in
Primary Membrane Stress Intensity		
PM_Z =	3.63	ksi
τ =	-2.20	ksi
SI_{max} =	5.71	ksi
SI_{max} =	5707.97	psi

Blend Radius External Piping Loads		
Parameters		
F_x =	3.00	kips
F_y =	15.00	kips
F_Z =	3.20	kips
M_x =	336.00	in-kips
M_y =	156.00	in-kips
M_Z =	480.00	in-kips
OD=	22.67	in
ID=	10.750	in
R_N =	8.35	in
L =	27.57	in
t_N =	5.96	in
$(M_x)_2$ =	-77.58	in-kips
$(M_y)_2$ =	238.72	in-kips
M_{xy} =	251.01	in-kips
F_{xy} =	15.30	kips
N_Z =	1.21	kips/in
q_N =	-0.51	kips/in
Primary Membrane Stress Intensity		
PM_Z =	0.20	ksi
τ =	-0.09	ksi
SI_{max} =	0.27	ksi
SI_{max} =	265.47	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 4: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
2	0	0	0	70	0	0	0	0	0	0.00	0.00	120
	1680	0	0	100	1100	41506.3	40318.3	15.77042	15.77042	41522.07	40334.07	120
	10880	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	120
3	0	29166	23676	100	50	1886.65	1832.65	15.77042	15.77042	31068.42	25524.42	300
	16782.8	-3577	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34281.53	33629.73	300
	21164	-3532	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34326.53	33629.73	300
4	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	1801.9	29465	22266	244.004	1010	38110.33	37019.53	91.47053	91.47053	67666.80	59377.00	300
	8602	7720	6749	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43937.80	300
5	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
	2229.8	13598	11941	311.002	1010	38110.33	37019.53	126.6901	126.6901	51835.02	49087.22	10000
	8600	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
6	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
	2820.3	15742	13892	280.691	1010	38110.33	37019.53	110.7562	110.7562	53963.09	51022.29	2000
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
9	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	2524	29006	23417	118.311	1010	38110.33	37019.53	25.39616	25.39616	67141.73	60461.93	10
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
10	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
	1632.4	16828	14701	267.399	1010	38110.33	37019.53	103.7688	103.7688	55042.10	51824.30	70
	7070	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
11	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	3.5	6620	6632	565	1190	44902.27	43617.07	260.2119	260.2119	51782.48	50509.28	10
	4.5	6190	6608	50	1185	44713.61	43433.81	10.51361	10.51361	50914.12	50052.32	10
	194.5	31720	21067	109.348	1135	42826.96	41601.16	20.68448	20.68448	74567.64	62688.84	10
	2166.3	-4761	-1859	513.483	972	36676.48	35626.72	-233.1304	-233.1304	31682.35	33534.59	10
	2362.5	31268	22070	102.255	1010	38110.33	37019.53	16.95583	16.95583	69395.29	59106.49	10
	6728.3	-4913	-3149	513.448	1010	38110.33	37019.53	-233.112	-233.112	32964.22	33637.42	10
	7149.9	32114	21472	83.333	1010	38110.33	37019.53	7.0089	7.0089	70231.34	58498.54	10
	18213.3	-3565	-3162	503.978	1010	38110.33	37019.53	-228.1338	-228.1338	34317.20	33629.40	10
	19122.6	29156	23083	100.048	1010	38110.33	37019.53	15.79565	15.79565	67282.13	60118.33	10
	26814.5	7720	6410	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43598.80	10
12	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	60
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	60
	2033.7	28648	25301	132.007	940	35469.02	34453.82	32.59588	32.59588	64149.62	59787.42	60
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
13	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	10	7720	6752	392	1375	51882.88	50397.88	169.2692	169.2692	59772.14	57319.14	1
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	1
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	1
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
14	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	5960	28487	25650	100	50	1886.65	1832.65	15.77042	15.77042	30389.42	27498.42	1
15	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	228
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	228
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	228
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
19	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	300
	6800	16752	14971	265	1010	38110.33	37019.53	102.5077	102.5077	54964.84	52093.04	300
20	0	17151	13815	265	1010	38110.33	37019.53	102.5077	102.5077	55363.84	50937.04	300
	8925	-3531	-3146	549	1010	38110.33	37019.53	-251.801	-251.801	34327.53	33621.73	300
20A	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	183	28102	12153	233	1010	38110.33	37019.53	85.68595	85.68595	66298.02	49258.22	300
	5451	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
21-23	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	20144	29168	23656	100	50	1886.65	1832.65	15.77042	15.77042	31070.42	25504.42	300
24	0	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
	600	0	0	100	1563	58976.68	57288.64	15.77042	15.77042	58992.45	57304.41	1
	2400	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
25	0	0	0	100	0	0	0	15.77042	15.77042	15.77	15.77	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

Table 4: Blend Radius Stress Summary (Continue)

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 1.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 37733)/1000 [Table3, 1].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 36653)/1000 [Table 3, 1].
Column 9: Total external stress from calculation in Table 3, 265.47 psi*(Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 5: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
2	0	0	0	70	0	0	0	0	0	0.00	0.00	120
	1680	0	0	100	1100	9780.1	9562.3	339.0875	339.0875	10119.19	9901.39	120
	6960	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	120
3	0	-170	-165	100	50	444.55	434.65	-339.0875	-339.0875	-64.54	-69.44	300
	153.2	-235	-212	104.256	50	444.55	434.65	-387.1927	-387.1927	-177.64	-164.54	300
	16328.2	2	3	549	1010	8979.91	8779.93	5414.097	5414.097	14396.01	14197.03	300
	16664	-1	0	549	1010	8979.91	8779.93	-5414.097	5414.097	3564.81	14194.03	300
4	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	3.6	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	1804.6	-15889	-11224	260.286	1010	8979.91	8779.93	-2150.787	-2150.787	-9059.88	-4594.86	300
	4102	21	23	392	1010	8979.91	8779.93	3639.539	3639.539	12640.45	12442.47	300
5	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
	900.1	244	189	310	1010	8979.91	8779.93	2712.7	2712.7	11936.61	11681.63	10000
	3600	-169	-110	392	1010	8979.91	8779.93	-3639.539	-3639.539	5171.37	5030.39	10000
	3684.4	33	35	392	1010	8979.91	8779.93	3639.539	3639.539	12652.45	12454.47	10000
	4100	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
6	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
	1800.1	196	159	280	1010	8979.91	8779.93	2373.612	2373.612	11549.52	11312.54	2000
	5400.2	-108	-68	392	1010	8979.91	8779.93	-3639.539	-3639.539	5232.37	5072.39	2000
	5496.6	29	31	392	1010	8979.91	8779.93	3639.539	3639.539	12648.45	12450.47	2000
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
9	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
	97.3	180	137	385.135	1010	8979.91	8779.93	3561.945	3561.945	12721.85	12478.87	10
	1884.1	63	65	265	1010	8979.91	8779.93	2204.069	2204.069	11246.98	11049.00	10
	2059.2	1161	859	226.597	1010	8979.91	8779.93	1770.003	1770.003	11910.91	11408.93	10
	3420.1	-334	-211	265	1010	8979.91	8779.93	-2204.069	-2204.069	6441.84	6364.86	10
	3490.2	97	98	265	1010	8979.91	8779.93	2204.069	2204.069	11280.98	11082.00	10
	5400.1	-126	-80	392	1010	8979.91	8779.93	-3639.539	-3639.539	5214.37	5060.39	10
	5470.6	31	32	392	1010	8979.91	8779.93	3639.539	3639.539	12650.45	12451.47	10
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
10	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
	77.1	2308	3188	285.461	1010	8979.91	8779.93	2435.338	2435.338	13723.25	14403.27	70
	169.4	-12	-13	265	1010	8979.91	8779.93	-2204.069	-2204.069	6763.84	6562.86	70
	1890	74	72	265	1010	8979.91	8779.93	2204.069	2204.069	11257.98	11056.00	70
	1968.2	-1069	-1511	322.362	1010	8979.91	8779.93	-2852.427	-2852.427	5058.48	4416.50	70
	2147.2	91	90	392	1010	8979.91	8779.93	3639.539	3639.539	12710.45	12509.47	70
	2570	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
11	0	-29	-27	392	1010	8979.91	8779.93	-3639.539	-3639.539	5311.37	5113.39	10
	2.9	-20317	-13859	565	1147	10197.98	9970.871	-5594.944	-5594.944	-15713.97	-9483.07	10
	6.8	42852	29563	565	1172	10420.25	10188.2	5594.944	5594.944	58867.20	45346.14	10
	1567.4	-15216	-10526	565	1135	10091.29	9866.555	-5594.944	-5594.944	-10719.66	-6254.39	10
	2168.4	60377	41773	50	1134	10082.39	9857.862	-226.0583	-226.0583	70233.34	51404.80	10
	5409.4	-14924	-10329	565	1054	9371.114	9162.422	-5594.944	-5594.944	-11147.83	-6761.52	10
	6730.4	60377	41773	50	1133	10073.5	9849.169	-226.0583	-226.0583	70224.44	51396.11	10
	7243.2	-1965	-1434	128.917	675	6001.425	5867.775	-665.9339	-665.9339	3370.49	3767.84	10
	18215.4	52636	36417	100	1010	8979.91	8779.93	339.0875	339.0875	61955.00	45536.02	10
	20015.5	-24511	-16189	260.183	1010	8979.91	8779.93	-2149.623	-2149.623	-17680.71	-9558.69	10
	22314.5	22	23	392	937	8330.867	8145.341	3639.539	3639.539	11992.41	11807.88	10
12	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	60
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	60
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	60
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	60
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
13	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1
	10	23	22	392	1375	12225.13	11952.88	3639.539	3639.539	15887.66	15614.41	1
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	1
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	1
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	1
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1

Table 5: Safe End Stress Summary (continue)

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
14	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	1
	60	4383	3174	275	885	7868.535	7693.305	2317.098	2317.098	14568.63	13184.40	1
	148	420	300	258.492	803	7139.473	6980.479	2130.509	2130.509	9689.98	9410.99	1
	960	544	424	100	50	444.55	434.65	339.0875	339.0875	1327.64	1197.74	1
	1460	137	139	100	50	444.55	434.65	339.0875	339.0875	920.64	912.74	1
15	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	228
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	228
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	228
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	228
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
19	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	300
	1800	219	177	265	1010	8979.91	8779.93	2204.069	2204.069	11402.98	11161.00	300
	2300	72	74	265	1010	8979.91	8779.93	2204.069	2204.069	11255.98	11058.00	300
20	0	-109	-105	265	1010	8979.91	8779.93	-2204.069	-2204.069	6666.84	6470.86	300
	4	-17288	-12189	440.106	1010	8979.91	8779.93	-4183.277	-4183.277	-12491.37	-7592.35	300
	4425	-2	-1	549	1010	8979.91	8779.93	-5414.097	-5414.097	3563.81	3364.83	300
20A	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	4	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	241	-7461	-5525	290.247	1010	8979.91	8779.93	-2489.433	-2489.433	-970.52	765.50	300
	572	128	132	549	1010	8979.91	8779.93	5414.097	5414.097	14522.01	14326.03	300
	951	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
21-23	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	138	62	45	545.167	989	8793.199	8597.377	5370.773	5370.773	14225.97	14013.15	300
	6264	-5	-20	374.97	50	444.55	434.65	-3447.05	-3447.05	-3007.50	-3032.40	300
	6390	104	59	366.172	50	444.55	434.65	3347.607	3347.607	3896.16	3841.26	300
	15644	-173	-167	100	50	444.55	434.65	-339.0875	-339.0875	-67.54	-71.44	300
24	0	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
	600	0	0	100	1563	13896.63	13587.16	339.0875	339.0875	14235.72	13926.25	1
	2400	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
25	0	0	0	100	0	0	0	339.0875	339.0875	339.09	339.09	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 2.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 8891)/1000 [Table 3, 1].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 8693)/1000 [Table3, 1].
Column 9: Total external stress from calculation in Table 3, 5707.97 psi*(Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 6: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 2.0
n = .2
Sm = 26700. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.670E+07 psi
Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
74568.	0.	74568.	62689.	1.000	41892.	1.000E+01	7.488E+03	.0013
70231.	0.	70231.	58499.	1.000	39456.	1.000E+01	8.944E+03	.0011
69395.	0.	69395.	59106.	1.000	38986.	1.000E+01	9.268E+03	.0011
67667.	0.	67667.	59377.	1.000	38015.	9.300E+01	9.988E+03	.0093
67667.	0.	67667.	59377.	1.000	38015.	1.200E+02	9.988E+03	.0120
67667.	0.	67667.	59377.	1.000	38015.	8.700E+01	9.988E+03	.0087
67282.	0.	67282.	60118.	1.000	37799.	1.000E+01	1.018E+04	.0010
67142.	0.	67142.	60462.	1.000	37720.	1.000E+01	1.025E+04	.0010
66791.	0.	66791.	62353.	1.000	37523.	1.000E+00	1.044E+04	.0001
66791.	0.	66791.	62353.	1.000	37523.	1.500E+01	1.044E+04	.0014
66791.	16.	66775.	62337.	1.000	37514.	1.230E+02	1.045E+04	.0118
66791.	1902.	64889.	60505.	1.000	36454.	9.000E+01	1.152E+04	.0078
66298.	1902.	64396.	47410.	1.000	36177.	3.000E+01	1.182E+04	.0025
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	30389.	35909.	21760.	1.000	20173.	1.000E+00	9.581E+04	.0000
66298.	31068.	35230.	23734.	1.000	19792.	2.670E+02	1.038E+05	.0026
64150.	31068.	33081.	34263.	1.000	18585.	3.300E+01	1.303E+05	.0003
64150.	31070.	33079.	34283.	1.000	18584.	2.700E+01	1.303E+05	.0002
59772.	31070.	28702.	31815.	1.000	16125.	1.000E+00	2.222E+05	.0000
58992.	31070.	27922.	31800.	1.000	15687.	1.000E+00	2.519E+05	.0000
55364.	31070.	24293.	25433.	1.000	13648.	2.710E+02	4.757E+05	.0006
55364.	31682.	23681.	17402.	1.000	13304.	1.000E+01	5.703E+05	.0000
55364.	32964.	22400.	17300.	1.000	12584.	1.000E+01	9.414E+05	.0000
55364.	34282.	21082.	17307.	1.000	11844.	9.000E+00	1.912E+06	.0000
55042.	34282.	20761.	18195.	1.000	11663.	7.000E+01	2.231E+06	.0000
54965.	34282.	20683.	18463.	1.000	11620.	2.210E+02	2.310E+06	.0001
54965.	34317.	20648.	18464.	1.000	11600.	1.000E+01	2.348E+06	.0000
54965.	34327.	20638.	18463.	1.000	11595.	6.900E+01	2.358E+06	.0000
53963.	34327.	19637.	17393.	1.000	11032.	2.310E+02	3.757E+06	.0001
53963.	34328.	19636.	17401.	1.000	11031.	3.000E+02	3.758E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	41522.	12441.	10688.	1.000	6989.	1.200E+02	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	6.000E+01	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	1.000E+00	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	8.800E+01	1.000E+20	.0000
51835.	43358.	8477.	7712.	1.000	4762.	1.400E+02	1.000E+20	.0000
51835.	46000.	5835.	5149.	1.000	3278.	3.000E+02	1.000E+20	.0000
51835.	46000.	5835.	5146.	1.000	3278.	9.560E+03	1.000E+20	.0000
51782.	46000.	5783.	6568.	1.000	3249.	1.000E+01	1.000E+20	.0000

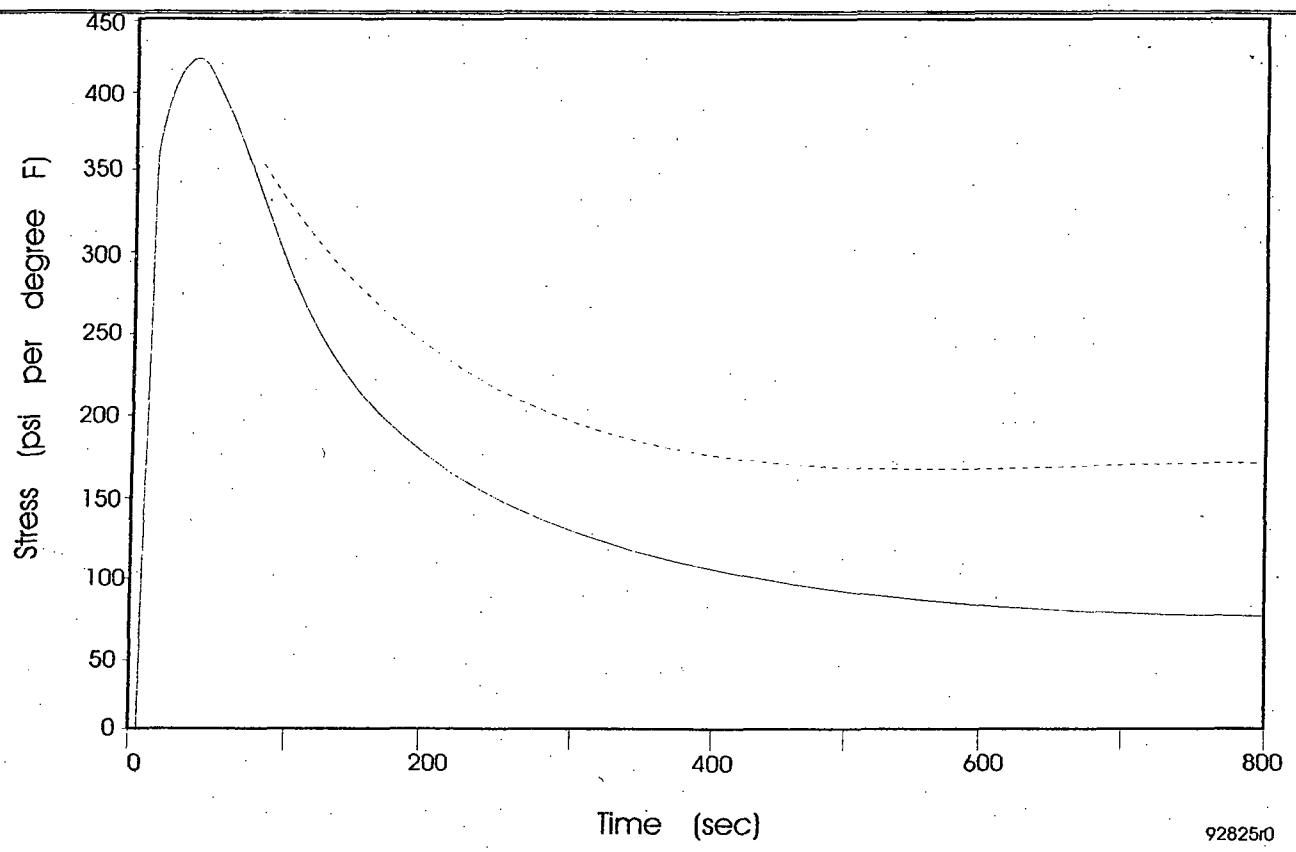
50914.	46000.	4915.	6112.	1.000	2761.	1.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	6.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.320E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+04	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
								=======
						TOTAL USAGE FACTOR =		.0636

Table 7: Fatigue Results for Safe End (60 Years)

LOCATION = LOCATION NO. 1 -- SAFE END
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 3.0
n = .2
Sm = 17800. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.810E+07 psi
Kt = 1.34

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
70233.	-17681.	87914.	60963.	1.283	74422.	1.000E+01	1.338E+03	.0075
70224.	-15714.	85938.	60879.	1.280	72869.	1.000E+01	1.415E+03	.0071
61955.	-12491.	74446.	53128.	1.000	49383.	1.000E+01	4.568E+03	.0022
58867.	-12491.	71359.	52938.	1.000	47700.	1.000E+01	5.094E+03	.0020
53379.	-12491.	65870.	47699.	1.000	43819.	2.800E+02	6.552E+03	.0427
53379.	-11148.	64527.	46869.	1.000	42951.	1.000E+01	6.953E+03	.0014
53379.	-10720.	64099.	46361.	1.000	42631.	1.000E+01	7.109E+03	.0014
53379.	-9972.	63351.	58588.	1.194	53087.	6.000E+01	3.628E+03	.0165
53379.	-9972.	63351.	58588.	1.194	53087.	1.000E+00	3.628E+03	.0003
53379.	-9972.	63351.	58588.	1.194	53087.	2.280E+02	3.628E+03	.0628
53379.	-9060.	62439.	44702.	1.000	41444.	1.100E+01	7.731E+03	.0014
15888.	-9060.	24948.	20209.	1.000	16985.	1.000E+00	1.802E+05	.0000
14569.	-9060.	23629.	17779.	1.000	15840.	1.000E+00	2.410E+05	.0000
14522.	-9060.	23582.	18921.	1.000	16022.	2.870E+02	2.287E+05	.0013
14522.	-3008.	17530.	17358.	1.000	12508.	1.300E+01	9.944E+05	.0000
14396.	-3008.	17404.	17229.	1.000	12417.	2.870E+02	1.083E+06	.0003
14396.	-971.	15367.	13432.	1.000	10641.	1.300E+01	5.165E+06	.0000
14236.	-971.	15206.	13161.	1.000	10506.	1.000E+00	5.563E+06	.0000
14226.	-971.	15196.	13248.	1.000	10516.	2.860E+02	5.531E+06	.0001
14226.	-178.	14404.	14178.	1.000	10262.	1.400E+01	6.379E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	6.000E+01	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	1.000E+00	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	2.250E+02	6.547E+06	.0000
13849.	-68.	13916.	14943.	1.000	10141.	3.000E+00	6.837E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	6.000E+01	8.117E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	2.280E+02	8.117E+06	.0000
13723.	-68.	13791.	14475.	1.000	9989.	9.000E+00	7.465E+06	.0000
13723.	-65.	13788.	14473.	1.000	9987.	6.100E+01	7.474E+06	.0000
12722.	-65.	12786.	12548.	1.000	9103.	1.000E+01	1.729E+07	.0000
12710.	-65.	12775.	12579.	1.000	9102.	7.000E+01	1.730E+07	.0000
12652.	-65.	12717.	12524.	1.000	9061.	1.590E+02	1.833E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.200E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	339.	12313.	12115.	1.000	8772.	1.230E+02	2.905E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.200E+02	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	921.	11732.	11542.	1.000	8357.	1.000E+00	5.462E+07	.0000
12652.	1328.	11325.	11257.	1.000	8088.	1.000E+00	7.100E+07	.0000
12652.	3370.	9282.	8687.	1.000	6531.	1.000E+01	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000

12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3564.	9089.	9090.	1.000	6501.	3.000E+02	1.000E+20	.0000
12652.	3565.	9088.	-1740.	1.000	4535.	3.000E+02	1.000E+20	.0000
12652.	3896.	8756.	8613.	1.000	6237.	3.000E+02	1.000E+20	.0000
12652.	5058.	7594.	8038.	1.000	5513.	7.000E+01	1.000E+20	.0000
12652.	5171.	7481.	7424.	1.000	5341.	7.048E+03	1.000E+20	.0000
12650.	5171.	7479.	7421.	1.000	5339.	1.000E+01	1.000E+20	.0000
12648.	5171.	7477.	7420.	1.000	5338.	2.000E+03	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12641.	5171.	7470.	7412.	1.000	5333.	2.240E+02	1.000E+20	.0000
12641.	5214.	7427.	7382.	1.000	5304.	1.000E+01	1.000E+20	.0000
12641.	5232.	7409.	7370.	1.000	5293.	2.000E+03	1.000E+20	.0000
12641.	5311.	7330.	7329.	1.000	5243.	1.000E+01	1.000E+20	.0000
12641.	6442.	6200.	6078.	1.000	4412.	1.000E+01	1.000E+20	.0000
12641.	6667.	5975.	5972.	1.000	4273.	3.000E+02	1.000E+20	.0000
12641.	6764.	5878.	5880.	1.000	4205.	7.000E+01	1.000E+20	.0000
12641.	9690.	2951.	3031.	1.000	2126.	1.000E+00	1.000E+20	.0000
12641.	10119.	2522.	2541.	1.000	1808.	1.200E+02	1.000E+20	.0000
12641.	11247.	1394.	1393.	1.000	997.	1.000E+01	1.000E+20	.0000
12641.	11256.	1385.	1384.	1.000	991.	3.000E+02	1.000E+20	.0000
12641.	11258.	1383.	1386.	1.000	990.	7.000E+01	1.000E+20	.0000
12641.	11281.	1360.	1360.	1.000	973.	1.000E+01	1.000E+20	.0000
12641.	11403.	1238.	1281.	1.000	894.	3.000E+02	1.000E+20	.0000
12641.	11550.	1092.	1130.	1.000	788.	2.000E+03	1.000E+20	.0000
12641.	11911.	731.	1034.	1.000	578.	1.000E+01	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	4.555E+03	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	5.445E+03	1.000E+20	.0000
12641.	11992.	649.	635.	1.000	462.	1.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	6.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	1.000E+00	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	2.280E+02	1.000E+20	.0000
12641.	12640.	1.	0.	1.000	1.	3.000E+02	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	3.956E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
								=======
							TOTAL USAGE FACTOR =	.1471



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1: Typical Green's Functions for Thermal Transient Stress

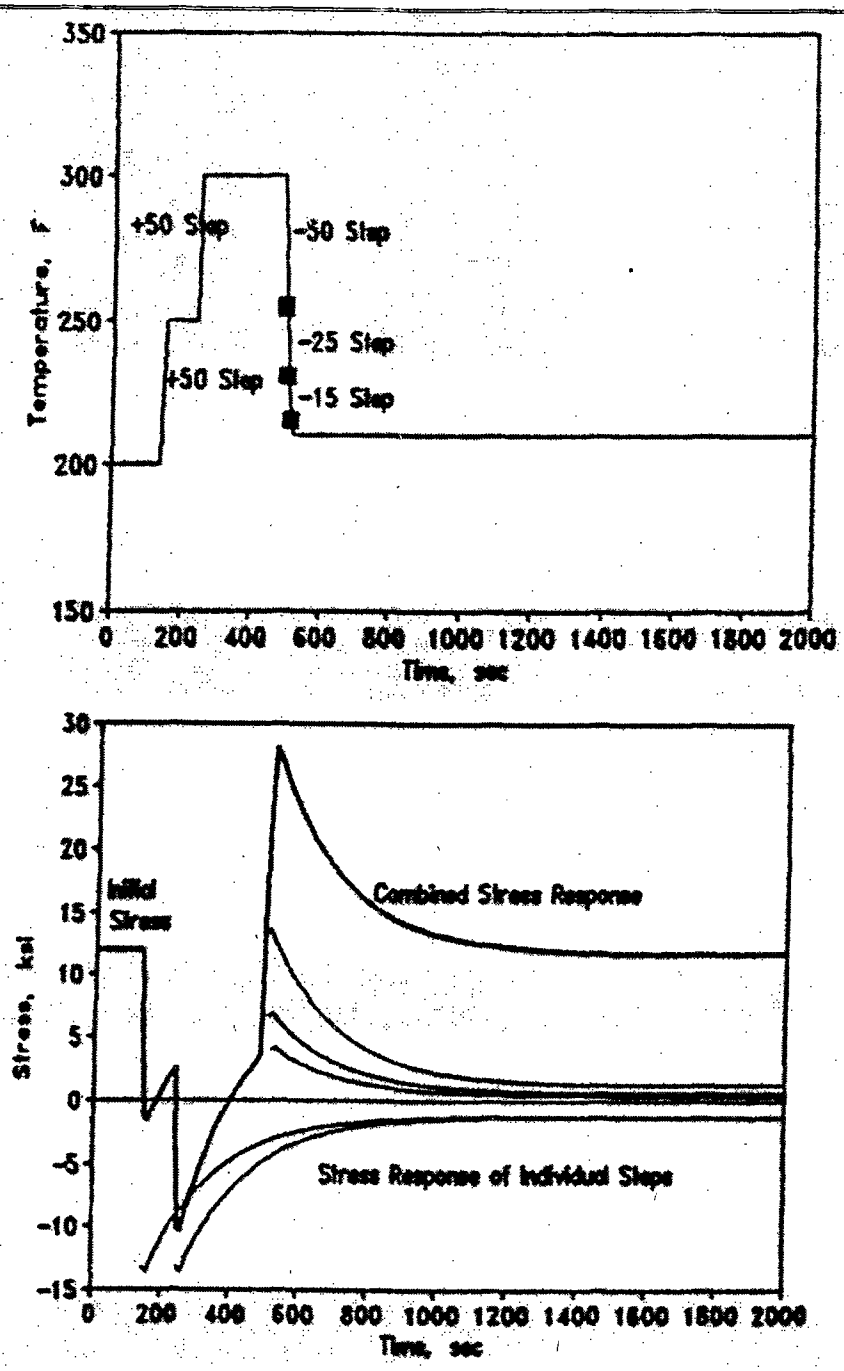


Figure 2: Typical Stress Response Using Green's Functions

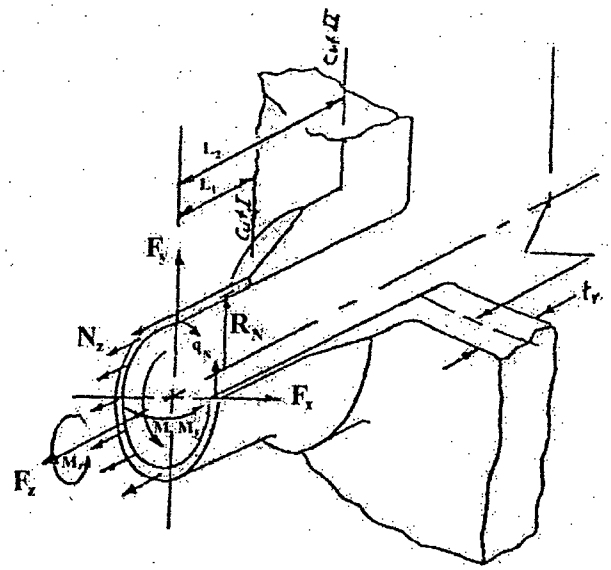


Figure 3: External Forces and Moments on the Feedwater Nozzle

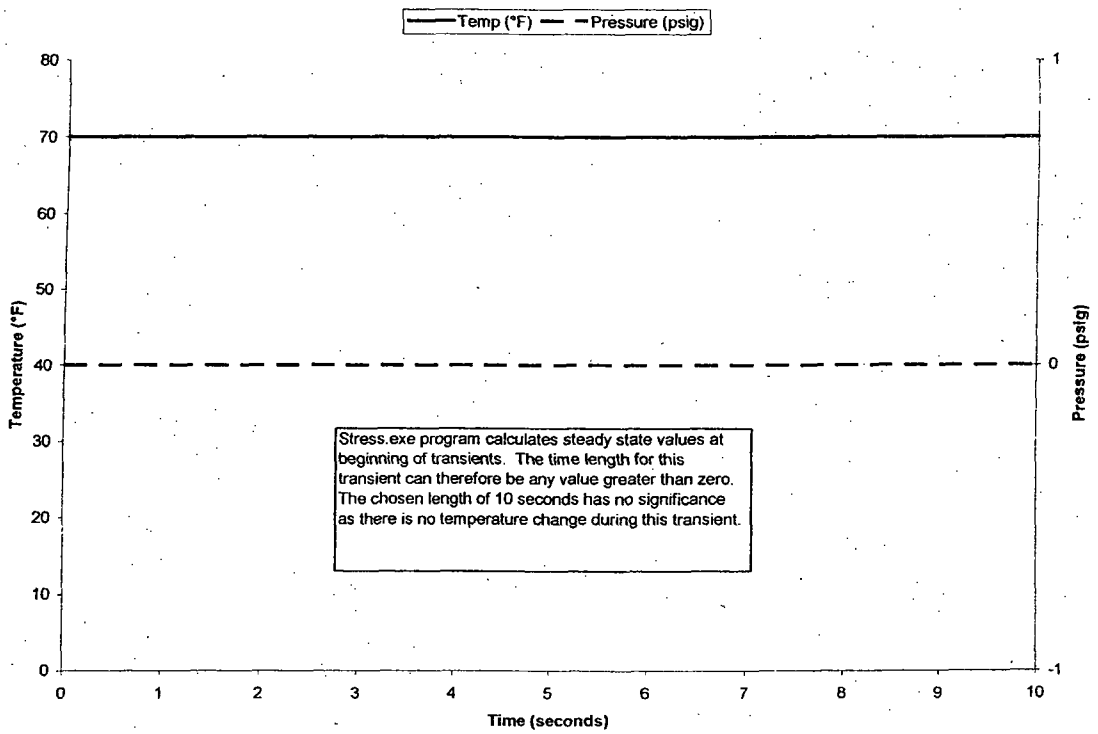


Figure 4: Transient 1, Bolt-up

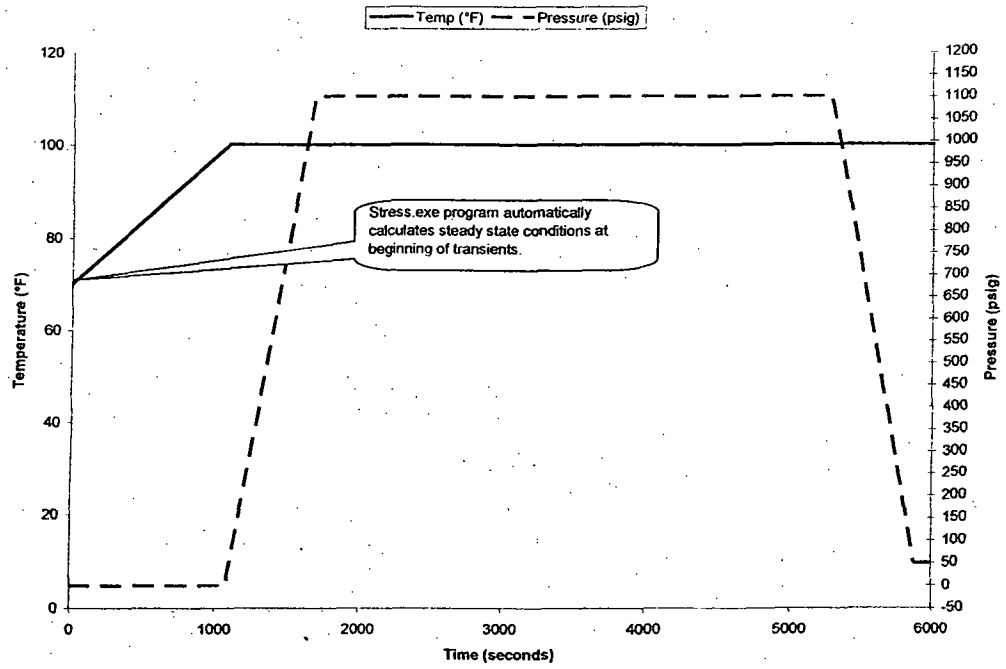


Figure 5: Transient 2, Design HYD Test

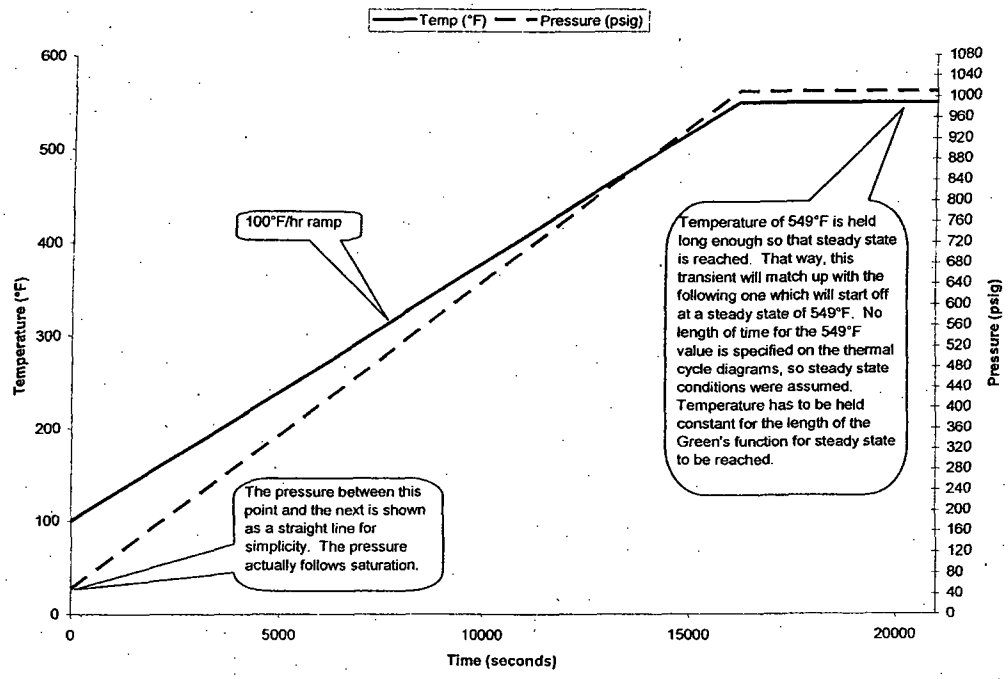


Figure 6: Transient 3, Startup

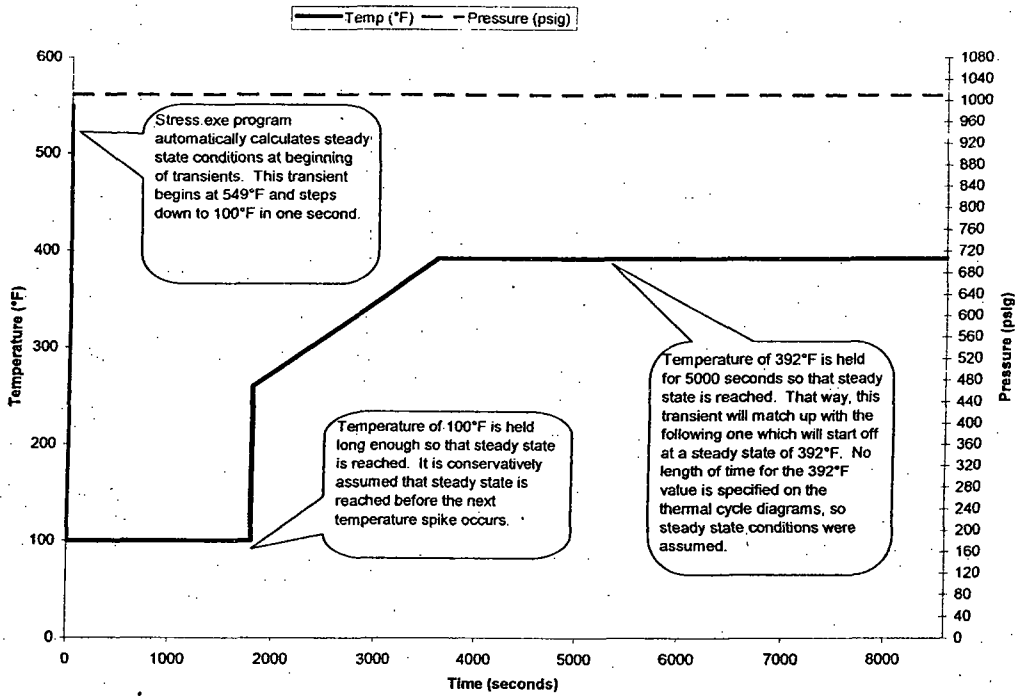


Figure 7: Transient 4, Turbine Roll and Increased to Rated Power

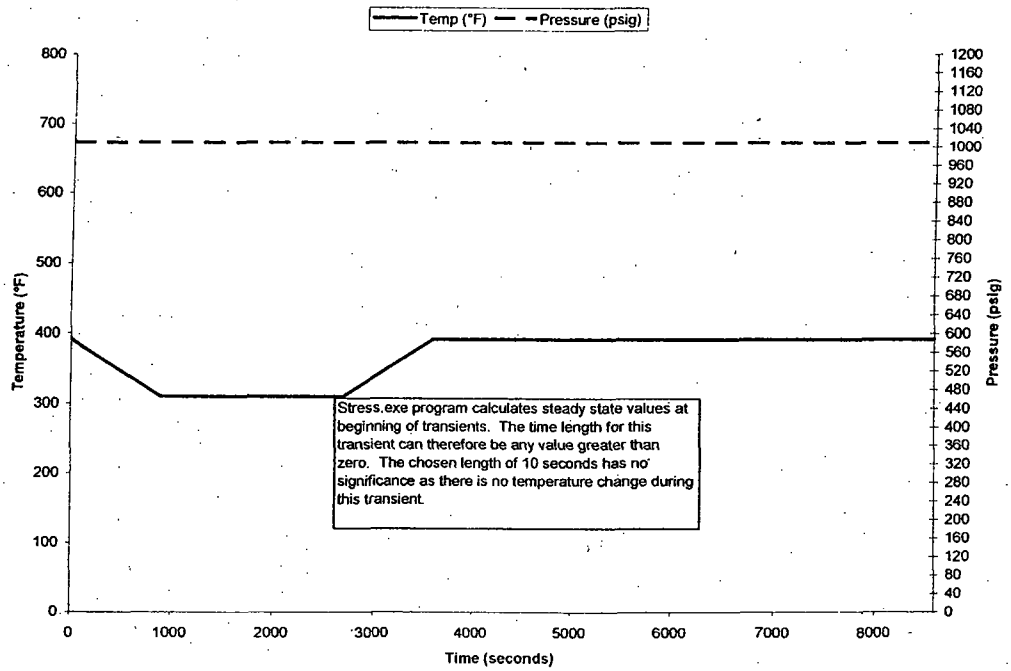


Figure 8: Transient 5, Daily Reduction 75% Power

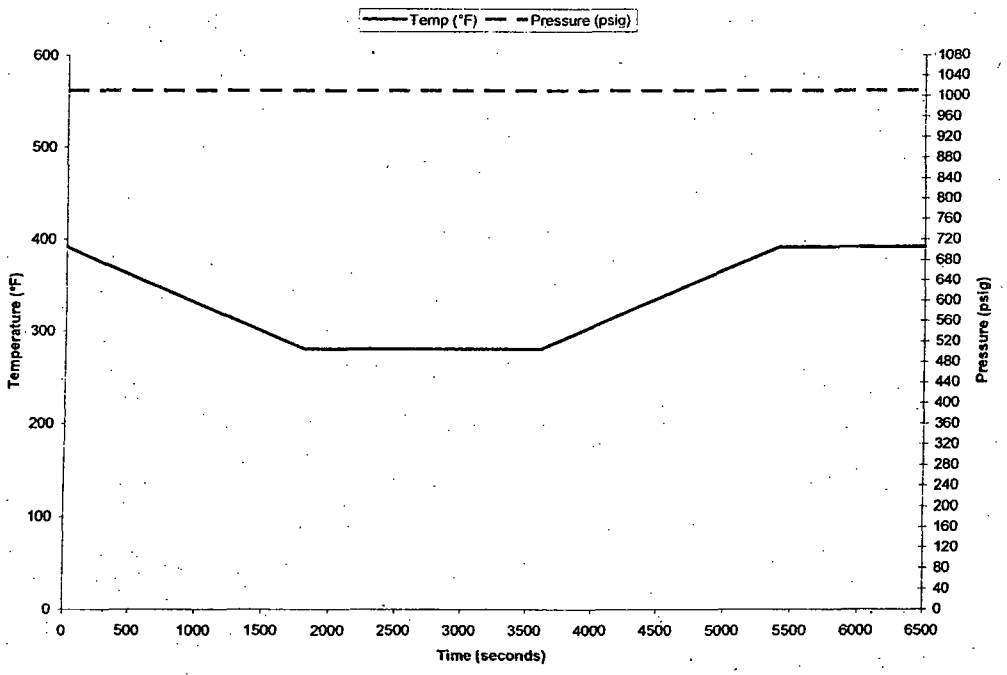


Figure 9: Transient 6, Weekly Reduction 50% Power

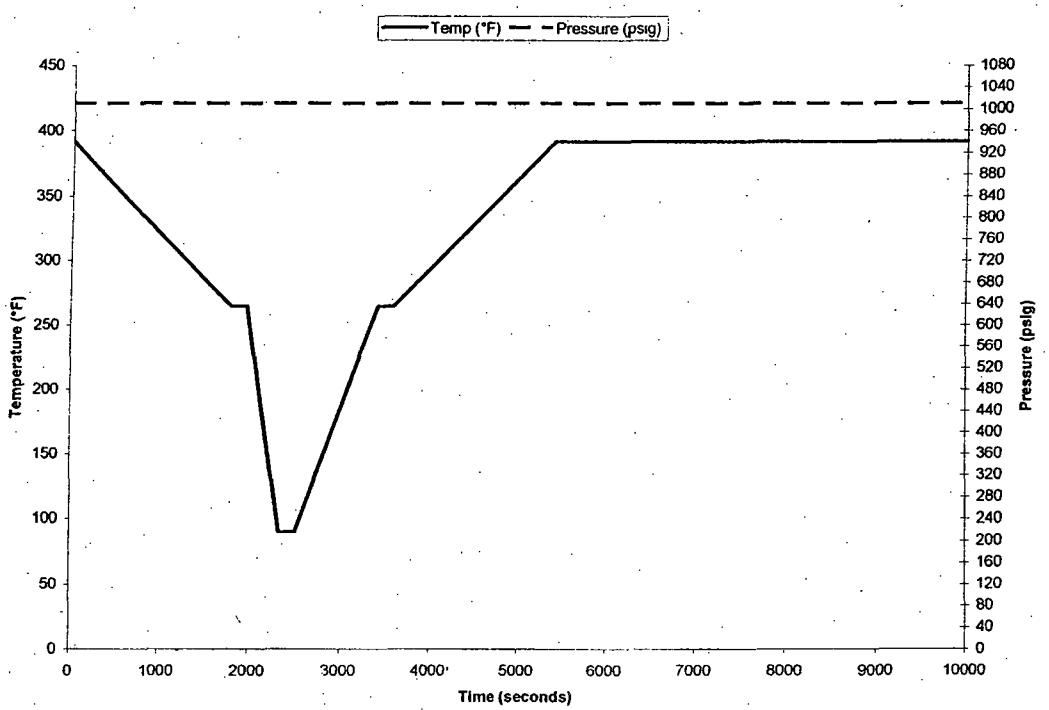


Figure 10: Transient 9, Turbine Trip at 25% Power

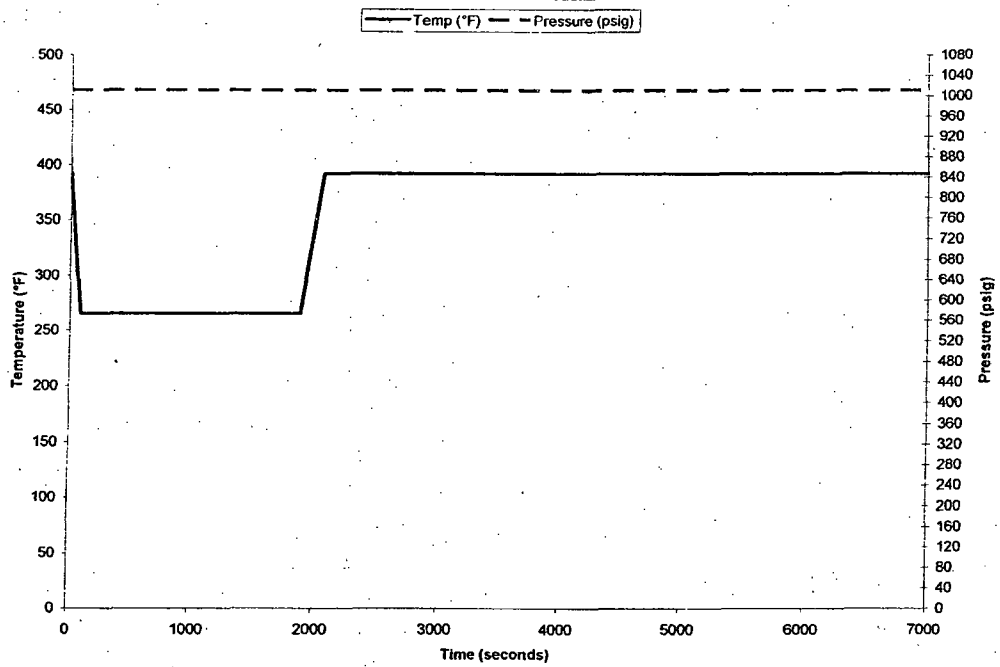


Figure 11: Transient 10, Feedwater Bypass

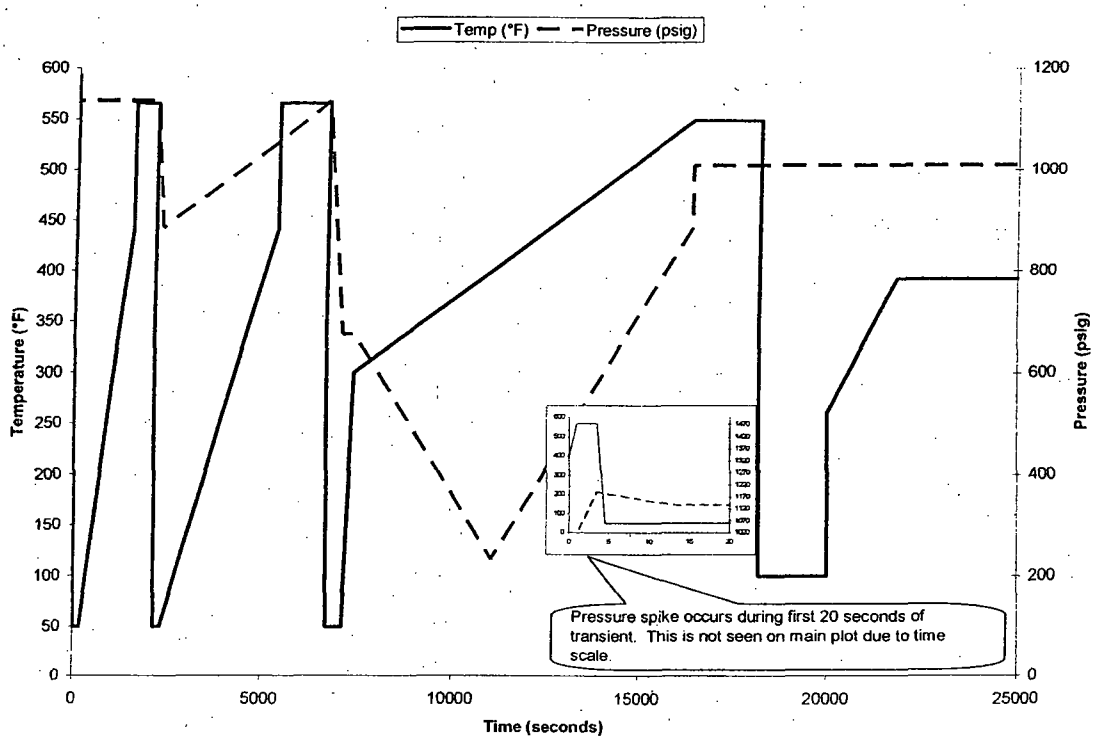


Figure 12: Transient 11, Loss of Feedwater Pumps

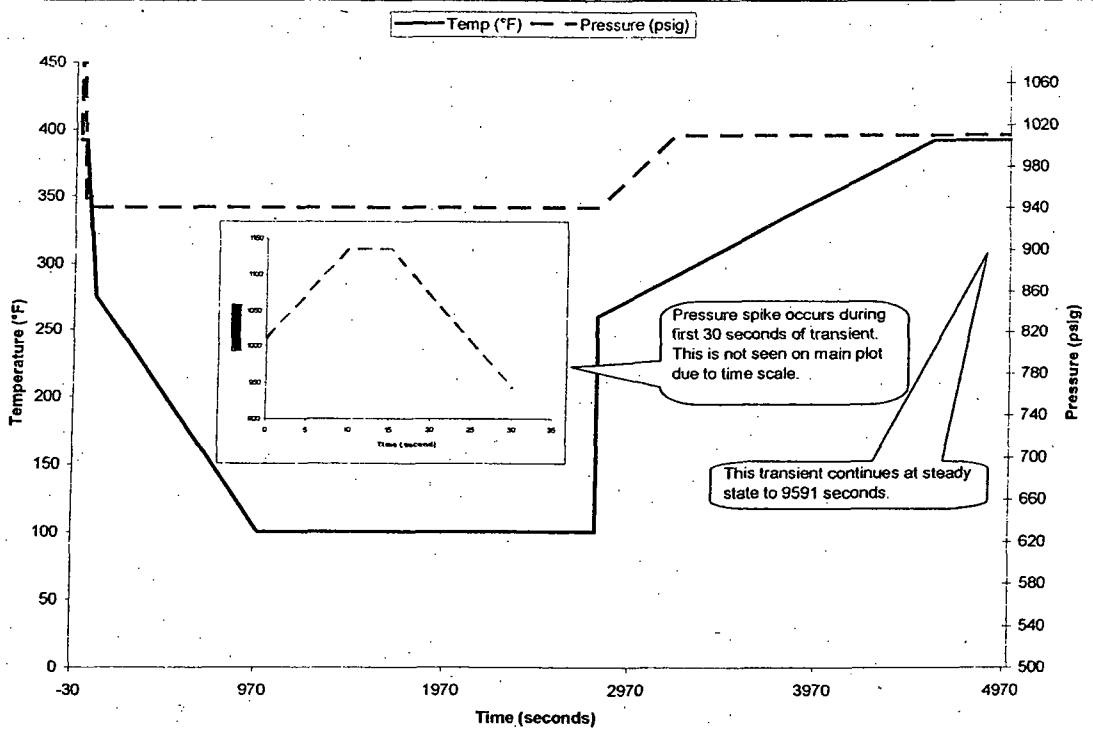


Figure 13: Transient 12, Turbine Generator Trip

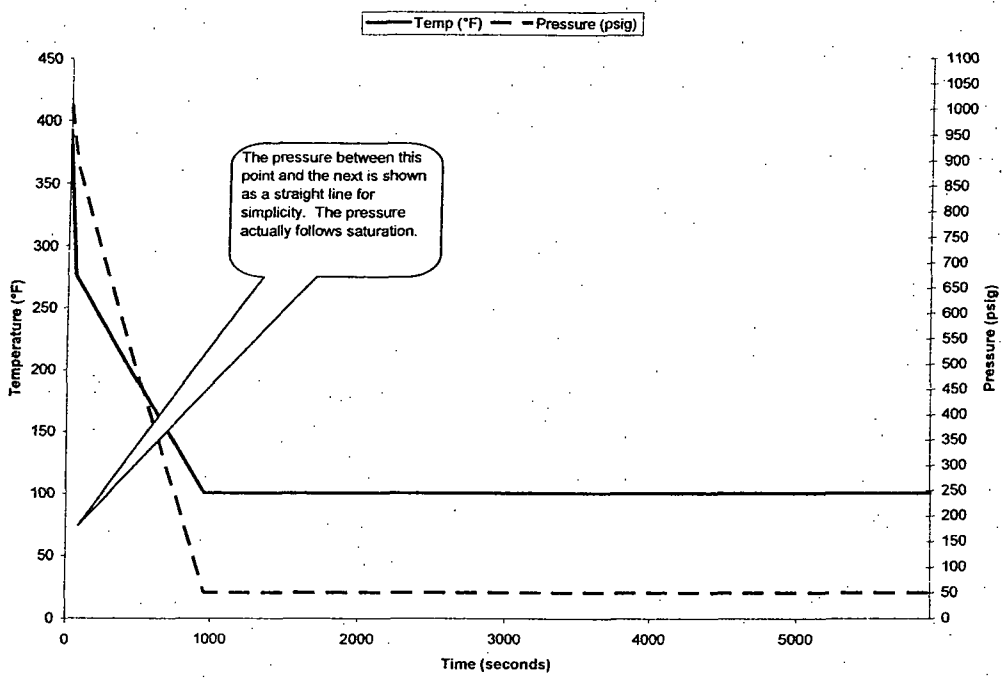


Figure 14: Transient 14, SRV Blowdown

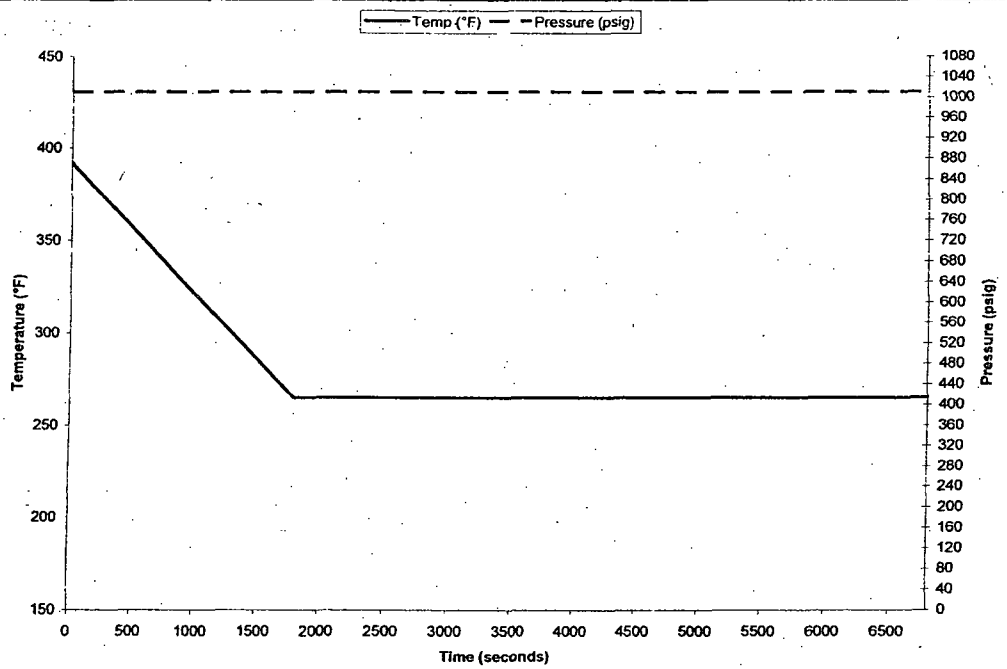


Figure 15: Transient 19, Reduction to 0% Power

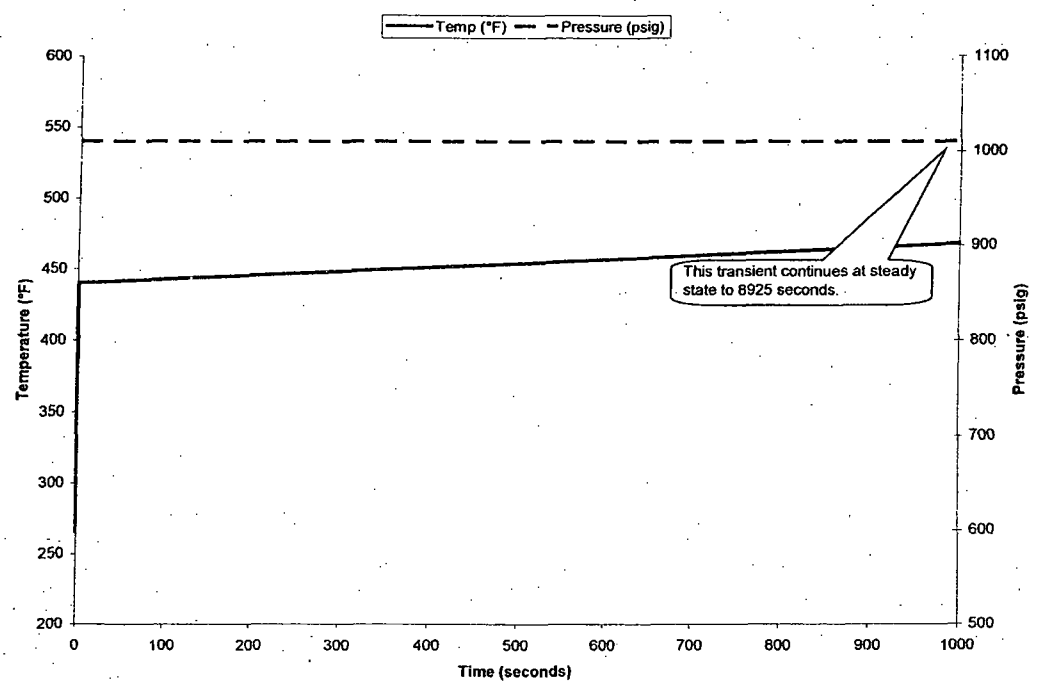


Figure 16: Transient 20, Hot Standby (Heatup Portion)

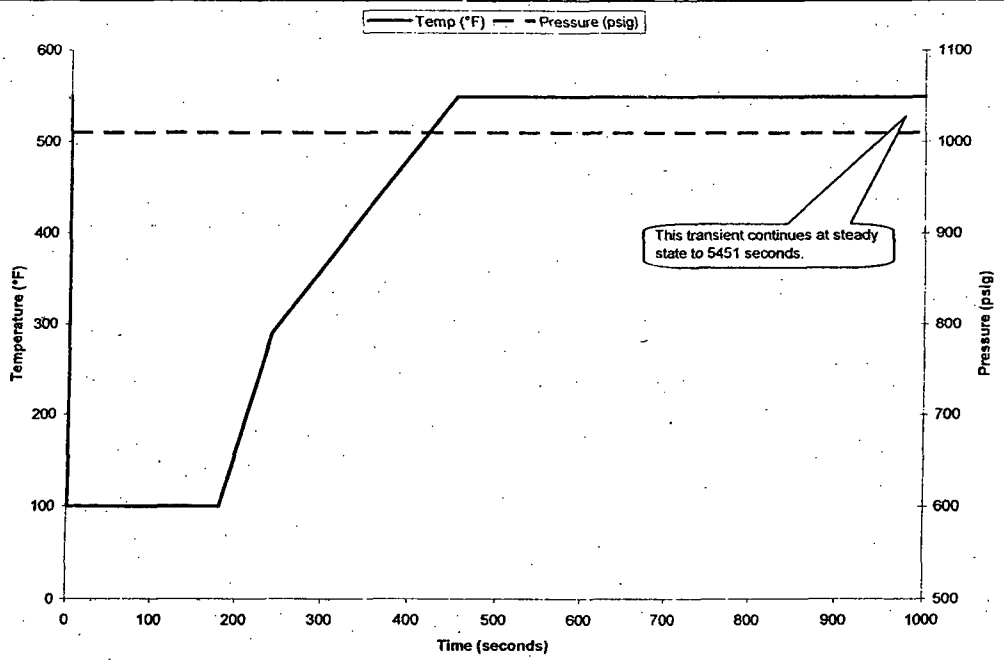


Figure 17: Transient 20A, Hot Standby (Feedwater Injection Portion)

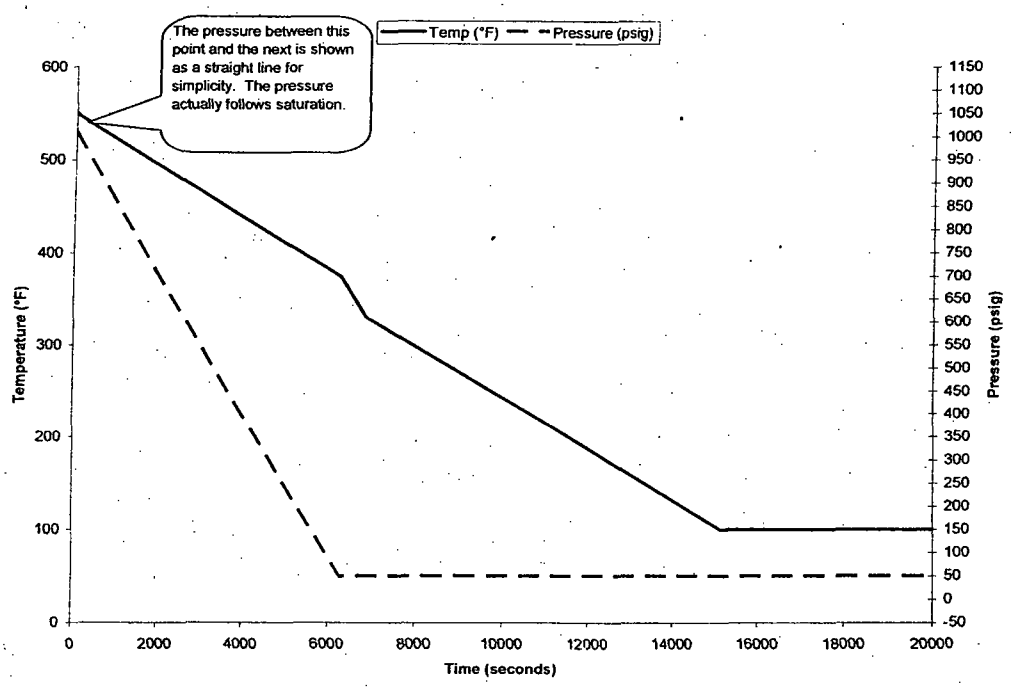


Figure 18: Transient 21-23, Shutdown

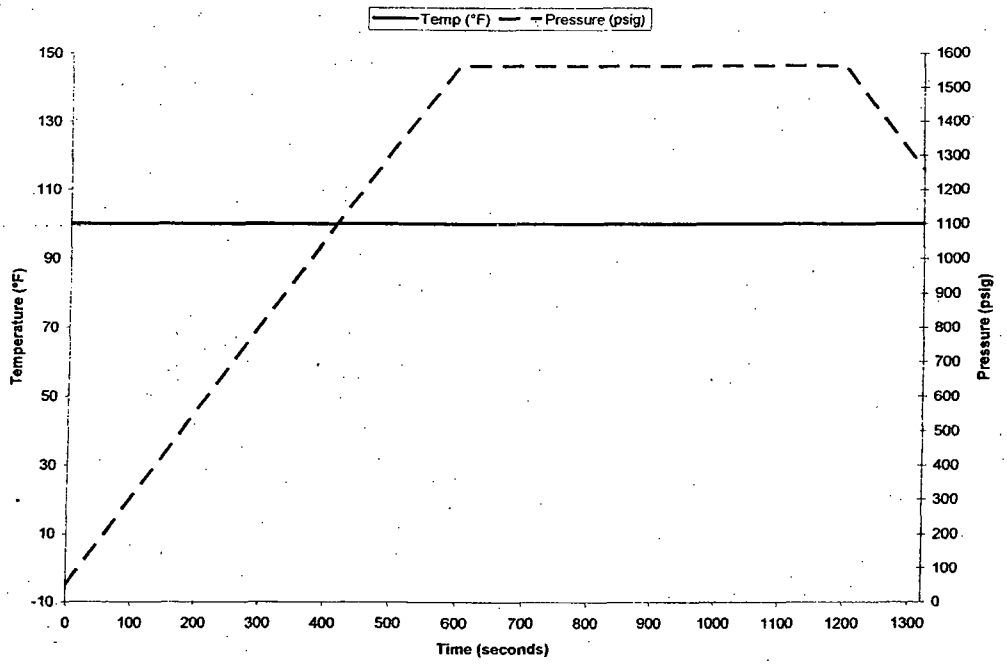


Figure 19: Transient 24, Hydrostatic Test

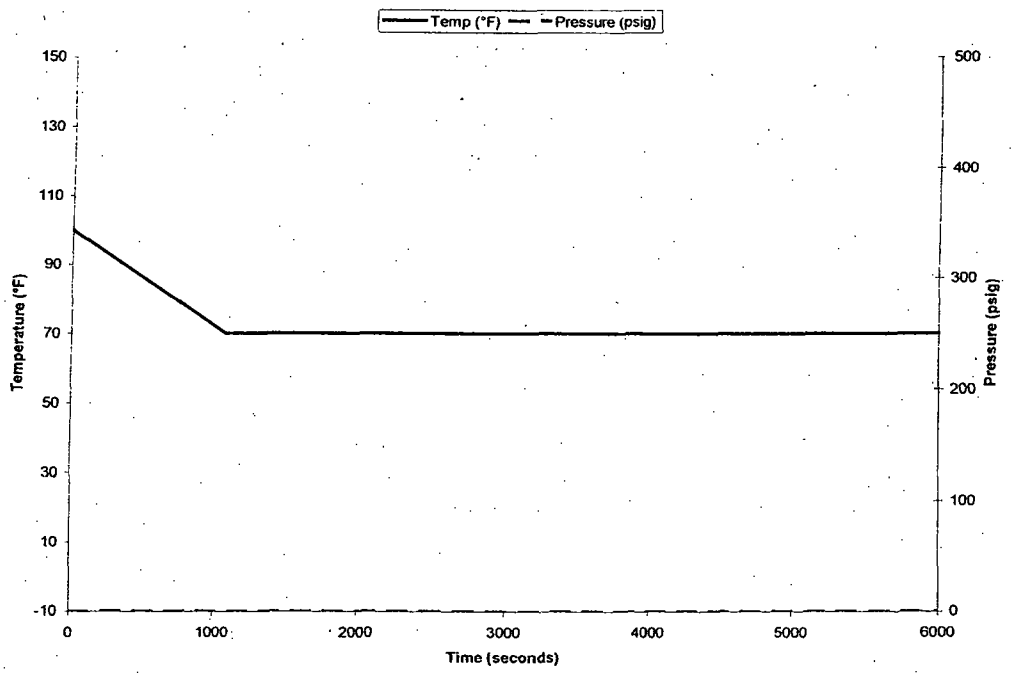


Figure 20: Transient 25, Unbolt

APPENDIX A

SUMMARY OF OUTPUT FILES

Transient Table.xls	Definition of Transients	In Computer files
BRresults.xls	Blend Radius Stress Summary	In Computer files
SEresults.xls	Safe End Stress Summary	In Computer files
TRANSNT_XX.INP	Input File for Each Transient	In Computer files
Green.dat	Input File for Green Functions	In Computer files
P-V_XX.OUT	Output File for Stress Analysis	In Computer files
GREEN.CFG	Input File for Defining Green Function	In Computer files
FATIGUE.CFG	Input File for Defining Fatigue Analysis	In Computer files
FATIGUE.DAT	Input File for Fatigue Curves	In Computer files
FATIGUE.inp	Input file for Fatigue Analysis from BRresults.xls or SEresults.xls	In Computer files
FATIGUE.OUT	Fatigue Output File	In Computer files

Where XX is defined for each transient.

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EXHIBIT C

Structural Integrity Associates, Inc.	File No.: VY-16Q-303
CALCULATION PACKAGE	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee

CALCULATION TITLE:

Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1 - 24, Appendices: A1 – A2, B1 – B2 In computer files	Initial issue.	Terry J. Herrmann 07/05/07	Gary L. Stevens 07/05/07 Terry J. Herrmann 07/05/07

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1.0 INTRODUCTION/STATEMENT OF PROBLEM/ OBJECTIVE

The purpose of this calculation is to perform a plant-specific evaluation of reactor water environmental effects for the reactor recirculation (RR) inlet nozzle and the reactor pressure vessel (RPV) shell/bottom head locations identified within NUREG/CR-6260 [1] for the older vintage General Electric (GE) plant for the Vermont Yankee Nuclear Power Plant (VY).

The water chemistry input used in this calculation covers several portions of the RPV, as well as the feedwater and recirculation lines. Although these regions encompass more areas than needed to address the two components of interest in this calculation, environmental fatigue multipliers are developed for all of these regions in this calculation for potential use in other evaluations associated with this project.

2.0 TECHNICAL APPROACH OR METHODOLOGY

Per Chapter X, "Time-Limited Aging Analyses Evaluation of Aging Management Programs Under 10 CFR 54.21(c)(1)(iii)," Section X.M1, "Metal Fatigue of Reactor Coolant Pressure Boundary," of the Generic Aging Lessons Learned (GALL) Report [2], detailed, vintage-specific, fatigue calculations are required for plants applying for license renewal for the locations identified for the appropriate vintage plant in NUREG/CR-6260.

In this calculation, detailed environmentally assisted fatigue (EAF) calculations are performed for VY for two of the locations associated with the older vintage GE plant in NUREG/CR-6260. The older-vintage GE plant is the appropriate comparison to VY since the original piping design at VY was in accordance with USAS B31.1 [3], as well as the fact that the older-vintage boiling water reactor (BWR) in NUREG/CR-6260 was a BWR-4 plant, which is the same as VY.

Entergy performed an initial assessment of EAF effects for VY in their License Renewal Application (LRA) that was submitted to the NRC in January 2006. Table 4.3-3 of the VY LRA provides the results of those evaluations. All but two of the VY locations evaluated for EAF in the LRA did not yield acceptable results for 60 years of operation. Further refined analyses are currently underway in other calculations associated with this project to address those components. This calculation documents the EAF evaluation for the RR inlet nozzle and RPV shell/bottom head locations, where it is expected that acceptable EAF results can be achieved based on the existing analyses without the need for additional refined evaluations.

3.0 ASSUMPTIONS / DESIGN INPUTS

Per Section X.M1 of the GALL Report [2], the EAF evaluation must use the appropriate F_{en} relationships from NUREG/CR-6583 [4] (for carbon/low alloy steels) and NUREG/CR-5704 [5] (for stainless steels), as appropriate for the material for each location. These expressions are:

For Carbon Steel [4, p. 69]: $F_{en} = \exp(0.585 - 0.00124T' - 0.101S^*T^*O^*\dot{\varepsilon}^*)$

Substituting T' = 25°C in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [6], the following is obtained:

$$F_{en} = \exp(0.585 - 0.00124(25°C) - 0.101\ S^*\ T^*\ O^*\ \dot{\varepsilon}^*)$$
$$= \exp(0.554 - 0.101\ S^*\ T^*\ O^*\ \dot{\varepsilon}^*)$$

For Low Alloy Steel [4, p. 69]: $F_{en} = \exp(0.929 - 0.00124T' - 0.101S^*T^*O^*\dot{\varepsilon}^*)$

Substituting T' = 25°C in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [6], the following is obtained:

$$F_{en} = \exp(0.929 - 0.00124(25°C) - 0.101\ S^*\ T^*\ O^*\dot{\varepsilon}^*)$$
$$= \exp(0.898 - 0.101\ S^*\ T^*\ O^*\dot{\varepsilon}^*)$$

where [4, pp. 60 and 65]:

F_{en}	=	fatigue life correction factor
S^*	=	S for 0 < sulfur content, $S \leq 0.015$ wt. %
	=	0.015 for $S > 0.015$ wt. %
T^*	=	0 for $T < 150°C$
	=	$(T - 150)$ for $150 \leq T \leq 350°C$
T	=	fluid service temperature (°C)
O^*	=	0 for dissolved oxygen, DO < 0.05 parts per million (ppm)
	=	$\ln(DO/0.04)$ for $0.05\ ppm \leq DO \leq 0.5\ ppm$
	=	$\ln(12.5)$ for $DO > 0.5$ ppm
$\dot{\varepsilon}^*$	=	0 for strain rate, $\dot{\varepsilon} > 1\%/sec$
	=	$\ln(\dot{\varepsilon}^*)$ for $0.001 \leq \dot{\varepsilon} \leq 1\%/sec$
	=	$\ln(0.001)$ for $\dot{\varepsilon} < 0.001\%/sec$

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For Types 304 and 316 Stainless Steel [5, p. 31]: $F_{en} = \exp(0.935 - T^* \dot{\varepsilon}^* O^*)$

where [5, pp. 25 and 31]:

F_{en}	=	fatigue life correction factor
T^*	=	0 for $T < 200°C$
	=	1 for $T \geq 200°C$
T	=	fluid service temperature (°C)
$\dot{\varepsilon}^*$	=	0 for strain rate, $\dot{\varepsilon} > 0.4\%/sec$
	=	$\ln(\dot{\varepsilon}/0.4)$ for $0.0004 \leq \dot{\varepsilon} \leq 0.4\%/sec$
	=	$\ln(0.0004/0.4)$ for $\dot{\varepsilon} < 0.0004\%/sec$
O^*	=	0.260 for dissolved oxygen, DO < 0.05 parts per million (ppm)
	=	0.172 for DO ≥ 0.05 ppm

Bounding F_{en} values are determined or, where necessary, computed for each load pair in the detailed fatigue calculation for each component. The environmental fatigue is then determined as $U_{env} = (U)(F_{en})$, where U is the original fatigue usage and U_{env} is the environmentally assisted fatigue (EAF) usage factor. All calculations can be found in Excel spreadsheet "*VY-16Q-303 (Env. Fat. Calcs).xls*" associated with this calculation.

From Reference [7], for the BWR, typical DO levels range from just over 200 ppb for normal water chemistry (NWC) conditions to less than 10 ppb for hydrogen water chemistry (HWC) conditions. Typical HWC system availabilities are greater than 90%. Based on VY-specific water chemistry input for Entergy [8], which is also contained in Appendix A of this calculation, the input shown in Table 1 is defined for use in this calculation.

The water chemistry input covers several portions of the RPV, as well as the feedwater and recirculation lines. Although these regions encompass more areas than needed to address the two components of interest in this calculation, environmental fatigue multipliers are developed for all of these regions in this calculation for potential use in other evaluations associated with this project.

Therefore, based on Table 1 and for the purposes of this calculation, the following is assumed:

- Over the 60-year operating life of the plant, HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time.
- All operation through 11/1/2003 was assumed as NWC using the dissolved oxygen values from the "Pre-NMCA" column in Appendix A, and all operation after 11/1/2003 was assumed as HWC using the maximum oxygen values from the "Post-NMCA + HWC (OLP)", "Post-NMCA + HWC (EPU)", and "Future Operation" columns in Appendix A.
- Recirculation line DO is 122 ppb pre-HWC and 48 ppb post-HWC.
- Feedwater line DO is 40 ppb for pre-HWC and 40 ppb for post-HWC conditions.
- RPV Upper Region DO is 114 ppb pre-HWC and 97 ppb post-HWC.
- RPV Beltline DO is 123 ppb pre-HWC and 46 ppb post-HWC.
- RPV Bottom Head Region DO is 128 ppb pre-HWC and 69 ppb post-HWC.

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Based on the above typical DO levels, bounding F_{en} multipliers for each of the three applicable materials (carbon, low alloy, and stainless steels) are shown in Tables 2 through 6 for the various RPV and piping regions.

The projected number of cycles used in this calculation is based on the number of cycles actually experienced by the plant in the past and forward-projected with some additional margin for 60 years of operation, as documented in Reference [9]. In addition, the latest governing stress analysis for each location was utilized, and any relevant effects of Extended Power Uprate (EPU) operation were incorporated as necessary. With these assumptions, the cumulative usage factor (CUF) values documented in this calculation are considered applicable for sixty years of operation including all relevant EAF and EPU effects.

4.0 CALCULATIONS

The analyses for the NUREG/CR-6260 locations identified in Section 2.0 are provided in this section. As previously noted, the fatigue calculations for 60 years for all locations make use of the 60-year projected cycles for VY from Reference [9], and incorporate EPU effects.

Since the F_{en} methodology documented in References [4] and [5] is relatively "new" technology, it is intended to apply to "modern-day" fatigue analyses, i.e., applied to fatigue analyses that use current ASME Code fatigue curves, etc. Therefore, to be consistent with this approach, the evaluation for the all locations will also utilize modern-day fatigue calculation methodology using the 1998 Edition, 2000 Addenda of the ASME Code [11]. This involves applying a Young's Modulus correction factor (i.e., $E_{fatigue\ curve}/E_{analysis}$) to the calculated stresses, applying K_e where appropriate, and utilizing the 2000 Addenda fatigue curve.

NOTE: It is recognized that some of the references used in this calculation are not the latest revision; for example, Reference [12] (VYC-378, Revision 0) has been revised. However, the details necessary to perform the evaluations in this calculation are not necessarily contained in the latest revision of all documents. Therefore, wherever necessary, the appropriate revision of the governing document is referenced in order to obtain all appropriate inputs necessary to perform the EAF calculations. So, it should be recognized that, despite using what appear to be outdated revisions of some references, use of these references is for input data use only. All calculations represent the latest available analyses for all locations.

NOTE: Hand calculations may yield results slightly different than the values shown in the tables of this calculation due to round-off based on the significant figures utilized by the spreadsheet used for these calculations.

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4.1 RPV Lower Head

The 60-year CUF value (without EAF effects) for the RPV shell/bottom head location was reported in Table 4.3-3 of the VY LRA submittal to be 0.400. The EAF CUF estimated by Entergy for this location was 0.98, based on an overall F_{en} of 2.45. Based on this result, further refined analysis would not normally be necessary to show acceptable EAF CUF results for this component. However, the calculation for this location is updated in this section to reflect the updated water chemistry information supplied for this project.

The CUF value reported in the VY LRA for the RPV shell/bottom head location is 0.400. This value is the original design basis CUF from the RPV Stress Report, as noted on page B8 of Reference [12]. However, as noted on page A61 of Reference [12], this CUF corresponds to Point 8, which is located on the outside surface of the RPV bottom head at the junction with the support skirt. Therefore, this location is not exposed to the reactor coolant, and EAF effects do not apply. Based on this, evaluation of the limiting location along the inside surface of the RPV bottom head was performed.

Based on a review of the primary plus secondary stresses tabulated for all locations along the bottom head on page A52 of Reference [12], Point 14 was selected for EAF evaluation. Per Section 3.2.1.2 of Reference [13], none of the CUF values for the RPV bottom head region were evaluated for the effects of EPU, as the CUF values are below the EPU screening criteria value of 0.5. Therefore, as a part of the evaluation for this location, EPU effects were included. Per References [14] and [19], the RPV shell material is low alloy steel (A-533, Grade B).

The new CUF calculation for Point 14 for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 7. The CUF for 40 years (without EAF effects) is 0.0057.

The fatigue calculation for 60 years for the RPV shell/bottom head location is also shown in Table 7. The results show a CUF (without EAF effects) of 0.0085 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RPV shell/bottom head location is shown in Table 7. Bounding F_{en} multipliers were applied in the calculations. RPV bottom head water chemistry conditions from Tables 1 and 6 are used for this location. The results show an EAF adjusted CUF of 0.0809 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The CUF determined for Point 14 is very low. Comparison to other locations of the RPV shell/bottom head region indicates it is not the limiting location from a fatigue perspective. Review of the CUF values in Table 3-1 of Reference [15] reveals that the shroud support (at vessel wall junction) location is potentially more limiting, so EAF evaluation of that location is also performed.

Per page S3-99f of Reference [16], the design basis CUF of 0.06 is for Point 9. Page S3-85 of Reference [16] reveals that this point is on the RPV shell at the junction of the shroud support plate. Per References [14] and [19], the RPV shell material is low alloy steel (A-533, Grade B).

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The revised and updated CUF calculation for Point 9 for 40 years, which includes the use of updated methodology and incorporates EPU effects, is shown at the top portion of Table 8. The CUF for 40 years (without EAF effects) is 0.0549. This CUF value is more limiting than the RPV shell/bottom head location evaluated in Table 7, so it is considered to be the governing location for VY with respect to the equivalent NUREG/CR-6260 RPV shell/bottom head location.

The fatigue calculation for 60 years for the RPV shell/shroud support location is also shown in Table 8. The results show a CUF (without EAF effects) of 0.0774 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RPV shell/shroud support location is shown in Table 8. Bounding F_{en} multipliers were applied in the calculations. RPV bottom head water chemistry conditions from Table 6 are used for this location. The results show an EAF adjusted CUF of 0.7364 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

4.2 RR Inlet Nozzle

For conservatism due to the different materials involved, two locations are evaluated for the RR inlet nozzle: (1) the limiting location in the nozzle forging, and (2) the limiting location in the safe end.

The 60-year CUF value (without EAF effects) for the RR inlet nozzle in the VY LRA submittal is 0.610. However, that analysis used conservative transient definitions and cyclic projections for 60 years of operation that have since been updated. The applicable CUF values are those shown in Table 3-1 of Reference [15] (0.1058 for the safe end, and 0.03 for the nozzle for 40-years), except that these values are pre-EPU.

For the RR inlet nozzle forging, the governing CUF calculation is shown on page B28 of Reference [12], where a value of 0.03 was obtained. From pages A269 and A270 of Reference [12], the CUF calculation corresponds to Point 12 in the nozzle forging, which is on the outside surface of the nozzle on the outboard end of the nozzle transition. Although this location is not exposed to the reactor coolant, it will be conservatively evaluated for EAF effects as it is the bounding fatigue location in the nozzle forging. As a part of the evaluation for this location, EPU effects were included. Per page I-S8-4 of Reference [17], the RR inlet nozzle material is low alloy steel (A-508 Class II).

The new CUF calculation for Point 12 for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 9. The CUF for 40 years (without EAF effects) is 0.0433.

The fatigue calculation for 60 years for the RR inlet nozzle forging location is also shown in Table 9. The results show a CUF (without EAF effects) of 0.0650 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RR inlet nozzle forging location is shown in Table 9. Bounding F_{en} multipliers were applied in the calculations. RPV beltline water chemistry conditions from Table 5 are used for this location. The results show an EAF adjusted CUF of 0.5034 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0)

For the RR inlet nozzle safe end, the governing CUF calculation is shown on page B27 of Reference [12], where a value of 0.1058 was obtained. From pages A257 and A259 of Reference [12], the CUF calculation corresponds to Line 6 at the inside surface of the safe end. Page A238 of Reference [12] reveals that this location is location at the nozzle-to-safe end weld. Per Section 3.2.1.2 of Reference [13], the CUF value for the RR inlet nozzle safe end was evaluated for the effects of EPU, since the original CUF calculated in Reference [18] was 0.551 (which was adjusted downward to 0.1058 by Entergy in Reference [12] based on further refined evaluation). Therefore, as a part of the evaluation for this location, EPU effects were included. Per page 8 of Reference [18], the RR inlet nozzle safe end material is 316L stainless steel.

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The new CUF calculation for the RR inlet nozzle safe end for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 10. The CUF for 40 years (without EAF effects) is 0.0017.

The fatigue calculation for 60 years for the RR inlet nozzle safe end location is also shown in Table 10. The results show a CUF (without EAF effects) of 0.0017 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RR inlet nozzle safe end location is shown in Table 10. Bounding F_{en} multipliers were applied in the calculations. Recirculation line water chemistry conditions from Table 2 are used for this location. The results show an EAF adjusted CUF of 0.0199 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0)

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5.0 RESULTS OF ANALYSIS

The final environmental fatigue results contained in Sections 4.1 and 4.2 (and associated Tables 7 through 10) for the RPV shell/bottom head and RR inlet nozzle locations are summarized in Table 11.

6.0 CONCLUSIONS AND DISCUSSION

In this calculation, EAF calculations were performed in accordance with the GALL Report [2] for the following VY locations:

- RR inlet nozzle, consisting of the following bounding locations:
  - Nozzle forging (low alloy steel)
  - Safe end (stainless steel)
- RPV shell/bottom head, consisting of the following bounding locations:
  - Limiting bottom head shell inside surface location (low alloy steel)
  - Limiting RPV shell/shroud support location (low alloy steel)

The above locations were selected based on the locations identified in NUREG/CR-6260 for the older vintage GE plant and plant-specific fatigue calculations that determined the limiting locations for VY. Calculations for the remaining NUREG/CR-6260 locations will be documented in other analyses performed under this project.

The EAF results for the locations identified above are shown in Table 11. These results indicate that the fatigue usage factors, including environmental effects, are within the allowable value for 60 years of operation for all locations evaluated. The calculations for all locations make use of the 60-year projected cycles for VY and incorporate EPU effects. Therefore, no additional evaluation is required for these components, and the GALL requirements are satisfied.

7.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.

2. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.

3. USAS B31.1.0 – 1967, USA Standard Code for Pressure Piping, "Power Piping," American Society of Mechanical Engineers, New York.

4. NUREG/CR-6583 (ANL-97/18), "Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels," March 1998.

5. NUREG/CR-5704 (ANL-98/31), "Effects of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steels," April 1999.

6. EPRI/BWRVIP Memo No. 2005-271, "Potential Error in Existing Fatigue Reactor Water Environmental Effects Analyses," July 1, 2005.

7. REDACTED

8. "Vermont Yankee Dissolved Oxygen (DO) Levels for Use in EAF Evaluations," page 11 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

9. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

10. VY LRA, page 1-4 (*included as Appendix B to this calculation*).

11. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, and Section II, Materials, Part D, "Properties (Customary)," 1998 Edition including the 2000 Addenda.

12. Yankee Atomic Electric Company Calculation No. VYC-378, Revision 0, "Vermont Yankee Reactor Cyclic Limits for Transient Events," 10/16/85, SI File No. VY-05Q-211.

13. REDACTED

14. GE Nuclear Energy Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," June 2, 2003, SI File No. VY-05Q-236.

15. Structural Integrity Associates Report No. SIR-01-130, Rev. 0, "System Review and Recommendations for a Transient and Fatigue Monitoring System at the Vermont Yankee Nuclear Power Station," February 2002, SI File No. W-VY-05Q-401.

16. CB&I RPV Stress Report, Section S3, Revision 4, "Stress Analysis, Shroud Support, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2-3-70, SI File No. VY-16Q-203.

17. CB&I RPV Stress Report, Section S8, Revision 4, "Stress Analysis, Recirculation Inlet Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2-3-70, SI File No. VY-16Q-203.

18. GE Nuclear Energy Certified Stress Report No. 23A4292, Revision 4, "Reactor Vessel – Recirculation Inlet Safe End Nozzle," March 12, 1986, SI File No. VY-16Q-203.

19. Entergy Drawing No. 5920-5752, Revision 3 (CB&I Drawing No. R15, Revision 1), "Vessel & Attachments Mat'l. Identifications," 1/20/88, SI File No. VY-16Q-209.

Table 1: Water Chemistry Calculations

Date of HWC Implementation:	11/01/2003	*(see Appendix A)*
Availability of HWC System Since HWC Implementation:	98.54%	*(see Appendix A)*
Projected Future HWC System Availability:	98.5%	*(see Appendix A, assume same as recent experience)*
Recirculation Line DO		
pre-HWC:	122	ppb *(see Appendix A)*
post-HWC:	48	ppb *(see Appendix A)*
Feedwater Line DO		
pre-HWC:	40	ppb *(see Appendix A)*
post-HWC:	40	ppb *(see Appendix A)*
RPV Upper Region DO		
pre-HWC:	114	ppb *(see Appendix A)*
post-HWC:	97	ppb *(see Appendix A)*
RPV Beltline Region DO		
pre-HWC:	123	ppb *(see Appendix A)*
post-HWC:	46	ppb *(see Appendix A)*
RPV Bottom Head Region DO		
pre-HWC:	128	ppb *(see Appendix A)*
post-HWC:	69	ppb *(see Appendix A)*
Plant Startup Date:	03/22/1972	*(see Appendix B)*
Time at pre-HWC Conditions:	31.61	years *(calculated, includes leap years.)*
Date of Calculations:	04/30/2007	
Time Since HWC Implementation:	3.49	years *(calculated, includes leap years.)*
Projected Future Time for HWC Operation:	24.90	years *(calculated, includes leap years.)*
Overall HWC Availability:	47%	

Note: All operation through 11/1/2003 was assumed as NWC using the dissolved oxygen values from the "Pre-NMCA" column in Appendix A, and all operation after 11/1/2003 was assumed as HWC using the maximum oxygen values from the "Post-NMCA + HWC (OLP)", "Post-NMCA + HWC (EPU)", and "Future Operation" columns in Appendix A.

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Table 2: Bounding F_{en} Multipliers for Recirculation Line

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
250	482	2.45
288	550	2.45

Thus, maximum F_{en} = 2.45

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm; so $O^* = \ln(0.122/0.04) = 1.115$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.40
250	482	7.89
288	550	12.29

Thus, maximum F_{en} = 12.29

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm, so $O^* = \ln(0.122/0.04) = 1.115$

Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	3.12
250	482	5.59
288	550	8.71

Thus, maximum F_{en} = 8.71

[$T^* = (T-150)$ for $T > 150°C$]

Stainless Steel:

$F_{en} = \exp(0.935 - T^*\varepsilon^*O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm < 0.050 ppm, so $O^* = 0.260$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	15.35
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	15.35
	Thus, maximum F_{en} =	15.35

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

so F_{en} =	2.55
so F_{en} ranges from	2.55
to	8.36
so F_{en} =	8.36
Thus, maximum F_{en} =	8.36

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Table 3: Bounding F_{en} Multipliers for Feedwater Line

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
250	482	2.45
288	550	2.45

Thus, maximum F_{en} = 2.45

For a BWR with NWC environment (pre-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
250	482	2.45
288	550	2.45

Thus, maximum F_{en} = 2.45

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

For a BWR with NWC environment (pre-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

[$T^* = (T-150)$ for $T > 150°C$]

There is no stainless steel in the Class 1 feedwater line.

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Table 4: Bounding F_{en} Multipliers for RPV Upper Region

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	3.90
250	482	6.20
288	550	8.82

Thus, maximum F_{en} = 8.82

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.25
250	482	7.35
288	550	11.14

Thus, maximum F_{en} = 11.14

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$

Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	2.77
250	482	4.40
288	550	6.25

Thus, maximum F_{en} = 6.25

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$

Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	3.01
250	482	5.21
288	550	7.90

Thus, maximum F_{en} = 7.90

[$T^* = (T-150)$ for $T > 150°C$]

Stainless Steel:

$F_{en} = \exp(0.935 - T^*\varepsilon^*O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm > 0.050 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	8.36
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	8.36
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

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Table 5: Bounding F_{en} Multipliers for RPV Beltline Region

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 46 ppb = 0.046 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
269.45	517.01	2.45
288	550	2.45

Thus, maximum F_{en} = 2.45

For a BWR with NWC environment (pre-HWC implementation):
DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.42
269.45	517.01	10.00
288	550	12.43

Thus, maximum F_{en} = 12.43

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 46 ppb = 0.046 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

For a BWR with NWC environment (pre-HWC implementation):
DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$

Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	3.13
250	482	5.64
288	550	8.81

Thus, maximum F_{en} = 8.81

[$T^* = (T-150)$ for $T > 150°C$]

Stainless Steel:

$F_{en} = \exp(0.935 - T^*\varepsilon^*O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 46 ppb = 0.046 ppm < 0.050 ppm, so $O^* = 0.260$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	15.35
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	15.35
	Thus, maximum F_{en} =	15.35

For a BWR with NWC environment (pre-HWC implementation):
DO = 123 ppb = 0.123 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	8.36
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

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Table 6: Bounding F_{en} Multipliers for RPV Bottom Head Region

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 69 ppb = 0.069 ppm, so $O^* = \ln(0.069/0.04) = 0.545$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	3.27
250	482	4.34
288	550	5.39

Thus, maximum F_{en} = 5.39

For a BWR with NWC environment (pre-HWC implementation):
DO = 128 ppb = 0.128 ppm, so $O^* = \ln(0.128/0.04) = 1.163$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.51
250	482	8.29
288	550	13.17

Thus, maximum F_{en} = 13.17

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 69 ppb = 0.069 ppm, so $O^* = \ln(0.069/0.04) = 0.545$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	2.31
250	482	3.08
288	550	3.82

Thus, maximum F_{en} = 3.82

For a BWR with NWC environment (pre-HWC implementation):
DO = 128 ppb = 0.128 ppm, so $O^* = \ln(0.128/0.04) = 1.163$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	3.20
250	482	5.88
288	550	9.34

Thus, maximum F_{en} = 9.34

[$T^* = (T-150)$ for $T > 150°C$]

Stainless Steel:

$F_{en} = \exp(0.935 - T^*\varepsilon^*O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 69 ppb = 0.069 ppm > 0.050 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	8.36
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

For a BWR with NWC environment (pre-HWC implementation):
DO = 128 ppb = 0.128 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

	so F_{en} =	2.55
	so F_{en} ranges from	2.55
	to	8.36
	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

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Table 7: EAF Evaluation for RPV Shell/Bottom Head Location

Component: RPV Shell/Bottom Head
NUREG/CR-6260 CUF: 0.032 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-102
Stress Report CUF: 0.0057 *(for Point 14, see below)*
Material: Low Alloy Steel *(Material = A-533 Gr. B per References [14] and [19])*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.149	*Conservatively used minimum E of 26.1 from Section S2 Appendix of RPV Stress Report.*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]*
K_t =	1.000	*stress concentration factor*
m =	2.0	*NB-3228.5 of ASME Code, Section III [11]*
n =	0.2	*NB-3228.5 of ASME Code, Section III [11]*
S_m =	26,700	psi *(ASME Code, Section II, Part D [11])*

P_L+P_B+Q *(see Note 1)*	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 4)*	N *(see Note 5)*	U
44,526	1.00	25,762	200	35,300	0.0057
				Total, U_{40} =	**0.0057**

Notes:
1. *P_L+P_B+Q is obtained for Point 14 from p. A52 of VYC-378, Rev. 0.*
2. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
3. *$S_{alt} = 0.5 * K_e * K_t * E_{fatigue\ curve}/E_{analysis} *$ Power Uprate $* (P_L+P_B+Q)$.*
4. *n for 40 years is the number of Heatup-Cooldown cycles, per p. B8 of VYC-378, Rev. 0.*
5. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
6. *n for 60 years is the projected number of Heatup-Cooldown cycles.*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 6)*	N *(see Note 4)*	U
44,526	1.00	25,762	300	35,300	0.0085
				Total, U_{60} =	**0.0085**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions = 5.39 *(from Table 6)*
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions = 13.17 *(from Table 6)*

$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47 =$ 0.0809
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ = 9.51

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Table 8: EAF Evaluation for Limiting RPV Shell/Shroud Support Location

Component: RPV Shell at Shroud Support
NUREG/CR-6260 CUF: 0.032 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-102
Stress Report CUF: 0.0549 *(for Point 9, see below)*
Material: Low Alloy Steel *(Material = A-533 Gr. B per References [14] and [19])*

Design Basis CUF Calculation for 40 years:

Hydrotest σ_ϕ =	26,240	psi	*(p. S3-97 of RPV Stress Report)*
Hydrotest σ_r =	-1,250	psi	*(p. S3-97 of RPV Stress Report)*
Stress Concentration Factor, K_t =	2.40		*(p. S3-99d of RPV Stress Report)*
Hydrotest $K_t\sigma_\phi$ =	62,976	psi	*(p. S3-97 of RPV Stress Report)*
Improper Startup σ_ϕ =	28,060	psi	*(p. S3-98 of RPV Stress Report)*
Improper Startup σ_r =	-1,025	psi	*(p. S3-98 of RPV Stress Report)*
Improper Startup Skin Stress =	156,099	psi	*(p. S3-98 of RPV Stress Report)*
Improper Startup $K_t\sigma_\phi$ + Skin Stress =	223,443	psi	*(p. S3-98 of RPV Stress Report)*
Warmup σ_ϕ =	-5,707	psi	*(p. S3-99a of RPV Stress Report)*
Warmup σ_r =	-102	psi	*(p. S3-99a of RPV Stress Report)*
Warmup $K_t\sigma_\phi$ =	-13,696	psi	*(p. S3-99a of RPV Stress Report)*
$E_{fatigue\ curve}/E_{analysis}$ =	1.0417		*30.0 / 28.8 per S3-99f of RPV Stress Report and ASME Code fatigue curve*
Power Uprate =	1.0067		*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]*
m =	2.0		*NB-3228.5 of ASME Code, Section III [11]*
n =	0.2		*NB-3228.5 of ASME Code, Section III [11]*
S_m =	26,700	psi	*(ASME Code, Section II, Part D [11])*

P_L+P_B+Q *(see Note 1)*	Events	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 4)*	N *(see Note 5)*	U
34,690	Improper Startup - Warmup	1.00	124,825	5	332	0.0151
33,095	Hydrotest - Warmup	1.00	40,804	322	8,095	0.0398
					Total, U_{40} =	**0.0549**

Notes:
1. *P_L+P_B+Q is computed for Point 9 based on the $[(\sigma_\phi - \sigma_r)_{Event\ 1} - (\sigma_\phi - \sigma_r)_{Event\ 2}]$ stress intensity.*
2. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
3. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis} *$ Power Uprate $* [(K_t\sigma_\phi - \sigma_r)_{Event\ 1} - (K_t\sigma_\phi - \sigma_r)_{Event\ 2}]$.*
4. *n for 40 years is the number of cycles as follows per p. S3-99e and S3-99f of the RPV Stress Report:*

Improper Startup =	*5*	*cycles*
Hydrotest =	*2*	*cycles*
Isothermal at 70°F and 1,000 psi =	*120*	*cycles (same as number of Startup events)*
Warmup-Cooldown =	*199*	*cycles*
Warmup-Blowdown =	*1*	*cycle*
TOTAL =	*327*	*cycles*

5. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
6. *n for 60 years is the projected number of cycles as follows:*

Improper Startup =	*1*	*cycles*
Hydrotest =	*1*	*cycles*
Isothermal at 70°F and 1,000 psi =	*300*	*cycles (same as number of Startup events)*
Warmup-Cooldown =	*300*	*cycles*
Warmup-Blowdown =	*1*	*cycle*
TOTAL =	*603*	*cycles*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*		K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 6)*	N *(see Note 4)*	U
34,690	Improper Startup - Warmup	1.00	124,825	1	332	0.0030
33,095	Hydrotest - Warmup	1.00	40,804	602	8,095	0.0744
					Total, U_{60} =	**0.0774**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions = 5.39 *(from Table 6)*
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions = 13.17 *(from Table 6)*

$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47 =$ 0.7364
Overall Multiplier = $U_{env\text{-}60}/U_{60} =$ 9.51

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Table 9: EAF Evaluation for RR Inlet Nozzle Forging Location

Component: Recirculation Inlet Nozzle Forging
NUREG/CR-6260 CUF: 0.310 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-105
Stress Report CUF: 0.0433 *(updated for Point 12, see below)*
Material: Low Alloy Steel *(Material = A-508 Cl. II per p. I-S8-4 of CBIN Stress Report Section S8)*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.1278	*= 30.0 / 26.6 (per p. I-S8-24 of CBIN Stress Report Section S8 and ASME Code fatigue curve)*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]*
K_t =	1.660	*stress concentration factor (p. A270 of VYC-378, Rev. 0 [12])*
m =	2.0	*NB-3228.5 of ASME Code, Section III [11]*
n =	0.2	*NB-3228.5 of ASME Code, Section III [11]*
S_m =	26,700	psi *(ASME Code, Section II, Part D [11])*

P_L+P_B+Q *(see Note 1)*	Skin Stress *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 6)*	U
43,110	15,145	1.00	49,224	200	4,614	0.0433
					Total, U_{40} =	**0.0433**

Notes:
1. *P_L+P_B+Q is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.*
2. *Skin Stress is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.*
3. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
4. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis}$ * Power Uprate * [$(P_L+P_B+Q)\ K_t$ + Skin Stress].*
5. *n for 40 years is the number of Heatup-Cooldown cycles, per p. B28 of VYC-378, Rev. 0.*
6. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
7. *n for 60 years is the projected number of Heatup-Cooldown cycles.*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	Skin Stress *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 7)*	N *(see Note 6)*	U
43,110	15,145	1.00	49,224	300	4,614	0.0650
					Total, U_{60} =	**0.0650**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions = 2.45 *(from Table 5)*
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions = 12.43 *(from Table 5)*

$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47$ = 0.5034
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ = 7.74

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Table 10: EAF Evaluation for RR Inlet Nozzle Safe End Location

Component: Recirculation Inlet Nozzle Safe End
NUREG/CR-6260 CUF: 0.310 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-105
Stress Report CUF: 0.0017 *(updated for Location 6-I, see below)*
Material: Stainless Steel *(316L per p. 8 of 23A4292, Rev. 4)*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.1076	*= 28.3 / 25.55 (per p. 62 of Reference [18] and ASME Code fatigue curve)*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]*
K_t =	1.280	*stress concentration factor (p. B27 of VYC-378, Rev. 0 [12])*
m =	1.7	*NB-3228.5 of ASME Code, Section III [11]*
n =	0.3	*NB-3228.5 of ASME Code, Section III [11]*
S_m =	16,600	psi *(ASME Code, Section II, Part D [11])*

P_L+P_B+Q *(see Note 1)*	P+Q+F *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 6)*	U
47,183	36,972	1.00	26,385	2,076	1,242,266	0.0017
					Total, U_{40} =	**0.0017**

Notes:
1. *P_L+P_B+Q is obtained for Surface I (after weld overlay) from p. 117 of Reference [18].*
2. *P+Q+F is obtained for Point 6-I from p. 118 of Reference [18] (BEFORE weld overlay).*
3. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
4. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis}$ * Power Uprate * [(P+Q+F) K_t].*
5. *n for 40 years is the number of cycles as follows per p. B26 of VYC-378, Rev. 0:*

Design Hydrotest =	*130*	
Loss of Feedpumps Composite:		
Startup/Shutdown =	*290*	
SRV Blowdown =	*8*	
Loss of Feedwater Pumps	*30*	*10 events x 3 up/down cycles per event*
SCRAM =	*270*	
Normal +/- Seismic =	*11*	*10 cycles of upset seismic, plus 1 Level C seismic event*
Normal =	*739*	*= Sum of all of above events*
Zeroload =	*598*	*= Startup/Shutdown + SRV Blowdown + Scram + LOFP*
Total number of cycles =	*2,076*	

6. *N obtained from Figure I-9.2 of Appendix I of ASME Code, Section III.*
7. *n for 60 years is the projected number of cycles as follows:*

Design Hydrotest =	*120*	
Loss of Feedpumps Composite:		
Startup/Shutdown =	*300*	
SRV Blowdown =	*1*	
Loss of Feedwater Pumps	*30*	*10 events x 3 up/down cycles per event*
SCRAM =	*289*	*All remaining scrams*
Normal +/- Seismic =	*11*	*Assume the same*
Normal =	*751*	*= Sum of all of above events*
Zeroload =	*620*	*= Startup/Shutdown + SRV Blowdown + Scram + LOFP*
Total number of cycles =	*2,122*	

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	P+Q+F *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 7)*	U
47,183	36,972	1.00	26,385	2,122	1,242,266	0.0017
					Total, U_{60} =	**0.0017**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions =	15.35	*(from Table 2)*
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions =	8.36	*(from Table 2)*
$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47$ =	**0.0199**	
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ =	**11.64**	

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Table 11: Summary of EAF Evaluation Results for VY

No.	Component	Material	40-Year Design CUF (1)	60-Year CUF (2)	Overall Environmental Multiplier	60-Year Environmental CUF (2,3)
1	RPV Shell/Bottom Head	Low Alloy Steel	0.0057	0.0085	9.51	0.0809
2	RPV Shell at Shroud Support	Low Alloy Steel	0.0549	0.0774	9.51	0.7364
3	Recirculation Inlet Nozzle Safe End	Stainless Steel	0.0017	0.0017	11.64	0.0199
4	Recirculation Inlet Nozzle Forging	Low Alloy Steel	0.0433	0.0650	7.74	0.5034

Notes:
1. Updated 40-year CUF calculation based on recent ASME Code methodology and design basis cycles.
2. CUF results using updated ASME Code methodology and actual cycles accumulated to-date and projected to 60 years.
3. An F_{en} multiplier was used for each respective component with the following conditions:
   + 47% HWC conditions and 53% NWC conditions

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APPENDIX A

VY WATER CHEMISTRY INFORMATION [8]

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Location	Pre-NMCA 1593 MWth (OLP)	Post-NMCA + HWC 1593 MWth (OLP)	Post-NMCA + HWC 1912 MWth (EPU)	Future Operation Post-NMCA + HWC 1912 MWth (EPU)
	---	Average Availability 98.5%	Average Availability 98.5%	Average Availability 99%
	Implementation Date = 11/1972	NMCA Application Date = 04/27/2001 HWC Implementation Date = 11/01/2003	EPU Implementation Date = 5/2006	---
FW Line	40 ppb	40 ppb	40 ppb	40 ppb
Recirc. Line	122 ppb	48 ppb	34 ppb	34 ppb
RPV Bottom Head **	128 ppb	69 ppb	55 ppb	55 ppb
RPV Upper Region	114 ppb	97 ppb	90 ppb	90 ppb
RPV Beltline Region	123 ppb	46 ppb	31 ppb	31 ppb

** RPV Bottom head at "Lower Plenum, Downflow" (i.e. outside core support columns)

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APPENDIX B

VY LICENSE DATE [10]

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Jay K. Thayer Vice President - Vermont Yankee Nuclear Power Station	Entergy Nuclear Vermont Yankee Corporate Office P.O. Box 0500 185 Old Ferry Road Brattleboro, VT 05302-0500

1.1.5 Class and Period of License Sought

ENO requests renewal of the facility operating license for VYNPS (facility operating license DPR-28) for a period of 20 years. The license was issued under Section 104b of the Atomic Energy Act of 1954 as amended. License renewal would extend the facility operating license from midnight March 21, 2012, to midnight March 21, 2032.

This application also applies to renewal of those NRC source materials, special nuclear material, and by-product material licenses that are subsumed or combined with the facility operating license.

1.1.6 Alteration Schedule

ENO does not propose to construct or alter any production or utilization facility in connection with this renewal application.

Structural Integrity Associates, Inc.	File No.: VY-16Q-304 EXHIBIT D
CALCULATION PACKAGE	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Outlet Nozzle Finite Element Model

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-6, Appendix: A1-A20	Initial Issue	Terry J. Herrmann 7/12/2007	Minghao Qin 7/12/2007 Jennifer E. Smith 7/12/2007

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1.0 OBJECTIVE

The objective of this calculation is to create a finite element model of the Vermont Yankee Nuclear Power Station recirculation outlet nozzle. This model will be used to develop a Green's Function to be used in a subsequent fatigue analysis.

2.0 GEOMETRY / MATERIAL PROPERTIES

A 2-D axisymmetric finite element model (FEM) of the nozzle was developed with element type PLANE182. The developed model includes the safe end, the nozzle forging, a portion of the vessel shell, and cladding. The model used the vessel radius multiplied by a factor 2.0 due to the model being axisymmetric.

The 2-D axisymmetric FEM was constructed using the dimensions and information from References [4 and 5] based on ANSYS [2] finite element software. Figure 1 shows the resulting finite element model.

The materials of the various components of the model are listed below:

- Safe End – SA182 F316 [4] (16Cr-12Ni-2Mo)
- Piping – SA376 TP316 [7] (16Cr-12Ni-2Mo)
- Nozzle Forging – SA508 Class 2 [5] (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B [6] (Mn-1/2Mo-1.2Ni)
- Cladding – SA240 Type 304 [1, Sheet 7] (18Cr-8Ni)

Material properties for these materials are based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [3] and are shown in Table 1. The properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FEM model for Green's Function development.

3.0 PROGRAM INPUT

The input file, RON_VY.INP (included in Appendix A), creates the finite element model for the recirculation outlet nozzle.

4.0 REFERENCES

1. GE. Stress Report No. 23A4316, Revision 0, "Reactor Vessel Recirculation Outlet Safe End," SI File No. VY-16Q-204.
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. Vermont Yankee Drawing 5920-06623, Rev. 0, (Hitachi, Ltd. Drawing No IOR290-127), "Recirc. Outlet Safe End," SI File No. VY-16Q-204.
5. Vermont Yankee Drawing 5920-00238, Rev. 4, (Chicago Bridge & Iron Company, Contract No. 9-6201, Drawing No. 21), "36"x28" Nozzles Mk N1A/B," SI File No. VY-16Q-204.
6. Vermont Yankee Drawing 5920-05752, Rev. 3, "Vessel & Attachments Material Identifications," SI File No. VY-16Q-209.
7. SI File No. VY-16Q-103, "Vermont Yankee Comments on VY-16Q-304."

Table 1: Material Properties @ 300°F (1)

Material	SA533 Grade B (Mn-1/2Mo-1/2Ni)	SA508 Class 2 (3/4Ni-1/2Mo-1/3Cr-V)	SA240 Type 304 (18Cr-8Ni)	SA182 F316/ SA376 TP316 (16Cr-12Ni-2Mo)
Modulus of Elasticity, e-6 psi	28.0	26.7	27.0	27.0
Coefficient of Thermal Expansion, e-6, in/in/°F	7.7	7.3	9.8	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	9.3
Thermal Diffusivity, ft^2/hr	0.401	0.401	0.160	0.150
Specific Heat, Btu/lb-°F (2)	0.119	0.119	0.125	0.127
Density, lb/in^3	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes:

1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values.
2. Calculated as $[k/(\rho d)]/12^3$.

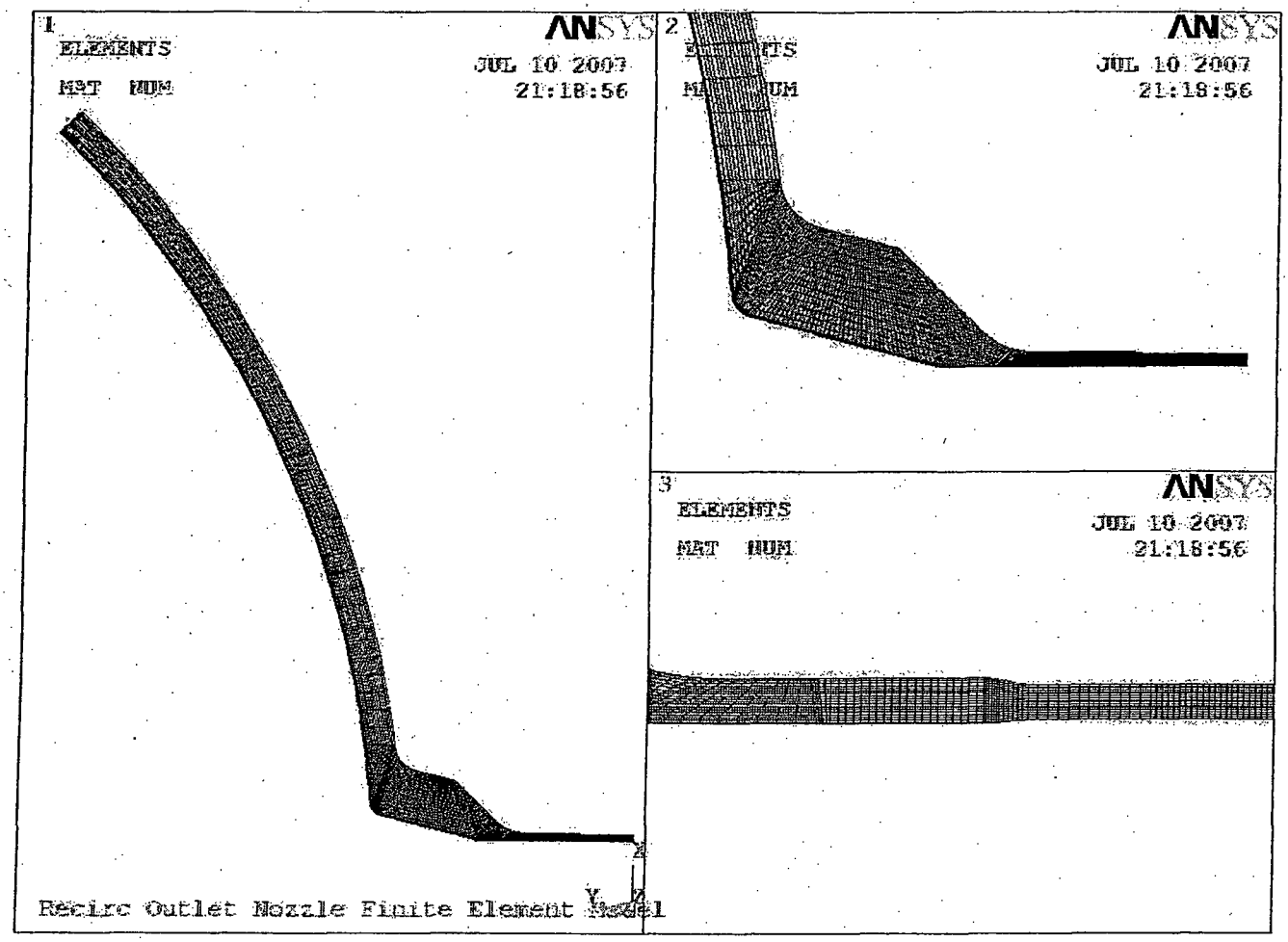

Figure 1: ANSYS Finite Element Model

APPENDIX A

RON_VY.inp

```
finish
/clear,start
/prep7
/title, Recirc Outlet Nozzle Finite Element Model

/com, PLANE182, 2-D Solid
et,1,PLANE182,,,1    !Axisymmetric

/com, ****************************
/com, Material Properties @T=300F
/com, ****************************

/COM, Material #1 (Safe-End and Piping)  SA-182 F316  (16Cr-12Ni-2Mo)
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mp,alpx,1,9.8E-06
mp,kxx,1,9.3/3600/12
mp,c,1,0.127
mp,nuxy,1,0.3
mp,dens,1,0.283

/COM, Material #2 (Nozzle Forging) SA-508 Class 2 (3/4Ni-1/2Mo-1/3Cr-V)
mp,ex,2,26.7E+06
mp,alpx,2,7.3E-06
mp,kxx,2,23.4/3600/12
mp,c,2,0.119
mp,nuxy,2,0.3
mp,dens,2,0.283

/COM, Material #3 (Cladding) SA-240 Type 304 (18Cr-8Ni)
mp,ex,3,27E+06
mp,alpx,3,9.8E-06
mp,kxx,3,9.8/3600/12
mp,c,3,0.125
mp,nuxy,3,0.3
mp,dens,3,0.283

/COM, Material #4 (Vessel) SA-533, GR. B (Mn-1/2Mo-1/2Ni)
mp,ex,4,28.0E+06
mp,alpx,4,7.7E-06
mp,kxx,4,23.4/3600/12
mp,c,4,0.119
mp,nuxy,4,0.3
mp,dens,4,0.283

*AFUN,DEG
/com, *** Geometric Parameters ***
*set,vira,(103+3/16)      !Actual Vessel Inner Radius to base metal used for model
*set,vir,2.0*vira          !2.0 time of  Vessel Inner Radius to base metal used for model
*set,tvw,5+5/8-3/16       !Vessel Wall Thickness
*set,ri1,25.75/2
*set,ro1,28.375/2
*set,L1,5
*set,ro2,28.375/2
```

```
*set,L2,4.25
*set,ro3,28.875/2
*set,ro4,48.75/2
*set,L3,1.5
*set,L4,5.25
*set,L5,7+1/16
*set,L6,12+13/16
*set,L7,9+7/8
*set,L8,9+3/8
*set,L9,31+15/16
*set,L10,L9-12-13/16-tvw
*set,ra,7
*set,rb,1
*set,rc,5.25
*set,rd,2.5
*set,tv,3/16
*set,dimA,vir-(tv*2.0)+L9+11+L1  !Vessel Centerline to End of Safe End used for model
*set,L21,1
*set,L22,4.25
*set,ri21,(25+15/16)/2

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/com, Geometry
/com,*********************************************************************
local,13,0,,dimA,,,,

csys,13

/com, Begin at end of Safe-End - Carbon Section

k, 1, ri1, -1*(dimA)
k, 2, ri1+tv, -1*(dimA)
k, 3, ro1, -1*(dimA)
k, 4, ri1, -1*(dimA-L1)
k, 5, ri1+tv, -1*(dimA-L1)
k, 6, ro1, -1*(dimA-L1)
k, 7, ri1, -1*(dimA-L1-L2)
k, 8, ri1+tv, -1*(dimA-L1-L2)
k, 9, ro2, -1*(dimA-L1-L2)
k, 10, ri1, -1*(dimA-L1-L2-L3)
k, 11, ri1+tv, -1*(dimA-L1-L2-L3)
k, 12, ro3, -1*(dimA-L1-L2-L3)
k, 13, ri1, -1*(dimA-L1-L2-L3-L4)
k, 14, ri1+tv, -1*(dimA-L1-L2-L3-L4)
k, 15, ro3, -1*(dimA-L1-L2-L3-L4)
k, 16, ri1, -1*(dimA-L1-L2-L3-L4-L5)
k, 17, ri1+tv, -1*(dimA-L1-L2-L3-L4-L5)
k, 18, ro3, -1*(dimA-L1-L2-L3-L4-L5)

k,19,    ro4, -1*(dimA-L1-L2-L3-L4-L5-L7)! Temporary Point
l,19,18
l,18,15
lfillt,1,2,ra
k,22,    ro4+(L8+6)*tan(15), -1*(dimA-L1-L2-L3-L4-L5-L7-(L8+6))
l,19,22
```

```
LFILLT,1,4,rb

k, 25, ri1, -1*(dimA-L1-L2-L3-L4-L6)
k, 26, ri1+tv, -1*(dimA-L1-L2-L3-L4-L6)

k, 27, ri1+(L10+tvw+tv+4)*tan(15), -1*(vir-tv-4)
k, 28, ri1+tv+(L10+tvw+tv+4)*tan(15), -1*(vir-tv-4)

k,29,    (vir+tvw+tv)*sin(45), -1*(vir+tvw+tv)*cos(45)
k,30,    0, -1*(vir+tvw+tv)  ! Temporary Point
k,31,    0, 0 ! Temporary Point

larc,29,30,31,vir+tvw+tv

k,32,    (vir+tv)*sin(45), -1*(vir+tv)*cos(45)
k,33,    0, -1*(vir+tv)  ! Temporary Point
larc,32,33,31,vir+tv

k,34,    vir*sin(45), -1*vir*cos(45)
k,35,    0, -1*vir  ! Temporary Point
larc,34,35,31,vir

LSTR,    4,    5
LSTR,    5,    6
LSTR,    6,    9
LSTR,    9,   12
LSTR,   12,   15
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LPTN,P51X

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FITEM,2,37
FITEM,2,40
FITEM,2,42
FITEM,2,44
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LFILLT,4,41,rd, ,
!*
LFILLT,43,8,rd, ,
!*
LFILLT,39,38,rc, ,

FLST,2,3,4,ORDE,3
FITEM,2,1
FITEM,2,3
FITEM,2,5
LCOMB,P51X, ,0
LSTR,    16,    17
LSTR,    17,    21
LSTR,    25,    26
LSTR,    26,    24
LSTR,    22,    30
LSTR,    30,    35
LSTR,    27,    28
LSTR,    28,    33
LSTR,    29,    32
LSTR,    32,    34

k,39,    0, -1*(vir+tvw+tv)

!Create Areas
FLST,2,4,4
FITEM,2,27
FITEM,2,30
FITEM,2,26
FITEM,2,9
AL,P51X
FLST,2,4,4
FITEM,2,28
FITEM,2,29
```

```
FITEM,2,10
FITEM,2,30
AL,P51X
FLST,2,4,4
FITEM,2,11
FITEM,2,32
FITEM,2,10
FITEM,2,14
AL,P51X
FLST,2,4,4
FITEM,2,15
FITEM,2,14
FITEM,2,9
FITEM,2,31
AL,P51X
FLST,2,4,4
FITEM,2,32
FITEM,2,33
FITEM,2,12
FITEM,2,17
AL,P51X
FLST,2,4,4
FITEM,2,16
FITEM,2,17
FITEM,2,31
FITEM,2,34
AL,P51X
FLST,2,4,4
FITEM,2,36
FITEM,2,13
FITEM,2,33
FITEM,2,18
AL,P51X
FLST,2,4,4
FITEM,2,19
FITEM,2,18
FITEM,2,35
FITEM,2,34
AL,P51X
FLST,2,4,4
FITEM,2,2
FITEM,2,5
FITEM,2,36
FITEM,2,21
AL,P51X
FLST,2,4,4
FITEM,2,20
FITEM,2,21
FITEM,2,3
FITEM,2,35
AL,P51X
FLST,2,4,4
FITEM,2,1
FITEM,2,37
FITEM,2,23
```

```
FITEM,2,5
AL,P51X
FLST,2,4,4
FITEM,2,22
FITEM,2,23
FITEM,2,25
FITEM,2,3
AL,P51X
FLST,2,4,4
FITEM,2,38
FITEM,2,42
FITEM,2,37
FITEM,2,8
AL,P51X
FLST,2,4,4
FITEM,2,4
FITEM,2,8
FITEM,2,25
FITEM,2,40
AL,P51X
FLST,2,4,4
FITEM,2,24
FITEM,2,45
FITEM,2,7
FITEM,2,42
AL,P51X
FLST,2,4,4
FITEM,2,6
FITEM,2,7
FITEM,2,44
FITEM,2,40
AL,P51X
FLST,2,4,4
FITEM,2,41
FITEM,2,43
FITEM,2,47
FITEM,2,44
AL,P51X
FLST,2,4,4
FITEM,2,39
FITEM,2,46
FITEM,2,45
FITEM,2,43
AL,P51X

! define materials
FLST,5,8,5,ORDE,2
FITEM,5,1
FITEM,5,-8
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
```

```
AATT,     1, ,  1,     0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,5,5,ORDE,5
FITEM,5,9
FITEM,5,11
FITEM,5,13
FITEM,5,15
FITEM,5,18
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,     2, ,  1,     0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,5,5,ORDE,5
FITEM,5,10
FITEM,5,12
FITEM,5,14
FITEM,5,16
FITEM,5,-17
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,     3, ,  1,     0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*

!/com, Map mesh areas
FLST,5,10,4,ORDE,10
FITEM,5,5
FITEM,5,10
FITEM,5,28
FITEM,5,32
FITEM,5,-33
FITEM,5,36
FITEM,5,-37
FITEM,5,42
FITEM,5,45
FITEM,5,-46
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
```

```
CMSEL,,_Y
!*
LESIZE,_Y1, , ,15, , , , ,1
!*
FLST,5,10,4,ORDE,10
FITEM,5,3
FITEM,5,9
FITEM,5,25
FITEM,5,27
FITEM,5,31
FITEM,5,34
FITEM,5,-35
FITEM,5,40
FITEM,5,44
FITEM,5,47
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,2, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,39
FITEM,5,41
FITEM,5,43
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,80, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,6
FITEM,5,-7
FITEM,5,24
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,20, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,4
FITEM,5,8
FITEM,5,38
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,40, , , , ,1
!*
```

```
FLST,5,3,4,ORDE,3
FITEM,5,1
FITEM,5,22
FITEM,5,-23
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,30, , , , ,1
!*
FLST,5,6,4,ORDE,6
FITEM,5,2
FITEM,5,20
FITEM,5,-21
FITEM,5,26
FITEM,5,29
FITEM,5,-30
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,40, , , , ,1
!*
FLST,5,9,4,ORDE,2
FITEM,5,11
FITEM,5,-19
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,20, , , , ,1
!*

! Meshing
FLST,5,18,5,ORDE,2
FITEM,5,1
FITEM,5,-18
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*
```

```
!Modify the safe end ID
FLST,2,6,5,ORDE,2
FITEM,2,1
FITEM,2,-6
ACLEAR,P51X
FLST,2,6,5,ORDE,2
FITEM,2,1
FITEM,2,-6
ADELE,P51X
FLST,2,9,4,ORDE,7
FITEM,2,9
FITEM,2,14
FITEM,2,-17
FITEM,2,26
FITEM,2,-27
FITEM,2,30
FITEM,2,-31
LDELE,P51X, , ,1

FLST,2,3,4,ORDE,3
FITEM,2,10
FITEM,2,28
FITEM,2,32
LDELE,P51X, , ,1
FLST,3,2,3,ORDE,2
FITEM,3,3
FITEM,3,6
KGEN,2,P51X, , ,-ro2+ri21, , , ,0
FLST,3,1,3,ORDE,1
FITEM,3,2
KGEN,2,P51X, , , ,L22, , ,0
FLST,3,3,3,ORDE,3
FITEM,3,1
FITEM,3,-2
FITEM,3,4
KGEN,2,P51X, , ,tv, , , ,0
FLST,3,2,3,ORDE,2
FITEM,3,10
FITEM,3,-11
KGEN,2,P51X, , , ,-(L3-L21), , ,0
FLST,3,1,3,ORDE,1
FITEM,3,23
KGEN,2,P51X, , ,5, , , ,0
LSTR,     23,     40
FLST,2,2,4,ORDE,2
FITEM,2,9
FITEM,2,12
LPTN,P51X
LDELE,     16, , ,1
FLST,2,4,3
FITEM,2,11
FITEM,2,23
FITEM,2,41
FITEM,2,12
A,P51X
```

```
FLST,2,4,3
FITEM,2,23
FITEM,2,8
FITEM,2,9
FITEM,2,41
A,P51X
FLST,2,4,3
FITEM,2,8
FITEM,2,7
FITEM,2,6
FITEM,2,9
A,P51X
FLST,2,4,3
FITEM,2,7
FITEM,2,5
FITEM,2,3
FITEM,2,6
A,P51X
FLST,2,4,3
FITEM,2,10
FITEM,2,20
FITEM,2,23
FITEM,2,11
A,P51X
FLST,2,4,3
FITEM,2,20
FITEM,2,4
FITEM,2,8
FITEM,2,23
A,P51X
FLST,2,4,3
FITEM,2,4
FITEM,2,2
FITEM,2,7
FITEM,2,8
A,P51X
FLST,2,4,3
FITEM,2,2
FITEM,2,1
FITEM,2,5
FITEM,2,7
A,P51X
FLST,5,8,5,ORDE,4
FITEM,5,1
FITEM,5,-6
FITEM,5,19
FITEM,5,-20
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,     1, ,  1,     0,
CMSEL,S,_Y
```

```
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,4,4,ORDE,4
FITEM,5,15
FITEM,5,-16
FITEM,5,26
FITEM,5,28
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,15, , , , ,1
!*
FLST,5,4,4,ORDE,4
FITEM,5,31
FITEM,5,48
FITEM,5,50
FITEM,5,52
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,2, , , , ,1
!*
FLST,5,6,4,ORDE,6
FITEM,5,9
FITEM,5,-10
FITEM,5,12
FITEM,5,14
FITEM,5,30
FITEM,5,32
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,6, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,11
FITEM,5,17
FITEM,5,49
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,12, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,27
FITEM,5,29
```

```
FITEM,5,51
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,25, , , , ,1
!*
FLST,5,8,5,ORDE,4
FITEM,5,1
FITEM,5,-6
FITEM,5,19
FITEM,5,-20
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*

FLST,2,2,5,ORDE,2
FITEM,2,17
FITEM,2,-18
ACLEAR,P51X

csys,0
k, 51,62/2,0,0
k, 52,62/2,60,0
LSTR,     51,     52
FLST,2,2,5,ORDE,2
FITEM,2,17
FITEM,2,-18
ADELE,P51X
lplo
FLST,2,4,4,ORDE,4
FITEM,2,39
FITEM,2,41
FITEM,2,43
FITEM,2,53
LPTN,P51X
FLST,2,2,4,ORDE,2
FITEM,2,60
FITEM,2,-61
LDELE,P51X, , ,1
FLST,2,4,4
FITEM,2,54
FITEM,2,62
```

```
FITEM,2,55
FITEM,2,44
AL,P51X
FLST,2,4,4
FITEM,2,55
FITEM,2,63
FITEM,2,58
FITEM,2,45
AL,P51X
FLST,2,4,4
FITEM,2,63
FITEM,2,56
FITEM,2,57
FITEM,2,46
AL,P51X
FLST,2,4,4
FITEM,2,47
FITEM,2,59
FITEM,2,57
FITEM,2,62
AL,P51X

CM,_Y,AREA
ASEL, , , ,    18
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,    2, ,  1,    0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,2,5,ORDE,2
FITEM,5,17
FITEM,5,22
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,    3, ,  1,    0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
CM,_Y,AREA
ASEL, , , ,    21
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,    4, ,  1,    0,
CMSEL,S,_Y
```

```
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,3,4,ORDE,3
FITEM,5,54
FITEM,5,-55
FITEM,5,58
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,8, , , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,56
FITEM,5,-57
FITEM,5,59
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,40, , , , ,1
!*
FLST,5,2,5,ORDE,2
FITEM,5,17
FITEM,5,-18
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*
FLST,5,2,5,ORDE,2
FITEM,5,21
FITEM,5,-22
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
```

```
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*

!Simulating Butter
FLST,2,2,5,ORDE,2
FITEM,2,9
FITEM,2,-10
ACLEAR,P51X
FLST,2,2,5,ORDE,2
FITEM,2,9
FITEM,2,-10
ADELE,P51X

KGEN,2,15, , , ,11/16, , ,0
KGEN,2,44, , , ,-0.25, , ,0
KGEN,2,14, , , ,11/16-1.375*tan(7.5), , ,0
KGEN,2,46, , , ,-0.25, , ,0

FLST,2,3,4,ORDE,3
FITEM,2,2
FITEM,2,20
FITEM,2,-21
LDELE,P51X
LSTR,    21,    44
LSTR,    44,    45
LSTR,    45,    15
LSTR,    17,    46
LSTR,    46,    47
LSTR,    47,    14
LSTR,    46,    44
LSTR,    45,    47
LSTR,    13,    16
FLST,3,2,3,ORDE,2
FITEM,3,46
FITEM,3,-47
KGEN,2,P51X, , ,-0.25, , , ,0
LSTR,    48,    46
LSTR,    49,    47
FLST,2,3,4,ORDE,3
FITEM,2,61
FITEM,2,64
FITEM,2,-65
LPTN,P51X
FLST,2,2,4,ORDE,2
FITEM,2,70
FITEM,2,-71
LDELE,P51X, , ,1
FLST,2,4,4
FITEM,2,67
FITEM,2,39
FITEM,2,68
FITEM,2,3
AL,P51X
```

```
FLST,2,4,4
FITEM,2,39
FITEM,2,5
FITEM,2,2
FITEM,2,53
AL,P51X
FLST,2,4,4
FITEM,2,20
FITEM,2,60
FITEM,2,53
FITEM,2,41
AL,P51X
FLST,2,4,4
FITEM,2,72
FITEM,2,68
FITEM,2,69
FITEM,2,41
AL,P51X
FLST,2,4,4
FITEM,2,21
FITEM,2,60
FITEM,2,36
FITEM,2,43
AL,P51X
FLST,2,4,4
FITEM,2,66
FITEM,2,69
FITEM,2,35
FITEM,2,43
AL,P51X

CM,_Y,AREA
ASEL, , , ,    10
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,    2, ,  1,    0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,3,5,ORDE,3
FITEM,5,9
FITEM,5,23
FITEM,5,-24
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,    3, ,  1,    0,
CMSEL,S,_Y
CMDELE,_Y
```

```
CMDELE,_Y1
!*
FLST,5,2,5,ORDE,2
FITEM,5,25
FITEM,5,-26
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT,     1, ,  1,     0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,3,4,ORDE,3
FITEM,5,2
FITEM,5,39
FITEM,5,67
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,10, , , , ,1
!*
FLST,5,6,4,ORDE,6
FITEM,5,20
FITEM,5,-21
FITEM,5,41
FITEM,5,43
FITEM,5,66
FITEM,5,72
CM,_Y,LINE
LSEL, , , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,2, , , , ,1
!*
FLST,5,2,5,ORDE,2
FITEM,5,9
FITEM,5,-10
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
```

```
CMDELE,_Y1
CMDELE,_Y2
!*
FLST,5,4,5,ORDE,2
FITEM,5,23
FITEM,5,-26
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*

save
finish
```

Structural Integrity Associates, Inc.	File No.: VY-16Q-305 EXHIBIT E
CALCULATION PACKAGE	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

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10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Outlet Stress History Development for Nozzle Green Function

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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%), mid (50%), and no (0%) flow thermal loading of the Vermont Yankee Nuclear Power Station recirculation outlet nozzle.

2.0 RECIRCULATION OUTLET NOZZLE MODEL

An axisymmetric finite element model of the recirculation outlet nozzle was developed in Reference [1] using ANSYS [2]. The geometry and model in Reference [1] is used in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FE model for Green's Function development. Table 1 listed the material properties at 300°F. The meshed model is shown in Figure 1.

3.0 APPLIED LOADS

Both pressure and thermal loads will be applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the recirculation outlet nozzle and the vessel wall. A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{cap} = P \cdot \frac{\left(D_i^2\right)}{\left(D_o^2 - D_i^2\right)}$$

where:

P = Pressure = 1,000 psi
D_i = Inner Radius = 12.96875 in
D_o = Outer Radius = 14.18750 in
P_{cap} = Tension stress on the end of the nozzle. (psi)

Therefore, the cap load is 5081.7 psi. The calculated value was given a negative sign in order for it to exert tension on the end of the model. The ANSYS input file VY_RON_P.INP, in the computer files, applies the pressure loading to the geometry in file RON_VY.INP. Figures 2, 3, and 4 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

3.2 Thermal Load

Thermal loads are applied to the recirculation outlet nozzle model. The heat transfer coefficients after power uprate were determined by scaling the values from Reference [4]. These values were determined for various regions of the finite element model and for 100% (28,294 GPM, converted from 12.3 Mlbm/hr [7]), 50% (14,147 GPM), and 0% (0 GPM) flow rates. The temperatures used are based upon a thermal shock from 500°F to 100°F. The calculated heat transfer coefficients for each region are shown below. The GPM values are calculated from the Mlbm/hr values at an average temperature of 300°F.

3.2.1 *Heat Transfer Coefficients*

The heat transfer coefficients for the 100% flow and 50% flow cases were calculated from Reference [4] as follows:

$$h_{Df} = h_{300}\left(\frac{f_{Df}}{25}\right)^{0.8}\left(\frac{26}{D_{Df}}\right)^{0.2}$$

Where:
h_{Df} = the heat transfer coefficient at a Diameter and flow rate
h_{300} = the heat transfer coefficient from Reference [4] at 300°F
f_{Df} = the flow rate corresponding to h_{Df} (ft/sec)
D_{Df} = the diameter corresponding to h_{Df} (in)

The heat transfer coefficients for 0% flow were calculated in spreadsheet Ht_coeffs.xls for natural convection and are shown in Tables 3 and 4.

As shown in Figure 5, the following heat transfer coefficients were applied:

<u>Region 1</u>

The heat transfer coefficient, h, for 100% flow is $4789\left(\frac{17.364}{25}\right)^{0.8} = 3577.8$ BTU/hr-ft^2-°F at 300°F. [4]

where 17.364 ft/sec is converted from 28,294 GPM and 25.8 in ID.

The heat transfer coefficient, h, for 50% flow is $4789\left(\frac{8.682}{25}\right)^{0.8} = 2054.9$ BTU/hr-ft^2-°F at 300°F. [4]

where 8.682 ft/sec is converted from 14,147 GPM and 25.8 in ID.

The heat transfer coefficient, h, for 0% flow is 112.34 BTU/hr-ft^2-°F at 300°F. [Table 3, for natural convection]

Region 2

The heat transfer coefficient for Region 2 is linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used for Region 3.

Region 3 (the point between Region 2 and Region 4)

The heat transfer coefficient, h, for 100% flow is $4789\left(\frac{17.364}{25}\right)^{0.8}\cdot\left(\frac{26}{35.49}\right)^{0.2}=3361$ BTU/hr-ft^2-°F at 300°F. [4]

where the flow rate is the same as that for Region 1, and the ID is 35.49 in.

The heat transfer coefficient, h, for 50% flow is $4789\left(\frac{8.682}{25}\right)^{0.8}\cdot\left(\frac{26}{35.49}\right)^{0.2}=1930.9$ BTU/hr-ft^2-°F at 300°F. [4]

where the flow rate is the same as that for Region 1, and the ID is 35.49 in.

The heat transfer coefficient, h, for 0% flow is 112.34 BTU/hr-ft^2-°F at 300°F. using the same HTC as Region 1 [Table 3, for natural convection]

Region 4

Per Reference [1], the heat transfer coefficient for Region 4 (Nozzle Blend Radius) is linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h, for 100% flow is 0.5 x 3577.8 = 1788.9 BTU/hr-ft^2-°F at 300°F. [4]

The heat transfer coefficient, h, for 50% flow is 0.5 x 2054.9 = 1027.4 BTU/hr-ft^2-°F at 300°F. [4]

The heat transfer coefficient, h, for 0% flow is 101 BTU/hr-ft^2-°F at 300°F. [Table 4, for natural convection] by using 40 in. hydraulic diameter [4].

Region 6

The heat transfer coefficient, h, is 0.4 BTU/hr-ft^2-°F [4].

3.2.2 *Boundary Fluid Temperatures*

For the Green's Functions, a 500°F to 100°F thermal shock is run to determine the stress response to a one-degree change in temperature. The following temperatures are valid when there is water flow. Values between defined points are linearly interpolated. For the 100%, 50%, and 0% flow cases, the thermal shock is run as follows:

Regions 1 to 5
T = 500°F – 100°F
Region 6
T = 120°F

4.0 THERMAL AND PRESSURE LOAD RESULTS

The three flow dependent thermal load cases outlined in Section 3.0 were run on the finite element model. Appendix A contains the thermal transient input files VY_RON_T_100.INP, VY_RON_T_50.INP, and VY_RON_T_0.INP for 100%, 50%, and 0% flow rates, respectively. The three flow dependent input files for the stress runs are also included in Appendix A. The stress filenames are VY_RON_S_100.INP, VY_RON_S_50.INP, and VY_RON_S_0.INP for 100%, 50%, and 0% flow rates, respectively.

The critical safe end location was chosen as node 6395, which has the highest stress intensity due to thermal loading under high flow conditions. As shown in Figures 6 and 7, Node 6395 is located on the inside diameter of the nozzle safe end of the model and the maximum stress occurs at 5.1 seconds.

The critical blend radius location was chosen, based upon the highest pressure stress. Assumed the cladding has cracked, therefore, as shown in Figures 8 and 9, the critical location is selected as node 3829 at base metal of the nozzle.

The stress intensity for use in the Green's functions are calculated from the component stresses (X, Y, and Z) and compared to the stress intensity reported by ANSYS. As seen in Figure 10, the Y-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the safe end. Therefore, the Y-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the safe end. As seen in Figure 11, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the blend radius in very beginning. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the blend radius.

The stress time history for the critical paths was extracted during the stress run for 100% flow rate. This produced two files, HFSE.OUT and HFBR.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files HFSE_Inside.RED and HFBR_Inside.RED, where SE and BR corresponded to the safe end and blend radius locations, respectively. The total stress intensity (SI) was extracted from these files to produce the files HFSE.CLD and HFBR.CLD, where SE and BR corresponded to the safe end and the blend radius, respectively.

The stress time history for the critical paths was extracted during the stress run for 50% flow rate. This produced two files, MFSE.OUT and MFBR.OUT which contains the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from the file to produce the file MFSE_Inside.RED, where SE corresponds to the safe end location.

The stress time history for the critical paths was extracted during the stress run for 0% flow rate. This produced two files, LFSE.OUT and LFBR.OUT which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from the file to produce the file LFSE_Inside.RED, where SE corresponds to the safe end location.

The stress time history for the recirculation outlet nozzle during 100% flow, 50% flow, and 0% flow are shown in Figures 12 to 23. The data for the Green's Functions is included in the files HFBR_M+B-Green.xls, HFBR_T-Green.xls, HFSE_M+B-Green.xls, HFSE_T-Green.xls, MFBR_M+B-Green.xls, MFBR_T_Green.xls, MFSE_M+B-Green.xls, MFSE_T-Green.xls, LFBR_M+B-Green.xls, LFBR_T-Green.xls, LFSE_M+B-Green.xls, and LFSE_T-Green.xls in the project Files. Where HF, MF, and LF corresponded to 100% flow, 50% flow, and 0% flow rate, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

The pressure stress intensities for the path were extracted during the pressure run. The pressure stresses were extracted along the nodal path as shown in Figures 7 and 9. This produced two files, PSE.OUT and PBR.OUT for the safe end and blend radius locations, respectively.

For the pressure loading specified (1000 psig), the total stress intensities at Node 6395 and Node 3829 were determined to be 11490 psi and 31300 psi, respectively. The membrane plus bending stress intensities at Node 6395 and Node 3829 were determined to be 11350 psi and 33640 psi, respectively. Table 2 shows the final pressure results.

Results were also extracted from the vessel portion of the model to verify the accuracy of the results obtained from the ANSYS model, and to check the results due to the use of the 2.0 multiplier on the vessel radius. These results are contained in the file PVESS.OUT. The radius of the finite element model (FEM) was multiplied by a factor of 2.0 [1] to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere but the true geometry is the intersection of two cylinders.

The equation for the membrane hoop stress for a sphere is:

$$\sigma = \left(\frac{(pressure) \times (radius)}{2 \times thickness} \right)$$

Considering a vessel base metal radius, R, of 105.906 inches increased by a factor of 2.0, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is PR/(2t) = 19,477 psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, PVESS.OUT, of 19,540 psi. Thus, considering the peak total pressure stress of 31,300 psi reported above, the stress concentrating effect of the nozzle corner is 31,300/19,477 = 1.61. In other words, the peak nozzle corner stress is 1.61 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \left(\frac{(pressure) \times (radius)}{thickness} \right).$$

Based on the previous dimensions, the calculated stress for a cylinder without the 2.0 factor is 19,477 psi. Increasing this by a factor of 1.61 yields an expected peak nozzle corner stress of 31,358 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (31,300 psi) is close to the peak nozzle corner stress for a cylindrical geometry because of the use of the 2.0 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.

5.0 REFERENCES

1. SI Calculation No. VY-16Q-304, Revision 0, "Recirculation Outlet Nozzle Finite Element Model"
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. CB&I, RPV Stress Report Section: T9 "Thermal Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
5. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
6. J. P. Holman, "Heat Transfer," 5th Edition, 1981.
7. Entergy Nuclear Northeast Engineering Report, Report No. VY-RPT-05-00022, "Task T0100 Reactor Heat Balance EPU Task Report for ER-04-1409," SI File No. VY-16Q-205.

Table 1: Material Properties @ 300°F[1]

Material	SA-533 Gr B (Mn-1/2Mo-1/2Ni)	SA-508 Cl 2 (3/4Ni-1/2Mo-1/3Cr-V)	SA-240 Type 304 (18Cr-8Ni)	SA-182 F316/ SA 376 TP316 (16Cr-12Ni-2Mo)
Modulus of Elasticity, e^{-6} psi	28.0	26.7	27.0	27.0
Coefficient of Thermal Expansion, e^{-6}, in/in/°F	7.7	7.3	9.8	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	9.3
Thermal Diffusivity, ft^2/hr	0.401	0.401	0.160	0.150
Calculated Specific Heat, Btu/lb-°F [2]	0.119	0.119	0.125	0.127
Density, lb/in³	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes: [1]The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values.
[2] Calculated as $[k/(\rho d)]/12^3$.

Table 2: Pressure Results

Location	Membrane Plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	11350	11490
Blend Radius	33640	31300

Table 3: 0% Flow Regions 1 and 3 Heat Transfer Coefficients

Pipe Inside Diameter, D =	25.800	inches =	2.150	ft					
		=	0.655	m					
Outer Pipe, Inside radius, r_o =	12.9	inches =	1.075	ft					
			0.328	m					
Inner Pipe Outside Diameter, D =	n/a	inches =	0.000	ft					
		=	0.000	m				0.000	
Inner Pipe, Outside radius, r_i =	0	inches =	0.000	ft					
			0.000	m					
Fluid Velocity, V =	17.364	ft/sec =	28293.595	gpm=		12.3 Mlb/hr			
Characteristic Length, L = D =	2.150	ft =	0.655	m					
(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F	
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C	

		Value at Fluid Temperature, T [3]							Units
	Conversion	70	100	200	300	400	500	600	°F
Water Property	Factor [1]	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m^3
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft^3
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m^3/m^3-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft^3/ft^3-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s^2
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s^2
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	3473691	5061789	10891437	16454670	21515912	26132199	27337904	—
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	2441754517	1.2697E+10	2.417E+11	1.252E+12	3.977E+12	1.034E+13	2.16049E+13	—
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	3.05E+08	1.59E+09	3.02E+10	1.57E+11	4.97E+11	1.29E+12	2.70E+12	—
Rayleigh Number, Ra	GrPr	17043446531	5.7265E+10	4.616E+11	1.528E+12	3.778E+12	8.883E+12	2.31172E+13	—
Rayleigh Number, Ra	$Gr_\delta Pr$	2.13E+09	7.16E+09	5.77E+10	1.91E+11	4.72E+11	1.11E+12	2.89E+12	—
From [1]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced}	$0.023Re^{0.8}Pr^{0.4}k/D$	7,823.02	9,326.34	13,148.12	15,405.24	16,705.40	17,126.15	16,275.32	W/m^2-°C
		1,377.74	1,642.50	2,315.56	2,713.07	2,942.05	3,016.15	2,866.31	$Btu/hr-ft^2$-°F
From [1]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25			
H_{free}	$C(GrPr)^n k/L$	181.85	258.65	469.34	637.89	773.57	875.17	933.22	W/m^2-°C
		32.03	45.55	82.66	112.34	136.24	154.13	164.35	$Btu/hr-ft^2$-°F

Table 4: 0% Flow Region 5 Heat Transfer Coefficient

Heat Transfer Coefficients

References: 1. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
2. J. P. Holman, "Heat Transfer," 5th Edition, 1981.
3. N. P. Cheremisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co., 1984.

(Required Inputs are Shaded!)

Title =	Piping						
Pipe Inside Diameter, D =	40.000	inches =	3.333	ft			
		=	1.016	m			
Outer Pipe, Inside radius, r_o =	20	inches =	1.667	ft			
			0.508	m			
Inner Pipe Outside Diameter, D =	n/a	inches =	0.000	ft			
		=	0.000	m		0.000	
Inner Pipe, Outside radius, r_i =	0	inches =	0.000	ft			
			0.000	m			
Fluid Velocity, V =	7.224	ft/sec =	26293.595	gpm=	12.3 Mlb/hr		
Characteristic Length, L = D =	3.333	ft =	1.016	m			

(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [3]							Units
	Conversion	70	100	200	300	400	500	600	°F
Water Property	Factor [1]	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m^3
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft^3
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m^3/m^3-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft^3/ft^3-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s^2
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s^2
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	°F
Reynold's Number, Re	$\rho VD/\mu$	2240531	3264854	7024977	10613262	13877763	16855268	17632948	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	9099611606	4.732E+10	9.01E+11	4.667E+12	1.48E+13	3.85E+13	8.05143E+13	---
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	1.14E+09	5.91E+09	1.13E+11	5.83E+11	1.85E+12	4.82E+12	1.01E+13	---
Rayleigh Number, Ra	GrPr	6.3515E+10	2.134E+11	1.72E+12	5.694E+12	1.41E+13	3.31E+13	8.61503E+13	---
Rayleigh Number, Ra	$Gr_\delta Pr$	7.94E+09	2.67E+10	2.15E+11	7.12E+11	1.76E+12	4.14E+12	1.08E+13	---
From [1]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced}	$0.023Re^{0.8}Pr^{0.4}k/D$	3,552.89	4,235.64	5,971.33	6,996.42	7,586.90	7,777.99	7,391.58	W/m^2-°C
		625.71	745.95	1,051.63	1,232.17	1,336.16	1,369.81	1,301.76	$Btu/hr\text{-}ft^2$-°F
From [1]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25			
H_{free}	$C(GrPr)^n k/L$	162.97	231.79	420.60	571.66	693.25	784.30	836.32	W/m^2-°C
		28.70	40.82	74.07	100.68	122.09	138.13	147.29	$Btu/hr\text{-}ft^2$-°F

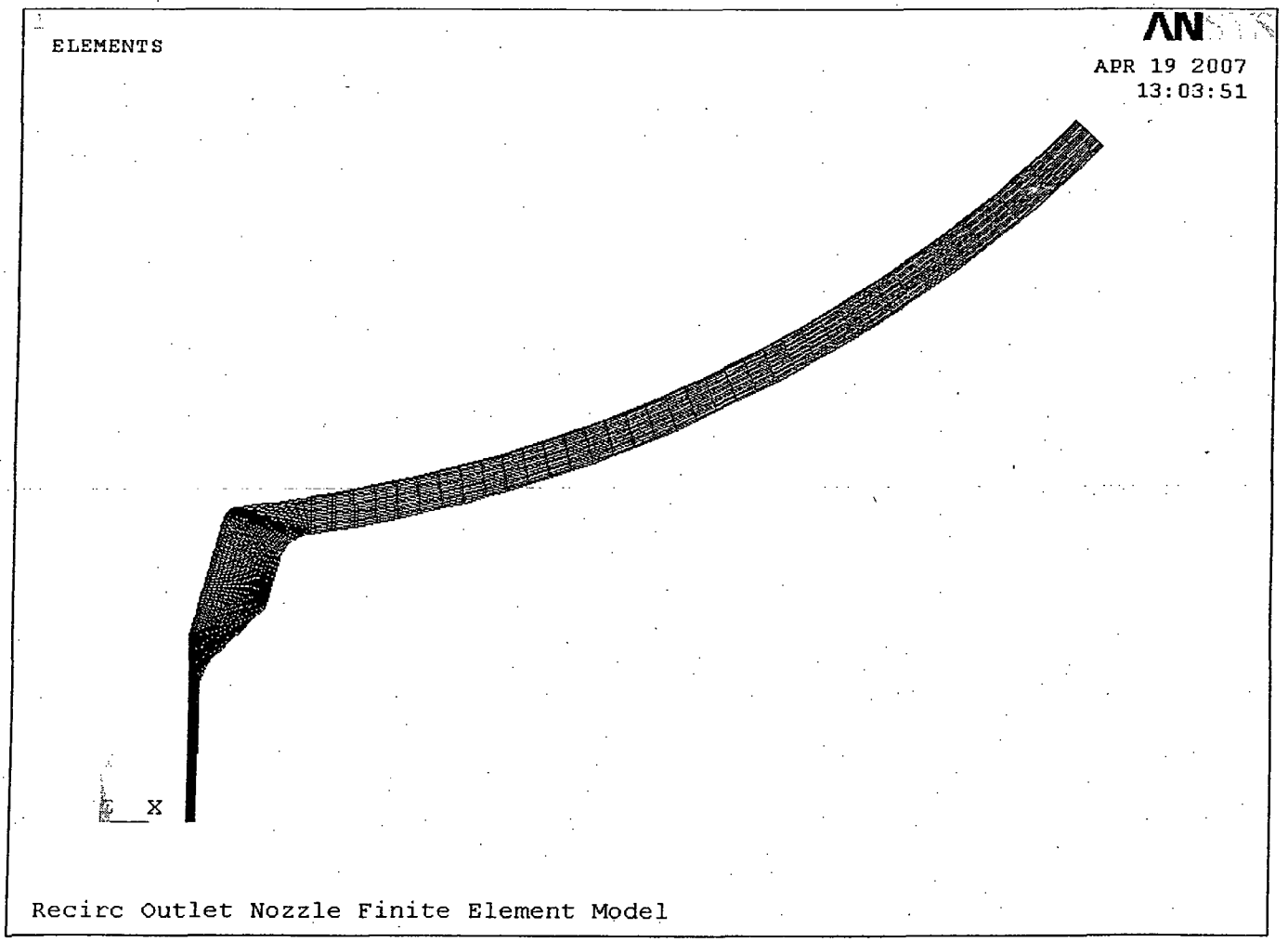

Figure 1: ANSYS Finite Element Model

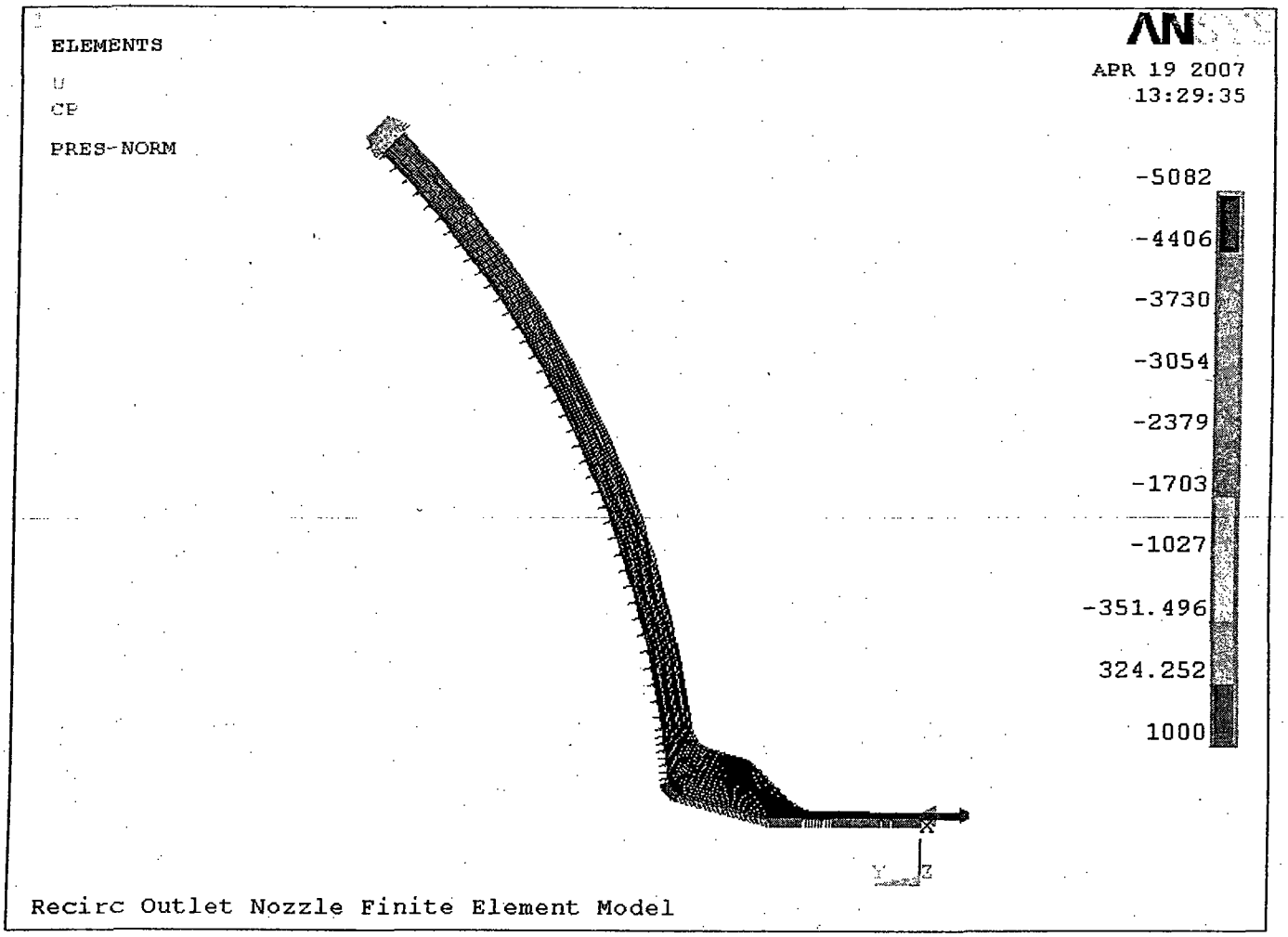

Figure 2: Recirculation Outlet Nozzle Internal Pressure Distribution

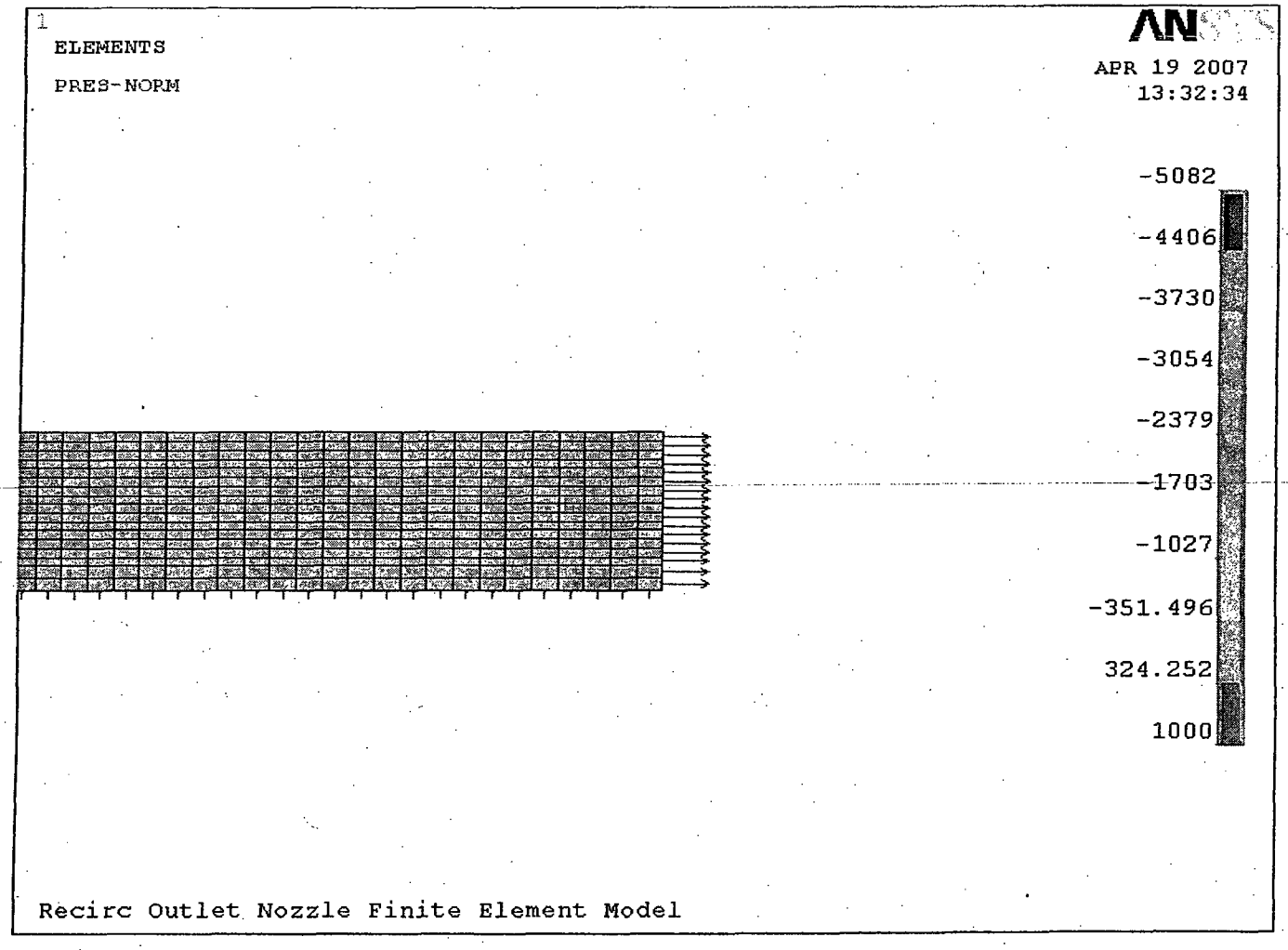

Figure 3: Recirculation Outlet Nozzle Pressure Cap Load

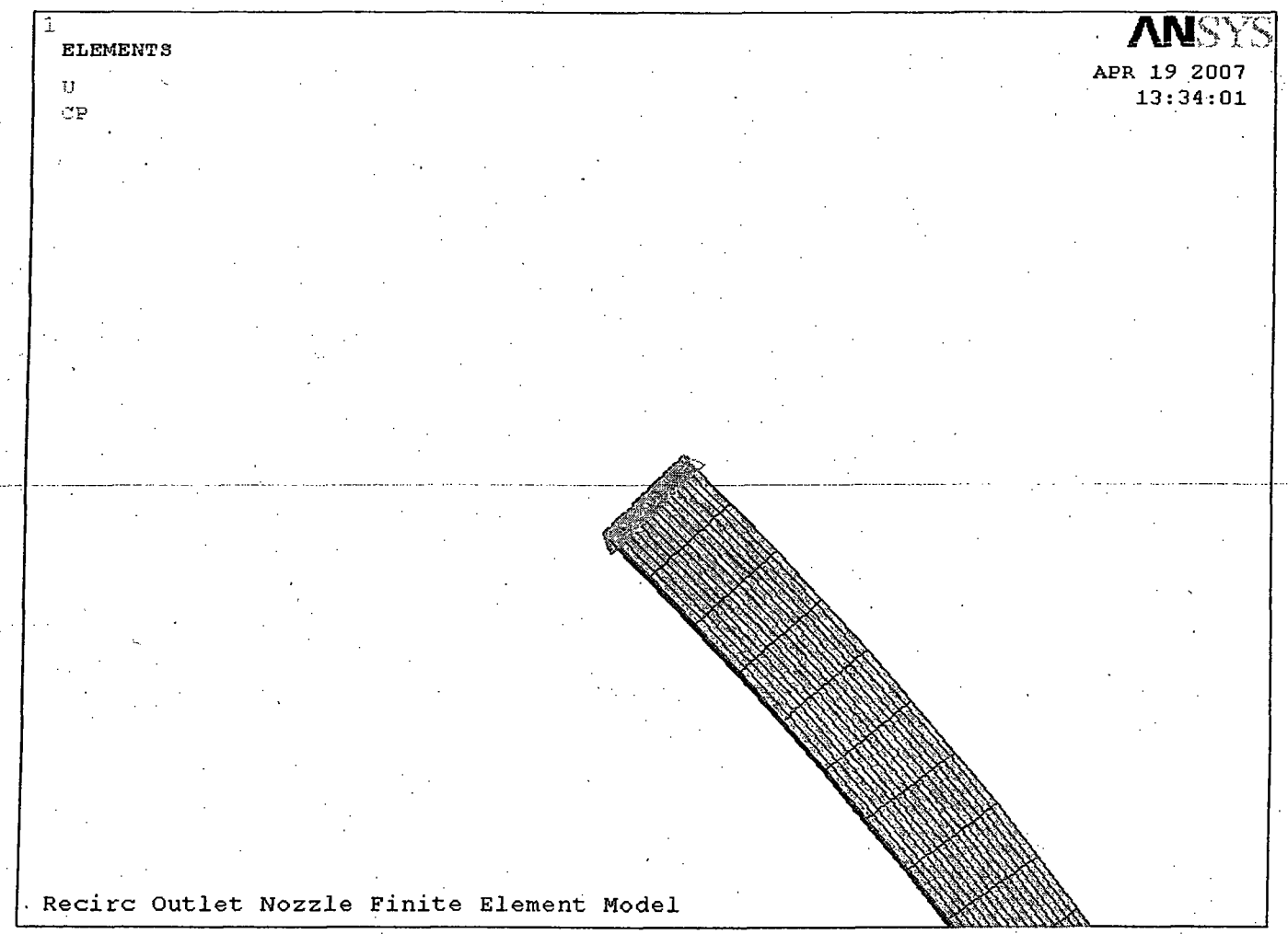

Figure 4: Recirculation Outlet Nozzle Vessel Boundary Conditions

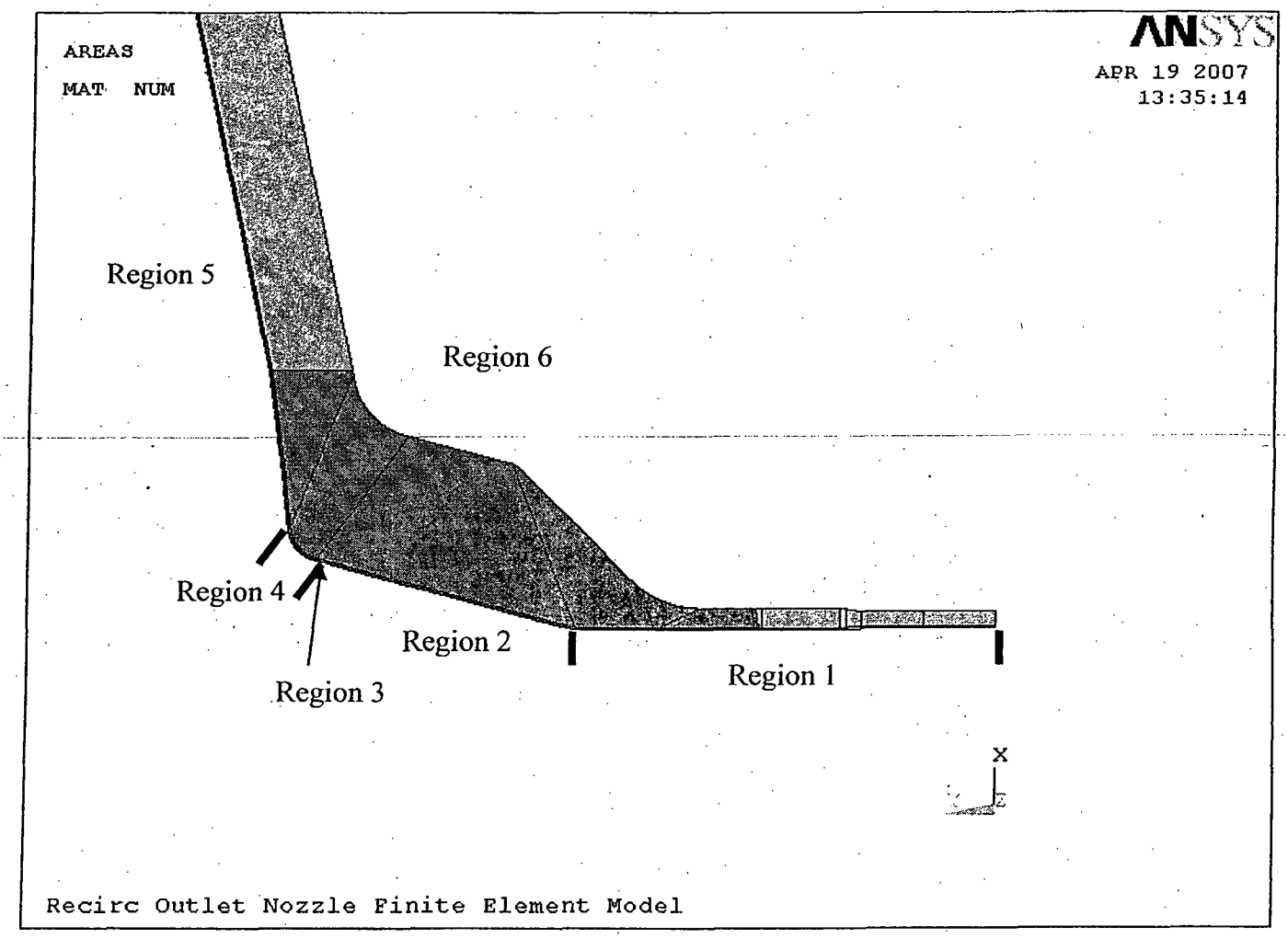

Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries

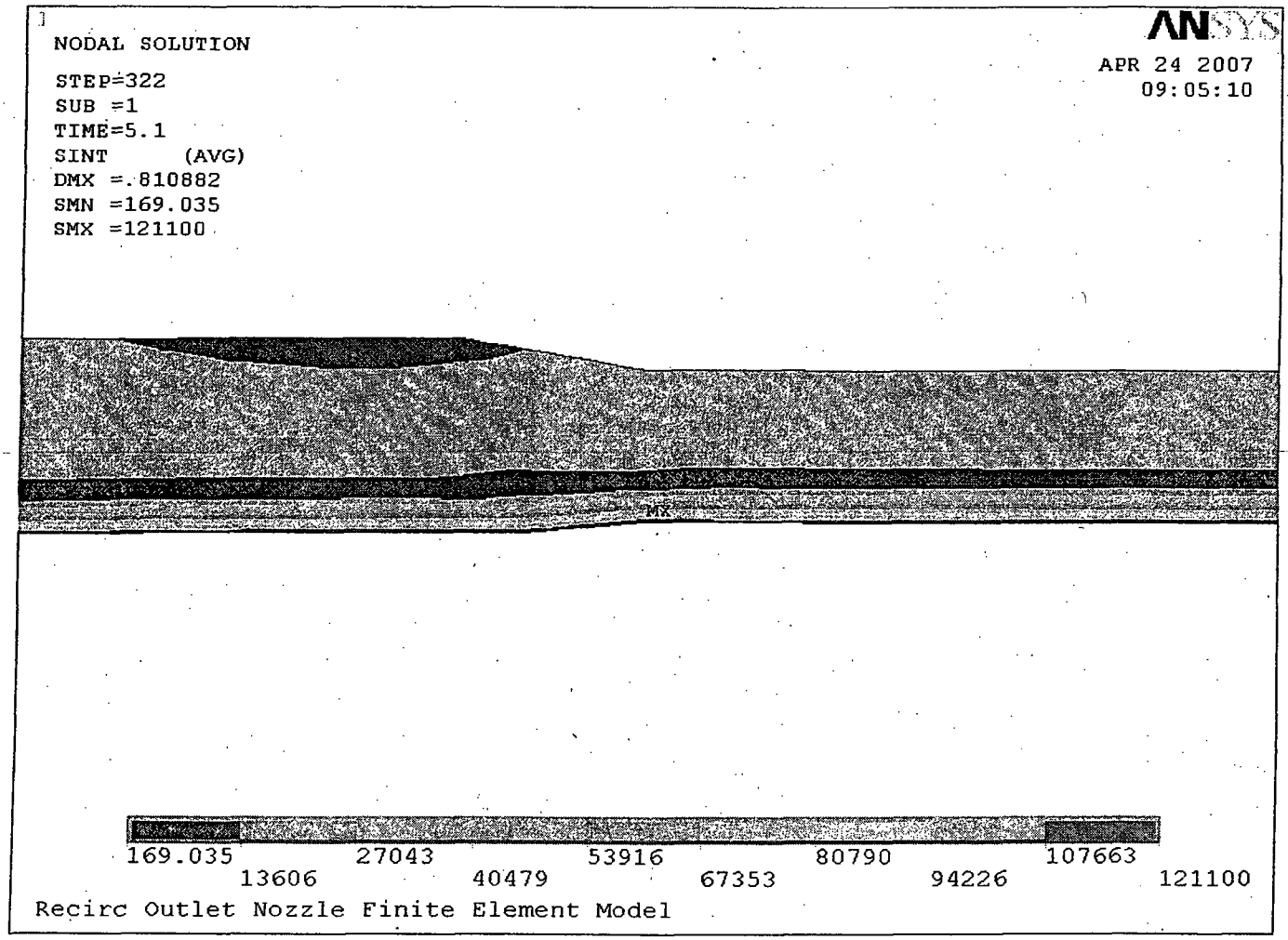

Figure 6: Safe End Critical Thermal Stress Location

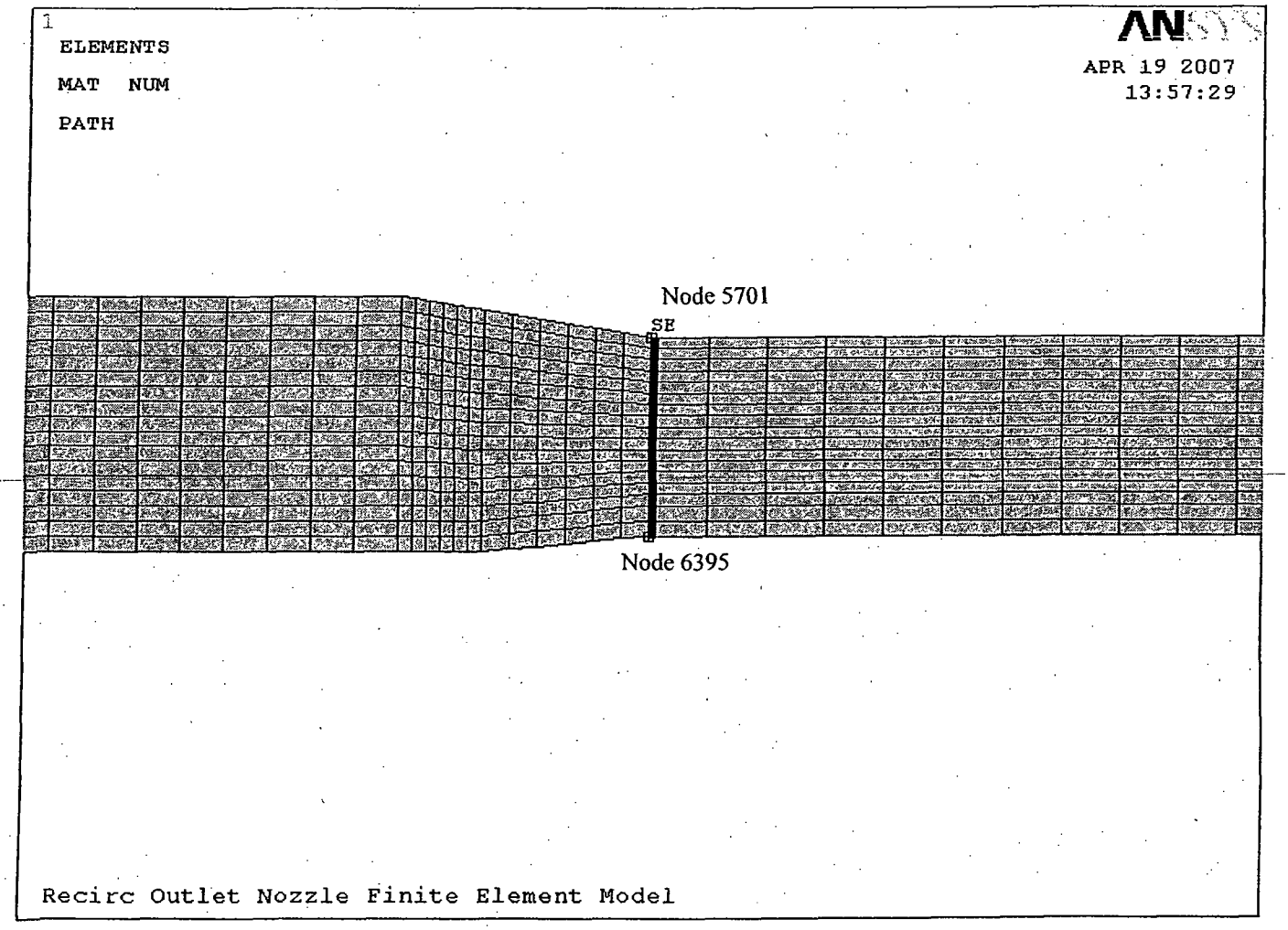

Figure 7: Safe End Limiting Linearized Stress Paths

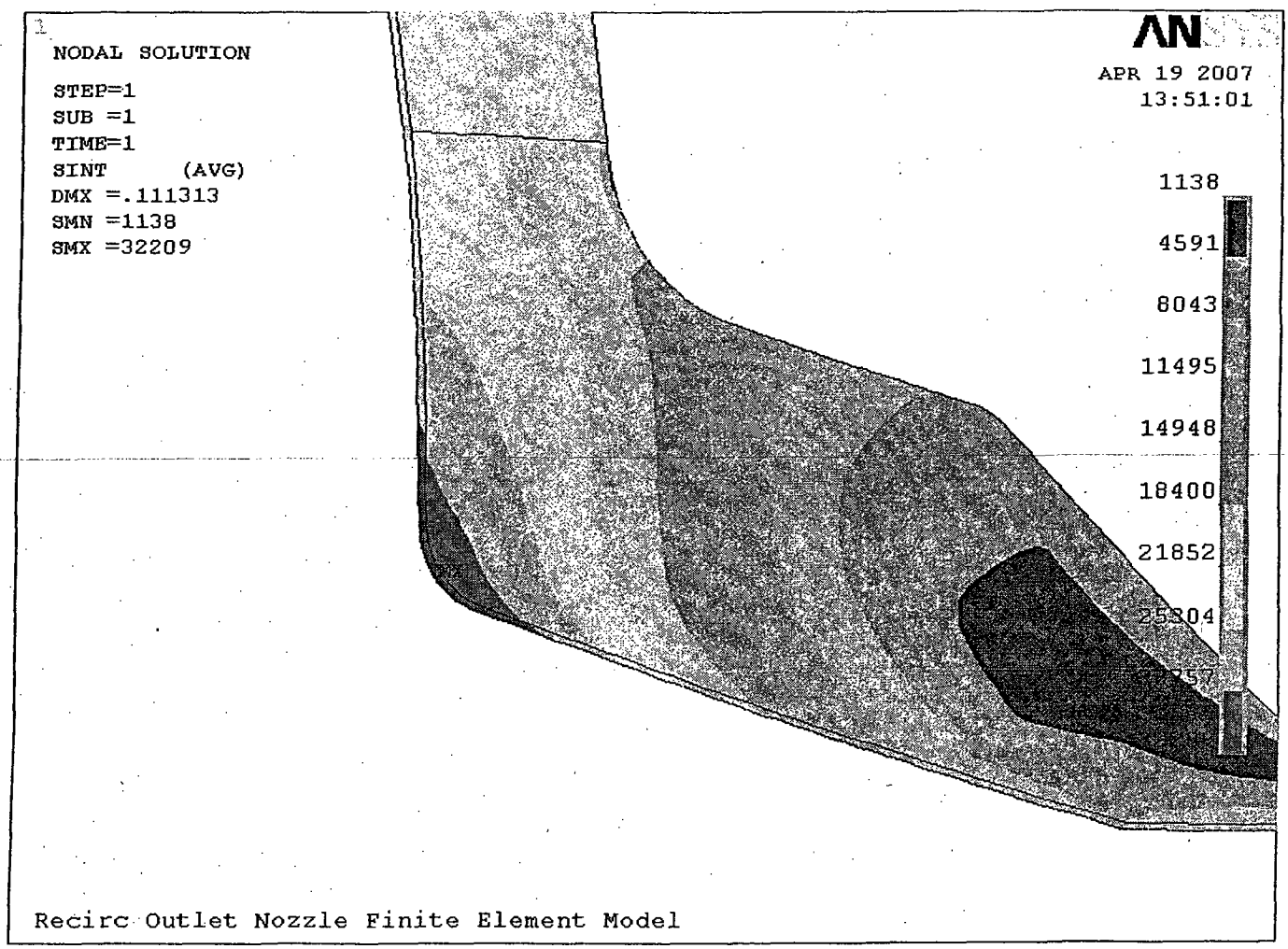

Figure 8: Blend Radius Limiting Pressure Stress Location

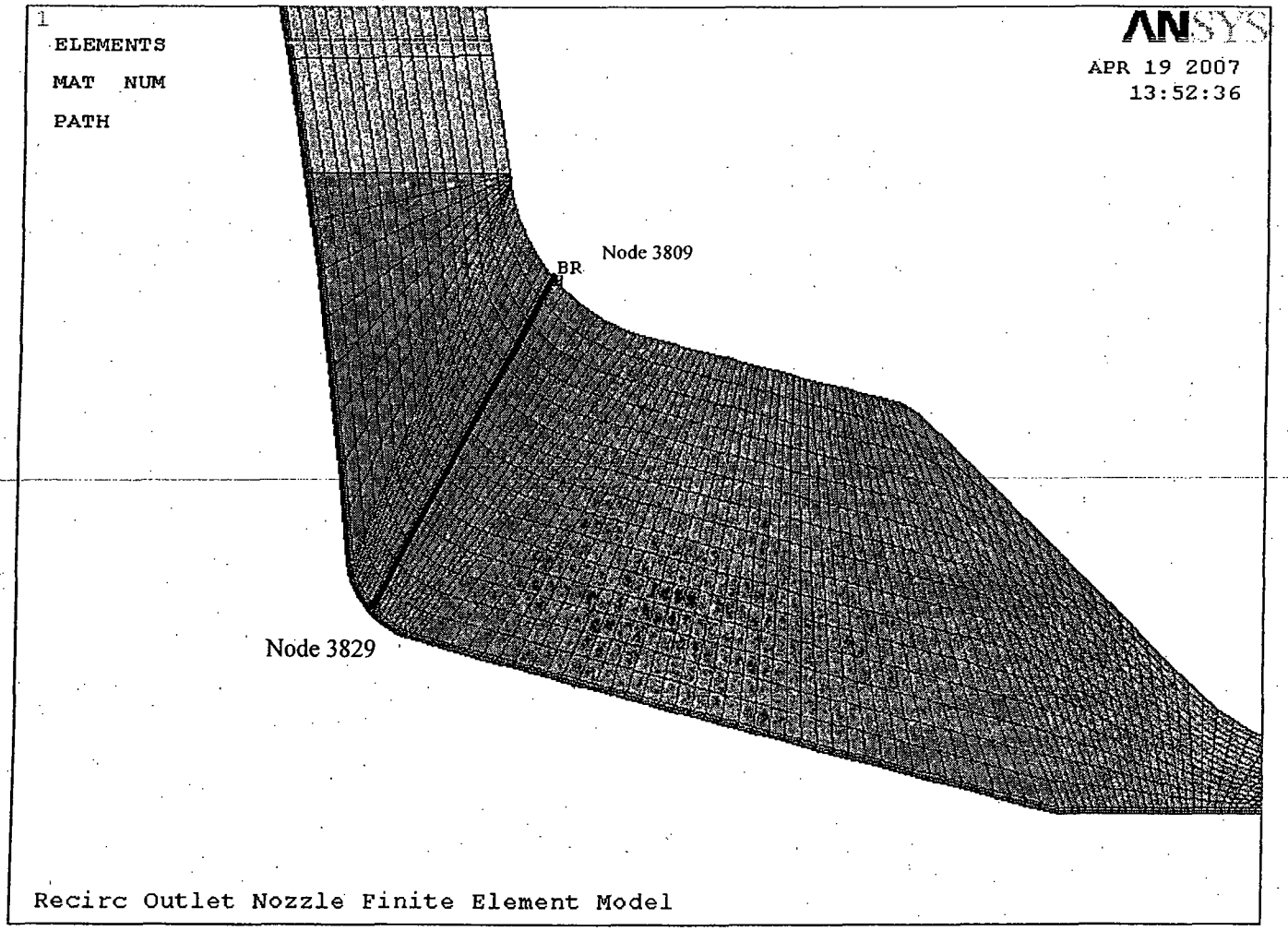

Figure 9: Blend Radius Linearized Stress Path

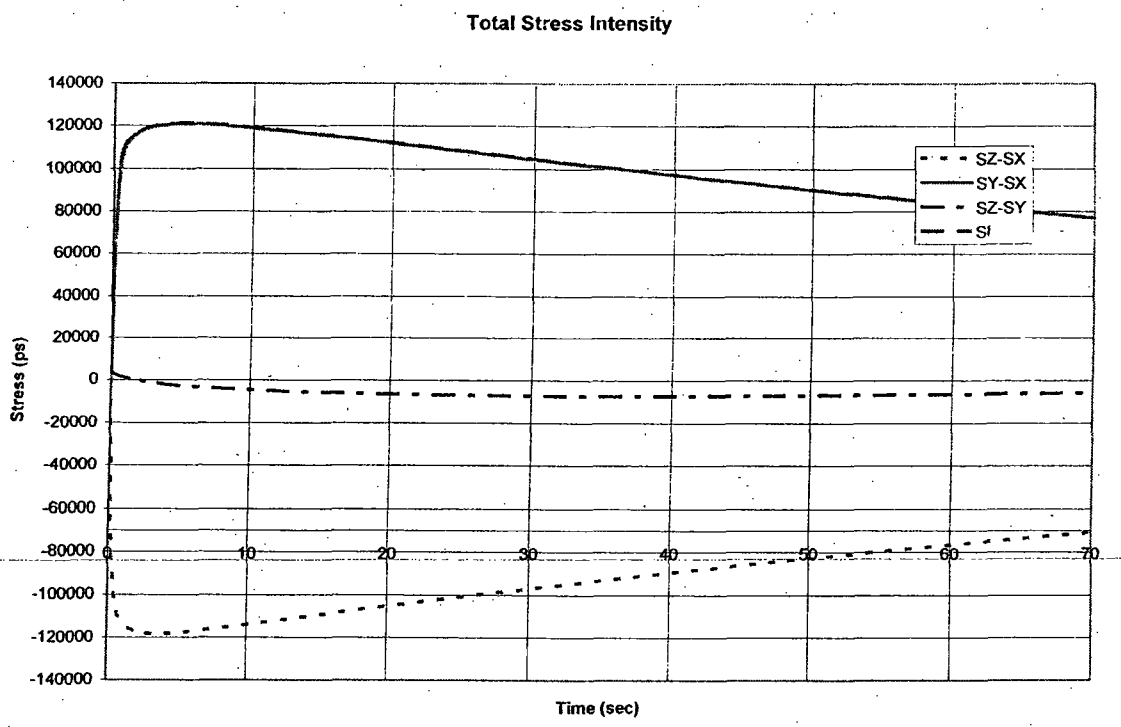

Figure 10: Safe End 100% Flow Total Stress Intensity

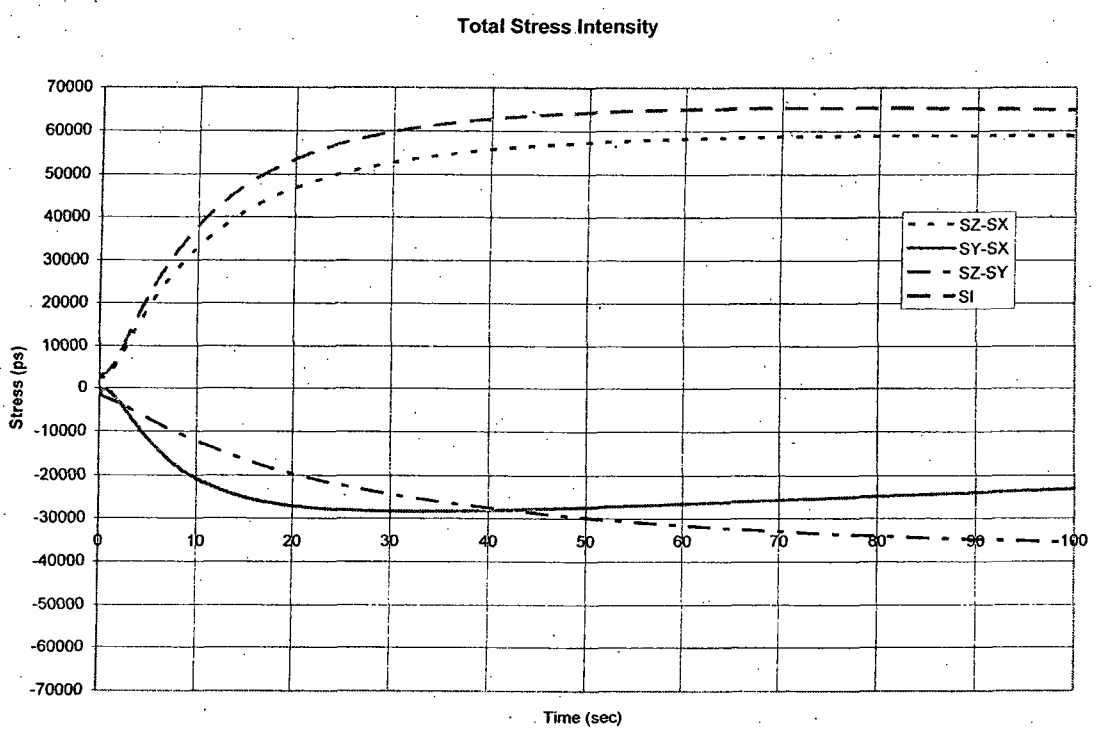

Figure 11: Blend Radius 100% Flow Total Stress Intensity

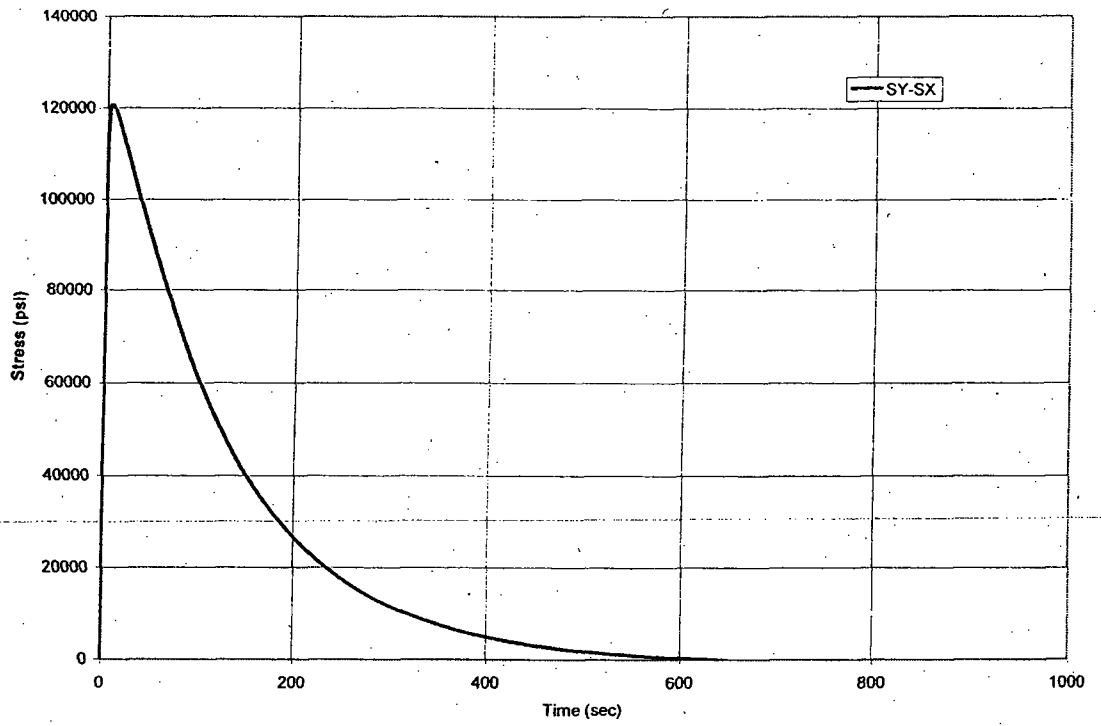

Figure 12: Safe End Total Stress History for 100% Flow

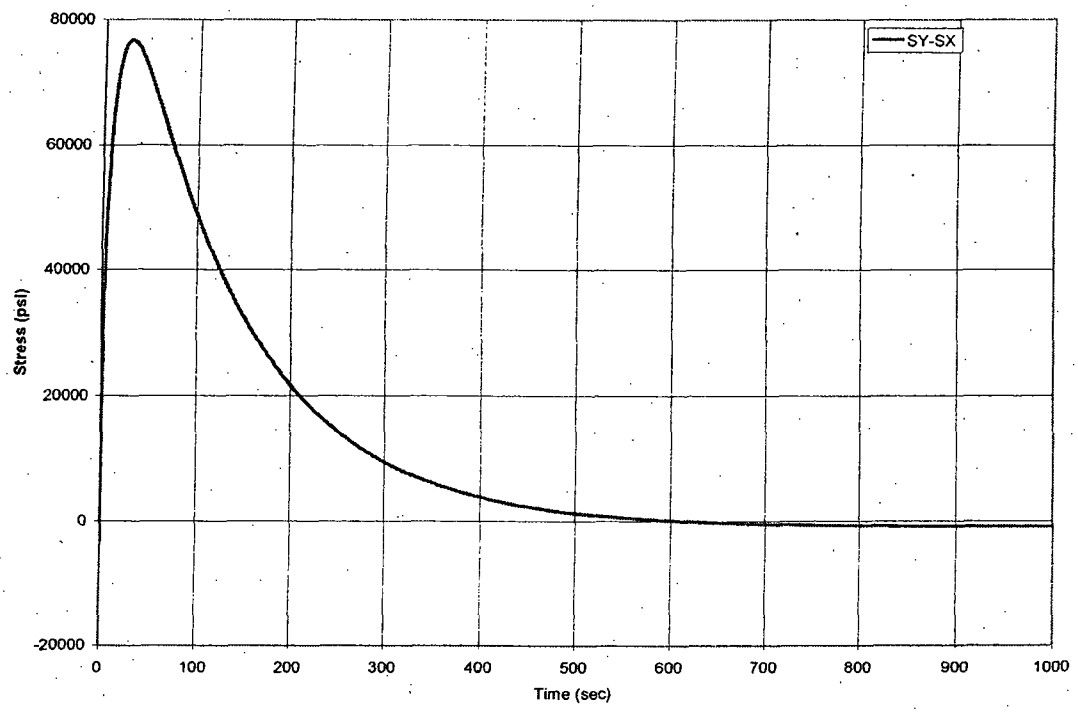

Figure 13: Safe End Membrane Plus Bending Stress History for 100% Flow

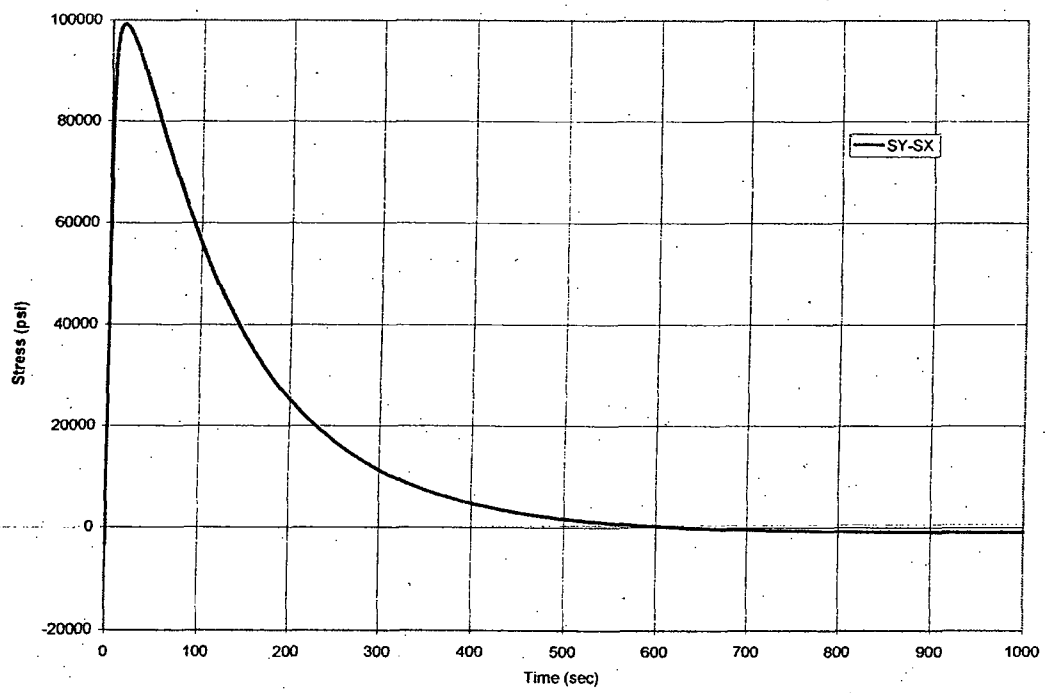

Figure 14: Safe End Total Stress History for 50% Flow

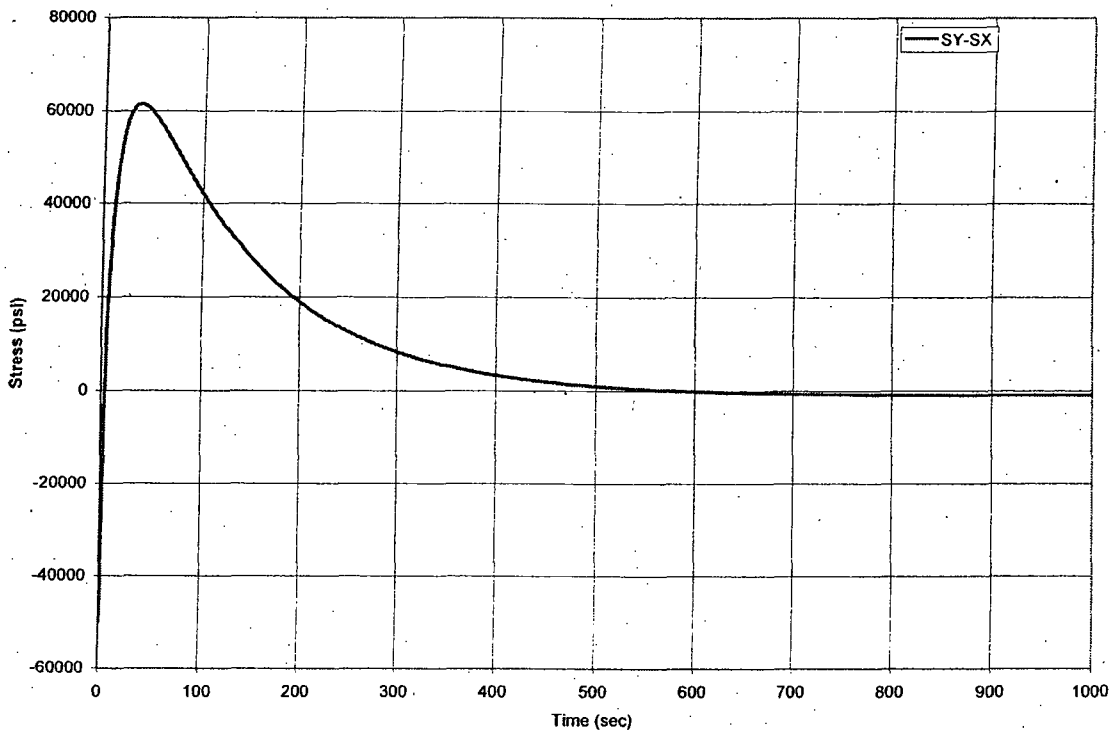

Figure 15: Safe End Membrane Plus Bending Stress History for 50% Flow

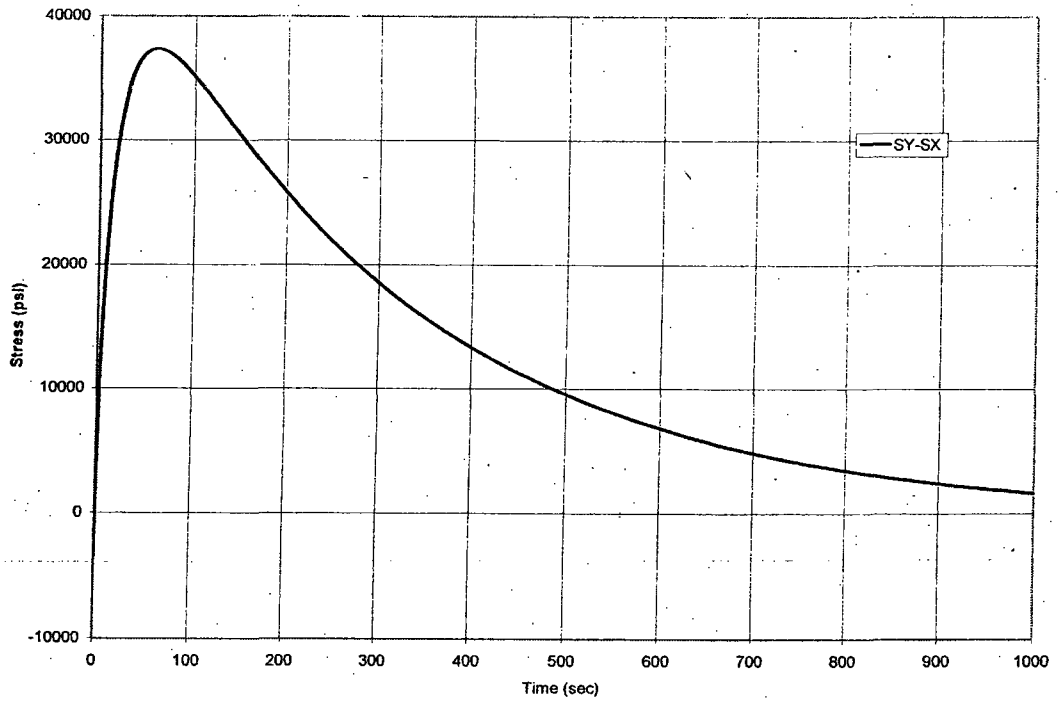

Figure 16: Safe End Total Stress History for 0% Flow

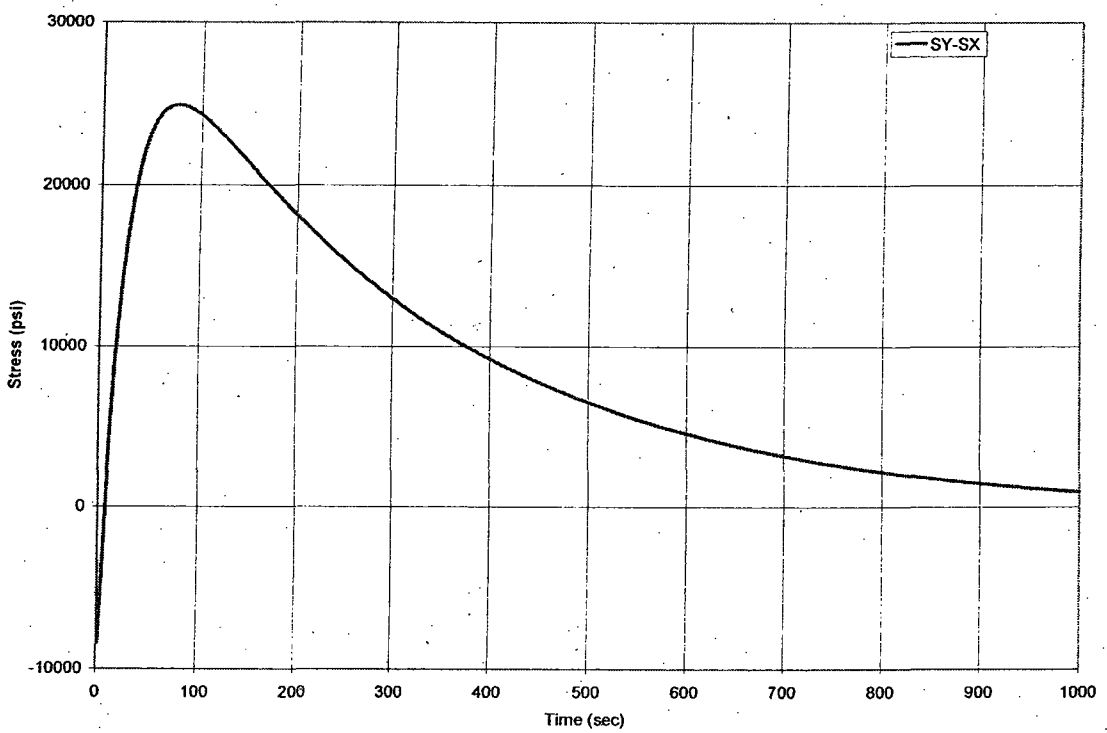

Figure 17: Safe End Membrane Plus Bending Stress History for 0% Flow

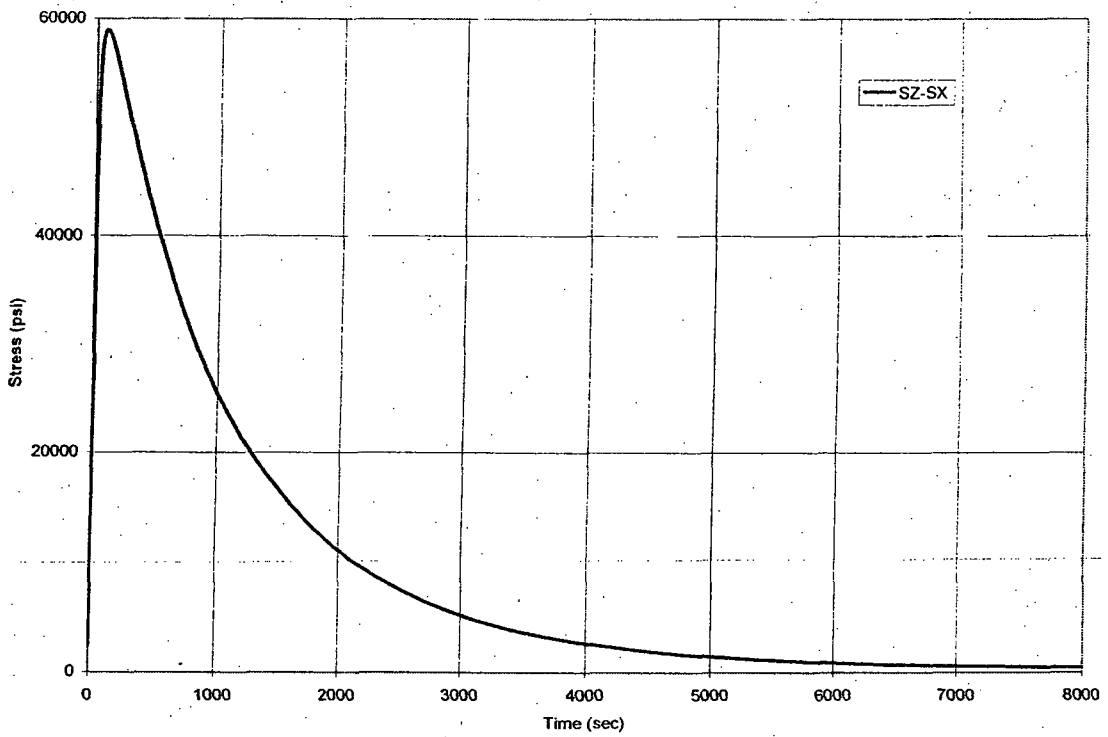

Figure 18: Blend Radius Total Stress History for 100% Flow

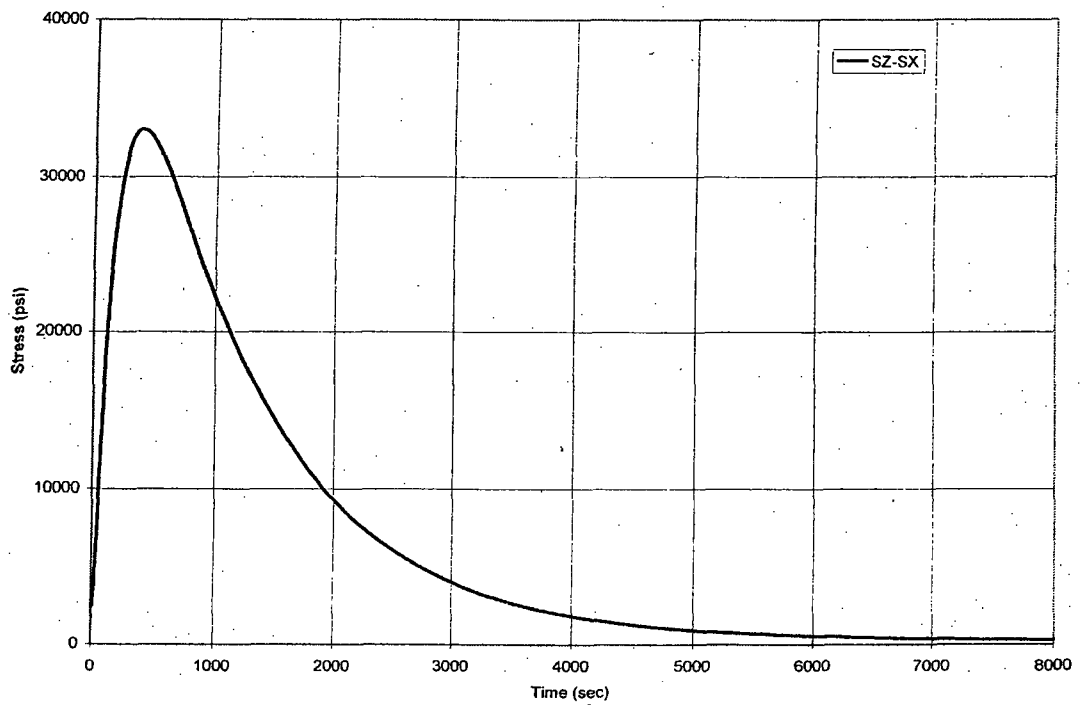

Figure 19: Blend Radius Membrane Plus Bending Stress History for 100% Flow

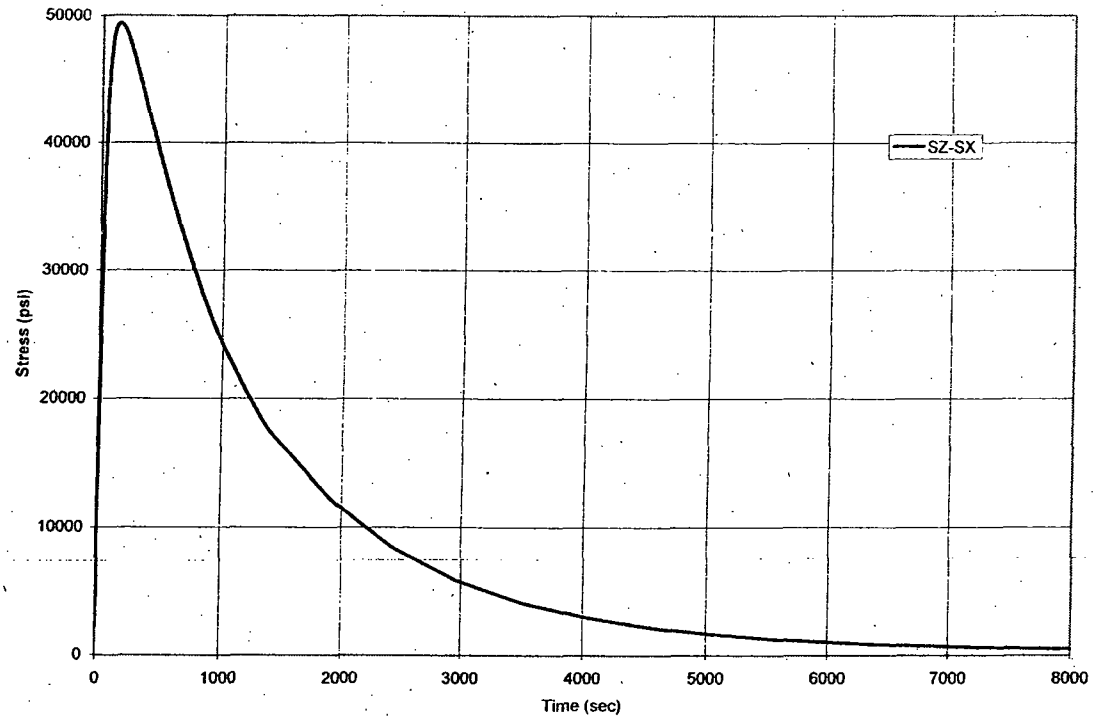

Figure 20: Blend Radius Total Stress History for 50% Flow

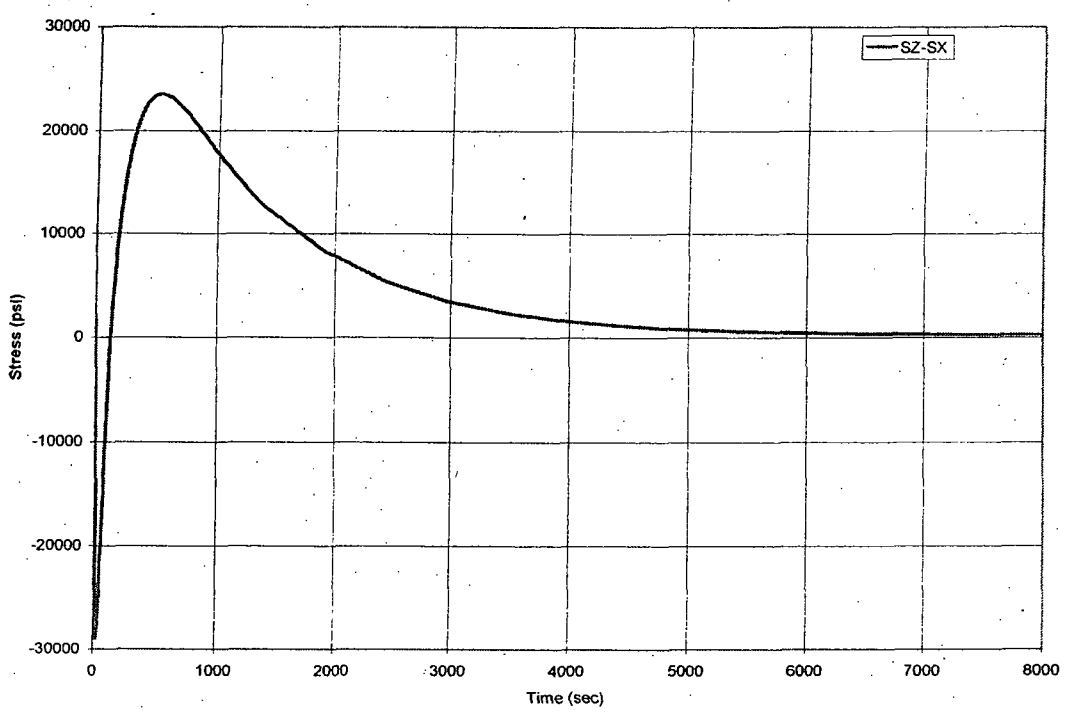

Figure 21: Blend Radius Membrane Plus Bending Stress History for 50% Flow

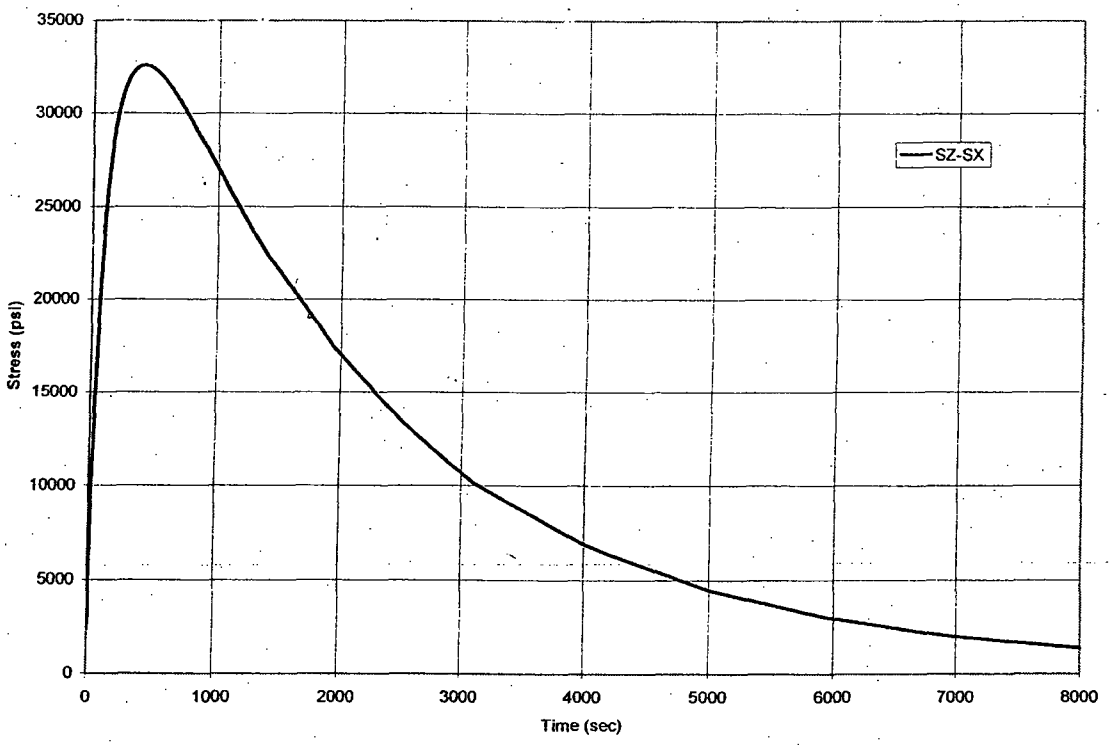

Figure 22: Blend Radius Total Stress History for 0% Flow

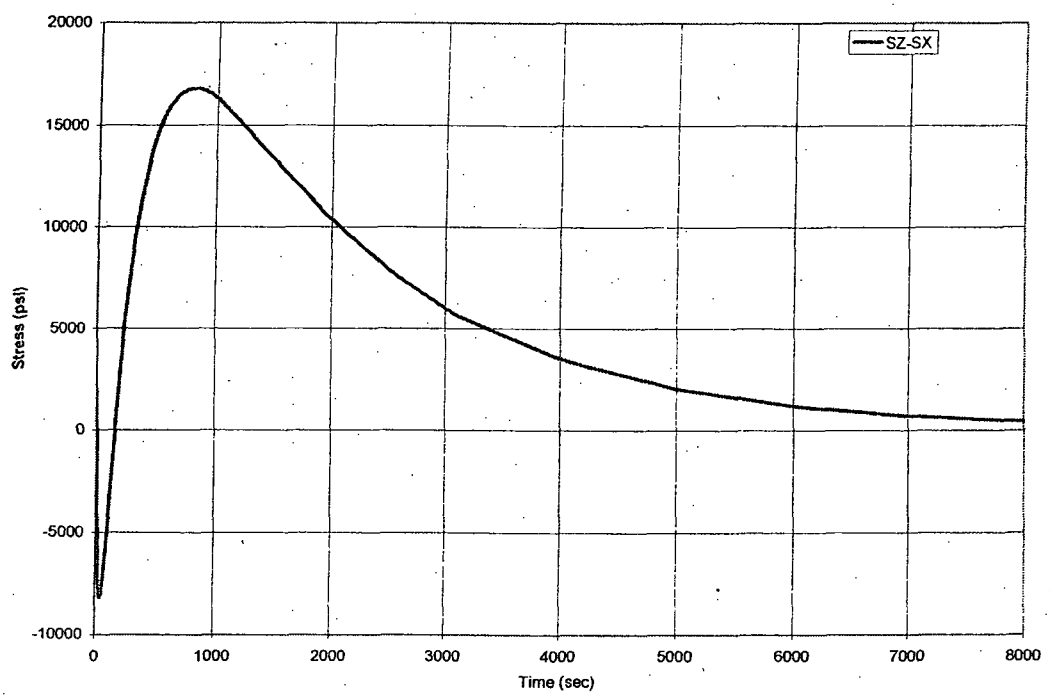

Figure 23: Blend Radius Membrane Plus Bending Stress History for 0% Flow

APPENDIX A

FINITE ELEMENT ANALYSIS FILES

RON_VY.INP	Input File for Pressure Load	In Computer files
VY_RON_T_100.INP	Input File for 100% Flow Thermal Analysis	In Computer files
VY_RON_S_100.INP	Input File for 100% Flow Stress Analysis	In Computer files
VY_RON_T_50.INP	Input File for 50% Flow Thermal Analysis	In Computer files
VY_RON_T_50.INP	Input File for 50% Flow Stress Analysis	In Computer files
VY_RON_0.INP	Input File for 0% Flow Thermal Analysis	In Computer files
VY_RON_0.INP	Input File for 0% Flow Stress Analysis	In Computer files
PVESS.OUT	Stress Output across the shell with Pressure Load	In Computer files
PSE.OUT	Stress Output at Safe End with Pressure Load	In Computer files
PBLEND.OUT	Stress Output at Blend Radius with Pressure Load	In Computer files
#FSE.OUT	Stress Output at Safe End	In Computer files
#FBR.OUT	Stress Output at Blend Radius	In Computer files
#FSE_INSIDE.RED	Stress Extracted at Safe End	In Computer files
#FBR_INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
#FSE_T-Green.XLS	Green Function with Total Stress at Safe End	In Computer files
#FSE_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Safe End	In Computer files
HFBR_T-Green.XLS	Green Function with Total Stress at Blend Radius at 100% flow	In Computer files
HFBR_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Blend Radius at 100% flow	In Computer files

Where # is H, M, L meaning 100%, 50%, and 0% flow rate, respectively.

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CALCULATION PACKAGE	**Project No.:** VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

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CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Recirculation Outlet Nozzle

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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the Entergy Vermont Yankee (VY) reactor pressure vessel (RPV) recirculation outlet nozzle. Two locations will be analyzed for fatigue acceptance: the safe end (SA182 F316) and the nozzle inner corner blend radius (SA508 Class 2). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. Fatigue usage will be determined for each location, the nozzle forging and safe end, respectively. An environmental fatigue usage factor will also be determined for each of these locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the recirculation outlet nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the recirculation outlet nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [14] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [13]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [17] methodology was originally used to evaluate the recirculation outlet nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original recirculation outlet nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more accurate CUF.

For the recirculation outlet nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data.

This Green's Function integration scheme is similar in concept to the Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [15]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 13 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 14. The input transient temperature history contains five step-changes of varying size, as shown in Figure 14. These five step changes produce the five successive stress responses in the second plot shown in Figure 14. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor recirculation outlet nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for these three Recirculation flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility computer programs are used to facilitate the fatigue analysis process: STRESS.EXE, P V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [5] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.
The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m, Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.

The 10 transients to be analyzed are described in Reference [2], for the recirculation outlet nozzle. Transients 11 and 12 are hydrostatic tests that have only a small temperature change and are not

modeled. Transients 1 to 10 are shown in Figures 3 – 12. The analysis of transient 9 is an exception to this process because there are two different thermal shocks at the nozzle and vessel regions. Transient 9 is analyzed separately using ANSYS instead of STRESS.EXE and P-V.EXE. The results from ANSYS are input directly into FATIGUE.EXE with the other transient stress results.

3.0 ANALYSIS

The fatigue analysis involves preparing the input files and running the three programs. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P-V.EXE. The ANSYS results from transient 9 are added to the P-V.EXE results for the other transients and input into FATIGUE.EXE.

The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- GREEN.DAT. There are 12 stress history functions (Green's Functions) obtained from Reference [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. The blend radius and the safe end have three stress history functions for the 100% flow, 50%, and no-flow conditions.
- GREEN.CFG is configured as described in Reference [5].
- Several TRANSNT.INP files are created to simulate the transients shown on Reference [2]. Tables 2 and 3 show the thermal history used to simulate each transient for the blend radius and safe end locations, respectively. The aforementioned transient information for each location is contained in EXCEL files *Blend_Radius_Transients.xls* and *Safe_End_Transients.xls,* which are contained in the computer files. Transients are split into the following groups based upon flow rate:
    - Transients 2, 3, 5, 6, 7, and 8 are run at 100% flow Green's Function
    - Transients 1 and 10 are run at 50% flow Green's Function
    - Transient 4 is run at no flow, 50% flow, and 100% flow Green's Functions, as shown in Tables 2 and 3.
    - Transient 9 is simulated by ANSYS [11] model and the thermal results are taken from ANSYS directly. See Section 4 for details.
    - Transients 11 and 12 have only small temperature change (70°F to 100°F). Therefore, the thermal stresses for these two transient are ignored. Only the piping load and the pressure load are considered in these two transients.
    - The loss of feedwater heaters (Feedwater Heater Bypass) event has a negligible temperature change (526 °F to 516 °F) associated with it. Therefore this transient is ignored.

3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

After STRESS.EXE runs are completed, the program P-V.EXE is run to extract only the peaks and valleys from the STRESS.OUT stress history file produced by the STRESS.EXE program. The only input required for this program is the stress history file (STRESS.OUT), and the program outputs all of the resulting peaks and valleys to output file P-V.OUT. The resulting peak and valley stress summaries for all transients are summarized in Tables 4 and 5 for both locations. Columns 2 through 5 of Tables 4 (for the blend radius) and 5 (for the safe end) show the final peak and valley output. These final peaks and valleys were selected from the total stress and membrane plus bending stress intensities that were calculated by STRESS.EXE and screened with P-V.EXE.

3.3 Pressure Load

The pressure stress associated with a 1,000 psi internal pressure was determined in Reference [1]. These values are as follows:

Pressure stress for the safe end:

- 11,350 psi membrane plus bending linearized stress intensity.
- 11,490 psi total stress intensity.

Pressure stress for the blend radius:

- 33,640 psi membrane plus bending linearized stress intensity.
- 31,300 psi total stress intensity.

The pressure stress intensity values for each transient were linearly scaled based on the pressure. The actual pressure for column 6 of Tables 4 and 5 is obtained from Tables 2 and 3, respectively. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4 and 5.

The pressure stress is combined with the peak and valley points to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 1.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

For Cut I:

$$(M_x)_1 = M_x - F_y L_1$$
$$(M_y)_1 = M_y + F_x L_1$$

For Cut II:
$$(M_x)_2 = M_x - F_y L_2$$
$$(M_y)_2 = M_y + F_x L_2$$

The total bending moment and shear loads are obtained using the equations below:

For Cut I:
$$M_{xy} = \sqrt{(M_x)_1^2 + (M_y)_1^2}$$
$$F_{xy} = \sqrt{(F_x)_1^2 + (F_y)_1^2}$$

For Cut II:
$$M_{xy} = \sqrt{(M_x)_2^2 + (M_y)_2^2}$$
$$F_{xy} = \sqrt{(F_x)_2^2 + (F_y)_2^2}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$N_z = \frac{1}{\pi R_N}\left[\frac{1}{2}F_z + \frac{M_{xy}}{R_N}\right]$$
$$q_N = \frac{1}{\pi R_N}\left[F_{xy} - \frac{M_z}{2R_N}\right]$$

To determine the primary stresses, P_M, due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$
$$(P_M)_\theta = \frac{Pa_N}{t_N}$$
$$(P_M)_R = -P$$
$$\tau_M = \frac{q_N}{t_N}$$
$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^2}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^2}$$

Because pressure was considered separately in this analysis, the equations used for Cut I are valid for Cut II.

where:
- L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
- L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.
- M_{xy} = The maximum bending moment in the xy plane.
- F_{yx} = The maximum shear force in the xy plane.
- N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
- q_N = The shear force per inch of circumference applied to the nozzle.
- R_N = The mid-wall nozzle radius.

Since the pressure was considered separately in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{Nz}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{Nz}{2t_N}\right)^2 + (\tau_M)_{z\theta}^{\ 2}}$$

Per Reference [7], the recirculation outlet nozzle piping loads (Total thermal, weight and seismic loads) are as follows:

F_x = 20,000 lbs	M_x = 2,004,000 in-lb
F_y = 20,000 lbs	M_y = 3,000,000 in-lb
F_z = 30,000 lbs	M_z = 2,004,000 in-lb

L_1 is equal to 4.25 inches and the L_2 is equal to 42.77 inches. The calculations for the safe end and blend radius are shown in Table 1. The first cut location is the same as the Green's Function cross section per [1] at the safe end, and the second cut is from Node 3829 (inside) to Node 3809 (outside). This gives the maximum ID and minimum OD for the cross section calculation. The maximum stress intensities due to the piping loads are 5708.89 psi at the safe end and 280.16 psi at the blend radius. The piping load sign is set as the same as the thermal stress sign.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F, and the full values are reached at reactor design temperature, 575°F [6]. The scaled piping stress values are shown in columns 9 and 10 of Tables 4 and 5. Columns 11 and 12 of Tables 4 and 5 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient [2]:

Column 13 in Tables 4 and 5 shows the number of cycles associated with each transient. The number of cycles for 60 years was obtained from Reference [2] unless otherwise noted.

The program FATIGUE.EXE performs the "ASME Code style" peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius	Safe End
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel) [13]	1.7 & 0.3 (stainless steel) [13]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17000 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [13]	28.3×10^6 psi [13]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [1]	27.0×10^6 psi [1]
The Geometric Stress Concentration Factor K_t	1.0	1.53 [3]

The results of the fatigue analyses are presented in Tables 6 and 7 for the blend radius and safe end for 60 years, respectively.

The fatigue run inputs described are contained in EXCEL files *BRresults.xls* and *SEresults.xls*, which are contained in the computer files.

4.0 CALCULATION OF THERMAL STRESSES FOR TRANSIENT 9

Per Tables 2 and 3, the thermal shocks are from 526°F to 268°F and from 526°F to 130°F at the blend radius and the safe end, respectively. Therefore, the average temperatures for these two locations are about 400°F and 330°F. Since there are two different temperature shocks in the same model, ANSYS [10] will be used to calculate stresses directly. In this section, ANSYS [10] is used to simulate this transient and the results will then be used as input to FATIGUE.EXE, as shown in Tables 4 and 5. This case corresponds to the downhill (RPV) side of the blend radius.

An additional case was also run to simulate the uphill (RPV) side of the blend radius, where the thermal shocks are from 526°F to 130°F at the safe end, and no temperature change at the blend

radius. This case at the uphill side of the blend radius was found to produce lower stresses than the previously mentioned downhill case. Due to this, the downhill case was used for the rest of the analysis in this calculation.

4.1 Thermal Load

Since the average temperatures in the blend radius and safe end respectively are 400°F and 330°F, the material properties for 400°F are used for the blend radius, cladding and vessel. Table 8 shows the material properties at 400°F. The flow rate at this transient is 3395.2 GPM (calculated from 12% of max flow rate [2]) and is shown in Tables 2 and 3.

Heat transfer coefficients listed on Reference [4] are for pre power uprate. The heat transfer coefficients can be scaled by power uprate flow rate and diameter to values corresponding to the flow and location conditions. Referring to Figure 2, heat transfer coefficients were applied as follows:

Region 1

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4911 \cdot \left(\frac{2.084}{25}\right)^{0.8} = 672.8 \text{ BTU/hr-ft}^2\text{-°F.}$$

Per [4], the heat transfer coefficient at 100°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$2250 \cdot \left(\frac{2.084}{25}\right)^{0.8} = 308.24 \text{ BTU/hr-ft}^2\text{-°F.}$$

The fluid temperature shock is:

T = 526°F - 130°F - 526°F

Region 2

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4911 \cdot \left(\frac{2.084}{25}\right)^{0.8} \left(\frac{26}{35.49}\right)^{0.2} = 632.21 \text{ BTU/hr-ft}^2\text{-°F.}$$

Per [4], the heat transfer coefficient at 300°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4789 \cdot \left(\frac{2.084}{25}\right)^{0.8} \left(\frac{26}{35.49}\right)^{0.2} = 616.57 \text{ BTU/hr-ft}^2\text{-°F.}$$

The fluid temperature shock is:

T = 526°F - 268°F - 526°F

Region 3

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM flow is

$672.8(0.5) = 336.4$ BTU/hr-ft^2-°F.

Per [4], the heat transfer coefficient at 300°F, h, for 3395.2 GPM flow is

$$336.4\left(\frac{4789}{4911}\right) = 328.04 \text{ BTU/hr-ft}^2\text{-°F}.$$

The fluid temperature shock is:

Case 1: T = 526°F - 268°F - 526°F
Case 2: T = 526°F

Region 4

The heat transfer coefficient, h, is 0.4 BTU/hr-ft^2-°F [4].

The temperature is:

T = 120°F

4.2 Thermal Results

The flow dependent thermal load case outlined in Section 4.1 was run on the finite element model. Appendix A contains the thermal transient input file VY_RON_T_T9.INP for 3395.2 GPM flow rate. The flow dependent input files for the stress run is also included in Appendix A. The stress filename is VY_RON_S_T9.INP for 3395.2 GPM flow rate.

The critical safe end and blend radius locations are defined in Reference [1] at nodes 6395 and 3829, respectively.

The stress time history for the critical paths was extracted during the stress run. This produced two files, T9SE.OUT and T9BR.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses were extracted from these files to produce the files T9SE_Inside.RED and T9BR_Inside.RED, where SE and BR corresponded to the safe end and blend radius locations, respectively.

The data for the stress results is included in the files T9BR_M+B.xls, T9BR_T.xls, T9SE_M+B.xls, and T9SE_T.xls in the project Files. Where SE and BR corresponded to the safe end and blend radius locations, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

5.0 FATIGUE USAGE RESULTS

The blend radius cumulative usage factor (CUF) from system cycling is 0.0108 for 60 years (Table 6). The safe end CUF is 0.0015 for 60 years (Table 7).

6.0 ENVIRONMENTAL FATIGUE ANALYSIS

The Recirculation Outlet nozzle has three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified, locations in the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the FEM analysis occur in the stainless steel safe end. Stainless steel Fen multipliers are significantly higher than Ni-Cr-Fe multipliers (Fen values are 2.55 or higher for stainless steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [16]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.
- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel Fen multipliers are higher than Ni-Cr-Fe multipliers (Fen values are 2.45 or higher for low alloy steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [16]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

Per Reference [12], the dissolved oxygen (DO) calculation shows the overall hydrogen water chemistry (HWC) availability is 47%. This means the time ratio under normal water chemistry (NWC, or pre-HWC) is 53%.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are 15.35 and 8.36 from Table 5 of Reference [12]. These result in an EAF adjusted CUF of (15.35 x 47% + 8.36 x 53%) x 0.0015 = 0.0175 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 11.6453.

For the blend radius location, the environmental fatigue factors for post-HWC and pre-HWC are 2.45 and 12.43 from Table 5 of Reference [12]. These result in an EAF adjusted CUF of (2.45 x 47% + 12.43 x 53%) x 0.0108 = 0.08358 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 7.739.

7.0 REFERENCES

1. Structural Integrity Associates Calculation No. VY-16Q-305, Revision 0, "Recirculation Outlet Nozzle Green's Functions."
2. Entergy Design Input Record (DIR), Rev. 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
3. CB&I, RPV Stress Report Section: S9 "Stress Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
4. CB&I, RPV Stress Report Section: T9 "Thermal Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
5. Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
6. VY Drawing, 5920-06623 Rev. 0, (Hitachi, Ltd. Drawing No IOR290-127, Revision 0), "Recirc. Outlet Safe End," SI File No. VY-16Q-204.
7. GE. Stress Report No. 23A4316, Revision 0, "Reactor Vessel Recirculation Outlet Safe End," SI File No. VY-16Q-204.
8. VY Drawing 5920-00238 Rev. 4, (Chicago Bridge & Iron Company, Contract No. 9-6201, Drawing No. 21, Revision 4), "36"x28" Nozzles Mk N1A/B," SI File No. VY-16QQ-204.
9. ASME Boiler and Pressure Vessel Code, Section II, Materials, Part D, Properties, 1998 Edition with 2000 Addenda.
10. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
11. Structural Integrity Associates Calculation No. VY-16Q-304, Revision 0, "Recirculation Outlet Nozzle Finite Element Model."
12. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."
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14. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
15. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
16. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.
17. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, 1965 Edition with Winter 1966 Addenda.

Table 1: Maximum Piping Stress Intensity Calculations

Blend Radius External Piping Loads		
Parameters		
F_x =	20.00	kips
F_y =	20.00	kips
F_Z =	30.00	kips
M_x =	2004.00	in-kips
M_y =	3000.00	in-kips
M_Z =	2004.00	in-kips
OD=	55.88	in
ID=	37.368	in
R_N =	23.31	in
L =	42.77	in
t_N =	9.25	in
$(M_x)_2$ =	1148.54	in-kips
$(M_y)_2$ =	3855.46	in-kips
M_{xy} =	4022.90	in-kips
F_{xy} =	28.28	kips
N_Z =	2.56	kips/in
q_N =	-0.20	kips/in
Primary Membrane Stress Intensity		
PM_Z =	0.28	ksi
τ =	-0.02	ksi
SI_{max} =	0.28	ksi
SI_{max} =	280.16	psi

Safe End External Piping Loads		
Parameters		
F_x =	20.00	kips
F_y =	20.00	kips
F_Z =	30.00	kips
M_x =	2004.00	in-kips
M_y =	3000.00	in-kips
M_Z =	2004.00	in-kips
OD=	28.38	in
ID=	25.938	in
R_N =	13.58	in
L =	4.25	in
t_N =	1.22	in
$(M_x)_1$ =	1919.00	in-kips
$(M_y)_1$ =	3085.00	in-kips
M_{xy} =	3633.15	in-kips
F_{xy} =	28.28	kips
N_Z =	6.62	kips/in
q_N =	-1.07	kips/in
Primary Membrane Stress Intensity		
PM_Z =	5.43	ksi
τ =	-0.88	ksi
SI_{max} =	5.71	ksi
SI_{max} =	5708.89	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 2: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0 (50%)'
	16164	549	16164	1010	
	22164	549	6000	1010	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294 (100%)'
	1	542	1	1010	
	601	542	600	1010	
	602	526	1	1010	
	6602	526	6000	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294 (100%)'
	1800	542	1800	1010	
	2100	542	300	1010	
	2460	526	360	1010	
	3060	526	600	1010	
	3960	542	900	1010	
	4260	542	300	1010	
	6060	526	1800	1010	
	12060	526	6000	1010	
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0 (0%)'
	3	526	3	1190	
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1136	
	2393	300	180	885	
	6773	500	4380	1135	
	7193	300	420	675	14147 (50%)
	7493	300	300	675	
	11093	400	3600	240	
	16457	549	5364	1010	
	16517	549	60	1010	
	16518	542	1	1010	28294 (100%)
	17118	542	600	1010	
	17119	526	1	1010	
	23119	526	6000	1010	
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294 (100%)'
	10	526	10	1135	
	15	526	5	1135	
	30	526	15	940	
	1830	526	1800	940	
	2250	549	420	1010	
	2310	549	60	1010	
	2311	542	1	1010	
	2911	542	600	1010	
	2912	526	1	1010	
	8912	526	6000	1010	

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
6. Reactor Overpressure 1 Cycle	0	526		1010	28294 (100%)'
	2	526	2	1375	
	32	526	30	940	
	1832	526	1800	940	
	2252	549	420	1010	
	2312	549	60	1010	
	2313	542	1	1010	
	2913	542	600	1010	
	2914	526	1	1010	
	8914	526	6000	1010	
7. SRV Blowdown 1 Cycle	0	526		1010	28294 (100%)'
	600	375	600	170	
	11580	70	10980	50	
	17580	70	6000	50	
8. SCRAM Other 228 Cycles	0	526		1010	28294 (100%)'
	15	526	15	940	
	1815	526	1800	940	
	2235	549	420	1010	
	2295	549	60	1010	
	2296	542	1	1010	
	2356	542	60	1010	
	2357	526	1	1010	
	8357	526	6000	1010	
9. Improper Startup 1 Cycle	0	526		1010	3395 (12%)'
	1	268 (3)	1	1010	
	27	268 (3)	26	1010	
	28	526	1	1010	
	6028	526	6000	1010	
10. Shutdown 300 Cycles	0	549		1010	14147 (50%)'
	6264	375	6264	170	
	6864	330	600	88	
	16224	70	9360	50	
	22224	70	6000	50	
11. Design Hydrostatic Test 120 cycles	—	100	—	50	1981 (7%)
				1563	
				50	
12. Hydrostatic Test 1 Cycle	—	100	—	0	1981 (7%)
				1100	
				50	

Notes:
1. The instant temperature change is assumed as 1 second time step.
2. The number of cycles is for 60 years [2].
3. 268°F is the blend radius temperature for this transient. The safe end has a different temperature for Transient 9. [2]

Table 3: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0 (50%)'
	16164	549	16164	1010	
	16864	549	700	1010	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294 (100%)'
	1	542	1	1010	
	601	542	600	1010	
	602	526	1	1010	
	1302	526	700	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294 (100%)'
	1800	542	1800	1010	
	2100	542	300	1010	
	2460	526	360	1010	
	3060	526	600	1010	
	3960	542	900	1010	
	4260	542	300	1010	
	6060	526	1800	1010	
	6760	526	700	1010	
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0 (0%)'
	3	526	3	1190	
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6773	500	4380	1135	
	7193	300	420	675	14147 (50%)'
	7493	300	300	675	
	11093	400	3600	240	
	16457	549	5364	1010	
	16517	549	60	1010	
	16518	542	1	1010	28294 (100%)'
	17118	542	600	1010	
	17119	526	1	1010	
	17819	526	700	1010	
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294 (100%)'
	10	526	10	1135	
	15	526	5	1135	
	30	526	15	940	
	1830	526	1800	940	
	2250	549	420	1010	
	2310	549	60	1010	
	2311	542	1	1010	
	2911	542	600	1010	
	2912	526	1	1010	
	3612	526	700	1010	

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
6. Reactor Overpressure 1 Cycle	0	526		1010	28294 (100%)'
	2	526	2	1375	
	32	526	30	940	
	1832	526	1800	940	
	2252	549	420	1010	
	2312	549	60	1010	
	2313	542	1	1010	
	2913	542	600	1010	
	2914	526	1	1010	
	3614	526	700	1010	
7. SRV Blowdown 1 Cycle	0	526		1010	28294 (100%)'
	600	375	600	170	
	11580	70	10980	50	
	12280	70	700	50	
8. SCRAM Other 228 Cycles	0	526		1010	28294 (100%)'
	15	526	15	940	
	1815	526	1800	940	
	2235	549	420	1010	
	2295	549	60	1010	
	2296	542	1	1010	
	2356	542	60	1010	
	2357	526	1	1010	
	3057	526	700	1010	
9. Improper Startup 1 Cycle	0	526		1010	3395 (12%)'
	1	130 (3)	1	1010	
	27	130 (3)	26	1010	
	28	526	1	1010	
	728	526	700	1010	
10. Shutdown 300 Cycles	0	549		1010	14147 (50%)'
	6264	375	6264	170	
	6864	330	600	88	
	16224	70	9360	50	
	16924	70	700	50	
11. Design Hydrostatic Test 120 Cycles	—	100	—	0	1981 (7%)'
				1100	
				50	
12. Hydrostatic Test 1 Cycle	—	100	—	50	1981 (7%)'
				1563	
				50	

Notes:
1. The instant temperature change is assumed as 1 second time step.
2. The number of cycles is for 60 years [2].
3. 130°F is the safe end temperature for this transient. The blend radius has a different temperature for Transient 9. [2]

Note: These transients are the same as in Table 2 with the exception of the 700 second steady state time increment that is used. The transients in Table 2 are plotted using a 6000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.

Table 4: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	459	388	100.00	0	0	0	16.64312	16.64312	475.64	404.64	300
	4303	-3417	-1594	219.53	1010	31613	33976.4	-82.95209	-82.95209	28113.05	32299.45	300
	22164	2713	2306	549.00	1010	31613	33976.4	265.7352	265.7352	34591.74	36548.14	300
2	0.00	3094	1934	549	1010	31613	33976.4	265.7352	265.7352	34972.74	36176.14	300
	94.30	4079	2481	542	1010	31613	33976.4	261.8518	261.8518	35953.85	36719.25	300
	601.70	3683	2435	538.8	1010	31613	33976.4	260.0765	260.0765	35556.08	36671.48	300
	680.10	5891	3489	526	1010	31613	33976.4	252.9754	252.9754	37756.98	37718.38	300
	6602.00	2977	1859	526	1010	31613	33976.4	252.9754	252.9754	34842.98	36088.38	300
3	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	10
	1807.20	1834	1043	542	1010	31613	33976.4	261.8518	261.8518	33708.85	35281.25	10
	2491.50	4425	2667	526	1010	31613	33976.4	252.9754	252.9754	36290.98	36896.38	10
	3974.40	1706	1060	542	1010	31613	33976.4	261.8518	261.8518	33580.85	35298.25	10
	6070.80	3971	2551	526	1010	31613	33976.4	252.9754	252.9754	35836.98	36780.38	10
	12060.00	2965	1852	526	1010	31613	33976.4	252.9754	252.9754	34830.98	36081.38	10
4	0	-2465	-703	526.00	1010	31613	33976.4	252.9754	-252.9754	34330.98	33020.42	10
	3	2465	-703	526.00	1190	37247	40031.6	252.9754	-252.9754	39964.98	39075.62	10
	13	2465	-703	526.00	1135	35525.5	38181.4	252.9754	-252.9754	38243.48	37225.42	10
	435.6	18138	9690	356.38	1135	35525.5	38181.4	158.8774	158.8774	53822.38	48030.28	10
	2222.5	-1169	-2598	489.44	1135	35525.5	38181.4	-232.6952	-232.6952	34123.80	35350.70	10
	2665.5	12763	6695	328.40	885	27700.5	29771.4	143.3539	143.3539	40606.85	36609.75	10
	6779.2	-4008	-2829	497.05	1010	31613	33976.4	-236.9137	-236.9137	27368.09	30910.49	10
	7243.8	19275	9965	302.91	1010	31613	33976.4	129.2122	129.2122	51017.21	44070.61	10
	13996	-2135	34	542.00	1010	31613	33976.4	-261.8518	261.8518	29216.15	34272.25	10
	17247	3413	2074	526.00	1010	31613	33976.4	252.9754	252.9754	35278.98	36303.38	10
	23119	2971	1855	526.00	1010	31613	33976.4	252.9754	252.9754	34836.98	36084.38	10
5	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	60
	10.00	2959	1849	526	1135	35525.5	38181.4	252.9754	252.9754	38737.48	40283.38	60
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	60
	2269.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	60
	3010.10	4407	2579	526	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	60
	8912.00	2968	1854	526	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	60
6	0.00	2959	1849	526.00	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	2.00	2959	1849	526.00	1375	43037.5	46255	252.9754	252.9754	46249.48	48356.98	1
	32.00	2959	1849	526.00	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	1
	2271.50	111	295	549.00	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	1
	3022.00	4407	2579	526.00	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	1
	8914.00	2968	1854	526.00	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	1
7	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	615.10	20260	12980	374.581	170	5321	5718.8	168.9726	168.9726	25749.97	18867.77	1
	17580.00	279	179	70	50	1565	1682	0	0	1844.00	1861.00	1
8	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	228
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	228
	2254.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	228
	2491.20	3792	2234	526	1010	31613	33976.4	252.9754	252.9754	35657.98	36463.38	228
	8357.00	2963	1851	526	1010	31613	33976.4	252.9754	252.9754	34828.98	36080.38	228
9	0	2058	961	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35190.26	1
	0.52	1956	734	525.6	1010	31613	33976.4	252.7535	252.7535	33821.75	34963.15	1
	28	23747	3188	504.5	1010	31613	33976.4	241.0479	241.0479	55601.05	37405.45	1
	425	1520	611	525.5	1010	31613	33976.4	252.698	252.698	33385.70	34640.10	1
	12400	2058	879	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35108.26	1
10	0	2767	2176	549	1010	31613	33976.4	265.7352	265.7352	34645.74	36418.14	300
	4240.8	6643	4158	445.775	441	13803.3	14835.24	208.469	208.469	20654.77	19201.71	300
	6268	6498	3675	374.7	170	5321	5718.8	169.0386	169.0386	11988.04	9562.84	300
	6891.8	9282	5241	329 228	88	2754.4	2960.32	143.8121	143.8121	12180.21	8345.13	300
	22224	361	120	70	50	1565	1682	0	0	1926.00	1802.00	300
11		0	0	100	0	0	0	16.64312	16.64312	16.64	16.64	120
		0	0	100	1100	34430	37004	16.64312	16.64312	34446.64	37020.64	120
		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	120
12		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1
		0	0	100	1563	48921.9	52579.32	16.64312	16.64312	48938.54	52595.96	1
		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 2.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 31300)/1000.
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 33640)/1000.
Column 9: Total external stress from calculation in Table 1, 280.16 psi*(Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 5: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	-925	-949	100.00	0	0	0	-339.1419	-339.1419	-1264.14	-1288.14	300
	16164	-4814	-4433	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	1375.93	1615.53	300
	16864	-3749	-3705	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	2440.93	2343.53	300
2	0	-3838	-3665	549	1010	11604.9	11463.5	-5414.966	-5414.966	2351.93	2383.53	300
	6	-1664	-2263	542	1010	11604.9	11463.5	-5335.833	-5335.833	4605.07	3864.67	300
	601	-3773	-3607	542	1010	11604.9	11463.5	-5335.833	-5335.833	2496.07	2520.67	300
	606.6	1196	-403	526	1010	11604.9	11463.5	5154.958	-5154.958	17955.86	5905.54	300
	1302	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	300
3	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	10
	1800.1	-4165	-3904	542	1010	11604.9	11463.5	-5335.833	-5335.833	2104.07	2223.67	10
	2460.2	-1932	-2200	526	1010	11604.9	11463.5	-5154.958	-5154.958	4517.94	4108.54	10
	3960.2	-4537	-4185	542	1010	11604.9	11463.5	-5335.833	-5335.833	1732.07	1942.67	10
	6060.2	-3315	-3241	526	1010	11604.9	11463.5	-5154.958	-5154.958	3134.94	3067.54	10
	6760	-3687	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2762.94	2786.54	10
4	0.00	-3756	-3716	526	1020	11719.8	11577	-5154.958	-5154.958	2808.84	2706.04	10
	3.00	-3756	-3716	526	1190	13673.1	13506.5	-5154.958	-5154.958	4762.14	4635.54	10
	13.00	-3756	-3716	526	1135	13041.15	12882.25	-5154.958	-5154.958	4130.19	4011.29	10
	242.30	15878	10049	302.374	1135	13041.15	12882.25	2626.926	2626.926	31546.08	25558.18	10
	2213.10	-6388	-5428	499.889	1135	13041.15	12882.25	-4859.78	-4859.78	1793.37	2594.47	10
	2408.60	13203	8265	301.443	885	10168.65	10044.75	2616.401	2616.401	25988.05	20926.15	10
	6773.40	-4763	-4312	499.809	1135	13041.15	12882.25	-4858.875	-4858.875	3419.27	3711.37	10
	7193.10	15374	9801	300	675	7755.75	7661.25	2600.088	2600.088	25729.84	20062.34	10
	16457.50	-4812	-5032	549	240	2757.6	2724	-5414.966	-5414.966	-7469.37	-7722.97	10
	16524.70	-2358	-2725	542	1010	11604.9	11463.5	-5335.833	-5335.833	3911.07	3402.67	10
	17118.00	-3778	-3610	541.996	1010	11604.9	11463.5	-5335.788	-5335.788	2491.11	2517.71	10
	17123.60	1192	-406	526	1010	11604.9	11463.5	5154.958	-5154.958	17951.86	5902.54	10
	17819.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	10
5	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	60
	10.00	-3688	-3522	526	1135	13041.15	12882.25	-5154.958	-5154.958	4198.19	4205.29	60
	30.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	60
	2250.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	60
	2319.90	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	60
	2911.00	-3782	-3613	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	60
	2916.70	1188	-408	526	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	60
	3612.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	60
6	0.00	-3688	-3522	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	2.00	-3688	-3522	5.26E+02	1375	15798.75	15606.25	-5154.958	-5154.958	6955.79	6929.29	1
	32.00	-3688	-3522	5.26E+02	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	1
	2252.10	-6054	-5337	5.49E+02	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	1
	2322.20	-2977	-3123	5.42E+02	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	1
	2913.00	-3782	-3613	5.42E+02	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	1
	2918.70	1188	-408	5.26E+02	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	1
	3614.00	-3670	-3509	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	1
7	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	600	7773	5336	375	170	1953.3	1929.5	3447.943	3447.943	13174.24	10713.44	1
	1367.9	-1390	-1567	354.172	162	1861.38	1838.7	-3212.488	-3212.488	-2741.11	-2940.79	1
	11580.1	454	190	70	50	574.5	567.5	0	0	1028.50	757.50	1
	12280	-707	-689	70	50	574.5	567.5	0	0	-132.50	-121.50	1
8	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	228
	15.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	228
	2235.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	228
	2305.20	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	228
	2356.00	-3183	-3151	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	3086.08	2976.68	228
	2361.50	1761	-28	526	1010	11604.9	11463.5	5154.958	-5154.958	18520.86	6280.54	228
	3057.00	-3667	-3506	526	1010	11604.9	11463.5	-5154.958	-5154.958	2782.94	2802.54	228
9	0	-2968	-2837	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3485.22	3474.82	1
	27	68473	45303	291.3	1010	11604.9	11463.5	2501.737	2501.737	82579.74	59268.34	1
	80.7	-11546	-8877	518.4	1010	11604.9	11463.5	-5069.042	-5069.042	-5010.04	-2482.14	1
	5200	-2967	-2832	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3486.21	3479.78	1
10	0	-3745	-3709	549	1010	11604.9	11463.5	-5414.966	-5414.966	2444.93	2339.53	300
	6864.2	501	-405	329.994	170	1953.3	1929.5	2939.162	-2939.162	5393.46	-1414.66	300
	7455.5	-1183	-1528	314.325	88	1011.12	998.8	-2762.029	-2762.029	-2933.91	-3291.23	300
	16224.1	334	-35	70	50	574.5	567.5	0	0	908.50	532.50	300
	16924	-731	-763	70	50	574.5	567.5	0	0	-156.50	-195.50	300
11		0	0	100	0	0	0	339.1419	339.1419	339.14	339.14	120
		0	0	100	1100	12639	12485	339.1419	339.1419	12978.14	12824.14	120
		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	120
12		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1
		0	0	100	1563	17958.87	17740.05	339.1419	339.1419	18298.01	18079.19	1
		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1

NOTES: Column 1: Transient number identification.

Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.

Column 3: Maxima or minima total stress intensity from P-V.OUT output file.

Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.

Column 5: Temperature per total stress intensity.

Column 6: Pressure per Table 3.

Column 7: Total pressure stress intensity from the quantity (Column 6 x 11490)/1000.

Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 11350)/1000.

Column 9: Total external stress from calculation in Table 1, 5708.89 psi*(Column 5-70°F)/(575°F -70°F).

Column 10: Same as Column 9, but for M+B stress.

Column 11: Sum of total stresses (Columns 3, 7, and 9).

Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).

Column 13: Number of cycles for the transient (60 years).

Table 6: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 2.0
n = .2
Sm = 26700. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.670E+07 psi
Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
55601.	17.	55584.	37389.	1.000	31227.	1.000E+00	1.951E+04	.0001
53822.	17.	53806.	48014.	1.000	30228.	1.000E+01	2.161E+04	.0005
51017.	17.	51001.	44054.	1.000	28652.	1.000E+01	2.547E+04	.0004
48939.	17.	48922.	52579.	1.000	27484.	1.000E+00	2.894E+04	.0000
46249.	17.	46233.	48340.	1.000	25974.	1.000E+00	3.443E+04	.0000
40607.	17.	40590.	36593.	1.000	22803.	1.000E+01	5.217E+04	.0002
39965.	17.	39948.	39059.	1.000	22443.	1.000E+01	5.647E+04	.0002
38737.	17.	38721.	40267.	1.000	21753.	6.000E+01	6.592E+04	.0009
38243.	17.	38227.	37209.	1.000	21476.	1.000E+01	7.025E+04	.0001
37757.	17.	37740.	37702.	1.000	21202.	7.000E+00	7.486E+04	.0001
37757.	476.	37281.	37314.	1.000	20945.	2.930E+02	7.954E+04	.0037
36291.	476.	35815.	36492.	1.000	20121.	7.000E+00	9.705E+04	.0001
36291.	1582.	34709.	35198.	1.000	19500.	3.000E+00	1.096E+05	.0000
36273.	1582.	34691.	35110.	1.000	19490.	6.000E+01	1.098E+05	.0005
36273.	1582.	34691.	35110.	1.000	19490.	1.000E+00	1.098E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	5.600E+01	1.135E+05	.0005
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1844.	34110.	34858.	1.000	19163.	1.000E+00	1.167E+05	.0000
35954.	1926.	34028.	34917.	1.000	19117.	2.410E+02	1.177E+05	.0020
35837.	1926.	33911.	34978.	1.000	19051.	1.000E+01	1.191E+05	.0001
35658.	1926.	33732.	34661.	1.000	18951.	4.900E+01	1.214E+05	.0004
35658.	11988.	23670.	26901.	1.000	13298.	1.790E+02	5.728E+05	.0003
35556.	11988.	23568.	27109.	1.000	13240.	1.210E+02	5.955E+05	.0002
35556.	12180.	23376.	28326.	1.000	13133.	1.790E+02	6.411E+05	.0003
35279.	12180.	23099.	27958.	1.000	12977.	1.000E+01	7.138E+05	.0000
34973.	12180.	22793.	27831.	1.000	12805.	1.110E+02	8.050E+05	.0001
34973.	20655.	14318.	16974.	1.000	8044.	1.890E+02	7.421E+07	.0000
34843.	20655.	14188.	16887.	1.000	7971.	1.110E+02	7.983E+07	.0000
34843.	25750.	9093.	17221.	1.000	5108.	1.000E+00	1.000E+20	.0000
34843.	27368.	7475.	5178.	1.000	4199.	1.000E+01	1.000E+20	.0000
34843.	28113.	6730.	3789.	1.000	3781.	1.780E+02	1.000E+20	.0000
34837.	28113.	6724.	3785.	1.000	3777.	1.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	6.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	1.000E+00	1.000E+20	.0000
34831.	28113.	6718.	3782.	1.000	3774.	1.000E+01	1.000E+20	.0000
34829.	28113.	6716.	3781.	1.000	3773.	4.100E+01	1.000E+20	.0000
34829.	29216.	5613.	1808.	1.000	3153.	1.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	6.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.000E+00	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.160E+02	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	6.000E+01	1.000E+20	.0000

34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	4.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	6.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.000E+00	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.270E+02	1.000E+20	.0000
34646.	32634.	2012.	2695.	1.000	1130.	1.010E+02	1.000E+20	.0000
34646.	33386.	1260.	1578.	1.000	708.	1.000E+00	1.000E+20	.0000
34646.	33581.	1065.	1120.	1.000	598.	1.000E+01	1.000E+20	.0000
34646.	33709.	937.	1137.	1.000	526.	1.000E+01	1.000E+20	.0000
34646.	33822.	824.	1455.	1.000	463.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1228.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1310.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	34124.	522.	1067.	1.000	293.	1.000E+01	1.000E+20	.0000
34646.	34331.	315.	3398.	1.000	177.	1.000E+01	1.000E+20	.0000
34646.	34447.	199.	-603.	1.000	112.	1.200E+02	1.000E+20	.0000
34646.	34592.	54.	-130.	1.000	30.	3.500E+01	1.000E+20	.0000
								=======
							TOTAL USAGE FACTOR =	.0108

Table 7: Fatigue Results for Safe End (60 Years)

LOCATION = LOCATION NO. 1 -- SAFE END
FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 1.7
n = .3
Sm = 17000. psi
Ecurve = 2.830E+07 psi
Eanalysis = 2.700E+07 psi
Kt = 1.53

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
82580.	-7469.	90049.	66991.	2.045	134573.	1.000E+00	6.765E+02	.0015
31546.	-7469.	39015.	33281.	1.000	29691.	9.000E+00	6.857E+05	.0000
31546.	-5010.	36556.	28040.	1.000	26947.	1.000E+00	1.160E+06	.0000
25988.	-2934.	28922.	24217.	1.000	21884.	1.000E+01	2.383E+06	.0000
25730.	-2934.	28664.	23354.	1.000	21509.	1.000E+01	2.566E+06	.0000
18521.	-2934.	21455.	9572.	1.000	13903.	2.280E+02	9.710E+08	.0000
18298.	-2934.	21232.	21370.	1.000	17063.	1.000E+00	7.876E+06	.0000
17956.	-2934.	20890.	9197.	1.000	13502.	5.100E+01	1.000E+20	.0000
17956.	-2741.	20697.	8846.	1.000	13304.	1.000E+00	1.000E+20	.0000
17956.	-1264.	19220.	7194.	1.000	12071.	2.480E+02	1.000E+20	.0000
17952.	-1264.	19216.	7191.	1.000	12068.	1.000E+01	1.000E+20	.0000
17948.	-1264.	19212.	7189.	1.000	12065.	4.200E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.800E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.000E+00	1.000E+20	.0000
13174.	-157.	13331.	10909.	1.000	10016.	1.000E+00	1.000E+20	.0000
12978.	-157.	13135.	13020.	1.000	10500.	1.200E+02	1.000E+20	.0000
6956.	-157.	7112.	7125.	1.000	5706.	1.000E+00	1.000E+20	.0000
5393.	-157.	5550.	-1219.	1.000	2570.	1.590E+02	1.000E+20	.0000
5393.	-133.	5526.	-1293.	1.000	2537.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	6.000E+01	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	7.900E+01	1.000E+20	.0000
4762.	136.	4626.	3924.	1.000	3514.	1.000E+01	1.000E+20	.0000
4605.	136.	4469.	3153.	1.000	3218.	1.390E+02	1.000E+20	.0000
4605.	339.	4266.	3526.	1.000	3215.	1.200E+02	1.000E+20	.0000
4605.	909.	3697.	3332.	1.000	2863.	4.100E+01	1.000E+20	.0000
4518.	909.	3609.	3576.	1.000	2885.	1.000E+01	1.000E+20	.0000
4198.	909.	3290.	3673.	1.000	2744.	6.000E+01	1.000E+20	.0000
4130.	909.	3222.	3479.	1.000	2655.	1.000E+01	1.000E+20	.0000
3911.	909.	3003.	2870.	1.000	2371.	1.000E+01	1.000E+20	.0000
3486.	909.	2578.	2947.	1.000	2170.	1.000E+00	1.000E+20	.0000
3485.	909.	2577.	2942.	1.000	2168.	1.000E+00	1.000E+20	.0000
3419.	909.	2511.	3179.	1.000	2199.	1.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	6.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	1.000E+00	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	9.600E+01	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.200E+02	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	1029.	2264.	2247.	1.000	1810.	1.000E+00	1.000E+20	.0000
3292.	1376.	1916.	1389.	1.000	1390.	9.000E+00	1.000E+20	.0000
3135.	1376.	1759.	1452.	1.000	1325.	1.000E+01	1.000E+20	.0000
3086.	1376.	1710.	1361.	1.000	1274.	2.280E+02	1.000E+20	.0000

2809.	1376.	1433.	1091.	1.000	1054.	1.000E+01	1.000E+20	.0000
2783.	1376.	1407.	1187.	1.000	1067.	4.300E+01	1.000E+20	.0000
2783.	1732.	1051.	860.	1.000	790.	1.000E+01	1.000E+20	.0000
2783.	1793.	990.	208.	1.000	576.	1.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	6.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.000E+00	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.040E+02	1.000E+20	.0000
2780.	1958.	822.	808.	1.000	655.	1.240E+02	1.000E+20	.0000
2780.	2104.	676.	576.	1.000	514.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.660E+02	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	6.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+00	1.000E+20	.0000
2763.	2352.	411.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	4.300E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.700E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	2.280E+02	1.000E+20	.0000
2496.	2441.	55.	177.	1.000	78.	5.300E+01	1.000E+20	.0000
2496.	2445.	51.	181.	1.000	77.	2.470E+02	1.000E+20	.0000
2491.	2445.	46.	178.	1.000	74.	1.000E+01	1.000E+20	.0000
2487.	2445.	42.	175.	1.000	71.	4.300E+01	1.000E+20	.0000
2487.	2487.	0.	0.	1.000	0.	1.700E+01	1.000E+20	.0000
								=======
							TOTAL USAGE FACTOR =	.0015

Table 8: Material Properties (For Transient 9)[1]

Material	SA-533 Gr B @400 °F (Mn-1/2Mo-1/2Ni)	SA-508 Cl 2 @400 °F (3/4Ni-1/2Mo-1/3Cr-V)	SA-240 Type 304 @400 °F (18Cr-8Ni)	SA-182 F316 @300 °F (16Cr-12Ni-2Mo)
Modulus of Elasticity, e^{-6} psi	27.4	26.1	26.5	27.0
Coefficient of Thermal Expansion, e^{-6}, in/in/°F	8.0	7.7	10.2	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.1	23.1	10.4	9.3
Thermal Diffusivity, ft^2/hr	0.378	0.378	0.165	0.150
Specific Heat, Btu/lb-°F[2]	0.125	0.125	0.129	0.127
Density, lb/in³	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes: [1] Material Properties are evaluated at 400°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. The safe end material properties were used for 300°F, the Code table values closest to the average temperature for the safe end for transient 9.

[2] Calculated as $[k/(\rho d)]/12^3$.

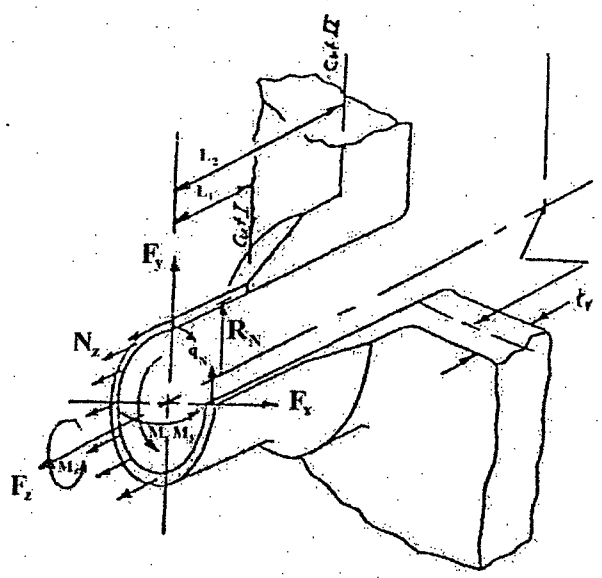


Figure 1: External Forces and Moments on the Recirculation Outlet Nozzle

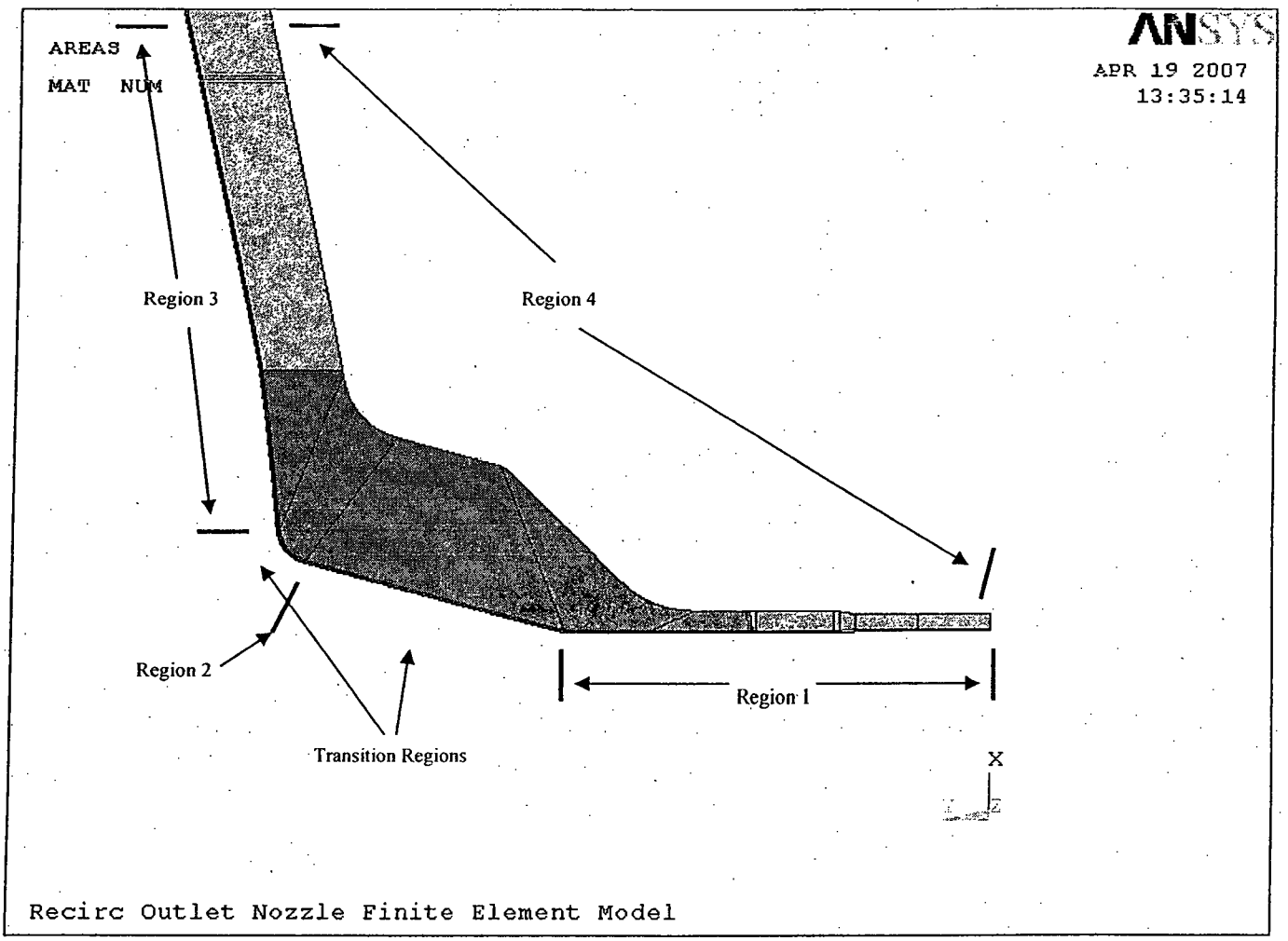


Figure 2: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries for Transient 9

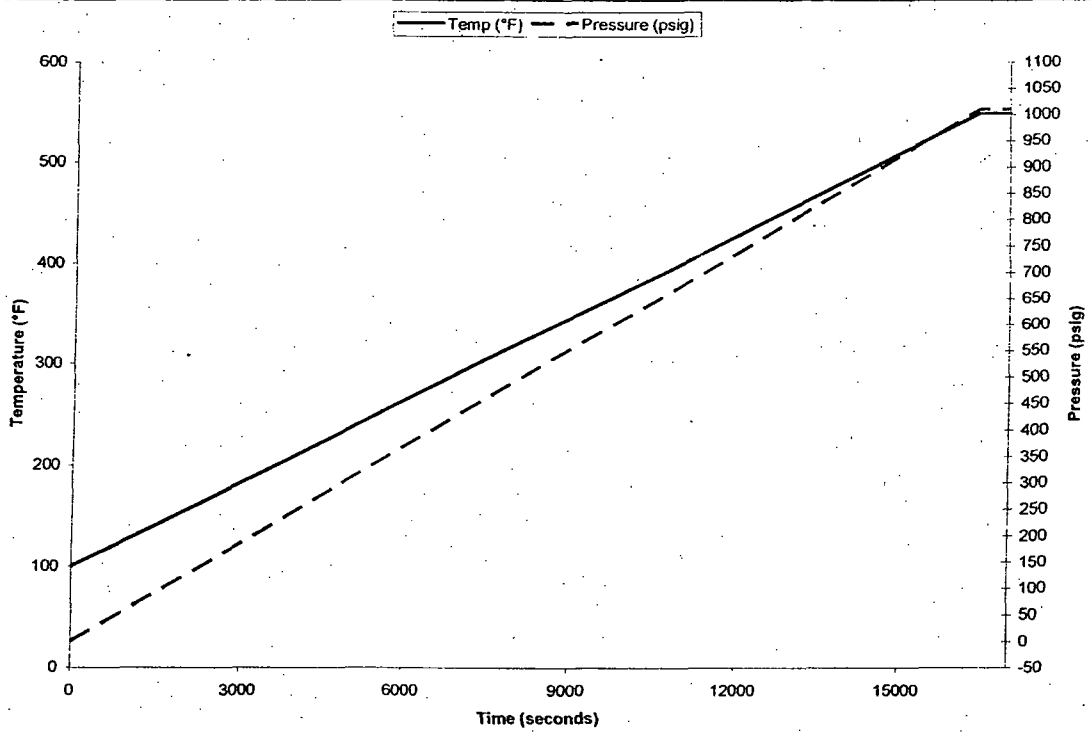


Figure 3: Transient 1 – Normal Startup at 100°F/hr

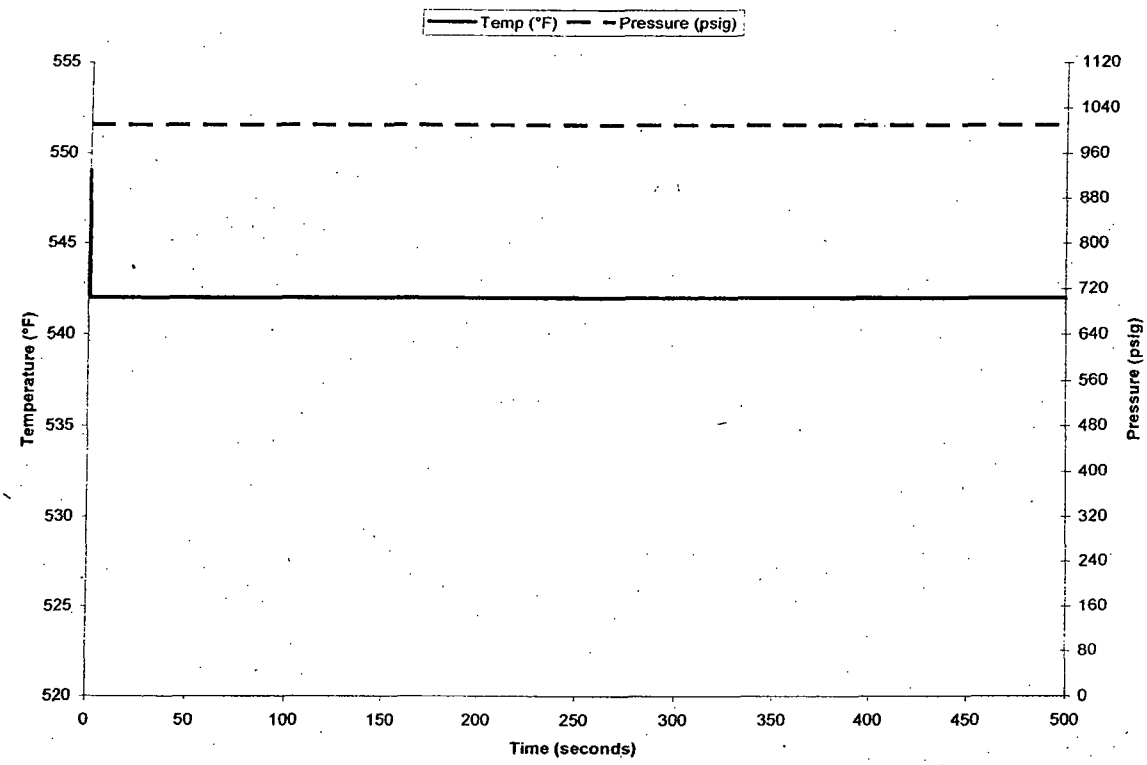


Figure 4: Transient 2 – Turbine Roll and Increase to Rated Power

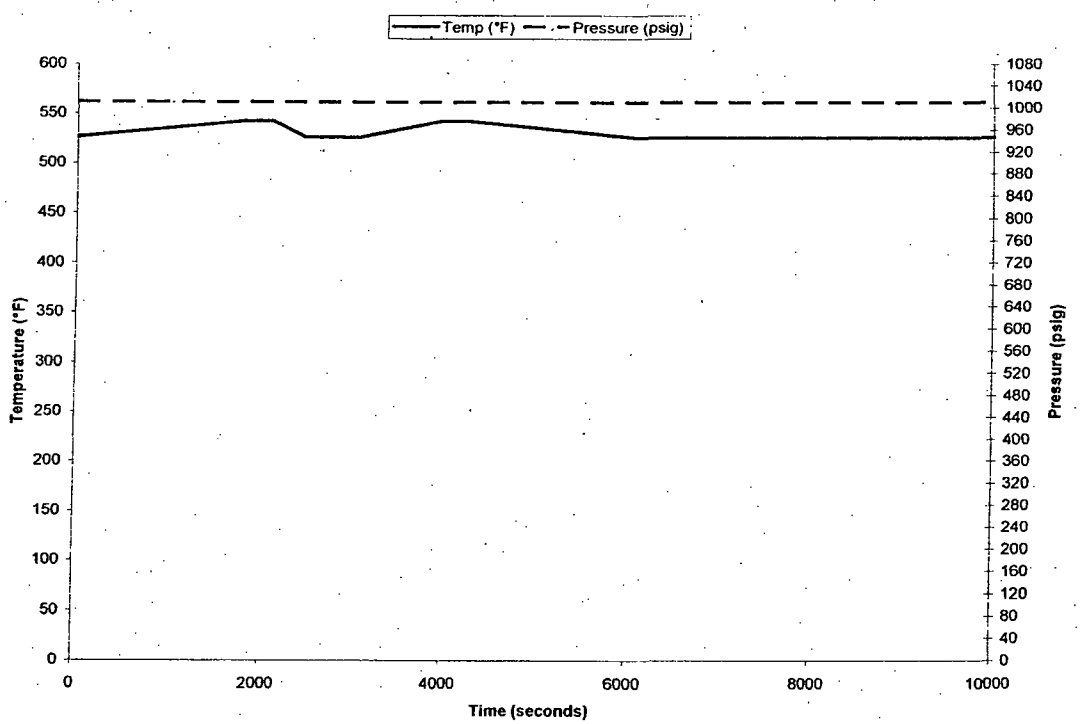


Figure 5: Transient 3 – Loss of Feedwater Heaters and Turbine Trip 25% Power

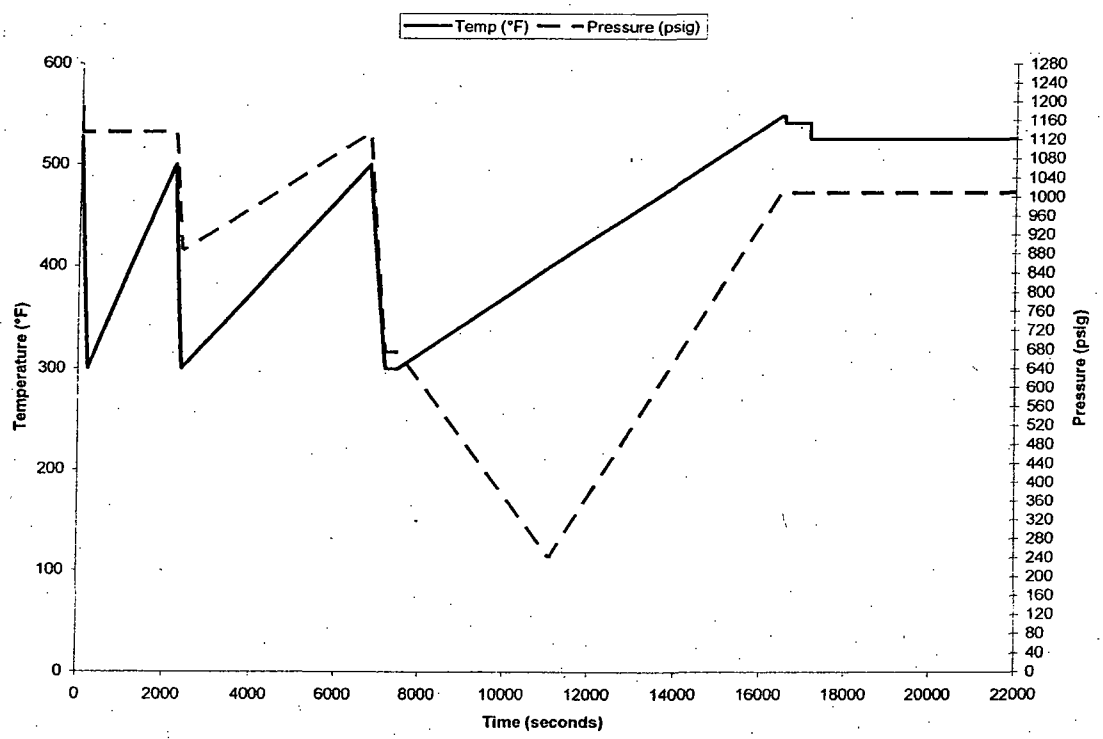


Figure 6: Transient 4 – Loss of Feedwater Pumps

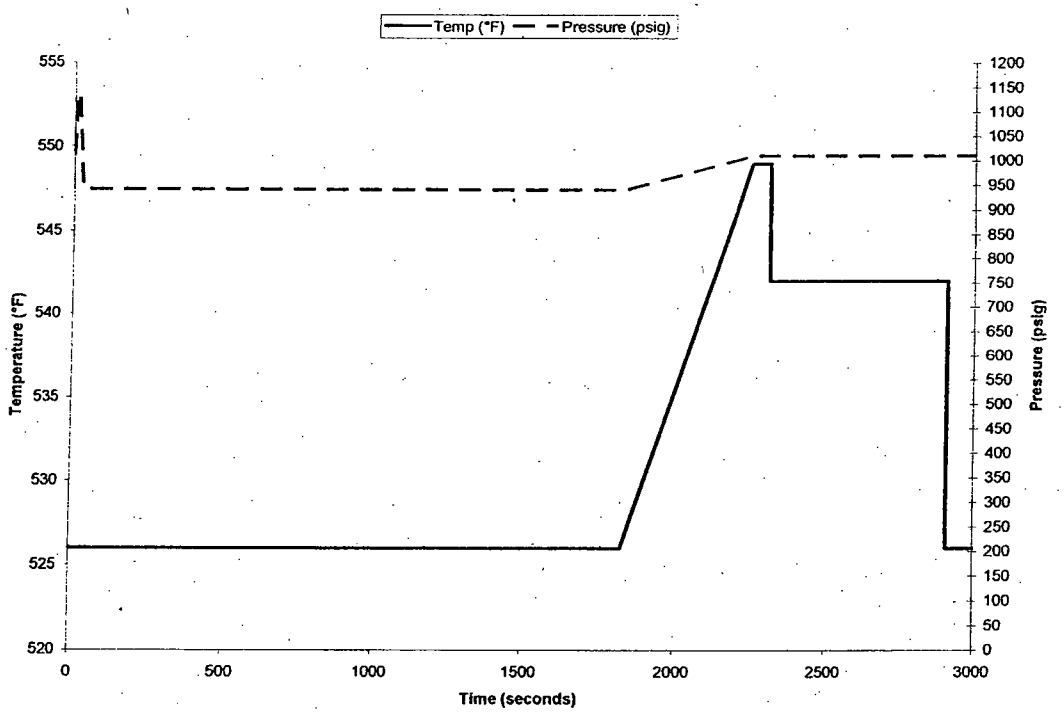


Figure 7: Transient 5 – Turbine Generator Trip

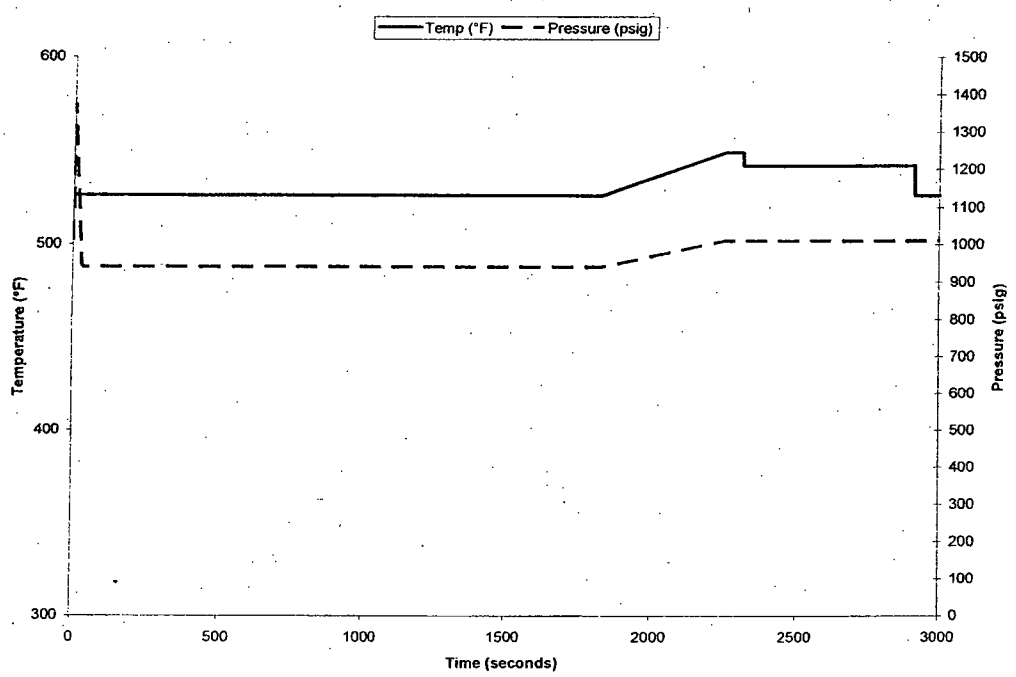


Figure 8: Transient 6 – Reactor Overpressure

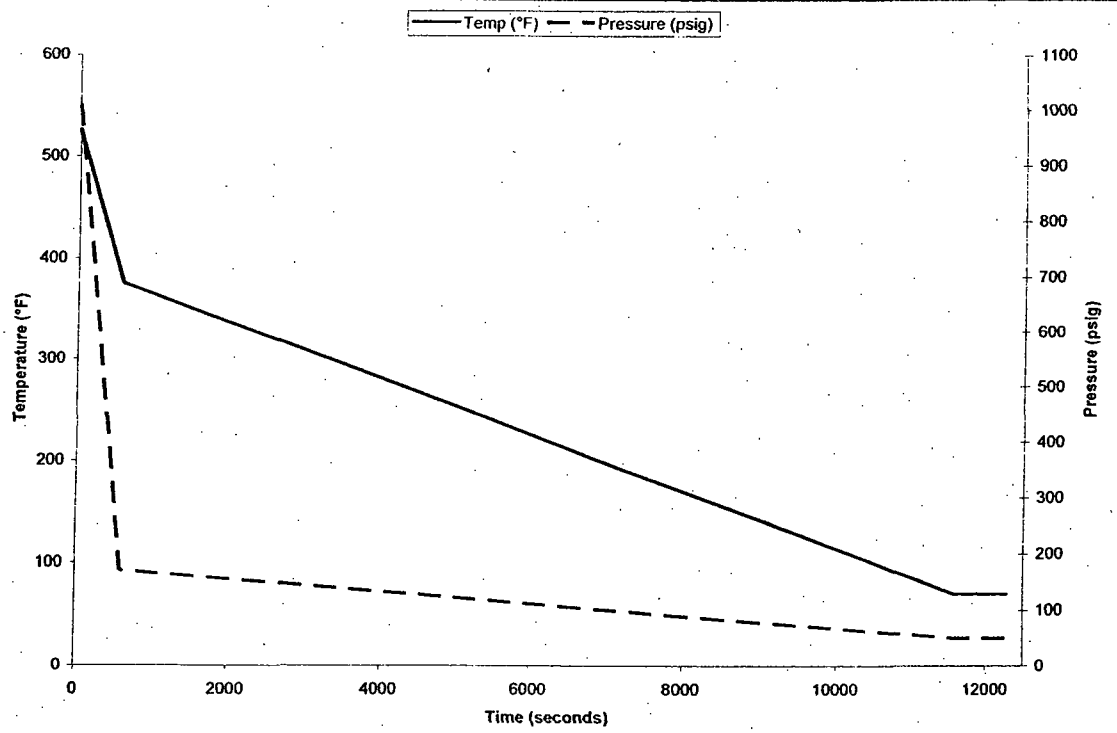


Figure 9: Transient 7 – SRV Blowdown

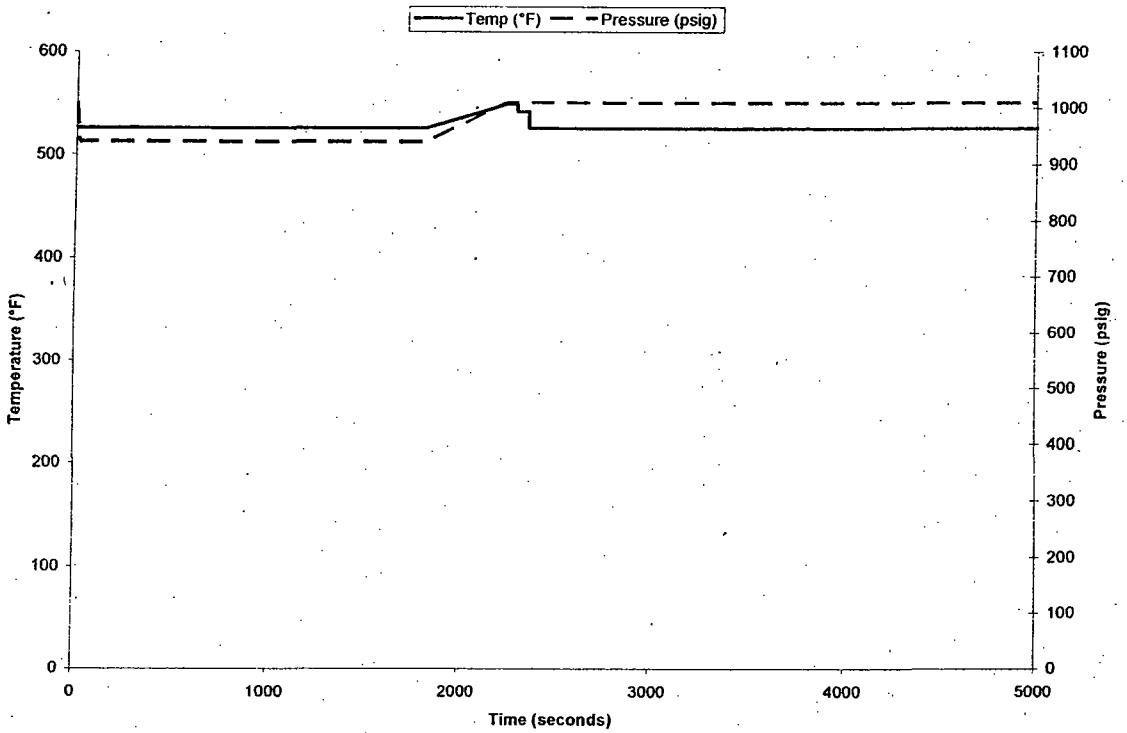


Figure 10: Transient 8 – SCRAM Other

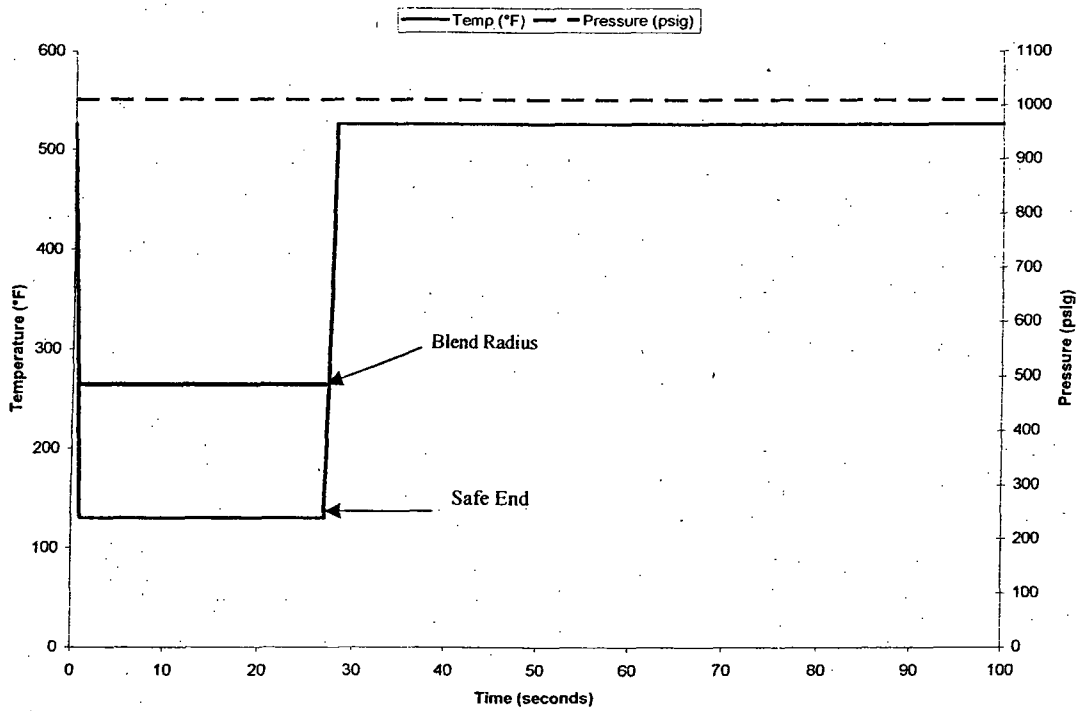


Figure 11: Transient 9 – Improper Startup

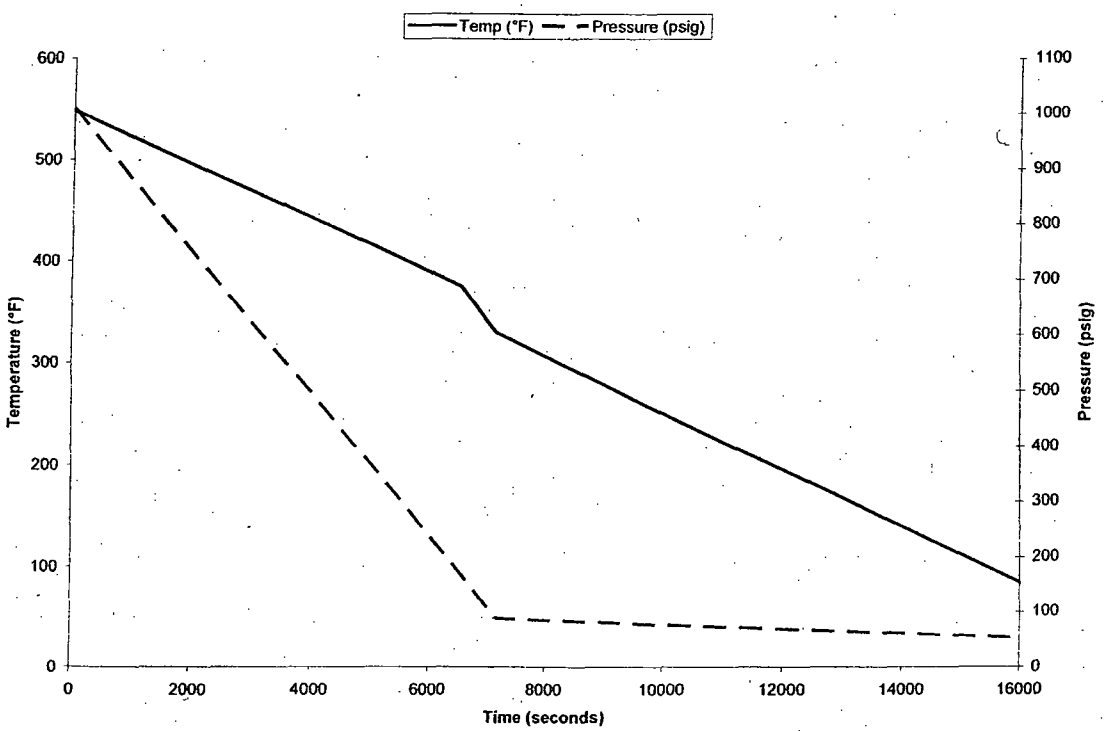
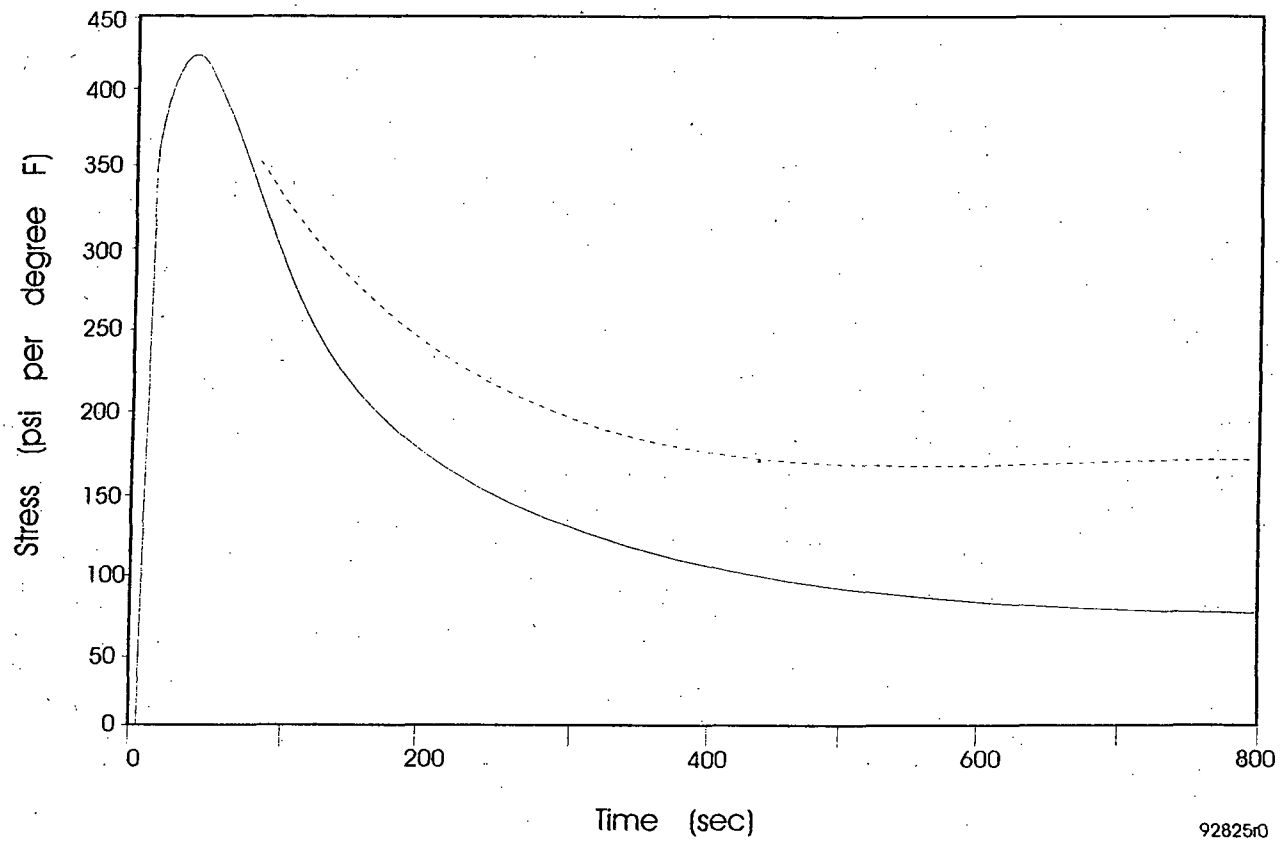


Figure 12: Transient 10 – Shutdown



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 13: Typical Green's Functions for Thermal Transient Stress

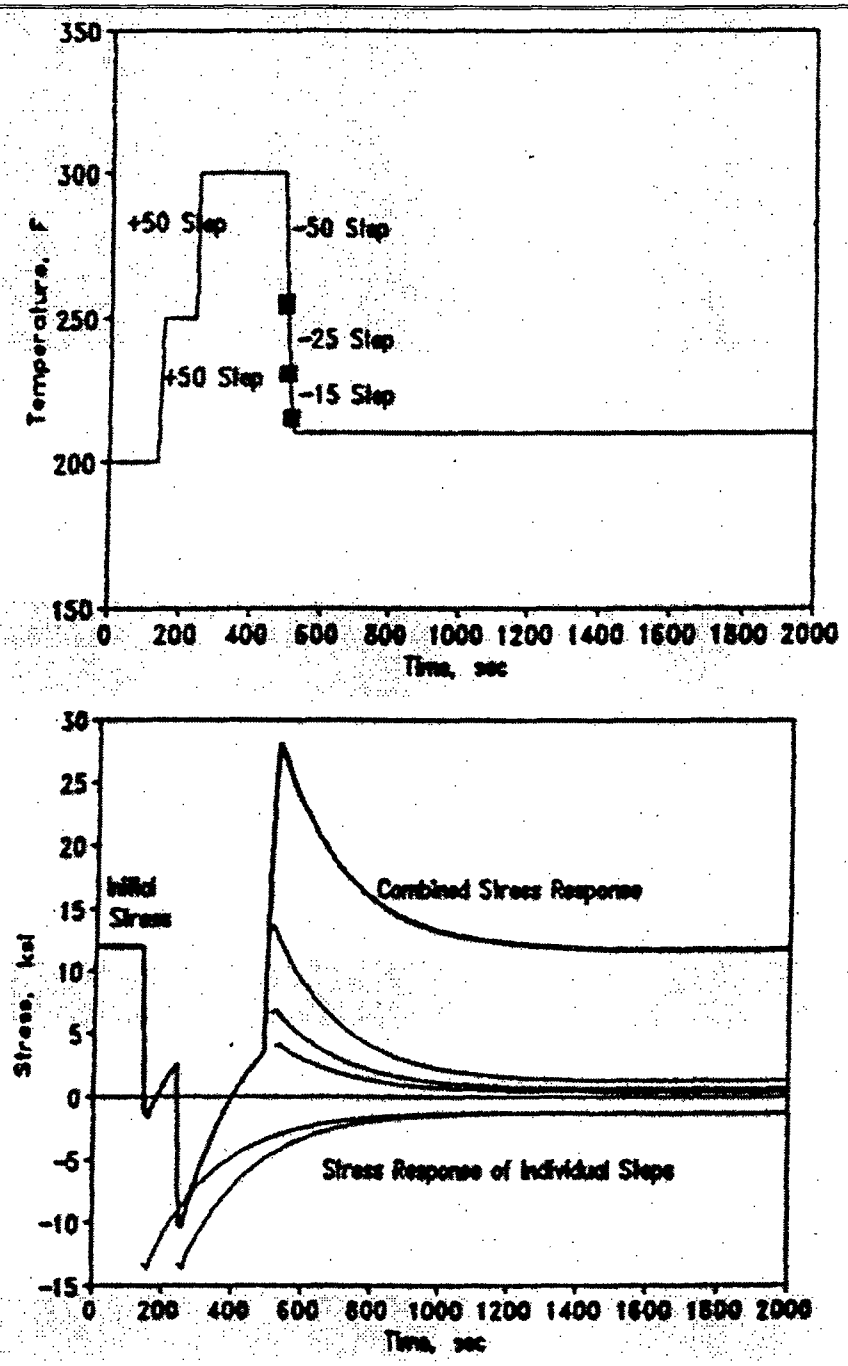


Figure 14: Typical Stress Response Using Green's Functions

APPENDIX A

SUMMARY OF OUTPUT FILES

VY_RON_T_T9.INP	Input File for Transient 9 Thermal Analysis	In Computer files
VY_RON_S_T9.INP	Input File for Transient 9 Stress Analysis	In Computer files
LFSE.OUT	Stress Output at Safe End	In Computer files
LFBR.OUT	Stress Output at Blend Radius	In Computer files
LFSE_INSIDE.RED	Stress Extracted at Safe End	In Computer files
LFBR_INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
LFSE_T.XLS	Stress Results with Total Stress at Safe End	In Computer files
LFSE_M+B.XLS	Stress Results with Membrane plus Bending Stress at Safe End	In Computer files
LFBR_T.XLS	Stress Results with Total Stress at Blend Radius	In Computer files
LFBR_M+B.XLS	Stress Results with Membrane plus Bending Stress at Blend Radius	In Computer files
T9SE.OUT	Transient 9 Safe End stress output	In Computer files
T9BR.OUT	Transient 9 Blend Radius stress output	In Computer files
T9SE_Inside.RED	Transient 9 Stress Extracted at Safe End	In Computer files
T9BR_Inside.RED	Transient 9 Stress Extracted at Blend Radius	In Computer files
T9BR_M+B.xls	Transient 9 Stress Results with Membrane plus Bending Stress at Blend Radius	In Computer files
T9BR_T.xls	Transient 9 Stress Results with Total Stress at Blend Radius	In Computer files
T9SE_M+B.xls	Transient 9 Stress Results with Membrane plus Bending Stress at Safe End	In Computer files
T9SE_T.xls	Transient 9 Stress Results with Total Stress at Safe End	In Computer files
FATIGUE.OUT	Output file from FATIGUE.EXE	In Computer files
FATIGUE.inp	Input file for FATIGUE.EXE	In Computer files
TRANSNT_XX.inp	Input files for STRESS.EXE	In Computer files
P-V_XX.OUT	Output file from P-V.EXE	In Computer files

Structural Integrity Associates, Inc.	File No.: VY-16Q-307 EXHIBIT G
CALCULATION PACKAGE	**Project No.:** VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Class 1 Piping Fatigue and EAF Analysis

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-16 A1-A51 B1-B5 Computer Files	Initial Issue	T. J. Herrmann 07/27/2007	R.V. Perry 07/27/2007 K.R. Evon 07/27/2007 P. Hirschberg 07/27/2007 C.J. Fourcade 07/27/2007

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1.0 OBJECTIVE

The purpose of this calculation is to perform an ASME Section III, NB-3600 fatigue usage calculation (including environmental fatigue) for the Loop A NUREG/CR-6260 locations in the Reactor Recirculation (RR) and Residual Heat Removal (RHR) piping.

The fatigue calculation performed herein is not a certified ASME Code NB-3600 stress and fatigue analysis. Rather, it is an evaluation for the purposes of establishing fatigue usage to accommodate fatigue monitoring of the subject B31.1 piping. Although the PIPESTRESS program implements all ASME Code NB-3600 equations, only the fatigue usage results are utilized. All stress limit checks, although calculated by the program, are ignored since satisfactory stress limit checks were performed as a part of the already existing governing B31.1 stress analyses for all piping systems.

2.0 METHODOLOGY

2.1 Background

Since ASME Section III Design Specifications do not exist for the subject piping systems, SI developed transient definitions and expected number of cycles for the subject piping in a previous evaluation. These definitions are based on SI's experience in piping analysis at other BWR plants, as well as review of VY-specific operating procedures, and are appropriate for BWR-4 plants and tailored specifically to VY. Those transient definitions will reflect current plant operating conditions as shown in references [7 through 10]. Using the PIPESTRESS computer code [5], heat transfer analysis will be performed for the transients defined to establish the necessary parameters for use in an NB-3600 fatigue evaluation. This will result in a detailed usage factor calculation for the RR and RHR NUREG/CR-6260 locations from which to base the environmental fatigue evaluation.

2.2 Design Transients and Fatigue Analysis

The temperature time histories are obtained from the reactor thermal cycle diagrams [7] [8]. These diagrams also provide the changes in flow rate and system pressures. These temperatures and pressures were updated to account for EPU [9].

The computer program PIPESTRESS [5] was used, which is a full function, verified piping analysis package. The ASME Code methodology for fatigue analysis of Class 1 piping systems requires determination of the through-wall thermal gradient terms ΔT_1 (linear gradient), ΔT_2 (nonlinear gradient), and T_a-T_b (transition gradient) for each transient containing a non-trivial ramp rate. PIPESTRESS calculates these terms for each thermal transient. Load sets were then developed for the critical time points of the transients, that include loads due to pressure, thermal expansion, OBE seismic, and thermal gradient stresses. PIPESTRESS was then used to determine the range of primary plus secondary plus peak stresses for each load set pair, and calculate the cumulative fatigue usage for the design numbers of cycles.

3.0 ASSUMPTIONS/DESIGN INPUTS

The Code of construction for VY is ANSI B31.1, 1967 Edition [3, 10]. In order to take advantage of improvements in the ASME Code that result in a lower calculated fatigue usage, this evaluation is done to the ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda [1]. The 1998 Edition of Section III (with 2000 Addenda) has been accepted by the US NRC for use in design analyses. Although there are a few restrictions on the application of this Edition, they involve the use of optional increased allowables that are not being used in this calculation.

The piping analysis input information was based on references provided by VY. The ADLPIPE input file [6] was the source for the piping geometry, and pipe support locations and types. Additional piping support information was obtained from plant drawings [15]. The pipe size, schedule, insulation, and weight per foot, were obtained from [3] (page 10). The flow element located between the pump and RHR return tee was not included in the model. The weight of the element would have no significant impact on the analysis and the element is remote from any areas of severe thermal transients such as the RHR return tee. The weight of the contents was automatically added by the PIPESTRESS program. The design temperature and piping material was obtained from reference [3] (page 9). Table 1 summarizes the material properties used in this analysis.

Reference [6] contains an SSE response spectrum. This spectrum was conservatively used as the OBE spectrum in this analysis. Code case N-411 damping is utilized and directional loading is combined by SRSS [3] (page 20).

Per Reference [9] (Item 14, section 3.2.1), the normal recirculation flow per loop, post EPU, is 12.3Mlbm/hr (at 526°F). Flow is converted to gpm as follows:

$$Q = 12{,}300{,}000\frac{lbm}{hr}\left(\frac{1ft^3}{47.45lbm}\right)\left(\frac{7.48gal}{ft^3}\right)\left(\frac{1hr}{60\min}\right) = 32{,}316gpm$$

Where flow is stopped, a flow rate that gives an equivalent natural convection heat transfer coefficient is calculated.

The applicable transients to consider for the RR and RHR systems are shown in the thermal cycle diagrams [7] and [8]. Level C transients are not required to be included in the fatigue analysis per NB-3224.4. Reference [3] describes which transients are considered level C. Note that a transient for RHR initiation is not accounted for on these diagrams. In order to account for this transient, RHR temperature data from RFO 25 [11] was used to conservatively determine an appropriate temperature change while reference [12] was used to determine flow rates and pressures. Table 2 describes each section and Figure 1 shows the piping model with node numbers. Table 3 contains a list of applicable transients. (Note that the transient RHR initiation contains a section 3B. This section accounts for the portion of the recirculation pump discharge piping that is affected by this transient.) OBE cycles are not listed in Table 3 but are included as Load Set 26 for +OBE and Load Set 27 for –OBE. A review of shutdown cooling mode operation since the recirculation piping was replaced in 1986 was performed by the station and the number of cycles per loop was conservatively estimated to be 150 through year 60 [10]. Based on this, the cycle counts for the Recirculation piping were reduced by a factor of 150/300 (50%) for all transients with the exception of transients that have fewer than 10 transient cycles.

To ensure this cycle reduction adequately considered the potential impact on carbon steel RHR piping, the full number of transient cycles [7] was initially applied to the PIPESTRESS model and the highest CUF for the carbon steel portion of the RHR piping, which has not been replaced, was lower than the value obtained for the recirculation piping with reduced cycles. The Recirculation and RHR line sizes are specified in reference [3] and are shown in Table 4.

Table 1: Material Properties [1] [3]

ASTM A-106 Grade B (C-Si)							
Temperature (°F)	Young's Modulus ($x10^6$ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft^2/hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	29.5	27.5	0.529	0.00	6.40	20.0	35.0
100	29.3	27.6	0.512	0.20		20.0	35.0
200	28.8	27.6	0.486	1.00		20.0	32.1
300	28.3	27.2	0.453	1.90		20.0	31.0
400	27.7	26.7	0.428	2.80		20.0	29.9
500	27.3	25.9	0.398	3.70		18.9	28.5
600	26.7	25.0	0.374	4.70		17.3	26.8

ASME SA-376 TP 316 (16Cr-12Ni-2Mo)							
Temperature (°F)	Young's Modulus ($x10^6$ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft^2/hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	28.3	8.2	0.139	0.00	8.50	20.0	30.0
100	28.1	8.3	0.140	0.30		20.0	30.0
200	27.6	8.8	0.145	1.40		20.0	25.9
300	27.0	9.3	0.150	2.50		20.0	23.4
400	26.5	9.8	0.155	3.70		19.3	21.4
500	25.8	10.2	0.160	5.00		18.0	20.0
600	25.3	10.7	0.165	6.30		17.0	18.9

ASME SA-403 WP 316 (16Cr-12Ni-2Mo)							
Temperature (°F)	Young's Modulus ($x10^6$ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft^2/hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	28.3	8.2	0.139	0.00	8.50	20.0	30.0
100	28.1	8.3	0.140	0.30		20.0	30.0
200	27.6	8.8	0.145	1.40		20.0	25.9
300	27	9.3	0.150	2.50		20.0	23.4
400	26.5	9.8	0.155	3.70		18.7	21.4
500	25.8	10.2	0.160	5.00		17.5	20.0
600	25.3	10.7	0.165	6.30		16.4	18.9

The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions.

Table 2: Recirculation and RHR Piping Segment Numbers

Piping Region	Node Points		Description
	Start	End	
1	3	500	Outlet
2	500	50	Pump suction
3	150	210	Pump discharge
3B*	188	210	Down Stream of RHR Return
4	210	340	Inlet Header
	210	320	Inlet Header
5A	340	365	Riser
5B	340	345	Riser
5C	210	334	Riser
5D	320	325	Riser
5E	320	315	Riser
6A	365	366	Inlet Nozzle
6B	345	346	Inlet Nozzle
6C	334	336	Inlet Nozzle
6D	325	326	Inlet Nozzle
6E	315	316	Inlet Nozzle
7A	500	550	RHR Supply; tee to valve
7B	550	565	RHR Supply; valve to penetration
8	152	176	4" Bypass
9A	600	660	RHR Return; valve to tee
9B	660	675	RHR Return; penetration to valve

*Only applicable for RHR initiation

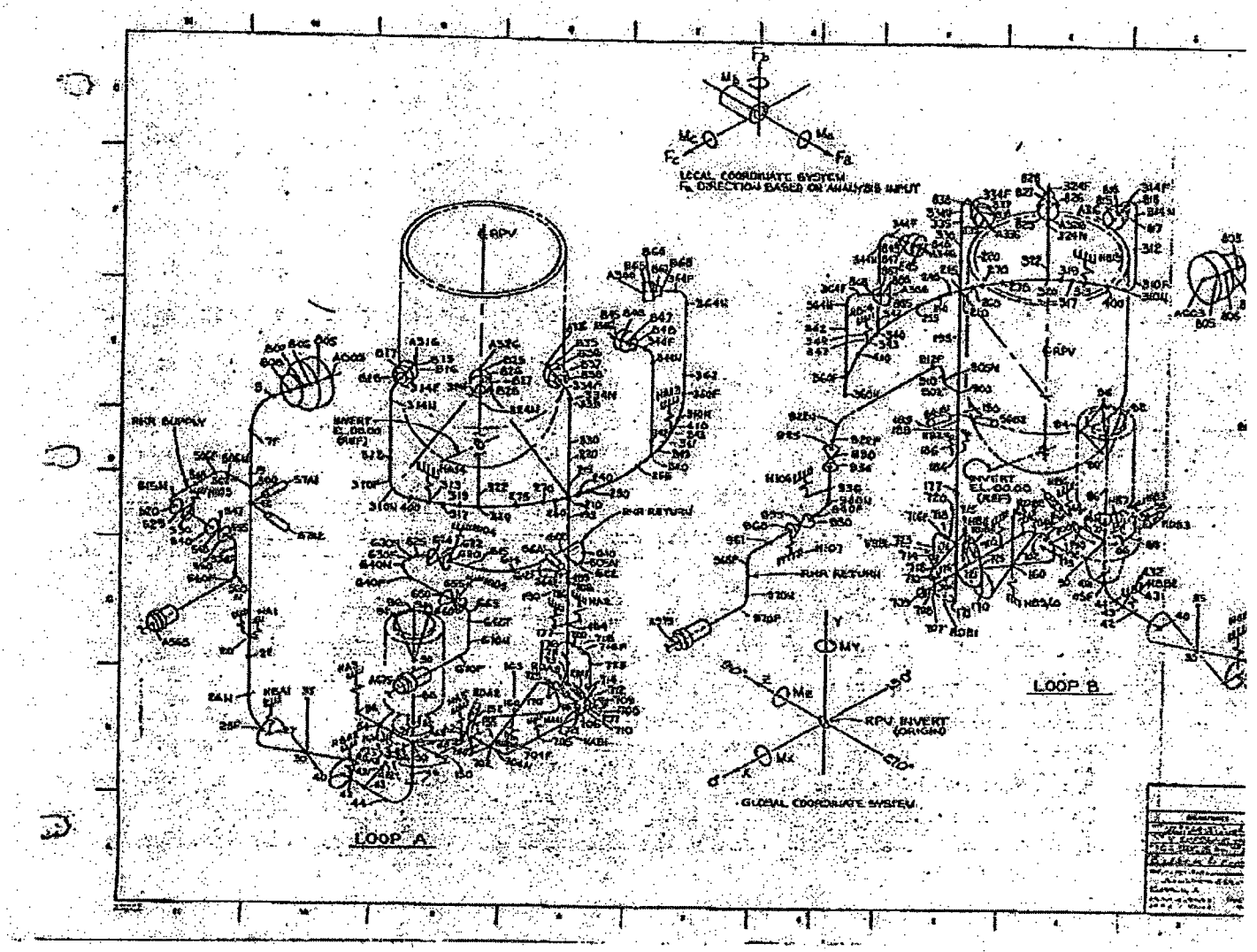

Figure 1. Recirculation and RHR Piping Diagram

Table 3: VY Thermal Transients

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10] Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Pressure Initial (psig)	Pressure Final (psig)	No. of Cycles [10]
1	Design Hydrotest (Leak Test) +	1	100	70	100	1800	60	2,262	0	1,100	60
		2	100	70	100	1800	60	2,262	0	1,100	
		3	100	70	100	1800	60	2,262	0	1,100	
		4	100	70	100	1800	60	905	0	1,100	
		5	100	70	100	1800	60	452	0	1,100	
		6	100	70	100	1800	60	452	0	1,100	
		7A	100	70	100	1800	60	0	0	1,100	
		7B	100	70	100	1800	60	0	0	120	
		8	100	70	100	1800	60	0	0	1,100	
		9A	100	70	100	1800	60	0	0	1,100	
		9B	100	70	100	1800	60	0	0	1,100	
2	Design Hydrotest (Leak Test) -	1	100	100	100	1800	0	2,262	1,100	50	60
		2	100	100	100	1800	0	2,262	1,100	50	
		3	100	100	100	1800	0	2,262	1,100	50	
		4	100	100	100	1800	0	905	1,100	50	
		5	100	100	100	1800	0	452	1,100	50	
		6	100	100	100	1800	0	452	1,100	50	
		7A	100	100	100	1800	0	0	1,100	50	
		7B	100	100	100	1800	0	0	120	50	
		8	100	100	100	1800	0	0	1,100	50	
		9A	100	100	100	1800	0	0	1,100	50	
		9B	100	100	100	1800	0	0	1,100	50	
3	Startup	1	549	100	549	16164	100	16,158	50	1,010	150
		2	549	100	549	16164	100	16,158	50	1,010	
		3	549	100	549	16164	100	16,158	50	1,035	
		4	549	100	549	16164	100	6,463	50	1,035	
		5	549	100	549	16164	100	3,232	50	1,035	
		6	549	100	549	16164	100	3,232	50	1,035	
		7A	549	100	549	16164	100	300	50	1,010	
		7B	150	100	150	16164	0	0	50	120	
		8	549	100	549	16164	100	168	50	1,035	
		9A	549	100	549	16164	100	0	50	1,035	
		9B	150	100	150	16164	11	0	50	1,035	
4	Turbine Roll & Increase to Rated Power + SCRAM - 1	1	542	549	542	0	STEP	32,316	1,010	1,010	290
		2	542	549	542	0	STEP	32,316	1,010	1,010	
		3	542	549	542	0	STEP	32,316	1,035	1,035	
		4	542	549	542	0	STEP	12,926	1,035	1,035	
		5	542	549	542	0	STEP	6,463	1,035	1,035	
		6	542	549	542	0	STEP	6,463	1,035	1,035	
		7A	542	549	542	0	STEP	364	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	542	549	542	0	STEP	335	1,035	1,035	
		9A	542	549	542	0	STEP	520	1,035	1,035	
		9B	150	150	150	0	STEP	0	1,035	1,035	
5	Turbine Roll & Increase to Rated Power + SCRAM - 2	1	526	542	526	0	STEP	32,316	1,010	1,010	290
		2	526	542	526	0	STEP	32,316	1,010	1,010	
		3	526	542	526	0	STEP	32,316	1,035	1,035	
		4	526	542	526	0	STEP	12,926	1,035	1,035	
		5	526	542	526	0	STEP	6,463	1,035	1,035	
		6	526	542	526	0	STEP	6,463	1,035	1,035	
		7A	526	542	526	0	STEP	358	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	526	542	526	0	STEP	335	1,035	1,035	
		9A	526	542	526	0	STEP	511	1,035	1,035	
		9B	150	150	150	0	STEP	0	1,035	1,035	
6	Loss of Feedwater Heater, Turbine Trip (+)	1	542	526	542	900	64	32,316	1,010	1,010	5 x 2
		2	542	526	542	900	64	32,316	1,010	1,010	
		3	542	526	542	900	64	32,316	1,035	1,035	
		4	542	526	542	900	64	12,926	1,035	1,035	
		5	542	526	542	900	64	6,463	1,035	1,035	
		6	542	526	542	900	64	6,463	1,035	1,035	
		7A	542	526	542	900	64	358	1,010	1,010	
		7B	150	150	150	900	0	0	120	120	
		8	542	526	542	900	64	335	1,035	1,035	
		9A	542	526	542	900	64	511	1,035	1,035	
		9B	150	150	150	900	0	0	1,035	1,035	
7	Loss of Feedwater Heater, Turbine Trip (-)	1	526	542	526	360	160	32,316	1,010	1,010	5 x 2
		2	526	542	526	360	160	32,316	1,010	1,010	
		3	526	542	526	360	160	32,316	1,035	1,035	
		4	526	542	526	360	160	12,926	1,035	1,035	
		5	526	542	526	360	160	6,463	1,035	1,035	
		6	526	542	526	360	160	6,463	1,035	1,035	
		7A	526	542	526	360	160	358	1,010	1,010	
		7B	150	150	150	360	0	0	120	120	
		8	526	542	526	360	160	335	1,035	1,035	
		9A	526	542	526	360	160	511	1,035	1,035	
		9B	150	150	150	360	0	0	1,035	1,035	

Table 3: VY Thermal Transients (continued)

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10] Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Pressure Initial (psig)	Pressure Final (psig)	No. of Cycles [10]
8	Loss of Feedwater Heater, Partial FW Heater Bypass (-)	1	516	526	516	0	STEP	32,316	1,010	1,010	35
		2	516	526	516	0	STEP	32,316	1,010	1,010	
		3	516	526	516	0	STEP	32,316	1,035	1,035	
		4	516	526	516	0	STEP	12,926	1,035	1,035	
		5	516	526	516	0	STEP	6,463	1,035	1,035	
		6	516	526	516	0	STEP	6,463	1,035	1,035	
		7A	516	526	516	0	STEP	351	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	516	526	516	0	STEP	335	1,035	1,035	
		9A	516	526	516	0	STEP	502	1,035	1,035	
		9B	150	150	150	0	STEP	0	1,035	1,035	
9	Loss of Feedwater Heater, Partial FW Heater Bypass (+)	1	526	516	526	0	STEP	32,316	1,010	1,010	35
		2	526	516	526	0	STEP	32,316	1,010	1,010	
		3	526	516	526	0	STEP	32,316	1,035	1,035	
		4	526	516	526	0	STEP	12,926	1,035	1,035	
		5	526	516	526	0	STEP	6,463	1,035	1,035	
		6	526	516	526	0	STEP	6,463	1,035	1,035	
		7A	526	516	526	0	STEP	351	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	526	516	526	0	STEP	335	1,035	1,035	
		9A	526	516	526	0	STEP	502	1,035	1,035	
		9B	150	150	150	0	STEP	0	1,035	1,035	
10	Loss of Feedwater Pumps (Isolation Valves Close) 1st step down	1	300	526	300	220	3698	600	1,190	1,135	5
		2	300	526	300	220	3698	600	1,190	1,135	
		3	300	526	300	220	3698	600	1,215	1,160	
		4	300	526	300	220	3698	400	1,215	1,160	
		5	300	526	300	220	3698	200	1,215	1,160	
		6	300	526	300	220	3698	200	1,215	1,160	
		7A	300	526	300	220	3698	306	1,190	1,135	
		7B	150	150	150	0.01	0	0	120	120	
		8	300	526	300	220	3698	6	1,215	1,160	
		9A	300	526	300	220	3698	437	1,215	1,160	
		9B	150	150	150	0.01	0	0	1,215	1,160	
11	Loss of Feedwater Pumps (Isolation Valves Close) 1st & 2nd step up	1	500	300	500	1980	364	600	885	1,135	5 X 2
		2	500	300	500	1980	364	600	885	1,135	
		3	500	300	500	1980	364	600	910	1,160	
		4	500	300	500	1980	364	400	910	1,160	
		5	500	300	500	1980	364	200	910	1,160	
		6	500	300	500	1980	364	200	910	1,160	
		7A	500	300	500	1980	364	301	885	1,135	
		7B	150	150	150	0.01	0	0	120	120	
		8	500	300	500	1980	364	6	910	1,160	
		9A	500	300	500	1980	364	429	910	1,160	
		9B	150	150	150	0.01	0	0	910	1,160	
12	Loss of Feedwater Pumps (Isolation Valves Close) 2nd & 3rd step down	1	300	500	300	180	4000	600	1,135	675	5 X 2
		2	300	500	300	180	4000	600	1,135	675	
		3	300	500	300	180	4000	600	1,160	700	
		4	300	500	300	180	4000	400	1,160	700	
		5	300	500	300	180	4000	200	1,160	700	
		6	300	500	300	180	4000	200	1,160	700	
		7A	300	500	300	180	4000	301	1,135	675	
		7B	150	150	150	0.01	0	0	120	121	
		8	300	500	300	180	4000	6	1,160	700	
		9A	300	500	300	180	4000	429	1,160	700	
		9B	150	150	150	0.01	0	0	1,160	700	
13	Loss of Feedwater Pumps (Isolation Valves Close) last step up	1	549	300	549	8964	100	16,158	240	1,010	5
		2	549	300	549	8964	100	16,158	240	1,010	
		3	549	300	549	8964	100	16,158	265	1,035	
		4	549	300	549	8964	100	6,463	265	1,035	
		5	549	300	549	8964	100	3,232	265	1,035	
		6	549	300	549	8964	100	3,232	265	1,035	
		7A	549	300	549	8964	100	310	240	1,010	
		7B	150	150	150	8964	100	0	120	120	
		8	549	300	549	8964	100	168	265	1,035	
		9A	549	300	549	8964	100	443	265	1,035	
		9B	150	150	150	8967	100	0	265	1,035	
14	Reduction to 0% Power	1	549	526	549	0	STEP	32,316	1,010	1,010	150
		2	549	526	549	0	STEP	32,316	1,010	1,010	
		3	549	526	549	0	STEP	32,316	1,035	1,035	
		4	549	526	549	0	STEP	12,926	1,035	1,035	
		5	549	526	549	0	STEP	6,463	1,035	1,035	
		6	549	526	549	0	STEP	6,463	1,035	1,035	
		7A	549	526	549	0	STEP	360	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	549	526	549	0	STEP	335	1,035	1,035	
		9A	549	526	549	0	STEP	514	1,035	1,035	
		9B	150	150	150	0	STEP	0	1,035	1,035	

Table 3: VY Thermal Transients (continued)

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10] Oper. Temp. (°F)	T_{bulk} (°F)	T_{fluid} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Pressure Initial (psig)	Pressure Final (psig)	No. of Cycles [10]
15	Shutdown 1	1	375	549	375	6264	100	16,158	1,010	170	150
		2	375	549	375	6264	100	16,158	1,010	170	
		3	375	549	375	6264	100	16,158	1,035	195	
		4	375	549	375	6264	100	6,463	1,035	195	
		5	375	549	375	6264	100	3,232	1,035	195	
		6	375	549	375	6264	100	3,232	1,035	195	
		7A	375	549	375	6264	100	320	1,035	170	
		7B	150	150	150	0.01	0	0	120	120	
		8	375	549	375	6264	100	168	1,035	195	
		9A	375	549	375	6264	100	458	1,035	195	
		9B	150	150	150	0.01	0	0	1,035	195	
16	Shutdown 2	1	330	375	330	600	270	16,158	170	90	150
		2	330	375	330	600	270	16,158	170	90	
		3	330	375	330	600	270	16,158	195	115	
		4	330	375	330	600	270	6,463	195	115	
		5	330	375	330	600	270	3,232	195	115	
		6	330	375	330	600	270	3,232	195	115	
		7A	330	375	330	600	270	282	170	90	
		7B	150	150	150	600	0	0	120	90	
		8	330	375	330	600	270	168	195	115	
		9A	330	375	330	600	270	403	195	115	
		9B	150	150	150	600	0	0	195	115	
17	Shutdown 3	1	225	330	225	3780	100	16,158	90	0	150
		2	225	330	225	3780	100	16,158	90	0	
		3	225	330	225	3780	100	16,158	115	25	
		4	225	330	225	3780	100	6,463	115	25	
		5	225	330	225	3780	100	3,232	115	25	
		6	225	330	225	3780	100	3,232	115	25	
		7A	225	330	225	3780	100	260	90	0	
		7B	150	150	150	0	0	0	90	0	
		8	225	330	225	3780	100	168	115	25	
		9A	225	330	225	3780	100	260	115	25	
		9B	150	150	150	0	0	0	115	25	
18	Shutdown 4 (RHR SDC Flow is on)	1	100	225	100	4500	100	22,858	0	0	150
		2	100	225	100	4500	100	16,158	0	0	
		3	100	225	100	4500	100	16,158	25	25	
		3B	100	225	100	4500	100	22,858	25	25	
		4	100	225	100	4500	100	9,143	25	25	
		5	100	225	100	4500	100	4,572	25	25	
		6	100	225	100	4500	100	4,572	25	25	
		7A	100	225	100	4500	100	6,700	0	0	
		7B	100	225	100	4500	100	6,700	0	0	
		8	100	225	100	4500	100	168	25	25	
		9A	100	225	100	4500	100	6,700	100	100	
		9B	100	225	100	4500	100	6,700	100	100	
19	Code Hydro	1	100	100	100	0.01	0	2,262	25	1,563	1
		2	100	100	100	0.01	0	2,262	25	1,563	
		3	100	100	100	0.01	0	2,262	25	1,563	
		4	100	100	100	0.01	0	905	25	1,563	
		5	100	100	100	0.01	0	452	25	1,563	
		6	100	100	100	0.01	0	452	25	1,563	
		7A	100	100	100	0.01	0	158	25	1,563	
		7B	100	100	100	0.01	0	0	0	450	
		8	100	100	100	0.01	0	23	25	1,563	
		9A	100	100	100	0.01	0	226	25	1,563	
		9B	100	100	100	0.01	0	0	25	1,563	
20	RHR Initiation (+)	1	225	225	225	0.01	0	22,858	0	0	150
		2	225	225	225	0.01	0	16,158	0	0	
		3	225	225	225	0.01	0	16,158	25	25	
		3B	225	180	225	60	2700	22,858	25	25	
		4	225	180	225	60	2700	9,143	25	25	
		5	225	180	225	60	2700	4,572	25	25	
		6	225	180	225	60	2700	4,572	25	25	
		7A	225	225	225	60	0	6,700	0	0	
		7B	225	150	225	60	4500	6,700	0	0	
		8	225	225	225	0.01	0	237	25	25	
		9A	225	70	225	60	9300	6,700	25	25	
		9B	225	70	225	60	9300	6,700	25	25	
21	RHR Initiation (-)	1	225	225	225	0.01	0	22,858	0	0	150
		2	225	225	225	0.01	0	16,158	0	0	
		3	225	225	225	0.01	0	16,158	25	25	
		3B	180	225	180	60	2700	22,858	25	25	
		4	180	225	180	60	2700	9,143	25	25	
		5	180	225	180	60	2700	4,572	25	25	
		6	180	225	180	60	2700	4,572	25	25	
		7A	225	225	225	60	0	6,700	0	0	
		7B	150	150	150	0.01	0	6,700	0	0	
		8	225	225	225	0.01	0	237	25	25	
		9A	70	225	70	60	9300	6,700	25	25	
		9B	70	150	70	60	9290	6,700	25	25	

Table 4: Recirc/RHR Piping Size Information [3]

Regions	1, 2	3	4	5	6	7A, 7B	8	9A, 9B
Piping Nom. O.D. (in.)	28.169	28.339	21.878	12.748	14.17	20	4.5	24
Piping Nom. Wall (in.)	1.244	1.339	1.043	0.685	1.395	1.031	0.3385	1.217
Pipe Weight[1] (lb/ft)	386.1	415.1	257.2	103.4	207.5	221.9	23.2	316.5

Note:

1. Weight of contents automatically added by the PIPESTRESS Program.

4.0 ANALYSIS

Through-wall thermal gradient terms were calculated by the PIPESTRESS program for all of the transients. Thermal transient cases were modeled for each transient, as shown in Table 3. Some transients were similar in nature and were lumped together and the number of cycles added together. Listings of the PIPESTRESS input files are included as Appendix A.

The forces and moments due to thermal expansion need to be included in the fatigue evaluation. The thermal expansion cases as analyzed by the piping program, PIPESTRESS, correspond to the end temperature and pressure of the transient. Table 5 lists the thermal expansion cases.

The material properties were obtained from the ASME Code Section III, 1998 Edition, Appendix I, with 2000 Addenda [1]. E and α are taken at 70°F, and k, ρ, and c_p are taken at the average temperature over the range of the individual transients.

The internal heat transfer coefficient h for the transients with flow occurring in the pipe is calculated based on the following relation for forced convection [13]:

$$h = 0.023\ Re^{0.8}\ Pr^{0.4}\ k/D$$

Where Re = Reynolds number
Pr = Prandtl number
k = Thermal conductivity
D = Pipe diameter

The heat transfer coefficients were calculated by PIPESTRESS using the above relation. The flow rates described for each transient in Section 3 were used. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The formula for h is [13]:

$$h = 0.55\,(Gr\,Pr)^{0.25}k/L$$

Where Gr = Grashof Number
L = Pipe diameter

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same heat transfer coefficient as the free convection coefficient.

Since the replacement of the Recirculation piping [10], HWC conditions exist for 39% of the time, and NWC conditions exist for 61% of the time. This is based on 17.5 years of operation with NWC between March and July 1986 when the piping was replaced and November 2003 when HWC was implemented and the 46 years from March 1986 to the end of the period of extended operation in March 2032. Using the bounding EAF multipliers (8.36 for HWC and 15.35 for NWC) [14], an overall multiplier may be calculated as follows:

$$(15.35)0.61 + (8.36)0.39 = 12.62$$

Table 5: Thermal Cycle Load Sets

Load Set	Transient Case	Region Temperatures (°F)												Region Pressures (psig)			
		1	2	3	3B	4	5	6	7A	7B	8	9A	9B	1, 2, 7A	3, 4, 5, 6, 8, 9A, 9B	7B	3B
1	1	100	100	100	-	100	100	100	100	100	100	100	100	1,100	1,100	120	-
2	2	100	100	100	-	100	100	100	100	100	100	100	100	50	50	50	-
3	3	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
4	4	542	542	542	-	542	542	542	542	150	542	542	150	1,010	1,035	120	-
5	5	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
6	6	542	542	542	-	542	542	542	542	150	542	542	150	1,010	1,035	120	-
7	7	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
8	8	516	516	516	-	516	516	516	516	150	516	516	150	1,010	1,035	120	-
9	9	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
10	10	300	300	300	-	300	300	300	300	150	300	300	150	1,135	1,160	120	-
11	11	500	500	500	-	500	500	500	500	150	500	500	150	1,135	1,160	120	-
12	12	300	300	300	-	300	300	300	300	150	300	300	150	675	700	121	-
13	13	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
14	14	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
15	15	375	375	375	-	375	375	375	375	150	375	375	150	170	195	120	-
16	16	330	330	330	-	330	330	330	330	150	330	330	150	90	115	90	-
17	17	225	225	225	-	225	225	225	225	150	225	225	150	0	25	0	-
18	18	100	100	100	100	100	100	100	100	100	100	100	100	0	25	0	25
19	19	100	100	100	-	100	100	100	100	100	100	100	100	1,563	1,563	450	-
20	20	225	225	225	225	225	225	225	225	225	225	225	225	0	25	0	25
21	21	225	225	225	180	180	180	180	225	150	225	70	70	0	25	0	25

5.0 RESULTS OF ANALYSIS

To perform the fatigue analysis, program PIPESTRESS [5] was used. PIPESTRESS calculates the thermal expansion and seismic moments, the ASME Code Equation 10, 12, and 13 stresses, performs the thermal stress ratchet check, and performs fatigue analysis per Equation 11 and 14. For each operating state of the recirculation/RHR piping, load sets are created. A load set includes the coincident pressure, thermal expansion moment, through-wall thermal gradient terms, number of cycles, and temperature at which the allowable S_m is taken. In general, the pressures and thermal expansion moments are taken at the end point of the transient, the thermal gradients taken at the point of maximum total thermal gradient stress during the transient, and the S_m allowable is initially conservatively taken at the highest temperature of the transient. Table 5 lists the inputs to the load sets.

In calculating fatigue, the range of stress in going from one load set to another is determined. Since the Code assumes that any transient could follow any other, all pairs of load sets are evaluated to determine the range of stresses for the Code stress equations. The number of allowable cycles for each load set pair is determined. The incremental fatigue usage is obtained by dividing the number of design cycles by the allowable cycles. The incremental fatigue usages for all load set pairs are then summed to obtain the total fatigue usage.

The cumulative fatigue usage for the Loop A recirculation RHR return isolation valve-to-pipe location (Node 641), prior to considering environmental effects, is 0.0128. Taking into account environmental effects, the bounding multiplier for stainless steel is 12.62. This results in a total fatigue usage of 0.1615. (Note that since the RHR carbon steel piping has not been replaced, these results represent the full projected 60 year cycle count.)

The cumulative fatigue usage for the RHR return tee (Node 600), prior to considering environmental effects, is 0.0590. Taking into account environmental effects, the bounding multiplier for stainless steel is 12.62. This results in a total fatigue usage of 0.7446.

Appendix A contains the PIPESTRESS input files. Appendix B contains the fatigue usage summary for both locations.

6.0 REFERENCES

1. ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda.
2. ASME Boiler and Pressure Vessel Code, Section XI, 1998 Edition.
3. Vermont Yankee Calculation 23A5569, "Recirculation System Stress Analysis Loop A", Revision 0, SI File No. VY-05Q-227.
4. Email from Jim Fitzpatrick (Entergy) to Terry Herrmann (SI), "RE: RHR Thermal Transients," dated: June 29, 2007 11:19AM, SI File Number VY-09Q-209.
5. Program PIPESTRESS, Version 3.5.1+26, DST Computer Services, S.A., June 2004.
6. ADLPIPE Model Input Listing, Vermont Yankee Calculation VYC-2030, Rev. 0, "Temporary Shielding Recirculation & RHR Piping Loop A," File c2030n2, SI File No. W-VY-05Q-227.
7. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
8. "Nozzle Thermal Cycles (Recirculation Outlet)," Attachment 1, page 4, of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
9. Entergy Nuclear Report VY-RPT-05-00022, "Task T0100 Reactor Heat Balance EPU Task Report for ER-0401409", Revision 0, SI File No. VY-16Q-205.
10. Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
11. "RHR Shutdown Cooling Temperature Data," page 8, of Entergy Design Input Record (DIR) EC No. 1773, Revision 1, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
12. "RHR Shutdown Cooling Flow Rate and Pressure Data," page 9, of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
13. Holman, J.P., *Heat Transfer*, Fifth Edition, McGraw-Hill, 1981.

14. SI Calculation, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head," Revision 0, SI File Number VY-16Q-303.

15. VY Drawings, SI File No. VY-16Q-205:

    a. 5920-6801, Sheet 1, Revision 1.
    b. 5920-6802, Sheet 1, Revision 2, Sheet2, Revision 2, Sheet 3, Revision 3, Sheet 4, Revision 2, Sheet 5, Revision 2, Sheet 6, Revision 2.
    c. 5920-6808 Sheet 1, Revision 0.

APPENDIX A

PIPESTRESS Input Files

Input File	Description
Recirc_15.fre	Piping model and general input for reduced cycle count
RHR_15.fre	Piping model and general input for 60 year cycle count
Reg1.inp	Region 1 transient definitions
Reg2.inp	Region 2 transient definitions
Reg3.inp	Region 3 transient definitions
Reg4.inp	Region 4 transient definitions
Reg5.inp	Region 5 transient definitions
Reg6.inp	Region 6 transient definitions
Reg7A.inp	Region 7A transient definitions
Reg7B.inp	Region 7B transient definitions
Reg8.inp	Region 8 transient definitions
Reg9A.inp	Region 9A transient definitions
Reg9B.inp	Region 9B transient definitions

Recirc 15.fre

```
IDEN JB=3    *Job number (1 to 9999)
     CD=1    *1=ASME Class 1
     GR=-Y   *Direction of gravity
     VA=0    *0=Calculate            2=Verify
     IU=1    *Input units            1=USA
     OU=1    *Output units           1=USA
     CH=$    *Delimiter character
     AB=T    *FREE errors = abort
     PL=$Vermont Yankee$
     EN=$RVP$
TITL BL=3    *Modeling option:
             *  3 = uniform mass for static analysis
             *      lumped mass for dynamic analysis
             *      rotational inertia ignored
     GL=1    *Report forces/moment        0=Global     1=Local     2=G et L
     SU=1    *Support summary             0=No         1=Yes
     CV=15   *Code version - See Manual
     HS=1    *Highest 20 stress ratios for each case
     MD=1    *Hot modulus
     J6=1    *File generated by program
     TI=$Vermont Yankee Recirculation $
        $Fatigue Analysis$
FREQ RF=1 RP=8 FR=36 MP=20 RC=0 MX=70 TI=$SEISMIC$
*
*************************************
**** THERMAL CYCLE LOAD CASES****
*************************************
LCAS RF=0 CA=1 TY=0 TI=$LC-1$      *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$      *TC-2
LCAS RF=0 CA=3 TY=0 TI=$LC-3$      *TC-3
LCAS RF=0 CA=4 TY=0 TI=$LC-4$      *TC-4
LCAS RF=0 CA=5 TY=0 TI=$LC-5$      *TC-5
LCAS RF=0 CA=6 TY=0 TI=$LC-6$      *TC-6
LCAS RF=0 CA=7 TY=0 TI=$LC-7$      *TC-7
LCAS RF=0 CA=8 TY=0 TI=$LC-8$      *TC-8
LCAS RF=0 CA=9 TY=0 TI=$LC-9$      *TC-9
LCAS RF=0 CA=10 TY=0 TI=$LC-10$    *TC-10
LCAS RF=0 CA=11 TY=0 TI=$LC-11$    *TC-11
LCAS RF=0 CA=12 TY=0 TI=$LC-12$    *TC-12
LCAS RF=0 CA=13 TY=0 TI=$LC-13$    *TC-13
LCAS RF=0 CA=14 TY=0 TI=$LC-14$    *TC-14
LCAS RF=0 CA=15 TY=0 TI=$LC-15$    *TC-15
LCAS RF=0 CA=16 TY=0 TI=$LC-16$    *TC-16
LCAS RF=0 CA=17 TY=0 TI=$LC-17$    *TC-17
LCAS RF=0 CA=18 TY=0 TI=$LC-18$    *TC-18
LCAS RF=0 CA=19 TY=0 TI=$LC-19$    *TC-19
LCAS RF=0 CA=20 TY=0 TI=$LC-20$    *TC-20
LCAS RF=0 CA=21 TY=0 TI=$LC-21$    *TC-21
LCAS RF=0 CA=22 TY=0 TI=$LC-22$    *TC-22
LCAS RF=0 CA=23 TY=0 TI=$LC-23$    *TC-23
LCAS RF=0 CA=24 TY=0 TI=$LC-24$    *TC-24
LCAS RF=0 CA=25 TY=0 TI=$LC-25$    *TC-25
*
*********************
**** WEIGHT CASES****
*********************
LCAS CA=101  RF=1  TY=3  TI=$OPERATING WEIGHT$
LCAS CA=102  RF=2  TY=4  TI=$HYDROTEST WEIGHT$
*
```

```
*********************************
**** THERMAL TRANSIENT CASES****
*********************************
TCAS CA=201 TI=$Design Hydrotest (+) $
TCAS CA=202 TI=$Design Hydrotest (-) $
TCAS CA=203 TI=$Startup              $
TCAS CA=204 TI=$TRoll & Inc. PWR1    $
TCAS CA=205 TI=$TRoll & Inc. PWR2    $
TCAS CA=206 TI=$LOFWH+TT PWR1        $
TCAS CA=207 TI=$LOFWH+TT PWR2        $
TCAS CA=208 TI=$LOFWH+PFWHTR Byp1    $
TCAS CA=209 TI=$LOFWH+PFWHTR Byp2    $
TCAS CA=210 TI=$LOFWP, ISO Cl DN 1   $
TCAS CA=211 TI=$LOFWP, ISO Cl UP 1   $
TCAS CA=212 TI=$LOFWP, ISO Cl DN 2   $
TCAS CA=213 TI=$LOFWP, ISO Cl UP 2   $
TCAS CA=214 TI=$Reduction to 0% PWR  $
TCAS CA=215 TI=$Shutdown1            $
TCAS CA=216 TI=$Shutdown2            $
TCAS CA=217 TI=$Shutdown3            $
TCAS CA=218 TI=$Shutdown4            $
TCAS CA=219 TI=$Code Hydrotest       $
TCAS CA=220 TI=$RHR Initiation UP    $
TCAS CA=221 TI=$RHR Initiation DN    $
TCAS CA=222 TI=$Inadvert. Inj. DOWN  $
TCAS CA=223 TI=$Inadvert. Inj. UP    $
TCAS CA=224 TI=$Single Relief BD DN  $
TCAS CA=225 TI=$Single Relief BD UP  $
*
**********************
**** SEISMIC CASES****
**********************
RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=$OBE INERTIA$
*
** *************************
**** LOAD COMBINATION CASES *
** *************************
CCAS RF=1 CA=104      ME=1 FL=1  C1=103 CY=10  TI=$OBE$
CCAS RF=1 CA=401 SS=1 ME=1 EQ=3  C1=101 C2=103 TI=$EQUATION 9 LEVEL B$
CCAS RF=1 CA=402 SS=1 ME=3 F1=1  C1=103 C2=1   TI=$NORMAL+OBE$
CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=1   TI=$NORMAL-OBE$
*
*****************
**** LOAD SETS****
*****************
*
LSET RF=1  FC=0 RP=1 CY=60   PR=1  MO=1  TR=201  TI=$Design Hydrotest (+)LS-1$
LSET RF=2  FC=0 RP=1 CY=60   PR=2  MO=2  TR=-202 TI=$Design Hydrotest (-)LS-2$
LSET RF=3  FC=0 RP=1 CY=150  PR=3  MO=3  TR=203  TI=$Startup            LS-3$
LSET RF=3  FC=0 RP=1 CY=290  PR=4  MO=4  TR=-204 TI=$TRoll & Inc. PWR1  LS-4$
LSET RF=4  FC=0 RP=1 CY=290  PR=5  MO=5  TR=-205 TI=$TRoll & Inc. PWR2  LS-5$
LSET RF=4  FC=0 RP=1 CY=10   PR=6  MO=6  TR=206  TI=$LOFWH+TT PWR1      LS-6$
LSET RF=4  FC=0 RP=1 CY=10   PR=7  MO=7  TR=-207 TI=$LOFWH+TT PWR2      LS-7$
LSET RF=5  FC=0 RP=1 CY=35   PR=8  MO=8  TR=-208 TI=$LOFWH+PFWHTR Byp1  LS-8$
LSET RF=5  FC=0 RP=1 CY=35   PR=9  MO=9  TR=209  TI=$LOFWH+PFWHTR Byp2  LS-9$
LSET RF=5  FC=0 RP=1 CY=5    PR=10 MO=10 TR=-210 TI=$LOFWP, ISO Cl DN 1 LS-10$
LSET RF=11 FC=0 RP=1 CY=10   PR=11 MO=11 TR=211  TI=$LOFWP, ISO Cl UP 1 LS-11$
LSET RF=11 FC=0 RP=1 CY=10   PR=12 MO=12 TR=-212 TI=$LOFWP, ISO Cl DN 2 LS-12$
LSET RF=3  FC=0 RP=1 CY=5    PR=13 MO=13 TR=213  TI=$LOFWP, ISO Cl UP 2 LS-13$
LSET RF=3  FC=0 RP=1 CY=150  PR=14 MO=14 TR=214  TI=$Reduction to 0% PWR LS-14$
```

```
LSET RF=5  FC=0 RP=1 CY=150  PR=15 MO=15 TR=-215 TI=$Shutdown1          LS-15$
LSET RF=15 FC=0 RP=1 CY=150  PR=16 MO=16 TR=-216 TI=$Shutdown2          LS-16$
LSET RF=16 FC=0 RP=1 CY=150  PR=17 MO=17 TR=-217 TI=$Shutdown3          LS-17$
LSET RF=20 FC=0 RP=1 CY=150  PR=18 MO=18 TR=-218 TI=$Shutdown4          LS-18$
LSET RF=19 FC=0 RP=1 CY=1    PR=19 MO=19 TR=219  TI=$Code Hydrotest     LS-19$
LSET RF=20 FC=0 RP=1 CY=150  PR=20 MO=20 TR=220  TI=$RHR Initiation UP  LS-20$
LSET RF=20 FC=0 RP=1 CY=150  PR=21 MO=21 TR=-221 TI=$RHR Initiation DN  LS-21$
LSET RF=5  FC=0 RP=1 CY=0    PR=22 MO=22 TR=-222 TI=$Inadvert. Inj. DOWN LS-22$
LSET RF=5  FC=0 RP=1 CY=0    PR=23 MO=23 TR=223  TI=$Inadvert. Inj. UP  LS-23$
LSET RF=23 FC=0 RP=1 CY=0    PR=24 MO=24 TR=-224 TI=$Single Relief BD DN LS-24$
LSET RF=24 FC=0 RP=1 CY=0    PR=25 MO=25 TR=225  TI=$Single Relief BD UP LS-25$
*
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=402 TI=$NORMAL+OBE  LS-26$
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=403 TI=$NORMAL-OBE  LS-27$
******
*FATG AT=500 AF=502
*FATG AT=600 AF=602
*
**************************
**** RESPONSE SPECTRA****
**************************
SPEC FS=OBE EV=1 ME=3 FP=0  TI=$RESPONSE$
     LV=1 DX=1 DY=1 DZ=1
     DI=X
       0.30/0.100  0.40/0.100  0.90/0.200  1.25/0.400  2.25/0.450  2.30/0.700
       3.30/0.700  4.40/0.750  4.41/0.900  4.75/1.100  5.20/1.100  5.80/1.600
       8.70/1.600 12.00/0.650 17.00/0.400 20.00/0.350 30.00/0.350 36.00/0.350
     DI=Y
       0.30/0.030  0.40/0.030  0.50/0.050  0.60/0.075  1.00/0.075  1.20/0.100
       2.00/0.220  2.40/0.350  3.50/0.350  3.60/0.300  5.30/0.300  5.75/0.330
       8.25/0.330  8.75/0.250 17.50/0.250 25.00/0.120 30.00/0.120 36.00/0.120
     DI=Z
       0.30/0.100  0.40/0.100  0.50/0.130  0.90/0.150  1.00/0.250  1.60/0.250
       1.90/0.600  3.50/0.600  3.75/0.700  4.40/0.700  4.50/0.800  6.25/1.500
       8.50/1.500 12.50/0.500 20.00/0.350 30.00/0.350 36.00/0.350
*
****************************
**** MATERIAL PROPERTIES ****
****************************
* ASTM A-106 Grade B, PIPE *
MATH CD=106      EX=0       TY=1      *C-Si
MATD TE=70       EH=29.5    EX=0.0    SM=20.0   SY=35
MATD TE=100      EH=29.3    EX=0.20   SM=20.0   SY=35
MATD TE=200      EH=28.8    EX=1.00   SM=20.0   SY=32.1
MATD TE=300      EH=28.3    EX=1.90   SM=20.0   SY=31
MATD TE=400      EH=27.7    EX=2.80   SM=20.0   SY=29.9
MATD TE=500      EH=27.3    EX=3.70   SM=18.9   SY=28.5
MATD TE=600      EH=26.7    EX=4.70   SM=17.3   SY=26.8
* ASME SA-376 Grade TP316, PIPE *
MATH CD=376.316 EX=0        TY=4      *16Cr-12Ni-2Mo
MATD TE=70       EH=28.3    EX=0.0    SM=20.0   SY=30.0
MATD TE=100      EH=28.1    EX=0.30   SM=20.0   SY=30.0
MATD TE=200      EH=27.6    EX=1.40   SM=20.0   SY=25.9
MATD TE=300      EH=27.0    EX=2.50   SM=20.0   SY=23.4
MATD TE=400      EH=26.5    EX=3.70   SM=19.3   SY=21.4
MATD TE=500      EH=25.8    EX=5.00   SM=18.0   SY=20.0
MATD TE=600      EH=25.3    EX=6.30   SM=17.0   SY=18.9
* ASME SA-403 Grade WP316, ELBOWS *
MATH CD=403.316 EX=0        TY=4      *16Cr-12Ni-2Mo
MATD TE=70       EH=28.3    EX=0.0    SM=20.0   SY=30.0
```

```
MATD TE=100     EH=28.1    EX=0.30  SM=20.0   SY=30.0
MATD TE=200     EH=27.6    EX=1.40  SM=20.0   SY=25.9
MATD TE=300     EH=27.0    EX=2.50  SM=20.0   SY=23.4
MATD TE=400     EH=26.5    EX=3.70  SM=18.7   SY=21.4
MATD TE=500     EH=25.8    EX=5.00  SM=17.5   SY=20.0
MATD TE=600     EH=25.3    EX=6.30  SM=16.4   SY=18.9
*** Cross Sectional Properties
CROS CD=1  OD=50.0    WT=8.87    MA=3977.2 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0               *RECIRCULATION OUTLET NOZZLE
CROS CD=2  OD=37.85   WT=6.1     MA=2122.2 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=3  OD=28.875  WT=1.56    MA=484.9  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=4  OD=28.638  WT=1.45    MA=450.4  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=5  OD=28.169  WT=1.244   MA=386.1  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=7  OD=28.166  WT=2.125   MA=0.001  *VALVE
           SO=1       ST=1.0     KL=1
CROS CD=8  OD=42.507  WT=2.486   MA=0.001  *PUMP
           SO=.001    ST=.001    KL=1
CROS CD=11 OD=6.625   WT=0.432   MA=0.001  *PUMP RIGID STRUTS
           SO=0.001   ST=0.001   KL=1
CROS CD=13 OD=28.339  WT=1.339   MA=415.1  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=14 OD=28.339  WT=2.67    MA=0.001  *VALVE
           SO=1       ST=1.0     KL=1
CROS CD=15 OD=12.748  WT=0.685   MA=103.4  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=16 OD=14.17   WT=1.395   MA=207.5  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=17 OD=15.5    WT=2       MA=307.7  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=18 OD=21.88   WT=4.06    MA=803.2  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=19 OD=28.25   WT=7.25    MA=1673.1 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=20 OD=21.878  WT=1.043   MA=257.2  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=25 OD=20      WT=1.031   MA=221.9  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=26 OD=20      WT=1.875   MA=0.001  *VALVE
           SO=1       ST=1       KL=1
CROS CD=27 OD=4.5     WT=0.3385  MA=23.2   *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1       KL=1      *4 inch bypass line
CROS CD=28 OD=4.5     WT=0.67    MA=0.001  *VALVE V2-54A
           SO=1       ST=1       KL=1
CROS CD=29 OD=24      WT=1.217   MA=316.5  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=30 OD=24      WT=2.43    MA=0.001  *VALVE
           SO=1       ST=1       KL=1
CROS CD=40 OD=4.5     WT=0.3385  MA=0.001  *4 inch bypass STRUTS
           SO=0.001   ST=0.001   KL=1
CROS CD=41 OD=2.875   WT=0.276   MA=0.001  *STRUT RDA1, RDA5, & VBA1
           SO=0.001   ST=0.001   KL=1
CROS CD=42 OD=28.339  WT=1.339   MA=0.001  *RIGID FROM RECIRC ELBOW TO RDA1 STRUT
           SO=0.001   ST=0.001   KL=1
**********************
* STRUCTURE AND LOADS
**********************
```

```
*-----------------------------------------------------------------------------
DESN  TE=575.0  PR=1250.0 *Reference 12 GE Design Requirements Rpt VY-05Q-227
*-----------------------------------------------------------------------------
*-----------------------------------------------------------------------
*BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE
*-----------------------------------------------------------------
*
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG1.INP
*
* RUN   1      FROM ANCHOR TO REACTOR VESSEL N3B
*GROUP 1 FROM ANCHOR TO REACTOR VESSEL N3B
*NOTE
*NOTE NODE 003 - RECIRC SUCTION NOZZLE N1A (EL. 279'5 INCH)
*NOTE NODE 003 IS AT THE SAFE END TO VESSEL NOZZLE CONNECTION
*NOTE
*NOTE SAFE END FROM NODES 003 TO 808
*NOTE CONNECTION TO VESSEL AT NODE 003
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT FOR SAFE END BASED ON THICKNESS
*NOTE
MATL CD=376.316
CROS CD=1
COOR PT=3    AX=0  AY=0  AZ=0
ANCH PT=3
AMVT    CA=1    PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=2    PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=3    PT=3    DX=0.0000  DY=0.3112  DZ=-0.3568
AMVT    CA=4    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=5    PT=3    DX=0.0000  DY=0.3112  DZ=-0.3568
AMVT    CA=6    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=7    PT=3    DX=0.0000  DY=0.2922  DZ=-0.3350
AMVT    CA=8    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=9    PT=3    DX=0.0000  DY=0.1422  DZ=-0.1630
AMVT    CA=10   PT=3    DX=0.0000  DY=0.2807  DZ=-0.3218
AMVT    CA=11   PT=3    DX=0.0000  DY=0.1422  DZ=-0.1630
AMVT    CA=12   PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=13   PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=14   PT=3    DX=0.0000  DY=0.1928  DZ=-0.2521
AMVT    CA=15   PT=3    DX=0.0000  DY=0.1624  DZ=-0.1986
AMVT    CA=16   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=17   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=18   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=19   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=20   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=21   PT=3    DX=0.0000  DY=0.0361  DZ=-0.0413
AMVT    CA=22   PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=23   PT=3    DX=0.0000  DY=0.1928  DZ=-0.2521
AMVT    CA=24   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201


*
TANG PT=805  DZ=-1.017  EW=1
CROS CD=2
TANG PT=806  DZ=-0.823  EW=1
CROS CD=3
TANG PT=807  DZ=-0.58  EW=1
CROS CD=4
TANG PT=808  DZ=-0.47
CROS CD=5
TANG PT=5    DZ=-5.59 EW=1
```

```
MATL CD=403.316
BRAD PT=7    RA=3.5 EW=1
MATL CD=376.316
TANG PT=9    DY=-6.69  EW=1
TANG PT=500  DY=-2.31
*-----------------------------------------------
*END REGION 1 GEOMETRY FROM RHR SUPPLY TO TEE
*-----------------------------------------------
*
*-----------------------------------------------------------------------------
*BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----------------------------------------------------------------------------
*GROUP 2 RHR SUPPLY TEE TO PUMP
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG2.INP
*
TANG PT=11   DY=-2.22  EW=1
CROS CD=5
TANG PT=12   DY=-1.78
TANG PT=20   DY=-6.77
TANG PT=22   DY=-3.25
TANG PT=25   DY=-15.49  EW=1
MATL CD=403.316
BRAD PT=26   RA=3.5  EW=1
MATL CD=376.316
TANG PT=27   DX=-3.3 DZ=1.27  EW=1
CROS CD=7
VALV PT=30   DX=-2.28 DZ=0.89 MA=10.368 PL=1
JUNC PT=30
VALV PT=40   DX=-2.31 DZ=0.9 PL=2  EW=1
JUNC PT=30
RIGD PT=35   DY=7
LUMP PT=35   MA=1.132
JUNC PT=40
CROS CD=5
TANG PT=42 DX=-1.18 DZ=0.46
TANG PT=43 DX=-0.55 DZ=0.21
TANG PT=44 DX=-3.31 DZ=1.28  EW=1
MATL CD=403.316
BRAD PT=46 RA=2.33  EW=1
MATL CD=376.316
CROS CD=8
TANG PT=50 DY=4.33  EW=0
LUMP PT=50 MA=28  *NOTE WEIGHT OF PUMP FLOODED 28K (EXCLUDING MOTOR)
TANG PT=75 DY=0.5
TANG PT=83 DY=2.13
TANG PT=86 DY=3.38
LUMP PT=86 MA=32  *NOTE TOTAL WEIGHT OF PUMP MOTOR 32000 LBS
TANG PT=90 DY=4.08 *TOP OF PUMP
*NOTE SNUBBERS ON TOP OF PUMPS WERE DELETED DURING
*NOTE THE RECIRC PIPE REPLACEMENT PROJECT
*NOTE - RIGID LINKS FOR CONSTANT SUPPORTS AT PUMP FOLLOW
*---------------------------
*END REGION 2 GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*---------------------------
*
*----------------------------------------
*BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
*----------------------------------------
*GROUP 3 FROM PUMP DISCHARGE TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3.INP
```

```
*
JUNC PT=50
CROS CD=8
RIGD PT=54 DX=1.06 DZ=1.06
RIGD PT=56 DX=1.06 DY=0.75 DZ=1.06  *NOTE CONSTANT SUPPORT HA3 AT NODE 56
JUNC PT=50
RIGD PT=66 DZ=-3.83
RIGD PT=69 DY=1                     *NOTE CONSTANT SUPPORT HA4 AT NODE 69
JUNC PT=50
CROS CD=8
RIGD PT=60 DX=-3.83
RIGD PT=63 DY=1                     *CONSTANT SUPPORT HA5 AT NODE 63
*
* *** CODING FOR PUMP RIGID STRUTS FOLLOW ***
*     CODED FROM PUMP CENTERLINE
CROS CD=11
JUNC PT=66
RIGD PT=15    DY=0.7071 DZ=-0.7071
*
JUNC PT=60
RIGD PT=16    DX=-0.7071 DY=0.7071
* *** END OF CODING FOR PUMP SUPPORTS ***
*PUMP INLET
CROS CD=8
JUNC PT=50
TANG PT=150 DX=-2.17
BRAN PT=151 DZ=2.333  TE=1
*NOTE PUMP DISCHARGE CONNECTION TO PIPE AT NODE 151
CROS CD=13
TANG PT=152 DZ=1.25
TANG PT=155 DZ=1  EW=1
CROS CD=14
VALV PT=160 PL=1 DX=0.0 DY=0.0  DZ=2.52 MA=6.8285
JUNC PT=160
RIGD PT=163 DX=0.0 DY=7.12 DZ=0.0
LUMP PT=163 MA=0.9715
JUNC PT=160
VALV PT=170 PL=2 DX=0.0 DY=0.0 DZ=6.18  EW=1
CROS CD=13
MATL CD=403.316
BRAD PT=175 RA=3.5  EW=1
MATL CD=376.316
TANG PT=176 DY=5.95
TANG PT=177 DY=4.42
*NOTE ***WEIGHT OF FLOW ELEMENT NOT INCLUDED***
*NOTE ***REF. DWG. 5920-6800 FOR DIMENSIONS***
TANG PT=184 DY=4.42
TANG PT=186 DY=3.02
TANG PT=188 DY=1.51
TANG PT=189 DY=0.74
TANG PT=190 DY=1.15  EW=1
TANG PT=600 DY=1.06

***INPUT FILE TO INCLUDE EFFECTS OF RHR INITIATION ON LINE NEAR RHR RETURN TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3B.INP

JUNC PT=600
TANG PT=195 DY=2.08  EW=1
TANG PT=210 DX=0.0  DY=1.83 DZ=0.0  KL=1 *CENTER OF CROSS, RECIRC HEADER
*MUST HAVE INDI CARD FOR EACH MEMBER CONNECTED TO CROSS CENTER
```

```
*---------------------------
*END REGION 3 GEOMETRY FROM PUMP DISCHARGE TO HEADER
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 336
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
*NOTE CROSS AND REDUCER DIMENSIONS TAKEN FROM 5920-6632 SHT.3
CROS CD=13
MATL CD=376.316
TANG PT=215 DX=0.0 DY=2.59  DZ=0.0  EW=0
CRED PT=220 DY=1.29  AN=30  EW=1  *AL=$CONC. REDUCER$
CROS CD=15
TANG PT=330 DY=4.58
TANG PT=335 DY=3.29  EW=1
MATL CD=403.316
BRAD PT=334 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 336
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 336
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=838 DX=3.875
CROS CD=16
TANG PT=837 DX=0.875  EW=1
CROS CD=17
TANG PT=836 DX=0.37  EW=1
CROS CD=18
TANG PT=835 DX=0.53  EW=1
CROS CD=19
TANG PT=336 DX=0.704  EW=1
NOZZ PT=336
AMVT    CA=1    PT=336  DX=-0.0201  DY=0.0246  DZ=0.0000
AMVT    CA=2    PT=336  DX=-0.3602  DY=0.4398  DZ=0.0000
AMVT    CA=3    PT=336  DX=-0.3568  DY=0.4316  DZ=0.0000
AMVT    CA=4    PT=336  DX=-0.3434  DY=0.4152  DZ=0.0000
AMVT    CA=5    PT=336  DX=-0.3568  DY=0.4050  DZ=0.0000
AMVT    CA=6    PT=336  DX=-0.3434  DY=0.2940  DZ=0.0000
AMVT    CA=7    PT=336  DX=-0.3350  DY=0.3229  DZ=0.0000
AMVT    CA=8    PT=336  DX=-0.3434  DY=0.2700  DZ=0.0000
AMVT    CA=9    PT=336  DX=-0.1630  DY=0.1991  DZ=0.0000
AMVT    CA=10   PT=336  DX=-0.3218  DY=0.1626  DZ=0.0000
AMVT    CA=11   PT=336  DX=-0.1630  DY=0.0246  DZ=0.0000
AMVT    CA=12   PT=336  DX=-0.3602  DY=0.4398  DZ=0.0000
AMVT    CA=13   PT=336  DX=-0.3602  DY=0.4316  DZ=0.0000
AMVT    CA=14   PT=336  DX=-0.2193  DY=0.4152  DZ=0.0000
AMVT    CA=15   PT=336  DX=-0.1862  DY=0.4050  DZ=0.0000
AMVT    CA=16   PT=336  DX=-0.1084  DY=0.2940  DZ=0.0000
AMVT    CA=17   PT=336  DX=-0.0201  DY=0.3229  DZ=0.0000
AMVT    CA=18   PT=336  DX=-0.0201  DY=0.2700  DZ=0.0000
AMVT    CA=19   PT=336  DX=-0.1084  DY=0.1991  DZ=0.0000
```

```
AMVT    CA=20    PT=336  DX=-0.0201  DY=0.1626  DZ=0.0000
AMVT    CA=21    PT=336  DX=-0.0413  DY=0.3229  DZ=0.0000
AMVT    CA=22    PT=336  DX=-0.3434  DY=0.2700  DZ=0.0000
AMVT    CA=23    PT=336  DX=-0.2211  DY=0.1991  DZ=0.0000
AMVT    CA=24    PT=336  DX=-0.0201  DY=0.1626  DZ=0.0000


*NOTE SAFE END FROM NODES 838 TO 336
*NOTE CONNECTION TO VESSEL AT NODE 336
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT BASED ON THICKNESS
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 336
*---------------------------
*
*----------------------------------------
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*----------------------------------------
*GROUP 4 HEADER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*
JUNC PT=210
CROS CD=20
BRAN PT=240 DX=0.1786 DY=0.0 DZ=1.7
TANG PT=250 DX=0.3 DZ=2.853  EW=0
BRAD PT=255 RA=4.578  EW=0 *NOTE BEND RADIUS IS 4.578 FEET
TANG PT=340 DX=1.799 DZ=3.108
*---------------------------
*END REGION 4 GEOMETRY HEADER TO NOZZLE NODE 366
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 366
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
TANG PT=349 DX=0.71 DZ=1.23  EW=0
CRED PT=347 DX=0.75 DZ=1.3  AN=30
CROS CD=15
TANG PT=343 DX=0.5525 DZ=0.957  EW=1
BRAD PT=410 RA=1.5  EW=1
TANG PT=360 DX=3.483 DZ=2.011  EW=1
MATL CD=403.316
BRAD PT=361 RA=1.5  EW=1
MATL CD=376.316
CROS CD=15
TANG PT=362 DY=3.18
TANG PT=364 DY=8.56  EW=1
MATL CD=403.316
BRAD PT=365 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 366
*---------------------------

*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 366
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
```

```
*
MATL CD=376.316
TANG PT=868 DX=1.8 DZ=-3.1
CROS CD=16
TANG PT=867 DX=0.4375 DZ=-0.76  EW=1
CROS CD=17
TANG PT=866 DX=0.185 DZ=-0.32  EW=1
CROS CD=18
TANG PT=865 DX=0.265 DZ=-0.46  EW=1
CROS CD=19
TANG PT=366 DX=0.352 DZ=-0.61  EW=1
NOZZ PT=366
AMVT    CA=1    PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=2    PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=3    PT=366  DX=-0.1783  DY=0.4357  DZ=0.3091
AMVT    CA=4    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=5    PT=366  DX=-0.1783  DY=0.4357  DZ=0.3091
AMVT    CA=6    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=7    PT=366  DX=-0.1674  DY=0.4091  DZ=0.2902
AMVT    CA=8    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=9    PT=366  DX=-0.0815  DY=0.1991  DZ=0.1412
AMVT    CA=10   PT=366  DX=-0.1609  DY=0.3930  DZ=0.2788
AMVT    CA=11   PT=366  DX=-0.0815  DY=0.1991  DZ=0.1412
AMVT    CA=12   PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=13   PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=14   PT=366  DX=-0.1097  DY=0.2678  DZ=0.1899
AMVT    CA=15   PT=366  DX=-0.0931  DY=0.2275  DZ=0.1613
AMVT    CA=16   PT=366  DX=-0.0542  DY=0.1324  DZ=0.0939
AMVT    CA=17   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=18   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=19   PT=366  DX=-0.0542  DY=0.1324  DZ=0.0939
AMVT    CA=20   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=21   PT=366  DX=-0.0207  DY=0.0505  DZ=0.0358
AMVT    CA=22   PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=23   PT=366  DX=-0.1105  DY=0.2700  DZ=0.1915
AMVT    CA=24   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174

*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 366
*---------------------------

*----------------------------------------
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*----------------------------------------
*GROUP 4 HEADER TO NOZZLES NODE 326 & 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*
JUNC PT=210
CROS CD=20
BRAN PT=260 DX=0.1786 DY=0.0 DZ=-1.7 TE=2
TANG PT=270 DX=0.3 DZ=-2.853  EW=0
BRAD PT=275 RA=4.578  EW=0
TANG PT=320 DX=1.799 DZ=-3.108
*---------------------------
*END REGION 4 GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*---------------------------

*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 316
*----------------------------------------
```

```
*GROUP 5 RISER TO NOZZLE NODE 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
TANG PT=319 DX=0.71 DZ=-1.23  EW=1
CRED PT=317 DX=0.75 DZ=-1.3  AN=30
CROS CD=15
TANG PT=313 DX=0.5525 DZ=-0.957  EW=1
BRAD PT=400 RA=1.5  EW=1
TANG PT=310 DX=3.483 DZ=-2.011  EW=1
MATL CD=403.316
BRAD PT=311 RA=1.5  EW=1
MATL CD=376.316
CROS CD=15
TANG PT=312 DY=4.74
TANG PT=314 DY=6.99  EW=1
MATL CD=403.316
BRAD PT=315 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 316
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 316
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=818 DX=1.84 DZ=3.19
CROS CD=16
TANG PT=817 DX=0.4375 DZ=0.76  EW=1
CROS CD=17
TANG PT=816 DX=0.185 DZ=0.32  EW=1
CROS CD=18
TANG PT=815 DX=0.265 DZ=0.46  EW=1
CROS CD=19
TANG PT=316 DX=0.352 DZ=0.61  EW=1
NOZZ PT=316
AMVT    CA=1    PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=2    PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
AMVT    CA=3    PT=316  DX=-0.1783  DY=0.4357  DZ=-0.3091
AMVT    CA=4    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=5    PT=316  DX=-0.1783  DY=0.4357  DZ=-0.3091
AMVT    CA=6    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=7    PT=316  DX=-0.1674  DY=0.4091  DZ=-0.2902
AMVT    CA=8    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=9    PT=316  DX=-0.0815  DY=0.1991  DZ=-0.1412
AMVT    CA=10   PT=316  DX=-0.1609  DY=0.3930  DZ=-0.2788
AMVT    CA=11   PT=316  DX=-0.0815  DY=0.1991  DZ=-0.1412
AMVT    CA=12   PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
AMVT    CA=13   PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
AMVT    CA=14   PT=316  DX=-0.1097  DY=0.2678  DZ=-0.1899
AMVT    CA=15   PT=316  DX=-0.0931  DY=0.2275  DZ=-0.1613
AMVT    CA=16   PT=316  DX=-0.0542  DY=0.1324  DZ=-0.0939
AMVT    CA=17   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=18   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=19   PT=316  DX=-0.0542  DY=0.1324  DZ=-0.0939
AMVT    CA=20   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=21   PT=316  DX=-0.0207  DY=0.0505  DZ=-0.0358
AMVT    CA=22   PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
```

```
AMVT    CA=23   PT=316  DX=-0.1105  DY=0.2700  DZ=-0.1915
AMVT    CA=24   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 316
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 346
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=340
CROS CD=15
BRAN PT=342 DY=1.36  TE=2
TANG PT=344 DY=10.39  EW=0
MATL CD=403.316
BRAD PT=345 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 346
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 346
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=848 DX=3.17 DZ=-1.83
CROS CD=16
TANG PT=847 DX=0.758 DZ=-0.4375  EW=1
CROS CD=17
TANG PT=846 DX=0.32 DZ=-0.185  EW=1
CROS CD=18
TANG PT=845 DX=0.46 DZ=-0.265  EW=1
CROS CD=19
TANG PT=346 DX=0.61 DZ=-0.352  EW=1
NOZZ PT=346
AMVT    CA=1    PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=2    PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=3    PT=346  DX=-0.3091  DY=0.4357  DZ=0.1783
AMVT    CA=4    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=5    PT=346  DX=-0.3091  DY=0.4357  DZ=0.1783
AMVT    CA=6    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=7    PT=346  DX=-0.2902  DY=0.4091  DZ=0.1674
AMVT    CA=8    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=9    PT=346  DX=-0.1412  DY=0.1991  DZ=0.0815
AMVT    CA=10   PT=346  DX=-0.2788  DY=0.3930  DZ=0.1609
AMVT    CA=11   PT=346  DX=-0.1412  DY=0.1991  DZ=0.0815
AMVT    CA=12   PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=13   PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=14   PT=346  DX=-0.1899  DY=0.2678  DZ=0.1097
AMVT    CA=15   PT=346  DX=-0.1613  DY=0.2275  DZ=0.0931
AMVT    CA=16   PT=346  DX=-0.0939  DY=0.1324  DZ=0.0542
AMVT    CA=17   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=18   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=19   PT=346  DX=-0.0939  DY=0.1324  DZ=0.0542
AMVT    CA=20   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=21   PT=346  DX=-0.0358  DY=0.0505  DZ=0.0207
```

```
AMVT    CA=22   PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=23   PT=346  DX=-0.1915  DY=0.2700  DZ=0.1105
AMVT    CA=24   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 346
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 326
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=320
CROS CD=15
BRAN PT=322 DY=1.42  TE=2
TANG PT=324 DY=10.33  EW=1
MATL CD=403.316
BRAD PT=325 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 326
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 326
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=828 DX=3.18 DZ=1.84
CROS CD=16
TANG PT=827 DX=0.758 DZ=0.4375  EW=1
CROS CD=17
TANG PT=826 DX=0.32 DZ=0.185  EW=1
CROS CD=18
TANG PT=825 DX=0.46 DZ=0.265  EW=1
CROS CD=19
TANG PT=326 DX=0.61 DZ=0.352  EW=1
NOZZ PT=326
AMVT    CA=1    PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=2    PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=3    PT=326  DX=-0.3091  DY=0.4357  DZ=-0.1783
AMVT    CA=4    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=5    PT=326  DX=-0.3091  DY=0.4357  DZ=-0.1783
AMVT    CA=6    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=7    PT=326  DX=-0.2902  DY=0.4091  DZ=-0.1674
AMVT    CA=8    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=9    PT=326  DX=-0.1412  DY=0.1991  DZ=-0.0815
AMVT    CA=10   PT=326  DX=-0.2788  DY=0.3930  DZ=-0.1609
AMVT    CA=11   PT=326  DX=-0.1412  DY=0.1991  DZ=-0.0815
AMVT    CA=12   PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=13   PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=14   PT=326  DX=-0.1899  DY=0.2678  DZ=-0.1097
AMVT    CA=15   PT=326  DX=-0.1613  DY=0.2275  DZ=-0.0931
AMVT    CA=16   PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT    CA=17   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=18   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=19   PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT    CA=20   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
```

```
AMVT    CA=21   PT=326  DX=-0.0358  DY=0.0505  DZ=-0.0207
AMVT    CA=22   PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=23   PT=326  DX=-0.1915  DY=0.2700  DZ=-0.1105
AMVT    CA=24   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 326
*---------------------------
*
*----------------------------------------
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*----------------------------------------
*GROUP 7 TO RHR SUPPLY VALVE NODE 550
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7A.INP
*
MATL CD=376.316
JUNC PT=500
CROS CD=25
BRAN PT=502 DX=1.67  EW=0  TE=1
TANG PT=506 DX=2.53  EW=0
MATL CD=403.316
BRAD PT=507 RA=1.67  EW=1
MATL CD=376.316
TANG PT=508 DZ=-4.01
TANG PT=515 DZ=-4.53  EW=1
MATL CD=403.316
BRAD PT=520 RA=1.67  EW=1
MATL CD=376.316
CROS CD=26
VALV PT=525 DX=-3.34 PL=1
JUNC PT=525
VALV PT=530 DX=-1.99 PL=2  EW=1
JUNC PT=525
RIGD PT=526 DY=2.5
LUMP PT=526 MA=7.569
JUNC PT=530
CROS CD=25
TANG PT=540 DX=-1.13  EW=1
CROS CD=26
VALV PT=545 DX=-1.97 PL=1
JUNC PT=545
RIGD PT=547 DY=2.5
LUMP PT=547 MA=7.355
JUNC PT=545
VALV PT=550 DX=-1.98 PL=2  EW=1
*---------------------------
*END REGION 7A GEOMETRY TO RHR SUPPLY VALVE NODE 550
*---------------------------
*
*----------------------------------------
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*----------------------------------------
*GROUP 17 FROM RHR SUPPLY VALVE TO PENET. NODE 565
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7B.INP
*
CROS CD=25
MATL CD=106
TANG PT=555 DX=-3.36  EW=1
BRAD PT=556 RA=1.67  EW=1
TANG PT=560 DY=-10.17  EW=1
BRAD PT=561 RA=1.67  EW=1
```

```
TANG PT=563 DZ=-6.92
TANG PT=565 DZ=-6.92
*---------------------------
*END REGION 7B GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*---------------------------
*
*----------------------------------------
*BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
*----------------------------------------
*GROUP 8 4 INCH BYPASS
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG8.INP
*
*NOTE CODING FOR 4 INCH BYPASS STARTS HERE
JUNC PT=152
CROS CD=27
MATL CD=376.316
BRAN PT=700 DX=-1.19  TE=4
TANG PT=702 DX=-0.61
TANG PT=703 DX=-1.43  EW=0
MATL CD=403.316
BRAD PT=704 RA=0.5  EW=0
MATL CD=376.316
TANG PT=705 DZ=5.08
*NOTE CONSTANT SUPPORT HA11 AT NODE 705
TANG PT=721 DZ=1.12
TANG PT=706 DZ=2.47
TANG PT=707 DZ=1.03
TANG PT=708 DZ=0.34
TANG PT=709 DZ=0.38
JUNC PT=707
BRAN PT=710 DY=0.34 TE=1
CROS CD=28
VALV PT=712 DY=0.71 MA=0.3669 PL=1  *AL=$VALVE V2-54A$
VALV PT=715 DZ=-3.5 MA=0.1831 PL=3
JUNC PT=712
VALV PT=714 DY=0.71          PL=2
CROS CD=27
TANG PT=723 DY=4.19
MATL CD=403.316
BRAD PT=716 RA=0.5
MATL CD=376.316
TANG PT=718 DX=1.48
TANG PT=720 DX=0.56
BRAN PT=176 DX=1.19  TE=4
************CODING FOR STRUTS RDA5 AND VAB1 FOLLOW
JUNC PT=170
CROS CD=40  *OD=4.5 inch
RIGD PT=725 DP=0 DX=-0.583 DY=1.84  *AL=$RDA5$
CROS CD=41  *OD=2.875 inch
RIGD PT=715 DP=0 DX=-2.67 DY=-0.79
RIGD PT=721 DP=0 DY=-1.05  *AL=$VAB1$
*************CODING FOR RDA1 STRUT FOLLOWS
CROS CD=42  *OD=28.339 inch
JUNC PT=175
RIGD PT=173 DP=0 DY=-3.5 DZ=0.34
CROS CD=41  *OD=2.875 inch
RIGD PT=708 DP=0 DX=-3.21  *AL=$RDA1$
*---------------------------
*END REGION 8 GEOMETRY  FOR 4 INCH BYPASS
*---------------------------
```

```
*
*----------------------------------------
*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*----------------------------------------
*GROUP 9 RHR RETURN FROM TEE TO VALVE NODE 660
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9A.INP
*
*NOTE CODING FOR RHR RETURN STARTS HERE
CROS CD=29
JUNC PT=600
MATL CD=376.316
BRAN PT=602 DX=-3.8123  TE=1
MATL CD=403.316
BRAD PT=610 RA=2  EW=1
TANP        DY=4
BRAD PT=612 RA=2  EW=1
MATL CD=376.316
TANG PT=614 DZ=-10.38  EW=1
MATL CD=403.316
BRAD PT=615 RA=10  EW=1
MATL CD=376.316
TANG PT=620 DX=5.98 DZ=-3.45  EW=1
*NOTE
*NOTE VARIABLE SPRING H104 AT NODE 620
*NOTE
*NOTE VALVE V10-81A DATA FROM 5920-4590 WEIGHT - 6845.#
*NOTE WEIGHT APPLIED AT ESTIMATED CENTER OF GRAVITY (NODE 623)
CROS CD=30
VALV PT=622 DX=1.98 DZ=-1.15 PL=1  *AL=$VALVE V10-81A$
JUNC PT=622
VALV PT=624 DX=1.98 DZ=-1.15 PL=2  EW=1
JUNC PT=622
RIGD PT=623 DY=2.5
LUMP PT=623 MA=7.32 *VALVE ACTUATOR
CROS CD=29
JUNC PT=624
TANG PT=625 DX=1.867 DZ=-1.078
TANG PT=630 DX=2.598 DZ=-1.5  EW=1
MATL CD=403.316
BRAD PT=631 RA=3  EW=1
MATL CD=376.316
TANG PT=640 DZ=-4.54  EW=1
MATL CD=403.316
BRAD PT=641 RA=2  EW=1
MATL CD=376.316
*NOTE VALVE V10-46A DATA FROM 5920-4718 WEIGHT - 5295.#
CROS CD=30
VALV PT=655 DX=-3.79 PL=1 TA=2 *AL=$VALVE V10-46A$
LUMP PT=655 MA=5.77

*----------------------------
*END REGION 9A GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*----------------------------
*
*----------------------------------------
*BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE
675
*----------------------------------------
*GROUP 19 RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9B.INP
```

```
*
*NOTE
*NOTE VARIABLE SPRING H105 AT NODE 655
*NOTE
VALV PT=660 DX=-1.79 PL=2  EW=1
*NOTE SPEC CHANGE TO CARBON STEEL
MATL CD=106
CROS CD=29
TANG PT=661 DX=-1
TANG PT=663 DX=-3.31  EW=1
BRAD PT=665 RA=2  EW=1
TANG PT=670 DY=-10.5 DZ=0.38  EW=1
BRAD PT=671 RA=2  EW=1
TANG PT=673 DZ=-7.74
TANG PT=675 DZ=-7.74
*---------------------------
*END REGION 9B GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
*---------------------------
*
*---------------------------
***STRESS INDICES AT CROSS POINT
*---------------------------
*
INDI AT=210 AF=195 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=215 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=240 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=260 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
*---------------------------
*** SUPPORTS
*---------------------------
RSTN PT=675  DX=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=675  DY=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=675  DZ=1  SP=23000          *RHR SUPPLY PENET.
ROTR PT=675  RX=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=675  RY=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=675  RZ=1  SP=340000         *RHR SUPPLY PENET.
RSTN PT=565  DX=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=565  DY=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=565  DZ=1  SP=23000          *RHR SUPPLY PENET.
ROTR PT=565  RX=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=565  RY=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=565  RZ=1  SP=340000         *RHR SUPPLY PENET.
*
SNUB PT=12   DZ=-1 SP=1000    *AL=$SNUBBER SS-7A-1$
SNUB PT=12   DX=1  SP=1000    *AL=$SNUBBER SS-7A-2$
SNUB PT=190  DX=-1 SP=1000    *AL=$SNUBBER SS-6-A1$
SNUB PT=190  DZ=1  SP=1000    *AL=$SNUBBER SS-6-A2$
*
VSUP PT=20   DY=1  FO=24.8  SP=2.664  *AL=$VARI. SUPT. HA-1$
*
CSUP PT=27   DY=1  FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A1$
CSUP PT=42   DY=1  FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A2$
CSUP PT=56   DY=1  FO=18.05 KP=0.01   *AL=$CONST. SUPT. HA3 FOR PUMP$
CSUP PT=69   DY=1  FO=18.0  KP=0.01   *AL=$CONST. SUPT. HA4 FOR PUMP$
CSUP PT=63   DY=1  FO=18.02 KP=0.01   *AL=$CONST. SUPT. HA5 FOR PUMP$
CSUP PT=160  DY=-1 FO=11.8  KP=0.01   *AL=$CONST. SUPT. HA-9 & HA-10$
CSUP PT=705  DY=1  FO=0.960 KP=0.01   *AL=$CONST. SUPT. HA-11 ON 4 INCH BYPASS$
*
VSUP PT=184  DY=1  FO=36.0 SP=3.542   *AL=$VARI. SUPT. HA-2$
VSUP PT=343  DY=1  FO=7.1  SP=3.014   *AL=$VARI. SUPT. HA13$
```

```
VSUP PT=313  DY=1  FO=7.1  SP=3.014   *AL=$VARI. SUPT. HA14$
*
VSUP PT=530  DY=1  SP=9.420 FO=26.0   *AL=$HANGER H109 RHR SUPPLY VALVE$
VSUP PT=620  DY=1  SP=7.084 FO=14.9   *AL=$HANGER H104 RHR RETURN VALVE$
VSUP PT=655  DY=1  SP=4.710 FO=22.0   *AL=$HANGER H105 RHR RETURN VALVE$
*
RSTN PT=15   DY=0.7071 DZ=-0.7071  SP=6000    *RECIRC PUMP
RSTN PT=16   DX=-0.7071 DY=0.7071  SP=6000    *RECIRC PUMP
ENDP
```

RHR_15.inp

```
IDEN JB=3    *Job number (1 to 9999)
     CD=1    *1=ASME Class 1
     GR=-Y   *Direction of gravity
     VA=0    *0=Calculate            2=Verify
     IU=1    *Input units            1=USA
     OU=1    *Output units           1=USA
     CH=$    *Delimiter character
     AB=T    *FREE errors = abort
     PL=$Vermont Yankee$
     EN=$RVP$
TITL BL=3    *Modeling option:
             *  3 = uniform mass for static analysis
             *      lumped mass for dynamic analysis
             *      rotational inertia ignored
     GL=1    *Report forces/moment        0=Global     1=Local     2=G et L
     SU=1    *Support summary             0=No         1=Yes
     CV=15   *Code version - See Manual
     HS=1    *Highest 20 stress ratios for each case
     MD=1    *Hot modulus
     J6=1    *File generated by program
     TI=$Vermont Yankee Recirculation $
        $Fatigue Analysis$
FREQ RF=1 RP=8 FR=36 MP=20 RC=0 MX=70 TI=$SEISMIC$
*
*************************************
**** THERMAL CYCLE LOAD CASES****
*************************************
LCAS RF=0 CA=1 TY=0 TI=$LC-1$      *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$      *TC-2
LCAS RF=0 CA=3 TY=0 TI=$LC-3$      *TC-3
LCAS RF=0 CA=4 TY=0 TI=$LC-4$      *TC-4
LCAS RF=0 CA=5 TY=0 TI=$LC-5$      *TC-5
LCAS RF=0 CA=6 TY=0 TI=$LC-6$      *TC-6
LCAS RF=0 CA=7 TY=0 TI=$LC-7$      *TC-7
LCAS RF=0 CA=8 TY=0 TI=$LC-8$      *TC-8
LCAS RF=0 CA=9 TY=0 TI=$LC-9$      *TC-9
LCAS RF=0 CA=10 TY=0 TI=$LC-10$    *TC-10
LCAS RF=0 CA=11 TY=0 TI=$LC-11$    *TC-11
LCAS RF=0 CA=12 TY=0 TI=$LC-12$    *TC-12
LCAS RF=0 CA=13 TY=0 TI=$LC-13$    *TC-13
LCAS RF=0 CA=14 TY=0 TI=$LC-14$    *TC-14
LCAS RF=0 CA=15 TY=0 TI=$LC-15$    *TC-15
LCAS RF=0 CA=16 TY=0 TI=$LC-16$    *TC-16
LCAS RF=0 CA=17 TY=0 TI=$LC-17$    *TC-17
LCAS RF=0 CA=18 TY=0 TI=$LC-18$    *TC-18
LCAS RF=0 CA=19 TY=0 TI=$LC-19$    *TC-19
LCAS RF=0 CA=20 TY=0 TI=$LC-20$    *TC-20
LCAS RF=0 CA=21 TY=0 TI=$LC-21$    *TC-21
LCAS RF=0 CA=22 TY=0 TI=$LC-22$    *TC-22
```

```
LCAS RF=0 CA=23 TY=0 TI=$LC-23$   *TC-23
LCAS RF=0 CA=24 TY=0 TI=$LC-24$   *TC-24
LCAS RF=0 CA=25 TY=0 TI=$LC-25$   *TC-25
*
*********************
**** WEIGHT CASES****
*********************
LCAS CA=101  RF=1  TY=3  TI=$OPERATING WEIGHT$
LCAS CA=102  RF=2  TY=4  TI=$HYDROTEST WEIGHT$
*
*********************************
**** THERMAL TRANSIENT CASES****
*********************************
TCAS CA=201 TI=$Design Hydrotest (+) $
TCAS CA=202 TI=$Design Hydrotest (-) $
TCAS CA=203 TI=$Startup              $
TCAS CA=204 TI=$TRoll & Inc. PWR1    $
TCAS CA=205 TI=$TRoll & Inc. PWR2    $
TCAS CA=206 TI=$LOFWH+TT PWR1        $
TCAS CA=207 TI=$LOFWH+TT PWR2        $
TCAS CA=208 TI=$LOFWH+PFWHTR Byp1    $
TCAS CA=209 TI=$LOFWH+PFWHTR Byp2    $
TCAS CA=210 TI=$LOFWP, ISO Cl DN 1   $
TCAS CA=211 TI=$LOFWP, ISO Cl UP 1   $
TCAS CA=212 TI=$LOFWP, ISO Cl DN 2   $
TCAS CA=213 TI=$LOFWP, ISO Cl UP 2   $
TCAS CA=214 TI=$Reduction to 0% PWR  $
TCAS CA=215 TI=$Shutdown1            $
TCAS CA=216 TI=$Shutdown2            $
TCAS CA=217 TI=$Shutdown3            $
TCAS CA=218 TI=$Shutdown4            $
TCAS CA=219 TI=$Code Hydrotest       $
TCAS CA=220 TI=$RHR Initiation UP    $
TCAS CA=221 TI=$RHR Initiation DN    $
TCAS CA=222 TI=$Inadvert. Inj. DOWN  $
TCAS CA=223 TI=$Inadvert. Inj. UP    $
TCAS CA=224 TI=$Single Relief BD DN  $
TCAS CA=225 TI=$Single Relief BD UP  $
*
**********************
**** SEISMIC CASES****
**********************
RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=$OBE INERTIA$
*
** *************************
**** LOAD COMBINATION CASES *
** *************************
CCAS RF=1 CA=104      ME=1 FL=1  C1=103 CY=10  TI=$OBE$
CCAS RF=1 CA=401 SS=1 ME=1 EQ=3  C1=101 C2=103 TI=$EQUATION 9 LEVEL B$
CCAS RF=1 CA=402 SS=1 ME=3 F1=1  C1=103 C2=1   TI=$NORMAL+OBE$
CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=1   TI=$NORMAL-OBE$
*
******************
**** LOAD SETS****
******************
*
LSET RF=1  FC=0 RP=1 CY=120  PR=1  MO=1  TR=201  TI=$Design Hydrotest (+)LS-1$
LSET RF=2  FC=0 RP=1 CY=120  PR=2  MO=2  TR=-202 TI=$Design Hydrotest (-)LS-2$
LSET RF=3  FC=0 RP=1 CY=300  PR=3  MO=3  TR=203  TI=$Startup            LS-3$
LSET RF=3  FC=0 RP=1 CY=579  PR=4  MO=4  TR=-204 TI=$TRoll & Inc. PWR1   LS-4$
```

```
LSET RF=4  FC=0 RP=1 CY=579  PR=5  MO=5  TR=-205 TI=$TRoll & Inc. PWR2   LS-5$
LSET RF=4  FC=0 RP=1 CY=20   PR=6  MO=6  TR=206  TI=$LOFWH+TT PWR1       LS-6$
LSET RF=4  FC=0 RP=1 CY=20   PR=7  MO=7  TR=-207 TI=$LOFWH+TT PWR2       LS-7$
LSET RF=5  FC=0 RP=1 CY=70   PR=8  MO=8  TR=-208 TI=$LOFWH+PFWHTR Byp1   LS-8$
LSET RF=5  FC=0 RP=1 CY=70   PR=9  MO=9  TR=209  TI=$LOFWH+PFWHTR Byp2   LS-9$
LSET RF=5  FC=0 RP=1 CY=10   PR=10 MO=10 TR=-210 TI=$LOFWP, ISO Cl DN 1  LS-10$
LSET RF=11 FC=0 RP=1 CY=20   PR=11 MO=11 TR=211  TI=$LOFWP, ISO Cl UP 1  LS-11$
LSET RF=11 FC=0 RP=1 CY=20   PR=12 MO=12 TR=-212 TI=$LOFWP, ISO Cl DN 2  LS-12$
LSET RF=3  FC=0 RP=1 CY=10   PR=13 MO=13 TR=213  TI=$LOFWP, ISO Cl UP 2  LS-13$
LSET RF=3  FC=0 RP=1 CY=300  PR=14 MO=14 TR=214  TI=$Reduction to 0% PWR LS-14$
LSET RF=5  FC=0 RP=1 CY=300  PR=15 MO=15 TR=-215 TI=$Shutdown1           LS-15$
LSET RF=15 FC=0 RP=1 CY=300  PR=16 MO=16 TR=-216 TI=$Shutdown2           LS-16$
LSET RF=16 FC=0 RP=1 CY=300  PR=17 MO=17 TR=-217 TI=$Shutdown3           LS-17$
LSET RF=20 FC=0 RP=1 CY=300  PR=18 MO=18 TR=-218 TI=$Shutdown4           LS-18$
LSET RF=19 FC=0 RP=1 CY=1    PR=19 MO=19 TR=219  TI=$Code Hydrotest      LS-19$
LSET RF=20 FC=0 RP=1 CY=300  PR=20 MO=20 TR=220  TI=$RHR Initiation UP   LS-20$
LSET RF=20 FC=0 RP=1 CY=300  PR=21 MO=21 TR=-221 TI=$RHR Initiation DN   LS-21$
LSET RF=5  FC=0 RP=1 CY=0    PR=22 MO=22 TR=-222 TI=$Inadvert. Inj. DOWN LS-22$
LSET RF=5  FC=0 RP=1 CY=0    PR=23 MO=23 TR=223  TI=$Inadvert. Inj. UP   LS-23$
LSET RF=23 FC=0 RP=1 CY=0    PR=24 MO=24 TR=-224 TI=$Single Relief BD DN LS-24$
LSET RF=24 FC=0 RP=1 CY=0    PR=25 MO=25 TR=225  TI=$Single Relief BD UP LS-25$
*
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=402 TI=$NORMAL+OBE  LS-26$
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=403 TI=$NORMAL-OBE  LS-27$
******
*FATG AT=500 AF=502
*FATG AT=600 AF=602
*
***************************
**** RESPONSE SPECTRA****
*************************
SPEC FS=OBE EV=1 ME=3 FP=0  TI=$RESPONSE$
      LV=1 DX=1 DY=1 DZ=1
      DI=X
        0.30/0.100  0.40/0.100  0.90/0.200  1.25/0.400  2.25/0.450  2.30/0.700
        3.30/0.700  4.40/0.750  4.41/0.900  4.75/1.100  5.20/1.100  5.80/1.600
        8.70/1.600 12.00/0.650 17.00/0.400 20.00/0.350 30.00/0.350 36.00/0.350
      DI=Y
        0.30/0.030  0.40/0.030  0.50/0.050  0.60/0.075  1.00/0.075  1.20/0.100
        2.00/0.220  2.40/0.350  3.50/0.350  3.60/0.300  5.30/0.300  5.75/0.330
        8.25/0.330  8.75/0.250 17.50/0.250 25.00/0.120 30.00/0.120 36.00/0.120
      DI=Z
        0.30/0.100  0.40/0.100  0.50/0.130  0.90/0.150  1.00/0.250  1.60/0.250
        1.90/0.600  3.50/0.600  3.75/0.700  4.40/0.700  4.50/0.800  6.25/1.500
        8.50/1.500 12.50/0.500 20.00/0.350 30.00/0.350 36.00/0.350
*
*****************************
**** MATERIAL PROPERTIES ****
*****************************
* ASTM A-106 Grade B, PIPE *
MATH CD=106      EX=0       TY=1      *C-Si
MATD TE=70       EH=29.5    EX=0.0    SM=20.0   SY=35
MATD TE=100      EH=29.3    EX=0.20   SM=20.0   SY=35
MATD TE=200      EH=28.8    EX=1.00   SM=20.0   SY=32.1
MATD TE=300      EH=28.3    EX=1.90   SM=20.0   SY=31
MATD TE=400      EH=27.7    EX=2.80   SM=20.0   SY=29.9
MATD TE=500      EH=27.3    EX=3.70   SM=18.9   SY=28.5
MATD TE=600      EH=26.7    EX=4.70   SM=17.3   SY=26.8
* ASME SA-376 Grade TP316, PIPE *
MATH CD=376.316 EX=0        TY=4      *16Cr-12Ni-2Mo
```

```
MATD TE=70       EH=28.3    EX=0.0    SM=20.0    SY=30.0
MATD TE=100      EH=28.1    EX=0.30   SM=20.0    SY=30.0
MATD TE=200      EH=27.6    EX=1.40   SM=20.0    SY=25.9
MATD TE=300      EH=27.0    EX=2.50   SM=20.0    SY=23.4
MATD TE=400      EH=26.5    EX=3.70   SM=19.3    SY=21.4
MATD TE=500      EH=25.8    EX=5.00   SM=18.0    SY=20.0
MATD TE=600      EH=25.3    EX=6.30   SM=17.0    SY=18.9
* ASME SA-403 Grade WP316, ELBOWS *
MATH CD=403.316 EX=0        TY=4      *16Cr-12Ni-2Mo
MATD TE=70       EH=28.3    EX=0.0    SM=20.0    SY=30.0
MATD TE=100      EH=28.1    EX=0.30   SM=20.0    SY=30.0
MATD TE=200      EH=27.6    EX=1.40   SM=20.0    SY=25.9
MATD TE=300      EH=27.0    EX=2.50   SM=20.0    SY=23.4
MATD TE=400      EH=26.5    EX=3.70   SM=18.7    SY=21.4
MATD TE=500      EH=25.8    EX=5.00   SM=17.5    SY=20.0
MATD TE=600      EH=25.3    EX=6.30   SM=16.4    SY=18.9
*** Cross Sectional Properties
CROS CD=1  OD=50.0    WT=8.87    MA=3977.2 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0                *RECIRCULATION OUTLET NOZZLE
CROS CD=2  OD=37.85   WT=6.1     MA=2122.2 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=3  OD=28.875  WT=1.56    MA=484.9  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=4  OD=28.638  WT=1.45    MA=450.4  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=5  OD=28.169  WT=1.244   MA=386.1  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=7  OD=28.166  WT=2.125   MA=0.001  *VALVE
           SO=1       ST=1.0     KL=1
CROS CD=8  OD=42.507  WT=2.486   MA=0.001  *PUMP
           SO=.001    ST=.001    KL=1
CROS CD=11 OD=6.625   WT=0.432   MA=0.001  *PUMP RIGID STRUTS
           SO=0.001   ST=0.001   KL=1
CROS CD=13 OD=28.339  WT=1.339   MA=415.1  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=14 OD=28.339  WT=2.67    MA=0.001  *VALVE
           SO=1       ST=1.0     KL=1
CROS CD=15 OD=12.748  WT=0.685   MA=103.4  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=16 OD=14.17   WT=1.395   MA=207.5  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=17 OD=15.5    WT=2       MA=307.7  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=18 OD=21.88   WT=4.06    MA=803.2  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=19 OD=28.25   WT=7.25    MA=1673.1 *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=20 OD=21.878  WT=1.043   MA=257.2  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1.0
CROS CD=25 OD=20      WT=1.031   MA=221.9  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=26 OD=20      WT=1.875   MA=0.001  *VALVE
           SO=1       ST=1       KL=1
CROS CD=27 OD=4.5     WT=0.3385  MA=23.2   *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1       KL=1      *4 inch bypass line
CROS CD=28 OD=4.5     WT=0.67    MA=0.001  *VALVE V2-54A
           SO=1       ST=1       KL=1
CROS CD=29 OD=24      WT=1.217   MA=316.5  *CALC. PER GE SPEC. NO. 23A5569 [3]
           SO=1       ST=1
CROS CD=30 OD=24      WT=2.43    MA=0.001  *VALVE
```

```
            SO=1       ST=1       KL=1
CROS CD=40 OD=4.5     WT=0.3385 MA=0.001  *4 inch bypass STRUTS
            SO=0.001  ST=0.001  KL=1
CROS CD=41 OD=2.875   WT=0.276  MA=0.001  *STRUT RDA1, RDA5, & VBA1
            SO=0.001  ST=0.001  KL=1
CROS CD=42 OD=28.339 WT=1.339  MA=0.001  *RIGID FROM RECIRC ELBOW TO RDA1 STRUT
            SO=0.001  ST=0.001  KL=1
**********************
* STRUCTURE AND LOADS
**********************
*-----------------------------------------------------------------------------
DESN  TE=575.0  PR=1250.0 *Reference 12 GE Design Requirements Rpt VY-05Q-227
*-----------------------------------------------------------------------------
*-----------------------------------------------------------------------
*BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE
*-----------------------------------------------------------------------
*
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG1.INP
*
* RUN   1     FROM ANCHOR TO REACTOR VESSEL N3B
*GROUP 1 FROM ANCHOR TO REACTOR VESSEL N3B
*NOTE
*NOTE NODE 003 - RECIRC SUCTION NOZZLE N1A (EL. 279'5 INCH)
*NOTE NODE 003 IS AT THE SAFE END TO VESSEL NOZZLE CONNECTION
*NOTE
*NOTE SAFE END FROM NODES 003 TO 808
*NOTE CONNECTION TO VESSEL AT NODE 003
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT FOR SAFE END BASED ON THICKNESS
*NOTE
MATL CD=376.316
CROS CD=1
COOR PT=3    AX=0  AY=0  AZ=0
ANCH PT=3
AMVT    CA=1    PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=2    PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=3    PT=3    DX=0.0000  DY=0.3112  DZ=-0.3568
AMVT    CA=4    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=5    PT=3    DX=0.0000  DY=0.3112  DZ=-0.3568
AMVT    CA=6    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=7    PT=3    DX=0.0000  DY=0.2922  DZ=-0.3350
AMVT    CA=8    PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=9    PT=3    DX=0.0000  DY=0.1422  DZ=-0.1630
AMVT    CA=10   PT=3    DX=0.0000  DY=0.2807  DZ=-0.3218
AMVT    CA=11   PT=3    DX=0.0000  DY=0.1422  DZ=-0.1630
AMVT    CA=12   PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=13   PT=3    DX=0.0000  DY=0.3141  DZ=-0.3602
AMVT    CA=14   PT=3    DX=0.0000  DY=0.1928  DZ=-0.2521
AMVT    CA=15   PT=3    DX=0.0000  DY=0.1624  DZ=-0.1986
AMVT    CA=16   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=17   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=18   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
AMVT    CA=19   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=20   PT=3    DX=0.0000  DY=0.0946  DZ=-0.1084
AMVT    CA=21   PT=3    DX=0.0000  DY=0.0361  DZ=-0.0413
AMVT    CA=22   PT=3    DX=0.0000  DY=0.2995  DZ=-0.3434
AMVT    CA=23   PT=3    DX=0.0000  DY=0.1928  DZ=-0.2521
AMVT    CA=24   PT=3    DX=0.0000  DY=0.0176  DZ=-0.0201
```

```
*
TANG PT=805  DZ=-1.017  EW=1
CROS CD=2
TANG PT=806  DZ=-0.823  EW=1
CROS CD=3
TANG PT=807  DZ=-0.58  EW=1
CROS CD=4
TANG PT=808  DZ=-0.47
CROS CD=5
TANG PT=5    DZ=-5.59 EW=1
MATL CD=403.316
BRAD PT=7    RA=3.5 EW=1
MATL CD=376.316
TANG PT=9    DY=-6.69  EW=1
TANG PT=500  DY=-2.31
*-----------------------------------------------
*END REGION 1 GEOMETRY FROM RHR SUPPLY TO TEE
*-----------------------------------------------
*
*-----------------------------------------------------------------------
*BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----------------------------------------------------------------------
*GROUP 2 RHR SUPPLY TEE TO PUMP
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG2.INP
*
TANG PT=11   DY=-2.22  EW=1
CROS CD=5
TANG PT=12   DY=-1.78
TANG PT=20   DY=-6.77
TANG PT=22   DY=-3.25
TANG PT=25   DY=-15.49  EW=1
MATL CD=403.316
BRAD PT=26   RA=3.5  EW=1
MATL CD=376.316
TANG PT=27   DX=-3.3 DZ=1.27  EW=1
CROS CD=7
VALV PT=30   DX=-2.28 DZ=0.89 MA=10.368 PL=1
JUNC PT=30
VALV PT=40   DX=-2.31 DZ=0.9 PL=2  EW=1
JUNC PT=30
RIGD PT=35   DY=7
LUMP PT=35   MA=1.132
JUNC PT=40
CROS CD=5
TANG PT=42 DX=-1.18 DZ=0.46
TANG PT=43 DX=-0.55 DZ=0.21
TANG PT=44 DX=-3.31 DZ=1.28  EW=1
MATL CD=403.316
BRAD PT=46 RA=2.33  EW=1
MATL CD=376.316
CROS CD=8
TANG PT=50 DY=4.33  EW=0
LUMP PT=50 MA=28  *NOTE WEIGHT OF PUMP FLOODED 28K (EXCLUDING MOTOR)
TANG PT=75 DY=0.5
TANG PT=83 DY=2.13
TANG PT=86 DY=3.38
LUMP PT=86 MA=32  *NOTE TOTAL WEIGHT OF PUMP MOTOR 32000 LBS
TANG PT=90 DY=4.08 *TOP OF PUMP
*NOTE SNUBBERS ON TOP OF PUMPS WERE DELETED DURING
*NOTE THE RECIRC PIPE REPLACEMENT PROJECT
```

```
*NOTE - RIGID LINKS FOR CONSTANT SUPPORTS AT PUMP FOLLOW
*----------------------------
*END REGION 2 GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*----------------------------
*
*----------------------------------------
*BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
*----------------------------------------
*GROUP 3 FROM PUMP DISCHARGE TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3.INP
*
JUNC PT=50
CROS CD=8
RIGD PT=54 DX=1.06 DZ=1.06
RIGD PT=56 DX=1.06 DY=0.75 DZ=1.06  *NOTE CONSTANT SUPPORT HA3 AT NODE 56
JUNC PT=50
RIGD PT=66 DZ=-3.83
RIGD PT=69 DY=1                     *NOTE CONSTANT SUPPORT HA4 AT NODE 69
JUNC PT=50
CROS CD=8
RIGD PT=60 DX=-3.83
RIGD PT=63 DY=1                     *CONSTANT SUPPORT HA5 AT NODE 63
*
* *** CODING FOR PUMP RIGID STRUTS FOLLOW ***
*     CODED FROM PUMP CENTERLINE
CROS CD=11
JUNC PT=66
RIGD PT=15    DY=0.7071 DZ=-0.7071
*
JUNC PT=60
RIGD PT=16    DX=-0.7071 DY=0.7071
* *** END OF CODING FOR PUMP SUPPORTS ***
*PUMP INLET
CROS CD=8
JUNC PT=50
TANG PT=150 DX=-2.17
BRAN PT=151 DZ=2.333  TE=1
*NOTE PUMP DISCHARGE CONNECTION TO PIPE AT NODE 151
CROS CD=13
TANG PT=152 DZ=1.25
TANG PT=155 DZ=1  EW=1
CROS CD=14
VALV PT=160 PL=1 DX=0.0 DY=0.0  DZ=2.52 MA=6.8285
JUNC PT=160
RIGD PT=163 DX=0.0 DY=7.12 DZ=0.0
LUMP PT=163 MA=0.9715
JUNC PT=160
VALV PT=170 PL=2 DX=0.0 DY=0.0 DZ=6.18  EW=1
CROS CD=13
MATL CD=403.316
BRAD PT=175 RA=3.5  EW=1
MATL CD=376.316
TANG PT=176 DY=5.95
TANG PT=177 DY=4.42
*NOTE ***WEIGHT OF FLOW ELEMENT NOT INCLUDED***
*NOTE ***REF. DWG. 5920-6800 FOR DIMENSIONS***
TANG PT=184 DY=4.42
TANG PT=186 DY=3.02
TANG PT=188 DY=1.51
TANG PT=189 DY=0.74
```

```
TANG PT=190 DY=1.15  EW=1
TANG PT=600 DY=1.06

***INPUT FILE TO INCLUDE EFFECTS OF RHR INITIATION ON LINE NEAR RHR RETURN TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3B.INP

JUNC PT=600
TANG PT=195 DY=2.08  EW=1
TANG PT=210 DX=0.0  DY=1.83 DZ=0.0  KL=1 *CENTER OF CROSS, RECIRC HEADER
*MUST HAVE INDI CARD FOR EACH MEMBER CONNECTED TO CROSS CENTER
*----------------------------
*END REGION 3 GEOMETRY FROM PUMP DISCHARGE TO HEADER
*----------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 336
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
*NOTE CROSS AND REDUCER DIMENSIONS TAKEN FROM 5920-6632 SHT.3
CROS CD=13
MATL CD=376.316
TANG PT=215 DX=0.0 DY=2.59  DZ=0.0  EW=0
CRED PT=220 DY=1.29  AN=30  EW=1  *AL=$CONC. REDUCER$
CROS CD=15
TANG PT=330 DY=4.58
TANG PT=335 DY=3.29  EW=1
MATL CD=403.316
BRAD PT=334 RA=1.5  EW=1
*----------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 336
*----------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 336
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=838 DX=3.875
CROS CD=16
TANG PT=837 DX=0.875  EW=1
CROS CD=17
TANG PT=836 DX=0.37  EW=1
CROS CD=18
TANG PT=835 DX=0.53  EW=1
CROS CD=19
TANG PT=336 DX=0.704  EW=1
NOZZ PT=336
AMVT    CA=1    PT=336  DX=-0.0201  DY=0.0246  DZ=0.0000
AMVT    CA=2    PT=336  DX=-0.3602  DY=0.4398  DZ=0.0000
AMVT    CA=3    PT=336  DX=-0.3568  DY=0.4316  DZ=0.0000
AMVT    CA=4    PT=336  DX=-0.3434  DY=0.4152  DZ=0.0000
AMVT    CA=5    PT=336  DX=-0.3568  DY=0.4050  DZ=0.0000
AMVT    CA=6    PT=336  DX=-0.3434  DY=0.2940  DZ=0.0000
AMVT    CA=7    PT=336  DX=-0.3350  DY=0.3229  DZ=0.0000
AMVT    CA=8    PT=336  DX=-0.3434  DY=0.2700  DZ=0.0000
AMVT    CA=9    PT=336  DX=-0.1630  DY=0.1991  DZ=0.0000
```

```
AMVT    CA=10   PT=336  DX=-0.3218  DY=0.1626  DZ=0.0000
AMVT    CA=11   PT=336  DX=-0.1630  DY=0.0246  DZ=0.0000
AMVT    CA=12   PT=336  DX=-0.3602  DY=0.4398  DZ=0.0000
AMVT    CA=13   PT=336  DX=-0.3602  DY=0.4316  DZ=0.0000
AMVT    CA=14   PT=336  DX=-0.2193  DY=0.4152  DZ=0.0000
AMVT    CA=15   PT=336  DX=-0.1862  DY=0.4050  DZ=0.0000
AMVT    CA=16   PT=336  DX=-0.1084  DY=0.2940  DZ=0.0000
AMVT    CA=17   PT=336  DX=-0.0201  DY=0.3229  DZ=0.0000
AMVT    CA=18   PT=336  DX=-0.0201  DY=0.2700  DZ=0.0000
AMVT    CA=19   PT=336  DX=-0.1084  DY=0.1991  DZ=0.0000
AMVT    CA=20   PT=336  DX=-0.0201  DY=0.1626  DZ=0.0000
AMVT    CA=21   PT=336  DX=-0.0413  DY=0.3229  DZ=0.0000
AMVT    CA=22   PT=336  DX=-0.3434  DY=0.2700  DZ=0.0000
AMVT    CA=23   PT=336  DX=-0.2211  DY=0.1991  DZ=0.0000
AMVT    CA=24   PT=336  DX=-0.0201  DY=0.1626  DZ=0.0000


*NOTE SAFE END FROM NODES 838 TO 336
*NOTE CONNECTION TO VESSEL AT NODE 336
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT BASED ON THICKNESS
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 336
*---------------------------
*
*----------------------------------------
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*----------------------------------------
*GROUP 4 HEADER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*
JUNC PT=210
CROS CD=20
BRAN PT=240 DX=0.1786 DY=0.0 DZ=1.7
TANG PT=250 DX=0.3 DZ=2.853  EW=0
BRAD PT=255 RA=4.578  EW=0 *NOTE BEND RADIUS IS 4.578 FEET
TANG PT=340 DX=1.799 DZ=3.108
*---------------------------
*END REGION 4 GEOMETRY HEADER TO NOZZLE NODE 366
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 366
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
TANG PT=349 DX=0.71 DZ=1.23  EW=0
CRED PT=347 DX=0.75 DZ=1.3  AN=30
CROS CD=15
TANG PT=343 DX=0.5525 DZ=0.957  EW=1
BRAD PT=410 RA=1.5  EW=1
TANG PT=360 DX=3.483 DZ=2.011  EW=1
MATL CD=403.316
BRAD PT=361 RA=1.5  EW=1
MATL CD=376.316
CROS CD=15
TANG PT=362 DY=3.18
TANG PT=364 DY=8.56  EW=1
MATL CD=403.316
```

```
BRAD PT=365 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 366
*---------------------------

*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 366
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=868 DX=1.8 DZ=-3.1
CROS CD=16
TANG PT=867 DX=0.4375 DZ=-0.76  EW=1
CROS CD=17
TANG PT=866 DX=0.185 DZ=-0.32  EW=1
CROS CD=18
TANG PT=865 DX=0.265 DZ=-0.46  EW=1
CROS CD=19
TANG PT=366 DX=0.352 DZ=-0.61  EW=1
NOZZ PT=366
AMVT    CA=1    PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=2    PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=3    PT=366  DX=-0.1783  DY=0.4357  DZ=0.3091
AMVT    CA=4    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=5    PT=366  DX=-0.1783  DY=0.4357  DZ=0.3091
AMVT    CA=6    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=7    PT=366  DX=-0.1674  DY=0.4091  DZ=0.2902
AMVT    CA=8    PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=9    PT=366  DX=-0.0815  DY=0.1991  DZ=0.1412
AMVT    CA=10   PT=366  DX=-0.1609  DY=0.3930  DZ=0.2788
AMVT    CA=11   PT=366  DX=-0.0815  DY=0.1991  DZ=0.1412
AMVT    CA=12   PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=13   PT=366  DX=-0.1800  DY=0.4398  DZ=0.3120
AMVT    CA=14   PT=366  DX=-0.1097  DY=0.2678  DZ=0.1899
AMVT    CA=15   PT=366  DX=-0.0931  DY=0.2275  DZ=0.1613
AMVT    CA=16   PT=366  DX=-0.0542  DY=0.1324  DZ=0.0939
AMVT    CA=17   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=18   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=19   PT=366  DX=-0.0542  DY=0.1324  DZ=0.0939
AMVT    CA=20   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174
AMVT    CA=21   PT=366  DX=-0.0207  DY=0.0505  DZ=0.0358
AMVT    CA=22   PT=366  DX=-0.1716  DY=0.4193  DZ=0.2974
AMVT    CA=23   PT=366  DX=-0.1105  DY=0.2700  DZ=0.1915
AMVT    CA=24   PT=366  DX=-0.0101  DY=0.0246  DZ=0.0174

*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 366
*---------------------------

*----------------------------------------
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*----------------------------------------
*GROUP 4 HEADER TO NOZZLES NODE 326 & 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*
JUNC PT=210
CROS CD=20
BRAN PT=260 DX=0.1786 DY=0.0 DZ=-1.7 TE=2
```

```
TANG PT=270 DX=0.3 DZ=-2.853  EW=0
BRAD PT=275 RA=4.578  EW=0
TANG PT=320 DX=1.799 DZ=-3.108
*---------------------------
*END REGION 4 GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*---------------------------

*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 316
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
TANG PT=319 DX=0.71 DZ=-1.23  EW=1
CRED PT=317 DX=0.75 DZ=-1.3  AN=30
CROS CD=15
TANG PT=313 DX=0.5525 DZ=-0.957  EW=1
BRAD PT=400 RA=1.5  EW=1
TANG PT=310 DX=3.483 DZ=-2.011  EW=1
MATL CD=403.316
BRAD PT=311 RA=1.5  EW=1
MATL CD=376.316
CROS CD=15
TANG PT=312 DY=4.74
TANG PT=314 DY=6.99  EW=1
MATL CD=403.316
BRAD PT=315 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 316
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 316
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=818 DX=1.84 DZ=3.19
CROS CD=16
TANG PT=817 DX=0.4375 DZ=0.76  EW=1
CROS CD=17
TANG PT=816 DX=0.185 DZ=0.32  EW=1
CROS CD=18
TANG PT=815 DX=0.265 DZ=0.46  EW=1
CROS CD=19
TANG PT=316 DX=0.352 DZ=0.61  EW=1
NOZZ PT=316
AMVT    CA=1    PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=2    PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
AMVT    CA=3    PT=316  DX=-0.1783  DY=0.4357  DZ=-0.3091
AMVT    CA=4    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=5    PT=316  DX=-0.1783  DY=0.4357  DZ=-0.3091
AMVT    CA=6    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=7    PT=316  DX=-0.1674  DY=0.4091  DZ=-0.2902
AMVT    CA=8    PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=9    PT=316  DX=-0.0815  DY=0.1991  DZ=-0.1412
AMVT    CA=10   PT=316  DX=-0.1609  DY=0.3930  DZ=-0.2788
AMVT    CA=11   PT=316  DX=-0.0815  DY=0.1991  DZ=-0.1412
AMVT    CA=12   PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
```

```
AMVT    CA=13   PT=316  DX=-0.1800  DY=0.4398  DZ=-0.3120
AMVT    CA=14   PT=316  DX=-0.1097  DY=0.2678  DZ=-0.1899
AMVT    CA=15   PT=316  DX=-0.0931  DY=0.2275  DZ=-0.1613
AMVT    CA=16   PT=316  DX=-0.0542  DY=0.1324  DZ=-0.0939
AMVT    CA=17   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=18   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=19   PT=316  DX=-0.0542  DY=0.1324  DZ=-0.0939
AMVT    CA=20   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
AMVT    CA=21   PT=316  DX=-0.0207  DY=0.0505  DZ=-0.0358
AMVT    CA=22   PT=316  DX=-0.1716  DY=0.4193  DZ=-0.2974
AMVT    CA=23   PT=316  DX=-0.1105  DY=0.2700  DZ=-0.1915
AMVT    CA=24   PT=316  DX=-0.0101  DY=0.0246  DZ=-0.0174
*----------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 316
*----------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 346
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=340
CROS CD=15
BRAN PT=342 DY=1.36  TE=2
TANG PT=344 DY=10.39  EW=0
MATL CD=403.316
BRAD PT=345 RA=1.5  EW=1
*----------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 346
*----------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 346
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=848 DX=3.17 DZ=-1.83
CROS CD=16
TANG PT=847 DX=0.758 DZ=-0.4375  EW=1
CROS CD=17
TANG PT=846 DX=0.32 DZ=-0.185  EW=1
CROS CD=18
TANG PT=845 DX=0.46 DZ=-0.265  EW=1
CROS CD=19
TANG PT=346 DX=0.61 DZ=-0.352  EW=1
NOZZ PT=346
AMVT    CA=1    PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=2    PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=3    PT=346  DX=-0.3091  DY=0.4357  DZ=0.1783
AMVT    CA=4    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=5    PT=346  DX=-0.3091  DY=0.4357  DZ=0.1783
AMVT    CA=6    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=7    PT=346  DX=-0.2902  DY=0.4091  DZ=0.1674
AMVT    CA=8    PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=9    PT=346  DX=-0.1412  DY=0.1991  DZ=0.0815
AMVT    CA=10   PT=346  DX=-0.2788  DY=0.3930  DZ=0.1609
AMVT    CA=11   PT=346  DX=-0.1412  DY=0.1991  DZ=0.0815
```

```
AMVT    CA=12   PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=13   PT=346  DX=-0.3120  DY=0.4398  DZ=0.1800
AMVT    CA=14   PT=346  DX=-0.1899  DY=0.2678  DZ=0.1097
AMVT    CA=15   PT=346  DX=-0.1613  DY=0.2275  DZ=0.0931
AMVT    CA=16   PT=346  DX=-0.0939  DY=0.1324  DZ=0.0542
AMVT    CA=17   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=18   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=19   PT=346  DX=-0.0939  DY=0.1324  DZ=0.0542
AMVT    CA=20   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
AMVT    CA=21   PT=346  DX=-0.0358  DY=0.0505  DZ=0.0207
AMVT    CA=22   PT=346  DX=-0.2974  DY=0.4193  DZ=0.1716
AMVT    CA=23   PT=346  DX=-0.1915  DY=0.2700  DZ=0.1105
AMVT    CA=24   PT=346  DX=-0.0174  DY=0.0246  DZ=0.0101
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 346
*---------------------------
*
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 326
*----------------------------------------
*GROUP 5 RISER TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=320
CROS CD=15
BRAN PT=322 DY=1.42  TE=2
TANG PT=324 DY=10.33  EW=1
MATL CD=403.316
BRAD PT=325 RA=1.5  EW=1
*---------------------------
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 326
*---------------------------
*
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 326
*----------------------------------------
*GROUP 6 TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=828 DX=3.18 DZ=1.84
CROS CD=16
TANG PT=827 DX=0.758 DZ=0.4375  EW=1
CROS CD=17
TANG PT=826 DX=0.32 DZ=0.185  EW=1
CROS CD=18
TANG PT=825 DX=0.46 DZ=0.265  EW=1
CROS CD=19
TANG PT=326 DX=0.61 DZ=0.352  EW=1
NOZZ PT=326
AMVT    CA=1    PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=2    PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=3    PT=326  DX=-0.3091  DY=0.4357  DZ=-0.1783
AMVT    CA=4    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=5    PT=326  DX=-0.3091  DY=0.4357  DZ=-0.1783
AMVT    CA=6    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=7    PT=326  DX=-0.2902  DY=0.4091  DZ=-0.1674
AMVT    CA=8    PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=9    PT=326  DX=-0.1412  DY=0.1991  DZ=-0.0815
AMVT    CA=10   PT=326  DX=-0.2788  DY=0.3930  DZ=-0.1609
```

```
AMVT    CA=11   PT=326  DX=-0.1412  DY=0.1991  DZ=-0.0815
AMVT    CA=12   PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=13   PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT    CA=14   PT=326  DX=-0.1899  DY=0.2678  DZ=-0.1097
AMVT    CA=15   PT=326  DX=-0.1613  DY=0.2275  DZ=-0.0931
AMVT    CA=16   PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT    CA=17   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=18   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=19   PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT    CA=20   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT    CA=21   PT=326  DX=-0.0358  DY=0.0505  DZ=-0.0207
AMVT    CA=22   PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT    CA=23   PT=326  DX=-0.1915  DY=0.2700  DZ=-0.1105
AMVT    CA=24   PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
*---------------------------
*END REGION 6 GEOMETRY TO NOZZLE NODE 326
*---------------------------
*
*----------------------------------------
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*----------------------------------------
*GROUP 7 TO RHR SUPPLY VALVE NODE 550
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7A.INP
*
MATL CD=376.316
JUNC PT=500
CROS CD=25
BRAN PT=502 DX=1.67  EW=0  TE=1
TANG PT=506 DX=2.53  EW=0
MATL CD=403.316
BRAD PT=507 RA=1.67  EW=1
MATL CD=376.316
TANG PT=508 DZ=-4.01
TANG PT=515 DZ=-4.53  EW=1
MATL CD=403.316
BRAD PT=520 RA=1.67  EW=1
MATL CD=376.316
CROS CD=26
VALV PT=525 DX=-3.34 PL=1
JUNC PT=525
VALV PT=530 DX=-1.99 PL=2  EW=1
JUNC PT=525
RIGD PT=526 DY=2.5
LUMP PT=526 MA=7.569
JUNC PT=530
CROS CD=25
TANG PT=540 DX=-1.13  EW=1
CROS CD=26
VALV PT=545 DX=-1.97 PL=1
JUNC PT=545
RIGD PT=547 DY=2.5
LUMP PT=547 MA=7.355
JUNC PT=545
VALV PT=550 DX=-1.98 PL=2  EW=1
*---------------------------
*END REGION 7A GEOMETRY TO RHR SUPPLY VALVE NODE 550
*---------------------------
*
*----------------------------------------
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
```

```
*----------------------------------------
*GROUP 17 FROM RHR SUPPLY VALVE TO PENET. NODE 565
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7B.INP
*
CROS CD=25
MATL CD=106
TANG PT=555 DX=-3.36  EW=1
BRAD PT=556 RA=1.67  EW=1
TANG PT=560 DY=-10.17  EW=1
BRAD PT=561 RA=1.67  EW=1
TANG PT=563 DZ=-6.92
TANG PT=565 DZ=-6.92
*---------------------------
*END REGION 7B GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*---------------------------
*
*----------------------------------------
*BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
*----------------------------------------
*GROUP 8 4 INCH BYPASS
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG8.INP
*
*NOTE CODING FOR 4 INCH BYPASS STARTS HERE
JUNC PT=152
CROS CD=27
MATL CD=376.316
BRAN PT=700 DX=-1.19  TE=4
TANG PT=702 DX=-0.61
TANG PT=703 DX=-1.43  EW=0
MATL CD=403.316
BRAD PT=704 RA=0.5  EW=0
MATL CD=376.316
TANG PT=705 DZ=5.08
*NOTE CONSTANT SUPPORT HA11 AT NODE 705
TANG PT=721 DZ=1.12
TANG PT=706 DZ=2.47
TANG PT=707 DZ=1.03
TANG PT=708 DZ=0.34
TANG PT=709 DZ=0.38
JUNC PT=707
BRAN PT=710 DY=0.34 TE=1
CROS CD=28
VALV PT=712 DY=0.71 MA=0.3669 PL=1  *AL=$VALVE V2-54A$
VALV PT=715 DZ=-3.5 MA=0.1831 PL=3
JUNC PT=712
VALV PT=714 DY=0.71          PL=2
CROS CD=27
TANG PT=723 DY=4.19
MATL CD=403.316
BRAD PT=716 RA=0.5
MATL CD=376.316
TANG PT=718 DX=1.48
TANG PT=720 DX=0.56
BRAN PT=176 DX=1.19  TE=4
************CODING FOR STRUTS RDA5 AND VAB1 FOLLOW
JUNC PT=170
CROS CD=40  *OD=4.5 inch
RIGD PT=725 DP=0 DX=-0.583 DY=1.84  *AL=$RDA5$
CROS CD=41  *OD=2.875 inch
RIGD PT=715 DP=0 DX=-2.67 DY=-0.79
```

```
RIGD PT=721 DP=0 DY=-1.05  *AL=$VAB1$
*************CODING FOR RDA1 STRUT FOLLOWS
CROS CD=42  *OD=28.339 inch
JUNC PT=175
RIGD PT=173 DP=0 DY=-3.5 DZ=0.34
CROS CD=41  *OD=2.875 inch
RIGD PT=708 DP=0 DX=-3.21  *AL=$RDA1$
*--------------------------
*END REGION 8 GEOMETRY  FOR 4 INCH BYPASS
*--------------------------
*
*----------------------------------------
*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*----------------------------------------
*GROUP 9 RHR RETURN FROM TEE TO VALVE NODE 660
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9A.INP
*
*NOTE CODING FOR RHR RETURN STARTS HERE
CROS CD=29
JUNC PT=600
MATL CD=376.316
BRAN PT=602 DX=-3.8123  TE=1
MATL CD=403.316
BRAD PT=610 RA=2  EW=1
TANP        DY=4
BRAD PT=612 RA=2  EW=1
MATL CD=376.316
TANG PT=614 DZ=-10.38  EW=1
MATL CD=403.316
BRAD PT=615 RA=10  EW=1
MATL CD=376.316
TANG PT=620 DX=5.98 DZ=-3.45  EW=1
*NOTE
*NOTE VARIABLE SPRING H104 AT NODE 620
*NOTE
*NOTE VALVE V10-81A DATA FROM 5920-4590 WEIGHT - 6845.#
*NOTE WEIGHT APPLIED AT ESTIMATED CENTER OF GRAVITY (NODE 623)
CROS CD=30
VALV PT=622 DX=1.98 DZ=-1.15 PL=1  *AL=$VALVE V10-81A$
JUNC PT=622
VALV PT=624 DX=1.98 DZ=-1.15 PL=2  EW=1
JUNC PT=622
RIGD PT=623 DY=2.5
LUMP PT=623 MA=7.32 *VALVE ACTUATOR
CROS CD=29
JUNC PT=624
TANG PT=625 DX=1.867 DZ=-1.078
TANG PT=630 DX=2.598 DZ=-1.5  EW=1
MATL CD=403.316
BRAD PT=631 RA=3  EW=1
MATL CD=376.316
TANG PT=640 DZ=-4.54  EW=1
MATL CD=403.316
BRAD PT=641 RA=2  EW=1
MATL CD=376.316
*NOTE VALVE V10-46A DATA FROM 5920-4718 WEIGHT - 5295.#
CROS CD=30
VALV PT=655 DX=-3.79 PL=1 TA=2 *AL=$VALVE V10-46A$
LUMP PT=655 MA=5.77
```

```
*---------------------------
*END REGION 9A GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*---------------------------
*
*----------------------------------------
*BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE
675
*----------------------------------------
*GROUP 19 RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9B.INP
*
*NOTE
*NOTE VARIABLE SPRING H105 AT NODE 655
*NOTE
VALV PT=660 DX=-1.79 PL=2  EW=1
*NOTE SPEC CHANGE TO CARBON STEEL
MATL CD=106
CROS CD=29
TANG PT=661 DX=-1
TANG PT=663 DX=-3.31  EW=1
BRAD PT=665 RA=2  EW=1
TANG PT=670 DY=-10.5 DZ=0.38  EW=1
BRAD PT=671 RA=2  EW=1
TANG PT=673 DZ=-7.74
TANG PT=675 DZ=-7.74
*---------------------------
*END REGION 9B GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
*---------------------------
*
*---------------------------
***STRESS INDICES AT CROSS POINT
*---------------------------
*
INDI AT=210 AF=195 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=215 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=240 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
INDI AT=210 AF=260 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
*---------------------------
*** SUPPORTS
*---------------------------
RSTN PT=675  DX=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=675  DY=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=675  DZ=1  SP=23000          *RHR SUPPLY PENET.
ROTR PT=675  RX=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=675  RY=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=675  RZ=1  SP=340000         *RHR SUPPLY PENET.
RSTN PT=565  DX=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=565  DY=1  SP=16000          *RHR SUPPLY PENET.
RSTN PT=565  DZ=1  SP=23000          *RHR SUPPLY PENET.
ROTR PT=565  RX=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=565  RY=1  SP=300000         *RHR SUPPLY PENET.
ROTR PT=565  RZ=1  SP=340000         *RHR SUPPLY PENET.
*
SNUB PT=12   DZ=-1 SP=1000    *AL=$SNUBBER SS-7A-1$
SNUB PT=12   DX=1  SP=1000    *AL=$SNUBBER SS-7A-2$
SNUB PT=190  DX=-1 SP=1000    *AL=$SNUBBER SS-6-A1$
SNUB PT=190  DZ=1  SP=1000    *AL=$SNUBBER SS-6-A2$
*
VSUP PT=20   DY=1  FO=24.8  SP=2.664  *AL=$VARI. SUPT. HA-1$
*
```

```
CSUP PT=27   DY=1  FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A1$
CSUP PT=42   DY=1  FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A2$
CSUP PT=56   DY=1  FO=18.05 KP=0.01   *AL=$CONST. SUPT. HA3 FOR PUMP$
CSUP PT=69   DY=1  FO=18.0  KP=0.01   *AL=$CONST. SUPT. HA4 FOR PUMP$
CSUP PT=63   DY=1  FO=18.02 KP=0.01   *AL=$CONST. SUPT. HA5 FOR PUMP$
CSUP PT=160  DY=-1 FO=11.8  KP=0.01   *AL=$CONST. SUPT. HA-9 & HA-10$
CSUP PT=705  DY=1  FO=0.960 KP=0.01   *AL=$CONST. SUPT. HA-11 ON 4 INCH BYPASS$
*
VSUP PT=184  DY=1  FO=36.0 SP=3.542   *AL=$VARI. SUPT. HA-2$
VSUP PT=343  DY=1  FO=7.1  SP=3.014   *AL=$VARI. SUPT. HA13$
VSUP PT=313  DY=1  FO=7.1  SP=3.014   *AL=$VARI. SUPT. HA14$
*
VSUP PT=530  DY=1  SP=9.420 FO=26.0   *AL=$HANGER H109 RHR SUPPLY VALVE$
VSUP PT=620  DY=1  SP=7.084 FO=14.9   *AL=$HANGER H104 RHR RETURN VALVE$
VSUP PT=655  DY=1  SP=4.710 FO=22.0   *AL=$HANGER H105 RHR RETURN VALVE$
*
RSTN PT=15   DY=0.7071 DZ=-0.7071  SP=6000    *RECIRC PUMP
RSTN PT=16   DX=-0.7071 DY=0.7071  SP=6000    *RECIRC PUMP
ENDP
```

Reg1.inp

```
*-----------------------------------------------------------------
*BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE
*-----------------------------------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1010
OPER CA=4   TE=542  PR=1010
OPER CA=5   TE=526  PR=1010
OPER CA=6   TE=542  PR=1010
OPER CA=7   TE=526  PR=1010
OPER CA=8   TE=516  PR=1010
OPER CA=9   TE=526  PR=1010
OPER CA=10  TE=300  PR=1135
OPER CA=11  TE=500  PR=1135
OPER CA=12  TE=300  PR=675
OPER CA=13  TE=549  PR=1010
OPER CA=14  TE=549  PR=1010
OPER CA=15  TE=375  PR=170
OPER CA=16  TE=330  PR=90
OPER CA=17  TE=225  PR=0
OPER CA=18  TE=100  PR=0
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=0
OPER CA=21  TE=225  PR=0
OPER CA=22  TE=130  PR=1010
OPER CA=23  TE=526  PR=1010
OPER CA=24  TE=375  PR=200
OPER CA=25  TE=100  PR=0
*
TRAN  CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=2262  IP=15   FP=1115  TP=0
TRAN  CA=202  IS=1 FS=1 IT=100 FT=100  TT=1800  FL=2262  IP=1115 FP=65   TP=0
TRAN  CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=16158 IP=65   FP=1025  TP=0
TRAN  CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=32316 IP=1025 FP=1025  TP=0
```

```
TRAN   CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=600   IP=1205 FP=1150   TP=0
TRAN   CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=600   IP=900  FP=1150  TP=0
TRAN   CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=600   IP=1150 FP=690   TP=0
TRAN   CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=16158 IP=255  FP=1025  TP=0
TRAN   CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN   CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=16158 IP=1025 FP=185   TP=0
TRAN   CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=16158 IP=185  FP=105   TP=0
TRAN   CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=16158 IP=105  FP=15    TP=0
TRAN   CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=22858 IP=15   FP=15    TP=0
TRAN   CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=2262  IP=40   FP=1578  TP=0
TRAN   CA=220
TRAN   CA=221
TRAN   CA=222  IS=1 FS=1 IT=526 FT=130  TT=0     FL=32316 IP=1025  FP=1025 TP=0
TRAN   CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=32316 IP=1025  FP=1025 TP=0
TRAN   CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=32316 IP=1025  FP=215  TP=0
TRAN   CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=32316 IP=215   FP=15   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140  EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151  EX=8.5 *Tavg=324.5
PAIR  CA=204 CO=10.5 DI=0.162  EX=8.5 *Tavg=545.5
PAIR  CA=205 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=206 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=207 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=208 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=209 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=210 CO=9.9  DI=0.156  EX=8.5 *Tavg=413.0
PAIR  CA=211 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=212 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156  EX=8.5 *Tavg=424.5
PAIR  CA=214 CO=10.4 DI=0.162  EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158  EX=8.5 *Tavg=462.0
PAIR  CA=216 CO=9.5  DI=0.152  EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149  EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143  EX=8.5 *Tavg=162.5
PAIR  CA=219 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
*PAIR CA=220 CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
*PAIR CA=221 CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
PAIR  CA=222 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR  CA=223 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR  CA=224 CO=10.0 DI=0.157  EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147  EX=8.5 *Tavg=237.5
```

Reg2.inp

```
*-----------------------------------------------------------------------
*BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----------------------------------------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1010
OPER CA=4   TE=542  PR=1010
OPER CA=5   TE=526  PR=1010
OPER CA=6   TE=542  PR=1010
OPER CA=7   TE=526  PR=1010
OPER CA=8   TE=516  PR=1010
OPER CA=9   TE=526  PR=1010
OPER CA=10  TE=300  PR=1135
OPER CA=11  TE=500  PR=1135
OPER CA=12  TE=300  PR=675
```

```
OPER CA=13  TE=549  PR=1010
OPER CA=14  TE=549  PR=1010
OPER CA=15  TE=375  PR=170
OPER CA=16  TE=330  PR=90
OPER CA=17  TE=225  PR=0
OPER CA=18  TE=100  PR=0
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=0
OPER CA=21  TE=225  PR=0
OPER CA=22  TE=130  PR=1010
OPER CA=23  TE=526  PR=1010
OPER CA=24  TE=375  PR=200
OPER CA=25  TE=100  PR=0
*
TRAN  CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=2262  IP=15   FP=1115  TP=0
TRAN  CA=202  IS=1 FS=1 IT=100 FT=100  TT=1800  FL=2262  IP=1115 FP=65    TP=0
TRAN  CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=16158 IP=65   FP=1025  TP=0
TRAN  CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=600   IP=1205 FP=1150  TP=0
TRAN  CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=600   IP=900  FP=1150  TP=0
TRAN  CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=600   IP=1150 FP=690   TP=0
TRAN  CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=16158 IP=255  FP=1025  TP=0
TRAN  CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=32316 IP=1025 FP=1025  TP=0
TRAN  CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=16158 IP=1025 FP=185   TP=0
TRAN  CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=16158 IP=185  FP=105   TP=0
TRAN  CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=16158 IP=105  FP=15    TP=0
TRAN  CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=16158 IP=15   FP=15    TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=2262  IP=40   FP=1578  TP=0
TRAN  CA=220
TRAN  CA=221
TRAN  CA=222  *IS=1 FS=1 IT=526 FT=130  TT=0     FL=32316 IP=1025  FP=1025 TP=0
TRAN  CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=32316 IP=1025  FP=1025 TP=0
TRAN  CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=32316 IP=1025  FP=215  TP=0
TRAN  CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=32316 IP=215   FP=15   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140  EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151  EX=8.5 *Tavg=324.5
PAIR  CA=204 CO=10.5 DI=0.162  EX=8.5 *Tavg=545.5
PAIR  CA=205 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=206 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=207 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=208 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=209 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=210 CO=9.9  DI=0.156  EX=8.5 *Tavg=413.0
PAIR  CA=211 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=212 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156  EX=8.5 *Tavg=424.5
PAIR  CA=214 CO=10.4 DI=0.162  EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158  EX=8.5 *Tavg=462.0
PAIR  CA=216 CO=9.5  DI=0.152  EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149  EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143  EX=8.5 *Tavg=162.5
PAIR  CA=219 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
*PAIR CA=220 CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
```

```
*PAIR  CA=221 CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
*PAIR   CA=222 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157  EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147  EX=8.5 *Tavg=237.5
```

Reg3.inp

```
*----------------------------------------
*BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=25
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=225  PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN  CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=2262  IP=15   FP=1115  TP=0
TRAN  CA=202  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=2262  IP=1115 FP=65    TP=0
TRAN  CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=16158 IP=65   FP=1050  TP=0
TRAN  CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=600   IP=1230 FP=1175  TP=0
TRAN  CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=600   IP=925  FP=1175  TP=0
TRAN  CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=600   IP=1175 FP=715   TP=0
TRAN  CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=16158 IP=280  FP=1050  TP=0
TRAN  CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=32316 IP=1050 FP=1050  TP=0
TRAN  CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=16158 IP=1050 FP=210   TP=0
TRAN  CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=16158 IP=210  FP=130   TP=0
TRAN  CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=16158 IP=130  FP=40    TP=0
TRAN  CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=16158 IP=40   FP=40    TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=2262  IP=40   FP=1578  TP=0
TRAN  CA=220
TRAN  CA=221
TRAN  CA=222  *IS=1 FS=1 IT=526 FT=130  TT=0     FL=32316 IP=1050  FP=1050 TP=0
TRAN  CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=32316 IP=1050  FP=1050 TP=0
```

```
TRAN   CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=32316 IP=1050  FP=240  TP=0
TRAN  CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=32316 IP=240   FP=40   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR   CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR   CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR  CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR   CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR   CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR   CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR   CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
*PAIR  CA=220 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR  CA=221 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR   CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg3B.inp

```
*----------------------------------------
*BEGIN REGION 3B TRANSIENT CARDS & GEOMETRY AFFECTED BY RHR INITIATION
*----------------------------------------
*
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=180  PR=25
*
TRAN CA=220 IS=1 FS=1 IT=180  FT=225 TT=60 FL=22858 IP=40 FP=40
TRAN CA=221 IS=1 FS=1 IT=225 FT=180  TT=60 FL=22858 IP=40 FP=40
PAIR  CA=220 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
PAIR  CA=221 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
```

Reg4.inp

```
*----------------------------------------
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
```

```
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=25
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=180  PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN  CA=201 IS=1 FS=1 IT=70  FT=100 TT=1800  FL=905    IP=15   FP=1115 TP=0
TRAN  CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800  FL=905    IP=1115 FP=65   TP=0
TRAN  CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=6463   IP=65   FP=1050 TP=0
TRAN  CA=204 IS=1 FS=1 IT=549 FT=542 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=205 IS=1 FS=1 IT=542 FT=526 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=206 IS=1 FS=1 IT=526 FT=542 TT=900   FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=207 IS=1 FS=1 IT=542 FT=526 TT=360   FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=208 IS=1 FS=1 IT=526 FT=516 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=209 IS=1 FS=1 IT=516 FT=526 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=210 IS=1 FS=1 IT=526 FT=300 TT=220   FL=400    IP=1230 FP=1175 TP=0
TRAN  CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980  FL=400    IP=925  FP=1175 TP=0
TRAN  CA=212 IS=1 FS=1 IT=500 FT=300 TT=180   FL=400    IP=1175 FP=715  TP=0
TRAN  CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964  FL=6463   IP=280  FP=1050 TP=0
TRAN  CA=214 IS=1 FS=1 IT=526 FT=549 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264  FL=6463   IP=1050 FP=210  TP=0
TRAN  CA=216 IS=1 FS=1 IT=375 FT=330 TT=600   FL=6463   IP=210  FP=130  TP=0
TRAN  CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780  FL=6463   IP=130  FP=40   TP=0
TRAN  CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500  FL=9143   IP=40   FP=40   TP=0
TRAN  CA=219 IS=1 FS=1 IT=100 FT=100 TT=0     FL=905    IP=40   FP=1578 TP=0
TRAN  CA=220 IS=1 FS=1 IT=180 FT=225 TT=60    FL=9143   IP=40   FP=40   TP=0
TRAN  CA=221 IS=1 FS=1 IT=225 FT=180 TT=60    FL=9143   IP=40   FP=40   TP=0
TRAN  *CA=222 IS=1 FS=1 IT=526 FT=130 TT=0    FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=223 IS=1 FS=1 IT=130 FT=526 TT=0     FL=12926  IP=1050 FP=1050 TP=0
TRAN  CA=224 IS=1 FS=1 IT=526 FT=375 TT=600   FL=12926  IP=1050 FP=240  TP=0
TRAN  CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900  FL=12926  IP=240  FP=40   TP=0
*
PAIR CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=220 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
PAIR CA=221 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
```

```
*PAIR  CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR  CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR  CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg5.inp

```
*----------------------------------------
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 336
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=25
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=180  PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN  CA=201 IS=1 FS=1 IT=70  FT=100  TT=1800  FL=452   IP=15   FP=1115 TP=0
TRAN  CA=202 IS=1 FS=1 IT=100 FT=100  TT=1800  FL=452   IP=1115 FP=65   TP=0
TRAN  CA=203 IS=1 FS=1 IT=100 FT=549  TT=16164 FL=3232  IP=65   FP=1050 TP=0
TRAN  CA=204 IS=1 FS=1 IT=549 FT=542  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=205 IS=1 FS=1 IT=542 FT=526  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=206 IS=1 FS=1 IT=526 FT=542  TT=900   FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=207 IS=1 FS=1 IT=542 FT=526  TT=360   FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=208 IS=1 FS=1 IT=526 FT=516  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=209 IS=1 FS=1 IT=516 FT=526  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=210 IS=1 FS=1 IT=526 FT=300  TT=220   FL=200   IP=1230 FP=1175 TP=0
TRAN  CA=211 IS=1 FS=1 IT=300 FT=500  TT=1980  FL=200   IP=925  FP=1175 TP=0
TRAN  CA=212 IS=1 FS=1 IT=500 FT=300  TT=180   FL=200   IP=1175 FP=715  TP=0
TRAN  CA=213 IS=1 FS=1 IT=300 FT=549  TT=8964  FL=3232  IP=280  FP=1050 TP=0
TRAN  CA=214 IS=1 FS=1 IT=526 FT=549  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=215 IS=1 FS=1 IT=549 FT=375  TT=6264  FL=3232  IP=1050 FP=210  TP=0
TRAN  CA=216 IS=1 FS=1 IT=375 FT=330  TT=600   FL=3232  IP=210  FP=130  TP=0
TRAN  CA=217 IS=1 FS=1 IT=330 FT=225  TT=3780  FL=3232  IP=130  FP=40   TP=0
TRAN  CA=218 IS=1 FS=1 IT=225 FT=100  TT=4500  FL=4571  IP=40   FP=40   TP=0
TRAN  CA=219 IS=1 FS=1 IT=100 FT=100  TT=0     FL=452   IP=40   FP=1578 TP=0
TRAN  CA=220 IS=1 FS=1 IT=180 FT=225  TT=60    FL=4572  IP=40   FP=40   TP=0
TRAN  CA=221 IS=1 FS=1 IT=225 FT=180  TT=60    FL=4572  IP=40   FP=40   TP=0
TRAN  *CA=222 IS=1 FS=1 IT=526 FT=130 TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=223 IS=1 FS=1 IT=130 FT=526  TT=0     FL=6463  IP=1050 FP=1050 TP=0
TRAN  CA=224 IS=1 FS=1 IT=526 FT=375  TT=600   FL=6463  IP=1050 FP=240  TP=0
```

```
TRAN  CA=225 IS=1 FS=1 IT=375 FT=100  TT=9900  FL=6463  IP=240  FP=40   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR   CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR   CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR   CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR   CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR   CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR   CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR   CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR   CA=220 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
PAIR   CA=221 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
*PAIR   CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg6.inp

```
*----------------------------------------
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 336
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=25
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=180  PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=452  IP=15   FP=1115 TP=0
```

```
TRAN CA=202  IS=1 FS=1 IT=100 FT=100  TT=1800  FL=452  IP=1115 FP=65   TP=0
TRAN CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=3232 IP=65   FP=1050 TP=0
TRAN  CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=200  IP=1230 FP=1175 TP=0
TRAN CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=200  IP=925  FP=1175 TP=0
TRAN  CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=200  IP=1175 FP=715  TP=0
TRAN CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=3232 IP=280  FP=1050 TP=0
TRAN  CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=3232 IP=1050 FP=210  TP=0
TRAN  CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=3232 IP=210  FP=130  TP=0
TRAN CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=3232 IP=130  FP=40   TP=0
TRAN CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=4572 IP=40   FP=40   TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=452  IP=40   FP=1578 TP=0
TRAN  CA=220  IS=1 FS=1 IT=180 FT=225  TT=60    FL=4572 IP=40   FP=40   TP=0
TRAN  CA=221  IS=1 FS=1 IT=225 FT=180  TT=60    FL=4572 IP=40   FP=40   TP=0
TRAN  CA=222  IS=1 FS=1 IT=526 FT=130  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=6463 IP=1050 FP=1050 TP=0
TRAN  CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=6463 IP=1050 FP=240  TP=0
TRAN CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=6463 IP=240  FP=40   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR   CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR   CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR   CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR   CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR  CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR   CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR   CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR   CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR   CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR   CA=220 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
PAIR   CA=221 CO=8.8  DI=0.145 EX=8.5 *Tavg=202.5
PAIR   CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg7A.inp

```
*----------------------------------------
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1010
OPER CA=4   TE=542  PR=1010
```

```
OPER CA=5   TE=526  PR=1010
OPER CA=6   TE=542  PR=1010
OPER CA=7   TE=526  PR=1010
OPER CA=8   TE=516  PR=1010
OPER CA=9   TE=526  PR=1010
OPER CA=10  TE=300  PR=1135
OPER CA=11  TE=500  PR=1135
OPER CA=12  TE=300  PR=675
OPER CA=13  TE=549  PR=1010
OPER CA=14  TE=549  PR=1010
OPER CA=15  TE=375  PR=170
OPER CA=16  TE=330  PR=90
OPER CA=17  TE=225  PR=0
OPER CA=18  TE=100  PR=0
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=0
OPER CA=21  TE=225  PR=0
OPER CA=22  TE=130  PR=1010
OPER CA=23  TE=526  PR=1010
OPER CA=24  TE=375  PR=200
OPER CA=25  TE=100  PR=0
*
TRAN CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=143  IP=15   FP=1115 TP=0
TRAN CA=202  IS=1 FS=1 IT=100 FT=100  TT=1800  FL=143  IP=1115 FP=65   TP=0
TRAN CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=300  IP=65   FP=1025 TP=0
TRAN CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=364  IP=1025 FP=1025 TP=0
TRAN CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=358  IP=1025 FP=1025 TP=0
TRAN CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=358  IP=1025 FP=1025 TP=0
TRAN CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=358  IP=1025 FP=1025 TP=0
TRAN CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=351  IP=1025 FP=1025 TP=0
TRAN CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=351  IP=1025 FP=1025 TP=0
TRAN CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=306  IP=1205 FP=1150 TP=0
TRAN CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=301  IP=900  FP=1150 TP=0
TRAN CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=301  IP=1150 FP=690  TP=0
TRAN CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=310  IP=255  FP=1025 TP=0
TRAN CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=360  IP=1025 FP=1025 TP=0
TRAN CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=320  IP=1050 FP=185  TP=0
TRAN CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=282  IP=185  FP=105  TP=0
TRAN CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=260  IP=105  FP=15   TP=0
TRAN CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=6700 IP=15   FP=15   TP=0
TRAN CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=158  IP=40   FP=1578 TP=0
TRAN CA=220 *IS=1 FS=1 IT=225 FT=225  TT=60    FL=6700 IP=15   FP=15   TP=0
TRAN CA=221 *IS=1 FS=1 IT=225 FT=225  TT=60    FL=6700 IP=15   FP=15   TP=0
TRAN CA=222 *IS=1 FS=1 IT=526 FT=130  TT=0     FL=272  IP=1025 FP=1025 TP=0
TRAN CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=272  IP=1025 FP=1025 TP=0
TRAN CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=320  IP=1025 FP=215  TP=0
TRAN CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=234  IP=215  FP=15   TP=0
*
PAIR CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
```

```
PAIR  CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR   CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR   CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR   CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
*PAIR   CA=220 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR   CA=221 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR   CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg7B.inp

```
*----------------------------------------
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*----------------------------------------
*
OPER CA=1   TE=100  PR=120
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=150  PR=120
OPER CA=4   TE=150  PR=120
OPER CA=5   TE=150  PR=120
OPER CA=6   TE=150  PR=120
OPER CA=7   TE=150  PR=120
OPER CA=8   TE=150  PR=120
OPER CA=9   TE=150  PR=120
OPER CA=10  TE=150  PR=120
OPER CA=11  TE=150  PR=120
OPER CA=12  TE=150  PR=120
OPER CA=13  TE=150  PR=120
OPER CA=14  TE=150  PR=120
OPER CA=15  TE=150  PR=120
OPER CA=16  TE=150  PR=120
OPER CA=17  TE=150  PR=100
OPER CA=18  TE=100  PR=0
OPER CA=19  TE=100  PR=450
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=150  PR=25
OPER CA=22  TE=150  PR=100
OPER CA=23  TE=150  PR=1035
OPER CA=24  TE=150  PR=100
OPER CA=25  TE=150  PR=100
*
TRAN CA=201 IS=1 FS=1 IT=70  FT=100 TT=1800  FL=143 IP=15  FP=135 TP=0
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800  FL=143 IP=135 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=300 IP=65  FP=135  TP=0
TRAN  CA=204
TRAN  CA=205
TRAN  CA=206
TRAN  CA=207
TRAN  CA=208
TRAN  CA=209
TRAN  CA=210
TRAN CA=211
TRAN  CA=212
TRAN CA=213
TRAN  CA=214
TRAN CA=215
```

```
TRAN  CA=216
TRAN CA=217
TRAN CA=218 IS=1 FS=1 IT=225 FT=100  TT=4500 FL=6700 IP=15  FP=15 TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100  TT=0  FL=143  IP=15  FP=465 TP=0
TRAN  CA=220  IS=1 FS=1 IT=150 FT=225  TT=60 FL=6700 IP=40  FP=40 TP=0
TRAN  CA=221 *IS=1 FS=1 IT=150 FT=150   TT=0 FL=6700 IP=40  FP=40 TP=0
TRAN  CA=222
TRAN  CA=223
TRAN  CA=224
TRAN CA=225
*
PAIR  CA=201 CO=27.6 DI=0.529 EX=6.4 *Tavg=85.0
PAIR  CA=202 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
PAIR  CA=203 CO=27.6 DI=0.506 EX=6.4 *Tavg=125.0
*PAIR  CA=204 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=205 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=206 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=207 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=208 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=209 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=210 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=211 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=212 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=213 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=214 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=215 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=216 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=217 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR  CA=218 CO=27.6 DI=0.496 EX=6.4 *Tavg=162.5
PAIR  CA=219 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
PAIR  CA=220 CO=27.6 DI=0.489 EX=6.4 *Tavg=187.5
PAIR  CA=221 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=222 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=223 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=224 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=225 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
```

Reg8.inp

```
*----------------------------------------
*BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=25
```

```
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=225  PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN CA=201 IS=1 FS=1 IT=70  FT=100  TT=1800  FL=23.5  IP=15   FP=1115 TP=0
TRAN CA=202 IS=1 FS=1 IT=100 FT=100  TT=1800  FL=23.5  IP=1115 FP=65   TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549  TT=16164 FL=168   IP=65   FP=1050 TP=0
TRAN  CA=204 IS=1 FS=1 IT=549 FT=542  TT=0     FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=205 IS=1 FS=1 IT=542 FT=526  TT=0     FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=206 IS=1 FS=1 IT=526 FT=542  TT=900   FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=207 IS=1 FS=1 IT=542 FT=526  TT=360   FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=208 IS=1 FS=1 IT=526 FT=516  TT=0     FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=209 IS=1 FS=1 IT=516 FT=526  TT=0     FL=335   IP=1050 FP=1050 TP=0
TRAN  CA=210 IS=1 FS=1 IT=526 FT=300  TT=220   FL=6     IP=1230 FP=1175 TP=0
TRAN CA=211 IS=1 FS=1 IT=300 FT=500  TT=1980  FL=6     IP=925  FP=1175 TP=0
TRAN  CA=212 IS=1 FS=1 IT=500 FT=300  TT=180   FL=6     IP=1175 FP=715  TP=0
TRAN CA=213 IS=1 FS=1 IT=300 FT=549  TT=8964  FL=167.5 IP=280  FP=1050 TP=0
TRAN  CA=214 IS=1 FS=1 IT=526 FT=549  TT=0     FL=335   IP=1050 FP=1050 TP=0
TRAN CA=215 IS=1 FS=1 IT=549 FT=375  TT=6264  FL=167.5 IP=1050 FP=210  TP=0
TRAN  CA=216 IS=1 FS=1 IT=375 FT=330  TT=600   FL=167.5 IP=210  FP=130  TP=0
TRAN CA=217 IS=1 FS=1 IT=330 FT=225  TT=3780  FL=167.5 IP=130  FP=40   TP=0
TRAN CA=218 IS=1 FS=1 IT=225 FT=100  TT=4500  FL=167.5 IP=40   FP=40   TP=0
TRAN  CA=219 IS=1 FS=1 IT=100 FT=100  TT=0     FL=23.5  IP=40   FP=1578 TP=0
TRAN  CA=220
TRAN  CA=221
TRAN  CA=222 *IS=1 FS=1 IT=526 FT=130  TT=0     FL=335 IP=1050  FP=1050 TP=0
TRAN  CA=223 IS=1 FS=1 IT=130 FT=526  TT=0     FL=335 IP=1050  FP=1050 TP=0
TRAN  CA=224 IS=1 FS=1 IT=526 FT=375  TT=600   FL=335 IP=1050  FP=240  TP=0
TRAN CA=225 IS=1 FS=1 IT=375 FT=100  TT=9900  FL=335 IP=240   FP=40   TP=0
*
PAIR CA=201 CO=8.3  DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4  DI=0.151 EX=8.5 *Tavg=324.5
PAIR  CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR  CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR  CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR  CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR  CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR  CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR  CA=210 CO=9.9  DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR  CA=212 CO=9.8  DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=213 CO=9.9  DI=0.156 EX=8.5 *Tavg=424.5
PAIR  CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR  CA=216 CO=9.5  DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2  DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7  DI=0.143 EX=8.5 *Tavg=162.5
PAIR  CA=219 CO=8.3  DI=0.140 EX=8.5 *Tavg=100.0
*PAIR CA=220 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR CA=221 CO=9.0  DI=0.146 EX=8.5 *Tavg=225.0
*PAIR  CA=222 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR  CA=223 CO=9.4  DI=0.151 EX=8.5 *Tavg=328.0
PAIR  CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0  DI=0.147 EX=8.5 *Tavg=237.5
```

Reg9A.inp

```
*----------------------------------------
*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=549  PR=1035
OPER CA=4   TE=542  PR=1035
OPER CA=5   TE=526  PR=1035
OPER CA=6   TE=542  PR=1035
OPER CA=7   TE=526  PR=1035
OPER CA=8   TE=516  PR=1035
OPER CA=9   TE=526  PR=1035
OPER CA=10  TE=300  PR=1160
OPER CA=11  TE=500  PR=1160
OPER CA=12  TE=300  PR=700
OPER CA=13  TE=549  PR=1035
OPER CA=14  TE=549  PR=1035
OPER CA=15  TE=375  PR=195
OPER CA=16  TE=330  PR=115
OPER CA=17  TE=225  PR=25
OPER CA=18  TE=100  PR=100
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=70   PR=25
OPER CA=22  TE=130  PR=1035
OPER CA=23  TE=526  PR=1035
OPER CA=24  TE=375  PR=225
OPER CA=25  TE=100  PR=25
*
TRAN  CA=201  IS=1 FS=1 IT=70  FT=100  TT=1800  FL=204  IP=15   FP=1115 TP=0
TRAN  CA=202  IS=1 FS=1 IT=100 FT=100  TT=1800  FL=204  IP=1115 FP=65   TP=0
TRAN  CA=203  IS=1 FS=1 IT=100 FT=549  TT=16164 FL=247  IP=65   FP=1050 TP=0
TRAN  CA=204  IS=1 FS=1 IT=549 FT=542  TT=0     FL=520  IP=1050 FP=1050 TP=0
TRAN  CA=205  IS=1 FS=1 IT=542 FT=526  TT=0     FL=511  IP=1050 FP=1050 TP=0
TRAN  CA=206  IS=1 FS=1 IT=526 FT=542  TT=900   FL=511  IP=1050 FP=1050 TP=0
TRAN  CA=207  IS=1 FS=1 IT=542 FT=526  TT=360   FL=511  IP=1050 FP=1050 TP=0
TRAN  CA=208  IS=1 FS=1 IT=526 FT=516  TT=0     FL=502  IP=1050 FP=1050 TP=0
TRAN  CA=209  IS=1 FS=1 IT=516 FT=526  TT=0     FL=502  IP=1050 FP=1050 TP=0
TRAN  CA=210  IS=1 FS=1 IT=526 FT=300  TT=220   FL=437  IP=1230 FP=1175 TP=0
TRAN  CA=211  IS=1 FS=1 IT=300 FT=500  TT=1980  FL=429  IP=925  FP=1175 TP=0
TRAN  CA=212  IS=1 FS=1 IT=500 FT=300  TT=180   FL=429  IP=1175 FP=715  TP=0
TRAN  CA=213  IS=1 FS=1 IT=300 FT=549  TT=8964  FL=443  IP=280  FP=1050 TP=0
TRAN  CA=214  IS=1 FS=1 IT=526 FT=549  TT=0     FL=514  IP=1050 FP=1050 TP=0
TRAN  CA=215  IS=1 FS=1 IT=549 FT=375  TT=6264  FL=458  IP=1050 FP=210  TP=0
TRAN  CA=216  IS=1 FS=1 IT=375 FT=330  TT=600   FL=403  IP=210  FP=130  TP=0
TRAN  CA=217  IS=1 FS=1 IT=330 FT=225  TT=3780  FL=260  IP=130  FP=40   TP=0
TRAN  CA=218  IS=1 FS=1 IT=225 FT=100  TT=4500  FL=6700 IP=115  FP=115  TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100  TT=0     FL=226  IP=40   FP=1578 TP=0
TRAN  CA=220  IS=1 FS=1 IT=70  FT=225  TT=60    FL=6700 IP=40   FP=40   TP=0
TRAN  CA=221  IS=1 FS=1 IT=225 FT=70   TT=60    FL=6700 IP=40   FP=40   TP=0
TRAN  CA=222  *IS=1 FS=1 IT=526 FT=130 TT=0     FL=389  IP=1050 FP=1050 TP=0
TRAN  CA=223  IS=1 FS=1 IT=130 FT=526  TT=0     FL=389  IP=1050 FP=1050 TP=0
TRAN  CA=224  IS=1 FS=1 IT=526 FT=375  TT=600   FL=458  IP=1050 FP=240  TP=0
TRAN  CA=225  IS=1 FS=1 IT=375 FT=100  TT=9900  FL=334  IP=240  FP=40   TP=0
*
PAIR  CA=201 CO=8.3  DI=0.140  EX=8.5 *Tavg=85.0
PAIR  CA=202 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
PAIR  CA=203 CO=9.4  DI=0.151  EX=8.5 *Tavg=324.5
```

```
PAIR   CA=204 CO=10.5 DI=0.162  EX=8.5 *Tavg=545.5
PAIR   CA=205 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR   CA=206 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR   CA=207 CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR   CA=208 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR   CA=209 CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR   CA=210 CO=9.9  DI=0.156  EX=8.5 *Tavg=413.0
PAIR  CA=211 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR   CA=212 CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=213 CO=9.9  DI=0.156  EX=8.5 *Tavg=424.5
PAIR   CA=214 CO=10.4 DI=0.162  EX=8.5 *Tavg=537.5
PAIR  CA=215 CO=10.0 DI=0.158  EX=8.5 *Tavg=462.0
PAIR   CA=216 CO=9.5  DI=0.152  EX=8.5 *Tavg=352.5
PAIR  CA=217 CO=9.2  DI=0.149  EX=8.5 *Tavg=277.5
PAIR  CA=218 CO=8.7  DI=0.143  EX=8.5 *Tavg=162.5
PAIR   CA=219 CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
PAIR  CA=220 CO=8.6  DI=0.142  EX=8.5 *Tavg=147.5
PAIR  CA=221 CO=8.6  DI=0.142  EX=8.5 *Tavg=147.5
*PAIR   CA=222 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR   CA=223 CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR   CA=224 CO=10.0 DI=0.157  EX=8.5 *Tavg=450.5
PAIR  CA=225 CO=9.0  DI=0.147  EX=8.5 *Tavg=237.5
```

Reg9B.inp

```
*----------------------------------------
*BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE
675
*----------------------------------------
*
OPER CA=1   TE=100  PR=1100
OPER CA=2   TE=100  PR=50
OPER CA=3   TE=150  PR=1035
OPER CA=4   TE=150  PR=1035
OPER CA=5   TE=150  PR=1035
OPER CA=6   TE=150  PR=1035
OPER CA=7   TE=150  PR=1035
OPER CA=8   TE=150  PR=1035
OPER CA=9   TE=150  PR=1035
OPER CA=10  TE=150  PR=1160
OPER CA=11  TE=150  PR=1160
OPER CA=12  TE=150  PR=700
OPER CA=13  TE=150  PR=1035
OPER CA=14  TE=150  PR=1035
OPER CA=15  TE=150  PR=195
OPER CA=16  TE=150  PR=115
OPER CA=17  TE=150  PR=25
OPER CA=18  TE=100  PR=100
OPER CA=19  TE=100  PR=1563
OPER CA=20  TE=225  PR=25
OPER CA=21  TE=70   PR=25
OPER CA=22  TE=150  PR=1035
OPER CA=23  TE=150  PR=1035
OPER CA=24  TE=150  PR=225
OPER CA=25  TE=150  PR=25
*
TRAN CA=201 IS=1 FS=1 IT=70  FT=100 TT=1800  FL=204  IP=15   FP=1115 TP=0
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800  FL=204  IP=1115 FP=65   TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=247  IP=65   FP=1050 TP=0
TRAN  CA=204
TRAN  CA=205
```

```
TRAN  CA=206
TRAN  CA=207
TRAN  CA=208
TRAN  CA=209
TRAN  CA=210
TRAN CA=211
TRAN  CA=212 IS=1 FS=1 IT=150 FT=150 TT=0     FL=429  IP=1175 FP=715  TP=0
TRAN CA=213
TRAN  CA=214
TRAN CA=215
TRAN  CA=216 IS=1 FS=1 IT=150 FT=150 TT=600   FL=403  IP=210  FP=130  TP=0
TRAN CA=217
TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500  FL=6700 IP=115  FP=115  TP=0
TRAN  CA=219 IS=1 FS=1 IT=100 FT=100 TT=0     FL=247  IP=40   FP=1578 TP=0
TRAN  CA=220 IS=1 FS=1 IT=70  FT=225 TT=60    FL=6700 IP=40   FP=40   TP=0
TRAN  CA=221 IS=1 FS=1 IT=150 FT=70  TT=60    FL=6700 IP=40   FP=40   TP=0
TRAN  CA=222
TRAN  CA=223
TRAN  CA=224 IS=1 FS=1 IT=150 FT=150 TT=0     FL=458  IP=1040 FP=240  TP=0
TRAN  CA=225 IS=1 FS=1 IT=150 FT=150 TT=0     FL=334  IP=240  FP=40   TP=0
*
PAIR  CA=201 CO=27.6 DI=0.521 EX=6.4 *Tavg=85.0
PAIR  CA=202 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
PAIR  CA=203 CO=27.6 DI=0.506 EX=6.4 *Tavg=125.0
*PAIR  CA=204 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=205 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=206 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=207 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=208 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=209 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=210 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=211 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR   CA=212 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=213 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=214 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=215 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR   CA=216 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=217 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR  CA=218 CO=27.6 DI=0.496 EX=6.4 *Tavg=162.5
PAIR   CA=219 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
PAIR   CA=220 CO=27.6 DI=0.500 EX=6.4 *Tavg=147.5
PAIR   CA=221 CO=27.6 DI=0.509 EX=6.4 *Tavg=110.0
*PAIR  CA=222 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
*PAIR  CA=223 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR   CA=224 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
PAIR   CA=225 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
```

APPENDIX B

PIPESTRESS Output

Output File	Description
Recirc_15.prf	Fatigue results for reduced cycle count
RHR_15.prf	Fatigue results for full 60 year cycle count

Recirc 15.prf

CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998 RVP 2007/07/26 08:42:12
Vermont Yankee Recirculation Fatigue Analysis

FATIGUE ANALYSIS AT POINT 600, WELDING TEE 600 TO 602

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	OCCURENCES NI	NJ		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
11	21	88675.	10	150		10	11	2801.	0.0036	
			0	140						
14	21	84491.	150	140		140	21	3359.	0.0417	
			10	0						
19	20	67184.	1	150		1	19	8221.	0.0001	
			0	149						
14	27	65876.	10	5		5	27	8900.	0.0006	
			5	0	45					
10	20	62914.	5	149		5	10	10868.	0.0005	
			0	144						
14	20	55493.	5	144		5	14	20012.	0.0002	
			0	139						
9	20	54186.	35	139		35	9	22574.	0.0016	
			0	104						
5	20	53739.	290	104		104	20	23541.	0.0044	
			186	0						
2	3	46216.	60	150		60	2	50502.	0.0012	
			0	90						
1	26	46169.	60	5		5	26	50782.	0.0001	
			55	0	45					
3	17	42799.	90	150		90	3	76939.	0.0012	
			0	60						
6	17	42219.	10	60		10	6	82911.	0.0001	
			0	50						
5	18	42196.	186	150		150	18	83156.	0.0018	
			36	0						
13	17	42131.	5	50		5	13	83864.	0.0001	
			0	45						
8	17	41392.	35	45		35	8	92408.	0.0004	
			0	10						

CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998 RVP 2007/07/26 08:42:12 [4
Vermont Yankee Recirculation Fatigue Analysis

FATIGUE ANALYSIS AT POINT 600, WELDING TEE 600 TO 602

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET I	PAIR J	SALT EQN.14	OCCURENCES NI	OCCURENCES NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
7	17	41326.	10	10	10	7,17	93222.	0.0001	
			0	0					
1	15	38663.	55	150	55	1	133841.	0.0004	
			0	95					
4	16	35177.	290	150	150	16	227086.	0.0007	
			140	0					
4	15	34727.	140	95	95	15	246096.	0.0004	
			45	0					
4	12	25167.	45	10	10	12	1449206.	0.0000	
			35	0					
26	27	23758.	45	45	45	,27 26	1748766.	0.0000	DYN. RANGE OF EVENT NO.
			0	0					
4	5	2773.	35	36	35	4	>100000000000.	0.0000	
			0	1					

TOTAL USAGE FACTOR = 0.0590

Notes a: Fails
f: Weld ISI
j: Rupture Location

RHR 15.prf

CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998 RVP 2007/07/26 08:44:06
Vermont Yankee Recirculation Fatigue Analysis

FATIGUE ANALYSIS AT POINT 641, SR ELBOW 640 TO 641

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	OCCURENCES NI	NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
10	20	100252.	10	300	10	10	1788.	0.0056	
			0	290					
11	21	68706.	20	300	20	11	7511.	0.0027	
			0	280					
12	20	64893.	20	290	20	12	9456.	0.0021	
			0	270					
20	21	39947.	270	280	270	20	112120.	0.0024	
			0	10					
3	21	35368.	300	10	10	21	219528.	0.0000	
			290	0					
3	17	20439.	290	300	290	3	3190872.	0.0000	
			0	10					
13	17	19221.	10	10	10	13,17	4148766.	0.0000	
			0	0					
14	18	14925.	300	300	300	14,18	>100000000000.	0.0000	
			0	0					
15	19	14877.	300	1	1	19	>100000000000.	0.0000	
			299	0					
1	15	14434.	120	299	120	1	>100000000000.	0.0000	
			0	179					
6	15	12896.	20	179	20	6	>100000000000.	0.0000	
			0	159					
9	15	12506.	70	159	70	9	>100000000000.	0.0000	
			0	89					
4	15	11542.	579	89	89	15	>100000000000.	0.0000	
			490	0					
7	27	10017.	20	5	5	27	>100000000000.	0.0000	
			15	0	45				
4	16	9993.	490	300	300	16	>100000000000.	0.0000	
			190	0					

CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998 RVP 2007/07/26 08:44:06
Vermont Yankee Recirculation Fatigue Analysis

FATIGUE ANALYSIS AT POINT 641, SR ELBOW 640 TO 641

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	OCCURENCES NI	NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
2	7	9353.	120	15	15	7	>100000000000.	0.0000	
			105	0					
2	5	9353.	105	579	105	2	>100000000000.	0.0000	
			0	474					
5	26	8235.	474	5	5	26	>100000000000.	0.0000	
			469	0	45				
26	27	2019.	45	45	45	,27 26	>100000000000.	0.0000	DYN. RANGE OF EVENT NO.
			0	0					
4	5	1570.	190	469	190	4	>100000000000.	0.0000	
			0	279					
5	8	1512.	279	70	70	8	>100000000000.	0.0000	
			209	0					

TOTAL USAGE FACTOR = 0.0128

Notes a: Fails
f: Weld ISI
j: Rupture Location

Structural Integrity Associates, Inc.	File No.: VY-16Q-308 EXHIBIT H
CALCULATION PACKAGE	**Project No.:** VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Core Spray Nozzle Finite Element Model

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-7, Appendix: A1-A17	Initial Issue	Terry J. Herrmann 07/19/2007	Roland Horvath 07/12/2007 John Staples 07/12/2007

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1.0 OBJECTIVE

The objective of this calculation is to create a finite element model of the Vermont Yankee Nuclear Power Station Core Spray Nozzle. This model will be used to develop a Green's Function to be used in a subsequent fatigue analysis.

2.0 GEOMETRY / MATERIAL PROPERTIES

A 2-D axisymmetric finite element model (FEM) of the nozzle was developed with element type PLANE82. The developed model includes the part of the pipe, the safe end, the nozzle forging, a portion of the vessel shell, and the cladding. The radius of the vessel in the finite element model was multiplied by a factor of 2 to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere, but the true geometry is a cylinder. The equation for the membrane hoop stress for a sphere is:

$$\sigma = \frac{(pressure) \times (radius)}{2 \times thickness}$$

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \frac{(pressure) \times (radius)}{thickness}$$

The factor of two was verified in Reference [11] where actual stress results were compared to the results of this analytical form.

The 2-D axisymmetric FEM was constructed using the dimensions and information from References [1 -8] based on ANSYS [9] finite element software. Figure 1 shows the resulting finite element model.

The materials of the various components of the model are listed below:

- Safe End – SB 166 [1] (72Ni-15Cr-8Fe, N06600)
- 8Ø x 10Ø Conc. Reduction – SA312 TP304 [7] (18Cr-8Ni)
- Nozzle Forging – SA508 Class II [1] (¾ Ni-1/2Mo-1/3 Cr-V)
- Vessel – SA533 Grade B [7] (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 TP 304 [7] (18Cr-8Ni)

Note: In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original design [7]. Later, this piping section was replaced by a new material (SA403 T316L) [10]. These two stainless steels have the same modulus of elasticity and thermal coefficient properties.

Material properties for these materials are based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8] and are shown in Table 1. The properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which will be applied to the FEM model for Green's Function development.

3.0 PROGRAM INPUT

The input file, VY_CSN_GEOM.inp (included in Appendix A), creates the finite element model for the core spray nozzle.

4.0 REFERENCES

1. Reactor 8 In. Dia. Nozzles Mk. N5A & B, 5920-00624 Rev. 8, SI File No. VY-16Q-207.

2. Core Spray Nozzle Weld Overlay Profile N5A & N5B, 5920-06813, Sh. 1 of 2 Rev. 0, SI File No. VY-16Q-206.

3. N5A/B Thermal Sleeve Details, 5920-00898, Rev 1, SI File No. VY-16Q-206.

4. Special Safe End Forging for Nozzles N2A/B & N5A/B, 5920-00655, Rev. 6, SI File No. VY-16Q-206.

5. Special Forgings for Nozzles N5A & N5B, 5920-00069, Rev 1, SI File No. VY-16Q-206.

6. Core Spray Nozzle Weld Overlay Profile N5A & N5B, 5920-06813, Sh. 2 of 2 Rev. 0, SI File No. VY-16Q-204.

7. CB&I RPV Stress Report, Section S7, "Stress Analysis Core Spray and Flooding Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201, SI File No. VY-16Q-206.

8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.

9. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.

10. "10" x 8" SA403 T316L CONC REDUCER", Page 18 of Attachment 2 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

11. SI Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions".

Table 1: Material Properties @ 300°F (1)

Material ID	Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft²/hr	Specific Heat, Btu/lb-°F [C] (3)
2	Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
2	Weld Overlay	INCONEL 82	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
1	Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193
3	Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193
4	3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252
4	8Ø x 10Ø Conc. Reduction(2)	SA312 TP304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252

Notes:

1. The material properties applied in the analyses are taken from ASME Code, Section II, Part D 1998 Edition, with 2000 information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a for the original design code is acceptable, since later editions typically reflect more accurate material properties than wa editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for dens assumed typical values [8].
2. In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original c was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity and the
3. Calculated as $[k/(\rho d)]/12^3$.

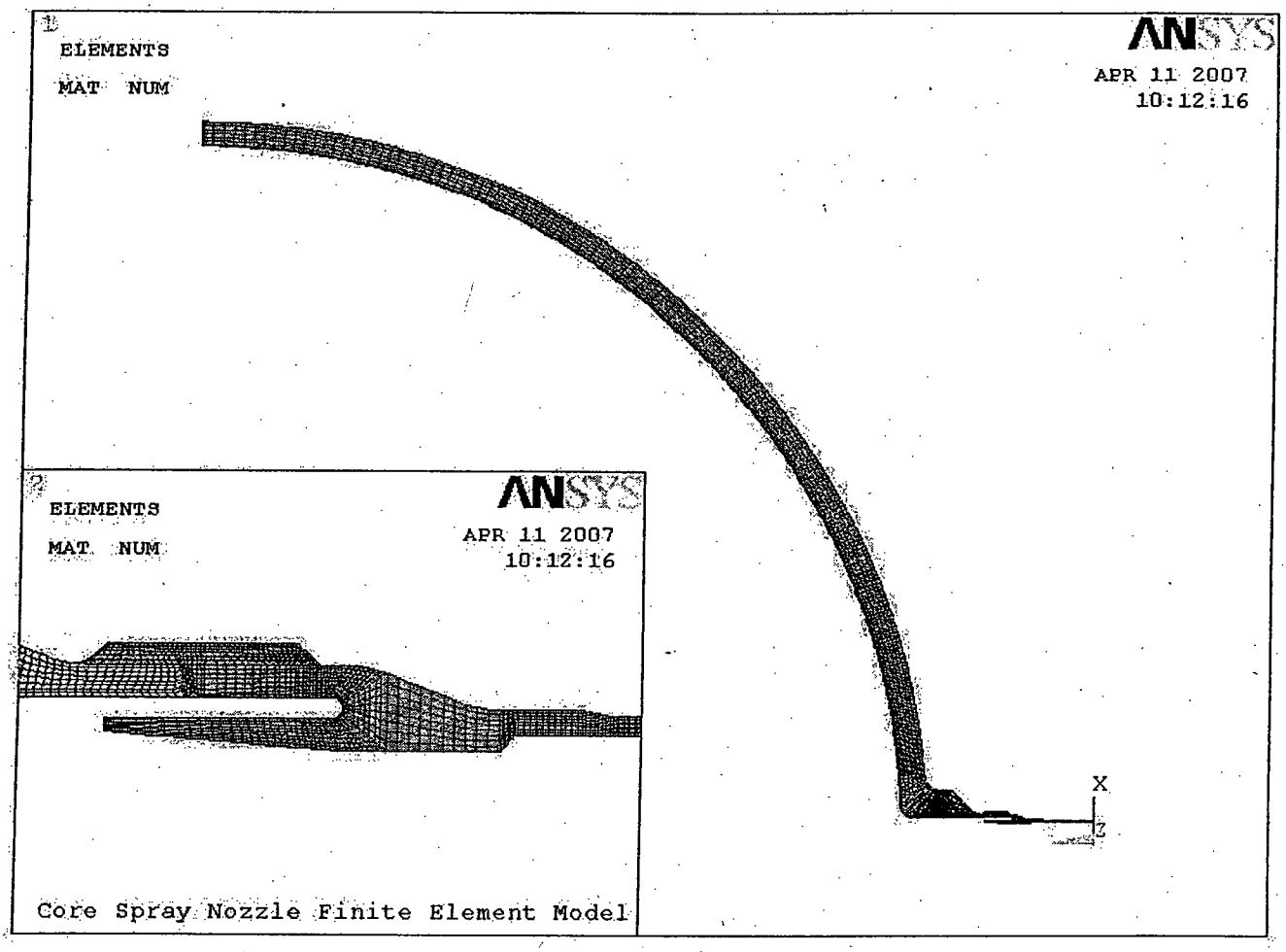

Figure 1: ANSYS Finite Element Model

APPENDIX A

VY_CSN_GEOM.INP

```
!finish
!/clear,start
!/prep7

et,1,PLANE182,,,1    !Axisymmetric
/com, ****************************
/com, Material Properties @T=300F
/com, ****************************
/COM, Material #1 (Nozzle: SA-508 Class II, 3/4Ni-1/2Mo-1/3Cr-V)
mp,ex  ,1,26.7E+06
mp,alpx,1,7.3E-06
mp,kxx ,1,23.4   /3600/12
mp,c   ,1,0.1193277
mp,nuxy,1,0.3
mp,dens,1,0.283

/COM, Material #2 (Safe End: N06600, Inconel 82 Weld Overlay)
mp,ex  ,2,29.8E+06
mp,alpx,2,7.9E-06
mp,kxx ,2,9.6   /3600/12
mp,c   ,2,0.1157407
mp,nuxy,2,0.29
mp,dens,2,0.3

/COM, Material #3 (Vessel: SA-533 Grade B, Mn-1/2Mo-1/2Ni)
mp,ex  ,3,28.00E+06
mp,alpx,3,7.7E-06
mp,kxx ,3,23.4   /3600/12
mp,c   ,3,0.1193277
mp,nuxy,3,0.3
mp,dens,3,0.283

/COM, Material #4 (3/16 Clad: SA-240 TP304, 8-10 Diam. Conc. Red.: SA-312 TP 304, Thermal
Sleeve: SA-312 TP304)
mp,ex  ,4,27.0E+06
mp,alpx,4,9.8E-06
mp,kxx ,4,9.8   /3600/12
mp,c   ,4,0.1252495
mp,nuxy,4,0.3
mp,dens,4,0.283


/com, **********************
/com, Geometric Parameters
/com, **********************
*AFUN,deg
```

```
/com, pipe parameters
*set, pID, 9.834
*set, pOD, 10.815
*set, pL,        8

k, 1, pID/2, 0
k, 2, POD/2, 0
k, 3, POD/2, pL
k, 4, PID/2, pL

l, 1, 2
l, 2, 3
l, 3, 4
l, 4, 1


/com, *******************
/com, Safe End Parameters
/com, *******************
*set, seBX, pL
*set, seID01, 9.834
*set, seID02, 9
*set, seID03, 9 + 31/32
*set, seID04, 11+ 3/4

*set, seOD01, 10.815
*set, seOD02, 11 + 1/6
*set, seOD03, 13 + 27/64
*set, seOD04, 10 + 11/16

*set, seL01,  3 + 1/32
*set, seL02,  7/8
*set, seL03,  1+11/16
*set, seL04,  13/32
*set, seL05,  4
*set, seL06,  3+1/2
*set, seL07,  12+4+1/16
*set, seL08,  seL07-(seL01+seL02+seL03+seL04+seL05+seL06)
*set, seR01,  3
*set, seR02,  3/4
*set, seR03,  1/4
*set, seR04,  1/8


k, 5, seOD01/2, seBX+seL01
k, 6, seOD02/2, seBX+seL01+seL02
```

```
k, 7, seOD02/2, seBX+seL01+seL02+seL03+seL04 +.496
k, 8, (seOD02+seOD03)/4, seBX+seL01+seL02+seL03+seL04+seL05/2
k, 9, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 10, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06
k, 11, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06
k, 12, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 13, seOD04/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 14, seOD04/2, seBX+seL07
k, 15, seID03/2, seBX+seL07
k, 16, seID02/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 17, seID02/2, seBX+seL01+seL02+seL03+seL04
k, 18, seID01/2, seBX+seL01+seL02+seL03

l, 3, 5
l, 5, 6
l, 6, 7
l, 9, 10
l, 10, 11
l, 11, 12
l, 12, 13
l, 13, 14
l, 14, 15
l, 15, 16
l, 16, 17
l, 17, 18
l, 18, 4

k, 19, seOD02/2+seR01, seBX+seL01+seL02+seL03+seL04 +.496
k, 8, seOD02/2+seR01, seBX+seL01+seL02+seL03+seL04+seR01 +.496
larc, 7, 8, 19, seR01

k, 20, seOD03/2-seR01, seBX+seL01+seL02+seL03+seL04+seL05
k, 21, seOD03/2-seR01, seBX+seL01+seL02+seL03+seL04+seL05-seR01
larc, 9, 21, 20, seR01
L2ANG,19,18,0,0,,,
ldele, 20, 21, ,1

lfillt, 5, 6, seR02
lfillt, 6, 7, seR02
lfillt, 10, 11, seR03
lfillt, 11, 12, seR03
lfillt, 15, 16, seR04
lfillt, 16, 17, seR04

/com, weld 1/8 gap
*set, wgap, 1/8
```

```
k, 40, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06 + wgap
k, 41, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06 + wgap

l, 10, 40
l, 40, 41
l, 41, 11

/com, **********************
/com, Nozzle
/com, **********************

*set, nID01, seOD01
*set, nOD01, seOD03
*set, nOD02, 24+1/4
*set, nOD03, 2*12+7.25
*set, nOD04, 2*12+7.25-1-1/8

*set, nL01, 4+5/16
*set, nL02, 5+3/8
*set, nL03, 5+1/8+5+5/8
*set, nW01, 1/16
*set, wClad, 3/16
*set, wReactor, 5+5/8-wClad
*set, nL04, 7/16
*set, nR01, 1/4
*set, nR02, 3/16
*set, nR03, (8*12+7)*2
*set, nR04, 2.5
*set, nR05, nR04-wClad
*set, nR06, 3.5
*set, nR07, 3 +7/8
*set, nR08, 0.5

K, 42, KX(11) + nW01, KY(11)
K, 43, KX(41) + wClad, KY(41)+nL04+nR01
K, 44, KX(43) + nR01, KY(43)
K, 46, KX(44) + nR01*sin(15), KY(44)-nR01*cos(15)
K, 47, KX(46) + 10*nR01*cos(15), KY(46)+10*nR01*sin(15)
K, 48, KX(43), KY(43) + 24
K, 49, KX(41), KY(41) + 24
K, 50, KX(40), KY(40) + nL01
K, 51, KX(44) + (nR01+wClad)*sin(15), KY(44)-(nR01+wClad)*cos(15)
K, 52, KX(51) + 10*nR01*cos(15), KY(51)+10*nR01*sin(15)
K, 53, KX(51) - 10*nR01*cos(15), KY(51)-10*nR01*sin(15)
K, 54, KX(42), KY(42)+wClad*2
larc, 43, 46, 44, nR01
```

```
L, 46, 47
L, 43, 48
L, 41, 49
L, 40, 50
L, 53, 52
L, 42, 54

LOVLAP, 35,36
LDELE, 39, 40,,0

LOVLAP, 31,34,38
LDELE, 40,42,2,0

lfillt, 37, 35, nR02

K, 60, nOD02/2, KY(40) + nL01+nL02
K, 61, nOD02/2, KY(40) + nL01+nL02+nL03
K, 62, 0, KY(40) + nL01+nL02+nL03+nR03
K, 63, 0, KY(62) - nR03
K, 64, nR03, KY(62)
K, 65, 0, KY(63)-wClad
K, 66, nR03+wClad, KY(62)
K, 67, 0, KY(65)-wReactor
K, 68, nR03+wReactor, KY(62)

LARC, 63, 64, 62, nR03
LARC, 65, 66, 62, nR03+wClad
LARC, 67, 68, 62, nR03+wReactor

L, 64, 66
L, 66, 68

LOVLAP, 34, 33
LDELE, 46,47

LOVLAP, 32, 38
LDELE, 34
LDELE, 46

LFILLT,45,48,nR04, ,
LFILLT,33,47,nR05, ,

L, 50, 60
L, 60, 61

LOVLAP, 40, 46
```

```
LDELE, 50, 51
LFILLT,49,52,nR06
LFILLT,38,41,nR07
LFILLT,49,38,nR08

/com, Nozzle and Vessel border
K,      80,     nOD03/2, KY(60)+2*nL03
K,      81, nOD03/2, KY(60)
K,      82,     nOD04/2, KY(60)+2*nL03
K,      83, nOD04/2, KY(60)
L,      80, 81
L,      82, 83

LPTN, 53, 48
LPTN, 51, 52
LDELE, 56, 59,1,0
LSTR,  76, 75
LPTN, 51, 47
KL,40,0.5, ,
KL,34,0.5, ,
KL,32,0.5, ,

LSTR,  78, 79
LSTR,  79, 84

K, 90, KX(73)+wReactor*2*cos(160), KY(73)+wReactor*2*sin(160)
L, 73, 90
LPTN, 59, 33
LPTN, 63, 45
LDELE, 65

K, 91, KX(71)+wReactor*2*cos(170), KY(71)+wReactor*2*sin(170)
L, 71, 91
LPTN, 45, 60
LPTN, 33, 67
LDELE, 69
KCENTER,KP,69,78,70,0
LSTR,     89,     58
LSTR,     89,     57

LPTN, 40, 33, 67
LDELE, 73, 74

L, 58, 56
L, 57, 55
```

```
/com, **********************
/com, Weld Overlay
/com, **********************
*set, woA, 3.100
*set, woB, 0.781
*set, woC, 2.500
*set, woD, 3.734
*set, woE, 3.480
*set, woF, 6.310
*set, woG, 8.313
*set, woH, 0.535
*set, woR01, 7/16

K, 80, KX(40), KY(40)-wgap/2-woA
K, 81, KX(80)+woH, KY(80)+woH
K, 83, KX(40), KY(40)-wgap/2+woB/2+woC
K, 82, KX(83)+woH, KY(83)-woH

L, 80, 81
L, 81, 82
L, 82, 83


LPTN, 74, 46
LDELE, 79
LFILLT,78,76,woR01, ,
LSTR,     94,     96

/com, *************************
/com, Heat transfer coef. points
/com, *************************
*set, tsL01, 2.25
*set, tsL02, 3.5

K, 100, KX(41),      KY(11)+seL08+tsL01
K, 101, KX(41)+wClad, KY(11)+seL08+tsL01

K, 102, KX(41),      KY(11)+seL08+tsL01+tsL02
K, 103, KX(41)+wClad, KY(11)+seL08+tsL01+tsL02
L, 100, 101
L, 102, 103

LDELE, 51
LDELE, 47
```

```
K, 104, KX(103)+wReactor*cos(-20), KY(103)+wReactor*sin(-20)
K, 105, KX(101)+wReactor*cos(-10), KY(101)+wReactor*sin(-10)
L, 103, 104
L, 101, 105
LPTN, 38, 47
LPTN, 76, 51

LDELE, 86
LDELE, 84
LDELE, 65
LDELE, 68

LDELE, 63
LDELE, 45
LDELE, 66
LDELE, 60

LSTR,   43,   101
LSTR,  101,   103
LSTR,  103,    85
LSTR,   86,   102
LSTR,  102,   100
LSTR,  100,    41

LDIV,30,0.5, ,2,0
K, 106, KX(99)+wReactor*cos(200), KY(99)+wReactor*sin(200)
K, 107, KX(38)+wReactor*cos(160), KY(38)+wReactor*sin(160)
L, 99, 106
L, 38, 107
LPTN, 66, 84
LPTN, 88, 76

LDELE, 89,90
LSTR,   99,   38
LDELE, 28
LSTR,   26,    9
LSTR,   29,   16

LCOMB,11,23,0
LCOMB,11,24,0
LDIV,11, , ,3,0

K, 110, KX(22)+wReactor*cos(180), KY(22)+wReactor*sin(180)
L, 110, 22
LPTN, 15, 89
LDELE, 94
```

```
LSTR,    28,    111
LSTR,    27,    22
LSTR,    17,    7
K, 112, KX(33)+wReactor, KY(33)
L, 33, 112
LPTN, 7, 95
LDELE, 99
K, 114, KX(25)+wReactor*cos(180), KY(25)+wReactor*sin(180)
L, 114,25
K, 115, KX(8)+wReactor*cos(180), KY(8)+wReactor*sin(180)
L, 115, 8
LPTN, 95,17
LPTN, 7,101
LDELE, 102,103


/com, *************************
/com, Creating Areas and Meshing
/com, *************************
allsel,all,all

MSHKEY,1    ! MAPPED MESHING
AL,1,2,3,4
MAT,4                    ! Pipe
LESIZE,1,,,8
LESIZE,3,,,8
LESIZE,2,,,20
LESIZE,4,,,20
AMESH, 1

MAT,2                    ! Safe End
AL, 3, 5, 100, 99
LESIZE,3,,,8
LESIZE,100,,,8
LESIZE,5,,,20
LESIZE,99,,,20
AMESH, 2

LCOMB, 20,6,0
LCOMB, 6,21,0
AL, 100, 6, 17, 104
LESIZE,100,,,8
LESIZE,17,,,8
LESIZE,6,,,10
LESIZE,104,,,10
AMESH, 3
```

```
AL, 17, 97, 98, 95
LESIZE,17,,,8
LESIZE,98,,,8
LESIZE,97,,,10
LESIZE,95,,,10
AMESH, 4

LDELE, 94
LSTR,     7,     30
LCOMB, 26,16,0
LCOMB, 16,25,0
AL, 98, 96, 7, 16
LESIZE,98,,,8
LESIZE,7,,,8
LESIZE,96,,,8
LESIZE,16,,,8
AMESH, 5


LCOMB, 18, 22
AL, 7, 18, 92, 93
LESIZE,7,,,8
LESIZE,92,,,8
LESIZE,18,,,10
LESIZE,93,,,10
AMESH, 6

AL, 92, 89, 23, 15
LESIZE,92,,,8
LESIZE,23,,,8
LESIZE,89,,,8
LESIZE,15,,,8
AMESH, 7

AL, 15, 24, 88, 90
LESIZE,15,,,8
LESIZE,24,,,8
LESIZE,88,,,8
LESIZE,90,,,8
AMESH, 8

AL, 89, 19, 28, 11
LESIZE,89,,,8
LESIZE,19,,,8
LESIZE,28,,,8
```

```
LESIZE,11,,,8
AMESH, 9


AL, 88, 12, 13, 14
LESIZE,88,,,8
LESIZE,13,,,8
LESIZE,12, , ,28,5, , , ,1
LESIZE,14, , ,28,0.2, , , ,1
AMESH, 10


K, 118, KX(80)+wReactor*cos(180), KY(80)+wReactor*sin(180)
L, 118, 80
LPTN, 10,20,8
LDELE, 101
AL, 28, 21, 94 , 26
LESIZE,28,,,8
LESIZE,94,,,8
LESIZE,21,,,6
LESIZE,26,,,6
AMESH, 11

LDELE, 9
LSTR,    42,    11
LSTR,    42,    10

LESIZE,8, , ,2, , , , ,1
LESIZE,9, , ,6, , , , ,1
LESIZE,22, , ,20,0.2, , , ,1
LESIZE,25, , ,20,0.2, , , ,1
AL, 94, 22, 9, 8, 25
AMAP,12,11,10,80,21


LCOMB, 37, 31
LCOMB, 27, 36
AL, 9, 27, 35, 31
LESIZE,35,,,6
LESIZE,27,,,4
LESIZE,31,,,4
AMESH, 13

MAT,4                    ! Clad
LCOMB, 68, 39
```

```
LCOMB, 29, 87
AL, 8, 31, 86, 29
LESIZE,8,,,2
LESIZE,86,,,2
LESIZE,31,,,4
LESIZE,29,,,4
AMESH, 14

AL, 35, 43, 39, 76
LESIZE,35,,,6
LESIZE,39,,,6
LESIZE,43,,,4
LESIZE,76,,,4
AMESH, 15


AL, 86, 76, 66, 91
LESIZE,86,,,2
LESIZE,66,,,2
LESIZE,76,,,4
LESIZE,91,,,4
AMESH, 16

MAT,1                    ! Nozzle
LCOMB, 41, 77,
LCOMB, 41, 74,
LCOMB, 41, 47,
LDELE, 41
LDELE, 47

LESIZE,45, , ,19, , , , ,1
LESIZE,30, , ,1, , , , ,1
LESIZE,10,,,20
LESIZE,85,,,6
AL, 39, 10, 85, 45, 30
AMAP,17,101,98,36,99

MAT,4                    ! Clad
LESIZE, 79,,,2
LESIZE,84,,,20
AL, 66, 30, 45, 79, 84
AMAP,18,100,101,99,109

MAT,1                    ! Nozzle
LCOMB, 38, 81
LESIZE, 38,,,14
```

```
LESIZE, 83,,,6
LESIZE, 51,,,14
AL, 85, 38, 83, 51
AMESH, 19

MAT,4                    ! Clad
LESIZE, 80,,,2
LESIZE, 65,,,14
AL, 79, 51, 80, 65
AMESH, 20

MAT,1                    ! Nozzle
LCOMB, 82, 50
LESIZE, 50,,,20
LESIZE, 62,,,6
LESIZE, 60,,,20
AL, 83, 50, 62, 60
AMESH, 21

MAT,4                    ! Clad
LESIZE, 64,,,2
LESIZE, 63,,,20
AL, 80, 60, 64, 63
AMESH, 22

MAT,1                    ! Nozzle
LCOMB, 49, 71
LESIZE, 49,,, 20
LESIZE, 69,,,6
LESIZE, 61,,,20
AL, 62,49,69,61
AMESH, 23

MAT,4                    ! Clad
LESIZE, 40,,,2
LESIZE, 59,,,20
AL, 64, 61, 40, 59
AMESH, 24

MAT,1                    ! Nozzle
LESIZE, 75,,,6
LESIZE, 70,,,6
LESIZE, 34,,,6
AL, 69, 75, 70, 34
AMESH, 25
```

```
MAT,4                    ! Clad
LESIZE, 33,,,2
LESIZE, 32,,,6
AL, 40, 34, 33, 32
AMESH, 26

MAT,1                    ! Nozzle
LCOMB, 53, 72
LESIZE, 53,,,8
LESIZE, 58,,,6
LESIZE, 52,,,8
AL, 70, 53, 58, 52
AMESH, 27

MAT,4                    ! Clad
LESIZE, 57,,,2
LESIZE, 54,,,8
AL, 33, 52, 57, 54
AMESH, 28

MAT,3                    ! Vessel
LESIZE, 57,,,2
LESIZE, 54,,,8
LESIZE, 48,,,100,0.2, , , ,1
LESIZE, 55,,,100,0.2, , , ,1
LESIZE, 56,,,100,0.2, , , ,1
AL, 48, 44, 56, 58
AMESH, 29

MAT,4                    ! Clad
LESIZE, 42,,,2
AL, 57, 56, 42, 55
AMESH, 30


MAT,1                    ! Nozzle
ACLEAR,    17
ADELE,    17
LOVLAP,10, 46
NUMMRG,KP, , , ,LOW
LCOMB,37,41 ,0
AL, 20, 37, 85, 45, 30, 39
AMAP,17,101,98,36,99
```

```
MAT,2                    ! Safe End
LCOMB, 36, 78
LESIZE, 67,,,6
LESIZE, 36,,,6, 0.2,,,1
AL, 67, 73,36,20,43,27,22
AMAP,31,23,82,81,80


/COM,*********************
/COM, HTC point of Region 3
/COM,*********************
ACLEAR, 4
ADELE,  4
LDELE,  95
LDELE,  97
K, 120, KX(18), KY(18) - 3/8
K, 121, KX(25), KY(120)
L, 25, 121
L, 121, 113
L, 117, 120
L, 120, 33
L, 120, 121

MAT,2                    ! Safe End
AL, 17, 10, 68, 46
LESIZE,10,,,12
LESIZE,68,,,8
LESIZE,46,,,12
AMESH, 4

AL, 68, 41, 98, 47
LESIZE,41,,,4
LESIZE,98,,,8
LESIZE,47,,,4
AMESH, 32


/COM,*********************
/COM, HTC point of Region 7
/COM,*********************
MAT,4                    ! Clad
ACLEAR,     18
ADELE,      18
K, 122, KX(14)+wReactor, KY(14)
L, 14,122
LSBL, 84, 71
```

```
LESIZE, 72,,,10
LESIZE, 74,,,10
AL, 66, 30,45,79,72,74
AMAP,18,100,101,99,109


/COM,*************************************
/COM, HTC point of Region 3
/COM,*************************************
MAT,2                          ! Safe End
ACLEAR,  2
ADELE,   2
K, 123, KX(120), KY(120)-3
K, 124, KX(4), KY(4)+1+1/16
LDELE, 99
L, 4, 124
L, 124, 123
L, 123, 116
AL, 3, 5, 100, 78, 77, 71
AMAP,2,116,8,3,4



/COM,*********************************
/COM, Define DOF constraints on lines
/COM,*********************************
DL,42, ,SYMM
DL,44, ,SYMM

FLST,4,9,1,ORDE,2
FITEM,4,1
FITEM,4,-9
CP,1,UY,P51X
```

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PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

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10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Core Spray Nozzle Green's Functions

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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%) and no (0%) flow thermal loading of the Vermont Yankee Nuclear Power Station Core Spray Nozzle.

2.0 CORE SPRAY NOZZLE MODEL DESCRIPTION

An axisymmetric finite element model of the core spray nozzle was developed in Reference [1] using ANSYS [2]. The geometry used in Reference [1] was utilized in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which will be applied to the FE model for Green's Function development. Table 1 lists the material properties at 300°F. The meshed model is shown in Figure 1.

Table 1: Material Properties @ 300°F [1]

Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft^2/hr	Specific Heat, Btu/lb-°F [C] [3]	Poi R [N
Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157	0
Weld Overlay	INCONEL 82							
Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193	(
Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193	(
3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252	(
8Ø x 10Ø Conc. Reduction[2]	SA312 TP304							
Thermal Sleeve	SA312 TP304							

Notes:

1. The material properties applied in the analyses are taken from ASME Code, Section II, Part D 1998 Edition, with 2000 A information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a l for the original design code is acceptable, since later editions typically reflect more accurate material properties than was Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for density and Poiss values [3].
2. In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original de was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity and thern
3. Calculated as $[k/(\rho d)]/12^3$.

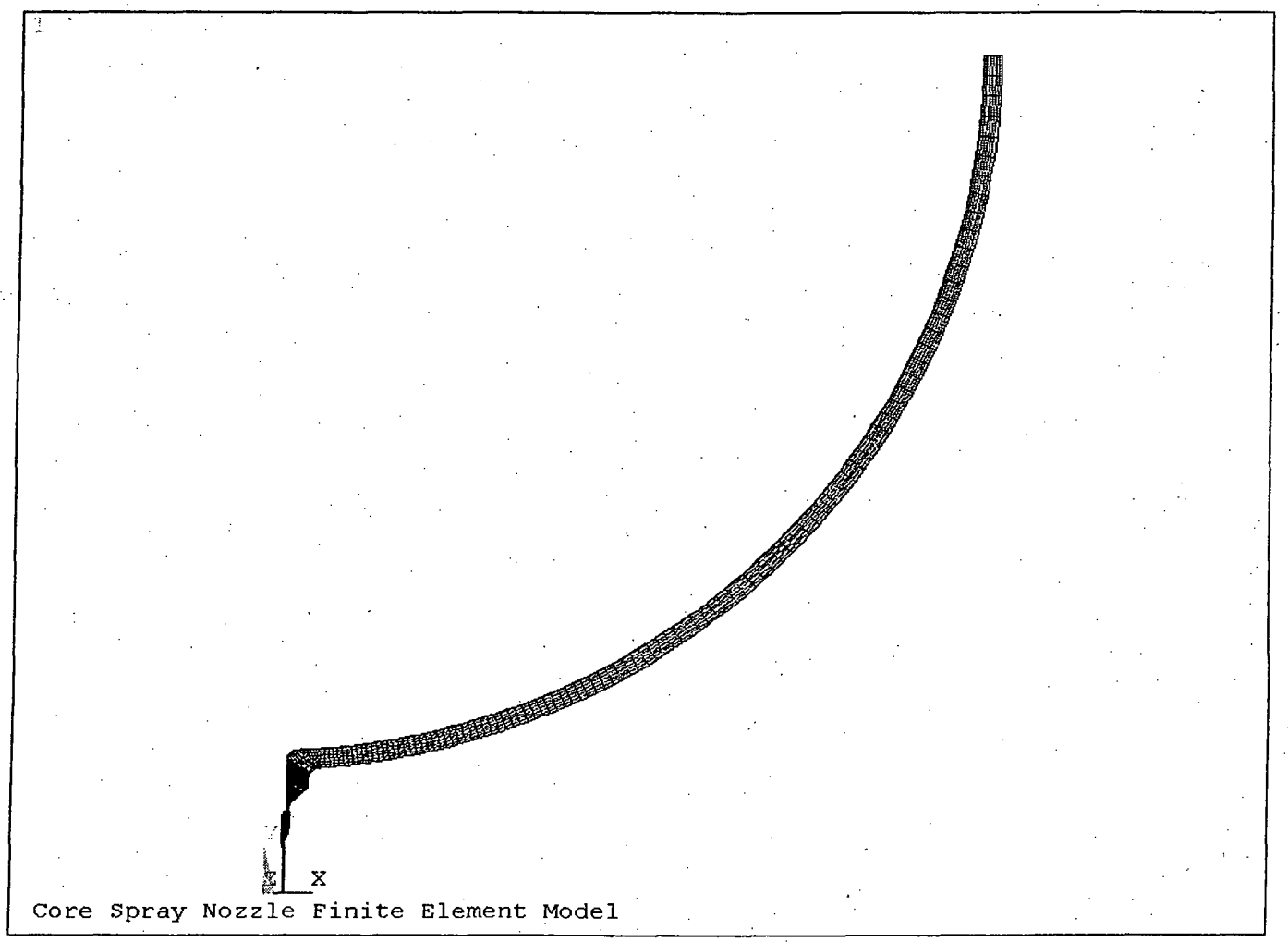

Figure 1: ANSYS Finite Element Model

3.0 APPLIED LOADS

Both pressure and thermal loads were applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the core spray nozzle and the reactor vessel wall (Figure 2). A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{CAP} = \frac{P \cdot D_i^2}{D_o^2 - D_i^2}$$

where:

P = Pressure = 1,000 psi
D_i = Inside Diameter = 9.834 in
D_o = Outside Diameter = 10.815 in

Therefore, the cap load is 4,774 psi. The calculated value was given a negative sign in order for it to exert tension on the end of the model. The nodes on the end of the safe end are coupled in the axial direction (UY, Figure 4) to ensure mutual displacement of the end of the nozzle due to attached piping. The boundary conditions at the end of the modeled portion of the reactor pressure vessel wall constructed to be "symmetric" (Figure 3).

The ANSYS input file VY_16Q_P.inp generates the core spray nozzle geometry from VY_CSN_Geom.inp [1] and performs the internal pressure load case just described. Figure 2, 3 and 4 show the internal pressure distribution, cap load, and symmetry conditions applied to the vessel end of the model, respectively.

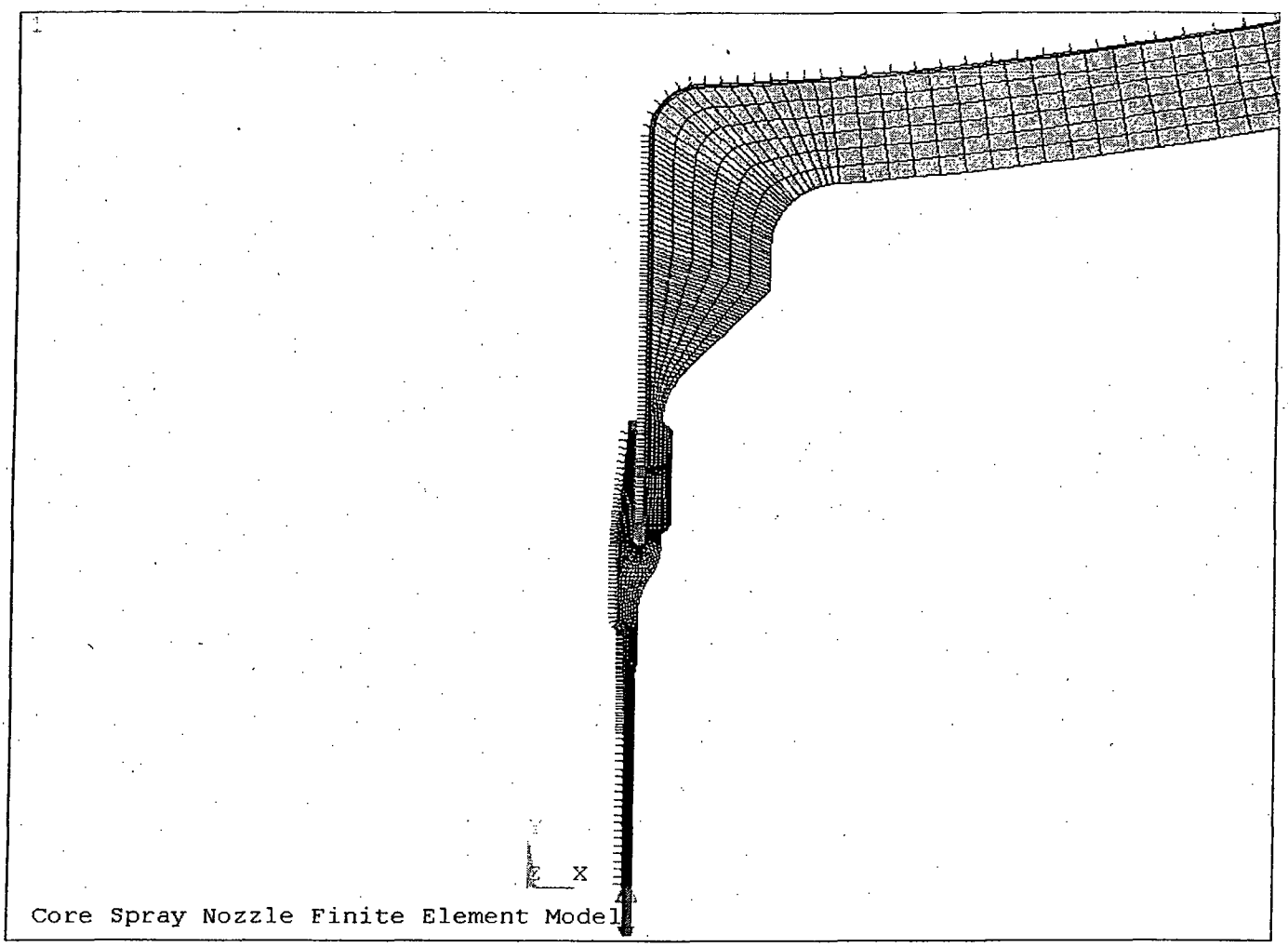

Figure 2: Core Spray Nozzle Internal Pressure Distribution

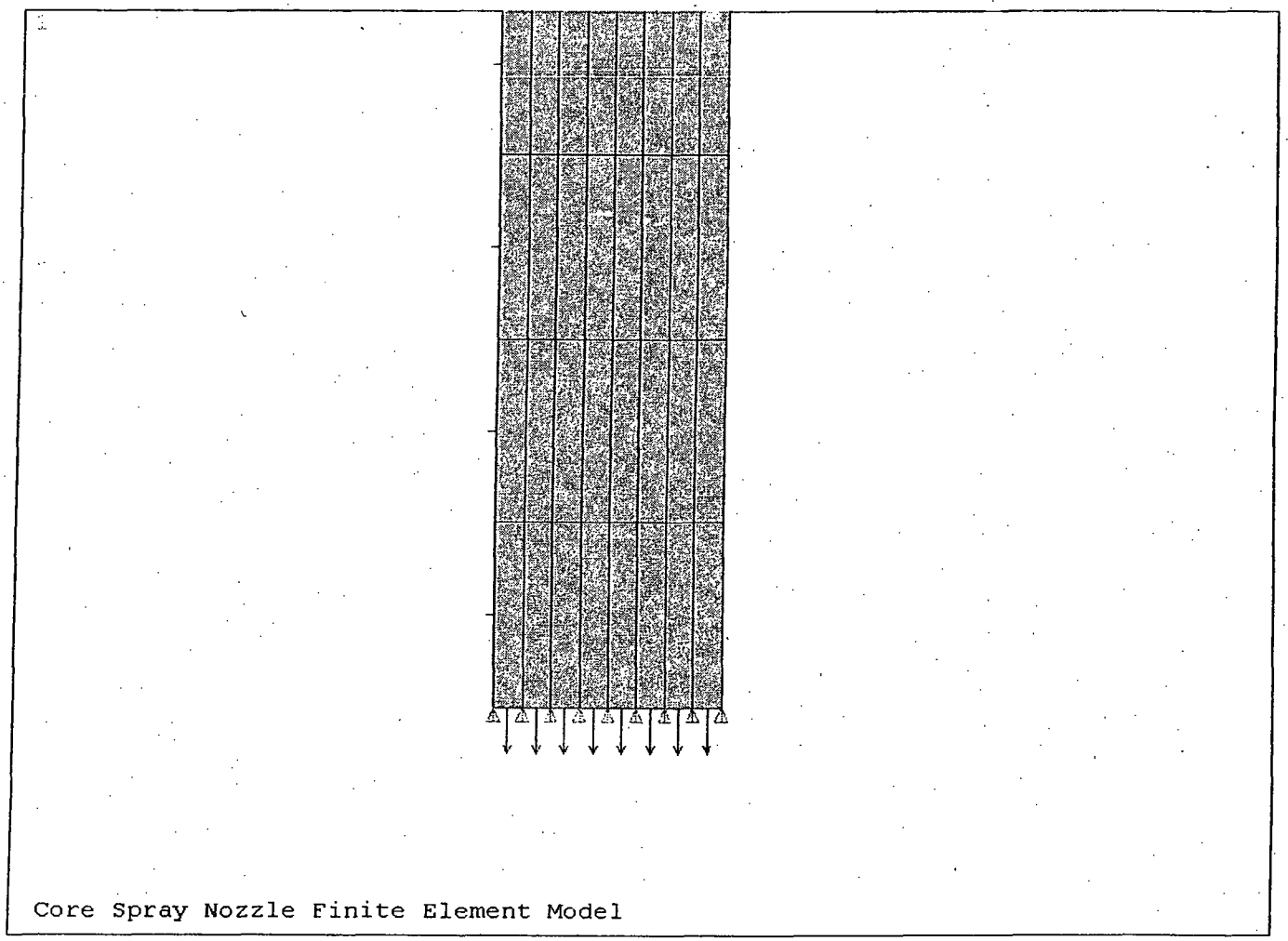

Figure 3: Core Spray Nozzle Pressure Cap Load

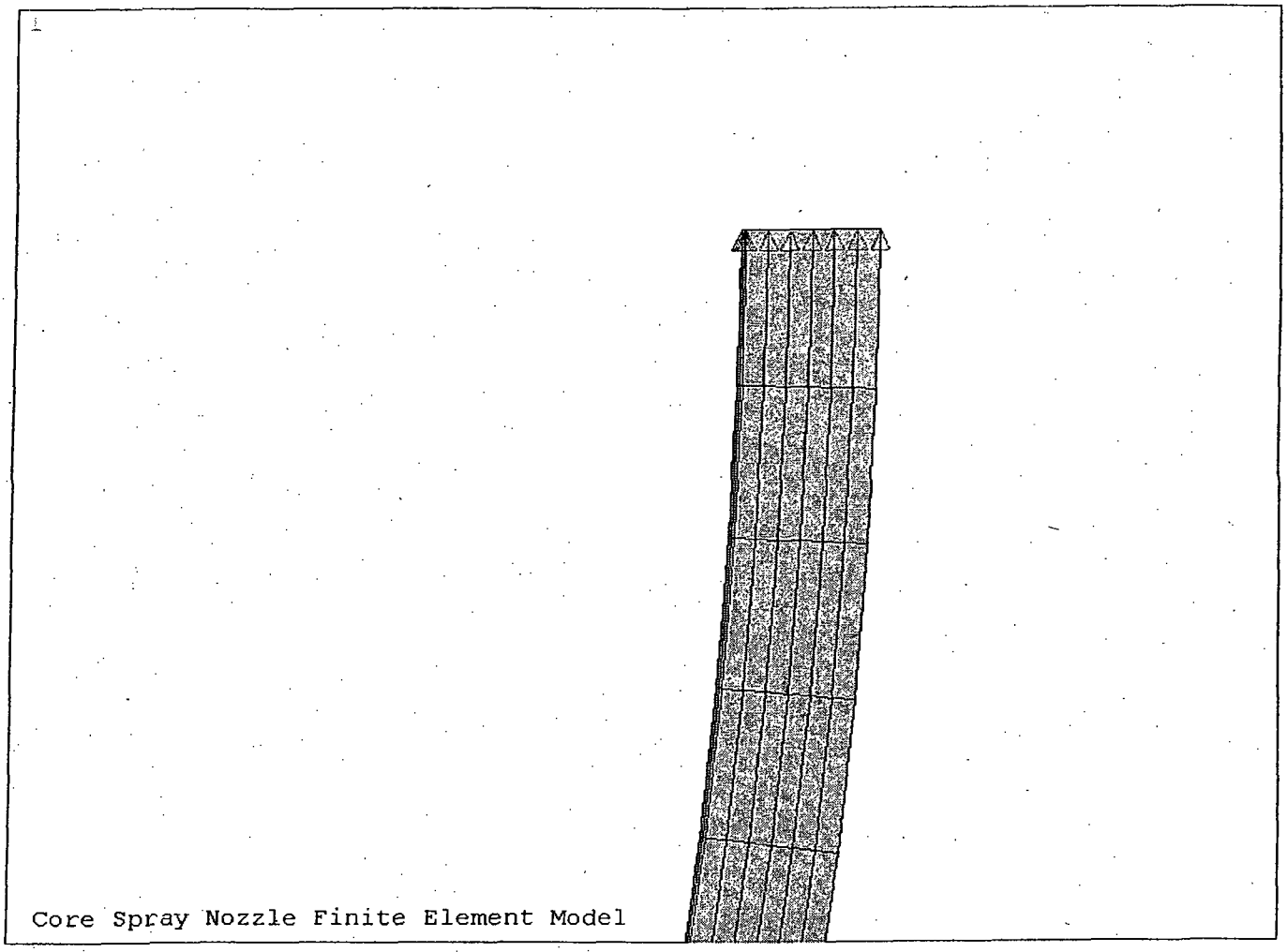

Figure 4: Core Spray Nozzle Vessel Wall Boundary Conditions

3.2 Thermal Load

Thermal loads are applied to the core spray nozzle model. The heat transfer coefficients (HTC) were determined using the methodology in the Excel spreadsheet "Heat Transfer Coefficients.xls", which is included in the project files. The HTCs were determined for various regions of the core spray FEM, (see Figure 5) for two different flow cases. The flow cases are for 100% (3200 gpm [6]) and 0% core spray flow through the nozzle.

The 0% flow case simulates a stagnant condition of the core spray nozzle when not in operation (i.e., the entire core spray nozzle is at the same temperature as the reactor pressure vessel due to reflooding). The HTCs for the no flow case are for free convection (stagnant) at the temperature of the reactor pressure vessel 500°F. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle in the stagnant condition.

The 100% flow case simulates operational condition of the core spray nozzle (i.e., the entire core spray nozzle experiences 100°F water due to injection). The HTCs for the high flow case are for forced and free convection depending on the region of the FEM. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle due to injection.

For both Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response. For the 0% flow case, the entire inside surface of the FEM was shocked. For the 100% flow case, only the nozzle flow path was shocked.

3.2.1 Boundary Fluid Temperatures

For the Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response to a degree change in temperature. The temperature on the exterior of the reactor, nozzle, safe end and the pipe is assumed to be 120 °F (ambient).

3.2.2 Heat Transfer Coefficients

Figure 5 shows where the heat transfer coefficients were applied to the FEM for the 0% (steady-state) and 100% core spray flow injection load case. For all the regions, the applied heat transfer coefficients and the initial temperatures are summarized in Table 2. The heat transfer coefficient for outside the reactor vessel wall is 0.2 BTU/hr-ft^2-°F and the heat transfer coefficient for inside the reactor vessel wall is 500 BTU/Hr-ft^2-°F, from page I-T7-5 of Reference [8].

Table 3 through Table 12 show the excel spreadsheets to calculate the HTC for regions 1, 3, 5, 7, and 9 respectively. These tables calculate the HTC for a certain part of the nozzle using the geometry of the bounding piping, the flow rate, and other physical fluid parameters. These tables calculate the Reynolds, Grashof and Rayleigh numbers in order to determine the HTC for inside surface/annulus forced and natural convection [4]. For several regions, the resultant HTCs had to be calculated from the partial heat transfer coefficients. These resultant HTCs are summarized in Table 13. In regions 2, 4, 6, 8, and 10 the HTCs are interpolated because of the complexity of the material profile.

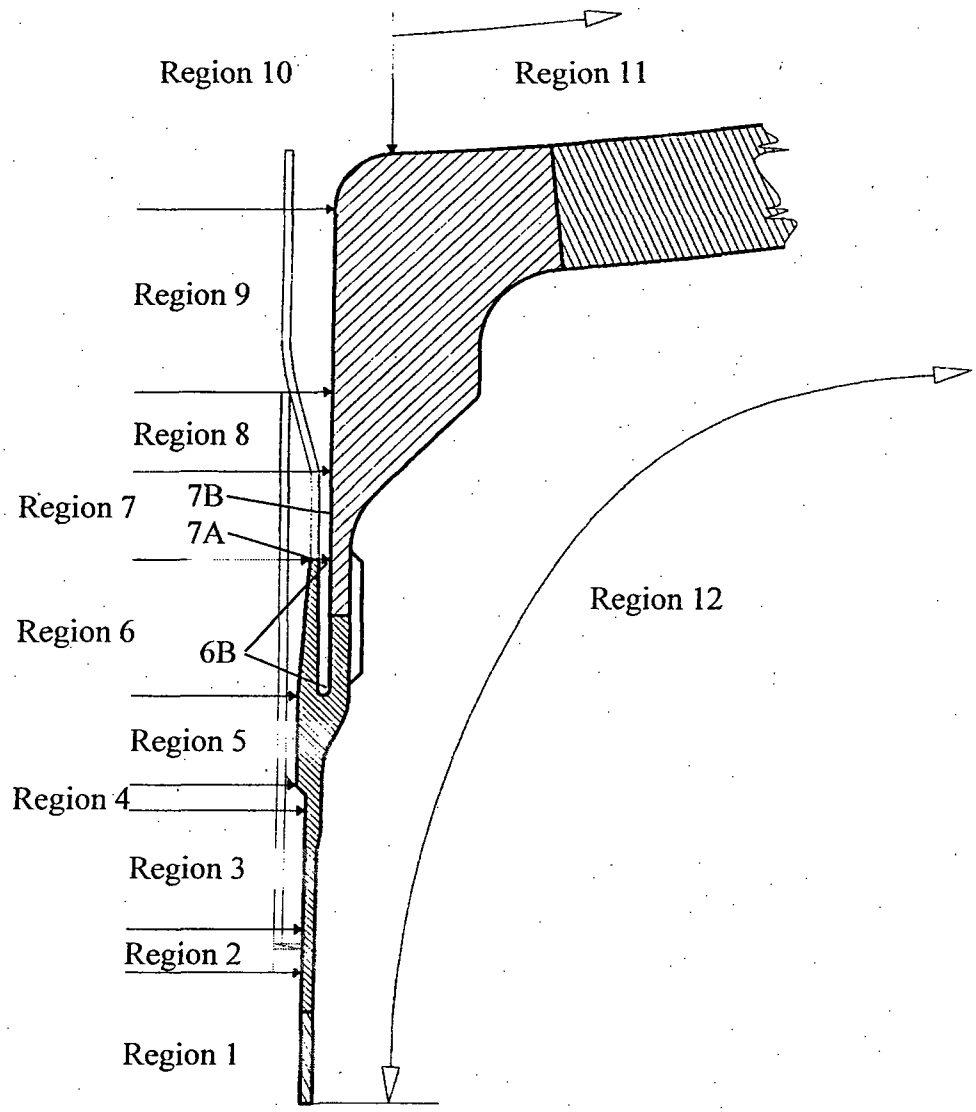

Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries (not to scale)

Table 2: Heat Transfer Coefficients

Regions	0% Flow		100% Flow	
	Initial Temperature °F	HTC Btu/hr-ft^2-°F	Initial Temperature °F	HTC Btu/hr-ft^2-°F
R1	500	143	500	2693
R2	500	Interpolated	500	Interpolated
R3 (1)	500	39	500	52
R4	500	Interpolated	500	Interpolated
R5 (1)	500	47	500	66
R6	500	Interpolated	500	Interpolated
R6B	500	97	500	97
R7A (1)	500	38	500	50
R7B (1)	500	20	500	23
R8	500	Interpolated	500	Interpolated
R9 (1)	500	33	500	41
R10	500	Interpolated	500	Interpolated
R11	500	500	500	500
R12	120	0.20	120	0.2

(1) See Table 13

Table 3: Heat Transfer Coefficients for Region 1

Pipe Inside Diameter, D =	9.834	inches =	0.820	ft	100% rated flow =	3,200	gpm
		=	0.250	m	@ T =	549	°F
Flow, % of rated =	100%				Density, ρ =	48.087	lbm/ft³
Fluid Velocity, V =	13.517	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr	
Characteristic Length, L = D =	0.820	ft =	0.250	m			

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

Note: The above assumption is based on experience with past RPV heat transfer analyses.

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	ρVD/μ	1.0307E+06	1.5019E+06	3.2317E+06	4.8825E+06	6.3843E+06	7.7540E+06	8.1118E+06	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	1.3522E+08	7.0314E+08	1.3383E+10	6.9351E+10	2.2021E+11	5.7264E+11	1.1964E+12	---
Rayleigh Number, Ra	GrPr	9.4382E+08	3.1712E+09	2.5562E+10	8.4608E+10	2.0920E+11	4.9189E+11	1.2802E+12	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	7,765.07	9,257.25	13,050.72	15,291.12	16,581.64	16,999.27	16,154.74	W/m²-°C
		1,367.53	1,630.33	2,298.41	2,692.98	2,920.25	2,993.80	2,845.07	Btu/hr-ft²-°F
		2.638E-03	3.145E-03	4.434E-03	5.195E-03	5.633E-03	5.775E-03	5.488E-03	Btu/sec-in²-°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 256 of [4])		
H_{free} =	$C(GrPr)^n k/L$	231.44	329.18	597.32	811.84	984.52	1,113.82	1,187.70	W/m²-°C
		40.76	57.97	105.20	142.98	173.39	196.16	209.17	Btu/hr-ft²-°F
		7.863E-05	1.118E-04	2.029E-04	2.758E-04	3.345E-04	3.784E-04	4.035E-04	Btu/sec-in²-°F

Table 4: First Partial Heat Transfer Coefficients for Region 3

Pipe Inside Diameter, D =	7.981	inches =	0.665	ft	100% rated flow =	3,200	gpm
		=	0.203	m	@ T =	549	°F
Flow, % of rated =	100%				Density, ρ =	48.087	lbm/ft³
Fluid Velocity, V =	20.522	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr	
Characteristic Length, L = D =	0.665	ft =	0.203	m			

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

Note: The above assumption is based on experience with past RPV heat transfer analyses

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	---
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m²-°C
		1,991.36	2,374.03	3,346.67	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft²-°F
		3.641E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in²-°F
From [4].									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [4])		
H_{free} =	$C(GrPr)^n k/L$	243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m²-°C
		42.94	61.08	110.83	150.64	182.68	206.67	220.38	Btu/hr-ft²-°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in²-°F

Table 5: Second Partial Heat Transfer Coefficients for Region 3

Pipe Inside Diameter, D =	9.834	inches =	0.820	ft	
		=	0.250	m	
Outer Pipe, Inside radius, r_o =	4.917	inches =	0.410	ft	
			0.125	m	
Inner Pipe Outside Diameter, D =	8.625	inches =	0.719	ft	
		=	0.219	m	
Inner Pipe, Outside radius, r_i =	4.3125	inches =	0.359	ft	
			0.110	m	
Fluid Velocity, V =	13.517	ft/sec =	3200.000	gpm	
Characteristic Length, L = D =	0.820	ft =	0.250	m	

(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m^3
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft^3
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m^3/m^3-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft^3/ft^3-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s^2
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s^2
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	1030724	1501950	3231741	4882481	6384268	7754027	8111787	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	135217684.2	703144247.6	13383382850	69350803914	2.2021E+11	5.72636E+11	1.19642E+12	---
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	3.14E+04	1.63E+05	3.11E+06	1.61E+07	5.11E+07	1.33E+08	2.78E+08	---
Rayleigh Number, Ra	GrPr	943819435.9	3171180557	25562261244	84607980776	2.092E+11	4.91894E+11	1.28017E+12	---
Rayleigh Number, Ra	$Gr_\delta Pr$	2.19E+05	7.37E+05	5.94E+06	1.97E+07	4.86E+07	1.14E+08	2.97E+08	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder		C =	0.40	n =	0.20	(see page 291 of [5])		
H_{free}	$C(Gr_\delta Pr)^n k/(r_o-r_i)$	182.78	244.68	400.00	512.07	593.51	643.36	653.99	W/m^2-°C
		32.19	43.09	70.45	90.18	104.53	113.30	115.18	$Btu/hr-ft^2$-°F

Table 6: First Partial Heat Transfer Coefficients for Region 5

Pipe Inside Diameter, D =	7.981	inches =	0.665	ft		100% rated flow =	3,200	gpm
		=	0.203	m		@ T =	549	°F
Flow, % of rated =	100%					Density, ρ =	48.087	lbm/ft³
Fluid Velocity, V =	20.522	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr		
Characteristic Length, L = D =	0.665	ft =	0.203	m				

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
Note. The above assumption is based on experience with past RPV heat transfer analyses. =	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	Conversion Factor [4]	70	100	200	300	400	500	600	°F
		21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	---
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m²-°C
		1.991E+06	2.374E+06	3.346E+06	3.921E+06	4.252E+06	4.359E+06	4.142E+06	Btu/hr-ft²-°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in²-°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [4])		
H_{free} =	$C(GrPr)^n k/L$	243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m²-°C
		42.94	61.08	110.83	150.64	182.68	206.67	220.38	Btu/hr-ft²-°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in²-°F

Table 7: Second Partial Heat Transfer Coefficients for Region 5

Title =	No Flow Region between nozzle and thermal sleeve				
Pipe Inside Diameter, D =	9.000	inches =	0.750	ft	
		=	0.229	m	
Outer Pipe, Inside radius, r_o =	4.5	inches =	0.375	ft	
			0.114	m	
Inner Pipe Outside Diameter, D =	8.625	inches =	0.719	ft	
		=	0.219	m	
Inner Pipe, Outside radius, r_i =	4.3125	inches =	0.359	ft	
			0.110	m	
Fluid Velocity, V =	16.138	ft/sec =	3200.000	gpm	
Characteristic Length, L = D =	0.750	ft =	0.229	m	

(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	°F
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	°C
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m^3
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft^3
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m^3/m^3-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft^3/ft^3-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s^2
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s^2
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	°F
Reynold's Number, Re	$\rho VD/\mu$	1126238	1641130	3531215	5334924	6975877	8472567	8863480	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	103650263.4	538990790.5	10258947757	53160421562	1.68801E+11	4.3895E+11	9.17108E+11	---
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	9.37E+02	4.87E+03	9.28E+04	4.81E+05	1.53E+06	3.97E+06	8.29E+06	---
Rayleigh Number, Ra	GrPr	723478838.9	2430848465	19594590215	64855714305	1.60361E+11	3.77058E+11	9.81306E+11	---
Rayleigh Number, Ra	$Gr_\delta Pr$	6.54E+03	2.20E+04	1.77E+05	5.86E+05	1.45E+06	3.41E+06	8.87E+06	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder	C =	0.40	n =	0.20	(see page 291 of [5])			
H_{free}	$C(Gr_\delta Pr)^n k/(r_o-r_i)$	291.94	390.80	638.87	817.88	947.95	1,027.56	1,044.55	W/m^2-°C
		51.41	68.83	112.51	144.04	166.95	180.97	183.96	$Btu/hr-ft^2$-°F

Table 8: First Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, D =	7.981	inches =	0.665	ft	100% rated flow =	3,200	gpm
		=	0.203	m	@ T =	549	°F
Flow, % of rated =	100%				Density, ρ =	48.087	lbm/ft³
Fluid Velocity, V =	20.522	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr	
Characteristic Length, L = D =	0.665	ft =	0.203	m			

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

Note. The above assumption is based on experience with past RPV heat transfer analyses.

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	---
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m²-°C
		1,991.36	2,374.03	3,346.87	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft²-°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in²-°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [4])		
H_{free} =	$C(GrPr)^n k/L$	243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m²-°C
		42.94	61.08	110.83	150.64	182.68	206.67	220.38	Btu/hr-ft²-°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in²-°F

Table 9: Second Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, D =	10.020	inches =	0.835	ft	
		=	0.255	m	
Outer Pipe, Inside radius, r_o =	5.01	inches =	0.418	ft	
			0.127	m	
Inner Pipe Outside Diameter, D =	8.625	inches =	0.719	ft	
		=	0.219	m	
Inner Pipe, Outside radius, r_i =	4.3125	inches =	0.359	ft	
			0.110	m	
Fluid Velocity, V =	13.020	ft/sec =	3200.000	gpm	
Characteristic Length, L = D =	0.835	ft =	0.255	m	
(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	**Conversion Factor [4]**	**70.**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
		21.11	**37.78**	**93.33**	**148.89**	**204.44**	**260.00**	**315.56**	**°C**
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m^3
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft^3
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m^3/m^3-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft^3/ft^3-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s^2
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s^2
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	1011591	1474069	3171750	4791848	6265758	7610090	7961209	—
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	143036227.2	743801381.9	14157235436	73360798959	2.32943E+11	6.05747E+11	1.2656E+12	—
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	4.82E+04	2.51E+05	4.78E+06	2.47E+07	7.86E+07	2.04E+08	4.27E+08	—
Rayleigh Number, Ra	GrPr	998392866.2	3354544233	27040319682	89500174730	2.21296E+11	5.20336E+11	1.35419E+12	—
Rayleigh Number, Ra	$Gr_\delta Pr$	3.37E+05	1.13E+06	9.12E+06	3.02E+07	7.46E+07	1.76E+08	4.57E+08	—

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder		C =	0.40	n =	0.20	(see page 291 of [5])		
H_{free}	$C(Gr_\delta Pr)^n k/(r_o-r_i)$	172.61	231.07	377.74	483.58	560.49	607.56	617.61	W/m^2-°C
		30.40	40.69	66.53	85.17	98.71	107.00	108.77	$Btu/hr-ft^2$-°F

Table 10: Third Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, D =	11.750	inches =	0.979	ft	
		=	0.298	m	
Outer Pipe, Inside radius, r_o =	5.875	inches =	0.490	ft	
			0.149	m	
Inner Pipe Outside Diameter, D =	10.750	inches =	0.896	ft	
		=	0.273	m	
Inner Pipe, Outside radius, r_i =	5.375	inches =	0.448	ft	
			0.137	m	
Fluid Velocity, V =	9.468	ft/sec =	3200.000	gpm	
Characteristic Length, L = D =	0.979	ft =	0.298	m	

(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	Conversion Factor [4]	70 21.11	100 37.78	200 93.33	300 148.89	400 204.44	500 260.00	600 315.56	°F °C
k (Thermal Conductivity)	1.7307	0.5997 0.3465	0.6300 0.3640	0.6784 0.3920	0.6836 0.3950	0.6611 0.3820	0.6040 0.3490	0.5071 0.2930	W/m-°C Btu/hr-ft-°F
c_p (Specific Heat)	4.1869	4.185 1.000	4.179 0.998	4.229 1.010	4.313 1.030	4.522 1.080	4.982 1.190	6.322 1.510	kJ/kg-°C Btu/lbm-°F
ρ (Density)	16.018	997.1 62.3	994.7 62.1	962.7 60.1	917.8 57.3	858.6 53.6	784.9 49.0	679.2 42.4	kg/m^3 lbm/ft^3
β (Volumetric Rate of Expansion)	1.8	1.89E-04 1.05E-04	3.24E-04 1.80E-04	6.66E-04 3.70E-04	1.01E-03 5.60E-04	1.40E-03 7.80E-04	1.98E-03 1.10E-03	3.15E-03 1.75E-03	m^3/m^3-°C ft^3/ft^3-°F
g (Gravitational Constant)	0.3048	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	m/s^2 ft/s^2
μ (Dynamic Viscosity)	1.4881	9.96E-04 6.69E-04	6.82E-04 4.58E-04	3.07E-04 2.06E-04	1.93E-04 1.30E-04	1.38E-04 9.30E-05	1.04E-04 7.00E-05	8.62E-05 5.79E-05	kg/m-s lbm/ft-s
Pr (Prandtl Number)		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	862650	1257036	2704761	4086325	5343225	6489626	6789048	—
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	230651605.4	1199409312	22829105215	1.18297E+11	3.75631E+11	9.76791E+11	2.04083E+12	—
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	1.78E+04	9.24E+04	1.76E+06	9.12E+06	2.89E+07	7.53E+07	1.57E+08	—
Rayleigh Number, Ra	GrPr	1609948206	5409335995	43603590961	1.44323E+11	3.56849E+11	8.39063E+11	2.18369E+12	—
Rayleigh Number, Ra	$Gr_\delta Pr$	1.24E+05	4.17E+05	3.36E+06	1.11E+07	2.75E+07	6.47E+07	1.68E+08	—

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder		C =	0.40	n =	0.20	(see page 291 of [5])		
H_{free}	$C(Gr_\delta Pr)^n k/(r_o-r_i)$	197.20	263.98	431.55	552.46	640.32	694.10	705.57	W/m^2-°C
		34.73	46.49	76.00	97.30	112.77	122.24	124.26	Btu/hr-ft²-°F

Table 11: First Partial Heat Transfer Coefficients for Region 9

Pipe Inside Diameter, D =	7.981	inches =	0.665	ft					
		=	0.203	m					
Flow, % of rated =	100%								
Fluid Velocity, V =	20.522	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr			
Characteristic Length, L = D =	0.665	ft =	0.203	m					

100% rated flow =	3,200 gpm
@ T =	549 °F
Density, ρ =	48.087 lbm/ft³

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

Note: The above assumption is based on experience with past RPV heat transfer analyses.

		Value at Fluid Temperature, T [7]							Units
	Conversion	70	100	200	300	400	500	600	°F
Water Property	Factor [4]	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	---
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m²-°C
		1,991.36	2,374.03	3,346.87	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft²-°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in²-°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [4])		
H_{free} =	$C(GrPr)^n k/L$	243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m²-°C
		42.94	61.08	110.83	150.64	182.68	206.67	220.38	Btu/hr-ft²-°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in²-°F

Table 12: Second Partial Heat Transfer Coefficients for Region 9

Pipe Inside Diameter, D =	11.750	inches =	0.979	ft	
		=	0.298	m	
Outer Pipe, Inside radius, r_o =	5.875	inches =	0.490	ft	
			0.149	m	
Inner Pipe Outside Diameter, D =	8.625	inches =	0.719	ft	
		=	0.219	m	
Inner Pipe, Outside radius, r_i =	4.3125	inches =	0.359	ft	
			0.110	m	
Fluid Velocity, V =	9.468	ft/sec =	3200.000	gpm	
Characteristic Length, L = D =	0.979	ft =	0.298	m	

(Outside) T_{fluid} - $T_{surface}$, ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

		Value at Fluid Temperature, T [7]							Units
Water Property	Conversion Factor [4]	70 21.11	100 37.78	200 93.33	300 148.89	400 204.44	500 260.00	600 315.56	°F °C
k (Thermal Conductivity)	1.7307	0.5997 0.3465	0.6300 0.3640	0.6784 0.3920	0.6836 0.3950	0.6611 0.3820	0.6040 0.3490	0.5071 0.2930	W/m-°C Btu/hr-ft-°F
c_p (Specific Heat)	4.1869	4.185 1.000	4.179 0.998	4.229 1.010	4.313 1.030	4.522 1.080	4.982 1.190	6.322 1.510	kJ/kg-°C Btu/lbm-°F
ρ (Density)	16.018	997.1 62.3	994.7 62.1	962.7 60.1	917.8 57.3	858.6 53.6	784.9 49.0	679.2 42.4	kg/m^3 lbm/ft^3
β (Volumetric Rate of Expansion)	1.8	1.89E-04 1.05E-04	3.24E-04 1.80E-04	6.66E-04 3.70E-04	1.01E-03 5.60E-04	1.40E-03 7.80E-04	1.98E-03 1.10E-03	3.15E-03 1.75E-03	m^3/m^3-°C ft^3/ft^3-°F
g (Gravitational Constant)	0.3048	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	9.806 32.17	m/s^2 ft/s^2
μ (Dynamic Viscosity)	1.4881	9.96E-04 6.69E-04	6.82E-04 4.58E-04	3.07E-04 2.06E-04	1.93E-04 1.30E-04	1.38E-04 9.30E-05	1.04E-04 7.00E-05	8.62E-05 5.79E-05	kg/m-s lbm/ft-s
Pr (Prandtl Number)		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	$\rho VD/\mu$	862650	1257036	2704761	4086325	5343225	6489626	6789048	---
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	230651605.4	1199409312	22829105215	1.18297E+11	3.75631E+11	9.76791E+11	2.04083E+12	---
Grashof Number, Gr_δ	$g\beta\Delta T(r_o-r_i)^3/(\mu/\rho)^3$	5.42E+05	2.82E+06	5.37E+07	2.78E+08	8.83E+08	2.30E+09	4.80E+09	---
Rayleigh Number, Ra	GrPr	1609948206	5409335995	43603590961	1.44323E+11	3.56849E+11	8.39063E+11	2.18369E+12	---
Rayleigh Number, Ra	$Gr_\delta Pr$	3.79E+06	1.27E+07	1.03E+08	3.39E+08	8.39E+08	1.97E+09	5.13E+09	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder		C =	0.40	n =	0.20	(see page 291 of 15)		
H_{free}	$C(Gr_\delta Pr)^n k/(r_o-r_i)$	125.02	167.35	273.58	350.24	405.94	440.03	447.30	W/m^2-°C
		22.02	29.47	48.18	61.68	71.49	77.50	78.78	$Btu/hr-ft^2$-°F

Although the thermal sleeve was excluded from the analysis, its effect had to be included in the finite element model. For several thermal regions, the resultant HTCs had to be calculated from the partial heat transfer coefficients (HTC_i in Table 13). These are generated by "Heat Transfer Coefficients.xls".

$$\frac{1}{HTC_{Res}} = \left(\frac{1}{HTC_I} + \frac{T_I}{TC_I}\right) + \left(\frac{1}{HTC_{II}} + \frac{T_{II}}{TC_{II}}\right) + \left(\frac{1}{HTC_{III}} + \frac{T_{III}}{TC_{III}}\right)$$

Where:

HTC_{Res}	=	Resultant HTC
HTC_i	=	HTC of ith material
T_i	=	Thickness of ith material
TC_i	=	Thermal Conductivity of ith material

The reference for this equation is [4].

Table 13: Resultant Heat Transfer Coefficients for the Regions

100% Flow

Regions	HTC I	Material		HTCII	Material		HTCIII	HTC resultant
		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		
R1	2,692.98							2693
R2								
R3	3,921.42	9.8	0.0268	90.18				71
R4								
R5	3,921.42	9.8	0.0268	144.04				101
R6								
R6B							97.30	97
R7A	3,921.42	9.8	0.0268	85.17				68
R7B	3,921.42	9.8	0.0268	85.17	9.8	0.0304	97.30	36
R8								
R9	3,921.42	9.8	0.0268	61.68				52
R10								
R11	500							500
R12	0.2							0.2

0% Flow

Regions	HTC I	Material		HTCII	Material		HTCIII	HTC resultant
		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		
R1	142.98							143
R2								
R3	150.64	9.8	0.0268	90.18				49
R4								
R5	150.64	9.8	0.0268	144.04				61
R6								
R6B							97.30	97
R7A	150.64	9.8	0.0268	85.17				47
R7B	150.64	9.8	0.0268	85.17	9.8	0.0304	97.30	29
R8								
R9	150.64	9.8	0.0268	61.68				39
R10								
R11	500							500
R12	0.2							0.2

4.0 THERMAL AND PRESSURE LOAD RESULTS

The two flow dependent thermal load cases outlined in previous section were run on the core spray FEM. For ANSYS, the thermal transient input files "VY_16Q_T100.inp," "VY_16Q_T0.inp," for 100% and 0% flow, respectively. The stress input filenames are "VY_16Q_ST100.inp"and "VY_16Q_ST0.inp," respectively.

The limiting safe end location was chosen based on the highest thermal stress intensity at 100% flow. Node 3719 on the inside surface of the core spray nozzle was selected for the safe end analysis and shown in Figure 6.

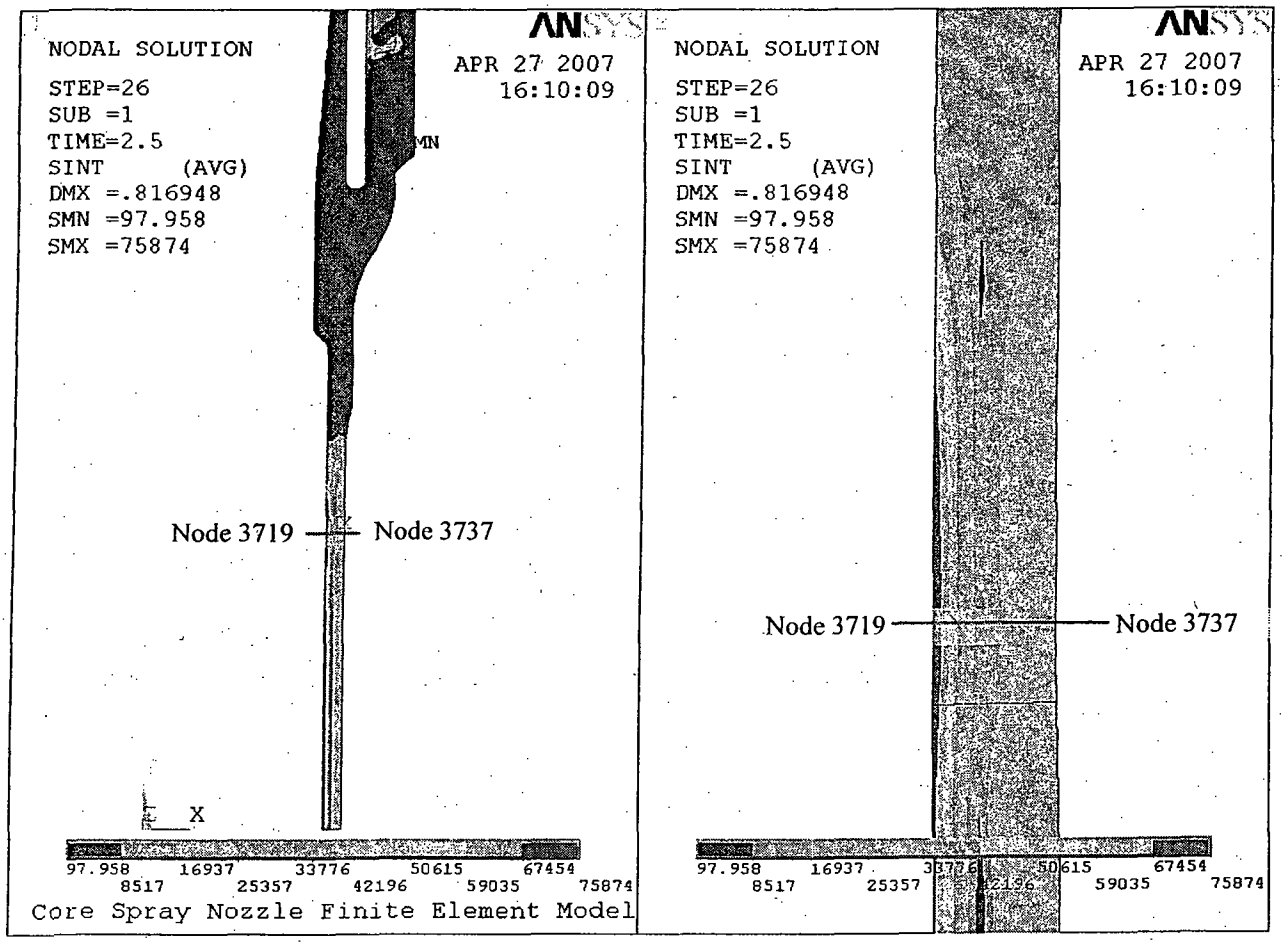

Figure 6: Safe End Critical Thermal Stress Location, Node 3719

The limiting blend radius location was chosen based upon the highest pressure stress intensity. Node 2166 on the inside surface of the blend radius was therefore selected for the nozzle forging analysis and shown in Figure 7. The highest thermal stress and pressure stress occur very close to the same location in the nozzle forging region. Therefore, this location is a reasonable choice for the limiting location.

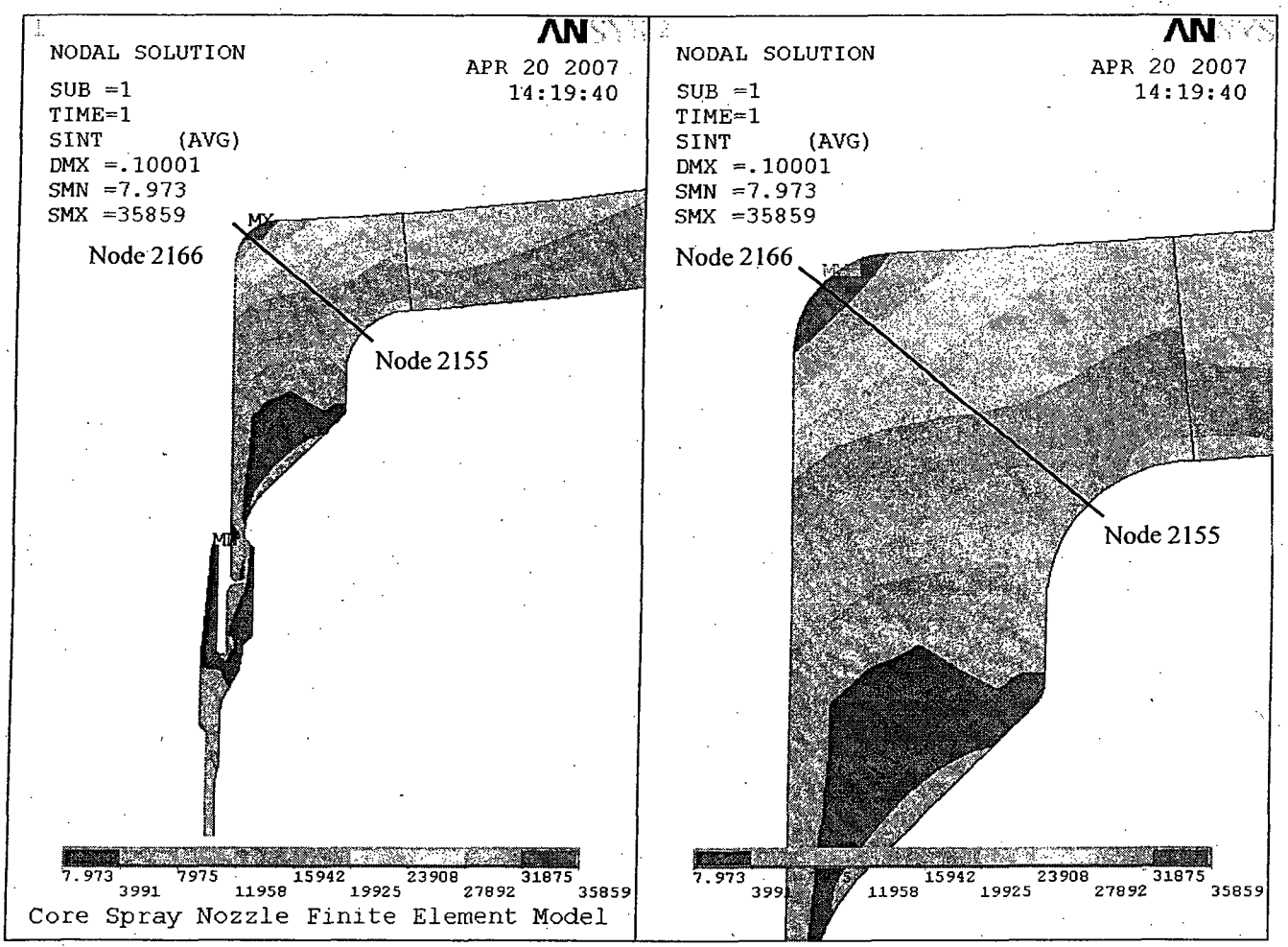

Figure 7: Blend Radius Limiting Pressure Stress Location, Node 2166

The stress intensity time history for the critical safe end and blend radius paths were extracted using the ANSYS post-processing file "extract100.inp" for 100% flow. This produced the two files, "SE_F100.out" and "BR_F100.out," which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the four files "SE_F100.cln, BR_F100.cln" and "SE_F100_INSIDE.RED, BR_F100_ INSIDE.RED."

The stress intensity time history for the critical safe end and blend radius paths were extracted using the ANSYS post-processing file "extract0.inp" for 0% flow. This produced the two files, "SE_F0.out" and "BR_F0.out," which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the four files "SE_F0.cln, BR_F0.cln" and "SE_F0_INSIDE.RED, BR_F0_ INSIDE.RED."

As the models were run with a 400°F step change in temperature, and the Green's Functions are for a 1°F step change in temperature, all data values were divided by 400. The governing Green's Functions for the core spray nozzle during 100% flow and 0% flow are shown in Figure 8 through Figure 11. The data for the Green's Functions is included in the files:

0% Flow Rate:

SE_Flow0_T_Green.xls
SE_Flow0_M+B-Green.xls
BLEND_Flow0_M+B_Green.xls
BLEND_Flow0_T_Green.xls

100 Flow Rate:

SE_Flow100_T_Green.xls
SE_Flow100_M+B-Green.xls
BLEND_Flow100_M+B_Green.xls
BLEND_Flow100_T_Green.xls

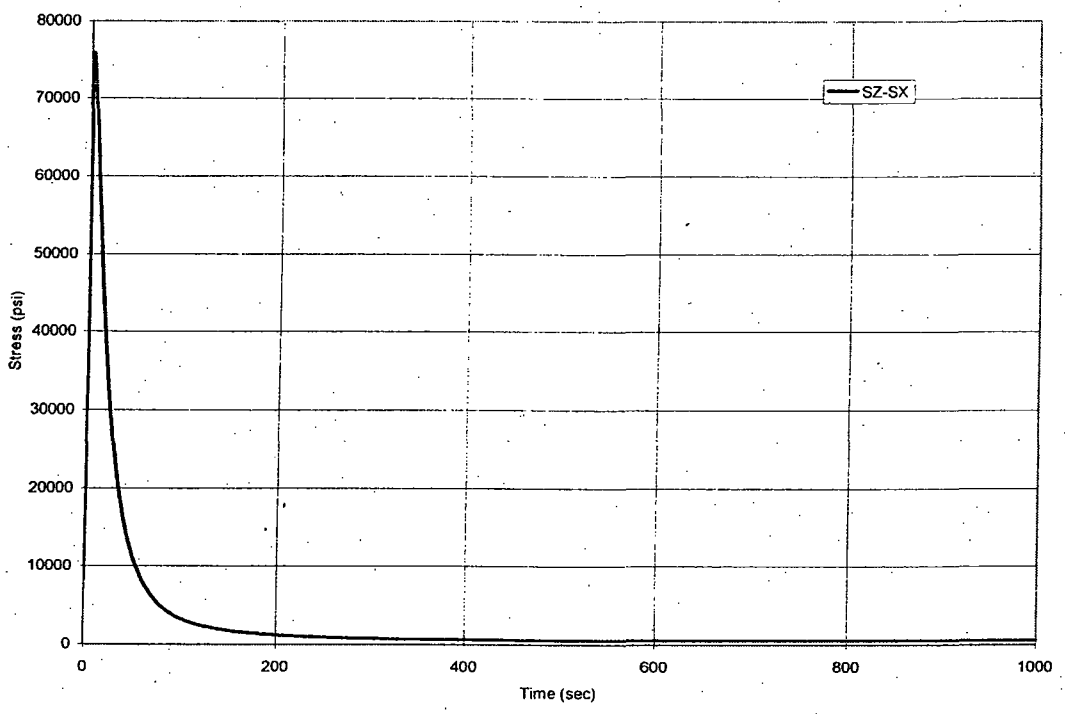

Figure 8: Safe End Total Stress History, 100% Flow

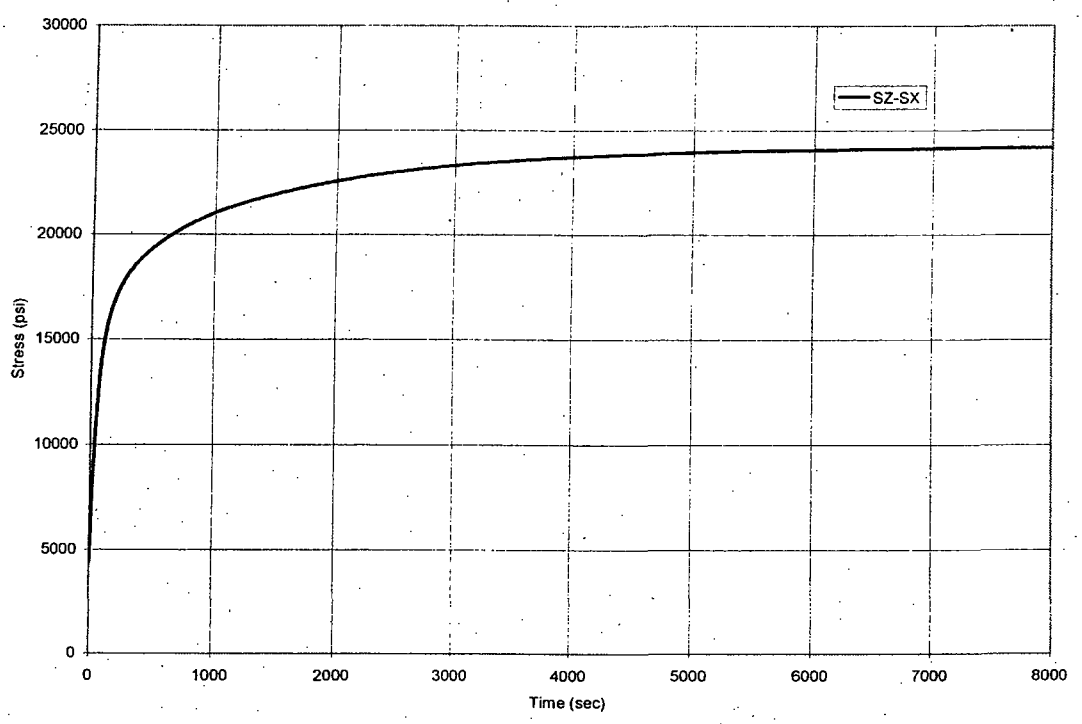

Figure 9: Blend Radius Total Stress History, 100% Flow

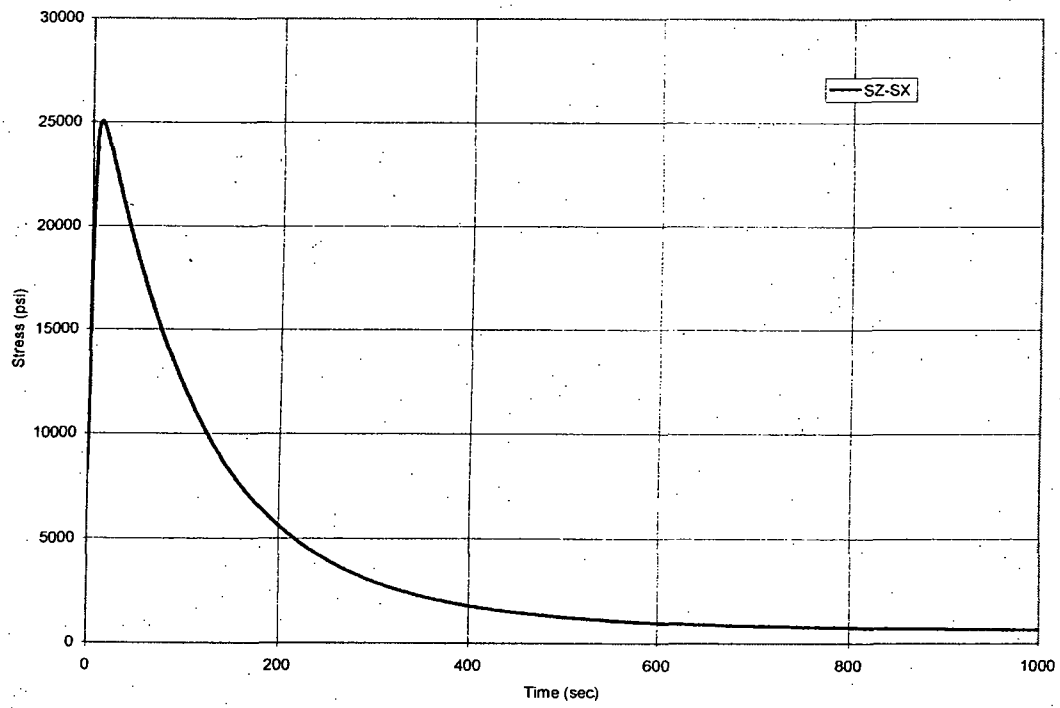

Figure 10: Safe End Total Stress History, 0% Flow

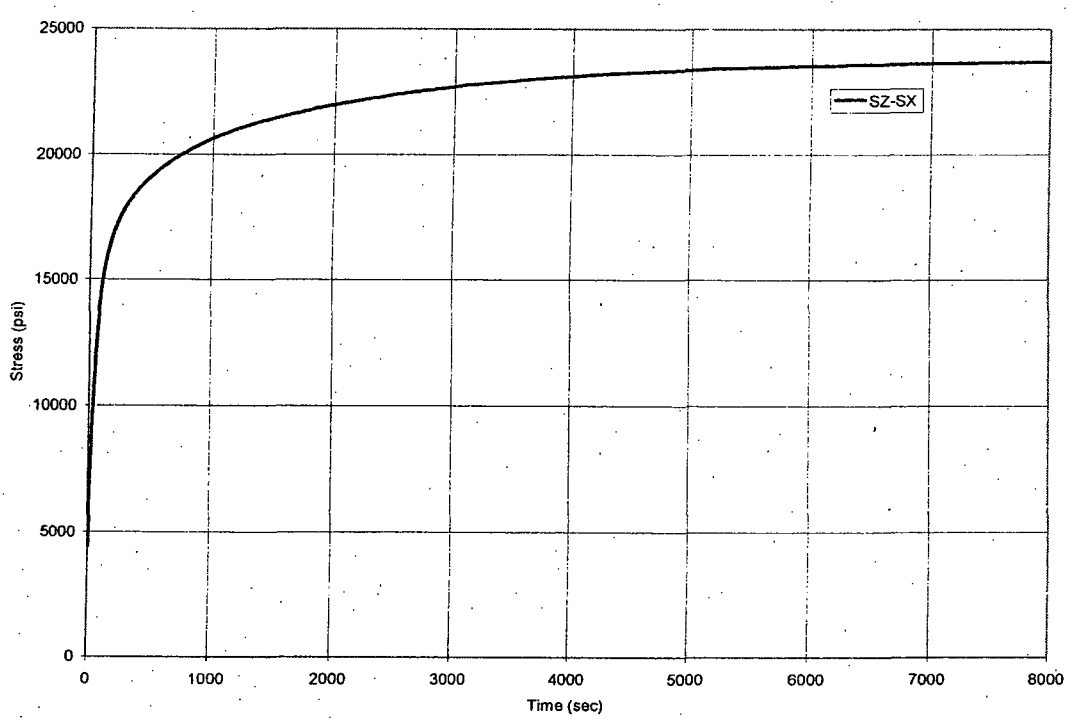

Figure 11: Blend Radius Total Stress History, 0% Flow

The pressure stress intensities for the safe end and blend radius paths were extracted using the ANSYS post-processing file "extractP.inp." This produced two files, SE_P.OUT for the safe end and BR_P.OUT for the blend radius.

Results of the internal pressure load case for Node 2166 (blend radius) is a total stress intensity of 35,860 psi and for Node 3719 (safe end), a total stress intensity of 12,030 psi. The membrane plus bending stress intensity at Node 2166 and Node 3719 are 34970 psi and 12,020 psi, respectively. Table 14 shows the final pressure results for the safe end and blend radius.

Table 14: Pressure Results (1,000 psi)

Location	Membrane plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	12,020	12,030
Blend Radius	34,970	35,860

Results were also extracted from the vessel portion of the model to verify the accuracy of the results obtained from the ANSYS model, and to check the results due to the use of the 2.0 multiplier on the vessel radius. These results are contained in the file VESSEL_P.OUT. The radius of the finite element model (FEM) was multiplied by a factor of 2.0 [1] to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere, but the true geometry is the intersection of two cylinders.

The equation for the membrane hoop stress in a sphere is:

$$\sigma = \left(\frac{(pressure) \times (radius)}{2 \times thickness} \right)$$

Considering a vessel base metal radius, R, of 105.906 inches increased by a factor of 2.0, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is PR/(2t) = 19,477 psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, VESSEL_P.OUT, of 18,530 psi. Thus, considering the peak total pressure stress of 35,860 psi reported above, the stress concentrating effect of the nozzle corner is 35,860/19,477 = 1.84. In other words, the peak nozzle corner stress is 1.84 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \left(\frac{(pressure) \times (radius)}{thickness} \right).$$

Based on the previous dimensions, the calculated stress for a cylinder without the 2.0 factor is 19,477 psi. Increasing this by a factor of 1.84 yields an expected peak nozzle corner stress of 35,838 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (35,860 psi) is close to the peak nozzle corner stress for a cylindrical geometry because of the use of the 2.0 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.

5.0 REFERENCES

1. SI Calculation No. VY-16Q-308, Revision 0, "Core Spray Nozzle Finite Element Model".

2. ANSYS, Release 8.1A1 (w/Service Pack 1), ANSYS, Inc., June 2004.

3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.

4. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.

5. J. P. Holman, "Heat Transfer," 5th Edition, 1981.

6. Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

7. N. P. Cheremisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co., 1984.

8. CB&I RPV Stress Report, Section T7, "Thermal Analysis of Core Spray Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201, SI File No. VY-16Q-206.

APPENDIX A

FINITE ELEMENT ANALYSIS FILES

ANSYS Input Files

File Name	Description
vy_csn_geom.inp	ANSYS input file includes the geometry and material properties
Heat Transfer Coefficients.xls	Excel file to calculate Heat Transfer coefficients
VY_16Q_P.inp	ANSYS input file for the pressure stress analysis
VY_16Q_T100.inp	ANSYS input file for the thermal analysis, 100% flow rate
VY_16Q_T0.inp	ANSYS input file for the thermal analysis, 0% flow rate
VY_16Q_ST100.inp	ANSYS input file for the thermal stress analysis, 100% flow rate
VY_16Q_ST0.inp	ANSYS input file for the thermal stress analysis, 0% flow rate
extract100.inp	ANSYS input file to extract the limiting paths, 100% flow rate
extract0.inp	ANSYS input file to extract the limiting paths, 100% flow rate
extractP.inp	ANSYS input file to extract the limiting paths
extractVessel.inp	ANSYS input file to extract the membrane stress in the vessel wall

ANSYS Output Files

File Name	Description
BR_F100.out	ANSYS output file, Results of running: extract100.inp, Blend Radius 100% Flow
SE_F100.out	ANSYS output file, Results of running: extract100.inp, Safe End 100% Flow
BR_F100.cln	Reduced ANSYS output file, contains the stress values in time, Blend Radius 100% Flow
SE_F100.cln	Reduced ANSYS output file, contains the stress values in time, Safe End 100% Flow
BR_F100_ INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Blend Radius 100% Flow
SE_F100_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Safe End 100% Flow
BR_F0.out	ANSYS output file, Results of running: extract0.inp, Blend Radius 0% Flow
SE_F0.out	ANSYS output file, Results of running: extract0.inp, Safe End 0% Flow
BR_F100.cln	Reduced ANSYS output file, contains the stress values in time, Blend Radius 0% Flow
SE_F100.cln	Reduced ANSYS output file, contains the stress values in time, Safe End 0% Flow
BR_F0_ INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Blend Radius 0% Flow
SE_F0_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Safe End 0% Flow

Structural Integrity Associates, Inc.	File No.: VY-16Q-310 EXHIBIT J
CALCULATION PACKAGE	**Project No.:** VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Core Spray Nozzle

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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the core spray nozzle. Two locations will be analyzed for fatigue acceptance: the blend radius (SA508 Class II) and the safe end (SB 166 N06600). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. A revised fatigue usage will be determined for both locations, the nozzle forging and safe end, respectively. In the end, the environmental fatigue usage factors will be determined for the limiting locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the core spray nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the core spray nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL), Report [12] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [8]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [14] methodology was originally used to evaluate the core spray nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original core spray nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more refined CUF.

For the core spray nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in

structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [11]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for all thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 8 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 9. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 9. These five step changes produce the five successive stress responses in the lower plot shown in Figure 9. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY core spray nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end

and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for these two locations and 60-year projected cycle counts.

Three Structural Integrity utility programs will be used to perform the fatigue analysis. The first two calculate stresses in response to transients. The transients analyzed are those described in the thermal cycle diagrams [2] for the core spray nozzle. These transients are shown in Figure 1– Figure 6. The temperatures and pressures for these transients have been modified to account for power uprate [3]. The power uprate pressures and temperatures were used for this analysis. The last program calculates fatigue based on the stress output. The three programs are STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [4] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient definition for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.

The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE, and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m, Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.

3.0 ANALYSIS

The transients analyzed for the core spray nozzle were developed based on the definitions in the original RPV Design Specification [10], as modified for EPU [3], as well as more recent definitions based on BWR operating experience [2] for BWR. The final transients evaluated in the stress and fatigue analyses are shown in Figure 1 thru Figure 6.

The fatigue analysis involves the preparing of input files for, and running of three programs [4]. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P_V.EXE.

The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 8 stress history functions obtained from References [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for flow condition of 0% and 100%.
- Green.cfg is configured as described in Reference [4].
- Transnt.inp. These files are created to represent the selected transients obtained from the thermal cycle diagrams [2] and redefined by power uprate [3]. Table 1 and Table 2 contain the loading defined for each transient. Based upon the thermal cycle diagram for the RPV and the core spray nozzle, the transients are split into the following groups based upon flow rate:
    - Transients 02, 03, 11, 14, 21-23 and 24 are run at 0% flow.
    - Transient 30 runs at 100% flow rate per [3]. The transient of emergency shutdown is numbered as 30.

The remaining transients are not included in this analysis, as temperature changes from them are considered negligible to have impact on the results.

3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

The program P-V.exe is then run to extract the peaks and valleys from the STRESS.OUT file produced by the STRESS.EXE program. The only input required for this program is STRESS.OUT and it outputs all the peaks and valleys to P-V.OUT. Columns 2 through 5 of Table 5 (for the blend radius) and Table 6 (for the safe end) show the final peak and valley output. The pressure for column six is then filled in using the thermal cycle diagrams. Pressure and piping loads have to be added to the peak and valley points to calculate the final stress values used for fatigue analysis.

3.3 Pressure Load

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [1]. These values are as follows:

Location	Membrane plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	12,020	12,030
Blend Radius	34,970	35,860

These pressure stress values for each location were linearly scaled according to the pressure of the transient. The actual pressure for column 6 of Table 5 and Table 6 is obtained from Reference [2] and shown in Tables 1 and 2. The scaled pressure stress values are shown in columns 7 and 8 of Table 5 (for the blend radius) and Table 6 (for the safe end).

The pressure stress is combined with the peak and valley points to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 7.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first cut (blend radius) and second cut (safe end) locations using the following equations:

For Cut I:
$$(M_x)_1 = M_x - F_y L_1$$
$$(M_y)_1 = M_y + F_x L_1$$

For Cut II:
$$(M_x)_2 = M_x - F_y L_2$$
$$(M_y)_2 = M_y + F_x L_2$$

The total bending moment and shear loads are obtained using the equations below:

For Cut I:

$$M_{xy} = \sqrt{(M_x)_1^{\,2} + (M_y)_1^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_1^{\,2} + (F_y)_1^{\,2}}$$

For Cut II:

$$M_{xy} = \sqrt{(M_x)_2^{\,2} + (M_y)_2^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_2^{\,2} + (F_y)_2^{\,2}}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$N_z = \frac{1}{\pi R_N}\left[\frac{1}{2}F_z + \frac{M_{xy}}{R_N}\right]$$
$$q_N = \frac{1}{\pi R_N}\left[F_{xy} - \frac{M_z}{2R_N}\right]$$

To determine the primary stresses, PM, due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$
$$(P_M)_\theta = \frac{Pa_N}{t_N}$$
$$(P_M)_R = -P$$
$$\tau_M = \frac{q_N}{t_N}$$
$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\,2}}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\,2}}$$

Where:

L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.

L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
M_{xy} = The maximum bending moment in the xy plane.
F_{yx} = The maximum shear force in the xy plane.
N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
q_N = The shear force per inch of circumference applied to the nozzle.
R_N = The mid-wall nozzle radius.

Because pressure was not considered in this analysis, the equations used for Cut I are valid for Cut II. In addition, the equations can be simplified as follows:

$$(P_M)_z = \frac{Nz}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{Nz}{2t_N}\right)^2 + (\tau_M)_{z\theta}{}^2}$$

Per Reference [5], the core spray nozzle piping loads are as follows:

F_x =	2,500	lbs	M_x =	22,000	ft-lb =	264,000	in-lb
F_y =	4,600	lbs	M_y =	7,100	ft-lb =	85,200	in-lb
F_z =	1,700	lbs	M_z =	8,800	ft-lb =	150,600	in-lb

The location of the nozzle piping loads is assumed to be at the end of the connection of the safe end and the attached pipe. Therefore, the L1 is equal to 30.817 inches and the L2 is equal to 0.303 inches. The calculations for the blend radius and safe end are shown in Table 3 and Table 4. The first cut location is the middle of Green's Function cross section for the blend radius (Node 2181) per [1], and the second cut is from Node 3719 (inside) to Node 3737 (outside). The maximum stress intensities, due to piping loads are 322.52 psi at the blend radius and 6949.94 psi at the safe end, respectively. The piping load sign is set as the same as the thermal stress sign.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F [2]. The scaled piping stress values are shown in columns 9 and 10 of Table 5 and Table 6. Columns 11 and 12 of Table 5 and Table 6 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient, as obtained from Reference [2]. Column 13 in Table 5 and Table 6 shows the number of cycles associated with each transient.

The program FATIGUE.EXE performs the "ASME Code style" peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius (SA508 Class II)	Safe End (N06600)	Piping (Stainless Steel)
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel) [8]	1.7 & 0.3 [8]	1.7 & 0.3 [8]
Design Stress Intensity Values, S_m	26,700 psi [6] @ 600°F	23,300 psi [6] @ 600°F	17,000 psi [6] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [8]	28.3×10^6 psi [8]	28.3×10^6 psi [8]
Elastic Modulus Used in Finite Element Model (300°F)	26.7×10^6 psi [1]	29.8×10^6 psi [1]	27.0×10^6 psi [1]
The Geometric Stress Concentration Factor K_t	1.0	4.0 See Note	1.8 [14]

Note: Conservative bounding value per ASME Code, Section NB-3600 to cover thread and weld regions.

The results of the fatigue analyses are presented in Table 7 through Table 9 for the blend radius, safe end and stainless steel piping for 60 years, respectively.

The Core Spray piping adjacent to the safe end was also analyzed because of its proximity to the maximum safe end thermal stress location. For this fatigue analysis, the stress results of the safe end were used with stainless steel material properties and a value of 1.8 was selected for K_t at the weld location, based on the maximum value given in ASME Code, Section III, table NB-3681(a)-1 [8].

The results described are contained in EXCEL files BRresults.xls and SEresults.xls, which are contained in the computer files.

4.0 FATIGUE USAGE RESULTS

The blend radius Cumulative Usage Factor (CUF) from system cycling is 0.0043 for 60 years. The safe end CUF is 0.0184 and the CUF of stainless steel piping is 0.0005 for 60 years.

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Per Reference [7], the dissolved Oxygen (DO) calculation shows the overall HWC availability is 47%. It means the pre-HWC is 53%.

The fatigue calculation will be re-performed for the nozzle base material, since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [8]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [9]. Therefore, the cladding will be neglected and EAF assessment of the nozzle base material is performed.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82 from Table 4 of Reference [7]. It results in an EAF adjusted CUF of (11.14 x 53% + 8.82 x 47%) x 0.0043 = 0.0432 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 10.05.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are all 1.49 from Reference [13]. It results in an EAF adjusted CUF of 1.49 x 0.0184 = 0.0274 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 1.49.

For the stainless steel piping, the environmental fatigue factors for post-HWC and pre-HWC are all 8.36 from Table 4 of Reference [7]. It results in an EAF adjusted CUF of 8.36 x 0.0005 = 0.00418 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 8.36.

A Fatigue Environmental Multiplier of 1.49 for Ni-Cr-Fe was applied to the safe end fatigue usage and 8.36 for stainless steel to the piping. This results in the safe end being the limiting location for fatigue.

6.0 REFERENCES

1 SI Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions."

2 "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR), Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.

3 GE Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," August 29, 2003, SI File No. VY-05Q-236.

4 Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."

5 VY Drawing 5920-0024, Revision 11, Sht. No. 7, "Reactor Vessel," (GE Drawing No. 919D294), SI File No. VY-05Q-241.

6 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.

7 SI Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell Bottom Head."

8 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III Subsection NB, 1998 Edition, 2000 Addenda.

9 NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.

10 GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.

11 Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.

12 NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.

13 EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.

14 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, Article 4, 1965 Edition with Winter 1966 Addenda.

Table 1: Blend Radius Transients [1,2,3]

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test	—	100	—	0	
				1100	
120 Cycles				50	
3. Startup	0	100		0	0
300 Cycles	16164	549	16164	1010	(0%)
	24164	549	8000	1010	
11. Loss of Feedwater	0	526		1010	0
Pumps	3	526	3	1190	(0%)
10 Cycles	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
	16698	542	60	1010	
	16699	526	1	1010	
	24699	526	8000	1010	
14. SRV Blowdown	0	526		1010	0
1 Cycle	600	375	600	400	(0%)
	11580	70	10980	50	
	19580	70	8000	50	
21-23. Shutdown	0	549		1010	0
300 Cycles	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
	24224	100	8000	50	
24. Hydrostatic Test	—	100	—	50	
1 Cycle				1563	
				50	
30. Emergency Shut Down	0	549		1010	3200
1 Cycle	10	406	10	250	(100%)
	11	70	1	250	
	8011	70	8000	0	

Note:

1. Instant temperature change is 1 sec.
2. This is due to the length of the Green's Function. The transients are plotted using an 8000 second steady state increment.
3. The number of cycles for 60 years is from Reference [2].

Table 2: Safe End Transients [1,2,3]

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test	---	100	---	0	
120 Cycles				1100	
				50	
3. Startup	0	100		0	0
300 Cycles	16164	549	16164	1010	(0%)
	17164	549	1000	1010	
11. Loss of Feedwater	0	526		1010	0
Pumps	3	526	3	1190	(0%)
10 Cycles	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
	16698	542	60	1010	
	16699	526	1	1010	
	17699	526	1000	1010	
14. SRV Blowdown	0	526		1010	0
1 Cycle	600	375	600	400	(0%)
	11580	70	10980	50	
	12580	70	1000	50	
21-23. Shutdown	0	549		1010	0
300 Cycles	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
	17224	100	1000	50	
12. Hydrostatic Test	---	100	---	50	
1 cycle				1563	
				50	
30. Emergency Shut Down	0	549		1010	3200
1 Cycle	10	406	10	250	(100%)
	11	70	1	250	
	1011	70	1000	0	

Note:

1. Instant temperature change is 1 sec.
2. The transients are plotted using a 1000 second steady state increment. The difference is due to the length of the Green's Function for the safe end.
3. The number of cycles for 60 years is from Reference [2].

Table 3: Maximum Piping Stress Intensity Calculations for Blend Radius

Blend Radius External Piping Loads		
Parameters		
F_x =	2.50	kips
F_y =	4.60	kips
F_Z =	1.70	kips
M_x =	264.00	in-kips
M_y =	85.20	in-kips
M_Z =	105.60	in-kips
OD=	18.87	in
ID=	11.750	in
R_N =	7.65	in
L =	30.82	in
t_N =	3.56	in
$(M_x)_2$ =	122.24	in-kips
$(M_y)_2$ =	162.24	in-kips
M_{xy} =	203.14	in-kips
F_{xy} =	5.24	kips
N_Z =	1.14	kips/in
q_N =	-0.07	kips/in
Primary Membrane Stress Intensity		
PM_Z =	0.32	ksi
τ =	-0.02	ksi
SI_{max} =	0.32	ksi
SI_{max} =	322.52	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 4: Maximum Piping Stress Intensity Calculations for Safe End

Safe End External Piping Loads		
Parameters		
F_x =	2.50	kips
F_y =	4.60	kips
F_Z =	1.70	kips
M_x =	264.00	in-kips
M_y =	85.20	in-kips
M_Z =	105.60	in-kips
OD=	10.82	in
ID=	9.834	in
R_N =	5.16	in
L =	0.30	in
t_N =	0.49	in
$(M_x)_1$ =	262.60	in-kips
$(M_y)_1$ =	85.96	in-kips
M_{xy} =	276.31	in-kips
F_{xy} =	5.24	kips
N_Z =	3.35	kips/in
q_N =	-0.31	kips/in
Primary Membrane Stress Intensity		
PM_Z =	6.84	ksi
τ =	-0.63	ksi
SI_{max} =	6.95	ksi
SI_{max} =	6949.94	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 5: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	19	19	19	19	120
				100	1100	39446	38467	19	19	39465	38486	120
				100	50	1793	1749	19	19	1812	1768	120
3	0	23700	12600	100	0	0	0	19	19	23719	12619	300
	24164	2100	3180	549	1010	36219	35320	306	306	38625	38806	300
11	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
	3	3209	3644	526	1190	42673	41614	291	291	46174	45550	10
	526	10458	5374	330	1135	40701	39691	166	166	51325	45231	10
	2222	5488	1664	490	1122	40235	39236	268	268	45991	41168	10
	2860	11776	7444	321	911	32668	31858	160	160	44605	39462	10
	6903	5435	3621	495	1124	40307	39306	272	272	46013	43199	10
	8012	12577	6791	390	627	22484	21926	204	204	35265	28921	10
	16640	2772	4370	542	1010	36219	35320	301	301	39292	39991	10
	16991	3389	4115	526	1010	36219	35320	291	291	39899	39726	10
	24699	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
14	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	1
	19580	25122	13197	70	50	1793	1749	0	0	26915	14946	1
21-23	0	2103	3161	549	1010	36219	35320	306	306	38628	38787	300
	24224	23680	12568	100	50	1793	1749	19	19	25492	14336	300
24				100	50	1793	1749	19	19	1812	1768	1
				100	1563	56049	54658	19	19	56068	54677	1
				100	50	1793	1749	19	19	1812	1768	1
30	0	2040	2950	549	1010	36219	35320	306	306	38565	38576	1
	8011	25700	14900	70	0	0	0	0	0	25700	14900	1

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 1.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 35,860)/1000 [1].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 34,970)/1000 [1].
Column 9: Total external stress from calculation in Table 3, 322.52 x (Column 5 -70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 6: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	413	413	413	413	120
				100	1100	13233	13222	413	413	13646	13635	120
				100	50	602	601	413	413	1014	1014	120
3	0	661	759	100	0	0	0	413	413	1074	1172	300
	17164	9240	10700	549	1010	12150	12140	6592	6592	27982	29432	300
11	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
	3	8802	10236	526	1190	14316	14304	6276	6276	29393	30815	10
	13	8802	10236	526	1135	13654	13643	6276	6276	28732	30154	10
	164	11645	11598	408	1135	13654	13643	4657	4657	29956	29898	10
	672	4808	5791	344	1135	13654	13643	3775	3775	22237	23209	10
	2374	11140	10841	361	912	10971	10962	4005	4005	26116	25808	10
	2955	4722	5577	325	916	11019	11010	3509	3509	19250	20096	10
	7054	9518	10162	441	959	11537	11527	5100	5100	26155	26789	10
	7930	4491	5276	309	637	7663	7657	3287	3287	15441	16219	10
	16709	9960	11116	526	1010	12150	12140	6276	6276	28386	29532	10
	17699	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
14	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	1
	152	9499	10570	497	855	10286	10277	5880	5880	25664	26727	1
	12580	91	95	70	50	602	601	0	0	693	696	1
21-23	0	9242	0	549	1010	12150	12140	6592	6592	27984	18732	300
	17224	664	0	100	50	602	601	413	413	1678	1014	300
24				100	50	602	601	413	413	1014	1014	1
				100	1563	18803	18787	413	413	19216	19200	1
				100	50	602	601	413	413	1014	1014	1
30	0	9280	10800	549	1010	12150	12140	6592	6592	28022	29532	1
	13	85600	44694	162	250	3002	3000	1260	1260	89862	48953	1
	1011	-12	-10	70	0	0	0	0	0	-12	-10	1

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 2.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 12,030)/1000.
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 12,020)/1000.
Column 9: Total external stress from calculation in Table 4, 6949.94 x (Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 7: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 2.0
n = .2
Sm = 26700. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.670E+07 psi
Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
56068.	19.	56049.	54658.	1.000	31488.	1.000E+00	1.896E+04	.0001
51325.	19.	51306.	45212.	1.000	28824.	1.000E+01	2.501E+04	.0004
46174.	19.	46155.	45531.	1.000	25930.	1.000E+01	3.460E+04	.0003
46013.	19.	45994.	43180.	1.000	25839.	1.000E+01	3.498E+04	.0003
45991.	19.	45972.	41149.	1.000	25827.	1.000E+01	3.503E+04	.0003
44605.	19.	44586.	39443.	1.000	25048.	1.000E+01	3.848E+04	.0003
39899.	19.	39880.	39707.	1.000	22404.	1.000E+01	5.695E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+00	5.824E+04	.0000
39465.	19.	39446.	38467.	1.000	22161.	3.800E+01	6.012E+04	.0006
39465.	1812.	37653.	36718.	1.000	21153.	8.200E+01	7.572E+04	.0011
39292.	1812.	37480.	38223.	1.000	21056.	1.000E+01	7.747E+04	.0001
38628.	1812.	36816.	37019.	1.000	20683.	2.800E+01	8.466E+04	.0003
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	23719.	14909.	26168.	1.000	8376.	2.700E+02	5.366E+07	.0000
38625.	23719.	14906.	26187.	1.000	8374.	3.000E+01	5.375E+07	.0000
38625.	25492.	13133.	24470.	1.000	7378.	2.700E+02	3.042E+08	.0000
38565.	25492.	13073.	24240.	1.000	7344.	1.000E+00	3.374E+08	.0000
35265.	25492.	9773.	14585.	1.000	5490.	1.000E+01	1.000E+20	.0000
26915.	25492.	1423.	610.	1.000	799.	1.000E+00	1.000E+20	.0000
25700.	25492.	208.	564.	1.000	117.	1.000E+00	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0043

Table 8: Fatigue Results for Safe End (60 Years)

LOCATION = LOCATION NO. 1 -- SAFE END
FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 1.7
n = .3
Sm = 23300. psi
Ecurve = 2.830E+07 psi
Eanalysis = 2.980E+07 psi
Kt = 4.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	112423.	1.000E+00	1.213E+03	.0008
29956.	413.	29543.	29485.	1.000	56029.	1.000E+01	1.910E+04	.0005
29393.	413.	28980.	30402.	1.000	57068.	1.000E+01	1.746E+04	.0006
28732.	413.	28319.	29741.	1.000	55813.	1.000E+01	1.946E+04	.0005
28386.	413.	27973.	29119.	1.000	54762.	1.000E+01	2.140E+04	.0005
28022.	413.	27609.	29119.	1.000	54590.	1.000E+00	2.174E+04	.0000
27984.	413.	27571.	18319.	1.000	39187.	7.900E+01	1.244E+05	.0006
27984.	693.	27291.	18036.	1.000	38651.	1.000E+00	1.341E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.200E+02	1.460E+05	.0008
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1074.	26910.	17560.	1.000	37792.	9.800E+01	1.514E+05	.0006
27982.	1074.	26908.	28260.	1.000	53033.	2.020E+02	2.517E+04	.0080
27982.	1678.	26304.	28418.	1.000	52971.	9.800E+01	2.532E+04	.0039
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+00	2.919E+04	.0000
26155.	1678.	24477.	25775.	1.000	48339.	1.000E+01	4.021E+04	.0002
26116.	1678.	24438.	24794.	1.000	46923.	1.000E+01	4.673E+04	.0002
25664.	1678.	23986.	25713.	1.000	48017.	1.000E+00	4.159E+04	.0000
22237.	1678.	20559.	22195.	1.000	41379.	1.000E+01	9.257E+04	.0001
19250.	1678.	17572.	19082.	1.000	35526.	1.000E+01	2.135E+05	.0000
19216.	1678.	17538.	18186.	1.000	34234.	1.000E+00	2.691E+05	.0000
15441.	1678.	13763.	15205.	1.000	28195.	1.000E+01	1.001E+06	.0000
13646.	1678.	11968.	12621.	1.000	23661.	1.200E+02	1.772E+06	.0001
							TOTAL USAGE FACTOR =	.0184

Table 9: Fatigue Results for Stainless Steel Piping (60 Years)

LOCATION = LOCATION NO. 1 -- SS Piping
FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 1.7
n = .3
Sm = 17000. psi
Ecurve = 2.830E+07 psi
Eanalysis = 2.700E+07 psi
Kt = 1.80

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	67629.	1.000E+00	8.006E+03	.0001
29956.	413.	29543.	29485.	1.000	27845.	1.000E+01	1.042E+06	.0000
29393.	413.	28980.	30402.	1.000	27934.	1.000E+01	1.031E+06	.0000
28732.	413.	28319.	29741.	1.000	27310.	1.000E+01	1.110E+06	.0000
28386.	413.	27973.	29119.	1.000	26868.	1.000E+01	1.171E+06	.0000
28022.	413.	27609.	29119.	1.000	26678.	1.000E+00	1.198E+06	.0000
27984.	413.	27571.	18319.	1.000	22130.	7.900E+01	2.272E+06	.0000
27984.	693.	27291.	18036.	1.000	21864.	1.000E+00	2.392E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.200E+02	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1074.	26910.	17560.	1.000	21465.	9.800E+01	2.588E+06	.0000
27982.	1074.	26908.	28260.	1.000	25950.	2.020E+02	1.311E+06	.0002
27982.	1678.	26304.	28418.	1.000	25700.	9.800E+01	1.354E+06	.0001
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+00	1.485E+06	.0000
26155.	1678.	24477.	25775.	1.000	23634.	1.000E+01	1.779E+06	.0000
26116.	1678.	24438.	24794.	1.000	23202.	1.000E+01	1.889E+06	.0000
25664.	1678.	23986.	25713.	1.000	23351.	1.000E+00	1.850E+06	.0000
22237.	1678.	20559.	22195.	1.000	20080.	1.000E+01	3.442E+06	.0000
19250.	1678.	17572.	19082.	1.000	17209.	1.000E+01	7.481E+06	.0000
19216.	1678.	17538.	18186.	1.000	16816.	1.000E+00	8.600E+06	.0000
15441.	1678.	13763.	15205.	1.000	13588.	1.000E+01	1.000E+20	.0000
13646.	1678.	11968.	12621.	1.000	11564.	1.200E+02	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0005

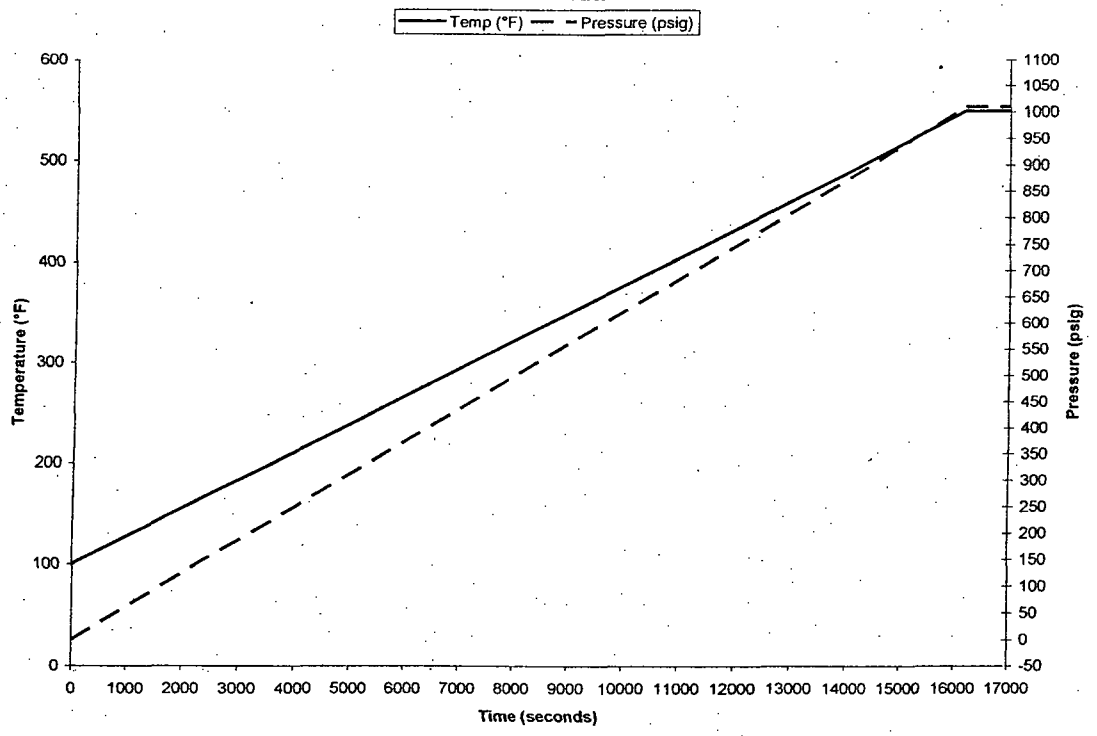

Figure 1: Transient 03: Start Up

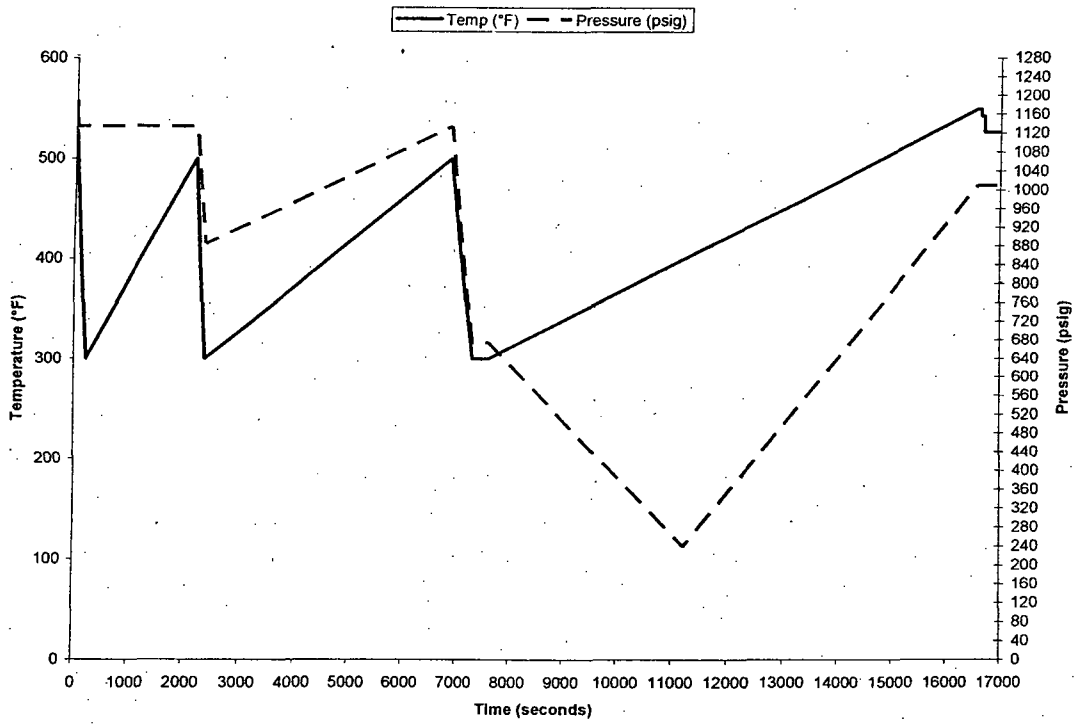

Figure 2: Transient 11: Loss of Feedwater Pumps, Isolation Valves Close

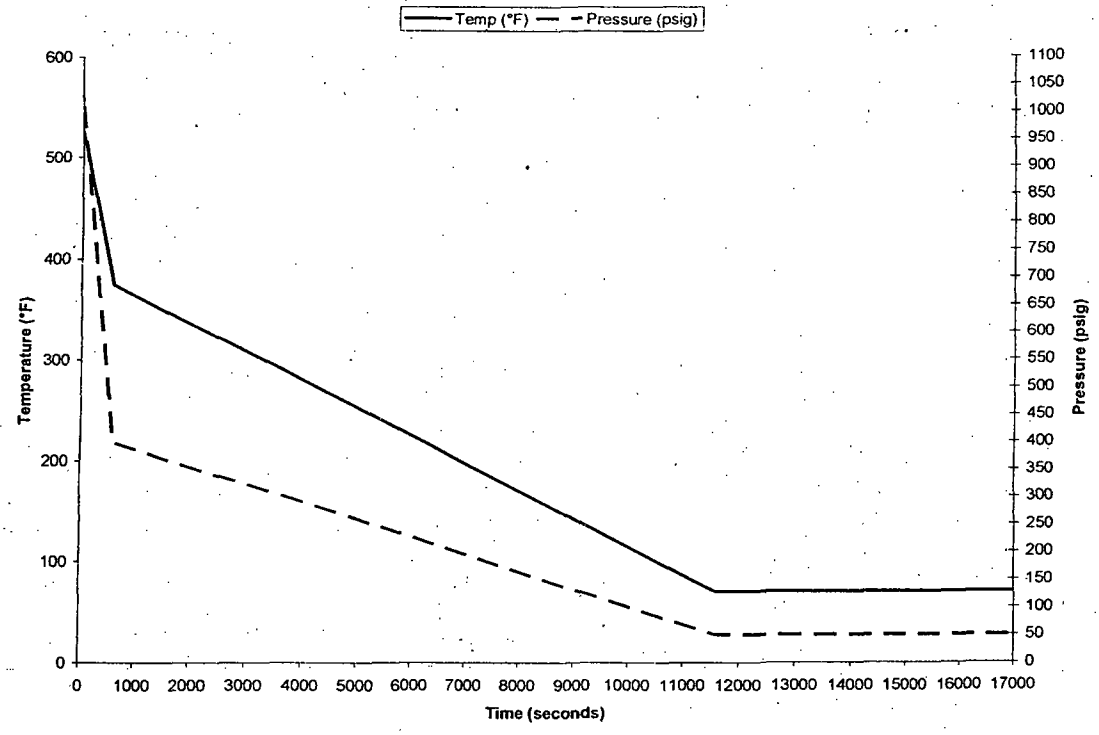

Figure 3: Transient 14: Single Relief of Safety Valve Blow Down

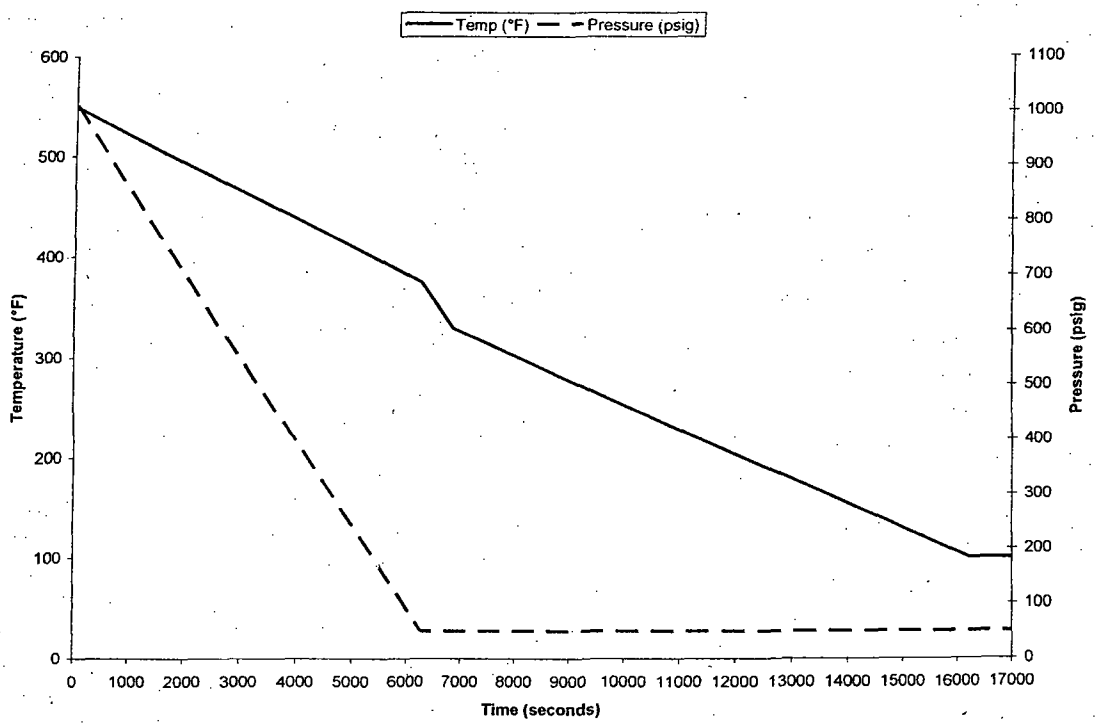

Figure 4: Transient 21-23: Shut Down Vessel Flooding

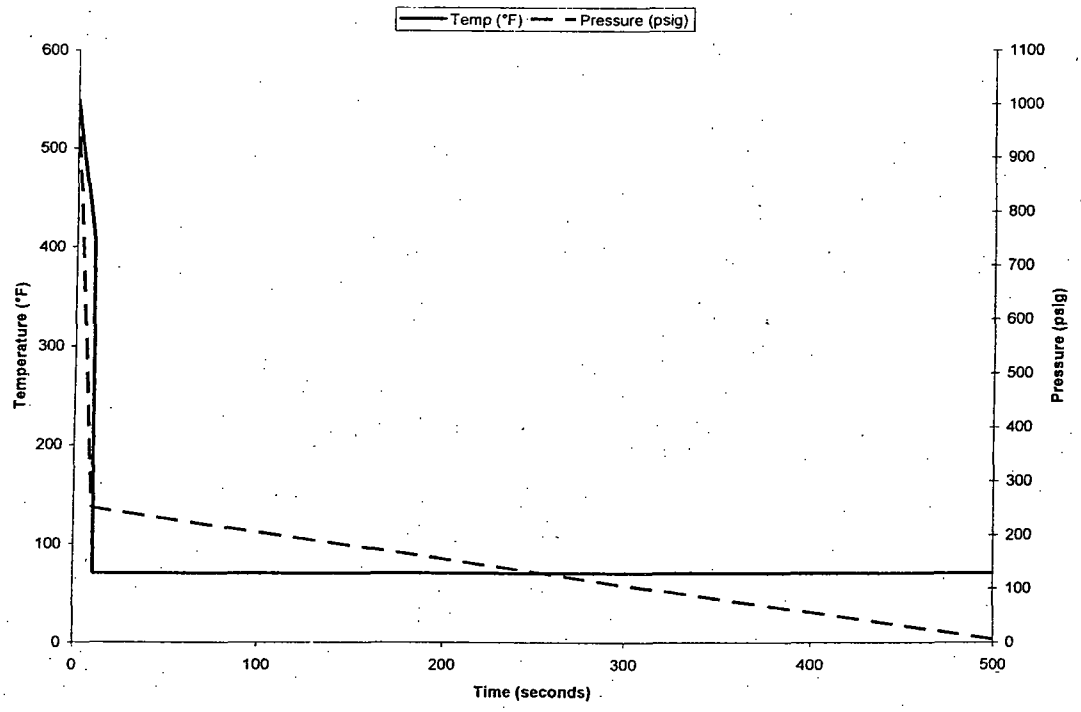

Figure 5: Transient 30: Emergency Shut Down 100% Flow (Safe End)

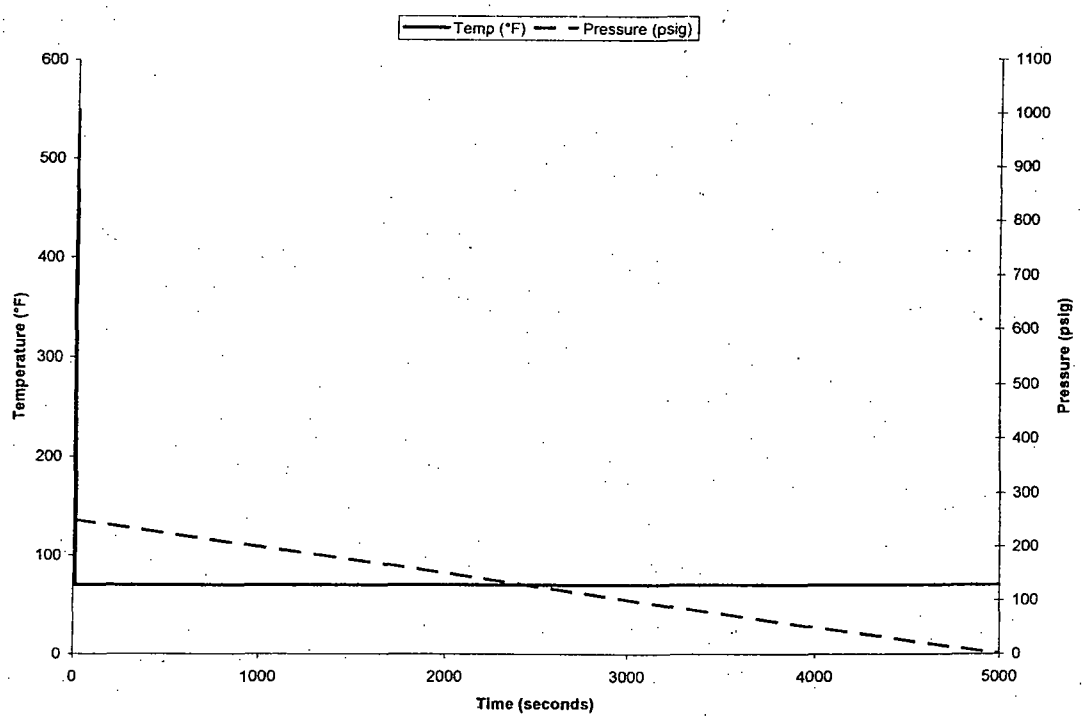

Figure 6: Transient 30: Emergency Shut Down 100% Flow (Blend Radius)

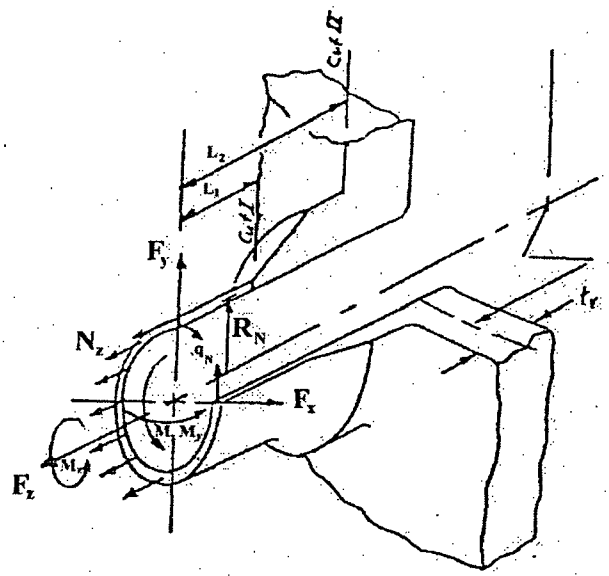

Figure 7: External Forces and Moments on the Core Spray Nozzle

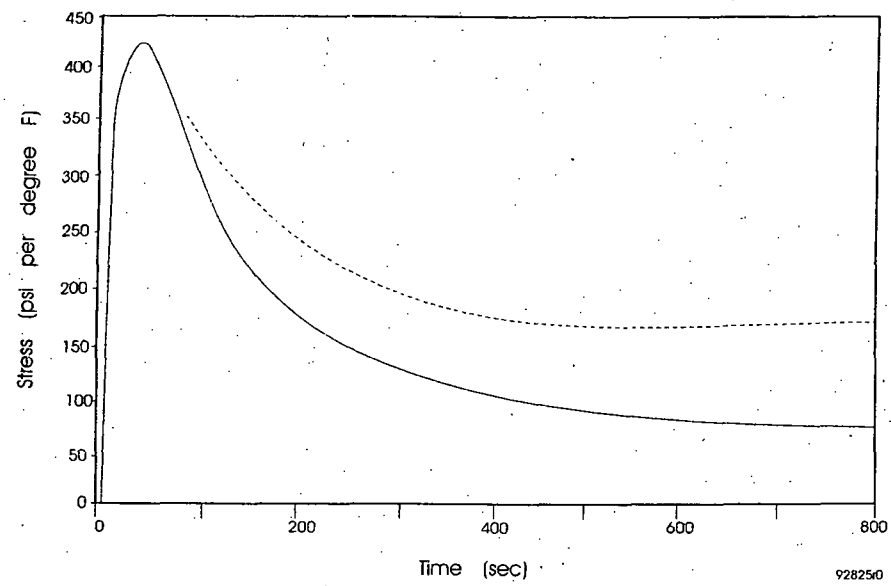

Figure 8: Typical Green's Functions for Thermal Transient Stress

Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

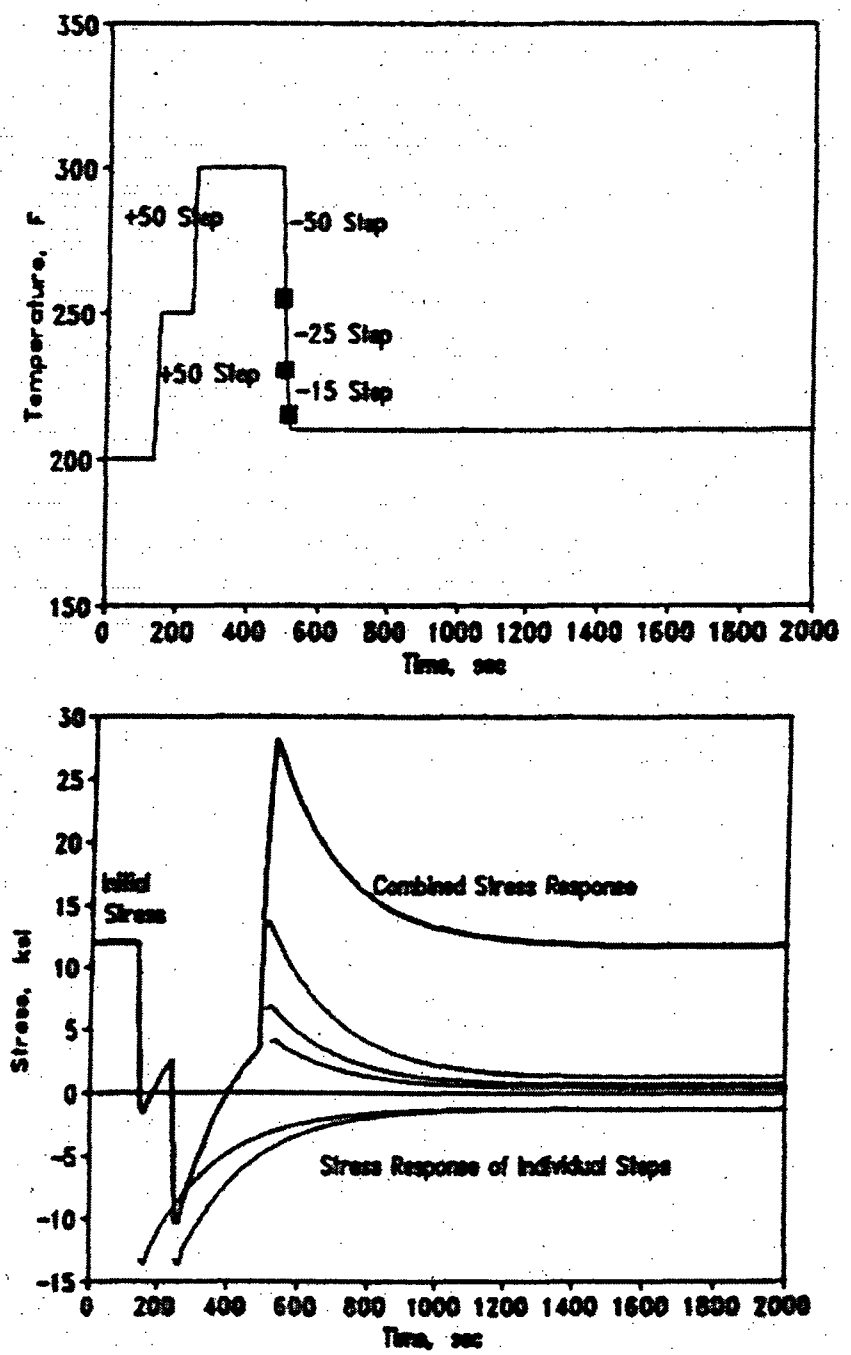

Figure 9: Typical Stress Response Using Green's Functions

APPENDIX A

INPUT AND OUTPUT FILES

Input Files

File Name	Description
TRANSNT_03.INP	Text file describing transient 03 for STRESS.EXE
TRANSNT_11.INP	Text file describing transient 11 for STRESS.EXE
TRANSNT_14.INP	Text file describing transient 14 for STRESS.EXE
TRANSNT_21_22_23.INP	Text file describing transients 21-23 for STRESS.EXE
TRANSNT_30.INP	Text file describing transient 30 for STRESS.EXE

Output Files

File Name	Description
P-V_03.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 03
P-V_11.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 11
P-V_14.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 14
P-V_21_22_23.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transients 21-23
P-V_30.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 30

EXHIBIT K

Structural Integrity Associates, Inc. CALCULATION PACKAGE	File No.: VY-16Q-311
	Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:	**PLANT:**
Entergy Nuclear Operations, Inc.	Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Feedwater Class 1 Piping Fatigue Analysis

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-17, A1 – A38, B1 – B45 In Computer Files	Initial Issue	Terry J. Herrmann 7/20/2007	Keith R. Evon 7/16/2007 Ryan V. Perry 7/16/2007

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1.0 OBJECTIVE

The purpose of this calculation is to perform an ASME Section III, NB-3600 fatigue calculation (Including environmental fatigue) of the Vermont Yankee (VY) Class 1 feedwater piping located inside the drywell (originally analyzed to B31.1 requirements). This section of piping was originally identified in the Recommendation Report [6] for installing a fatigue monitoring system at VY.

The fatigue calculation performed herein is not a certified ASME Code NB-3600 stress and fatigue analysis. Rather, it is an evaluation for the purposes of establishing fatigue usage to accommodate fatigue monitoring of the subject B31.1 piping. Although the PIPESTRESS program implements all ASME Code NB-3600 equations, only the fatigue usage results are utilized. All stress limit checks, although calculated by the program, are ignored since satisfactory stress limit checks were performed as a part of the already existing governing B31.1 stress analyses for all piping systems.

2.0 METHODOLOGY

The Class 1 Loop A feedwater piping system line extending from anchor HD-36 to reactor pressure vessel (RPV) nozzles N-4A and N-4B was evaluated. This includes a portion of the HPCI line to support HPCI-HD35A [7], so that the appropriate stiffness affects of this line on the feedwater piping are included. This evaluation is also considered valid for the Loop B line extending from anchor HD-39 to RPV nozzles N-4C and N-4D for the following reasons:

1. The Class 1 sections of Loop A and Loop B are mirror images of each other. This evaluation includes piping beyond the Class 1 boundary check valve so that its influence on the Class 1 piping is taken into account. The final fatigue analysis will only consider points on the Class 1 portion of the piping.
2. A 14" HPCI line tees into Loop A and a 4" RCIC line tees into Loop B. The HPCI line is more than three times the size of the RCIC line and will therefore have a greater influence on the feedwater piping.
3. The transients defined in this calculation are the bounding set for the two loops.

The operating conditions for the Class 1 portion of the feedwater line were defined based on References [11 and 12]. The resulting piping transient definitions are specified in Table 1. For each thermal cycle, the operating temperatures for Regions I through V define the conditions to be applied to the model.

Region boundaries are defined at branches, transitions, or locations where temperature and flow conditions change. These boundary locations are also shown in Figure 1. A listing of the PIPESTRESS input file "FWHPCI.FRE" is given in Appendix A and is also included in the project computer files.

Table 1: Thermal Cycle Definitions for Feedwater Line

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2) Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	T_{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pressure Conditions Pinit (psig)	Pfinal (psig)	No. of Cycles (1)
1	Design Hydrotest (Leak Test) (+)	I	100	70	100	1800	60	85	0	1	200.0	0.0	1100	120
		IIa	100	70	100	1800	60	85	0	1	150.0	0.0	1100	
		IIb	100	70	100	1800	60	85	0	1	150.0	0.0	1100	
		II	100	70	100	1800	60	85	0	1	150.0	0.0	50	
		III	100	70	100	1800	60	85	0	1	200.0	0.0	1100	
		IV	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
		IVa	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
		IVb	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
		V	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
2	Design Hydrotest (Leak Test) (-)	I	100	100	100	0	0	100	0	1	200.0	1100.0	50	120
		IIa	100	100	100	0	0	100	0	1	150.0	1100.0	50	
		IIb	100	100	100	0	0	100	0	1	150.0	1100.0	50	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	1100.0	50	
		IV	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
		IVa	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
		IVb	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
		V	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
3	Startup (+)	I	150	100	150	16164	11.1	125	0	1	200.0	50.0	1010	300
		IIa	150	100	150	16164	11.1	125	0	1	150.0	50.0	1010	
		IIb	125	100	125	16164	5.6	113	0	1	150.0	50.0	1010	
		II	100	100	100	16164	0.0	100	0	1	150.0	50.0	50	
		III	150	100	150	16164	11.1	125	0	1	200.0	50.0	1010	
		IV	150	100	150	16164	11.1	125	0	1/2	100.0	50.0	1010	
		IVa	283	100	283	16164	40.8	192	0	1/2	100.0	50.0	1010	
		IVb	416	100	416	16164	70.4	258	0	1/2	100.0	50.0	1010	
		V	549	100	549	16164	100	325	0	1/2	100.0	50.0	1010	
4	Turbine Roll & Increase to Rated Power 1 (-) (Includes 10 SCRAM, Loss of Feedwater Pumps and 300 Hot Standby - Feedwater Cycling)	I	100	150	100	0	STEP	125	15	1	1377.0	1010.0	1010	610
		IIa	100	150	100	0	STEP	125	0	1	150.0	1010.0	1010	
		IIb	100	125	100	0	STEP	113	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	100	150	100	0	STEP	125	15	1	1377.0	1010.0	1010	
		IV	100	150	100	0	STEP	125	15	1/2	688.5	1010.0	1010	
		IVa	100	283	100	0	STEP	192	15	1/2	688.5	1010.0	1010	
		IVb	100	416	100	0	STEP	258	15	1/2	688.5	1010.0	1010	
		V	100	549	100	0	STEP	325	15	1/2	688.5	1010.0	1010	
5	Turbine Roll & Increase to Rated Power 2 (+) (Includes 10 SCRAM, Loss of Feedwater Pumps, 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	260	100	260	0	STEP	180	15	1	1377.0	1010.0	1010	599
		IIa	260	100	260	0	STEP	180	0	1	150.0	1010.0	1010	
		IIb	180	100	180	0	STEP	140	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	260	100	260	0	STEP	180	15	1	1377.0	1010.0	1010	
		IV	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
		IVa	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
		IVb	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
		V	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
6	Turbine Roll & Increase to Rated Power 3 (+) (Includes 10 SCRAM, Loss of Feedwater Pumps, 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	260	392	1800	264	326	100	1	9180.0	1010.0	1010	599
		IIa	392	260	392	1800	264	326	0	1	150.0	1010.0	1010	
		IIb	246	180	246	1800	132	213	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	260	392	1800	264	326	100	1	9180.0	1010.0	1010	
		IV	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
		IVa	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
		IVb	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
		V	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
7	Daily Reduction to 75% Power (-)	I	310	392	310	900	-328	351	75	1	6885.0	1010.0	1010	10000
		IIa	310	392	310	900	-328	351	0	1	150.0	1010.0	1010	
		IIb	205	246	205	900	-164	226	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	310	392	310	900	-328	351	75	1	6885.0	1010.0	1010	
		IV	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
		IVa	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
		IVb	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
		V	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
8	Daily Reduction to 75% Power (+)	I	392	310	392	900	328	351	75	1	6885.0	1010.0	1010	10000
		IIa	392	310	392	900	328	351	0	1	150.0	1010.0	1010	
		IIb	246	205	246	900	164	226	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	310	392	900	328	351	75	1	6885.0	1010.0	1010	
		IV	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
		IVa	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
		IVb	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
		V	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
9	Weekly Reduction to 50% Power (-)	I	280	392	280	1800	-224	336	50	1	4590.0	1010.0	1010	2000
		IIa	280	392	280	1800	-224	336	0	1	150.0	1010.0	1010	
		IIb	190	246	190	1800	-112	218	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	280	392	280	1800	-224	336	50	1	4590.0	1010.0	1010	
		IV	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
		IVa	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
		IVb	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
		V	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2) Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	T_{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pressure Conditions Pinit (psig)	Pfinal (psig)	No. of Cycles (1)
10	Weekly Reduction to 50% Power (+)	I	392	280	392	1800	224	336	50	1	4590.0	1010.0	1010	2000
		IIa	392	280	392	1800	224	336	0	1	150.0	1010.0	1010	
		IIb	246	190	246	1800	112	218	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	280	392	1800	224	336	50	1	4590.0	1010.0	1010	
		IV	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
		IVa	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
		IVb	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
		V	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
11	Loss of Feedwater Heater, Turbine Trip 1 (-) (Includes 10 Loss of Feedwater Heater Turbine Trip, and 300 Reduction to 0% Power)	I	265	392	265	1800	-254	329	50	1	4590.0	1010.0	1010	310
		IIa	265	392	265	1800	-254	329	0	1	150.0	1010.0	1010	
		IIb	182.5	246	182.5	1800	-127	214	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	265	392	265	1800	-254	329	50	1	4590.0	1010.0	1010	
		IV	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
		IVa	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
		IVb	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
		V	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
12	Loss of Feedwater Heater, Turbine Trip 2 (-)	I	90	265	90	360	-1750	178	15	1	1377.0	1010.0	1010	10
		IIa	90	265	90	360	-1750	178	0	1	150.0	1010.0	1010	
		IIb	95	182.5	95	360	-875	139	0	1	150.0	1010.0	1010	
		II	100	100	100	360	0	100	0	1	150.0	50.0	50	
		III	90	265	90	360	-1750	178	15	1	1377.0	1010.0	1010	
		IV	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
		IVa	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
		IVb	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
		V	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
13	Loss of Feedwater Heater, Turbine Trip 3 (+)	I	265	90	265	900	700	178	15	1	1377.0	1010.0	1010	10
		IIa	265	90	265	900	700	178	0	1	150.0	1010.0	1010	
		IIb	182.5	95	182.5	900	350	139	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	265	90	265	900	700	178	15	1	1377.0	1010.0	1010	
		IV	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
		IVa	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
		IVb	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
		V	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
14	Loss of Feedwater Heater, Turbine Trip 4 (+)	I	392	265	392	1800	254	329	50	1	4590.0	1010.0	1010	10
		IIa	392	265	392	1800	254	329	0	1	150.0	1010.0	1010	
		IIb	246	182.5	246	1800	127	214	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	265	392	1800	254	329	50	1	4590.0	1010.0	1010	
		IV	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
		IVa	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
		IVb	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
		V	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
15	Loss of Feedwater Heater, FW Heater Bypass (-)	I	265	392	265	90	-5080	329	100	1	9180.0	1010.0	1010	70
		IIa	265	392	265	90	-5080	329	0	1	150.0	1010.0	1010	
		IIb	182.5	246	182.5	90	-2540	214	0	1	150.0	1010.0	1010	
		II	100	100	100	90	0	100	0	1	150.0	50.0	50	
		III	265	392	265	90	-5080	329	100	1	9180.0	1010.0	1010	
		IV	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
		IVa	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
		IVb	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
		V	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
16	Loss of Feedwater Heater, FW Heater Bypass (+)	I	392	265	392	180	2540	329	100	1	9180.0	1010.0	1010	70
		IIa	392	265	392	180	2540	329	0	1	150.0	1010.0	1010	
		IIb	246	182.5	246	180	1270	214	0	1	150.0	1010.0	1010	
		II	100	100	100	180	0	100	0	1	150.0	50.0	50	
		III	392	265	392	180	2540	329	100	1	9180.0	1010.0	1010	
		IV	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
		IVa	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
		IVb	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
		V	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
17	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 1 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	275	392	275	60	-7020	334	110	1	10098.0	1010.0	1010	289
		IIa	275	392	275	60	-7020	334	0	1	150.0	1010.0	1010	
		IIb	187.5	246	187.5	60	-3510	217	0	1	150.0	1010.0	1010	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	275	392	275	60	-7020	334	110	1	10098.0	1010.0	1010	
		IV	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
		IVa	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
		IVb	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
		V	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
18	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 2 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	100	275	100	900	-700	188	3	1	275.4	1010.0	1010	289
		IIa	100	275	100	900	-700	188	0	1	150.0	1010.0	1010	
		IIb	100	187.5	100	900	-350	144	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	100	275	100	900	-700	188	3	1	275.4	1010.0	1010	
		IV	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	
		IVa	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	
		IVb	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	
		V	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2) Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	T_{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pressure Conditions Pinit (psig)	Pfinal (psig)	No. of Cycles (1)
19	Hot Standby 1 (+)	I	265	265	265	0	STEP	265	0	1	200.0	1010.0	1010	300
		IIa	265	265	265	0	STEP	265	0	1	150.0	1010.0	1010	
		IIb	182.5	182.5	182.5	0	STEP	183	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	265	265	265	0	STEP	265	0	1	200.0	1010.0	1010	
		IV	265	265	265	0	STEP	265	0	1/2	100.0	1010.0	1010	
		IVa	323	265	323	0	STEP	294	0	1/2	100.0	1010.0	1010	
		IVb	382	265	382	0	STEP	324	0	1/2	100.0	1010.0	1010	
		V	440	265	440	0	STEP	353	0	1/2	100.0	1010.0	1010	
20	Hot Standby 2 (+)	I	265	265	265	0	0	265	0	1	200.0	1010.0	1010	300
		IIa	265	265	265	0	0	265	0	1	150.0	1010.0	1010	
		IIb	182.5	182.5	182.5	0	0	183	0	1	150.0	1010.0	1010	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	265	265	265	0	0	265	0	1	200.0	1010.0	1010	
		IV	265	265	265	0	0	265	0	1/2	100.0	1010.0	1010	
		IVa	360	323	360	3924	34	342	0	1/2	100.0	1010.0	1010	
		IVb	454	382	454	3924	66	418	0	1/2	100.0	1010.0	1010	
		V	549	440	549	3924	100	495	0	1/2	100.0	1010.0	1010	
21	Hot Standby 3 (-)	I	150	265	150	4140	-100	208	0	1	200.0	1010.0	1010	300
		IIa	150	265	150	4140	-100	208	0	1	150.0	1010.0	1010	
		IIb	125	182.5	125	4140	-50	154	0	1	150.0	1010.0	1010	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	265	150	4140	-100	208	0	1	200.0	1010.0	1010	
		IV	150	265	150	4140	-100	208	0	1/2	100.0	1010.0	1010	
		IVa	283	360	283	4140	-67	322	0	1/2	100.0	1010.0	1010	
		IVb	416	454	416	4140	-33	435	0	1/2	100.0	1010.0	1010	
		V	549	549	549	0	0	549	0	1/2	100.0	1010.0	1010	
22	Shutdown 1 (-)	I	150	150	150	0	0	150	0	1	200.0	1010.0	170	300
		IIa	150	150	150	0	0	150	0	1	150.0	1010.0	170	
		IIb	125	125	125	0	0	125	0	1	150.0	1010.0	170	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	150	150	0	0	150	0	1	200.0	1010.0	170	
		IV	150	150	150	0	0	150	0	1/2	100.0	1010.0	170	
		IVa	225	283	225	6264	-33	254	0	1/2	100.0	1010.0	170	
		IVb	300	416	300	6264	-67	358	0	1/2	100.0	1010.0	170	
		V	375	549	375	6264	-100	462	0	1/2	100.0	1010.0	170	
23	Shutdown 2 (-)	I	150	150	150	0	0	150	0	1	200.0	170.0	88	300
		IIa	150	150	150	0	0	150	0	1	150.0	170.0	88	
		IIb	125	125	125	0	0	125	0	1	150.0	170.0	88	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	150	150	0	0	150	0	1	200.0	170.0	88	
		IV	150	150	150	0	0	150	0	1/2	100.0	170.0	88	
		IVa	210	225	210	600	-90	218	0	1/2	100.0	170.0	88	
		IVb	270	300	270	600	-180	285	0	1/2	100.0	170.0	88	
		V	330	375	330	600	-270	353	0	1/2	100.0	170.0	88	
24	Shutdown 3 (-)	I	100	150	100	8280	-22	125	0	1	200.0	88.0	50	300
		IIa	100	150	100	8280	-22	125	0	1	150.0	88.0	50	
		IIb	100	125	100	8280	-11	113	0	1	150.0	88.0	50	
		II	100	100	100	8280	0	100	0	1	150.0	50.0	50	
		III	100	150	100	8280	-22	125	0	1	200.0	88.0	50	
		IV	100	150	100	8280	-22	125	0	1/2	100.0	88.0	50	
		IVa	100	210	100	8280	-48	155	0	1/2	100.0	88.0	50	
		IVb	100	270	100	8280	-74	185	0	1/2	100.0	88.0	50	
		V	100	330	100	8280	-100	215	0	1/2	100.0	88.0	50	
25	SCRAM, Loss of Feedwater Pumps 1 (+)	I	392	392	392	12	0	392	0	1	200.0	1010.0	1190	10
		IIa	392	392	392	12	0	392	0	1	150.0	1010.0	1190	
		IIb	246	246	246	12	0	246	0	1	150.0	1010.0	1190	
		II	100	100	100	12	0	100	0	1	150.0	50.0	50	
		III	392	392	392	12	0	392	0	1	200.0	1010.0	1190	
		IV	392	392	392	12	0	392	0	1/2	100.0	1010.0	1190	
		IVa	450	392	450	12	17400	421	0	1/2	100.0	1010.0	1190	
		IVb	507	392	507	12	34500	450	0	1/2	100.0	1010.0	1190	
		V	565	392	565	12	51900	479	0	1/2	100.0	1010.0	1190	
26	SCRAM, Loss of Feedwater Pumps 2 (-) (First HPCI)	I	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	10
		IIa	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	
		IIb	50	246	50	0	STEP	148	40	1	3672.0	1190.0	1135	
		II	50	100	50	0	STEP	75	40	1	3672.0	50.0	1135	
		III	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	
		IV	50	392	50	0	STEP	221	40	1/2	1836.0	1190.0	1135	
		IVa	50	450	50	0	STEP	250	40	1/2	1836.0	1190.0	1135	
		IVb	50	507	50	0	STEP	279	40	1/2	1836.0	1190.0	1135	
		V	50	565	50	0	STEP	308	40	1/2	1836.0	1190.0	1135	
27	SCRAM, Loss of Feedwater Pumps 3 (+)	I	150	50	150	1380	261	100	0	1	200.0	1135.0	1135	10
		IIa	150	50	150	1380	261	100	0	1	150.0	1135.0	1135	
		IIb	125	50	125	1380	196	88	0	1	150.0	1135.0	1135	
		II	100	50	100	1380	130	75	0	1	150.0	1135.0	50	
		III	150	50	150	1380	261	100	0	1	200.0	1135.0	1135	
		IV	150	50	150	1380	261	100	0	1/2	100.0	1135.0	1135	
		IVa	247	50	247	1380	514	149	0	1/2	100.0	1135.0	1135	
		IVb	343	50	343	1380	764	197	0	1/2	100.0	1135.0	1135	
		V	440	50	440	1380	1017	245	0	1/2	100.0	1135.0	1135	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2) Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	T_{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pressure Conditions Pinit (psig)	Pfinal (psig)	No. of Cycles (1)
28	SCRAM, Loss of Feedwater Pumps 4 (+)	I	150	150	150	0	STEP	150	0	1	200.0	1135.0	1135	10
		IIa	150	150	150	0	STEP	150	0	1	150.0	1135.0	1135	
		IIb	125	125	125	0	STEP	125	0	1	150.0	1135.0	1135	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	150	150	150	0	STEP	150	0	1	200.0	1135.0	1135	
		IV	150	150	150	0	STEP	150	0	1/2	100.0	1135.0	1135	
		IVa	288	247	288	0	STEP	268	0	1/2	100.0	1135.0	1135	
		IVb	427	343	427	0	STEP	385	0	1/2	100.0	1135.0	1135	
		V	565	440	565	0	STEP	503	0	1/2	100.0	1135.0	1135	
29	SCRAM, Loss of Feedwater Pumps 5 (-) (Second HPCI)	I	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885	10
		IIa	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885	
		IIb	50	125	50	0	STEP	88	30	1	2754.0	1135.0	885	
		II	50	100	50	0	STEP	75	30	1	2754.0	1135.0	885	
		III	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885	
		IV	50	150	50	0	STEP	100	30	1/2	1377.0	1135.0	885	
		IVa	50	288	50	0	STEP	169	30	1/2	1377.0	1135.0	885	
		IVb	50	427	50	0	STEP	239	30	1/2	1377.0	1135.0	885	
		V	50	565	50	0	STEP	308	30	1/2	1377.0	1135.0	885	
30	SCRAM, Loss of Feedwater Pumps 6 (+)	I	150	50	150	3060	118	100	0	1	200.0	885.0	1060	10
		IIa	150	50	150	3060	118	100	0	1	150.0	885.0	1060	
		IIb	125	50	125	3060	88	88	0	1	150.0	885.0	1060	
		II	100	50	100	3060	59	75	0	1	150.0	885.0	50	
		III	150	50	150	3060	118	100	0	1	200.0	885.0	1060	
		IV	150	50	150	3060	118	100	0	1/2	100.0	885.0	1060	
		IVa	247	50	247	3060	232	149	0	1/2	100.0	885.0	1060	
		IVb	343	50	343	3060	345	197	0	1/2	100.0	885.0	1060	
		V	440	50	440	3060	459	245	0	1/2	100.0	885.0	1060	
31	SCRAM, Loss of Feedwater Pumps 7 (+)	I	150	150	150	0	STEP	150	0	1	200.0	1060.0	1135	10
		IIa	150	150	150	0	STEP	150	0	1	150.0	1060.0	1135	
		IIb	125	125	125	0	STEP	125	0	1	150.0	1060.0	1135	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	150	150	150	0	STEP	150	0	1	200.0	1060.0	1135	
		IV	150	150	150	0	STEP	150	0	1/2	100.0	1060.0	1135	
		IVa	283	247	283	0	STEP	265	0	1/2	100.0	1060.0	1135	
		IVb	416	343	416	0	STEP	380	0	1/2	100.0	1060.0	1135	
		V	549	440	549	0	STEP	495	0	1/2	100.0	1060.0	1135	
32	SCRAM, Loss of Feedwater Pumps 8 (-) (Third HPCI)	I	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675	10
		IIa	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675	
		IIb	50	125	50	0	STEP	88	17	1	1560.6	1135.0	675	
		II	50	100	50	0	STEP	75	17	1	1560.6	50.0	675	
		III	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675	
		IV	50	150	50	0	STEP	100	17	1/2	780.3	1135.0	675	
		IVa	50	283	50	0	STEP	167	17	1/2	780.3	1135.0	675	
		IVb	50	416	50	0	STEP	233	17	1/2	780.3	1135.0	675	
		V	50	549	50	0	STEP	300	17	1/2	780.3	1135.0	675	
33	SCRAM, Loss of Feedwater Pumps 9 (+)	I	150	50	150	300	1200	100	0	1	200.0	675.0	675	10
		IIa	150	50	150	300	1200	100	0	1	150.0	675.0	675	
		IIb	125	50	125	300	900	88	0	1	150.0	675.0	675	
		II	100	50	100	300	600	75	0	1	150.0	675.0	50	
		III	150	50	150	300	1200	100	0	1	200.0	675.0	675	
		IV	150	50	150	300	1200	100	0	1/2	100.0	675.0	675	
		IVa	200	50	200	300	1800	125	0	1/2	100.0	675.0	675	
		IVb	250	50	250	300	2400	150	0	1/2	100.0	675.0	675	
		V	300	50	300	300	3000	175	0	1/2	100.0	675.0	675	
34	SCRAM, Loss of Feedwater Pumps 10 (+)	I	150	150	150	8964	0	150	0	1	200.0	240.0	1010	10
		IIa	150	150	150	8964	0	150	0	1	150.0	240.0	1010	
		IIb	125	125	125	8964	0	125	0	1	150.0	240.0	1010	
		II	100	100	100	8964	0	100	0	1	150.0	50.0	50	
		III	150	150	150	8964	0	150	0	1	200.0	240.0	1010	
		IV	150	150	150	8964	0	150	0	1/2	100.0	240.0	1010	
		IVa	283	200	283	8964	33	242	0	1/2	100.0	240.0	1010	
		IVb	416	250	416	8964	67	333	0	1/2	100.0	240.0	1010	
		V	549	300	549	8964	100	425	0	1/2	100.0	240.0	1010	
35	SCRAM, SRV Blowdown 1 (-)	I	275	392	275	60	-7020	334	110	1	10098.0	1010.0	885	1
		IIa	275	392	275	60	-7020	334	0	1	150.0	1010.0	885	
		IIb	187.5	246	187.5	60	-3510	217	0	1	150.0	1010.0	885	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	275	392	275	60	-7020	334	110	1	10098.0	1010.0	885	
		IV	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885	
		IVa	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885	
		IVb	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885	
		V	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885	
36	SCRAM, SRV Blowdown 2 (-)	I	100	275	100	900	-700	188	3	1	275.4	885.0	50	1
		IIa	100	275	100	900	-700	188	0	1	150.0	885.0	50	
		IIb	100	187.5	100	900	-350	144	0	1	150.0	885.0	50	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	100	275	100	900	-700	188	3	1	275.4	885.0	50	
		IV	100	275	100	900	-700	188	3	1/2	137.7	885.0	50	
		IVa	100	275	100	900	-700	188	3	1/2	137.7	885.0	50	
		IVb	100	275	100	900	-700	188	3	1/2	137.7	885.0	50	
		V	100	275	100	900	-700	188	3	1/2	137.7	885.0	50	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2) Oper. Temp. (°F)	T_{init} (°F)	T_{final} (°F)	Time (sec.)	Rate (°F/hr)	T_{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pressure Conditions Pinit (psig)	Pfinal (psig)	No. of Cycles (1)
37	Hydrostatic Test (+)	I	100	100	100	0	0	100	0	1	200.0	50.0	1563	1
		IIa	100	100	100	0	0	100	0	1	150.0	50.0	1563	
		IIb	100	100	100	0	0	100	0	1	150.0	50.0	1563	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	50.0	1563	
		IV	100	100	100	0	0	100	0	1/2	100.0	50.0	1563	
		IVa	100	100	100	0	0.0	100	0	1/2	100.0	50.0	1563	
		IVb	100	100	100	0	0.0	100	0	1/2	100.0	50.0	1563	
		V	100	100	100	0	0	100	0	1/2	100.0	50.0	1563	
38	Hydrostatic Test (-)	I	100	100	100	0	0	100	0	1	200.0	1563.0	50	1
		IIa	100	100	100	0	0	100	0	1	150.0	1563.0	50	
		IIb	100	100	100	0	0	100	0	1	150.0	1563.0	50	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	1563.0	50	
		IV	100	100	100	0	0	100	0	1/2	100.0	1563.0	50	
		IVa	100	100	100	0	0 0	100	0	1/2	100.0	1563.0	50	
		IVb	100	100	100	0	0.0	100	0	1/2	100.0	1563.0	50	
		V	100	100	100	0	0	100	0	1/2	100.0	1563.0	50	
39	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 1 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	60	0	392	110	1	10098.0	1010.0	1375	289
		IIa	392	392	392	60	0	392	0	1	150.0	1010.0	1375	
		IIb	246	246	246	60	0	246	0	1	150 0	1010.0	1375	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	392	392	392	60	0	392	110	1	10098.0	1010.0	1375	
		IV	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
		IVa	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
		IVb	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
		V	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
40	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 2 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	900	0	392	3	1	275.4	1375.0	940	289
		IIa	392	392	392	900	0	392	0	1	150.0	1375.0	940	
		IIb	246	246	246	900	0	246	0	1	150.0	1375.0	940	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	392	392	900	0	392	3	1	275 4	1375.0	940	
		IV	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
		IVa	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
		IVb	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
		V	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
41	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 3 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	900	0	392	3	1	275.4	940.0	1010	289
		IIa	392	392	392	900	0	392	0	1	150.0	940.0	1010	
		IIb	246	246	246	900	0	246	0	1	150.0	940.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	392	392	900	0	392	3	1	275.4	940.0	1010	
		IV	392	392	392	900	0	392	3	1/2	137.7	940 0	1010	
		IVa	392	392	392	900	0	392	3	1/2	137.7	940.0	1010	
		IVb	392	392	392	900	0	392	3	1/2	137.7	940.0	1010	
		V	392	392	392	900	0	392	3	1/2	137.7	940.0	1010	
42	Hot Standby, Feedwater Cycling 1 (+)	I	125	100	125	60	1500	113	0	1	200.0	1010.0	1010	300
		IIa	125	100	125	60	1500	113	0	1	150.0	1010.0	1010	
		IIb	112 5	100	112.5	60	750	106	0	1	150.0	1010.0	1010	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	125	100	125	60	1500	113	0	1	200.0	1010.0	1010	
		IV	125	100	125	60	1500	113	0	1/2	100.0	1010 0	1010	
		IVa	180	100	180	60	4800	140	0	1/2	100.0	1010.0	1010	
		IVb	235	100	235	60	8100	168	0	1/2	100.0	1010.0	1010	
		V	290	100	290	60	11400	195	0	1/2	100.0	1010.0	1010	
43	Hot Standby, Feedwater Cycling 2 (+)	I	150	125	150	210	429	138	0	1	200.0	1010.0	1010	300
		IIa	150	125	150	210	429	138	0	1	150.0	1010.0	1010	
		IIb	125	112.5	125	210	214	119	0	1	150.0	1010.0	1010	
		II	100	100	100	210	0	100	0	1	150.0	50.0	50	
		III	150	125	150	210	429	138	0	1	200.0	1010.0	1010	
		IV	150	125	150	210	429	138	0	1/2	100 0	1010.0	1010	
		IVa	283	180	283	210	1766	232	0	1/2	100.0	1010 0	1010	
		IVb	416	235	416	210	3103	326	0	1/2	100.0	1010.0	1010	
		V	549	290	549	210	4440	420	0	1/2	100.0	1010.0	1010	

For notes, see next page.

Table 1: Thermal Cycle Definitions for Feedwater Line (concluded)

Notes:

1. From Reference [13].

2. Normal operating conditions are 1,010 psig, 549°F (steam dome), 392°F (feedwater), and 4590 gpm (feedwater nozzle) [14].

3. See Figure 1.

4. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The same approximate value was used within each region. These values are:

   - 200 gpm for Regions I and III.
   - 150 gpm for Regions II, IIa, and IIb.
   - 100 gpm for Regions IV and V.

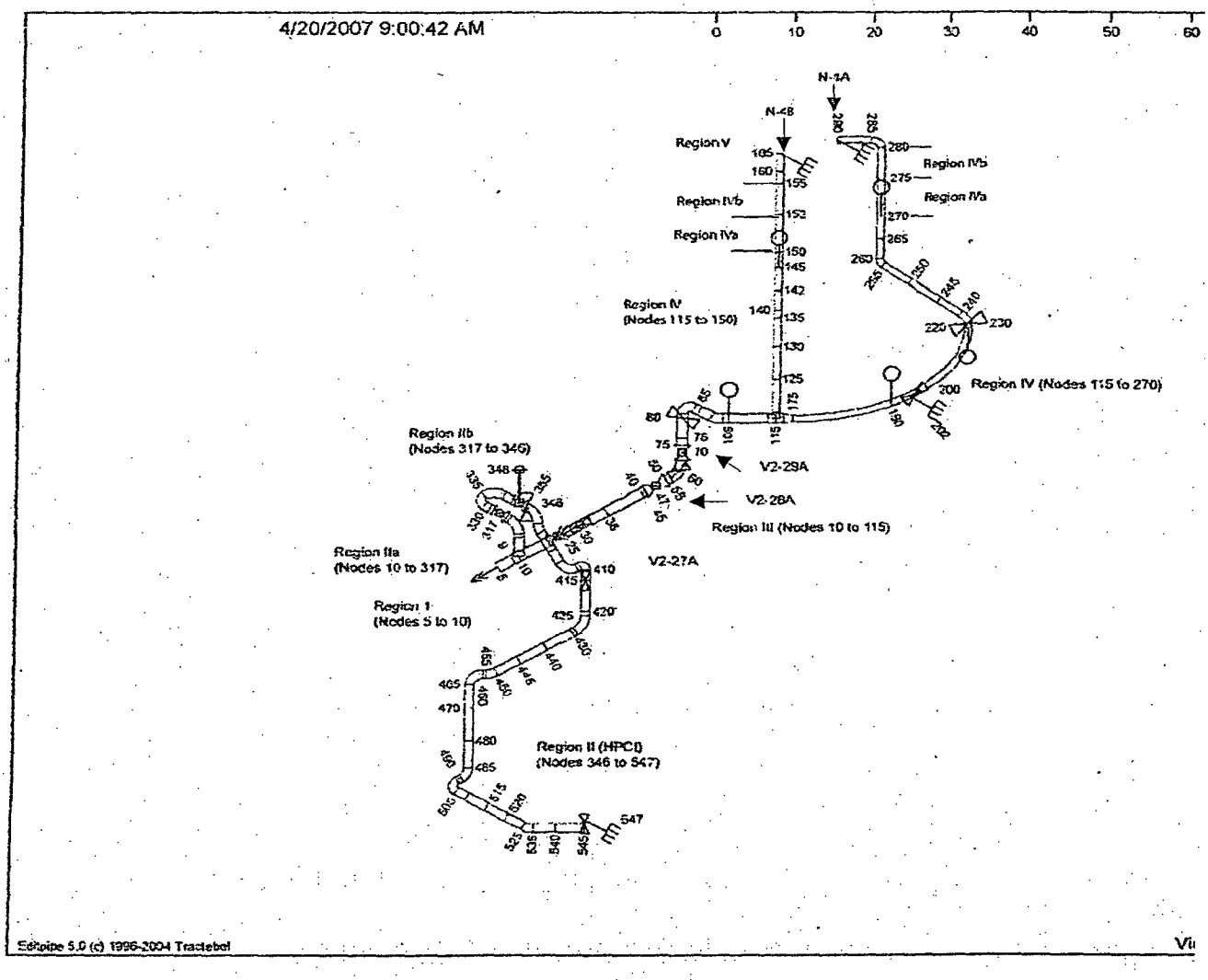

Figure 1: Feedwater/HPCI Piping from Anchor HD-36 to RPV Nozzles N-4A and

3.0 ASSUMPTIONS/DESIGN INPUT

In order to take advantage of improvements in the ASME Code that result in a lower calculated fatigue usage, this evaluation is done to the ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda [9]. The 1998 Edition of Section III (with 2000 Addenda) has been accepted by the US NRC for use in design analyses. Although there are a few restrictions on the application of this Edition, they involve the use of optional increased allowables that are not being used in this calculation.

A piping model was created using PIPESTRESS [1]. The calculation [2] that had previously analyzed the subject Class 1 feedwater piping contains the ADLPIPE input file used to create the PIPESTRESS input file for this evaluation. Valve dimensions and properties were also obtained from the ADLPIPE input file. The piping model is composed of one carbon steel grade (maximum carbon content of 0.30 %) [2]. Temperature dependent material properties were used with values obtained from Reference [5]. Table 2 summarizes these values. The resulting PIPESTRESS model (including boundary conditions) is shown in Figure 1. The drawings for both feedwater loops [3, 4] and the HPCI line [7] were also consulted to aid in building the PIPESTRESS model.

Assumptions:

1) The weight of insulation is included in the analysis and PIPESTRESS calculates the heat transfer effects of insulation.
2) Node 545 is the end of the as-modeled HPCI piping system. This is appropriate because of the distance from the HPCI/Feedwater tee, six pipe supports in the segment and multiple pipe direction changes.

The feedwater and HPCI line sizes are specified in the previous calculation [2] and are shown in Table 3.

Table 2: Material Properties for Feedwater System Class 1 Piping [2 App. E, 5]

SA 106 B and SA-234 WPB (Carbon Silicon Steel, C-Si)							
Temperature (°F)	Young's Modulus ($x10^6$ psi)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion(1) (10^{-6} /in/in/°F)	Thermal Conductivity(1) (btu/hr/ft/°F)	Thermal Diffusivity(1) (ft^2/hr)	Yield Stress S_y (ksi)	Design Stress Intensity S_m (ksi)
50	29.6	0(2)				35.0	20.0
70	29.5	0	6.4	27.5	0.529	35.0	20.0
100	29.3	0.2		27.6	0.512	35.0	20.0
150				27.6	0.496		
200	28.8	1.0		27.6	0.486	32.1	20.0
250				27.4	0.467		
300	28.3	1.9		27.2	0.453	31.0	20.0
350				27.0	0.440		
400	27.7	2.8		26.7	0.428	29.9	20.0
450				26.3	0.413		
500	27.3	3.7		25.9	0.398	28.5	18.9
550				25.5	0.387		
600	26.7	4.7		25.0	0.374	26.8	17.3

Notes:

1. These properties are used for the transient analysis only.
2. Assumed equivalent to the value at 70°F.

The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions.

Table 3: Feedwater/HPCI Piping Size Information [2]

	16" FW Downstream of V2-29A	16" FW Upstream of V2-29A	10" FW	14" HPCI
Pipe Schedule	80	120	120	120
Fittings Schedule	120	---	120	---
Piping O.D. (in.)	16.0	16.0	10.75	14.0
Piping Nom. Wall (in.)	0.843	1.218	0.843	1.093
Fitting Nom. Wall (in.)	1.218	---	0.843	---
Pipe Weight[1] (lb/ft)	136.46	192.3	89.20	150.7
Insulation Weight (lb/ft)	14.64	11.98	8.92	10.65[2]

Note:
1. Weight of contents automatically added by the PIPESTRESS Program.
2. Insulation weight assumed to be consistent with thickness (2 inches) and composition of insulation on the 16" FW upstream of V2-29A.

4.0 ANALYSIS

Through-wall thermal gradient terms were calculated by the PIPESTRESS program for all of the transients. Table 1 defines each thermal cycle definition (i.e,, transient load case) and the region of the modeled piping those conditions are applicable.

The forces and moments due to differential thermal expansion need to be included in the fatigue evaluation. The differential thermal expansion cases as analyzed by the piping program, PIPESTRESS, correspond to the end temperature and pressure of the transient. Table 4 lists the thermal expansion cases.

The material properties were obtained from the ASME Code Section II, 1998 Edition, Part D, with 2000 Addenda [5]. E and α are taken at 70°F, and k, ρ, and c_p are taken at the average temperature over the range of the individual transients.

The internal heat transfer coefficient h for the transients with flow occurring in the pipe is calculated based on the following relation for forced convection [8]:

$$h = 0.023\ Re^{0.8}\ Pr^{0.4}\ k/D$$

Where Re = Reynolds number
Pr = Prandtl number

The heat transfer coefficients were calculated by PIPESTRESS using the above relation. The flow rates described for each transient in Table 1 were used. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The formula for h is [8]:

$$h = 0.55\ (Gr\ Pr)^{0.25} k/L$$

Where Gr = Grashof Number
L = pipe diameter

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same heat transfer coefficient as the free convection coefficient.

As discussed in the next section, the PIPESTRESS input file "FWHPCI.FRE" will be run and analyzed to Section III, Subsection NB-3600 of ASME 1998 Edition [9] in order to evaluate acceptable fatigue usage values for the Class 1 feedwater loop A system. The code option available in PIPESTRESS is the 1998 edition without addenda. This is acceptable as the 1999 and 2000 addenda to the 1998 code did not change the fatigue analysis method which PIPESTRESS uses.

A Listing of the PIPESTRESS input is included as Appendix A.

Table 4: Thermal Cycle Load Cases

Load Set	Transients Represented	Region I Temp. (°F)	Region IIa Temp. (°F)	Region IIb Temp. (°F)	Region II Temp. (°F)	Region III Temp. (°F)	Region IV Temp. (°F)	Region IVa Temp. (°F)	Region IVb Temp. (°F)	Region V Temp. (°F)	Vessel Temp. (°F)	Region II Pressure (psig)	All other Regions Pressure (psig)
1	1	100	100	100	100	100	100	100	100	100	100	50	1100
2	2, 24, 36, 38	100	100	100	100	100	100	100	100	100	100	50	50
3	3, 21, 34, 43	150	150	125	100	150	150	283	416	549	549	50	1010
4	5	260	260	180	100	260	260	260	260	260	549	50	1010
5	6, 8, 10, 14, 16	392	392	246	100	392	392	392	392	392	549	50	1010
6	7	310	310	205	100	310	310	310	310	310	549	50	1010
7	9	280	280	190	100	280	280	280	280	280	549	50	1010
8	11, 13, 15	265	265	182.5	100	265	265	265	265	265	549	50	1010
9	12	90	90	95	100	90	90	90	90	90	549	50	1010
10	20	265	265	182.5	100	265	265	360	454	549	549	50	1010
11	22	150	150	125	100	150	150	225	300	375	375	50	170
12	23	150	150	125	100	150	150	210	270	330	330	50	88
13	25	392	392	246	100	392	392	450	507	565	565	50	1190
14	26	50	50	50	50	50	50	50	50	50	565	1135	1135
15	27	150	150	125	100	150	150	247	343	440	565	50	1135
16	28	150	150	125	100	150	150	288	427	565	565	50	1135
17	30	150	150	125	100	150	150	247	343	440	555	50	1060
18	31	150	150	125	100	150	150	283	416	549	565	50	1135
19	32	50	50	50	50	50	50	50	50	50	502	675	675
20	33	150	150	125	100	150	150	200	250	300	502	50	675
21	35	275	275	187.5	100	275	275	275	275	275	549	50	885
22	37	100	100	100	100	100	100	100	100	100	100	50	1563
23	39	392	392	246	100	392	392	392	392	392	600	50	1375
24	40	392	392	246	100	392	392	392	392	392	539	50	940
25	41	392	392	246	100	392	392	392	392	392	549	50	1010
26	17	275	275	187.5	100	275	275	275	275	275	539	50	1010
27	19	265	265	182.5	100	265	265	323	382	440	549	50	1010
28	4	100	100	100	100	100	100	100	100	100	549	50	1010
29	18	100	100	100	100	100	100	100	100	100	539	50	1010
30	42	125	125	112.5	100	125	125	180	235	290	549	50	1010
31	29	50	50	50	50	50	50	50	50	50	532	885	885

5.0 RESULTS OF ANALYSIS

Since the piping at VY was designed in accordance with USAS B31.1 methodology, fatigue analysis does not exist for the piping. Therefore, fatigue calculations are being developed for selected locations in the Class 1 piping systems at VY. This will result in detailed, Class 1 fatigue calculations for each selected location. Piping models and transient definitions have been developed for the Class 1 portion of the feedwater system, as documented in the previous sections of this calculation.

The limiting total fatigue usage for the analyzed feedwater/HPCI piping system occurs at Node 155 on the riser to the feedwater nozzle. The total usage at this location is U = 0.1661 (per the PIPESTRESS report FWHPCI.PRF) which passes Class 1 fatigue evaluation. The second highest total fatigue usage for the analyzed feedwater/HPCI piping system occurs at Node 175, the 16" to 10" reducer on the feedwater piping. The total usage at this location is U = 0.1114 (per the PIPESTRESS report FWHPCI.PRF) which passes Class 1 fatigue evaluation. The environmental fatigue multiplier to use from Reference [10] is 1.74. The total usage including environmental effects is therefore 0.289.

Appendix B contains the fatigue usage summary for node 155.

6.0 REFERENCES

1. PIPESTRESS, Version 3.5.1+0.26, DST Computer Services S.A., QA-1670-301, June, 2004.

2. HPCI/FW Piping Stress Information. ADLPIPE listing for FDW & HPCI piping from Calculation No. VYC-551, Rev. 2, Appendix A, SI File No. VY-05Q-229.

3. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-FDW-Part 5, Rev. 1, "Piping Isometric Feedwater: Drywell-Main Steam Tunnel (FDW) Part 5," SI File No. VY-05Q-221.

4. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-FDW-Part 5A, Rev. 1, "Piping Isometric Feedwater: Main Steam Tunnel and Drywell FDW-Part 5A," SI File No. VY-05Q-221.

5. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section II, Materials, Part D, "Properties (Customary)," 1998 Edition including the 2000 Addenda.

6. Structural Integrity Associates Report No. SIR-01-130, Revision 0, "System Review and Recommendations for a Transient and Fatigue Monitoring System at the Vermont Yankee Nuclear Power Station," February 2002, SI File No. VY-05Q-401.

7. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-HPCI-Part 5, Rev. 0, "Piping Isometric Drawing High Pressure Coolant Injection Main Steam Tunnel-Torus Area (HPCI) Part 5," SI File No. VY-05Q-223.

8. Holman, J.P., *Heat Transfer*, Fifth Edition, McGraw-Hill, 1981.

9. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, 1998 Edition including the 2000 Addenda.

10. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."

11. "Reactor Thermal Cycles," Attachment 1, page 2, of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

12. "Nozzle Thermal Cycles (Feedwater)," Attachment 1, page 3, of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

13. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.

14. GE Certified Design Specification No. 26A6019, Revision 1, "REACTOR VESSEL – EXTENDED POWER UPRATE," August 29, 2003, SI File No. VY-05Q-236.

APPENDIX A

PIPESTRESS INPUT FILE ("FWHPCI.FRE")

(Pages A1 – A38)

```
IDEN JB=2     *Job number (1 to 9999)
     CD=1     *1=ASME Section III
     VA=0     *0=Calculate            2=Verify
     GR=-Y    *Direction of gravity
     IU=1     *Input units            0=SIU        1=USA
     OU=1     *Output units           0=SIU        1=USA
     CH=$     *Delimiter character
     AB=T     *FREE errors =abort
     PL=$Vermont Yankee$
     EN=$KRE$
TITL BL=3     *Modeling option:
              *  3 =uniform mass for static analysis
              *      lumped mass for dynamic analysis
              *      rotational inertia ignored
     GL=1     *Report forces/moment         0=Global     1=Local     2=G et L
     SU=1     *Support summary              0=No         1=Yes
     CV=15    *Code version - See Manual
     HS=1     *Highest 20 stress ratios for each case
     MD=1     *Hot modulus
     TI=$Vermont Yankee Feedwater Piping$
        $SI Fatigue Analysis$
FREQ RF=1 RP=8 FR=33 MP=20 MX=70 TI=$SEISMIC$
*
*************************************
**** THERMAL CYCLE LOAD CASES****
*************************************
LCAS RF=0 CA=1 TY=0 TI=$LC-1$      *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$      *TC-2,24,36,38
LCAS RF=0 CA=3 TY=0 TI=$LC-3$      *TC-3,21,34,43
LCAS RF=0 CA=4 TY=0 TI=$LC-4$      *TC-5
LCAS RF=0 CA=5 TY=0 TI=$LC-5$      *TC-6,8,10,14,16
LCAS RF=0 CA=6 TY=0 TI=$LC-6$      *TC-7
LCAS RF=0 CA=7 TY=0 TI=$LC-7$      *TC-9
LCAS RF=0 CA=8 TY=0 TI=$LC-8$      *TC-11,13,15
LCAS RF=0 CA=9 TY=0 TI=$LC-9$      *TC-12
LCAS RF=0 CA=10 TY=0 TI=$LC-10$    *TC-20
LCAS RF=0 CA=11 TY=0 TI=$LC-11$    *TC-22
LCAS RF=0 CA=12 TY=0 TI=$LC-12$    *TC-23
LCAS RF=0 CA=13 TY=0 TI=$LC-13$    *TC-25
LCAS RF=0 CA=14 TY=0 TI=$LC-14$    *TC-26, 29
LCAS RF=0 CA=15 TY=0 TI=$LC-15$    *TC-27
LCAS RF=0 CA=16 TY=0 TI=$LC-16$    *TC-28
LCAS RF=0 CA=17 TY=0 TI=$LC-17$    *TC-30
LCAS RF=0 CA=18 TY=0 TI=$LC-18$    *TC-31
LCAS RF=0 CA=19 TY=0 TI=$LC-19$    *TC-32
LCAS RF=0 CA=20 TY=0 TI=$LC-20$    *TC-33
LCAS RF=0 CA=21 TY=0 TI=$LC-21$    *TC-35
LCAS RF=0 CA=22 TY=0 TI=$LC-22$    *TC-37
LCAS RF=0 CA=23 TY=0 TI=$LC-23$    *TC-39
LCAS RF=0 CA=24 TY=0 TI=$LC-24$    *TC-40
LCAS RF=0 CA=25 TY=0 TI=$LC-25$    *TC-41
LCAS RF=0 CA=26 TY=0 TI=$LC-26$    *TC-17
LCAS RF=0 CA=27 TY=0 TI=$LC-27$    *TC-19
LCAS RF=0 CA=28 TY=0 TI=$LC-28$    *TC-4
LCAS RF=0 CA=29 TY=0 TI=$LC-29$    *TC-18
LCAS RF=0 CA=30 TY=0 TI=$LC-30$    *TC-42
```

```
LCAS RF=0 CA=31 TY=0 TI=$LC-31     *TC-29
LCAS RF=6 CA=32 TY=6 TI=$SAM$
*********************
**** WEIGHT CASES****
*********************
LCAS CA=101  RF=1  TY=3  TI=$OPERATING WEIGHT$
LCAS CA=102  RF=2  TY=4  TI=$HYDROTEST WEIGHT$
*
********************************
**** THERMAL TRANSIENT CASES****
********************************
TCAS CA=201 RP=1 TI=$Design Hydrotest +$
TCAS CA=202 RP=1 TI=$Design Hydrotest -$
TCAS CA=203 RP=1 TI=$Startup +$
TCAS CA=204 RP=1 TI=$TRoll & Inc. PWR1 -$
TCAS CA=205 RP=1 TI=$TRoll & Inc. PWR2 +$
TCAS CA=206 RP=1 TI=$TRoll & Inc. PWR3 +$
TCAS CA=207 RP=1 TI=$DlyReduction to 75% -$
TCAS CA=208 RP=1 TI=$DlyReduction to 75% +$
TCAS CA=209 RP=1 TI=$WklyReduct to 50% -$
TCAS CA=210 RP=1 TI=$WklyReduct to 50% +$
TCAS CA=211 RP=1 TI=$LOFWH+TT 1 -$
TCAS CA=212 RP=1 TI=$LOFWH+TT 2 -$
TCAS CA=213 RP=1 TI=$LOFWH+TT 3 +$
TCAS CA=214 RP=1 TI=$LOFWH+TT 4 +$
TCAS CA=215 RP=1 TI=$LOFWH+PFWHTR Byp -$
TCAS CA=216 RP=1 TI=$LOFWH+PFWHTR Byp +$
TCAS CA=217 RP=1 TI=$SCRAM+TT+AllOtrScm -$
TCAS CA=218 RP=1 TI=$SCRAM+TT+AllOtrScm -$
TCAS CA=219 RP=1 TI=$HotStandby 1 +$
TCAS CA=220 RP=1 TI=$HotStandby 2 +$
TCAS CA=221 RP=1 TI=$HotStandby 3 -$
TCAS CA=222 RP=1 TI=$Shutdown 1 -$
TCAS CA=223 RP=1 TI=$Shutdown 2 -$
TCAS CA=224 RP=1 TI=$Shutdown 3 -$
TCAS CA=225 RP=1 TI=$SCRAM+LOFWP1 +$
TCAS CA=226 RP=1 TI=$SCRAM+LOFWP2 -$
TCAS CA=227 RP=1 TI=$SCRAM+LOFWP3 +$
TCAS CA=228 RP=1 TI=$SCRAM+LOFWP4 +$
TCAS CA=229 RP=1 TI=$SCRAM+LOFWP5 -$
TCAS CA=230 RP=1 TI=$SCRAM+LOFWP6 +$
TCAS CA=231 RP=1 TI=$SCRAM+LOFWP7 +$
TCAS CA=232 RP=1 TI=$SCRAM+LOFWP8 -$
TCAS CA=233 RP=1 TI=$SCRAM+LOFWP9 +$
TCAS CA=234 RP=1 TI=$SCRAM+LOFWP10+$
TCAS CA=235 RP=1 TI=$SCRAM+SRVBLDN1-$
TCAS CA=236 RP=1 TI=$SCRAM+SRVBLDN2-$
TCAS CA=237 RP=1 TI=$Hydro Test +$
TCAS CA=238 RP=1 TI=$Hydro Test -$
TCAS CA=239 RP=1 TI=$SCRAM+TG+OPres1 -$
TCAS CA=240 RP=1 TI=$SCRAM+TG+OPres2 -$
TCAS CA=241 RP=1 TI=$SCRAM+TG+OPres3 -$
TCAS CA=242 RP=1 TI=$HotSby_FWcyc +$
TCAS CA=243 RP=1 TI=$HotSby_FWcyc +$
**********************
**** SEISMIC CASES****
```

```
**********************
RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=$OBE INERTIA$
*
** *************************
**** LOAD COMBINATION CASES *
** *************************
CCAS RF=1 CA=104       ME=1 FL=1  C1=103 CY=10  TI=$OBE$
CCAS RF=1 CA=401 SS=1 ME=1 EQ=3  C1=102 C2=103 TI=$EQUATION 9 LEVEL B$
CCAS RF=1 CA=402 SS=1 ME=3 F1=1  C1=103 C2=6 C3=32 TI=$NORMAL+OBE$
CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=6 C3=32 TI=$NORMAL-OBE$
*
******************
**** LOAD SETS****
******************
*RF field is the highest temperature and pressure of the transient
*PR and MO fields are the final temperature and pressure of the transient
LSET RF=1  RP=1 CY=120   PR=1  MO=1  TR=+201 TI=$Design Hydrotest +    LS-1$
LSET RF=2  RP=1 CY=120   PR=2  MO=2  TR=-202 TI=$Design Hydrotest -    LS-2$
LSET RF=3  RP=1 CY=300   PR=3  MO=3  TR=+203 TI=$Startup +             LS-3$
LSET RF=3  RP=1 CY=610   PR=28 MO=28 TR=-204 TI=$TRoll & Inc. PWR1 -   LS-4$
LSET RF=4  RP=1 CY=599   PR=4  MO=4  TR=+205 TI=$TRoll & Inc. PWR2 +   LS-5$
LSET RF=5  RP=1 CY=599   PR=5  MO=5  TR=+206 TI=$TRoll & Inc. PWR3 +   LS-6$
LSET RF=5  RP=1 CY=10000 PR=6  MO=6  TR=-207 TI=$DlyReduction to 75% - LS-7$
LSET RF=5  RP=1 CY=10000 PR=5  MO=5  TR=+208 TI=$DlyReduction to 75% + LS-8$
LSET RF=5  RP=1 CY=2000  PR=7  MO=7  TR=-209 TI=$WklyReduct to 50% -   LS-9$
LSET RF=5  RP=1 CY=2000  PR=5  MO=5  TR=+210 TI=$WklyReduct to 50% +   LS-10$
LSET RF=5  RP=1 CY=310   PR=8  MO=8  TR=-211 TI=$LOFWH+TT 1 -          LS-11$
LSET RF=8  RP=1 CY=10    PR=9  MO=9  TR=-212 TI=$LOFWH+TT 2 -          LS-12$
LSET RF=8  RP=1 CY=10    PR=8  MO=8  TR=+213 TI=$LOFWH+TT 3 +          LS-13$
LSET RF=5  RP=1 CY=10    PR=5  MO=5  TR=+214 TI=$LOFWH+TT 4 +          LS-14$
LSET RF=5  RP=1 CY=70    PR=8  MO=8  TR=-215 TI=$LOFWH+PFWHTR Byp -    LS-15$
LSET RF=5  RP=1 CY=70    PR=5  MO=5  TR=+216 TI=$LOFWH+PFWHTR Byp +    LS-16$
LSET RF=5  RP=1 CY=289   PR=26 MO=26 TR=-217 TI=$SCRAM+TT+AllOtrScm -  LS-17$
LSET RF=26 RP=1 CY=289   PR=29 MO=29 TR=-218 TI=$SCRAM+TT+AllOtrScm -  LS-18$
LSET RF=27 RP=1 CY=300   PR=27 MO=27 TR=+219 TI=$HotStandby 1 +        LS-19$
LSET RF=10 RP=1 CY=300   PR=10 MO=10 TR=+220 TI=$HotStandby 2 +        LS-20$
LSET RF=10 RP=1 CY=300   PR=3  MO=3  TR=-221 TI=$HotStandby 3 -        LS-21$
LSET RF=3  RP=1 CY=300   PR=11 MO=11 TR=-222 TI=$Shutdown 1 -          LS-22$
LSET RF=11 RP=1 CY=300   PR=12 MO=12 TR=-223 TI=$Shutdown 2 -          LS-23$
LSET RF=12 RP=1 CY=300   PR=2  MO=2  TR=-224 TI=$Shutdown 3 -          LS-24$
LSET RF=13 RP=1 CY=10    PR=13 MO=13 TR=+225 TI=$SCRAM+LOFWP1 +        LS-25$
LSET RF=13 RP=1 CY=10    PR=14 MO=14 TR=-226 TI=$SCRAM+LOFWP2 -        LS-26$
LSET RF=15 RP=1 CY=10    PR=15 MO=15 TR=+227 TI=$SCRAM+LOFWP3 +        LS-27$
LSET RF=16 RP=1 CY=10    PR=16 MO=16 TR=+228 TI=$SCRAM+LOFWP4 +        LS-28$
LSET RF=16 RP=1 CY=10    PR=31 MO=31 TR=-229 TI=$SCRAM+LOFWP5 -        LS-29$
LSET RF=17 RP=1 CY=10    PR=17 MO=17 TR=+230 TI=$SCRAM+LOFWP6 +        LS-30$
LSET RF=18 RP=1 CY=10    PR=18 MO=18 TR=+231 TI=$SCRAM+LOFWP7 +        LS-31$
LSET RF=18 RP=1 CY=10    PR=19 MO=19 TR=-232 TI=$SCRAM+LOFWP8 -        LS-32$
LSET RF=20 RP=1 CY=10    PR=20 MO=20 TR=+233 TI=$SCRAM+LOFWP9 +        LS-33$
LSET RF=3  RP=1 CY=10    PR=3  MO=3  TR=+234 TI=$SCRAM+LOFWP10+        LS-34$
LSET RF=5  RP=1 CY=1     PR=21 MO=21 TR=-235 TI=$SCRAM+SRVBLDN1-       LS-35$
LSET RF=21 RP=1 CY=1     PR=2  MO=2  TR=-236 TI=$SCRAM+SRVBLDN2-       LS-36$
LSET RF=22 RP=1 CY=1     PR=22 MO=22 TR=+237 TI=$Hydro Test +          LS-37$
LSET RF=2  RP=1 CY=1     PR=2  MO=2  TR=-238 TI=$Hydro Test -          LS-38$
LSET RF=23 RP=1 CY=289   PR=23 MO=23 TR=-239 TI=$SCRAM+TG+OPres1 -     LS-39$
LSET RF=24 RP=1 CY=289   PR=24 MO=24 TR=-240 TI=$SCRAM+TG+OPres2 -     LS-40$
```

```
LSET RF=25 RP=1 CY=289   PR=25 MO=25 TR=-241 TI=$SCRAM+TG+OPres3 -     LS-41$
LSET RF=30 RP=1 CY=300   PR=30 MO=30 TR=+242 TI=$HotSbyFWcyc +         LS-42$
LSET RF=3  RP=1 CY=300   PR=3  MO=3  TR=+243 TI=$HotSbyFWcyc +         LS-43$
*
LSET RF=6 CY=5 FL=1 PR=6 MO=402 TI=$NORMAL+OBE  LS-132$
LSET RF=6 CY=5 FL=1 PR=6 MO=403 TI=$NORMAL-OBE  LS-133$

***************************
**** RESPONSE SPECTRA****
*************************
*SSE response spectra conservatively used
SPEC FS=OBE EV=1 ME=3 FP=1 TI=$RESPONSE$
      LV=1 DX=1 DY=1 DZ=1
       DI=X
         0.30/0.125  0.80/0.300  2.00/0.650  3.00/0.725  3.50/1.000  4.40/1.200
         5.00/1.900  5.75/2.850  6.00/3.375  8.25/3.375  9.00/3.000 10.00/2.400
        14.00/1.325 19.00/1.600 21.00/1.000 22.00/0.800 30.00/0.700 36.00/0.650
       DI=Y
         0.30/0.075  1.25/0.250  1.75/0.325  2.40/0.450  2.75/0.475  3.80/0.500
         4.40/0.500  4.80/0.600  7.25/0.600  7.50/0.700  8.50/0.700 10.00/0.925
        12.00/1.450 16.00/1.900 18.00/1.700 20.00/0.750 25.00/0.450 30.00/0.350
        36.00/0.325 36.10/0.325 36.20/0.325 36.30/0.325 36.40/0.325 36.50/0.325
       DI=Z
         0.30/0.150  1.00/0.350  2.00/0.625  4.00/1.000  4.50/1.400  5.00/2.000
         5.75/2.950  6.00/3.450  6.25/3.800  8.75/3.800 10.00/2.625  12.0/2.150
        15.00/1.300 17.50/1.450 20.00/0.875 30.00/0.650 36.00/0.650 36.10/0.650
****************************
**** MATERIAL PROPERTIES ****
****************************
* SA-106 Grade B and SA-234 WPB
MATH CD=106 EX=0 TY=1 *C-Si
*MATD TE=-100 EH=30.2 EX=0 SM=20 SY=35
MATD TE=50   EH=29.6 EX=0 SM=20 SY=35
MATD TE=70   EH=29.5 EX=0   SM=20   SY=35
MATD TE=100  EH=29.3 EX=0.2 SM=20.0 SY=35
MATD TE=200  EH=28.8 EX=1.0 SM=20.0 SY=32.1
MATD TE=300  EH=28.3 EX=1.9 SM=20.0 SY=31
MATD TE=400  EH=27.7 EX=2.8 SM=20.0 SY=29.9
MATD TE=500  EH=27.3 EX=3.7 SM=18.9 SY=28.5
MATD TE=600  EH=26.7 EX=4.7 SM=17.3 SY=26.8
*** Cross Sectional Properties
*REGION I- LINE 16 INCH FDW-16 SCH. 120 Run from 5 to 10
*Anchor HD36 to HPCI brnch
CROS CD=1 OD=16.0  WT=1.218 MA=204.28
          SO=1  ST=1  IN=0
*FEEDWATER Valves - V2-27A, V2-28A, V2-29A
CROS CD=2 OD=24.0  WT=2.436 MA=0.12
          SO=1  ST=1  IN=0  KL=1
*REGION III- LINE 16 INCH FDW-16 SCH. 80
*Piping Downstream of Valve V2-29A TO FW TEE
CROS CD=3 OD=16.0  WT=0.843 MA=151.1
          SO=1  ST=1  IN=0
*REGION III- LINE 16 INCH FDW-16 SCH. 120
*Fittings Downstream of Valve V2-29A TO FW TEE
CROS CD=4 OD=16.0  WT=1.218 MA=204.28
          SO=1  ST=1  IN=0
```

```
*REGION IV & V- LINES 10 INCH INCH FDW-21 AND 10 INCH FDW-19 SCH. 120
*Piping Downstream of FW TEE TO NOZZLES
CROS CD=5 OD=10.75 WT=0.843 MA=98.12
          SO=1  ST=1  IN=0
*REGION II- LINE 14 INCH HPCI-15A SCH. 120 FROM NODE 10 TO 547
CROS CD=6 OD=14.0  WT=1.093 MA=161.35
          SO=1  ST=1  IN=1
*REGION II- HPCI Valves
CROS CD=7 OD=21.0  WT=2.186 MA=0.12
          SO=1  ST=1  IN=1  KL=1
*
**********************
* STRUCTURE AND LOADS
*-------------------------------------------------------------------------------
DESN  TE=400.0  PR=1900.0 *FEEDWATER AND HPCI PIPING
*-------------------------------------------------------------------------------
*BEGIN REGION 1
*--------------
*Same for all regions except II
OPER CA=1  TE=100 PR=1100
OPER CA=22 TE=100 PR=1563
OPER CA=28 TE=100 PR=1010
OPER CA=29 TE=100 PR=1010

*Same for all regions
OPER CA=2  TE=100 PR=50
OPER CA=19 TE=50  PR=675
OPER CA=31 TE=50  PR=885

*Unique
OPER CA=3  TE=150 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=265 PR=1010
OPER CA=11 TE=150 PR=170
OPER CA=12 TE=150 PR=88
OPER CA=13 TE=392 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=150 PR=1135
OPER CA=16 TE=150 PR=1135
OPER CA=17 TE=150 PR=1060
OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=265 PR=1010
```

```
OPER CA=30 TE=125 PR=1010

*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=200 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=200 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=9180 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=6885 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=6885 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=1377 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=1377 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=9180 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=9180 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=200 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=200 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=200 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=200 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=200 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=200 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=200 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=10098 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=200 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=200 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
```

```
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138
*
*REGION I GEOMETRY
* RUN 1 FROM ANCHOR HD36 TO HPCI brnCH- FDW-16 LINE A
MATL CD=106
CROS CD=1
COOR PT=5   AX=0 AY=0 AZ=0 *ANCHOR HD36
JUNC PT=5
TANG PT=9   DZ=-2.75  EW=1
TANG PT=10  DZ=-1 *WELDING TEE PER ANSI B16.9
*-------------------------------------------------------------------------------
*END REGION I
*-------------------------------------------------------------------------------
*BEGIN REGION 3
*---------------
*OPER cards same as those for region I
*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=200 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=200 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=9180 IP=1025 FP=1025 TP=1800
```

```
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=6885 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=6885 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=1377 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=1377 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=9180 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=9180 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=200 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=200 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=200 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=200 IP=1150 FP=1150 TP=1380
TRAN CA=228 *IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=200 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=200 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=200 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=10098 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=200 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=200 IP=1025 FP=1025 TP=210

*
PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
```

```
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
*REGION III GEOMETRY
CROS CD=1
*JUNC PT=10
TANG PT=11  DZ=-1  EW=1
TANG PT=15  DZ=-4.17
TANG PT=20  DZ=-0.333  EW=1  *TA=1
CROS CD=2
VALV PT=22  DZ=-1.333 PL=1 MA=2.7  *VALVE V2-27A
VALV PT=25  DZ=-1.333 PL=2  EW=1  *TA=1
CROS CD=1
TANG PT=30  DZ=-2.792
LUMP PT=30  MA=1.285
TANG PT=38  DZ=-4.6
TANG PT=40  DZ=-6.317
TANG PT=45  DZ=-0.625  EW=1  *TA=1
CROS CD=2
VALV PT=47  DZ=-1.792 PL=1 MA=2.7  *VALVE V2-28A
VALV PT=50  DZ=-1.792 PL=2  EW=1  *TA=1
CROS CD=1
*TANG PT=55  DZ=-2.791  EW=1
TANG PT=55  DZ=-.791 EW=1
*BRAD PT=65  RA=2  SD=2  EW=1 Used this to determine midpoint viw .prd output
BEND PT=60 X1=0 Y1=0 Z1=-.828 X2=0 Y2=.586 Z2=-.586
BEND PT=65 X1=0 Y1=.586 Z1=-.586 X2=0 Y2=.828 Z2=0
*TANG PT=67  DY=2.084  EW=1  *TA=1
TANG PT=67  DY=.084
CROS CD=2
VALV PT=70  DY=1.333 PL=1 MA=3.25  *VALVE V2-29A
```

```
VALV PT=75  DY=1.333 PL=2  EW=1  *TA=1
CROS CD=3
TANG PT=78  DY=1.25
TANG PT=80  DY=3.5
TANG PT=82  DY=2.667  EW=1
CROS CD=4
BRAD PT=85  RA=2  EW=1
CROS CD=3
TANG PT=90  DX=2.875
TANG PT=95  DX=2.875  EW=1
CROS CD=4
BRAD PT=100 RA=2  EW=1
CROS CD=3
TANG PT=105 DX=1.12  DZ=-1.12
TANG PT=110 DX=3.477 DZ=-3.477  EW=1
CROS CD=4
TANG PT=115 DX=0.7071 DZ=-0.7071 EW=1
*
*-----------------------------------------------------------------------------
*END REGION III
*-----------------------------------------------------------------------------
*BEGIN REGION IV
*---------------
*OPER cards same as those for regions I and III
*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 *IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1075 FP=1150 TP=0
```

```
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=100 IP=1025 FP=1025 TP=210

*
PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
```

```
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
*REGION IV GEOMETRY DOWNSTREAM OF FW brnCH TEE/REDUCER - 10 INCH PIPING
*RUN FROM FW TEE TO ELBOW BEFORE NOZZLE NBA, NODE 275
CROS CD=4
TANG PT=170  DX=0.7071  DZ=-0.7071  EW=1
ERED PT=175  DX=0.825 DZ=-0.825  AN=30
CROS CD=5
*RUN FROM FW TEE TO ELBOW BEFORE NOZZLE N4B, NODE 152
BEND PT=190  X1=4.813  Y1=0  Z1=-4.813  X2=1.283  Y2=0  Z2=-6.685
BEND PT=200  X1=0.449  Y1=0  Z1=-2.342  X2=-0.059  Y2=0  Z2=-2.384
STRU PT=201 DX=.198 DZ=.9802
STRU PT=202 DX=.198 DZ=.9802
ANCH PT=202
JUNC PT=200
BEND PT=220  X1=-0.2196  Y1=0  Z1=-8.859  X2=-6.266  Y2=0  Z2=-6.266
TANG PT=225  DX=-0.3388  DZ=-0.3388
TANG PT=230  DX=-0.3388  DZ=-0.3388
TANG PT=235  DX=-1.002  DZ=-1.002
BRAD PT=240  RA=1.25
TANG PT=245  DX=-2.693  DY=3.196  DZ=2.693
TANG PT=250  DX=-2.693  DY=3.196  DZ=2.693
TANG PT=255  DX=-2.693  DY=3.196  DZ=2.693
BRAD PT=260  RA=1.25
TANG PT=265  DY=3.958
TANG PT=270  DY=3.959
*-----------------------------------------------------------------------------
*END REGION IV
*-----------------------------------------------------------------------------
*BEGIN REGION IVa
*-----------------------------------------------------------------------------
OPER CA=3  TE=283 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=360 PR=1010
OPER CA=11 TE=225 PR=170
OPER CA=12 TE=210 PR=88
OPER CA=13 TE=450 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=247 PR=1135
OPER CA=16 TE=288 PR=1135
OPER CA=17 TE=247 PR=1060
OPER CA=18 TE=283 PR=1135

OPER CA=20 TE=200 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
```

```
OPER CA=27 TE=323 PR=1010
OPER CA=30 TE=180 PR=1010
*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=283 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=283 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=323 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=323 FT=360 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 IS=1 FS=1 IT=360 FT=283 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=283 FT=225 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=225 FT=210 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=210 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=450 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=450 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=247 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=247 FT=288 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=288 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=247 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=247 FT=283 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=283 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=200 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=200 FT=283 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=180 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=180 FT=283 TT=210 FL=100 IP=1025 FP=1025 TP=210


PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=204 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
```

```
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.2 DI=0.455 EX=6.4 * Tavg=294
PAIR CA=220 CO=27.0 DI=0.442 EX=6.4 * Tavg=342
PAIR CA=221 CO=27.1 DI=0.447 EX=6.4 * Tavg=322
PAIR CA=222 CO=27.4 DI=0.466 EX=6.4 * Tavg=254
PAIR CA=223 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=224 CO=27.6 DI=0.495 EX=6.4 * Tavg=155
PAIR CA=225 CO=26.5 DI=0.422 EX=6.4 * Tavg=421
PAIR CA=226 CO=27.4 DI=0.467 EX=6.4 * Tavg=250
PAIR CA=227 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=228 CO=27.3 DI=0.462 EX=6.4 * Tavg=268
PAIR CA=229 CO=27.6 DI=0.492 EX=6.4 * Tavg=169
PAIR CA=230 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=231 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=232 CO=27.6 DI=0.493 EX=6.4 * Tavg=167
PAIR CA=233 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=234 CO=27.4 DI=0.470 EX=6.4 * Tavg=242
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.499 EX=6.4 * Tavg=140
PAIR CA=243 CO=27.5 DI=0.474 EX=6.4 * Tavg=232


TANG PT=275  DY=6.583  EW=0
*
*-----------------------------------------------------------------------------
*END REGION IVa
*-----------------------------------------------------------------------------
*BEGIN REGION IVb
*-----------------------------------------------------------------------------
OPER CA=3  TE=416 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=454 PR=1010
OPER CA=11 TE=300 PR=170
OPER CA=12 TE=270 PR=88
OPER CA=13 TE=507 PR=1190
OPER CA=14 TE=50  PR=1135
```

```
OPER CA=15 TE=343 PR=1135
OPER CA=16 TE=427 PR=1135
OPER CA=17 TE=343 PR=1060
OPER CA=18 TE=416 PR=1135

OPER CA=20 TE=250 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=382 PR=1010
OPER CA=30 TE=235 PR=1010
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=416 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=416 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=382 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=382 FT=454 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 IS=1 FS=1 IT=454 FT=416 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=416 FT=300 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=300 FT=270 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=270 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=507 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=507 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=343 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=343 FT=427 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=427 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=343 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=343 FT=416 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=416 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=250 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=250 FT=416 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=235 TT=60 FL=100 IP=1025 FP=1025 TP=60
```

```
TRAN CA=243 IS=1 FS=1 IT=235 FT=416 TT=210 FL=100 IP=1025 FP=1025 TP=210


PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=204 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.1 DI=0.447 EX=6.4 * Tavg=324
PAIR CA=220 CO=26.6 DI=0.423 EX=6.4 * Tavg=418
PAIR CA=221 CO=26.4 DI=0.418 EX=6.4 * Tavg=435
PAIR CA=222 CO=27.0 DI=0.438 EX=6.4 * Tavg=358
PAIR CA=223 CO=27.3 DI=0.457 EX=6.4 * Tavg=285
PAIR CA=224 CO=27.6 DI=0.489 EX=6.4 * Tavg=185
PAIR CA=225 CO=26.3 DI=0.413 EX=6.4 * Tavg=450
PAIR CA=226 CO=27.3 DI=0.459 EX=6.4 * Tavg=279
PAIR CA=227 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=228 CO=26.8 DI=0.432 EX=6.4 * Tavg=385
PAIR CA=229 CO=27.4 DI=0.471 EX=6.4 * Tavg=239
PAIR CA=230 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=231 CO=26.8 DI=0.433 EX=6.4 * Tavg=380
PAIR CA=232 CO=27.5 DI=0.473 EX=6.4 * Tavg=233
PAIR CA=233 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=234 CO=27.1 DI=0.444 EX=6.4 * Tavg=333
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.492 EX=6.4 * Tavg=168
PAIR CA=243 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
*
TANG PT=280  DY=6.583
*-----------------------------------------------------------------------------
*END REGION IVb
*-----------------------------------------------------------------------------
*BEGIN REGION V TO NOZZLE N4A, NODE 290
*-----------------------------------------------------------------------------

OPER CA=3  TE=549 PR=1010
OPER CA=4  TE=260 PR=1010
```

```
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=549 PR=1010
OPER CA=11 TE=375 PR=170
OPER CA=12 TE=330 PR=88
OPER CA=13 TE=565 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=440 PR=1135
OPER CA=16 TE=565 PR=1135
OPER CA=17 TE=440 PR=1060
OPER CA=18 TE=549 PR=1135

OPER CA=20 TE=300 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=440 PR=1010
OPER CA=30 TE=290 PR=1010
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=549 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=440 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=440 FT=549 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 *IS=1 FS=1 IT=549 FT=549 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=222 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=375 FT=330 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=330 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=565 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=440 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=440 FT=565 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=440 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=440 FT=549 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=549 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
```

```
TRAN CA=233 IS=1 FS=1 IT=50 FT=300 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=290 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=290 FT=549 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=204 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=220 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=221 CO=25.5 DI=0.387 EX=6.4 * Tavg=549
PAIR CA=222 CO=26.2 DI=0.409 EX=6.4 * Tavg=462
PAIR CA=223 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=224 CO=27.5 DI=0.480 EX=6.4 * Tavg=215
PAIR CA=225 CO=26.1 DI=0.404 EX=6.4 * Tavg=479
PAIR CA=226 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=227 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=228 CO=25.9 DI=0.397 EX=6.4 * Tavg=503
PAIR CA=229 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=230 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=231 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=232 CO=27.2 DI=0.453 EX=6.4 * Tavg=300
PAIR CA=233 CO=27.6 DI=0.491 EX=6.4 * Tavg=175
PAIR CA=234 CO=26.5 DI=0.421 EX=6.4 * Tavg=425
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.487 EX=6.4 * Tavg=195
PAIR CA=243 CO=26.5 DI=0.422 EX=6.4 * Tavg=420
```

```
BRAD PT=285  RA=1.25
TANG PT=290  DX=-4.007  DZ=4.007  EW=1
NOZZ PT=290   *NOZZLE N4A
AMVT  CA=1 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT  CA=2 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT  CA=3 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=4 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=5 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=6 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=7 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=8 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT  CA=9 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=10 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=11 PT=290 DX=0.1993 DY=1.0867 DZ=-0.1993
AMVT CA=12 PT=290 DX=0.1699 DY=0.9264 DZ=-0.1699
AMVT CA=13 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=14 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=15 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=16 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=17 PT=290 DX=0.3169 DY=1.7281 DZ=-0.3169
AMVT CA=18 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=19 PT=290 DX=0.2823 DY=1.5392 DZ=-0.2823
AMVT CA=20 PT=290 DX=0.2823 DY=1.5392 DZ=-0.2823
AMVT CA=21 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=22 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT CA=23 PT=290 DX=0.3463 DY=1.8884 DZ=-0.3463
AMVT CA=24 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=25 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=26 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=27 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=28 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=29 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=30 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=31 PT=290 DX=0.3019 DY=1.6461 DZ=-0.3019

AMVT CA=32 PT=290 DX=-.09   DY=.015   DZ=-.093
*-------------------------------------------------------------------------------
*END REGION V
*--------------
*BEGIN REGION IV
*--------------
OPER CA=3  TE=150 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=265 PR=1010
OPER CA=11 TE=150 PR=170
OPER CA=12 TE=150 PR=88
OPER CA=13 TE=392 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=150 PR=1135
OPER CA=16 TE=150 PR=1135
OPER CA=17 TE=150 PR=1060
```

```
OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=265 PR=1010

OPER CA=30 TE=125 PR=1010
*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=100 IP=1025 FP=1025 TP=210
```

```
*
PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
*REGION IV GEOMETRY DOWNSTREAM OF FW brnCH TEE/REDUCER - 10 INCH PIPING
*RUN FROM FW TEE TO ELBOW BEFORE NOZZLE N4A, NODE 155
JUNC PT=115
CROS CD=5
BRAN PT=120  DX=-0.5022  DY=0.596   DZ=-0.5022  TE=1 EW=1
TANG PT=125  DX=-2.594  DY=3.078  DZ=-2.594
TANG PT=130  DX=-2.594  DY=3.078  DZ=-2.594
TANG PT=135  DX=-2.594  DY=3.078  DZ=-2.594  EW=0
```

```
BRAD PT=140  RA=1.25  EW=0
TANG PT=142  DY=4
TANG PT=145  DY=4
TANG PT=150  DY=2.53
*-----------------------------------------------------------------------------
*END REGION IV
*-----------------------------------------------------------------------------
*BEGIN REGION IVa
*-----------------------------------------------------------------------------
OPER CA=3  TE=283 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=360 PR=1010
OPER CA=11 TE=225 PR=170
OPER CA=12 TE=210 PR=88
OPER CA=13 TE=450 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=247 PR=1135
OPER CA=16 TE=288 PR=1135
OPER CA=17 TE=247 PR=1060
OPER CA=18 TE=283 PR=1135

OPER CA=20 TE=200 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=323 PR=1010
OPER CA=30 TE=180 PR=1010
*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=283 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=283 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=323 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=323 FT=360 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
```

```
TRAN CA=221 IS=1 FS=1 IT=360 FT=283 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=283 FT=225 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=225 FT=210 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=210 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=450 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=450 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=247 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=247 FT=288 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=288 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=247 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=247 FT=283 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=283 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=200 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=200 FT=283 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=180 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=180 FT=283 TT=210 FL=100 IP=1025 FP=1025 TP=210


PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=204 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.2 DI=0.455 EX=6.4 * Tavg=294
PAIR CA=220 CO=27.0 DI=0.442 EX=6.4 * Tavg=342
PAIR CA=221 CO=27.1 DI=0.447 EX=6.4 * Tavg=322
PAIR CA=222 CO=27.4 DI=0.466 EX=6.4 * Tavg=254
PAIR CA=223 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=224 CO=27.6 DI=0.495 EX=6.4 * Tavg=155
PAIR CA=225 CO=26.5 DI=0.422 EX=6.4 * Tavg=421
PAIR CA=226 CO=27.4 DI=0.467 EX=6.4 * Tavg=250
PAIR CA=227 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=228 CO=27.3 DI=0.462 EX=6.4 * Tavg=268
PAIR CA=229 CO=27.6 DI=0.492 EX=6.4 * Tavg=169
PAIR CA=230 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=231 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
```

```
PAIR CA=232 CO=27.6 DI=0.493 EX=6.4 * Tavg=167
PAIR CA=233 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=234 CO=27.4 DI=0.470 EX=6.4 * Tavg=242
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.499 EX=6.4 * Tavg=140
PAIR CA=243 CO=27.5 DI=0.474 EX=6.4 * Tavg=232

*
TANG PT=152  DY=6.53  EW=0
*
*-----------------------------------------------------------------------------
*END REGION IVa
*-----------------------------------------------------------------------------
*BEGIN REGION IVb
*-----------------------------------------------------------------------------
OPER CA=3  TE=416 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=454 PR=1010
OPER CA=11 TE=300 PR=170
OPER CA=12 TE=270 PR=88
OPER CA=13 TE=507 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=343 PR=1135
OPER CA=16 TE=427 PR=1135
OPER CA=17 TE=343 PR=1060
OPER CA=18 TE=416 PR=1135

OPER CA=20 TE=250 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=382 PR=1010
OPER CA=30 TE=235 PR=1010
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=416 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=416 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
```

```
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=382 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=382 FT=454 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 IS=1 FS=1 IT=454 FT=416 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=416 FT=300 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=300 FT=270 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=270 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=507 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=507 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=343 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=343 FT=427 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=427 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=343 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=343 FT=416 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=416 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=250 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=250 FT=416 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=235 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=235 FT=416 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=204 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.1 DI=0.447 EX=6.4 * Tavg=324
PAIR CA=220 CO=26.6 DI=0.423 EX=6.4 * Tavg=418
PAIR CA=221 CO=26.4 DI=0.418 EX=6.4 * Tavg=435
```

```
PAIR CA=222 CO=27.0 DI=0.438 EX=6.4 * Tavg=358
PAIR CA=223 CO=27.3 DI=0.457 EX=6.4 * Tavg=285
PAIR CA=224 CO=27.6 DI=0.489 EX=6.4 * Tavg=185
PAIR CA=225 CO=26.3 DI=0.413 EX=6.4 * Tavg=450
PAIR CA=226 CO=27.3 DI=0.459 EX=6.4 * Tavg=279
PAIR CA=227 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=228 CO=26.8 DI=0.432 EX=6.4 * Tavg=385
PAIR CA=229 CO=27.4 DI=0.471 EX=6.4 * Tavg=239
PAIR CA=230 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=231 CO=26.8 DI=0.433 EX=6.4 * Tavg=380
PAIR CA=232 CO=27.5 DI=0.473 EX=6.4 * Tavg=233
PAIR CA=233 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=234 CO=27.1 DI=0.444 EX=6.4 * Tavg=333
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.492 EX=6.4 * Tavg=168
PAIR CA=243 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
*
TANG PT=155  DY=6.523
*--------------
*END REGION IVb
*-----------------------------------------------
*BEGIN REGION V GEOMETRY TO NOZZLE N4B, NODE 165
*-----------------------------------------------
OPER CA=3  TE=549 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=549 PR=1010
OPER CA=11 TE=375 PR=170
OPER CA=12 TE=330 PR=88
OPER CA=13 TE=565 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=440 PR=1135
OPER CA=16 TE=565 PR=1135
OPER CA=17 TE=440 PR=1060
OPER CA=18 TE=549 PR=1135

OPER CA=20 TE=300 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=440 PR=1010
OPER CA=30 TE=290 PR=1010
```

```
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=549 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=440 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=440 FT=549 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 *IS=1 FS=1 IT=549 FT=549 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=222 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=375 FT=330 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=330 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=565 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=440 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=440 FT=565 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=440 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=440 FT=549 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=549 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=300 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=290 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=290 FT=549 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=204 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
```

```
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=220 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=221 CO=25.5 DI=0.387 EX=6.4 * Tavg=549
PAIR CA=222 CO=26.2 DI=0.409 EX=6.4 * Tavg=462
PAIR CA=223 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=224 CO=27.5 DI=0.480 EX=6.4 * Tavg=215
PAIR CA=225 CO=26.1 DI=0.404 EX=6.4 * Tavg=479
PAIR CA=226 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=227 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=228 CO=25.9 DI=0.397 EX=6.4 * Tavg=503
PAIR CA=229 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=230 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=231 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=232 CO=27.2 DI=0.453 EX=6.4 * Tavg=300
PAIR CA=233 CO=27.6 DI=0.491 EX=6.4 * Tavg=175
PAIR CA=234 CO=26.5 DI=0.421 EX=6.4 * Tavg=425
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.487 EX=6.4 * Tavg=195
PAIR CA=243 CO=26.5 DI=0.422 EX=6.4 * Tavg=420
BRAD PT=160  RA=1.25
TANG PT=165  DX=-4.007  DZ=-4.007  EW=1
NOZZ PT=165   *NOZZLE N4B
AMVT CA=1  PT=165 DX=0.0196 DY=0.1069 DZ=0.0196
AMVT CA=2  PT=165 DX=0.0196 DY=0.1069 DZ=0.0196
AMVT CA=3  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=4  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=5  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=6  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=7  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=8  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=9  PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=10 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=11 PT=165 DX=0.1993 DY=1.0867 DZ=0.1993
AMVT CA=12 PT=165 DX=0.1699 DY=0.9264 DZ=0.1699
AMVT CA=13 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=14 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=15 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=16 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=17 PT=165 DX=0.3169 DY=1.7281 DZ=0.3169
AMVT CA=18 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=19 PT=165 DX=0.2823 DY=1.5392 DZ=0.2823
AMVT CA=20 PT=165 DX=0.2823 DY=1.5392 DZ=0.2823
AMVT CA=21 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=22 PT=165 DX=0.0196 DY=0.1069 DZ=0.0196
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AMVT CA=23 PT=165 DX=0.3463 DY=1.8884 DZ=0.3463
AMVT CA=24 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=25 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=26 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=27 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=28 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=29 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=30 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=31 PT=165 DX=0.3019 DY=1.6461 DZ=0.3019
AMVT CA=32 PT=165 DX=-.09   DY=.015   DZ=-.093
*-----------------------------------------------------------------------
*END REGION V
*-----------------------------------------------------------------------
*REGION II GEOMETRY - HPCI Line brnch
CROS CD=6
JUNC PT=10
*-----------------------------------------------------------------------
*BEGIN REGION IIa
*-----------------------------------------------------------------------
OPER CA=3  TE=150 PR=1010
OPER CA=4  TE=260 PR=1010
OPER CA=5  TE=392 PR=1010
OPER CA=6  TE=310 PR=1010
OPER CA=7  TE=280 PR=1010
OPER CA=8  TE=265 PR=1010
OPER CA=9  TE=90  PR=1010
OPER CA=10 TE=265 PR=1010
OPER CA=11 TE=150 PR=170
OPER CA=12 TE=150 PR=88
OPER CA=13 TE=392 PR=1190
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=150 PR=1135
OPER CA=16 TE=150 PR=1135
OPER CA=17 TE=150 PR=1060
OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=265 PR=1010

OPER CA=30 TE=125 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=150 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
```

```
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=150 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=150 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=150 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=150 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=150 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=150 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=150 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=150 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=150 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=150 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=150 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=150 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=150 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=150 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=150 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=150 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=150 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=150 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
```

```
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
BRAN PT=301  DY=1  TE=1 EW=1
TANG PT=302  DY=2.333
TANG PT=305  DY=2.333  EW=1
BRAD PT=310  RA=1.75  EW=1
TANG PT=315  DX=-2.333  EW=1
CROS CD=7
VALV PT=317  DX=-1.167 PL=1 MA=2.05
*-----------------------------------------------------------------------------
*BEGIN REGION IIb
*-----------------------------------------------------------------------------

OPER CA=3  TE=125   PR=1010
OPER CA=4  TE=180   PR=1010
OPER CA=5  TE=246   PR=1010
OPER CA=6  TE=205   PR=1010
OPER CA=7  TE=190   PR=1010
OPER CA=8  TE=182.5 PR=1010
OPER CA=9  TE=95    PR=1010
OPER CA=10 TE=182.5 PR=1010
OPER CA=11 TE=125   PR=170
OPER CA=12 TE=125   PR=88
OPER CA=13 TE=246   PR=1190
OPER CA=14 TE=50    PR=1135
OPER CA=15 TE=125   PR=1135
OPER CA=16 TE=125   PR=1135
OPER CA=17 TE=125   PR=1060
OPER CA=18 TE=125   PR=1135

OPER CA=20 TE=125   PR=675
OPER CA=21 TE=187.5 PR=885

OPER CA=23 TE=246   PR=1375
```

```
OPER CA=24 TE=246    PR=940
OPER CA=25 TE=246    PR=1010
OPER CA=26 TE=187.5 PR=1010
OPER CA=27 TE=182.5 PR=1010

OPER CA=30 TE=112.5 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=125 TT=16164 FL=150 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=125 FT=100 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=180 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=180 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=246 FT=205 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=205 FT=246 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=246 FT=190 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=190 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=246 FT=182.5 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=182.5 FT=95 TT=360 FL=150 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=95 FT=182.5 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=182.5 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=246 FT=182.5 TT=90 FL=150 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=182.5 FT=246 TT=180 FL=150 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=246 FT=187.5 TT=60 FL=150 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=187.5 FT=100 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=182.5 FT=182.5 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=182.5 FT=182.5 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=182.5 FT=125 TT=4140 FL=150 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=125 FT=100 TT=8280 FL=150 IP=103 FP=65 TP=8280
TRAN CA=225 *IS=1 FS=1 IT=246 FT=246 TT=12 FL=150 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=246 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=125 TT=1380 FL=150 IP=1150 FP=1150 TP=1380
TRAN CA=228 *IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=125 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=125 TT=3060 FL=150 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=125 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=125 TT=300 FL=150 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=125 FT=125 TT=8964 FL=150 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=246 FT=187.5 TT=60 FL=150 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=187.5 FT=100 TT=900 FL=150 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=246 FT=246 TT=60 FL=150 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=246 FT=246 TT=900 FL=150 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=246 FT=246 TT=900 FL=150 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=112.5 TT=60 FL=150 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=112.5 FT=125 TT=210 FL=150 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=204 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=205 CO=27.6 DI=0.499 EX=6.4 * Tavg=140
```

```
PAIR CA=206 CO=27.5 DI=0.481 EX=6.4 * Tavg=213
PAIR CA=207 CO=27.5 DI=0.476 EX=6.4 * Tavg=226
PAIR CA=208 CO=27.5 DI=0.476 EX=6.4 * Tavg=226
PAIR CA=209 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=210 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=211 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=212 CO=27.6 DI=0.500 EX=6.4 * Tavg=139
PAIR CA=213 CO=27.6 DI=0.500 EX=6.4 * Tavg=139
PAIR CA=214 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=215 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=216 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=217 CO=27.5 DI=0.480 EX=6.4 * Tavg=217
PAIR CA=218 CO=27.6 DI=0.498 EX=6.4 * Tavg=144
PAIR CA=219 CO=27.6 DI=0.489 EX=6.4 * Tavg=183
PAIR CA=220 CO=27.6 DI=0.489 EX=6.4 * Tavg=183
PAIR CA=221 CO=27.6 DI=0.495 EX=6.4 * Tavg=154
PAIR CA=222 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=223 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=224 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=225 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=226 CO=27.6 DI=0.497 EX=6.4 * Tavg=148
PAIR CA=227 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=228 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=229 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=230 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=231 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=232 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=233 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=234 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=235 CO=27.5 DI=0.480 EX=6.4 * Tavg=217
PAIR CA=236 CO=27.6 DI=0.498 EX=6.4 * Tavg=144
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=240 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=241 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=242 CO=27.6 DI=0.510 EX=6.4 * Tavg=106
PAIR CA=243 CO=27.6 DI=0.506 EX=6.4 * Tavg=119
VALV PT=320  DX=-1.167 PL=2  EW=1
CROS CD=6
TANG PT=325  DX=-0.666
TANG PT=330  DX=-2.667  EW=1
BRAD PT=335  RA=1.75  EW=1
TANP         DZ=-3.5
BRAD PT=340  RA=1.75  EW=1
TANG PT=345  DX=3.333  EW=1
CROS CD=7
VALV PT=346  DX=1.167 PL=1 MA=1.725
*-----------------------------------------------------------------------------
*END REGION IIb
*-----------------------------------------------------------------------------
*BEGIN REGION II
*-----------------------------------------------------------------
OPER CA=1  TE=100 PR=50

OPER CA=3  TE=100 PR=50
```

```
OPER CA=4  TE=100 PR=50
OPER CA=5  TE=100 PR=50
OPER CA=6  TE=100 PR=50
OPER CA=7  TE=100 PR=50
OPER CA=8  TE=100 PR=50
OPER CA=9  TE=100 PR=50
OPER CA=10 TE=100 PR=50
OPER CA=11 TE=100 PR=50
OPER CA=12 TE=100 PR=50
OPER CA=13 TE=100 PR=50
OPER CA=14 TE=50  PR=1135
OPER CA=15 TE=100 PR=50
OPER CA=16 TE=100 PR=50
OPER CA=17 TE=100 PR=50
OPER CA=18 TE=100 PR=50

OPER CA=20 TE=100 PR=50
OPER CA=21 TE=100 PR=50
OPER CA=22 TE=100 PR=50
OPER CA=23 TE=100 PR=50
OPER CA=24 TE=100 PR=50
OPER CA=25 TE=100 PR=50
OPER CA=26 TE=100 PR=50
OPER CA=27 TE=100 PR=50
OPER CA=28 TE=100 PR=50
OPER CA=29 TE=100 PR=50
OPER CA=30 TE=100 PR=50

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=65 TP=1800
TRAN CA=202 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=203 *IS=1 FS=1 IT=100 FT=100 TT=16164 FL=150 IP=65 FP=65 TP=16164
TRAN CA=204 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=205 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=206 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=207 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=208 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=209 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=210 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=211 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=212 *IS=1 FS=1 IT=100 FT=100 TT=360 FL=150 IP=65 FP=65 TP=360
TRAN CA=213 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=214 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=215 *IS=1 FS=1 IT=100 FT=100 TT=90 FL=150 IP=65 FP=65 TP=90
TRAN CA=216 *IS=1 FS=1 IT=100 FT=100 TT=180 FL=150 IP=65 FP=65 TP=180
TRAN CA=217 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=218 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=219 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=220 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=221 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=222 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=223 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=224 *IS=1 FS=1 IT=100 FT=100 TT=8280 FL=150 IP=65 FP=65 TP=8280
TRAN CA=225 *IS=1 FS=1 IT=100 FT=100 TT=12 FL=150 IP=65 FP=65 TP=12
TRAN CA=226 IS=1 FS=1 IT=100 FT=50 TT=0 FL=3672 IP=65 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=100 TT=1380 FL=150 IP=1150 FP=65 TP=1380
TRAN CA=228 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
```

```
TRAN CA=229 IS=1 FS=1 IT=100 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=100 TT=3060 FL=150 IP=900 FP=65 TP=3060
TRAN CA=231 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=232 IS=1 FS=1 IT=100 FT=50 TT=0 FL=1560.6 IP=65 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=100 TT=300 FL=150 IP=690 FP=65 TP=300
TRAN CA=234 *IS=1 FS=1 IT=100 FT=100 TT=8964 FL=150 IP=65 FP=65 TP=8964
TRAN CA=235 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=236 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=237 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=238 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=239 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=240 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=241 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=242 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=243 *IS=1 FS=1 IT=100 FT=100 TT=210 FL=150 IP=65 FP=65 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=204 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=205 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=206 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=207 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=208 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=209 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=210 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=211 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=212 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=213 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=214 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=215 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=216 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=217 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=218 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=219 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=220 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=221 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=222 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=223 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=224 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=225 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=226 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=227 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=228 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=229 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=230 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=231 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=232 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=233 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=234 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=235 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=236 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=240 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
```

```
PAIR CA=241 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=242 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=243 CO=27.6 DI=0.512 EX=6.4 * Tavg=100

VALV PT=350  DX=1.167 PL=2  EW=1
CROS CD=6
TANG PT=355  DX=0.167
TANG PT=360  DX=2.083  EW=1
BRAD PT=380  RA=1.75  EW=1
TANP         DY=-2.479
BRAD PT=390  RA=1.75  EW=1
TANG PT=392  DX=2.585 DY=-2.585
TANG PT=395  DX=2.585 DY=-2.585  EW=1
BRAD PT=400  RA=1.75  EW=1
TANG PT=405  DZ=-3.417
BRAD PT=410  RA=1.17  EW=1
TANG PT=415  DY=-3
TANG PT=420  DY=-5.25
TANG PT=425  DY=-2.417
BRAD PT=430  RA=1.75  EW=1
TANG PT=435  DZ=2.333
TANG PT=440  DZ=4.757
TANG PT=445  DZ=4.757
TANG PT=450  DZ=4.757
BRAD PT=455  RA=1.75  EW=1
TANG PT=460  DX=-1.989  DZ=1.989
BRAD PT=465  RA=1.75  EW=1
TANG PT=470  DY=-5.722
TANG PT=480  DY=-5.722
TANG PT=485  DY=-5.722
BRAD PT=490  RA=1.17  EW=1
TANG PT=495  DZ=1.667
TANG PT=500  DZ=2.0833
BRAD PT=505  RA=1.75  EW=1
TANG PT=510  DX=3.682
TANG PT=515  DX=3.682
TANG PT=520  DX=3.682
TANG PT=525  DX=3.682
BRAD PT=535  RA=1.75
TANG PT=540  DX=2.556  DZ=-2.556
TANG PT=545  DX=2.555  DZ=-2.555
STRU PT=546  DX=-.7071 DZ=-.7071
STRU PT=547  DX=-.7071 DZ=-.7071
ANCH PT=547
*-----------------------------------------------------------------------------
*END REGION II
*-----------------------------------------------------------------------------
*
*** VALVE OPERATOR ***
CROS CD=7
JUNC PT=346
VALV PT=348 DY=5.567 PL=3 MA=2.52
*** SUPPORTS AND ANCHORS ***
CSUP PT=105 DY=1  KP=5000  PI=0 *FW-9
CSUP PT=190 DY=1  KP=1000  PI=0 *FW-6
CSUP PT=220 DY=-1 KP=1000  PI=0 *FW-4
```

```
CSUP PT=270 DY=1  KP=1000  PI=0 *FW-2
CSUP PT=145 DY=1  KP=1000  PI=0 *FW-7
*
RSTN PT=230 DX=-0.6123  DY=-0.5  DZ=0.6123  SP=370  *FW-3
*
RSTN PT=80   DX=1.0                          SP=200  *FDW-H10

RSTN PT=201  DX=0.198 DZ=0.9802    SP=1000  *FDW-H23
RSTN PT=546  DX=-0.7071  DZ=-0.7071            SP=1000 *BELLOWS
RSTN PT=355   DY=1.0                                   *HPCI-H31
RSTN PT=415   DX=1.0  DZ=1.0                           *HPCI-H32
RSTN PT=30    DX=1.0  DY=1.0                           *FDW-HD37 FLUED HEAD
ROTR PT=30    RZ=1                                     *FDW_HD37 FLUED HEAD
RSTN PT=60  DX=1.0 DY=1.0                      SP=200  *FDW-H24
RSTN PT=5 DX=1 DY=1 DZ=1
ROTR PT=5 RZ=1
ENDP
```

APPENDIX B

PIPESTRESS OUTPUT FILE ("FWHPCI.PRF")

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [1

Vermont Yankee Feedwater PipingSI Fatigue Analysis

DELTA T1 IN DEGREES
PRESSURES IN PSI
STRESSES IN PSI

SUMMARY OF LOAD SETS AT POINT 155 LR ELBOW 155 TO 160 GLOBAL MOMENTS IN FT-LB

LOAD SET NO.	LOAD SET DESCRIPTION		CYCLES	DYNAM. CYCLES	PRESSURE	MOMENT X	MOMENT Y	MOMENT Z	TRANSIENT STRESSES EQ. 10	EQ. 11	EQ. 13	DELTA T1
1	Design Hydrotest +	LS-1	120		1100.	55.	22.	54.	0.	50.	0.	0
2	Design Hydrotest -	LS-2	120		50.	56.	21.	54.	0.	0.	0.	0
3	Startup +	LS-3	300		1010.	14505.	-695.	-12403.	25110.	25114.	12555.	0
4	TRoll & Inc. PWR1 -	LS-4	610		1010.	24012.	-983.	-21319.	-25109.	-54064.	-12555.	-169
5	TRoll & Inc. PWR2 +	LS-5	599		1010.	13760.	-442.	-11409.	0.	15049.	0.	78
6	TRoll & Inc. PWR3 +	LS-6	599		1010.	4661.	45.	-2630.	0.	252.	0.	1
7	DlyReduction to 75% -	LS-7	10000		1010.	10313.	-259.	-8082.	0.	-317.	0.	-1
8	DlyReduction to 75% +	LS-8	10000		1010.	4661.	45.	-2630.	0.	317.	0.	1
9	WklyReduct to 50% -	LS-9	2000		1010.	12381.	-369.	-10078.	0.	-215.	0.	-1
10	WklyReduct to 50% +	LS-10	2000		1010.	4661.	45.	-2630.	0.	215.	0.	1
11	LOFWH+TT 1 -	LS-11	310		1010.	13415.	-424.	-11077.	0.	-243.	0.	-1
12	LOFWH+TT 2 -	LS-12	10		1010.	24520.	-1009.	-21810.	0.	-1480.	0.	-7
13	LOFWH+TT 3 +	LS-13	10		1010.	13415.	-424.	-11077.	0.	619.	0.	3
14	LOFWH+TT 4 +	LS-14	10		1010.	4661.	45.	-2630.	0.	243.	0.	1
15	LOFWH+PFWHTR Byp -	LS-15	70		1010.	13415.	-424.	-11077.	0.	-4784.	0.	-25
16	LOFWH+PFWHTR Byp +	LS-16	70		1010.	4661.	45.	-2630.	0.	2433.	0.	13
17	SCRAM+TT+AllOtrScm -	LS-17	289		1010.	12192.	-365.	-9935.	0.	-6434.	0.	-34
18	SCRAM+TT+AllOtrScm -	LS-18	289		1010.	23480.	-960.	-20844.	0.	-593.	0.	-3
19	HotStandby 1 +	LS-19	300		1010.	10450.	-359.	-8332.	10951.	10969.	5476.	39
20	HotStandby 2 +	LS-20	300		1010.	8500.	-316.	-6524.	17948.	17967.	8974.	0
21	HotStandby 3 -	LS-21	300		1010.	14505.	-695.	-12403.	0.	0.	0.	0
22	Shutdown 1 -	LS-22	300		170.	7904.	-355.	-6594.	-25111.	-25113.	-12555.	-0
23	Shutdown 2 -	LS-23	300		88.	6239.	-269.	-5132.	-14160.	-14241.	-7080.	-1
24	Shutdown 3 -	LS-24	300		50.	56.	21.	54.	-11328.	-11330.	-5664.	-0
25	SCRAM+LOFWP1 +	LS-25	10		1190.	2347.	86.	-451.	10950.	10965.	5475.	42
26	SCRAM+LOFWP2 -	LS-26	10		1135.	26504.	-1110.	-23646.	-10950.	-57147.	-5475.	-251
27	SCRAM+LOFWP3 +	LS-27	10		1135.	17301.	-771.	-14962.	18318.	18515.	9159.	5
28	SCRAM+LOFWP4 +	LS-28	10		1135.	15136.	-727.	-12959.	26076.	26097.	13038.	31
29	SCRAM+LOFWP5 -	LS-29	10		885.	24686.	-1036.	-22025.	-26053.	-64906.	-13027.	-223
30	SCRAM+LOFWP6 +	LS-30	10		1060.	16738.	-747.	-14459.	18317.	18413.	9159.	2
31	SCRAM+LOFWP7 +	LS-31	10		1135.	15415.	-733.	-13217.	25130.	25158.	12565.	27

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

DELTA T1 IN DEGREE
PRESSURES IN PSI
STRESSES IN PSI

SUMMARY OF LOAD SETS AT POINT 155 LR ELBOW 155 TO 160 GLOBAL MOMENTS IN FT-LB

LOAD SET NO.	LOAD SET DESCRIPTION		CYCLES	DYNAM. CYCLES	PRESSURE	MOMENT X	MOMENT Y	MOMENT Z	TRANSIENT STRESSES EQ. 10	EQ. 11	EQ. 13	DELT T1
32	SCRAM+LOFWP8 -	LS-32	10		675.	23044.	-968.	-20560.	-25110.	-53661.	-12555.	-167
33	SCRAM+LOFWP9 +	LS-33	10		675.	16095.	-672.	-13953.	9507.	10417.	4754.	15
34	SCRAM+LOFWP10+	LS-34	10		1010.	14505.	-695.	-12403.	25119.	25126.	12559.	0
35	SCRAM+SRVBLDN1-	LS-35	1		885.	12714.	-388.	-10400.	0.	-6434.	0.	-34
36	SCRAM+SRVBLDN2-	LS-36	1		50.	56.	21.	54.	0.	-593.	0.	-3
37	Hydro Test +	LS-37	1		1563.	54.	22.	54.	0.	0.	0.	0
38	Hydro Test -	LS-38	1		50.	56.	21.	54.	0.	0.	0.	0
39	SCRAM+TG+OPres1 -	LS-39	289		1375.	7411.	-66.	-5088.	0.	0.	0.	0
40	SCRAM+TG+OPres2 -	LS-40	289		940.	4126.	66.	-2152.	0.	0.	0.	0
41	SCRAM+TG+OPres3 -	LS-41	289		1010.	4661.	45.	-2630.	0.	0.	0.	0
42	HotSbyFWcyc +	LS-42	300		1010.	20056.	-857.	-17600.	10427.	10652.	5214.	29
43	HotSbyFWcyc +	LS-43	300		1010.	14505.	-695.	-12403.	25130.	25446.	12565.	23
44	NORMAL+OBE LS-132		5	10(1)	1010.	14078.	-2008.	-11434.	0.	0.	0.	0
45	NORMAL-OBE LS-133		5	10(1)	1010.	10494.	1458.	-8380.	0.	0.	0.	0
101	WEIGHT					10013.	720.	-7200.				
104	DYNAMIC FLAG= 1					1792.	1733.	1527.				

B1	C1	K1	B2	C2	K2	C3	K3	C3PRIM	C4	Z	DIAM/TH	MATERIAL	E
0.106	1.247	1.000	2.022	3.034	1.000	1.000	1.000	0.500	1.100	0.60321E+02	12.752	CARBON STEEL	0.295

THIS ANALYSIS IS FOR THE BODY OF THE FITTING.

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
28	29	62066.b	7949.	255.1	35508.	100939.	1.317	66456.	1808.	53580.	759.3
29	31	60890.b	7720.	251.0	35035.	99771.	1.273	63498.	2050.	53580.	763.1
29	43	60634.b	8457.	247.4	34041.	99802.	1.263	63040.	2098.	53580.	763.1
29	34	60622.b	8457.	224.3	34035.	99482.	1.263	62816.	2122.	53580.	763.1
3	29	60614.b	8457.	224.3	34031.	99470.	1.263	62793.	2125.	53580.	763.1
28	32	61463.b	6621.	198.3	36705.	90035.	1.294	58265.	2697.	53580.	763.1
20	29	58527.b	13533.	224.4	30450.	97400.	1.185	57693.	2783.	53580.	763.1
25	29	58182.b	18755.	266.0	28382.	97049.	1.172	56861.	2915.	53580.	759.3
31	32	60288.b	6392.	194.2	36232.	88867.	1.219	54146.	3406.	54348.	766.8
4	28	59539.b	7360.	201.2	34042.	88515.	1.222	54103.	3415.	53580.	763.1
32	43	60031.b	7129.	190.6	35238.	88898.	1.209	53745.	3488.	54348.	766.8
32	34	60020.b	7129.	167.6	35233.	88578.	1.209	53533.	3532.	54348.	766.8
3	32	60011.b	7129.	167.6	35228.	88566.	1.208	53512.	3536.	54348.	766.8
4	31	58364.b	7131.	197.0	33569.	87347.	1.148	50128.	4354.	54348.	766.8
4	43	58108.b	7868.	193.5	32576.	87378.	1.138	49733.	4465.	54348.	766.8
4	34	58096.b	7868.	170.4	32570.	87058.	1.138	49533.	4523.	54348.	766.8
3	4	58088.b	7868.	170.4	32566.	87046.	1.138	49512.	4529.	54348.	766.8
25	32	57580.b	17427.	209.2	29579.	86146.	1.149	49505.	4531.	53580.	763.1
20	32	57925.b	12205.	167.6	31647.	86496.	1.132	48941.	4700.	54348.	766.8
26	43	47001.g	9927.	274.8	26489.	93513.	1.000	46757.	5404.	53580.	763.1
26	34	46989.g	9927.	251.8	26484.	93193.	1.000	46597.	5460.	53580.	763.1
3	26	46980.g	9927.	251.7	26479.	93181.	1.000	46591.	5462.	53580.	763.1
26	28	46445.g	9419.	282.5	25969.	92663.	1.000	46332.	5553.	53580.	759.3
27	29	52528.g	6169.	229.1	31629.	91578.	1.000	45789.	5750.	53580.	788.7
2	29	52662.g	19973.	223.8	27119.	91515.	1.000	45758.	5762.	53580.	849.3
24	29	52662.g	19973.	223.3	27119.	91515.	1.000	45758.	5762.	53580.	810.3

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	SP EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
29	36	52662.g	19973.	220.6	27119.	91515.	1.000	45758.	5762.	53580.	821.0
29	38	52662.g	19973.	223.8	27119.	91515.	1.000	45758.	5762.	53580.	849.3
22	28	64676.b	5819.	31.9	40719.	64699.	1.414	45748.	5766.	53580.	763.1
26	31	45270.g	9190.	278.4	25496.	91495.	1.000	45747.	5766.	53580.	763.1
29	30	52386.g	6625.	226.2	31032.	91335.	1.000	45668.	5796.	53580.	788.7
4	25	55656.b	18165.	212.0	26916.	84625.	1.077	45590.	5825.	53580.	763.1
20	26	44893.g	15003.	251.8	22898.	91110.	1.000	45555.	5838.	53580.	763.1
29	37	51415.g	19974.	223.8	25871.	90268.	1.000	45134.	6002.	53580.	784.2
5	29	36248.		302.3		90149.	1.000	45075.	6025.	53580.	823.9
4	20	56000.b	12943.	170.5	28984.	84975.	1.061	45071.	6026.	54348.	766.8
19	29	49926.g	11928.	263.5	26952.	88796.	1.000	44398.	6302.	53580.	788.7
25	26	42561.		293.4		88773.	1.000	44386.	6307.	53580.	759.3
2	26	41017.		251.2		87213.	1.000	43607.	6647.	53580.	849.3
24	26	41395.		250.8		87213.	1.000	43607.	6647.	53580.	810.3
26	36	41017.		248.0		87213.	1.000	43607.	6647.	53580.	821.0
26	38	41017.		251.2		87213.	1.000	43607.	6647.	53580.	849.3
1	29	47735.g	19973.	224.1	22191.	86638.	1.000	43319.	6779.	53580.	849.3
22	31	63959.b	6048.	27.7	40246.	63989.	1.354	43310.	6783.	54348.	766.8
23	29	47490.g	15103.	222.4	26817.	86343.	1.000	43172.	6848.	53580.	801.6
16	29	43884.g	16837.	236.9	21476.	85169.	1.000	42585.	7132.	53580.	798.3
22	29	45501.g	13765.	223.2	26165.	84354.	1.000	42177.	7338.	53580.	763.1
26	30	37958.		253.6		84251.	1.000	42125.	7365.	53580.	788.7
29	33	44347.g	7118.	238.8	26905.	84109.	1.000	42055.	7402.	53580.	816.1
5	26	22614.		329.7		83859.	1.000	41930.	7467.	53580.	823.9
29	39	44560.g	14613.	223.8	24377.	83413.	1.000	41706.	7587.	53580.	787.5
26	27	36908.		256.6		83301.	1.000	41651.	7617.	53580.	788.7

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
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DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
8	29	43884.g	16837.	225.5	21476.	83053.	1.000	41527.	7684.	53580.	798.3
6	29	43884.g	16837.	225.2	21476.	82988.	1.000	41494.	7702.	53580.	798.3
14	29	43884.g	16837.	225.1	21476.	82979.	1.000	41490.	7705.	53580.	798.3
10	29	43884.g	16837.	225.0	21476.	82951.	1.000	41475.	7713.	53580.	798.3
29	41	43884.g	16837.	223.8	21476.	82736.	1.000	41368.	7772.	53580.	798.3
29	40	43760.g	17270.	223.8	20920.	82613.	1.000	41307.	7807.	53580.	798.3
19	26	36292.		290.9		82507.	1.000	41253.	7836.	53580.	788.7
23	26	39055.		249.8		82041.	1.000	41021.	7969.	53580.	801.6
26	37	35796.		251.2		81993.	1.000	40996.	7983.	53580.	784.2
2	4	52124.g	19385.	169.9	27641.	81079.	1.000	40539.	8253.	54348.	851.7
4	24	52124.g	19385.	169.4	27641.	81079.	1.000	40539.	8253.	54348.	813.4
4	36	52124.g	19385.	166.7	27641.	81079.	1.000	40539.	8253.	54348.	824.1
4	38	52124.g	19385.	169.9	27641.	81079.	1.000	40539.	8253.	54348.	851.7
22	43	62228.b	5311.	24.1	39253.	62546.	1.290	40342.	8374.	54348.	766.8
27	32	51926.g	4842.	172.4	32826.	80674.	1.000	40337.	8377.	54348.	792.1
30	32	51784.g	5297.	169.4	32229.	80431.	1.000	40216.	8452.	54348.	792.1
29	42	41341.		253.0		80418.	1.000	40209.	8456.	53580.	818.0
22	34	62217.b	5311.	1.1	39247.	62226.	1.290	40123.	8510.	54348.	766.8
3	22	62208.b	5311.	1.1	39243.	62214.	1.289	40105.	8521.	54348.	766.8
22	26	48016.g	15236.	250.7	27681.	80052.	1.000	40026.	8571.	53580.	763.1
26	33	32701.		266.2		79807.	1.000	39904.	8649.	53580.	816.1
32	37	50812.g	18645.	167.0	27068.	79364.	1.000	39682.	8794.	54348.	788.7
5	32	35646.		245.5		79246.	1.000	39623.	8832.	54348.	827.0
4	27	50002.g	5580.	175.2	30163.	79153.	1.000	39577.	8863.	54348.	792.1
1	26	32672.		251.5		78918.	1.000	39459.	8942.	53580.	849.3
4	30	49860.g	6036.	172.2	29566.	78911.	1.000	39455.	8944.	54348.	792.1

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
16	26	30250.		264.3		78880.	1.000	39440.	8955.	53580.	798.3
7	29	39141.		222.1		77994.	1.000	38997.	9260.	53580.	798.3
19	32	49324.g	10600.	206.7	28149.	77893.	1.000	38946.	9296.	54348.	792.1
29	45	39023.		223.8		77876.	1.000	38938.	9302.	53580.	814.2
4	37	48890.g	19386.	169.9	24406.	77845.	1.000	38922.	9313.	54348.	788.7
4	5	33721.		248.4		77724.	1.000	38862.	9356.	54348.	827.0
26	40	31239.		251.2		77436.	1.000	38718.	9459.	53580.	798.3
2	32	48721.g	18644.	167.0	24978.	77272.	1.000	38636.	9519.	54348.	851.7
24	32	48721.g	18644.	166.6	24978.	77272.	1.000	38636.	9519.	54348.	813.4
32	36	48721.g	18644.	163.9	24978.	77272.	1.000	38636.	9519.	54348.	824.1
32	38	48721.g	18644.	167.0	24978.	77272.	1.000	38636.	9519.	54348.	851.7
8	26	30250.		252.9		76764.	1.000	38382.	9707.	53580.	798.3
6	26	30250.		252.6		76698.	1.000	38349.	9732.	53580.	798.3
14	26	30250.		252.5		76689.	1.000	38345.	9735.	53580.	798.3
10	26	30250.		252.4		76661.	1.000	38331.	9746.	53580.	798.3
26	41	30250.		251.2		76447.	1.000	38223.	9827.	53580.	798.3
17	29	37547.		189.4		76400.	1.000	38200.	9845.	53580.	798.3
4	19	47399.g	11338.	209.6	25486.	76372.	1.000	38186.	9856.	54348.	792.1
9	29	37405.		222.7		76258.	1.000	38129.	9899.	53580.	798.3
13	29	36537.		227.1		76009.	1.000	38004.	9996.	53580.	822.9
4	23	46952.g	14514.	168.5	27338.	75906.	1.000	37953.	10042.	54348.	804.7
1	32	47132.g	18645.	167.3	23388.	75733.	1.000	37867.	10120.	54348.	851.7
11	29	36537.		222.5		75390.	1.000	37695.	10278.	53580.	798.3
15	29	36537.		198.1		75390.	1.000	37695.	10278.	53580.	798.3
26	39	28939.		251.2		75136.	1.000	37568.	10397.	53580.	787.5
29	35	36132.		189.4		74985.	1.000	37492.	10468.	53580.	798.3

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
29	44	36112.		223.8		74965.	1.000	37482.	10477.	53580.	814.2
21	29	35503.		223.8		74356.	1.000	37178.	10772.	53580.	763.1
16	32	43282.		180.1		74266.	1.000	37133.	10817.	54348.	801.4
1	4	45210.g	19385.	170.1	20726.	74214.	1.000	37107.	10843.	54348.	851.7
26	42	27708.		280.4		74129.	1.000	37064.	10885.	53580.	818.0
4	22	44964.g	13177.	169.3	26687.	73917.	1.000	36959.	10992.	54348.	766.8
4	33	43808.g	6529.	184.9	27427.	73672.	1.000	36836.	11117.	54348.	819.2
4	16	41357.		182.9		72745.	1.000	36372.	11607.	54348.	801.4
32	39	43959.g	13286.	167.0	25574.	72510.	1.000	36255.	11735.	54348.	793.6
8	32	43282.		168.7		72150.	1.000	36075.	11936.	54348.	801.4
23	32	43549.g	13774.	165.6	24676.	72100.	1.000	36050.	11964.	54348.	804.7
6	32	43282.		168.4		72085.	1.000	36042.	11972.	54348.	801.4
14	32	43282.		168.3		72076.	1.000	36038.	11978.	54348.	801.4
10	32	43282.		168.2		72048.	1.000	36024.	11993.	54348.	801.4
32	41	43282.		167.0		71833.	1.000	35917.	12116.	54348.	801.4
32	40	43158.		167.0		71710.	1.000	35855.	12187.	54348.	801.4
7	26	25507.		249.5		71704.	1.000	35852.	12190.	53580.	798.3
26	45	25383.		251.2		71580.	1.000	35790.	12262.	53580.	814.2
4	40	42346.		169.9		71301.	1.000	35651.	12426.	54348.	801.4
4	39	42033.		169.9		70988.	1.000	35494.	12614.	54348.	793.6
26	35	24485.		216.8		70682.	1.000	35341.	12801.	53580.	798.3
4	8	41357.		171.6		70629.	1.000	35314.	12834.	54348.	801.4
4	6	41357.		171.2		70563.	1.000	35282.	12874.	54348.	801.4
4	14	41357.		171.2		70554.	1.000	35277.	12880.	54348.	801.4
4	10	41357.		171.0		70526.	1.000	35263.	12897.	54348.	801.4
4	41	41357.		169.9		70312.	1.000	35156.	13032.	54348.	801.4

Notes
b,d,e,k: Fails
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L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

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INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
22	32	41561.		166.5		70111.	1.000	35056.	13159.	54348.	766.8
17	26	23913.		216.8		70110.	1.000	35055.	13160.	53580.	798.3
9	26	23771.		250.1		69968.	1.000	34984.	13251.	53580.	798.3
32	33	40406.		182.1		69867.	1.000	34934.	13316.	54348.	819.2
13	26	22903.		254.6		69719.	1.000	34860.	13413.	53580.	822.9
32	42	40739.		196.2		69515.	1.000	34757.	13548.	54348.	821.2
11	26	22903.		249.9		69100.	1.000	34550.	13826.	53580.	798.3
15	26	22903.		225.6		69100.	1.000	34550.	13826.	53580.	798.3
26	44	22472.		251.2		68669.	1.000	34334.	14124.	53580.	814.2
26	29	29511.		27.4		68364.	1.000	34182.	14340.	53580.	759.3
22	27	58697.b	7598.	5.9	36840.	58896.	1.160	34161.	14370.	54348.	792.1
21	26	21870.		251.2		68067.	1.000	34034.	14554.	53580.	763.1
4	42	38815.		199.0		67994.	1.000	33997.	14607.	54348.	821.2
29	32	29051.		56.8		67904.	1.000	33952.	14674.	53580.	763.1
24	28	58057.b	12029.	31.8	34782.	58080.	1.167	33893.	14760.	53580.	810.3
7	32	38539.		165.3		67091.	1.000	33545.	15288.	54348.	801.4
32	45	38428.		167.0		66979.	1.000	33489.	15375.	54348.	817.3
18	29	28066.		220.6		66919.	1.000	33459.	15422.	53580.	821.0
4	29	27637.		53.9		66490.	1.000	33245.	15763.	53580.	763.1
12	29	27212.		215.9		66064.	1.000	33032.	16111.	53580.	822.9
4	7	36614.		168.2		65569.	1.000	32785.	16530.	54348.	801.4
17	32	36945.		132.6		65496.	1.000	32748.	16592.	54348.	801.4
4	45	36498.		169.9		65453.	1.000	32726.	16630.	54348.	817.3
9	32	36804.		165.9		65355.	1.000	32677.	16715.	54348.	801.4
13	32	35935.		170.4		65106.	1.000	32553.	16934.	54348.	826.0
22	30	57644.b	7142.	2.9	36243.	57742.	1.121	32373.	17257.	54348.	792.1

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
4	35	35592.		135.4		64547.	1.000	32273.	17438.	54348.	801.4
11	32	35935.		165.7		64487.	1.000	32243.	17493.	54348.	801.4
15	32	35935.		141.4		64487.	1.000	32243.	17493.	54348.	801.4
32	35	35530.		132.6		64082.	1.000	32041.	17873.	54348.	801.4
32	44	35518.		167.0		64070.	1.000	32035.	17884.	54348.	817.3
4	17	35020.		135.4		63975.	1.000	31987.	17974.	54348.	801.4
4	9	34878.		168.7		63833.	1.000	31917.	18111.	54348.	801.4
24	31	57340.b	12259.	27.6	34309.	57370.	1.110	31844.	18252.	54348.	813.4
26	32	31565.		84.2		63602.	1.000	31801.	18336.	53580.	763.1
4	13	34010.		173.2		63584.	1.000	31792.	18353.	54348.	826.0
21	32	34901.		167.0		63452.	1.000	31726.	18484.	54348.	766.8
4	11	34010.		168.6		62965.	1.000	31483.	18975.	54348.	801.4
4	15	34010.		144.2		62965.	1.000	31483.	18975.	54348.	801.4
4	44	33589.		169.9		62544.	1.000	31272.	19413.	54348.	817.3
4	21	32977.		169.9		61932.	1.000	30966.	20067.	54348.	766.8
18	26	14433.		248.0		60630.	1.000	30315.	21420.	53580.	821.0
23	28	55715.b	7157.	32.7	35896.	55816.	1.080	30132.	21821.	53580.	801.6
4	26	28162.		81.3		60199.	1.000	30100.	21893.	53580.	763.1
12	26	13576.		243.3		59773.	1.000	29886.	22376.	53580.	822.9
24	43	55610.b	11521.	24.0	33315.	55927.	1.046	29262.	23875.	54348.	813.4
24	34	55598.b	11521.	1.0	33309.	55607.	1.046	29083.	24329.	54348.	813.4
3	24	55590.b	11521.	1.0	33305.	55595.	1.046	29068.	24367.	54348.	813.4
12	32	28939.		159.1		57491.	1.000	28745.	25217.	54348.	826.0
4	32	28514.		2.8		57469.	1.000	28735.	25246.	54348.	766.8
18	32	28086.		163.9		56638.	1.000	28319.	26401.	54348.	824.1
23	31	54998.b	7386.	28.5	35423.	55107.	1.024	28212.	26708.	54348.	804.7

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [3

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
4	18	25540.		166.7		54495.	1.000	27247.	29720.	54348.	824.1
4	12	25536.		161.9		54491.	1.000	27245.	29726.	54348.	826.0
23	43	53267.g	6649.	25.0	34429.	53664.	1.000	26832.	31155.	54348.	804.7
23	34	53256.g	6649.	1.9	34423.	53344.	1.000	26672.	31732.	54348.	804.7
3	23	53247.g	6649.	1.9	34419.	53332.	1.000	26666.	31754.	54348.	804.7
22	42	52114.g	9899.	29.7	31901.	52341.	1.000	26170.	33637.	54348.	821.2
24	27	52078.g	13808.	5.8	30903.	52277.	1.000	26138.	33763.	58680.	833.3
5	22	36363.		79.1		51414.	1.000	25707.	35533.	54348.	827.0
24	30	51025.g	13352.	2.8	30306.	51123.	1.000	25561.	36158.	58680.	833.3
20	22	50097.g	363.	1.1	35661.	50120.	1.000	25060.	38425.	54348.	766.8
23	27	49736.g	8936.	6.7	32017.	50014.	1.000	25007.	38676.	58680.	825.8
22	25	49174.g	5006.	42.7	33593.	49190.	1.000	24595.	40698.	53580.	763.1
23	30	48683.g	8480.	3.8	31420.	48860.	1.000	24430.	41549.	58680.	825.8
28	36	46729.g	12029.	34.5	29118.	47343.	1.000	23672.	45771.	53580.	821.0
2	28	46729.g	12029.	31.3	29118.	46750.	1.000	23375.	47577.	53580.	849.3
28	38	46729.g	12029.	31.3	29118.	46750.	1.000	23375.	47577.	53580.	849.3
31	36	46012.g	12259.	30.3	28644.	46633.	1.000	23317.	47944.	54348.	824.1
22	33	45280.g	6649.	15.6	28778.	46192.	1.000	23096.	49364.	54348.	819.2
2	31	46012.g	12259.	27.2	28644.	46040.	1.000	23020.	49865.	54348.	851.7
31	38	46012.g	12259.	27.2	28644.	46040.	1.000	23020.	49865.	54348.	851.7
24	42	45494.		29.6		45720.	1.000	22860.	51538.	60000.	856.2
12	22	45390.g	13603.	7.4	26687.	45392.	1.000	22696.	53409.	54348.	826.0
36	43	44282.g	11521.	26.8	27651.	45190.	1.000	22595.	54602.	54348.	824.1
34	36	44270.g	11521.	3.7	27645.	44871.	1.000	22435.	56559.	54348.	824.1
3	36	44262.g	11521.	3.7	27641.	44859.	1.000	22429.	56635.	54348.	824.1
5	24	29744.		79.0		44794.	1.000	22397.	57040.	60000.	860.7

Notes b,d,e,k: Fails
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h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
19	22	44599.g	1861.	40.2	32163.	44619.	1.000	22310.	58159.	54348.	792.1
2	43	44282.g	11521.	23.6	27651.	44597.	1.000	22299.	58299.	54348.	851.7
38	43	44282.g	11521.	23.6	27651.	44597.	1.000	22299.	58299.	54348.	851.7
18	22	44533.g	12746.	2.6	26687.	44536.	1.000	22268.	58701.	54348.	824.1
2	34	44270.g	11521.	0.5	27645.	44278.	1.000	22139.	60417.	54348.	851.7
34	38	44270.g	11521.	0.5	27645.	44278.	1.000	22139.	60417.	54348.	851.7
2	3	44262.g	11521.	0.5	27641.	44266.	1.000	22133.	60499.	54348.	851.7
3	38	44262.g	11521.	0.5	27641.	44266.	1.000	22133.	60499.	54348.	851.7
23	42	43153.		30.5		43458.	1.000	21729.	66286.	60000.	849.3
20	24	43368.		1.0		43390.	1.000	21695.	66803.	54348.	813.4
5	23	27401.		79.9		42530.	1.000	21265.	73771.	60000.	853.9
22	37	42394.		0.6		42396.	1.000	21198.	74933.	54348.	788.7
27	36	40750.		8.5		41540.	1.000	20770.	82915.	58680.	841.7
28	37	41509.		31.3		41529.	1.000	20765.	83019.	53580.	784.2
37	43	41048.		23.6		41364.	1.000	20682.	84683.	54348.	788.7
20	23	41038.		2.0		41138.	1.000	20569.	87007.	54348.	804.7
34	37	41036.		0.5		41044.	1.000	20522.	88005.	54348.	788.7
3	37	41028.		0.5		41032.	1.000	20516.	88134.	54348.	788.7
2	27	40750.		5.3		40947.	1.000	20473.	89044.	58680.	868.4
27	38	40750.		5.3		40947.	1.000	20473.	89044.	58680.	868.4
31	37	40792.		27.2		40820.	1.000	20410.	90429.	54348.	788.7
30	36	39697.		5.6		40386.	1.000	20193.	95348.	58680.	841.7
2	30	39697.		2.4		39793.	1.000	19897.	101886.	58680.	868.4
30	38	39697.		2.4		39793.	1.000	19897.	101886.	58680.	868.4
24	33	38661.		15.5		39572.	1.000	19786.	103950.	60000.	854.6
12	24	38769.		7.5		38770.	1.000	19385.	111907.	60000.	860.0

Notes b,d,e,k: Fails
g: Weld ISI
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L: Information

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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
1	22	38714.		0.8		38766.	1.000	19383.	111954.	54348.	851.7
1	28	38385.		31.1		38406.	1.000	19203.	115784.	53580.	849.3
19	24	37973.		40.1		37992.	1.000	18996.	120392.	58680.	833.3
18	24	37912.		2.7		37914.	1.000	18957.	121286.	60000.	858.4
1	31	37668.		26.9		37696.	1.000	18848.	123834.	54348.	851.7
1	43	37368.		23.3		37683.	1.000	18842.	123981.	54348.	851.7
23	25	37549.		43.5		37644.	1.000	18822.	124447.	53580.	801.6
1	34	37356.		0.3		37364.	1.000	18682.	127846.	54348.	851.7
1	3	37348.		0.3		37352.	1.000	18676.	127996.	54348.	851.7
25	28	37321.		10.8		37342.	1.000	18671.	128119.	53580.	759.3
16	22	34887.		13.6		37322.	1.000	18661.	128359.	54348.	801.4
23	33	36319.		16.4		37309.	1.000	18655.	128517.	60000.	847.8
25	43	36861.		18.6		37176.	1.000	18588.	130181.	53580.	763.1
21	22	37098.		0.6		37100.	1.000	18550.	131149.	54348.	766.8
28	40	36950.		31.3		36971.	1.000	18485.	132812.	53580.	798.3
25	34	36849.		41.6		36857.	1.000	18428.	134297.	53580.	763.1
3	25	36841.		41.6		36845.	1.000	18422.	134456.	53580.	763.1
13	22	36074.		3.9		36696.	1.000	18348.	136430.	54348.	826.0
28	35	30200.		65.8		36655.	1.000	18328.	136979.	53580.	798.3
25	31	36604.		15.0		36632.	1.000	18316.	137291.	53580.	763.1
22	44	36626.		0.6		36628.	1.000	18314.	137342.	54348.	817.3
12	23	36429.		6.5		36509.	1.000	18255.	138959.	60000.	853.1
12	28	34857.		39.3		36357.	1.000	18179.	141064.	53580.	822.9
31	40	36233.		27.2		36261.	1.000	18130.	142421.	54348.	801.4
17	28	29626.		65.8		36082.	1.000	18041.	144986.	53580.	798.3
11	22	36074.		0.7		36077.	1.000	18038.	145056.	54348.	801.4

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15	22	36074.		25.1		36077.	1.000	18038.	145056.	54348.	801.4
28	41	35960.		31.3		35981.	1.000	17991.	146447.	53580.	798.3
6	28	35960.		30.0		35981.	1.000	17991.	146449.	53580.	798.3
8	28	35960.		29.6		35981.	1.000	17991.	146449.	53580.	798.3
10	28	35960.		30.2		35981.	1.000	17991.	146449.	53580.	798.3
14	28	35960.		30.0		35981.	1.000	17991.	146449.	53580.	798.3
16	28	35960.		18.3		35981.	1.000	17991.	146449.	53580.	798.3
31	35	29482.		61.6		35944.	1.000	17972.	146991.	54348.	801.4
30	37	35668.		2.4		35764.	1.000	17882.	149673.	58680.	819.8
19	23	35632.		41.1		35730.	1.000	17865.	150191.	58680.	825.8
27	37	35529.		5.3		35726.	1.000	17863.	150253.	58680.	819.8
22	39	35660.		0.6		35662.	1.000	17831.	151224.	54348.	793.6
18	23	35572.		1.8		35653.	1.000	17826.	151370.	60000.	851.6
17	31	28909.		61.6		35371.	1.000	17686.	155755.	54348.	801.4
31	41	35243.		27.2		35272.	1.000	17636.	157347.	54348.	801.4
6	31	35243.		25.8		35271.	1.000	17636.	157350.	54348.	801.4
8	31	35243.		25.5		35271.	1.000	17636.	157350.	54348.	801.4
10	31	35243.		26.0		35271.	1.000	17636.	157350.	54348.	801.4
14	31	35243.		25.9		35271.	1.000	17636.	157350.	54348.	801.4
16	31	35243.		14.1		35271.	1.000	17636.	157350.	54348.	801.4
12	43	33425.		31.5		35220.	1.000	17610.	158176.	54348.	826.0
9	22	35211.		0.6		35213.	1.000	17606.	158293.	54348.	801.4
8	22	34887.		2.3		35206.	1.000	17603.	158403.	54348.	801.4
12	31	33681.		35.1		35189.	1.000	17595.	158680.	54348.	826.0
6	22	34887.		1.9		35141.	1.000	17570.	159466.	54348.	801.4
14	22	34887.		1.9		35132.	1.000	17566.	159612.	54348.	801.4

Notes b,d,e,k: Fails
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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
10	22	34887.		1.7		35104.	1.000	17552.	160075.	54348.	801.4
17	22	35068.		33.9		35070.	1.000	17535.	160627.	54348.	801.4
36	42	34166.		32.3		34983.	1.000	17492.	162071.	60000.	864.5
12	34	33413.		8.5		34900.	1.000	17450.	163460.	54348.	826.0
22	41	34887.		0.6		34889.	1.000	17445.	163651.	54348.	801.4
3	12	33405.		8.5		34888.	1.000	17444.	163664.	54348.	826.0
40	43	34503.		23.6		34819.	1.000	17409.	164848.	54348.	801.4
22	40	34759.		0.6		34761.	1.000	17381.	165831.	54348.	801.4
28	39	34652.		31.3		34673.	1.000	17337.	167354.	53580.	787.5
18	28	34000.		34.5		34614.	1.000	17307.	168392.	53580.	821.0
39	43	34193.		23.6		34509.	1.000	17254.	170242.	54348.	793.6
35	43	27756.		58.0		34506.	1.000	17253.	170292.	54348.	801.4
34	40	34491.		0.5		34499.	1.000	17249.	170419.	54348.	801.4
22	35	34495.		33.9		34498.	1.000	17249.	170442.	54348.	801.4
3	40	34483.		0.5		34487.	1.000	17243.	170635.	54348.	801.4
2	42	34166.		29.2		34390.	1.000	17195.	172368.	60000.	893.7
38	42	34166.		29.2		34390.	1.000	17195.	172368.	60000.	893.7
34	39	34181.		0.5		34189.	1.000	17095.	176049.	54348.	793.6
34	35	27745.		35.0		34186.	1.000	17093.	176101.	54348.	801.4
3	39	34173.		0.5		34177.	1.000	17089.	176274.	54348.	793.6
3	35	27736.		35.0		34174.	1.000	17087.	176326.	54348.	801.4
5	36	18416.		81.7		34057.	1.000	17029.	178513.	60000.	869.1
22	45	33979.		0.6		33981.	1.000	16990.	179967.	54348.	817.3
31	39	33935.		27.2		33963.	1.000	16982.	180307.	54348.	793.6
17	43	27181.		58.0		33931.	1.000	16966.	180916.	54348.	801.4
41	43	33514.		23.6		33829.	1.000	16915.	182887.	54348.	801.4

Notes b,d,e,k: Fails
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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

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INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
6	43	33514.		22.2		33829.	1.000	16915.	182890.	54348.	801.4
8	43	33514.		21.9		33829.	1.000	16915.	182890.	54348.	801.4
10	43	33514.		22.4		33829.	1.000	16915.	182890.	54348.	801.4
14	43	33514.		22.3		33829.	1.000	16915.	182890.	54348.	801.4
16	43	33514.		10.5		33829.	1.000	16915.	182890.	54348.	801.4
17	34	27170.		35.0		33612.	1.000	16806.	187194.	54348.	801.4
3	17	27161.		35.0		33599.	1.000	16800.	187437.	54348.	801.4
34	41	33502.		0.5		33510.	1.000	16755.	189252.	54348.	801.4
6	34	33502.		0.8		33510.	1.000	16755.	189256.	54348.	801.4
8	34	33502.		1.2		33510.	1.000	16755.	189256.	54348.	801.4
10	34	33502.		0.6		33510.	1.000	16755.	189256.	54348.	801.4
14	34	33502.		0.8		33510.	1.000	16755.	189256.	54348.	801.4
16	34	33502.		12.5		33510.	1.000	16755.	189256.	54348.	801.4
7	22	33497.		1.1		33499.	1.000	16750.	189470.	54348.	801.4
3	41	33494.		0.5		33498.	1.000	16749.	189499.	54348.	801.4
3	6	33494.		0.8		33497.	1.000	16749.	189503.	54348.	801.4
3	8	33494.		1.2		33497.	1.000	16749.	189503.	54348.	801.4
3	10	33494.		0.6		33497.	1.000	16749.	189503.	54348.	801.4
3	14	33494.		0.8		33497.	1.000	16749.	189503.	54348.	801.4
3	16	33494.		12.5		33497.	1.000	16749.	189503.	54348.	801.4
18	43	32568.		26.8		33477.	1.000	16738.	189930.	54348.	824.1
2	5	18416.		78.5		33465.	1.000	16732.	190176.	60000.	905.8
5	38	18416.		78.5		33465.	1.000	16732.	190176.	60000.	905.8
18	31	32824.		30.3		33445.	1.000	16723.	190566.	54348.	824.1
15	28	28620.		57.0		33425.	1.000	16713.	190984.	53580.	798.3
18	34	32556.		3.7		33157.	1.000	16578.	196612.	54348.	824.1

Notes b,d,e,k: Fails
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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
3	18	32548.		3.7		33145.	1.000	16572.	196871.	54348.	824.1
12	20	31318.		8.5		32817.	1.000	16409.	205131.	54348.	826.0
24	25	32755.		42.6		32771.	1.000	16386.	206456.	53580.	810.3
15	31	27902.		52.8		32714.	1.000	16357.	208114.	54348.	801.4
20	28	32655.		30.7		32675.	1.000	16338.	209234.	53580.	763.1
20	36	32040.		3.8		32653.	1.000	16327.	209888.	54348.	824.1
1	27	32405.		5.1		32602.	1.000	16301.	211385.	58680.	868.4
12	25	30973.		50.1		32467.	1.000	16233.	215442.	53580.	822.9
2	22	32275.		0.6		32278.	1.000	16139.	221264.	54348.	851.7
22	24	32275.		0.1		32278.	1.000	16139.	221264.	54348.	813.4
22	36	32275.		2.6		32278.	1.000	16139.	221264.	54348.	824.1
22	38	32275.		0.6		32278.	1.000	16139.	221264.	54348.	851.7
1	30	31988.		2.1		32084.	1.000	16042.	227424.	58680.	868.4
2	20	32040.		0.6		32060.	1.000	16030.	228208.	54348.	851.7
20	38	32040.		0.6		32060.	1.000	16030.	228208.	54348.	851.7
20	31	31938.		26.6		31966.	1.000	15983.	231302.	54348.	766.8
25	30	31481.		39.8		31577.	1.000	15789.	244590.	53580.	788.7
7	28	31219.		33.0		31557.	1.000	15779.	245285.	53580.	798.3
25	27	31341.		36.8		31538.	1.000	15769.	245964.	53580.	788.7
15	43	26180.		49.2		31279.	1.000	15640.	255410.	54348.	801.4
28	45	31220.		31.3		31241.	1.000	15620.	256841.	53580.	814.2
28	42	31152.		2.2		31173.	1.000	15587.	259394.	53580.	818.0
27	40	30970.		5.3		31167.	1.000	15583.	259631.	58680.	822.5
37	42	30932.		29.2		31156.	1.000	15578.	260037.	60000.	868.4
18	20	30461.		3.8		31074.	1.000	15537.	263206.	54348.	824.1
19	28	31051.		8.4		31071.	1.000	15536.	263297.	53580.	788.7

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15	34	26168.		26.2		30959.	1.000	15480.	267678.	54348.	801.4
3	15	26159.		26.2		30947.	1.000	15474.	268157.	54348.	801.4
27	35	24217.		39.8		30848.	1.000	15424.	272130.	58680.	822.5
7	31	30502.		28.9		30847.	1.000	15424.	272146.	54348.	801.4
23	37	30758.		1.4		30838.	1.000	15419.	272503.	60000.	838.6
18	25	30116.		45.3		30723.	1.000	15362.	277200.	53580.	821.0
28	33	30567.		16.3		30588.	1.000	15294.	282853.	53580.	816.1
20	43	30208.		23.0		30523.	1.000	15262.	285593.	54348.	766.8
31	45	30493.		27.2		30521.	1.000	15260.	285693.	54348.	817.3
21	24	30480.		0.5		30481.	1.000	15241.	287400.	54348.	813.4
19	31	30333.		12.5		30362.	1.000	15181.	292600.	54348.	792.1
33	37	29424.		15.0		30334.	1.000	15167.	293826.	60000.	860.6
17	27	23644.		39.8		30276.	1.000	15138.	296416.	58680.	822.5
5	37	15182.		78.5		30231.	1.000	15115.	298432.	60000.	892.0
20	34	30196.		0.0		30203.	1.000	15102.	299660.	54348.	766.8
3	20	30188.		0.1		30191.	1.000	15096.	300210.	54348.	766.8
27	41	29980.		5.3		30178.	1.000	15089.	300835.	58680.	822.5
6	27	29980.		4.0		30177.	1.000	15089.	300844.	58680.	822.5
8	27	29980.		3.6		30177.	1.000	15089.	300844.	58680.	822.5
10	27	29980.		4.2		30177.	1.000	15089.	300844.	58680.	822.5
14	27	29980.		4.0		30177.	1.000	15089.	300844.	58680.	822.5
16	27	29980.		7.7		30177.	1.000	15089.	300844.	58680.	822.5
13	24	29455.		3.8		30076.	1.000	15038.	305510.	60000.	860.0
42	43	29721.		5.6		30036.	1.000	15018.	307361.	54348.	821.2
30	40	29917.		2.4		30014.	1.000	15007.	308417.	58680.	822.5
31	42	29977.		2.0		30005.	1.000	15003.	308818.	54348.	821.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
24	44	29966.		0.5		29968.	1.000	14984.	310575.	60000.	853.1
9	28	29486.		32.5		29721.	1.000	14861.	322516.	53580.	798.3
34	42	29709.		28.6		29716.	1.000	14858.	322754.	54348.	821.2
3	42	29701.		28.6		29704.	1.000	14852.	323355.	54348.	821.2
30	35	23164.		36.8		29695.	1.000	14847.	323829.	58680.	822.5
11	24	29455.		0.8		29457.	1.000	14728.	335946.	60000.	840.6
15	24	29455.		25.2		29457.	1.000	14728.	335946.	60000.	840.6
33	43	29133.		8.6		29449.	1.000	14724.	336356.	54348.	819.2
31	33	29392.		12.1		29420.	1.000	14710.	337846.	54348.	819.2
7	43	28774.		25.3		29406.	1.000	14703.	338595.	54348.	801.4
33	34	29122.		14.5		29129.	1.000	14565.	353547.	54348.	819.2
17	30	22592.		36.8		29122.	1.000	14561.	353924.	58680.	822.5
3	33	29113.		14.5		29117.	1.000	14559.	354220.	54348.	819.2
43	45	28797.		23.6		29112.	1.000	14556.	354495.	54348.	817.3
7	34	28762.		2.2		29086.	1.000	14543.	355926.	54348.	801.4
3	7	28753.		2.2		29074.	1.000	14537.	356604.	54348.	801.4
30	41	28928.		2.4		29024.	1.000	14512.	359424.	58680.	822.5
6	30	28928.		1.0		29024.	1.000	14512.	359435.	58680.	822.5
8	30	28928.		0.7		29024.	1.000	14512.	359435.	58680.	822.5
10	30	28928.		1.2		29024.	1.000	14512.	359435.	58680.	822.5
14	30	28928.		1.1		29024.	1.000	14512.	359435.	58680.	822.5
16	30	28928.		10.7		29024.	1.000	14512.	359435.	58680.	822.5
9	31	28768.		28.3		29011.	1.000	14505.	360192.	54348.	801.4
2	12	27441.		7.9		28921.	1.000	14460.	365335.	60000.	903.8
12	36	27441.		4.8		28921.	1.000	14460.	365335.	60000.	868.4
12	38	27441.		7.9		28921.	1.000	14460.	365335.	60000.	903.8

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
19	43	28604.		16.1		28920.	1.000	14460.	365397.	54348.	792.1
30	39	28811.		2.4		28907.	1.000	14454.	366116.	58680.	822.5
11	28	28620.		32.6		28884.	1.000	14442.	367461.	53580.	798.3
27	39	28671.		5.3		28868.	1.000	14434.	368386.	58680.	822.5
20	35	22397.		35.0		28851.	1.000	14425.	369387.	54348.	801.4
33	36	27333.		18.2		28836.	1.000	14418.	370288.	60000.	863.0
20	37	28806.		0.6		28826.	1.000	14413.	370834.	54348.	788.7
34	45	28785.		0.5		28792.	1.000	14396.	372829.	54348.	817.3
3	45	28777.		0.5		28780.	1.000	14390.	373546.	54348.	817.3
27	43	28412.		18.2		28727.	1.000	14364.	376708.	54348.	792.1
13	28	28620.		28.0		28641.	1.000	14321.	381903.	53580.	822.9
19	34	28592.		39.1		28600.	1.000	14300.	384425.	54348.	792.1
9	24	28590.		0.7		28591.	1.000	14296.	384960.	60000.	840.6
3	19	28584.		39.2		28588.	1.000	14294.	385169.	54348.	792.1
25	35	22050.		76.6		28499.	1.000	14249.	390691.	53580.	798.3
28	44	28431.		31.3		28451.	1.000	14226.	393669.	53580.	814.2
17	24	28447.		34.0		28449.	1.000	14224.	393854.	60000.	840.6
27	34	28400.		4.8		28407.	1.000	14204.	396461.	54348.	792.1
3	27	28392.		4.8		28395.	1.000	14198.	397234.	54348.	792.1
5	28	28333.		47.2		28354.	1.000	14177.	399912.	53580.	823.9
2	33	27333.		15.0		28243.	1.000	14121.	407140.	60000.	889.7
33	38	27333.		15.0		28243.	1.000	14121.	407140.	60000.	889.7
21	23	28137.		1.4		28218.	1.000	14109.	408793.	54348.	804.7
11	31	27902.		28.5		28173.	1.000	14086.	411773.	54348.	801.4
28	30	27997.		28.9		28018.	1.000	14009.	422270.	53580.	788.7
13	31	27902.		23.8		27930.	1.000	13965.	428376.	54348.	826.0

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
27	28	27856.		26.0		27877.	1.000	13939.	432081.	53580.	788.7
24	35	27875.		34.0		27877.	1.000	13938.	432125.	60000.	840.6
13	23	27112.		4.7		27812.	1.000	13906.	436721.	60000.	853.1
23	44	27648.		1.4		27729.	1.000	13864.	442732.	60000.	846.3
31	44	27673.		27.2		27701.	1.000	13851.	444748.	54348.	817.3
30	43	27360.		21.2		27675.	1.000	13838.	446650.	54348.	792.1
5	31	27614.		51.4		27642.	1.000	13821.	449150.	54348.	827.0
20	42	27614.		28.6		27634.	1.000	13817.	449757.	54348.	821.2
15	27	22635.		31.0		27616.	1.000	13808.	451074.	58680.	822.5
3	28	27578.		30.8		27599.	1.000	13799.	452344.	53580.	763.1
21	28	27578.		31.3		27599.	1.000	13799.	452344.	53580.	763.1
28	34	27578.		30.8		27599.	1.000	13799.	452344.	53580.	763.1
28	43	27578.		7.7		27599.	1.000	13799.	452344.	53580.	763.1
9	43	27041.		24.7		27572.	1.000	13786.	454392.	54348.	801.4
1	42	27252.		28.9		27476.	1.000	13738.	461639.	60000.	893.7
17	20	20982.		35.0		27436.	1.000	13718.	464734.	54348.	801.4
30	34	27348.		1.8		27356.	1.000	13678.	470989.	54348.	792.1
3	30	27340.		1.9		27344.	1.000	13672.	471943.	54348.	792.1
25	42	27269.		13.0		27283.	1.000	13642.	476732.	53580.	818.0
24	39	27275.		0.5		27276.	1.000	13638.	477271.	60000.	840.6
19	36	26645.		42.9		27255.	1.000	13628.	478974.	58680.	841.7
9	34	27030.		1.7		27252.	1.000	13626.	479249.	54348.	801.4
3	9	27021.		1.7		27240.	1.000	13620.	480224.	54348.	801.4
1	23	27078.		1.7		27208.	1.000	13604.	482793.	60000.	878.3
11	23	27112.		0.1		27193.	1.000	13597.	483997.	60000.	833.8
15	23	27112.		24.3		27193.	1.000	13597.	483997.	60000.	833.8

Notes b,d,e,k: Fails
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h,i: Rupture Location
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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

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INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
18	36	26584.		0.0		27177.	1.000	13589.	485280.	60000.	866.8
2	18	26584.		3.2		27177.	1.000	13589.	485286.	60000.	899.7
18	38	26584.		3.2		27177.	1.000	13589.	485286.	60000.	899.7
31	43	26861.		3.6		27177.	1.000	13588.	485321.	54348.	766.8
24	45	27103.		0.5		27105.	1.000	13552.	491227.	60000.	853.1
22	23	27100.		0.8		27103.	1.000	13551.	491420.	54348.	804.7
17	25	20635.		76.6		27084.	1.000	13542.	492958.	53580.	798.3
20	33	27026.		14.4		27046.	1.000	13523.	496131.	54348.	819.2
12	27	25319.		13.3		26996.	1.000	13498.	500727.	58680.	843.2
3	31	26861.		26.6		26889.	1.000	13445.	518868.	54348.	766.8
21	31	26861.		27.2		26889.	1.000	13445.	518868.	54348.	766.8
31	34	26861.		26.6		26889.	1.000	13445.	518868.	54348.	766.8
20	27	26676.		4.8		26872.	1.000	13436.	521787.	54348.	792.1
7	24	26861.		1.2		26862.	1.000	13431.	523582.	60000.	840.6
30	31	26821.		24.8		26850.	1.000	13425.	525802.	54348.	792.1
15	20	21992.		26.2		26795.	1.000	13398.	535458.	54348.	801.4
12	30	25177.		10.3		26753.	1.000	13376.	543158.	58680.	843.2
11	43	26180.		24.9		26738.	1.000	13369.	545912.	54348.	801.4
27	31	26681.		21.8		26709.	1.000	13355.	551258.	54348.	792.1
25	33	26682.		27.1		26696.	1.000	13348.	553668.	53580.	816.1
2	19	26645.		39.7		26662.	1.000	13331.	560023.	58680.	868.4
19	38	26645.		39.7		26662.	1.000	13331.	560023.	58680.	868.4
1	33	25744.		14.8		26654.	1.000	13327.	561655.	60000.	889.7
1	5	11502.		78.2		26550.	1.000	13275.	581619.	60000.	905.8
13	43	26180.		20.3		26495.	1.000	13248.	592663.	54348.	826.0
43	44	26148.		23.6		26464.	1.000	13232.	599029.	54348.	817.3

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15	30	21583.		28.0		26463.	1.000	13232.	599165.	58680.	822.5
15	25	21645.		67.8		26444.	1.000	13222.	603148.	53580.	798.3
11	34	26168.		1.8		26418.	1.000	13209.	608383.	54348.	801.4
3	11	26159.		1.8		26406.	1.000	13203.	610902.	54348.	801.4
9	23	26247.		0.2		26328.	1.000	13164.	627410.	60000.	833.8
28	31	26306.		4.2		26327.	1.000	13163.	627701.	53580.	763.1
5	43	25895.		54.9		26211.	1.000	13106.	653074.	54348.	827.0
17	23	26105.		33.1		26185.	1.000	13093.	658887.	60000.	833.8
13	34	26168.		2.8		26175.	1.000	13088.	661145.	54348.	826.0
3	13	26159.		2.8		26163.	1.000	13082.	663907.	54348.	826.0
34	44	26137.		0.5		26144.	1.000	13072.	668334.	54348.	817.3
3	44	26128.		0.5		26132.	1.000	13066.	671130.	54348.	817.3
5	25	21935.		36.4		26033.	1.000	13017.	694402.	53580.	823.9
5	34	25884.		78.0		25891.	1.000	12946.	729405.	54348.	827.0
3	5	25875.		78.0		25879.	1.000	12940.	732486.	54348.	827.0
16	23	23283.		14.4		25797.	1.000	12899.	753773.	60000.	833.8
7	27	25238.		7.0		25752.	1.000	12876.	765629.	58680.	822.5
20	30	25623.		1.8		25719.	1.000	12859.	774672.	54348.	792.1
12	37	24207.		7.9		25687.	1.000	12843.	783405.	60000.	888.0
23	35	25532.		33.1		25613.	1.000	12807.	803942.	60000.	833.8
5	12	9038.		86.5		25567.	1.000	12783.	817215.	60000.	870.6
3	43	25130.		23.1		25446.	1.000	12723.	852773.	54348.	766.8
21	43	25130.		23.6		25446.	1.000	12723.	852773.	54348.	766.8
34	43	25130.		23.0		25446.	1.000	12723.	852773.	54348.	766.8
27	45	25182.		5.3		25379.	1.000	12690.	873244.	58680.	836.3
19	27	25071.		34.3		25268.	1.000	12634.	908503.	58680.	813.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 (
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
18	27	24462.		8.5		25252.	1.000	12626.	913613.	58680.	841.7
23	39	25107.		1.4		25188.	1.000	12594.	934874.	60000.	833.8
1	20	25126.		0.3		25146.	1.000	12573.	948882.	54348.	851.7
3	34	25119.		0.0		25126.	1.000	12563.	955635.	54348.	766.8
21	34	25119.		0.5		25126.	1.000	12563.	955635.	54348.	766.8
3	21	25110.		0.5		25114.	1.000	12557.	959796.	54348.	766.8
18	30	24320.		5.5		25009.	1.000	12505.	996650.	58680.	841.7
23	45	24883.		1.4		24963.	1.000	12482.	>1000000.	60000.	846.3
20	25	24601.		41.6		24621.	1.000	12310.	>1000000.	53580.	763.1
40	42	24388.		29.2		24612.	1.000	12306.	>1000000.	60000.	846.7
16	24	22175.		13.5		24609.	1.000	12305.	>1000000.	60000.	840.6
7	23	24523.		0.3		24604.	1.000	12302.	>1000000.	60000.	833.8
7	30	24186.		4.1		24599.	1.000	12300.	>1000000.	58680.	822.5
39	42	24075.		29.2		24300.	1.000	12150.	>1000000.	60000.	846.7
35	42	17634.		63.6		24292.	1.000	12146.	>1000000.	60000.	846.7
30	45	24139.		2.4		24235.	1.000	12118.	>1000000.	58680.	836.3
12	19	22716.		47.6		24214.	1.000	12107.	>1000000.	58680.	843.2
19	30	24018.		37.3		24114.	1.000	12057.	>1000000.	58680.	813.2
18	37	23350.		3.2		23943.	1.000	11972.	>1000000.	60000.	880.1
9	27	23503.		6.5		23914.	1.000	11957.	>1000000.	58680.	822.5
5	18	8182.		81.7		23823.	1.000	11912.	>1000000.	60000.	869.1
17	42	17061.		63.6		23720.	1.000	11860.	>1000000.	60000.	846.7
8	23	23283.		3.1		23681.	1.000	11841.	>1000000.	60000.	833.8
5	40	8626.		78.5		23674.	1.000	11837.	>1000000.	60000.	851.3
41	42	23398.		29.2		23623.	1.000	11811.	>1000000.	60000.	846.7
6	42	23398.		27.8		23622.	1.000	11811.	>1000000.	60000.	846.7

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [1

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
8	42	23398.		27.5		23622.	1.000	11811.	>1000000.	60000.	846.7
10	42	23398.		28.0		23622.	1.000	11811.	>1000000.	60000.	846.7
14	42	23398.		27.8		23622.	1.000	11811.	>1000000.	60000.	846.7
16	42	23398.		16.1		23622.	1.000	11811.	>1000000.	60000.	846.7
6	23	23283.		2.7		23616.	1.000	11808.	>1000000.	60000.	833.8
14	23	23283.		2.7		23607.	1.000	11803.	>1000000.	60000.	833.8
10	23	23283.		2.5		23579.	1.000	11789.	>1000000.	60000.	833.8
2	17	17119.		34.4		23553.	1.000	11777.	>1000000.	60000.	875.7
17	36	17119.		31.3		23553.	1.000	11777.	>1000000.	60000.	849.0
17	38	17119.		34.4		23553.	1.000	11777.	>1000000.	60000.	875.7
33	39	22569.		15.0		23479.	1.000	11740.	>1000000.	60000.	845.2
19	37	23411.		39.7		23428.	1.000	11714.	>1000000.	58680.	819.8
23	41	23283.		1.4		23364.	1.000	11682.	>1000000.	60000.	833.8
5	39	8313.		78.5		23362.	1.000	11681.	>1000000.	60000.	851.3
24	37	23355.		0.5		23356.	1.000	11678.	>1000000.	60000.	851.8
5	35	1871.		113.0		23354.	1.000	11677.	>1000000.	60000.	851.3
23	40	23146.		1.4		23227.	1.000	11613.	>1000000.	60000.	833.8
27	33	22925.		9.7		23122.	1.000	11561.	>1000000.	58680.	837.9
11	27	22635.		6.6		23075.	1.000	11537.	>1000000.	58680.	822.5
20	21	23025.		0.6		23045.	1.000	11522.	>1000000.	54348.	766.8
2	35	16547.		34.4		22981.	1.000	11491.	>1000000.	60000.	875.7
35	36	16547.		31.3		22981.	1.000	11491.	>1000000.	60000.	849.0
35	38	16547.		34.4		22981.	1.000	11491.	>1000000.	60000.	875.7
2	15	18127.		25.7		22911.	1.000	11456.	>1000000.	60000.	875.7
15	36	18127.		22.5		22911.	1.000	11456.	>1000000.	60000.	849.0
15	38	18127.		25.7		22911.	1.000	11456.	>1000000.	60000.	875.7

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1, ASME-1998 KRE 2007/07/11 11:35:07

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
17	33	15555.		49.5		22900.	1.000	11450.	>1000000.	60000.	845.2
13	27	22635.		2.0		22832.	1.000	11416.	>1000000.	58680.	843.2
33	41	21891.		15.0		22801.	1.000	11400.	>1000000.	60000.	845.2
6	33	21891.		13.7		22801.	1.000	11400.	>1000000.	60000.	845.2
8	33	21891.		13.3		22801.	1.000	11400.	>1000000.	60000.	845.2
10	33	21891.		13.9		22801.	1.000	11400.	>1000000.	60000.	845.2
14	33	21891.		13.7		22801.	1.000	11400.	>1000000.	60000.	845.2
16	33	21891.		2.0		22801.	1.000	11400.	>1000000.	60000.	845.2
5	17	1299.		113.0		22783.	1.000	11391.	>1000000.	60000.	851.3
9	30	22450.		3.5		22761.	1.000	11381.	>1000000.	58680.	822.5
21	25	22681.		42.2		22695.	1.000	11348.	>1000000.	53580.	763.1
5	41	7636.		78.5		22685.	1.000	11342.	>1000000.	60000.	851.3
5	6	7636.		77.2		22685.	1.000	11342.	>1000000.	60000.	851.3
5	8	7636.		76.8		22685.	1.000	11342.	>1000000.	60000.	851.3
5	10	7636.		77.4		22685.	1.000	11342.	>1000000.	60000.	851.3
5	14	7636.		77.2		22685.	1.000	11342.	>1000000.	60000.	851.3
5	16	7636.		65.5		22685.	1.000	11342.	>1000000.	60000.	851.3
33	40	21767.		15.0		22677.	1.000	11339.	>1000000.	60000.	845.2
20	44	22548.		0.6		22567.	1.000	11284.	>1000000.	54348.	817.3
5	27	22346.		73.2		22543.	1.000	11271.	>1000000.	58680.	844.0
8	24	22175.		2.2		22493.	1.000	11247.	>1000000.	60000.	840.6
27	44	22291.		5.3		22488.	1.000	11244.	>1000000.	58680.	836.3
18	19	21860.		42.9		22470.	1.000	11235.	>1000000.	58680.	841.7
6	24	22175.		1.8		22428.	1.000	11214.	>1000000.	60000.	840.6
14	24	22175.		1.8		22419.	1.000	11210.	>1000000.	60000.	840.6
10	24	22175.		1.6		22391.	1.000	11195.	>1000000.	60000.	840.6

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [1

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
5	20	22281.		77.9		22301.	1.000	11150.	>1000000.	54348.	827.0
20	40	22243.		0.6		22263.	1.000	11132.	>1000000.	54348.	801.4
11	20	21992.		1.9		22254.	1.000	11127.	>1000000.	54348.	801.4
24	41	22175.		0.5		22176.	1.000	11088.	>1000000.	60000.	840.6
25	44	22161.		42.2		22175.	1.000	11088.	>1000000.	53580.	814.2
1	12	20527.		8.2		22056.	1.000	11028.	>1000000.	60000.	903.8
25	36	21427.		45.3		22035.	1.000	11017.	>1000000.	53580.	821.0
13	20	21992.		2.8		22011.	1.000	11006.	>1000000.	54348.	826.0
30	33	21873.		12.6		21969.	1.000	10984.	>1000000.	58680.	837.9
20	39	21947.		0.6		21967.	1.000	10983.	>1000000.	54348.	793.6
11	30	21583.		3.7		21922.	1.000	10961.	>1000000.	58680.	822.5
11	25	21645.		43.5		21902.	1.000	10951.	>1000000.	53580.	798.3
27	42	21615.		23.8		21812.	1.000	10906.	>1000000.	58680.	839.4
21	27	21600.		5.3		21797.	1.000	10898.	>1000000.	54348.	792.1
35	37	15300.		34.4		21734.	1.000	10867.	>1000000.	60000.	833.7
13	30	21583.		1.0		21679.	1.000	10840.	>1000000.	58680.	843.2
13	25	21645.		38.8		21660.	1.000	10830.	>1000000.	53580.	822.9
30	42	21473.		26.8		21569.	1.000	10784.	>1000000.	58680.	839.4
12	33	19125.		23.0		21514.	1.000	10757.	>1000000.	60000.	864.5
33	35	14141.		49.5		21486.	1.000	10743.	>1000000.	60000.	845.2
2	25	21427.		42.2		21442.	1.000	10721.	>1000000.	53580.	849.3
25	38	21427.		42.2		21442.	1.000	10721.	>1000000.	53580.	849.3
5	30	21294.		76.1		21390.	1.000	10695.	>1000000.	58680.	844.0
9	20	21124.		1.7		21358.	1.000	10679.	>1000000.	54348.	801.4
30	44	21261.		2.4		21358.	1.000	10679.	>1000000.	58680.	836.3
20	41	21254.		0.6		21274.	1.000	10637.	>1000000.	54348.	801.4

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
6	20	21254.		0.8		21274.	1.000	10637.	>1000000.	54348.	801.4
8	20	21254.		1.1		21274.	1.000	10637.	>1000000.	54348.	801.4
10	20	21254.		0.6		21274.	1.000	10637.	>1000000.	54348.	801.4
14	20	21254.		0.7		21274.	1.000	10637.	>1000000.	54348.	801.4
16	20	21254.		12.5		21274.	1.000	10637.	>1000000.	54348.	801.4
24	40	21195.		0.5		21197.	1.000	10598.	>1000000.	60000.	840.6
15	42	16052.		54.8		21060.	1.000	10530.	>1000000.	60000.	846.7
9	25	20777.		43.3		21006.	1.000	10503.	>1000000.	53580.	798.3
21	30	20547.		2.4		20643.	1.000	10321.	>1000000.	54348.	792.1
12	16	16674.		21.0		20587.	1.000	10294.	>1000000.	60000.	850.5
5	42	15762.		49.4		20384.	1.000	10192.	>1000000.	60000.	866.8
17	37	13885.		34.4		20319.	1.000	10160.	>1000000.	60000.	833.7
1	18	19670.		3.4		20313.	1.000	10156.	>1000000.	60000.	899.7
19	35	13795.		74.1		20247.	1.000	10124.	>1000000.	58680.	822.5
15	33	14547.		40.7		20241.	1.000	10121.	>1000000.	60000.	845.2
5	15	289.		104.2		20122.	1.000	10061.	>1000000.	60000.	851.3
20	45	19910.		0.6		19930.	1.000	9965.	>1000000.	54348.	817.3
5	33	14258.		63.5		19800.	1.000	9900.	>1000000.	60000.	865.3
18	33	18268.		18.2		19771.	1.000	9885.	>1000000.	60000.	863.0
1	19	19731.		39.4		19748.	1.000	9874.	>1000000.	58680.	868.4
21	36	19151.		3.2		19744.	1.000	9872.	>1000000.	54348.	824.1
7	20	19391.		2.3		19728.	1.000	9864.	>1000000.	54348.	801.4
1	24	19674.		0.7		19726.	1.000	9863.	>1000000.	60000.	885.1
15	37	14893.		25.7		19677.	1.000	9839.	>1000000.	60000.	833.7
19	20	19553.		39.1		19573.	1.000	9786.	>1000000.	54348.	792.1
27	30	19370.		3.0		19567.	1.000	9784.	>1000000.	58680.	813.2

Notes b,d,e,k: Fails
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L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2	23	19336.		1.4		19416.	1.000	9708.	>1000000.	60000.	878.3
23	24	19336.		0.9		19416.	1.000	9708.	>1000000.	60000.	843.2
23	36	19336.		1.8		19416.	1.000	9708.	>1000000.	60000.	851.6
23	38	19336.		1.4		19416.	1.000	9708.	>1000000.	60000.	878.3
7	25	19041.		43.9		19373.	1.000	9686.	>1000000.	53580.	798.3
13	36	18127.		6.5		19339.	1.000	9670.	>1000000.	60000.	868.4
25	45	19292.		42.2		19306.	1.000	9653.	>1000000.	53580.	814.2
36	44	18638.		3.2		19231.	1.000	9616.	>1000000.	60000.	861.5
19	25	19209.		2.5		19227.	1.000	9613.	>1000000.	53580.	788.7
7	42	18656.		30.9		19197.	1.000	9599.	>1000000.	60000.	846.7
2	21	19151.		0.0		19151.	1.000	9576.	>1000000.	54348.	851.7
21	38	19151.		0.0		19151.	1.000	9576.	>1000000.	54348.	851.7
12	40	17663.		7.9		19143.	1.000	9572.	>1000000.	60000.	850.5
19	42	19013.		10.5		19030.	1.000	9515.	>1000000.	58680.	839.4
16	18	15818.		16.2		18843.	1.000	9422.	>1000000.	60000.	849.0
17	19	12380.		74.1		18832.	1.000	9416.	>1000000.	58680.	822.5
12	39	17350.		7.9		18830.	1.000	9415.	>1000000.	60000.	850.5
42	45	18564.		29.2		18789.	1.000	9394.	>1000000.	60000.	859.2
2	13	18127.		3.3		18746.	1.000	9373.	>1000000.	60000.	903.8
13	38	18127.		3.3		18746.	1.000	9373.	>1000000.	60000.	903.8
11	36	18127.		1.9		18720.	1.000	9360.	>1000000.	60000.	849.0
2	44	18638.		0.0		18638.	1.000	9319.	>1000000.	60000.	888.2
38	44	18638.		0.0		18638.	1.000	9319.	>1000000.	60000.	888.2
8	12	16674.		9.6		18471.	1.000	9235.	>1000000.	60000.	850.5
19	33	18426.		24.7		18443.	1.000	9222.	>1000000.	58680.	837.9
6	12	16674.		9.3		18406.	1.000	9203.	>1000000.	60000.	850.5

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
12	14	16674.		9.2		18397.	1.000	9198.	>1000000.	60000.	850.5
7	33	17149.		16.7		18376.	1.000	9188.	>1000000.	60000.	845.2
2	11	18127.		1.3		18370.	1.000	9185.	>1000000.	60000.	875.7
11	38	18127.		1.3		18370.	1.000	9185.	>1000000.	60000.	875.7
10	12	16674.		9.1		18368.	1.000	9184.	>1000000.	60000.	850.5
5	7	2894.		80.2		18259.	1.000	9130.	>1000000.	60000.	851.3
15	19	13390.		65.3		18192.	1.000	9096.	>1000000.	58680.	822.5
12	41	16674.		7.9		18154.	1.000	9077.	>1000000.	60000.	850.5
1	35	11620.		34.7		18104.	1.000	9052.	>1000000.	60000.	875.7
33	45	17108.		15.0		18018.	1.000	9009.	>1000000.	60000.	857.7
5	45	2922.		78.5		17971.	1.000	8985.	>1000000.	60000.	863.8
9	36	17262.		2.0		17855.	1.000	8927.	>1000000.	60000.	849.0
5	19	13679.		38.8		17777.	1.000	8888.	>1000000.	58680.	844.0
2	9	17262.		1.2		17476.	1.000	8738.	>1000000.	60000.	875.7
9	38	17262.		1.2		17476.	1.000	8738.	>1000000.	60000.	875.7
18	40	16807.		3.2		17400.	1.000	8700.	>1000000.	60000.	849.0
9	42	16920.		30.3		17359.	1.000	8679.	>1000000.	60000.	846.7
12	35	10909.		26.5		17343.	1.000	8672.	>1000000.	60000.	850.5
18	39	16494.		3.2		17087.	1.000	8543.	>1000000.	60000.	849.0
19	40	16851.		39.7		16868.	1.000	8434.	>1000000.	58680.	822.5
12	17	10337.		26.5		16772.	1.000	8386.	>1000000.	60000.	850.5
8	18	15818.		4.9		16727.	1.000	8364.	>1000000.	60000.	849.0
1	17	10205.		34.7		16689.	1.000	8344.	>1000000.	60000.	875.7
6	18	15818.		4.5		16662.	1.000	8331.	>1000000.	60000.	849.0
14	18	15818.		4.5		16653.	1.000	8327.	>1000000.	60000.	849.0
10	18	15818.		4.3		16625.	1.000	8312.	>1000000.	60000.	849.0

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREE
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
16	35	7752.		47.5		16619.	1.000	8309.	>1000000.	60000.	831.2
25	39	16565.		42.2		16580.	1.000	8290.	>1000000.	53580.	787.5
33	42	16341.		14.1		16565.	1.000	8283.	>1000000.	60000.	860.7
19	39	16541.		39.7		16559.	1.000	8280.	>1000000.	58680.	822.5
36	39	15947.		3.2		16540.	1.000	8270.	>1000000.	60000.	849.0
9	33	15414.		16.2		16539.	1.000	8269.	>1000000.	60000.	845.2
11	42	16052.		30.5		16519.	1.000	8259.	>1000000.	60000.	846.7
18	35	10053.		31.3		16487.	1.000	8244.	>1000000.	60000.	849.0
5	9	1158.		79.7		16421.	1.000	8210.	>1000000.	60000.	851.3
18	41	15818.		3.2		16410.	1.000	8205.	>1000000.	60000.	849.0
36	45	15775.		3.2		16368.	1.000	8184.	>1000000.	60000.	861.5
13	42	16052.		25.8		16276.	1.000	8138.	>1000000.	60000.	866.1
7	36	15533.		1.5		16126.	1.000	8063.	>1000000.	60000.	849.0
1	15	11213.		25.9		16047.	1.000	8023.	>1000000.	60000.	875.7
5	44	964.		78.5		16013.	1.000	8007.	>1000000.	60000.	863.8
2	39	15947.		0.0		15947.	1.000	7974.	>1000000.	60000.	875.7
38	39	15947.		0.0		15947.	1.000	7974.	>1000000.	60000.	875.7
21	37	15918.		0.0		15918.	1.000	7959.	>1000000.	54348.	788.7
17	18	9481.		31.3		15915.	1.000	7958.	>1000000.	60000.	849.0
42	44	15657.		29.2		15881.	1.000	7940.	>1000000.	60000.	859.2
19	41	15861.		39.7		15879.	1.000	7940.	>1000000.	58680.	822.5
6	19	15861.		38.3		15879.	1.000	7939.	>1000000.	58680.	822.5
8	19	15861.		38.0		15879.	1.000	7939.	>1000000.	58680.	822.5
10	19	15861.		38.5		15879.	1.000	7939.	>1000000.	58680.	822.5
14	19	15861.		38.4		15879.	1.000	7939.	>1000000.	58680.	822.5
16	19	15861.		26.6		15879.	1.000	7939.	>1000000.	58680.	822.5

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2	7	15533.		1.7		15850.	1.000	7925.	>1000000.	60000.	875.7
7	38	15533.		1.7		15850.	1.000	7925.	>1000000.	60000.	875.7
12	42	14132.		37.1		15836.	1.000	7918.	>1000000.	60000.	866.1
5	21	765.		78.5		15814.	1.000	7907.	>1000000.	54348.	827.0
2	45	15775.		0.0		15775.	1.000	7888.	>1000000.	60000.	888.2
38	45	15775.		0.0		15775.	1.000	7888.	>1000000.	60000.	888.2
11	33	14547.		16.3		15700.	1.000	7850.	>1000000.	60000.	845.2
5	11	289.		79.8		15581.	1.000	7790.	>1000000.	60000.	851.3
13	37	14893.		3.3		15512.	1.000	7756.	>1000000.	60000.	888.0
13	33	14547.		11.7		15457.	1.000	7729.	>1000000.	60000.	864.5
37	44	15404.		0.0		15404.	1.000	7702.	>1000000.	60000.	857.6
25	37	15333.		42.2		15347.	1.000	7673.	>1000000.	53580.	784.2
5	13	289.		75.2		15338.	1.000	7669.	>1000000.	60000.	870.6
21	42	15018.		29.2		15242.	1.000	7621.	>1000000.	54348.	821.2
16	17	6337.		47.5		15204.	1.000	7602.	>1000000.	60000.	831.2
33	44	14277.		15.0		15187.	1.000	7594.	>1000000.	60000.	857.7
11	37	14893.		1.3		15136.	1.000	7568.	>1000000.	60000.	833.7
35	39	8429.		34.4		14863.	1.000	7431.	>1000000.	60000.	831.2
15	16	7347.		38.7		14564.	1.000	7282.	>1000000.	60000.	831.2
8	35	7752.		36.1		14503.	1.000	7251.	>1000000.	60000.	831.2
19	21	14425.		39.7		14443.	1.000	7221.	>1000000.	54348.	792.1
6	35	7752.		35.8		14438.	1.000	7219.	>1000000.	60000.	831.2
25	40	14423.		42.2		14437.	1.000	7218.	>1000000.	53580.	798.3
14	35	7752.		35.8		14429.	1.000	7214.	>1000000.	60000.	831.2
21	33	13510.		15.0		14420.	1.000	7210.	>1000000.	54348.	819.2
10	35	7752.		35.6		14400.	1.000	7200.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07

Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
25	41	14299.		42.2		14314.	1.000	7157.	>1000000.	53580.	798.3
6	25	14299.		40.8		14314.	1.000	7157.	>1000000.	53580.	798.3
8	25	14299.		40.5		14314.	1.000	7157.	>1000000.	53580.	798.3
10	25	14299.		41.0		14314.	1.000	7157.	>1000000.	53580.	798.3
14	25	14299.		40.8		14314.	1.000	7157.	>1000000.	53580.	798.3
16	25	14299.		29.1		14314.	1.000	7157.	>1000000.	53580.	798.3
9	37	14028.		1.2		14242.	1.000	7121.	>1000000.	60000.	833.7
35	41	7752.		34.4		14186.	1.000	7093.	>1000000.	60000.	831.2
12	15	9327.		17.7		14111.	1.000	7056.	>1000000.	60000.	850.5
18	42	13275.		32.3		14092.	1.000	7046.	>1000000.	60000.	864.5
35	40	7628.		34.4		14063.	1.000	7031.	>1000000.	60000.	831.2
19	44	13999.		39.7		14017.	1.000	7008.	>1000000.	58680.	836.3
16	36	10847.		16.2		13873.	1.000	6936.	>1000000.	60000.	849.0
17	40	7326.		34.4		13761.	1.000	6880.	>1000000.	60000.	831.2
11	19	13390.		41.0		13651.	1.000	6825.	>1000000.	58680.	822.5
17	39	7014.		34.4		13448.	1.000	6724.	>1000000.	60000.	831.2
7	12	11932.		6.2		13411.	1.000	6706.	>1000000.	60000.	850.5
13	19	13390.		36.3		13408.	1.000	6704.	>1000000.	58680.	843.2
12	45	11813.		7.9		13293.	1.000	6647.	>1000000.	60000.	863.0
2	16	10847.		13.1		13280.	1.000	6640.	>1000000.	60000.	875.7
16	38	10847.		13.1		13280.	1.000	6640.	>1000000.	60000.	875.7
15	18	8471.		22.5		13255.	1.000	6627.	>1000000.	60000.	849.0
15	40	8336.		25.7		13120.	1.000	6560.	>1000000.	60000.	831.2
1	25	13083.		41.9		13097.	1.000	6549.	>1000000.	53580.	849.3
8	17	6337.		36.2		13088.	1.000	6544.	>1000000.	60000.	831.2
6	17	6337.		35.8		13023.	1.000	6511.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
14	17	6337.		35.8		13014.	1.000	6507.	>1000000.	60000.	831.2
10	17	6337.		35.6		12986.	1.000	6493.	>1000000.	60000.	831.2
15	39	8023.		25.7		12807.	1.000	6404.	>1000000.	60000.	831.2
17	41	6337.		34.4		12771.	1.000	6386.	>1000000.	60000.	831.2
9	19	12522.		40.8		12755.	1.000	6377.	>1000000.	58680.	822.5
36	37	12027.		3.2		12620.	1.000	6310.	>1000000.	60000.	880.1
7	37	12299.		1.7		12616.	1.000	6308.	>1000000.	60000.	833.7
37	45	12541.		0.0		12541.	1.000	6271.	>1000000.	60000.	857.6
8	15	7347.		27.4		12448.	1.000	6224.	>1000000.	60000.	831.2
6	15	7347.		27.0		12382.	1.000	6191.	>1000000.	60000.	831.2
14	15	7347.		27.0		12373.	1.000	6187.	>1000000.	60000.	831.2
10	15	7347.		26.8		12345.	1.000	6173.	>1000000.	60000.	831.2
1	21	12237.		0.3		12287.	1.000	6144.	>1000000.	54348.	851.7
15	41	7347.		25.7		12131.	1.000	6065.	>1000000.	60000.	831.2
19	45	12049.		39.7		12066.	1.000	6033.	>1000000.	58680.	836.3
2	37	12027.		0.0		12027.	1.000	6013.	>1000000.	60000.	970.1
37	38	12027.		0.0		12027.	1.000	6013.	>1000000.	60000.	970.1
1	13	11213.		3.1		11832.	1.000	5916.	>1000000.	60000.	903.8
1	44	11724.		0.3		11774.	1.000	5887.	>1000000.	60000.	888.2
8	36	10847.		4.9		11757.	1.000	5878.	>1000000.	60000.	849.0
6	36	10847.		4.5		11691.	1.000	5846.	>1000000.	60000.	849.0
14	36	10847.		4.5		11682.	1.000	5841.	>1000000.	60000.	849.0
9	12	10196.		6.8		11676.	1.000	5838.	>1000000.	60000.	850.5
7	18	11075.		1.5		11668.	1.000	5834.	>1000000.	60000.	849.0
10	36	10847.		4.3		11654.	1.000	5827.	>1000000.	60000.	849.0
18	45	10961.		3.2		11553.	1.000	5777.	>1000000.	60000.	861.5

Notes b,d,e,k: Fails
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L: Information

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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREE
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
1	11	11213.		1.6		11506.	1.000	5753.	>1000000.	60000.	875.7
7	19	11134.		41.4		11469.	1.000	5734.	>1000000.	58680.	822.5
36	41	10847.		3.2		11440.	1.000	5720.	>1000000.	60000.	849.0
12	13	9327.		11.3		11426.	1.000	5713.	>1000000.	60000.	869.9
2	24	11328.		0.5		11330.	1.000	5665.	>1000000.	60000.	885.1
24	36	11328.		2.7		11330.	1.000	5665.	>1000000.	60000.	858.4
24	38	11328.		0.5		11330.	1.000	5665.	>1000000.	60000.	885.1
2	8	10847.		1.7		11164.	1.000	5582.	>1000000.	60000.	875.7
8	38	10847.		1.7		11164.	1.000	5582.	>1000000.	60000.	875.7
2	6	10847.		1.4		11098.	1.000	5549.	>1000000.	60000.	875.7
6	38	10847.		1.4		11098.	1.000	5549.	>1000000.	60000.	875.7
2	14	10847.		1.3		11090.	1.000	5545.	>1000000.	60000.	875.7
14	38	10847.		1.3		11090.	1.000	5545.	>1000000.	60000.	875.7
2	10	10847.		1.2		11061.	1.000	5531.	>1000000.	60000.	875.7
10	38	10847.		1.2		11061.	1.000	5531.	>1000000.	60000.	875.7
2	41	10847.		0.0		10847.	1.000	5423.	>1000000.	60000.	875.7
38	41	10847.		0.0		10847.	1.000	5423.	>1000000.	60000.	875.7
16	21	8383.		13.1		10816.	1.000	5408.	>1000000.	54348.	801.4
11	12	9327.		6.6		10807.	1.000	5404.	>1000000.	60000.	850.5
1	9	10348.		1.4		10612.	1.000	5306.	>1000000.	60000.	875.7
36	40	9867.		3.2		10460.	1.000	5230.	>1000000.	60000.	849.0
12	44	8904.		7.9		10384.	1.000	5192.	>1000000.	60000.	863.0
16	44	7878.		13.1		10311.	1.000	5156.	>1000000.	60000.	843.7
16	37	7613.		13.1		10046.	1.000	5023.	>1000000.	60000.	833.7
11	16	7347.		14.4		10023.	1.000	5011.	>1000000.	60000.	831.2
9	18	9339.		2.0		9932.	1.000	4966.	>1000000.	60000.	849.0

Notes b,d,e,k: Fails
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L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2	40	9867.		0.0		9867.	1.000	4934.	>1000000.	60000.	875.7
38	40	9867.		0.0		9867.	1.000	4934.	>1000000.	60000.	875.7
13	16	7347.		9.7		9780.	1.000	4890.	>1000000.	60000.	850.5
12	21	8294.		7.9		9774.	1.000	4887.	>1000000.	54348.	826.0
13	18	8471.		6.5		9683.	1.000	4841.	>1000000.	60000.	868.4
35	45	3119.		34.4		9554.	1.000	4777.	>1000000.	60000.	843.7
7	35	3009.		32.7		9443.	1.000	4722.	>1000000.	60000.	831.2
21	40	9372.		0.0		9373.	1.000	4686.	>1000000.	54348.	801.4
9	16	6478.		14.2		9126.	1.000	4563.	>1000000.	60000.	831.2
11	18	8471.		1.9		9064.	1.000	4532.	>1000000.	60000.	849.0
21	39	9063.		0.0		9063.	1.000	4531.	>1000000.	54348.	793.6
21	35	2626.		34.4		9060.	1.000	4530.	>1000000.	54348.	801.4
1	36	8346.		3.4		8989.	1.000	4494.	>1000000.	60000.	899.7
1	7	8619.		2.0		8985.	1.000	4493.	>1000000.	60000.	875.7
13	40	8336.		3.3		8955.	1.000	4478.	>1000000.	60000.	850.5
1	45	8861.		0.3		8911.	1.000	4455.	>1000000.	60000.	888.2
40	44	8864.		0.0		8865.	1.000	4432.	>1000000.	60000.	843.7
35	44	2416.		34.4		8850.	1.000	4425.	>1000000.	60000.	843.7
8	21	8383.		1.7		8700.	1.000	4350.	>1000000.	54348.	801.4
18	44	8053.		3.2		8646.	1.000	4323.	>1000000.	60000.	861.5
13	39	8023.		3.3		8643.	1.000	4321.	>1000000.	60000.	850.5
13	35	1582.		37.8		8635.	1.000	4318.	>1000000.	60000.	850.5
6	21	8383.		1.4		8635.	1.000	4317.	>1000000.	54348.	801.4
14	21	8383.		1.3		8626.	1.000	4313.	>1000000.	54348.	801.4
10	21	8383.		1.2		8598.	1.000	4299.	>1000000.	54348.	801.4
11	40	8336.		1.3		8579.	1.000	4289.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
39	44	8578.		0.0		8579.	1.000	4289.	>1000000.	60000.	843.7
17	21	2051.		34.4		8485.	1.000	4243.	>1000000.	54348.	801.4
1	2	8346.		0.3		8396.	1.000	4198.	>1000000.	60000.	970.1
1	38	8346.		0.3		8396.	1.000	4198.	>1000000.	60000.	970.1
21	41	8383.		0.0		8383.	1.000	4192.	>1000000.	54348.	801.4
11	39	8023.		1.3		8266.	1.000	4133.	>1000000.	60000.	831.2
17	45	1772.		34.4		8206.	1.000	4103.	>1000000.	60000.	843.7
8	44	7878.		1.7		8195.	1.000	4098.	>1000000.	60000.	843.7
17	44	1760.		34.4		8194.	1.000	4097.	>1000000.	60000.	843.7
6	44	7878.		1.4		8130.	1.000	4065.	>1000000.	60000.	843.7
14	44	7878.		1.3		8121.	1.000	4060.	>1000000.	60000.	843.7
10	44	7878.		1.2		8093.	1.000	4046.	>1000000.	60000.	843.7
13	17	1010.		37.8		8064.	1.000	4032.	>1000000.	60000.	850.5
18	21	7438.		3.2		8031.	1.000	4015.	>1000000.	54348.	824.1
7	17	1594.		32.7		8029.	1.000	4014.	>1000000.	60000.	831.2
11	35	1582.		33.1		8016.	1.000	4008.	>1000000.	60000.	831.2
15	35	1582.		8.8		8016.	1.000	4008.	>1000000.	60000.	831.2
13	41	7347.		3.3		7966.	1.000	3983.	>1000000.	60000.	850.5
6	13	7347.		2.0		7966.	1.000	3983.	>1000000.	60000.	850.5
8	13	7347.		1.6		7966.	1.000	3983.	>1000000.	60000.	850.5
10	13	7347.		2.2		7966.	1.000	3983.	>1000000.	60000.	850.5
13	14	7347.		2.0		7966.	1.000	3983.	>1000000.	60000.	850.5
8	37	7613.		1.7		7930.	1.000	3965.	>1000000.	60000.	833.7
8	11	7347.		3.0		7906.	1.000	3953.	>1000000.	60000.	831.2
41	44	7878.		0.0		7878.	1.000	3939.	>1000000.	60000.	843.7
6	37	7613.		1.4		7865.	1.000	3932.	>1000000.	60000.	833.7

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 [
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
14	37	7613.		1.3		7856.	1.000	3928.	>1000000.	60000.	833.7
17	35	1415.		0.0		7850.	1.000	3925.	>1000000.	60000.	831.2
6	11	7347.		2.7		7841.	1.000	3921.	>1000000.	60000.	831.2
11	14	7347.		2.6		7832.	1.000	3916.	>1000000.	60000.	831.2
10	37	7613.		1.2		7828.	1.000	3914.	>1000000.	60000.	833.7
10	11	7347.		2.5		7804.	1.000	3902.	>1000000.	60000.	831.2
37	40	7746.		0.0		7746.	1.000	3873.	>1000000.	60000.	833.7
9	35	1273.		33.3		7707.	1.000	3854.	>1000000.	60000.	831.2
9	40	7468.		1.2		7683.	1.000	3841.	>1000000.	60000.	831.2
1	39	7602.		0.3		7652.	1.000	3826.	>1000000.	60000.	875.7
37	41	7613.		0.0		7613.	1.000	3807.	>1000000.	60000.	833.7
11	41	7347.		1.3		7589.	1.000	3795.	>1000000.	60000.	831.2
16	39	5128.		13.1		7561.	1.000	3780.	>1000000.	60000.	831.2
7	16	4742.		14.8		7493.	1.000	3746.	>1000000.	60000.	831.2
16	45	5016.		13.1		7449.	1.000	3725.	>1000000.	60000.	843.7
11	17	1010.		33.1		7444.	1.000	3722.	>1000000.	60000.	831.2
15	17	1010.		8.8		7444.	1.000	3722.	>1000000.	60000.	831.2
15	45	2654.		25.7		7438.	1.000	3719.	>1000000.	60000.	843.7
7	15	2604.		24.0		7388.	1.000	3694.	>1000000.	60000.	831.2
9	39	7155.		1.2		7370.	1.000	3685.	>1000000.	60000.	831.2
8	9	6478.		2.9		7010.	1.000	3505.	>1000000.	60000.	831.2
6	9	6478.		2.5		6945.	1.000	3472.	>1000000.	60000.	831.2
9	14	6478.		2.5		6936.	1.000	3468.	>1000000.	60000.	831.2
37	39	6911.		0.0		6911.	1.000	3456.	>1000000.	60000.	833.7
9	10	6478.		2.3		6908.	1.000	3454.	>1000000.	60000.	831.2
9	41	6478.		1.2		6693.	1.000	3347.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
9	17	143.		33.3		6577.	1.000	3289.	>1000000.	60000.	831.2
1	16	3933.		12.8		6366.	1.000	3183.	>1000000.	60000.	875.7
39	40	6117.		0.0		6118.	1.000	3059.	>1000000.	60000.	831.2
7	40	5732.		1.7		6049.	1.000	3025.	>1000000.	60000.	831.2
40	45	5998.		0.0		5998.	1.000	2999.	>1000000.	60000.	843.7
15	44	1059.		25.7		5843.	1.000	2921.	>1000000.	60000.	843.7
15	21	1049.		25.7		5833.	1.000	2917.	>1000000.	54348.	801.4
39	45	5775.		0.0		5775.	1.000	2887.	>1000000.	60000.	843.7
7	39	5420.		1.7		5737.	1.000	2869.	>1000000.	60000.	831.2
9	15	868.		24.5		5652.	1.000	2826.	>1000000.	60000.	831.2
8	39	5128.		1.7		5445.	1.000	2722.	>1000000.	60000.	831.2
13	15	0.		29.0		5403.	1.000	2702.	>1000000.	60000.	850.5
6	39	5128.		1.4		5380.	1.000	2690.	>1000000.	60000.	831.2
7	8	4742.		3.4		5377.	1.000	2688.	>1000000.	60000.	831.2
14	39	5128.		1.3		5371.	1.000	2685.	>1000000.	60000.	831.2
10	39	5128.		1.2		5342.	1.000	2671.	>1000000.	60000.	831.2
8	45	5016.		1.7		5333.	1.000	2667.	>1000000.	60000.	843.7
6	7	4742.		3.1		5311.	1.000	2656.	>1000000.	60000.	831.2
7	14	4742.		3.0		5302.	1.000	2651.	>1000000.	60000.	831.2
7	10	4742.		2.9		5274.	1.000	2637.	>1000000.	60000.	831.2
6	45	5016.		1.4		5268.	1.000	2634.	>1000000.	60000.	843.7
14	45	5016.		1.3		5259.	1.000	2630.	>1000000.	60000.	843.7
10	45	5016.		1.2		5231.	1.000	2615.	>1000000.	60000.	843.7
39	41	5128.		0.0		5128.	1.000	2564.	>1000000.	60000.	831.2
7	41	4742.		1.7		5060.	1.000	2530.	>1000000.	60000.	831.2
41	45	5016.		0.0		5016.	1.000	2508.	>1000000.	60000.	843.7

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREE
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
11	15	0.		24.4		4784.	1.000	2392.	>1000000.	60000.	831.2
1	8	3933.		1.4		4250.	1.000	2125.	>1000000.	60000.	875.7
1	6	3933.		1.1		4184.	1.000	2092.	>1000000.	60000.	875.7
1	14	3933.		1.0		4176.	1.000	2088.	>1000000.	60000.	875.7
1	10	3933.		0.9		4147.	1.000	2074.	>1000000.	60000.	875.7
1	40	4066.		0.3		4116.	1.000	2058.	>1000000.	60000.	875.7
1	41	3933.		0.3		3983.	1.000	1991.	>1000000.	60000.	875.7
7	21	3643.		1.7		3960.	1.000	1980.	>1000000.	54348.	801.4
1	37	3680.		0.3		3730.	1.000	1865.	>1000000.	60000.	970.1
21	45	3666.		0.0		3666.	1.000	1833.	>1000000.	54348.	0.0
7	13	2604.		5.0		3540.	1.000	1770.	>1000000.	60000.	850.5
7	44	3221.		1.7		3538.	1.000	1769.	>1000000.	60000.	843.7
44	45	3528.		0.0		3528.	1.000	1764.	>1000000.	60000.	0.0
16	40	990.		13.1		3423.	1.000	1712.	>1000000.	60000.	831.2
13	45	2654.		3.3		3273.	1.000	1636.	>1000000.	60000.	863.0
7	11	2604.		0.4		2921.	1.000	1461.	>1000000.	60000.	831.2
11	45	2654.		1.3		2897.	1.000	1448.	>1000000.	60000.	843.7
16	41	0.		13.1		2433.	1.000	1217.	>1000000.	60000.	831.2
6	16	0.		11.7		2433.	1.000	1217.	>1000000.	60000.	831.2
8	16	0.		11.4		2433.	1.000	1217.	>1000000.	60000.	831.2
10	16	0.		11.9		2433.	1.000	1217.	>1000000.	60000.	831.2
14	16	0.		11.8		2433.	1.000	1217.	>1000000.	60000.	831.2
12	18	857.		4.8		2337.	1.000	1168.	>1000000.	60000.	868.4
9	21	1911.		1.2		2126.	1.000	1063.	>1000000.	54348.	801.4
9	45	1887.		1.2		2102.	1.000	1051.	>1000000.	60000.	843.7
7	9	1736.		0.5		2053.	1.000	1027.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREE
STRESSES IN PSI

LOAD SET PAIR		SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
9	44	1642.		1.2		1857.	1.000	929.	>1000000.	60000.	843.7
9	13	868.		4.5		1702.	1.000	851.	>1000000.	60000.	850.5
13	44	1059.		3.3		1678.	1.000	839.	>1000000.	60000.	863.0
13	21	1049.		3.3		1668.	1.000	834.	>1000000.	54348.	826.0
7	45	1057.		1.7		1374.	1.000	687.	>1000000.	60000.	843.7
8	40	990.		1.7		1307.	1.000	654.	>1000000.	60000.	831.2
11	44	1059.		1.3		1301.	1.000	651.	>1000000.	60000.	843.7
11	21	1049.		1.3		1292.	1.000	646.	>1000000.	54348.	801.4
6	40	990.		1.4		1242.	1.000	621.	>1000000.	60000.	831.2
14	40	990.		1.3		1233.	1.000	616.	>1000000.	60000.	831.2
10	40	990.		1.2		1205.	1.000	602.	>1000000.	60000.	831.2
9	11	868.		0.2		1111.	1.000	556.	>1000000.	60000.	831.2
21	44	1018.		0.0		1018.	1.000	509.	>1000000.	54348.	0.0
40	41	990.		0.0		990.	1.000	495.	>1000000.	60000.	831.2
11	13	0.		4.6		862.	1.000	431.	>1000000.	60000.	850.5
2	36	0.		3.2		593.	1.000	296.	>1000000.	60000.	899.7
36	38	0.		3.2		593.	1.000	296.	>1000000.	60000.	899.7
8	41	0.		1.7		317.	1.000	159.	>1000000.	60000.	831.2
6	8	0.		0.4		317.	1.000	159.	>1000000.	60000.	831.2
8	10	0.		0.5		317.	1.000	159.	>1000000.	60000.	831.2
8	14	0.		0.4		317.	1.000	159.	>1000000.	60000.	831.2
6	41	0.		1.4		252.	1.000	126.	>1000000.	60000.	831.2
6	10	0.		0.2		252.	1.000	126.	>1000000.	60000.	831.2
6	14	0.		0.0		252.	1.000	126.	>1000000.	60000.	831.2
14	41	0.		1.3		243.	1.000	121.	>1000000.	60000.	831.2
10	14	0.		0.2		243.	1.000	121.	>1000000.	60000.	831.2

Notes b,d,e,k: Fails
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07 (
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	SP EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
10 41	0.		1.2		215.	1.000	107.	>1000000.	60000.	831.2
2 38	0.		0.0		0.	1.000	0.	>1000000.	60000.	0.0

Notes
b,d,e,k: Fails
g: Weld ISI
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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREE
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	OCCURENCES NI	NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
28	29	66456.	10	10	10	28,29	1808.	0.0055	
			0	0					
31	32	54146.	10	10	10	31,32	3406.	0.0029	
			0	0					
4	43	49733.	610	300	300	43	4465.	0.0672	
			310	0					
4	34	49533.	310	10	10	34	4523.	0.0022	
			300	0					
3	4	49512.	300	300	300	3, 4	4529.	0.0662	
			0	0					
20	26	45555.	300	10	10	26	5838.	0.0017	
			290	0					
22	27	34161.	300	10	10	27	14370.	0.0007	
			290	0					
22	30	32373.	290	10	10	30	17257.	0.0006	
			280	0					
22	42	26170.	280	300	280	22	33637.	0.0083	
			0	20					
24	42	22860.	300	20	20	42	51538.	0.0004	
			280	0					
5	24	22397.	599	280	280	24	57040.	0.0049	
			319	0					
5	23	21265.	319	300	300	23	73771.	0.0041	
			19	0					
5	36	17029.	19	1	1	36	178513.	0.0000	
			18	0					
2	5	16732.	120	18	18	5	190176.	0.0001	
			102	0					
12	20	16409.	10	290	10	12	205131.	0.0000	
			0	280					

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	------ OCCURENCES ------ NI	NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
2	20	16030.	102	280	102	2	228208.	0.0004	
			0	178					
20	38	16030.	178	1	1	38	228208.	0.0000	
			177	0					
18	20	15537.	289	177	177	20	263206.	0.0007	
			112	0					
18	25	15362.	112	10	10	25	277200.	0.0000	
			102	0					
33	37	15167.	10	1	1	37	293826.	0.0000	
			9	0					
1	33	13327.	120	9	9	33	561655.	0.0000	
			111	0					
18	19	11235.	102	300	102	18	>1000000.	0.0000	
			0	198					
19	35	10124.	198	1	1	35	>1000000.	0.0000	
			197	0					
1	19	9874.	111	197	111	1	>1000000.	0.0000	
			0	86					
17	19	9416.	289	86	86	19	>1000000.	0.0000	
			203	0					
16	17	7602.	70	203	70	16	>1000000.	0.0000	
			0	133					
17	40	6880.	133	289	133	17	>1000000.	0.0000	
			0	156					
15	40	6560.	70	156	70	15	>1000000.	0.0000	
			0	86					
21	40	4686.	300	86	86	40	>1000000.	0.0000	
			214	0					
21	39	4531.	214	289	214	21	>1000000.	0.0000	
			0	75					

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Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

****
INDIVIDUAL STRESS RANGE CHECK
****

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR I	J	SALT EQN.14	OCCURENCES NI	NJ	NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
13	39	4321.	10	75	10	13	>1000000.	0.0000	
			0	65					
39	44	4289.	65	5	5	44	>1000000.	0.0000	
			60	0 45					
11	39	4133.	310	60	60	39	>1000000.	0.0000	
			250	0					
8	11	3953.	10000	250	250	11	>1000000.	0.0000	
			9750	0					
8	9	3505.	9750	2000	2000	9	>1000000.	0.0000	
			7750	0					
7	8	2688.	10000	7750	7750	8	>1000000.	0.0000	
			2250	0					
6	7	2656.	599	2250	599	6	>1000000.	0.0000	
			0	1651					
7	14	2651.	1651	10	10	14	>1000000.	0.0000	
			1641	0					
7	10	2637.	1641	2000	1641	7	>1000000.	0.0000	
			0	359					
10	45	2615.	359	5	5	45	>1000000.	0.0000	
			354	0 45					
44	45	1764.	45	45	45	,45 44	>1000000.	0.0000	DYN. RANGE OF EVENT NO.
			0	0					
10	41	107.	354	289	289	41	>1000000.	0.0000	
			65	0					

TOTAL USAGE FACTOR = 0.1661j

PROPRIETARY
REDACTED

EXHIBIT L

Report No.: SIR-07-130-NPS
Revision No.: 0
Project No.: VY-16Q
File No.: VY-16Q-401
July 2007

Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Feedwater Nozzles

NOTE

This document references vendor proprietary information. Such information is identified with -2xxP SI Project File numbers in the list of references. Any such references and the associated information in this document where those references are used are identified so that this information can be treated in accordance with applicable vendor proprietary agreements.

Prepared for:
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(Contract No. 10150394)

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Structural Integrity Associates, Inc.

REVISION CONTROL SHEET

Document Number: SIR-07-130-NPS

Title: Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Feedwater Nozzles

Client: Entergy Nuclear Vermont Yankee, LLC

SI Project Number: VY-16Q

Section	Pages	Revision	Date	Comments
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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee (VY) License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) value for the reactor pressure vessel (RPV) feedwater nozzle (FW) is reported as 0.750. Application of an environmentally assisted fatigue (EAF) multiplier, as required for the license renewal period, resulted in an unacceptable EAF CUF value of 2.86. Therefore, further refined analysis was necessitated to show acceptable EAF CUF results for this component.

REDACTED

The VY FW nozzles were re-evaluated in detail by SI in 2004 for EPU and 60 years of operation. However, that analysis used conservative transient definitions and cyclic projections for 60 years of operation that have since been updated as a part of LRA development.

This report documents a refined fatigue evaluation for the VY FW nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the FW nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [16] needs for EAF evaluation for license renewal. The resulting fatigue results will be used as a replacement to the values previously reported in the VY LRA.

This Page Contains Reference to Vendor Proprietary Information
(such information is marked with a "bar" in the right-hand margin)

The refined evaluation summarized in this report included development of a detailed finite element model of the FW nozzle, including relevant portions of the safe end, thermal sleeve, and the RPV wall. Thermal and pressure stress histories were developed for relevant transients affecting the FW nozzle, including any effects of EPU, as specified by the VY RPV Design Specification [3], the VY EPU Design Specification [17] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress and primary plus secondary stress for use in a subsequent fatigue evaluation. Stresses were also included due to loads from the attached piping for application in the stress/fatigue analysis based on the bounding reaction loads obtained from the relevant design documents. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code [15], and was performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

In order to provide an overall approach and strategy for evaluating the feedwater nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the feedwater nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [22] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [15]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress

are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges can be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, these are not required, as the stresses are calculated by finite element analysis and any applicable stress concentration factors. This generally results in a net reduction of the stress ranges and consequently, in the fatigue usage. Article 4 [27] methodology was originally used to evaluate the feedwater nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original feedwater nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve as accurate a CUF as reasonably achievable.

For the feedwater nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [4]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for all thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor feedwater nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the three limiting locations (two in the safe end and one in the nozzle forging, representing the three materials of the nozzle assembly) using the Green's Functions developed for the three feedwater flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility computer programs are used to facilitate the fatigue analysis process: STRESS.EXE, P V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [14] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

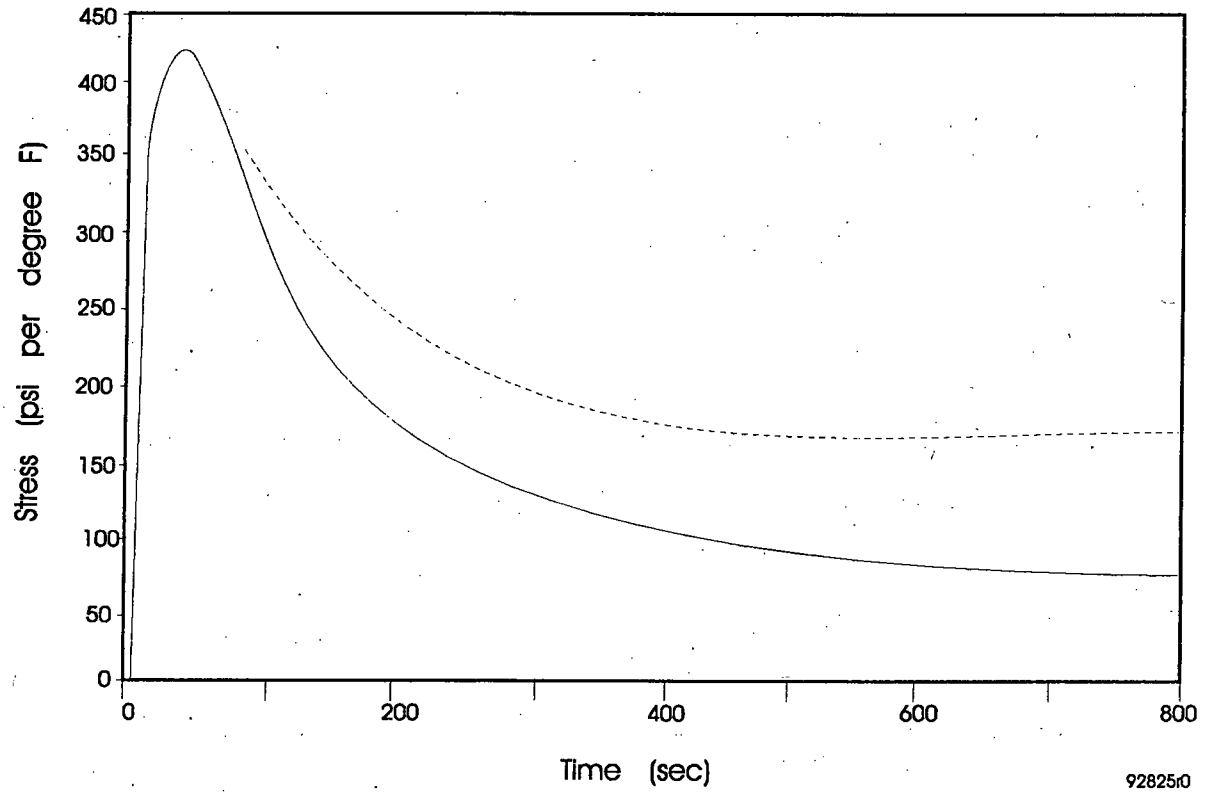
- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.

- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.
The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m, Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1. Typical Green's Functions for Thermal Transient Stress

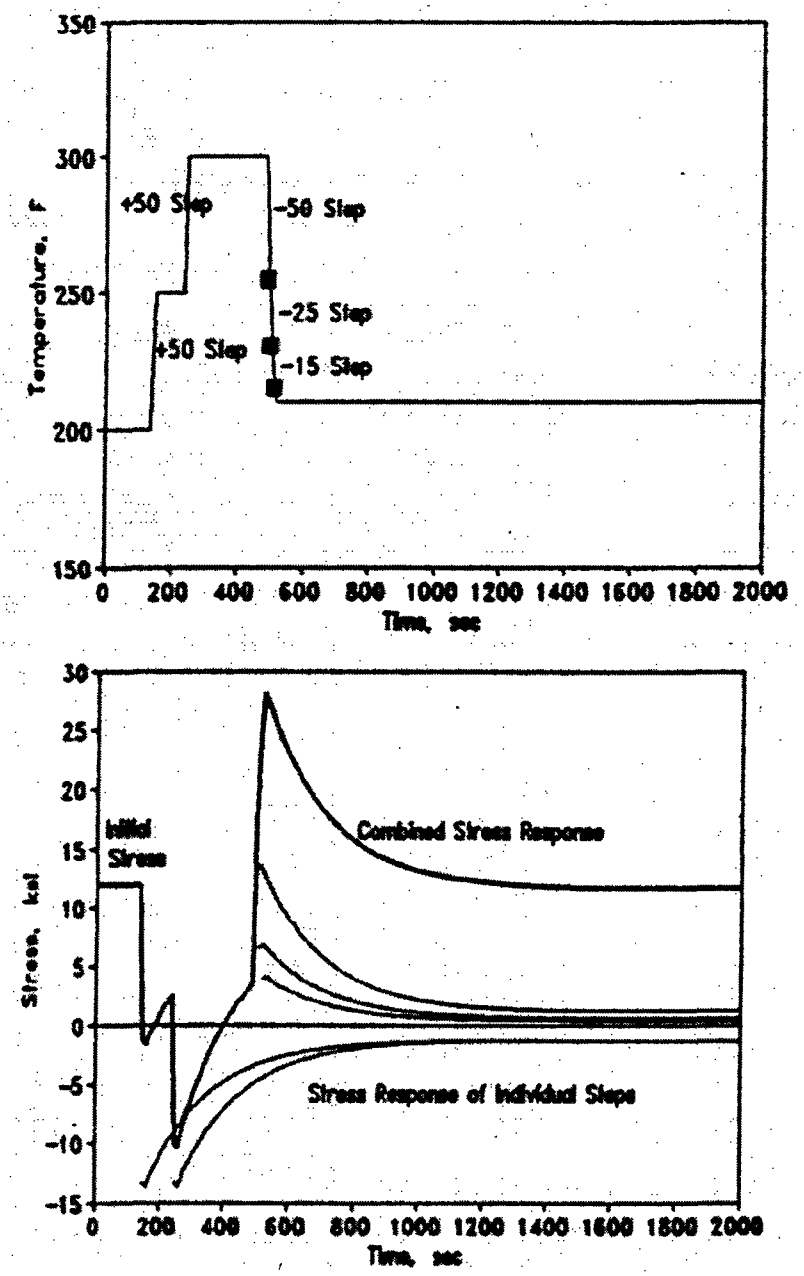


Figure 1-2. Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

A previously generated ANSYS [5] finite element model (FEM) of the VY feedwater nozzle and safe end was used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [6] calculation.

A few key points with respect to model development are as follows:

- The model is identical to the geometry and mesh of the model previously developed for feedwater nozzle fracture mechanics work performed for VY [7].
- The boundary condition corresponding to the location of the start of the thermal sleeve in the FEM are consistent with Reference [8].

The materials of the various components of the model are listed below:

- Reactor Pressure Vessel – SA533 Grade B
- Reactor Pressure Vessel Cladding – Stainless Steel
- Nozzle Forging – ASTM A508 Class II
- Safe End Forging – ASTM A508 Class I
- Feedwater Piping – ASTM A106 Grade B

The FEM model the radius of RPV was increased by a factor of two to account for the fact that the vessel portion of the finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], and are shown in Table 2-1. The properties were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which was applied to the FEM model for Green's Function development.

The finite element model is shown in Figures 2-1 and 2-2.

Table 2-1. Material Properties @ 300°F [1]

Material Ident.	Young's Modulus, E x 10^6 (psi)	Instantaneous Coefficient of Thermal Expansion, α x 10^{-6} (in/in-°F)	Density, ρ (lb/in^3) (assumed)	Conductivity, k (BTU/hr-ft-°F)	Diffusivity, d (ft^2/hr)	Specific Heat, c_p (BTU/lbm-°F) (see Note 5)	Poisson's Ratio (assumed)
SA533 Grade B, A508 Class II (see Note 2)	26.7	7.3	0.283	23.4	0.401	0.119	0.3
SS Clad (see Note 3)	27.0	9.8	0.283	9.8	0.160	0.125	0.3
A508 Class I (see Note 4)	28.1	7.3	0.283	32.3	0.561	0.118	0.3
A106 Grade B (see Note 4)	28.3	7.3	0.283	32.3	0.561	0.118	0.3

Notes

1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D [9], except for density and Poisson's ratio, which are assumed typical values.
2. Properties of A508 Class II are used (3/4Ni-1/2Mo-1/3Cr-V).
3. Properties of 18Cr - 8Ni austenitic stainless steel are used.
4. Composition = C-Si.
5. Calculated as $k/(\rho d)/12^3$.

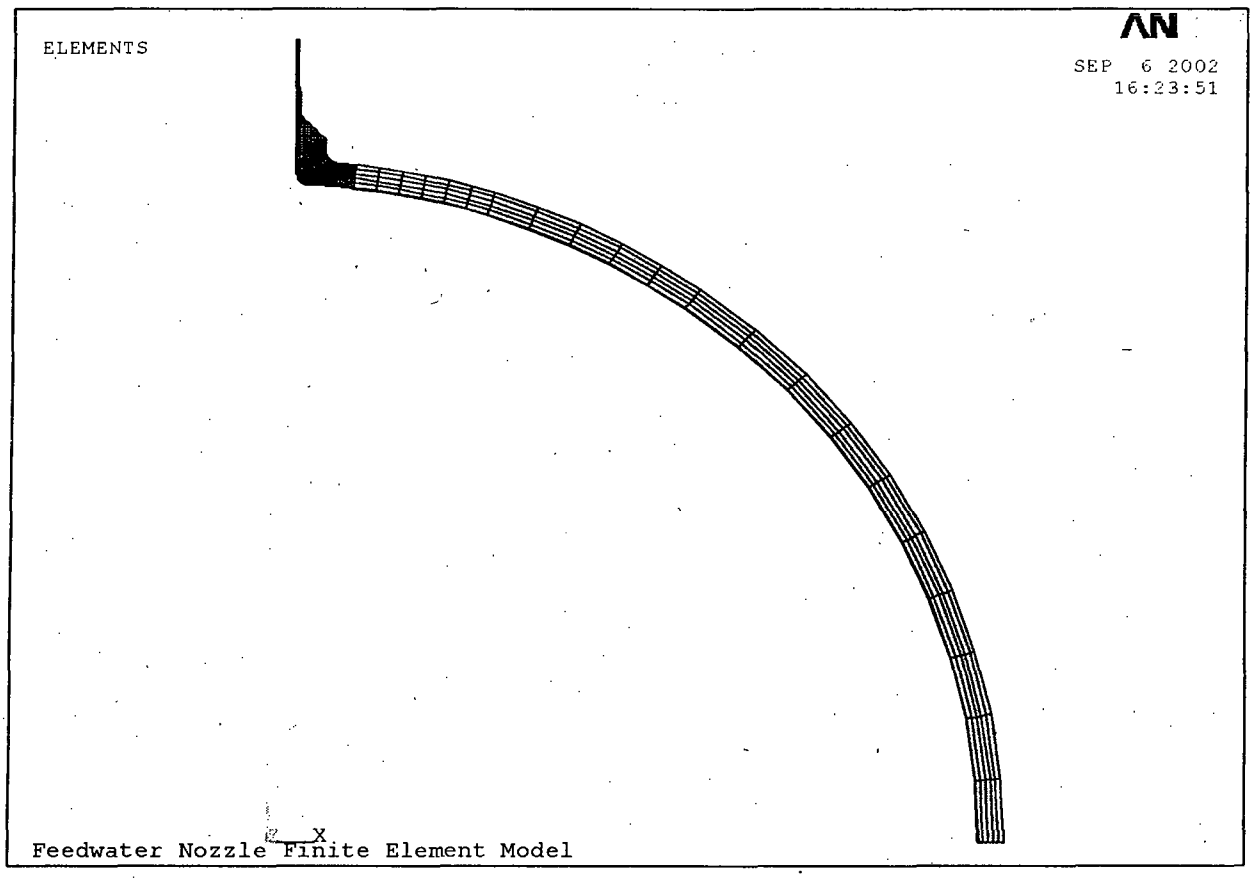


Figure 2-1: VY Feedwater Nozzle FEM

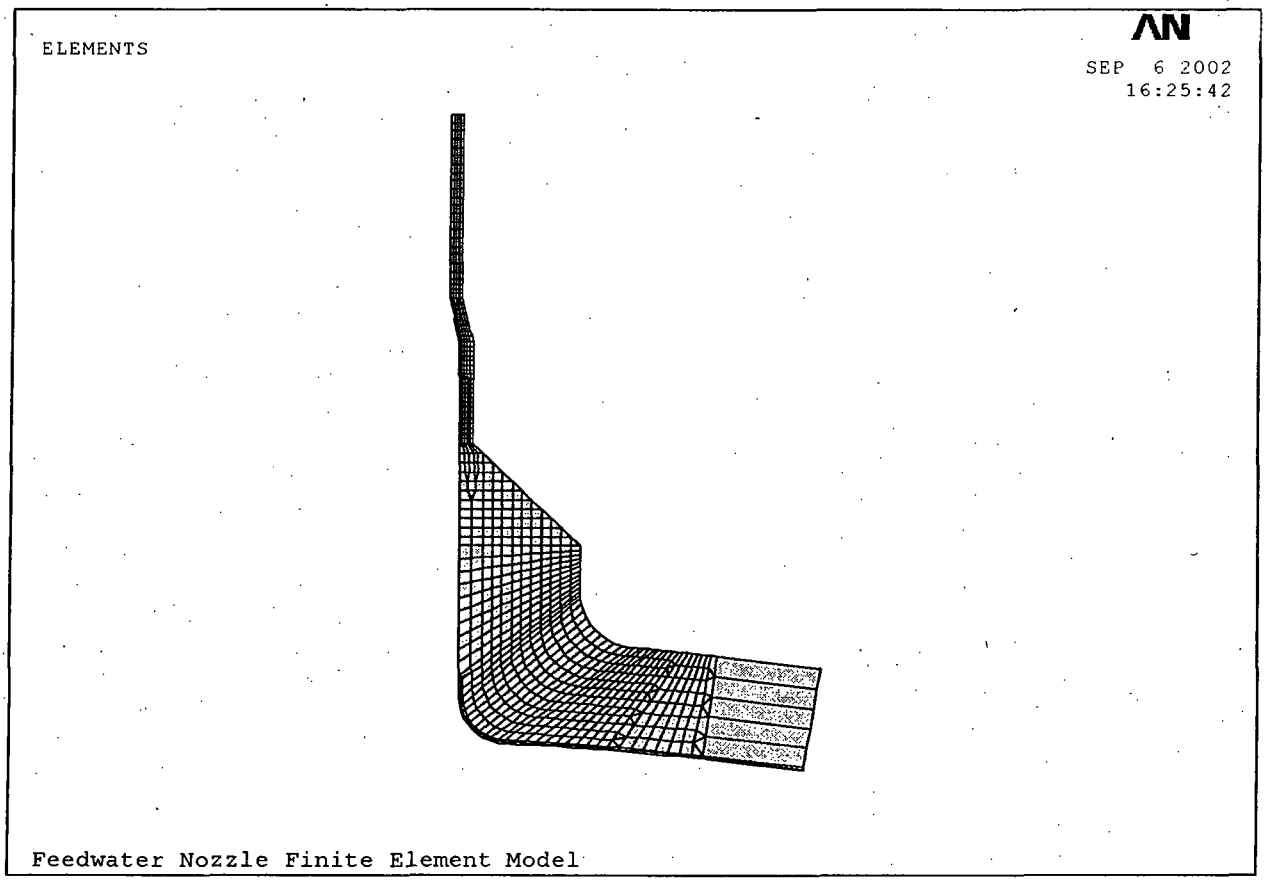


Figure 2-2: VY Feedwater Nozzle FEM – Safe End/Nozzle Region

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the feedwater nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model described in Section 2.0 of this report. The details of the Green's function development and associated stress evaluation are documented in the Reference [10] calculation.

3.1 Thermal Loading

Thermal loads are applied to the feedwater nozzle model. The heat transfer coefficients after power uprate were determined in Reference [10]. These values were determined for various regions of the finite element model and for 100% (4,590 GPM), 40% (1836 GPM) and 25% (1,148 GPM) [10]. The annulus leakage flow rate is assumed to be 25 GPM for non-EPU conditions and 31 GPM for EPU conditions. The 25 GPM value is calculated by scaling the 23 GPM [Page 6, 13] value up by approximately 9%. The 23 GPM value is scaled up to provide some conservatism and allow for inaccuracies in the determination of leakage flow. The 31 GPM value is calculated by multiplying the 25 GPM value by 1.25 [Page 6, 13]. Based on this, the annulus leakage flow rate is assumed to be 8 GPM for EPU conditions with 25% flow rate and 13 GPM for EPU condition with 40% flow rate. The temperatures used are based upon a thermal shock from 500°F to 100°F.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-4, heat transfer coefficients were applied as follows:

- The heat transfer coefficient for the outside surfaces of the FEM (Region 8) was a constant value of 0.2 BTU/hr-ft^2-°F (3.858×10^{-7} BTU/sec-in^2-°F).
- Table 3-3 shows a sampling of the heat transfer coefficient calculations for Region 1 for the 40% flow case.

For all Green's Functions, a 500°F to 100°F thermal shock was run to determine the stress response.

The applied heat transfer coefficients and the initial temperatures for all regions are contained in Reference [10].

3.1.2 Green's Function's

Three flow dependent thermal load cases were run on the FEM model with the heat transfer coefficients and the fluid temperature conditions listed above. Two locations were selected for analysis (see Figures 3-5 and 3-6):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under high flow conditions. The highest stress intensity due to thermal loading occurred at Node 192 (see Figure 3-5), on the inside diameter of the nozzle safe end, and therefore, this node was selected for analysis. Because the safe end stress response is affected by flow, three flow conditions were analyzed (100%, 40% and 25%).
2. The critical blend radius location was chosen, based upon the highest pressure stress. Conservatively assuming the cladding has cracked, the critical location is selected as node 657 at base metal of the nozzle, as shown in Figure 3-6. Because the blend radius stress response is affected by flow, three flow conditions were analyzed (100%, 40% and 25%).

Two stress intensity time history were developed for each location and each flow case: (1) total stress intensity, and (2) membrane plus bending stress intensity. The stress time histories for the safe end location, where the maximum stress was obtained for each of the flow conditions, are shown in Figures 3-7 through 3-12. The stress time histories for the blend radius location, where the maximum stress was obtained for each of the flow conditions, are shown in Figures 3-13 through 3-18.

3.1.3 Thermal Transients (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 12 stress history functions obtained from Reference [10]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for each of the following flow conditions; 100%, 40%, and 25% flow.
- Green.cfg is configured as described in Reference [14].
- Transnt.inp. These files are created to represent the transients shown on the thermal cycle diagrams and redefined by power uprate. Note that transients 12, 13, and 15 are nearly identical on the thermal cycle diagram [19] and the results from running transient 12 will be used for all three transients. Transient 16, 17 and 18 will not be considered since there is no temperature change. Tables 3-4 and 3-5 show the thermal history used to represent each transient. Based upon the thermal cycle diagram for the feedwater nozzle [19], the transients are split into the following groups based upon flow rate:
    - Transients 3, 20, 20A, and 21-23 are run at 25% flow. Although Reference [19] shows 15% flow rate, it is conservative to use 25% flow rate for these transients. Transient 20, Hot Standby, is split up into two parts. The first portion is "Heatup portion" and the second portion is "Feedwater Injection portion" that are defined from Reference [19].
    - Transient 11 is run at 40% flow. Transient 11 starts off and ends at 100% flow.
    - Transients 5, 6, 9, 10, and 19 are run at 100% flow.
    - Transient 4 is run at 100% flow only to obtain the last stress point. The remainder of the stress points for transient 4 is obtained from the 25% flow stress results. The results are pulled from the two flow case results based upon the flow rates defined in the thermal cycle diagram [19].
    - Transients 12, 13, 14 and 15 were run at 100% flow. Heat transfer coefficients were not re-calculated for the 1 minute intervals each of these transients is at 110% flow. The effect of this small flow rate increase for such a relatively short duration should be minor.

- Transients 1, 2, 24, and 25 are set as no thermal stress due to very small temperature changes (70°F to 100°F) at these transients.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the feedwater nozzle and the vessel wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. The nodal forces shown in Table 3-1 [10] are defined by the following equation:

$$F_{element} = \pi(IR)^2 P \cdot \frac{\pi\left(R_o^{\ 2} - R_i^{\ 2}\right)}{\pi\left(OR^2 - IR^2\right)}$$

where:

P	=	unit pressure load = 1,000 psi
IR	=	inner pipe radius = 4.8345 in
OR	=	outer pipe radius = 5.42 in
R_i	=	inside radius of element that node is attached to
R_o	=	outside radius of element that node is attached to
F_{node}	=	average of the element forces on either side of the node.

Note: The force on the innermost and outermost nodes is calculated as one half of the force on the element that they are attached to.

The calculated nodal forces were applied as positive values so they would exert tension on the end of the model. Figures 3-1, 3-2, and 3-3 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [10]. These values are as follows:

Pressure stress for the safe end:

- 8693 psi membrane plus bending stress intensity.
- 8891 psi total linearized stress intensity.

Pressure stress for the blend radius:

- 36653 psi membrane plus bending stress intensity.
- 37733 psi total linearized stress intensity.

These pressure stress values for each location were linearly scaled with pressure. The actual pressure for column 6 of Tables 4-1 and 4-2 is obtained from Tables 3-4 and 3-5. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4-1 and 4-2.

The pressure stress is combined with the thermal and piping loads to calculate the final stress values used for fatigue analysis. The piping load sign is set as the same as the thermal stress sign.

3.3 Piping Loading

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 3-36.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

For Cut I:
$$(M_x)_1 = M_x - F_y L_1$$
$$(M_y)_1 = M_y + F_x L_1$$

For Cut II:
$$(M_x)_2 = M_x - F_y L_2$$
$$(M_y)_2 = M_y + F_x L_2$$

The total bending moment and shear loads are obtained using the equations below:

For Cut I:

$$M_{xy} = \sqrt{(M_x)_1^{\,2} + (M_y)_1^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_1^{\,2} + (F_y)_1^{\,2}}$$

For Cut II:

$$M_{xy} = \sqrt{(M_x)_2^{\,2} + (M_y)_2^{\,2}}$$
$$F_{xy} = \sqrt{(F_x)_2^{\,2} + (F_y)_2^{\,2}}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$N_z = \frac{1}{\pi R_N}\left[\frac{1}{2}F_z + \frac{M_{xy}}{R_N}\right]$$
$$q_N = \frac{1}{\pi R_N}\left[F_{xy} - \frac{M_z}{2R_N}\right]$$

To determine the primary stresses, P_M, due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$

$$(P_M)_\theta = \frac{Pa_N}{t_N}$$

$$(P_M)_R = -P$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\ 2}}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^{\ 2}}$$

where: L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.

L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.

M_{xy} = The maximum bending moment in the xy plane.

F_{yx} = The maximum shear force in the xy plane.

N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.

q_N = The shear force per inch of circumference applied to the nozzle.

R_N = The mid-wall nozzle radius.

Because pressure was not considered in this analysis, the equations used for Cut I are valid for Cut II. Furthermore, since the pressure was not considered in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{Nz}{t_N}$$
$$(P_M)_\theta = 0$$
$$(P_M)_R = 0$$
$$\tau_M = \frac{q_N}{t_N}$$
$$SI_{MAX} = 2(\tau_M)_{z\theta}$$
or
$$SI_{MAX} = 2\sqrt{\left(\frac{Nz}{2t_N}\right)^2 + (\tau_M)_{z\theta}^{\ 2}}$$

Per Reference [11], the feedwater nozzle piping loads are as follows:

F_x = 3,000 lbs	M_x = 28,000 ft-lb = 336,000 in-lb
F_y = 15,000 lbs	M_y = 13,000 ft-lb = 156,000 in-lb
F_z = 3,200 lbs	M_z = 40,000 ft-lb = 480,000 in-lb

The loads are applied at the connection of the piping and safe end. Therefore, the L_1 is equal to 12.0871 inches and the L_2 is equal to 27.572 inches. The calculations for the safe end and blend radius are shown in Table 3-2. The first cut location is the same as the Green's Function cross section per [10] at the safe end, and the second cut is from Node 645 (outside) to Node 501 (inside). The maximum stress intensities due to piping loads are 5707.97 psi at the safe end and 265.47 psi at the blend radius, respectively.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F. The scaled piping stress values are shown in columns 9 and 10 of Tables 4-1 and 4-2. Columns 11 and 12 of Tables 4-1 and 4-1 show the summation of all stresses for each thermal peak and valley stress point.

Table 3-1: Nodal Force Calculation for End Cap Load

Node Number	Element Number	Radius (in)	Δ Radius (in)	R_o^2-Ri^2 (in^2)	$F_{element}$ (lb)	F_{node} (lb)
1		5.42				7678.0
	1022		0.1171	1.25565	15356.1	
2		5.3029				15188.4
	1021		0.1171	1.22823	15020.7	
3		5.1858				14853.0
	1020		0.1171	1.20080	14685.3	
4		5.0687				14517.6
	1019		0.1171	1.17338	14349.9	
5		4.9516				14182.2
	1018		0.1171	1.14595	14014.5	
6		4.8345				7007.3

Table 3-2: Maximum Piping Stress Intensity Calculations

Safe End External Piping Loads		
Parameters		
F_x =	3.00	kips
F_y =	15.00	kips
F_Z =	3.20	kips
M_x =	336.00	in-kips
M_y =	156.00	in-kips
M_Z =	480.00	in-kips
OD=	11.86	in
ID=	10.409	in
R_N =	5.57	in
L =	12.09	in
t_N =	0.72	in
$(M_x)_1$ =	154.69	in-kips
$(M_y)_1$ =	192.26	in-kips
M_{xy} =	246.77	in-kips
F_{xy} =	15.30	kips
N_Z =	2.63	kips/in
q_N =	-1.59	kips/in
Primary Membrane Stress Intensity		
PM_Z =	3.63	ksi
τ =	-2.20	ksi
SI_{max} =	5.71	ksi
SI_{max} =	5707.97	psi

Blend Radius External Piping Loads		
Parameters		
F_x =	3.00	kips
F_y =	15.00	kips
F_Z =	3.20	kips
M_x =	336.00	in-kips
M_y =	156.00	in-kips
M_Z =	480.00	in-kips
OD=	22.67	in
ID=	10.750	in
R_N =	8.35	in
L =	27.57	in
t_N =	5.96	in
$(M_x)_2$ =	-77.58	in-kips
$(M_y)_2$ =	238.72	in-kips
M_{xy} =	251.01	in-kips
F_{xy} =	15.30	kips
N_Z =	1.21	kips/in
q_N =	-0.51	kips/in
Primary Membrane Stress Intensity		
PM_Z =	0.20	ksi
τ =	-0.09	ksi
SI_{max} =	0.27	ksi
SI_{max} =	265.47	psi

Note: The locations for Cut I and Cut II were defined in Reference [10] for safe end and blend radius paths, respectively.

Table 3-3: Heat Transfer Coefficients for Region 1 (40% Flow)

Calculation of Heat Transfer Coefficients for Feedwater Nozzle Flow Path

Pipe Inside Diameter, D =	9.669	inches =	0.806	ft	100% rated flow =	4,590	gpm
		=	0.246	m	@ T =	391.9	°F
Flow, % of rated =	40%				Density, ρ =	53.8997	lbm/ft³
Fluid Velocity, V =	8.022	ft/sec =	1,836.0	gpm =	0.793742524	Mlb/hr	
Characteristic Length, L = D =	0.806	ft =	0.246	m			

$T_{fluid} - T_{surface}$, ΔT = assumed to be 12% of fluid temperature = 8.40, 12.00, 24.00, 36.00, 48.00, 60.00, 72.00 °F

Note: The above assumption is based on experience with past RPV heat transfer analyses. = 4.67, 6.67, 13.33, 20.00, 26.67, 33.33, 40.00 °C

		Value at Fluid Temperature, T [26]							Units
Water Property	Conversion Factor [24]	70	100	200	300	400	500	600	°F
		21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c_p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ (Density)	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m³
		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft³
β (Volumetric Rate of Expansion)	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m³/m³-°C
		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft³/ft³-°F
g (Gravitational Constant)	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s²
		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s²
μ (Dynamic Viscosity)	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr (Prandtl Number)		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—
Calculated Parameter	**Formula**	**70**	**100**	**200**	**300**	**400**	**500**	**600**	**°F**
Reynold's Number, Re	ρVD/μ	6.0147E+05	8.7645E+05	1.8859E+06	2.8491E+06	3.7255E+06	4.5248E+06	4.7336E+06	—
Grashof Number, Gr	$g\beta\Delta TL^3/(\mu/\rho)^2$	1.2852E+08	6.6834E+08	1.2721E+10	6.5918E+10	2.0931E+11	5.4429E+11	1.1372E+12	—
Rayleigh Number, Ra	GrPr	8.9710E+08	3.0142E+09	2.4297E+10	8.0420E+10	1.9885E+11	4.6755E+11	1.2168E+12	—
From [24]: ***Inside Surface Forced Convection Heat Transfer Coefficient:***									
H_{forced} =	$0.023Re^{0.8}Pr^{0.4}k/D$	5,132.76	6,119.10	8,626.61	10,107.53	10,960.57	11,236.63	10,678.39	W/m²-°C
		903.95	1,077.66	1,519.26	1,780.07	1,930.31	1,978.92	1,880.61	Btu/hr-ft²-°F
		1.744E-03	2.079E-03	2.931E-03	3.434E-03	3.724E-03	3.817E-03	3.628E-03	Btu/sec-in²-°F
From [24]: ***Inside Surface Natural Convection Heat Transfer Coefficient:***									
Case:	Enclosed cylinder		C =	0.55	n =	0.25	(see page 289 of [25])		
H_{free} =	$C(GrPr)^n k/L$	232.43	330.57	599.85	815.28	988.69	1,118.54	1,192.73	W/m²-°C
		40.93	58.22	105.64	143.58	174.12	196.99	210.06	Btu/hr-ft²-°F
		7.896E-05	1.123E-04	2.038E-04	2.770E-04	3.359E-04	3.800E-04	4.052E-04	Btu/sec-in²-°F

Table 3-4: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0
123 Cycles	10	70	10	0
2. Design	0	70		0
HYD Test	1080	100	1080	0
120 Cycles	1680	100	600	1100
	5280	100	3600	1100
	5880	100	600	50
	10880	100	5000	50
3. Startup	0	100		50
300 Cycles	16164	549	16164	1010
LF_25	21164	549	5000	1010
4. Turbine Roll	0	549		1010
and Increased to	1	100	1	1010
Rated	1801	100	1800	1010
Power	1802	260	1	1010
300 cycles	3602	392	1800	1010
LF_25, HF_100	8602	392	5000	1010
5. Daily	0	392		1010
Reduction	900	310	900	1010
75% Power	2700	310	1800	1010
10,000 Cycles	3600	392	900	1010
HF_100	8600	392	5000	1010
6. Weekly Reduct	0	392		1010
60% Power	1800	280	1800	1010
2,000 Cycles	3600	280	1800	1010
HF_100	5400	392	1800	1010
	10400	392	5000	1010
9. Turbine	0	392		1010
Trip at 25%	1800	265	1800	1010
Power	1980	265	180	1010
10 Cycles	2340	90	360	1010
HF_100	2520	90	180	1010
	3420	265	900	1010
	3600	265	180	1010
	5400	392	1800	1010
	10400	392	5000	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
10. FW Heater	0	392		1010
Bypass	90	265	90	1010
70 Cycles	1890	265	1800	1010
HF_100	2070	392	180	1010
	7070	392	5000	1010
11. Loss of	0	392		1010
FW Pumps	1	565	1	1010
10 Cycles	3.5	565	2.5	1190
MF_40, HF_100	4.5	50	1	1185
	13.5	50	9	1135
	184.5	50	171	1135
	1584.5	440	1380	1135
	1565.5	565	1	1135
	2165.5	565	600	1135
	2166.5	50	1	1135
	2346.5	50	180	885
	5406.5	440	3060	1055
	5407.5	565	1	1055
	6727.5	565	1320	1135
	6728.5	50	1	1135
	7148.5	50	420	675
	7448.5	300	300	675
	11048.5	400	3600	232
	16411.5	549	5363	885
	16412.5	549	1	1010
	18212.5	549	1800	1010
	18213.5	100	1	1010
	20013.5	100	1800	1010
	20014.5	260	1	1010
	21814.5	392	1800	1010
	26814.5	392	5000	1010
12. Turbine	0	392		1010
Generator Trip	10	392	10	1135/1375(2)
60 Cycles	15	392	5	1135/1375(2)
13. Reactor	30	392	15	940
Overpressure	90	275	60	940
1 Cycles	990	100	900	940
15. Other	2790	100	1800	940
SCRAMs	2791	260	1	940
228 cycles	3210	291	419	1010
HF_100	4591	392	1381	1010
	9591	392	5000	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
14. SRV	0	392		1010
Blowdown	60	275	60	885
1 Cycles	960	100	900	50
HF_100	5960	100	5000	50
19. Reduction to 0%	0	392		1010
Power 300 Cycles	1800	265	1800	1010
HF_100	6800	265	5000	1010
20. Hot Standby	0	265		1010
(Heatup Portion)	1	440	1	1010
300 Cycles	3925	549	3924	1010
LF_25	8925	549	5000	1010
20A. Hot Standby	0	549		1010
(FW Injection Portion)	1	100	1	1010
300 Cycles	181	100	180	1010
LF_25	241	290	60	1010
	451	549	210	1010
	5451	549	5000	1010
21-23. Shutdown	0	549		1010
300 Cycles	6264	375	6264	50
LF_25	6864	330	600	50
	15144	100	8280	50
	20144	100	5000	50
24. Hydrostatic	0	100		50
Test	600	100	600	1563
1 Cycles	1200	100	600	1563
	1800	100	600	50
	2400	100	600	50
25. Unbolt	0	100		0
123 Cycles	1080	70	1080	0
	6080	70	5000	0

Note: 1. The indicated time or pressure was assumed.
2. 1375 psi is for Transient 13 only.

Table 3-5: Safe End Transient

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0
123 Cycles	10	70	10	0
2. Design	0	70		0
HYD Test	1080	100	1080	0
120 Cycles	1680	100	600	1100
	5280	100	3600	1100
	5880	100	600	50
	6380	100	500	50
3. Startup	0	100		50
300 Cycles	16164	549	16164	1010
LF_25	16664	549	500	1010
4. Turbine Roll	0	549		1010
and Increased to	1	100	1	1010
Rated	1801	100	1800	1010
Power	1802	260	1	1010
300 cycles	3602	392	1800	1010
LF_25, HF_100	4102	392	500	1010
5. Daily	0	392		1010
Reduction	900	310	900	1010
75% Power	2700	310	1800	1010
10,000 Cycles	3600	392	900	1010
HF_100	4100	392	500	1010
6. Weekly Reduct	0	392		1010
50% Power	1800	280	1800	1010
2,000 Cycles	3600	280	1800	1010
HF_100	5400	392	1800	1010
	5900	392	500	1010
9. Turbine	0	392		1010
Trip at 25%	1800	265	1800	1010
Power	1980	265	180	1010
10 Cycles	2340	90	360	1010
HF_100	2520	90	180	1010
	3420	265	900	1010
	3600	265	180	1010
	5400	392	1800	1010
	5900	392	500	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
10. FW Heater	0	392		1010
Bypass	90	265	90	1010
70 Cycles	1890	265	1800	1010
HF_100	2070	392	180	1010
	2570	392	500	1010
11. Loss of	0	392		1010
FW Pumps	1	565	1	1010
10 Cycles	3.5	565	2.5	1190
MF_40, HF_100	4.5	50	1	1185
	13.5	50	9	1135
	184.5	50	171	1136
	1564.5	440	1380	1135
	1565.5	565	1	1135
	2165.5	565	600	1135
	2166.5	50	1	1135
	2346.5	50	180	885
	5406.5	440	3060	1055
	5407.5	565	1	1055
	6727.5	565	1320	1135
	6728.5	50	1	1135
	7148.5	50	420	675
	7448.5	300	300	675
	11048.5	400	3600	232
	16411.5	549	5363	885
	16412.5	549	1	1010
	18212.5	549	1800	1010
	18213.5	100	1	1010
	20013.5	100	1800	1010
	20014.5	260	1	1010
	21814.5	392	1800	1010
	22314.5	392	500	1010
12. Turbine	0	392		1010
Generator Trip	10	392	10	1135/1375[3]
60 Cycles	15	392	5	1135/1375[3]
13. Reactor	30	392	15	940
Overpressure	90	275	60	940
1 Cycles	990	100	900	940
15. Other	2790	100	1800	940
SCRAMs	2791	260	1	940
228 cycles	3210	291	419	1010
HF_100	4591	392	1381	1010
	5091	392	500	1010

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
14. SRV	0	392		1010
Blowdown	60	275	60	885
1 Cycles	960	100	900	50
HF_100	1460	100	500	50
19. Reduction to 0%	0	392		1010
Power 300 Cycles	1800	265	1800	1010
HF_100	2300	265	500	1010
20. Hot Standby	0	265		1010
(Heatup Portion)	1	440	1	1010
300 Cycles	3925	549	3924	1010
LF_25	4425	549	500	1010
20A. Hot Standby	0	549		1010
(FW Injection Portion)	1	100	1	1010
300 Cycles	181	100	180	1010
LF_26	241	290	60	1010
	451	549	210	1010
	951	549	500	1010
21-23. Shutdown	0	549		1010
300 Cycles	6264	375	6264	50
LF_25	6864	330	600	50
	15144	100	8280	50
	15644	100	500	50
24. Hydrostatic	0	100		50
Test	600	100	600	1563
1 Cycles	1200	100	600	1563
	1800	100	600	50
	2400	100	600	50
25. Unbolt	0	100		0
123 Cycles	1080	70	1080	0
	1580	70	500	0

Note: 1. These transients are the same as in Table 3-4 with the exception of the 500 second steady state time increment that is used. The transients in Table 3-4 are plotted using a 5000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.

2. The indicated time or pressure was assumed.

3. 1375 psi is for Transient 13 only.

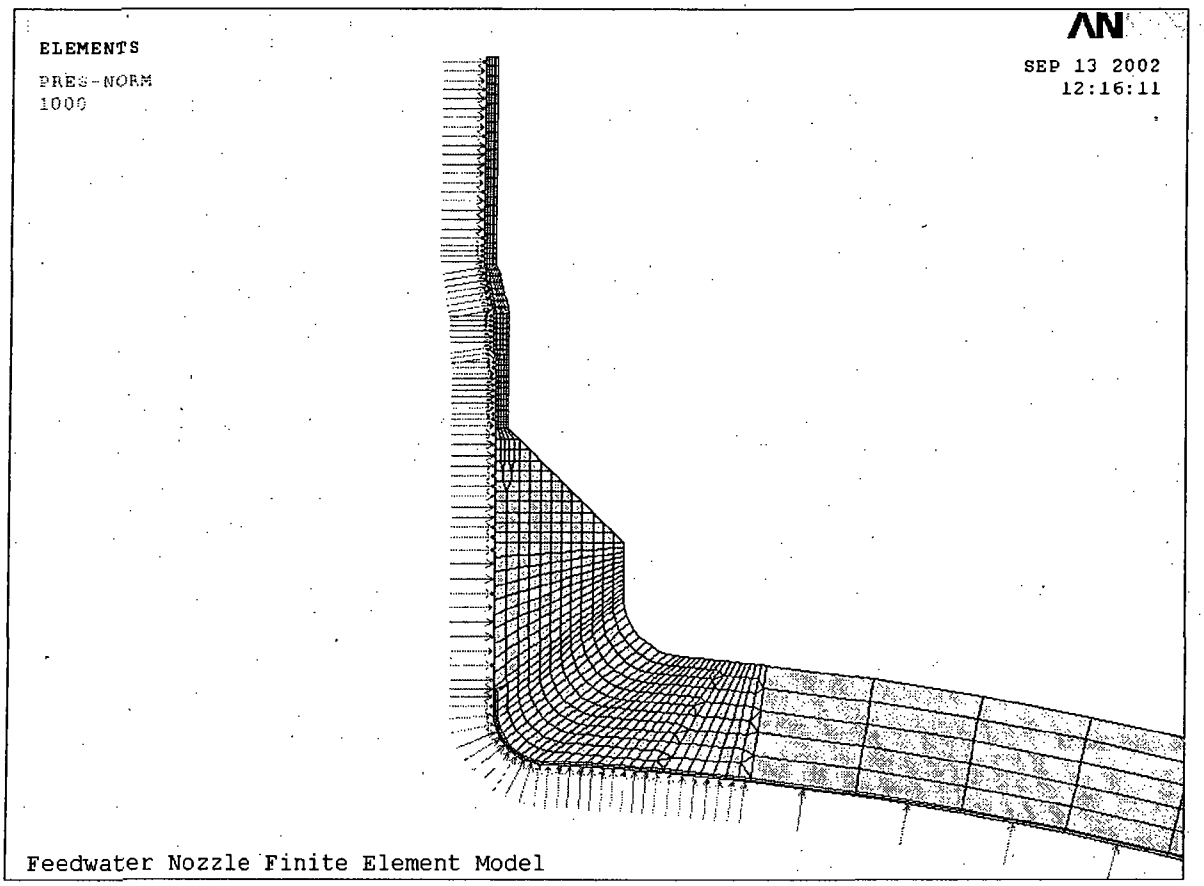


Figure 3-1: Feedwater Nozzle Internal Pressure Distribution

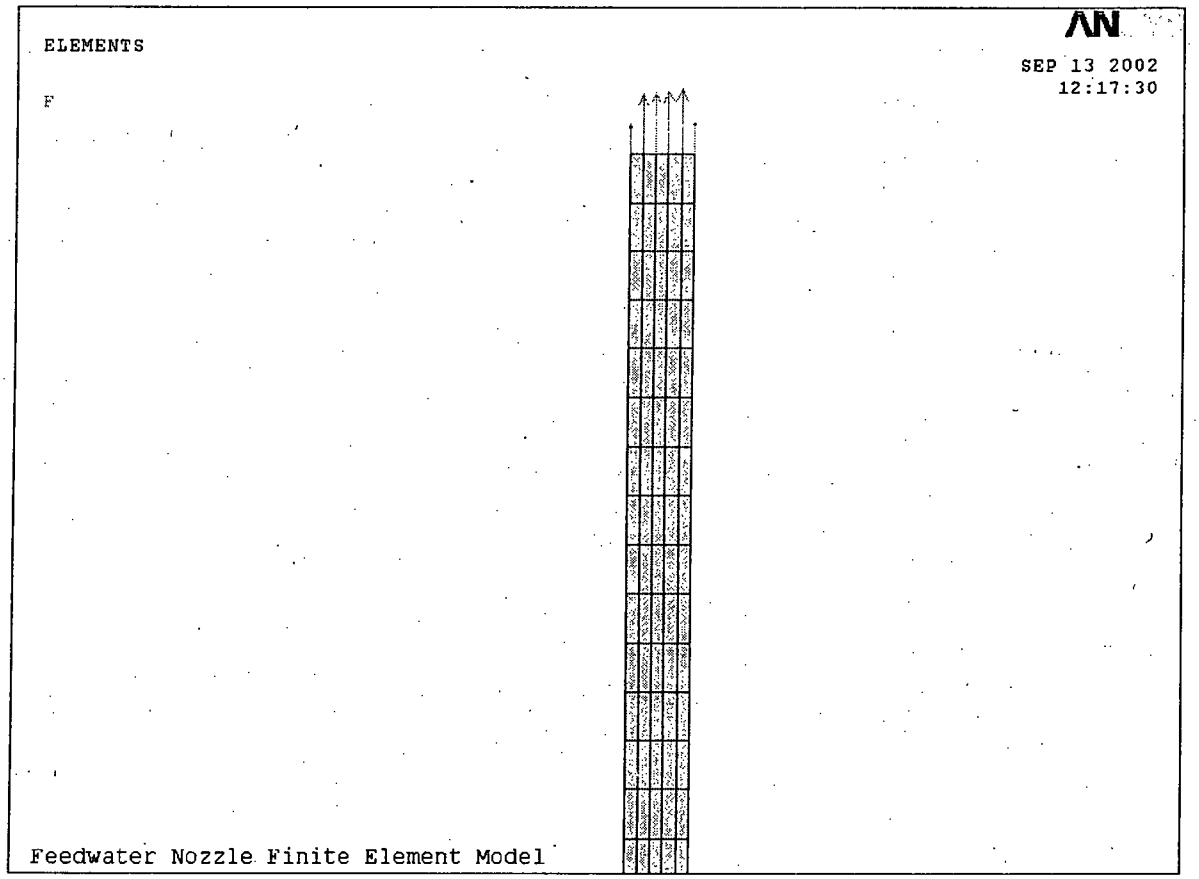


Figure 3-2: Feedwater Nozzle Pressure Cap Load

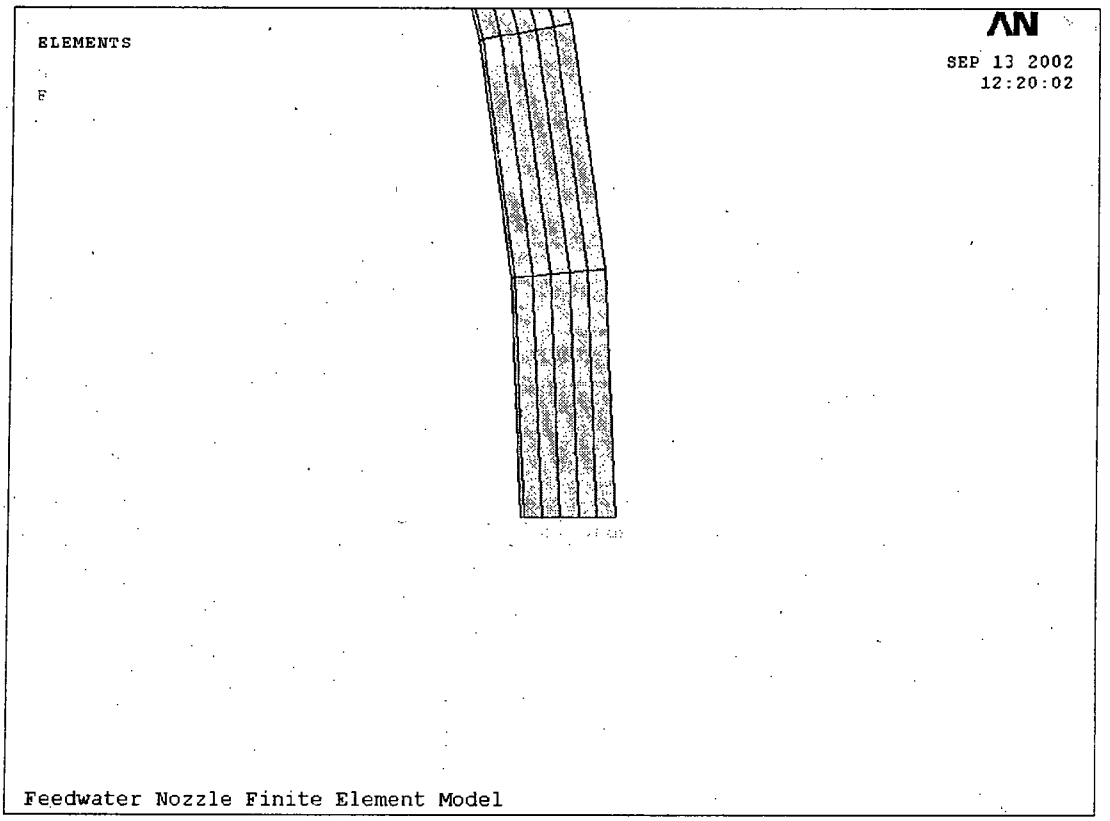
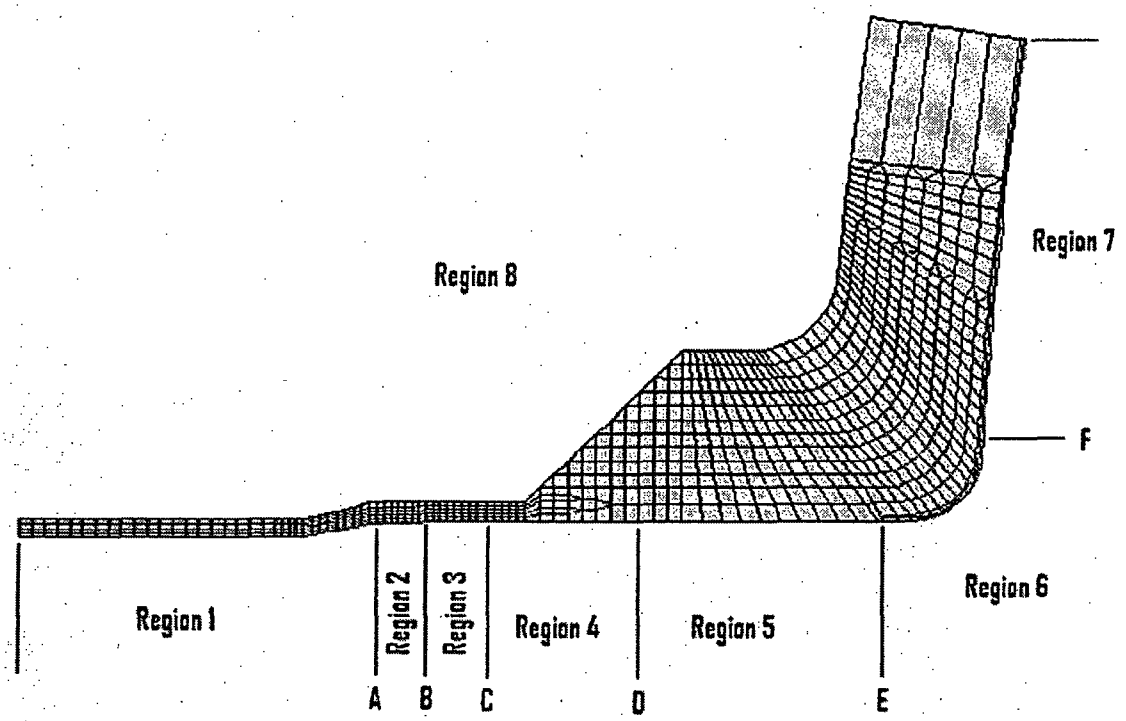


Figure 3-3: Feedwater Nozzle Vessel Boundary Condition



Notes: Point A: End of thermal sleeve = Node 204 = 0.25" from feedwater inlet side of thermal sleeve flat per Reference [8].
Point B: Beginning of annulus = Node 252.
Point C: Beginning of thermal sleeve transition = approximately 4.0" from Point A per Reference [8] = Node 294.
Point D: End of thermal sleeve transition = approximately 9.5" from Point A per Reference [8] = Node 387.
Point E: End of inner blend radius (nozzle side) = Node 553.
Point F: End of inner blend radius (vessel wall side) = Node 779.

Figure 3-4: Thermal Regions

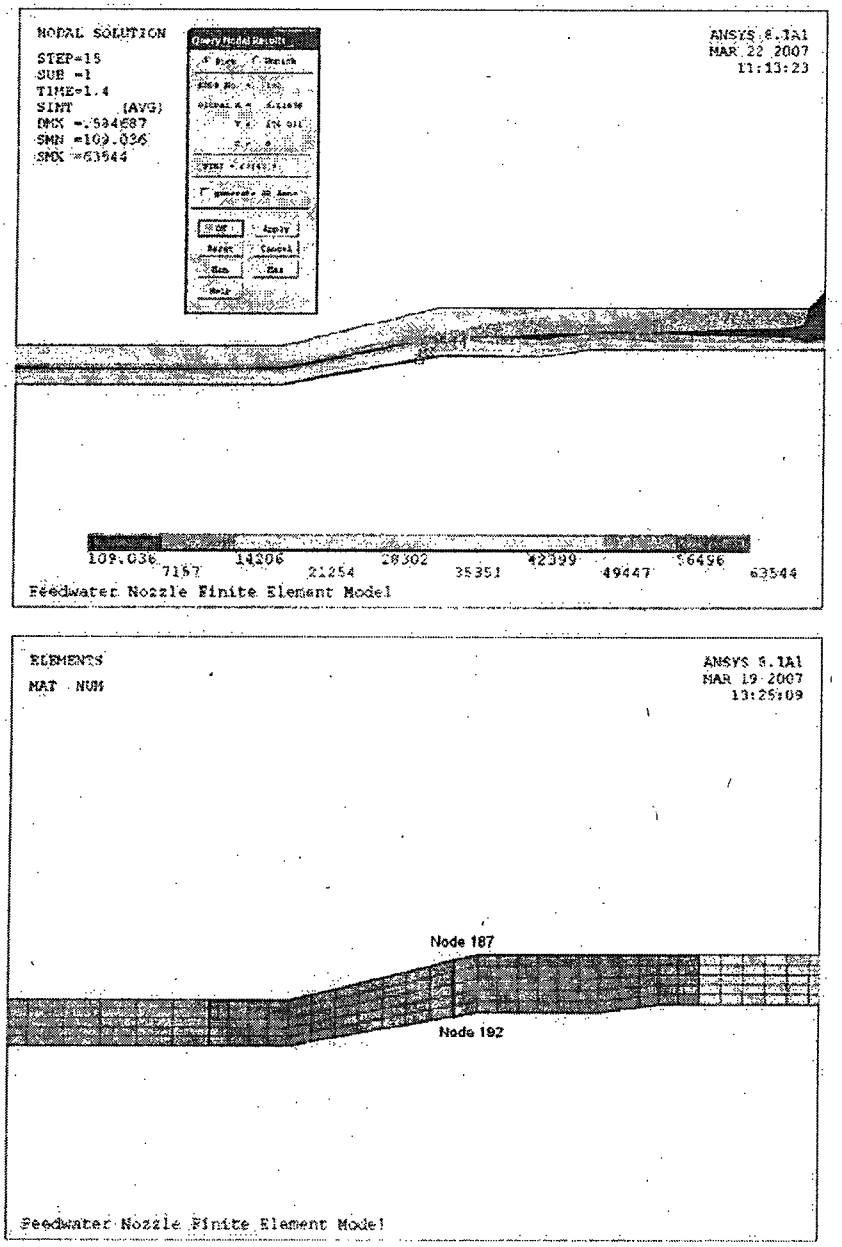
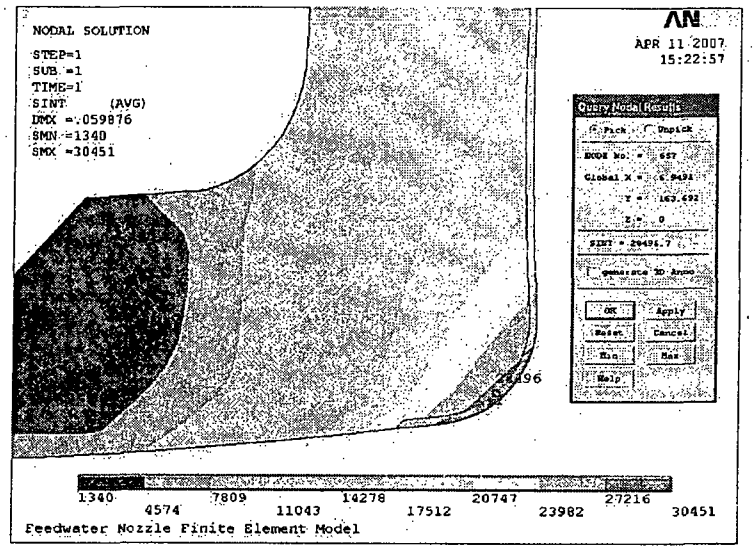


Figure 3-5: Safe End Critical Thermal Stress Location and Linearized Stress Paths



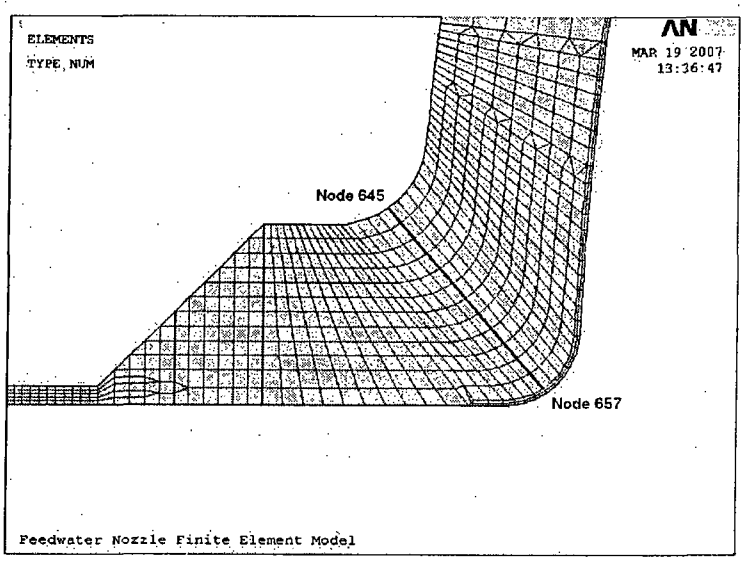


Figure 3-6: Brand Radius Critical Thermal Stress Location and Linearized Stress Paths

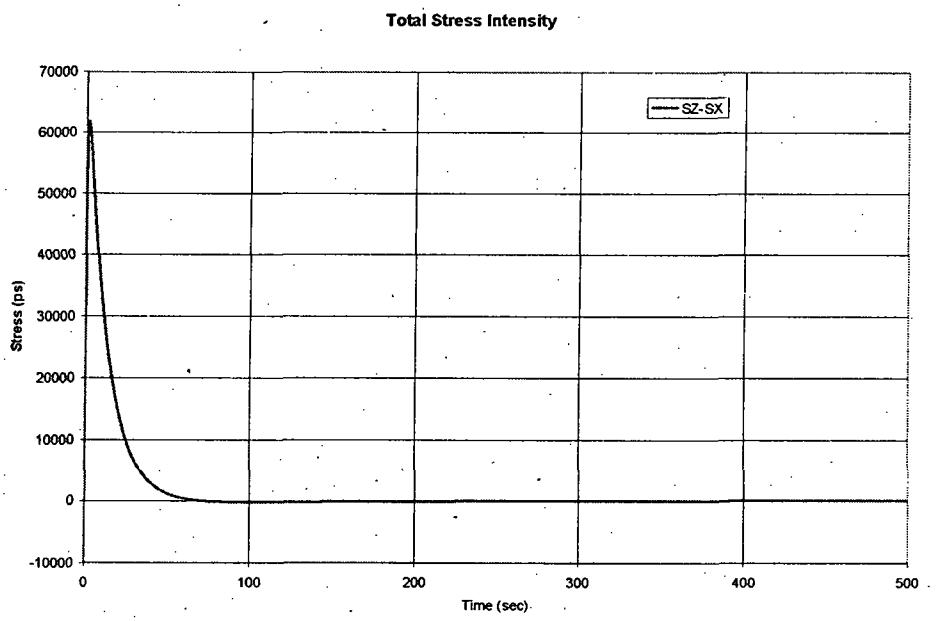


Figure 3-7: Safe End Total Stress History for 100% Flow

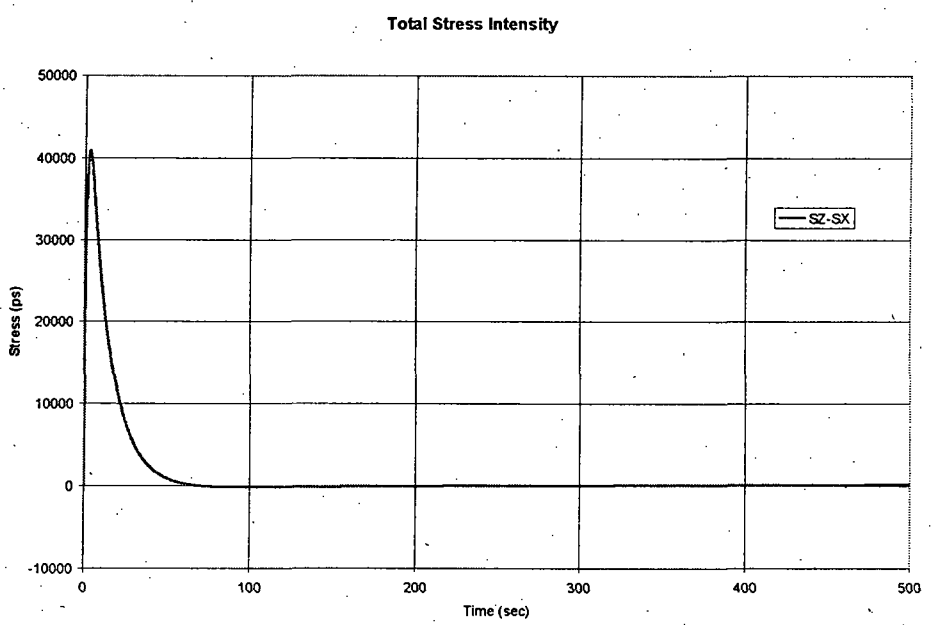


Figure 3-8: Safe End Membrane Plus Bending Stress History for 100% Flow

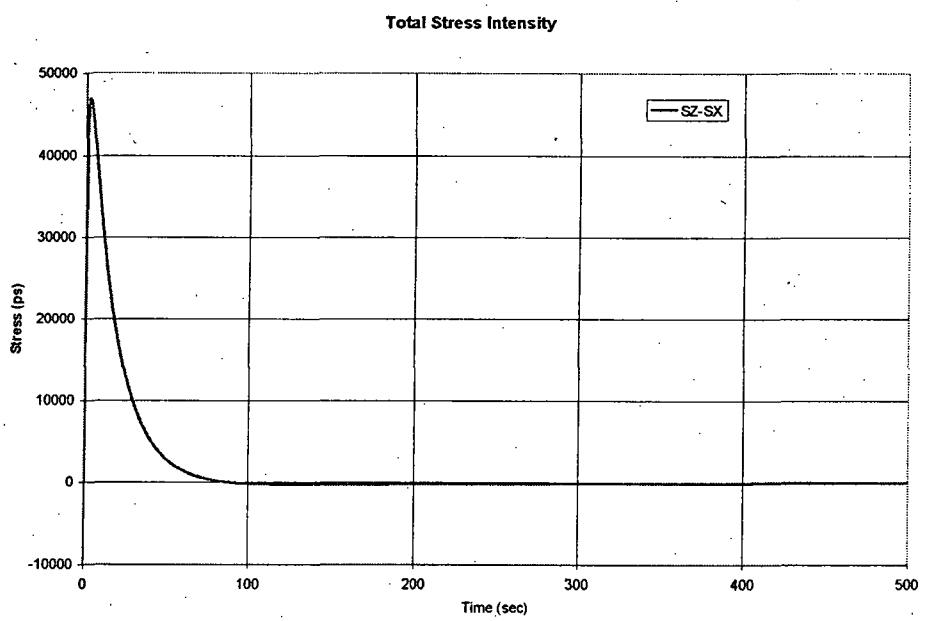


Figure 3-9: Safe End Total Stress History for 40% Flow

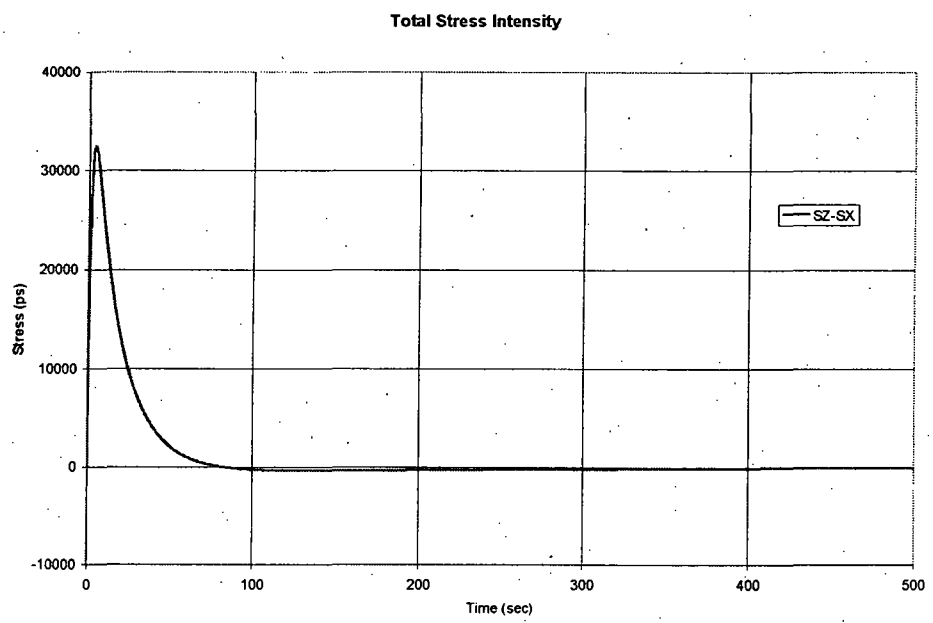


Figure 3-10: Safe End Membrane Plus Bending Stress History for 40% Flow

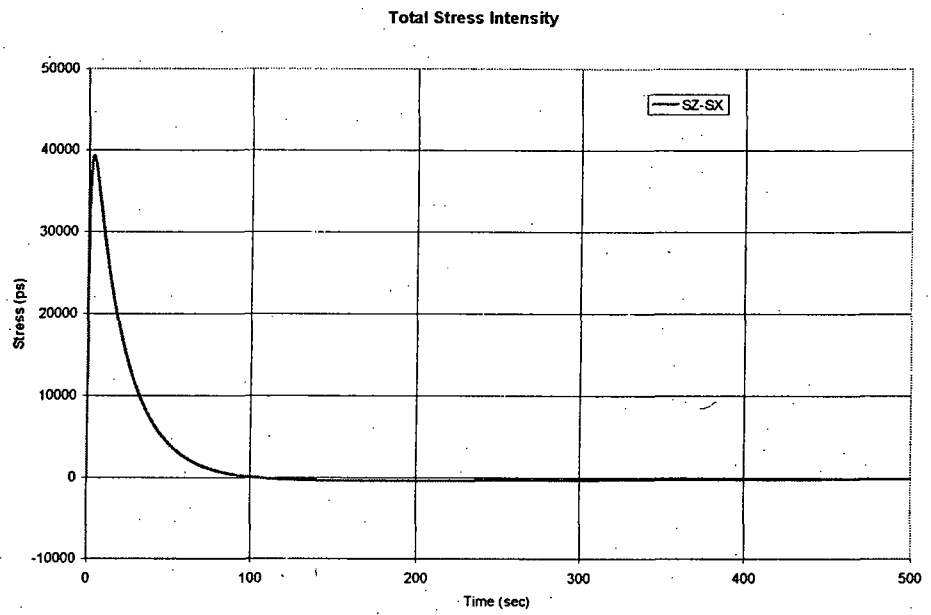


Figure 3-11: Safe End Total Stress History for 25% Flow

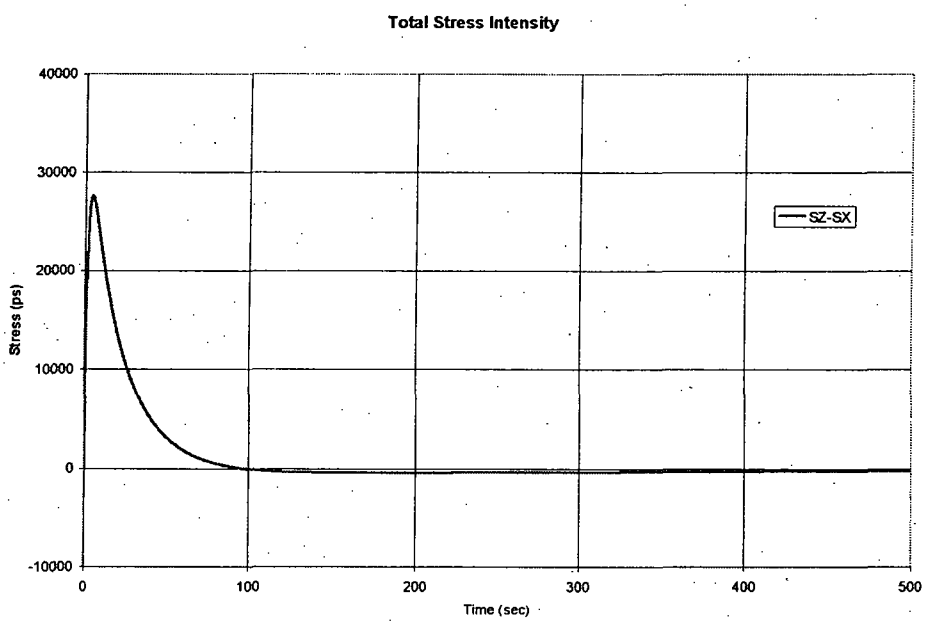


Figure 3-12: Safe End Membrane Plus Bending Stress History for 25% Flow

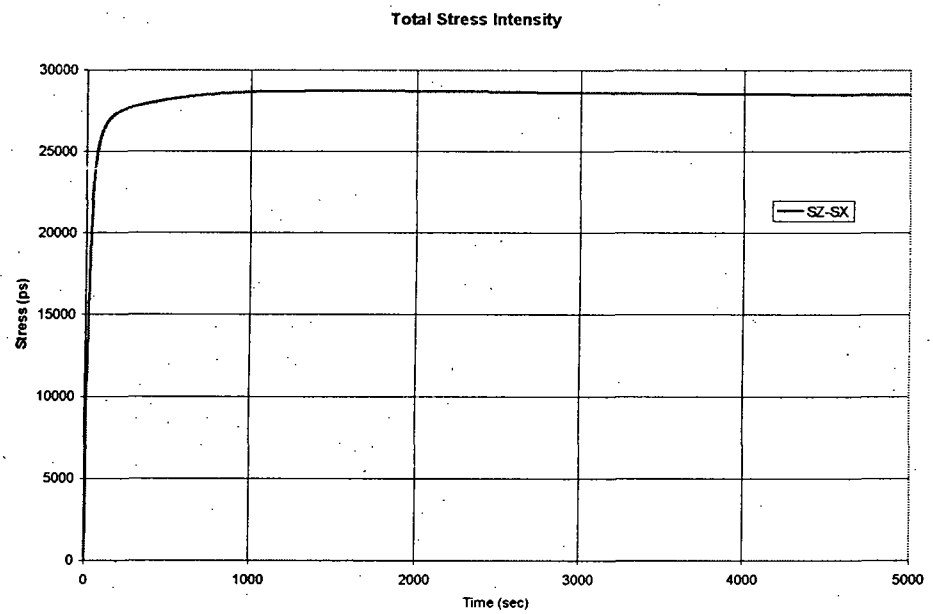


Figure 3-13: Blend Radius Total Stress History for 100% Flow

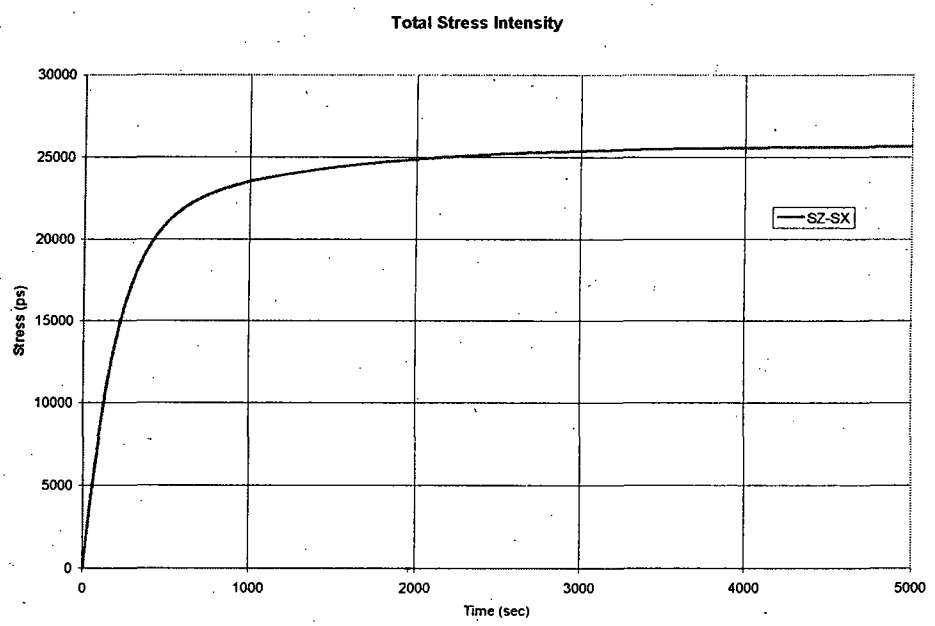


Figure 3-14: Blend Radius Membrane Plus Bending Stress History for 100% Flow

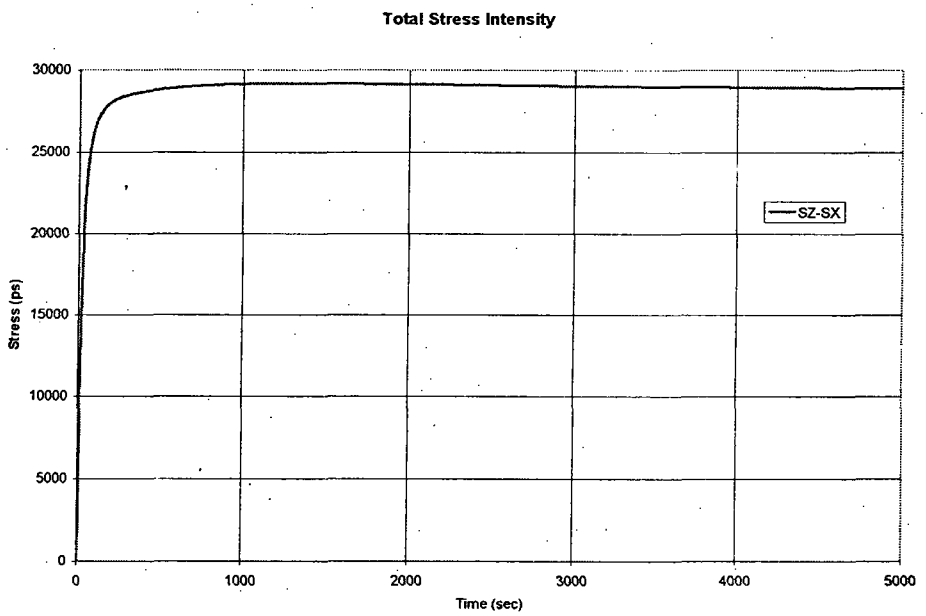


Figure 3-15: Blend Radius Total Stress History for 40% Flow

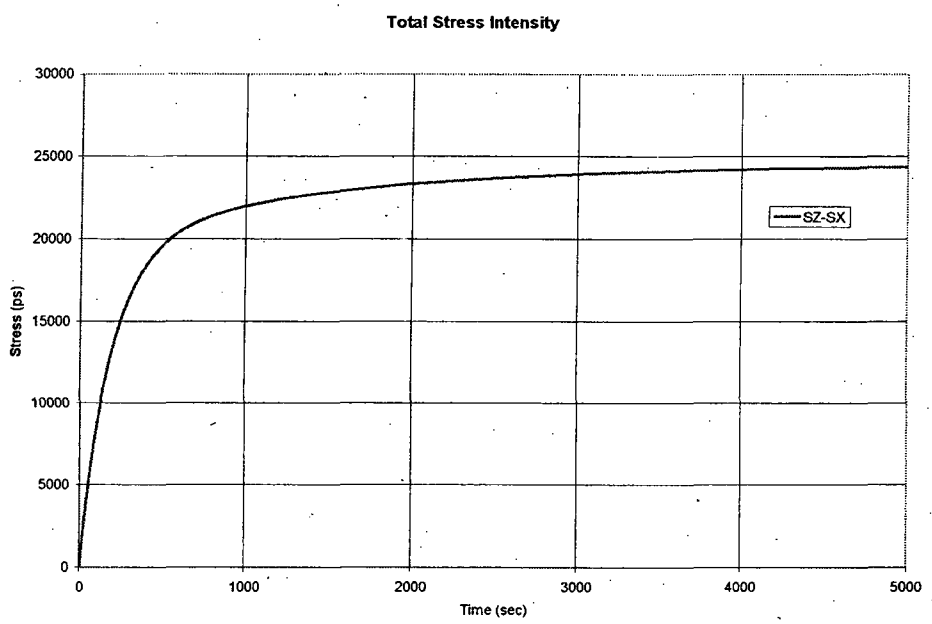


Figure 3-16: Blend Radius Membrane Plus Bending Stress History for 40% Flow

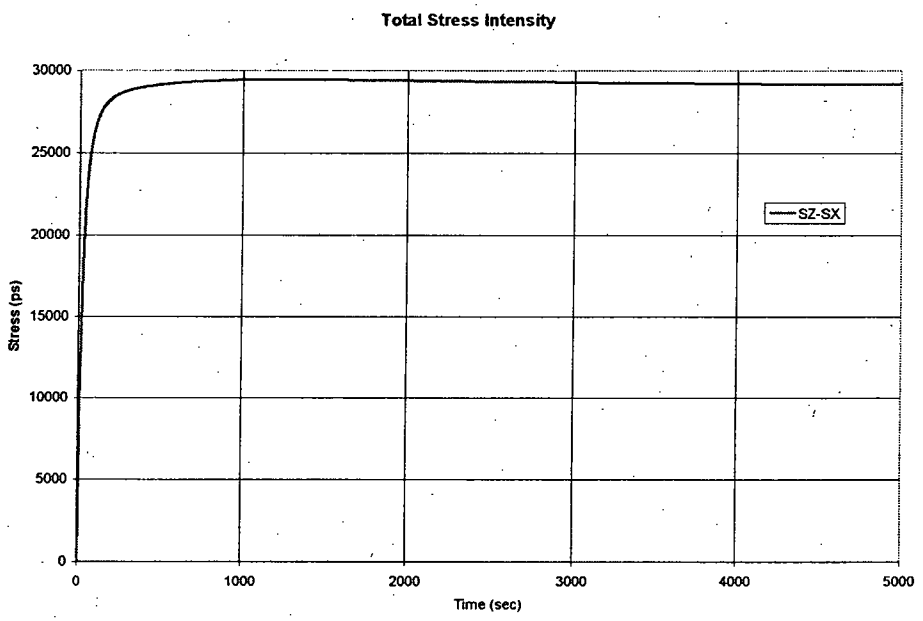


Figure 3-17: Blend Radius Total Stress History for 25% Flow

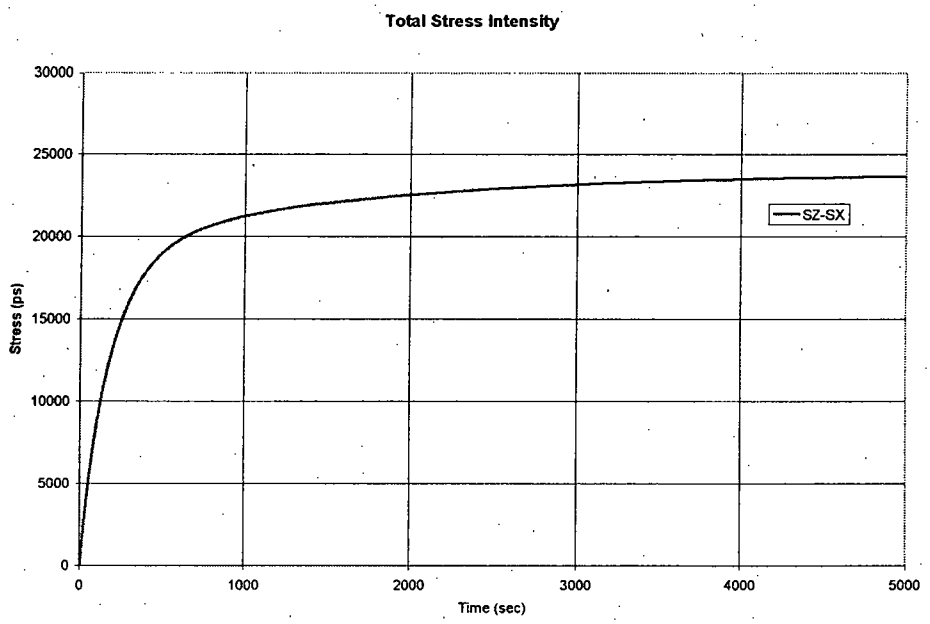


Figure 3-18: Blend Radius Membrane Plus Bending Stress History for 25% Flow

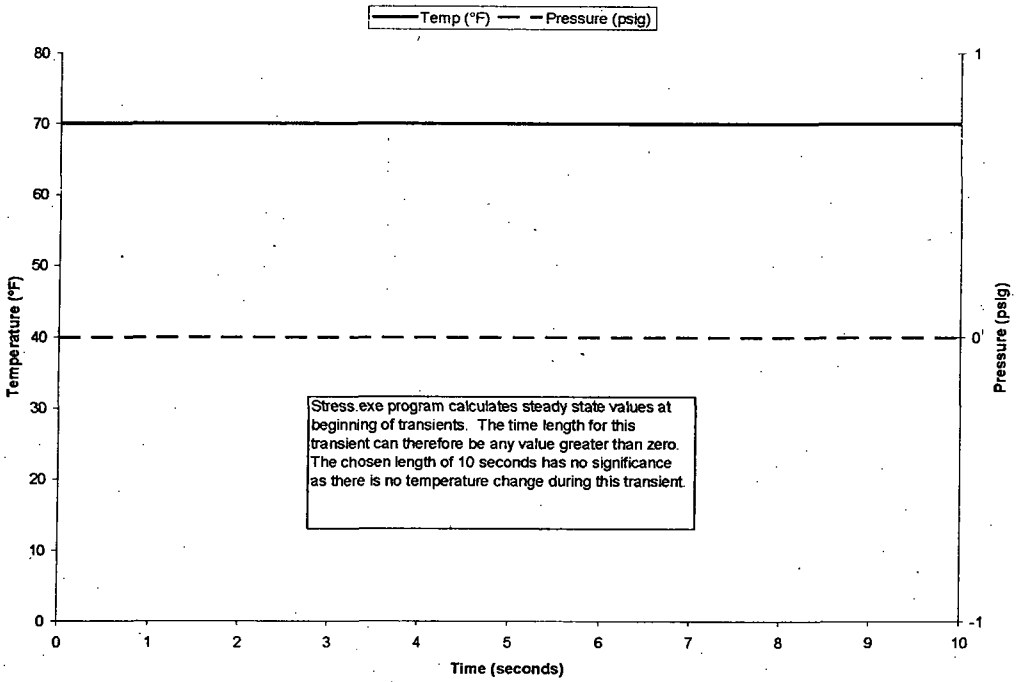


Figure 3-19: Transient 1, Bolt-up

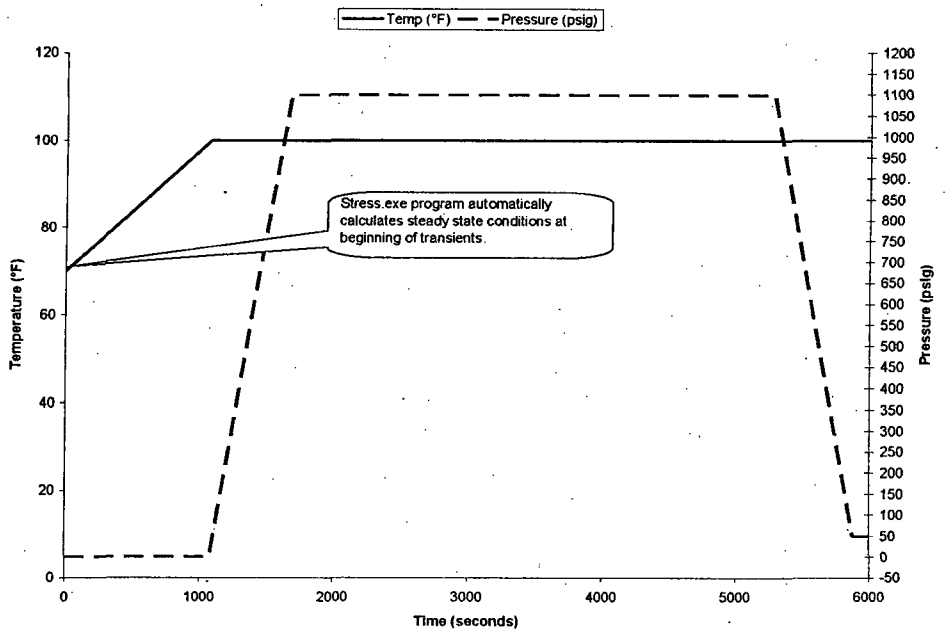


Figure 3-20: Transient 2, Design HYD Test

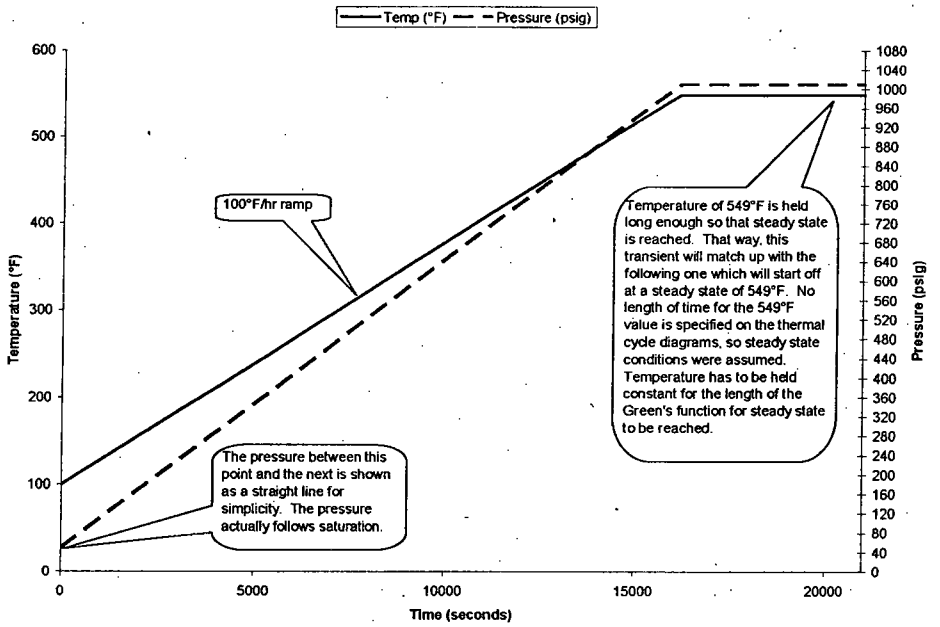


Figure 3-21: Transient 3, Startup

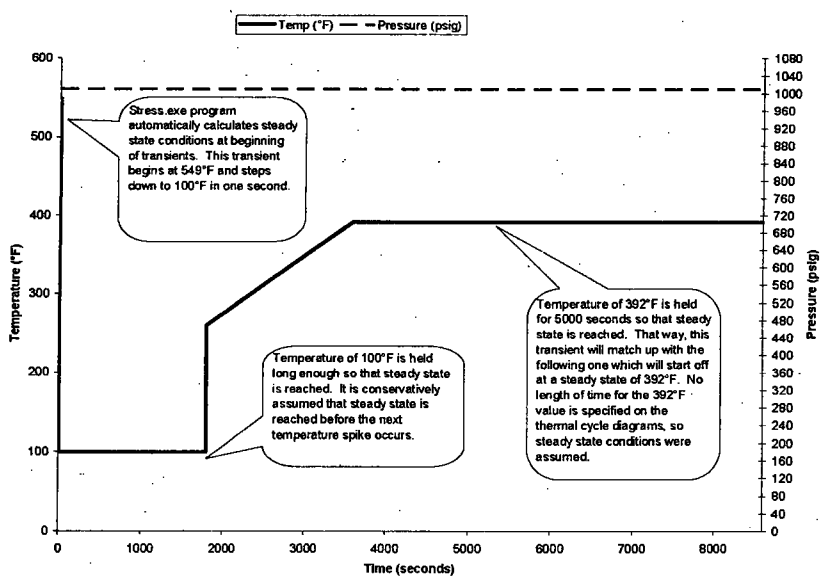


Figure 3-22: Transient 4, Turbine Roll and Increased to Rated Power

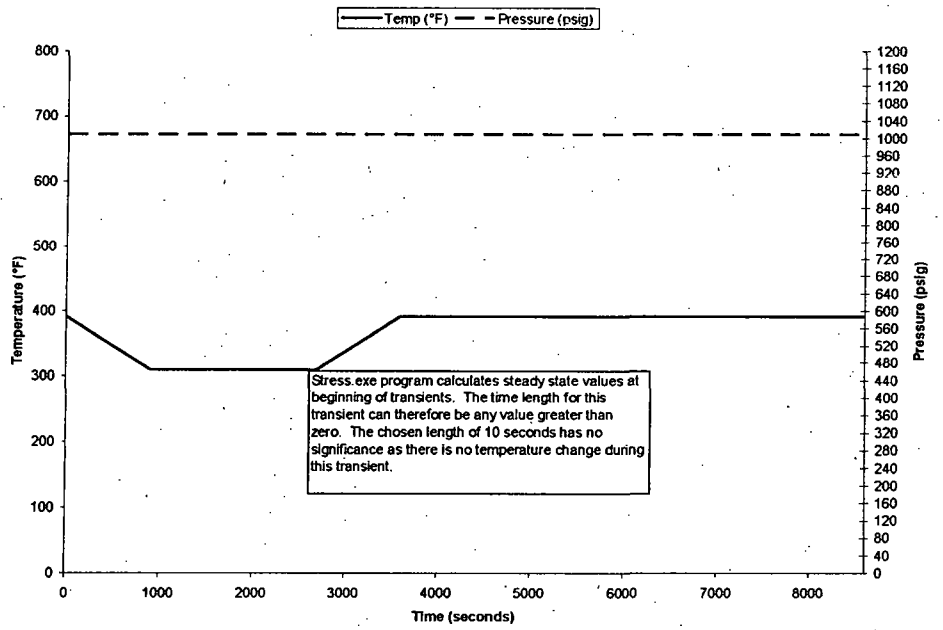


Figure 3-23: Transient 5, Daily Reduction 75% Power

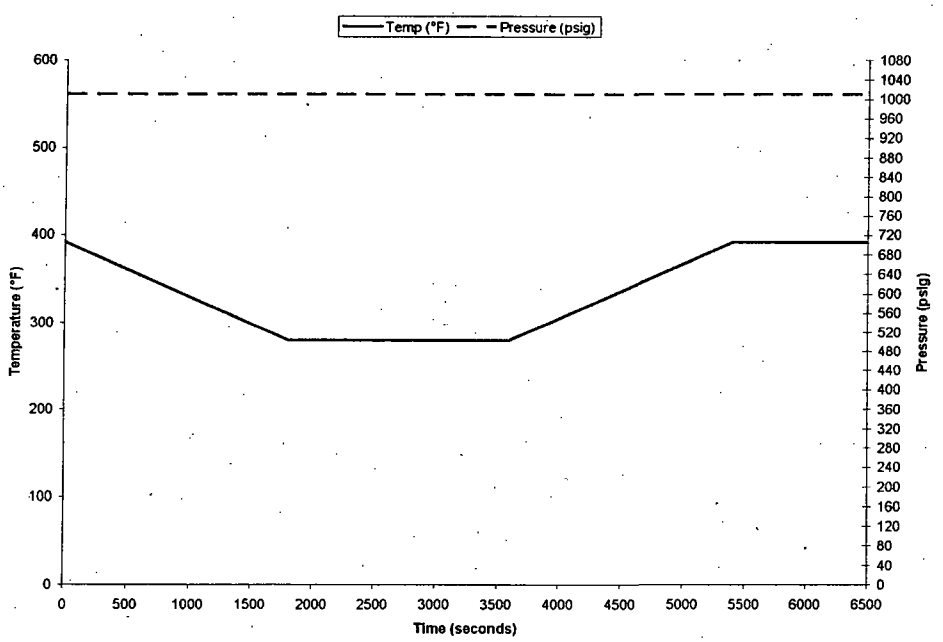


Figure 3-24: Transient 6, Weekly Reduction 50% Power

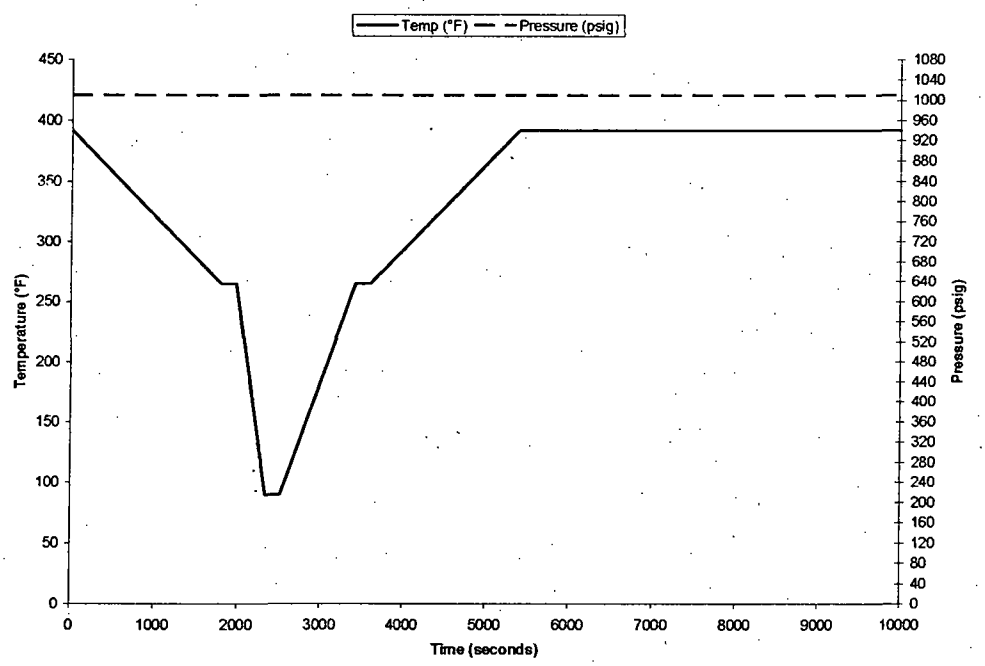


Figure 3-25: Transient 9, Turbine Trip at 25% Power

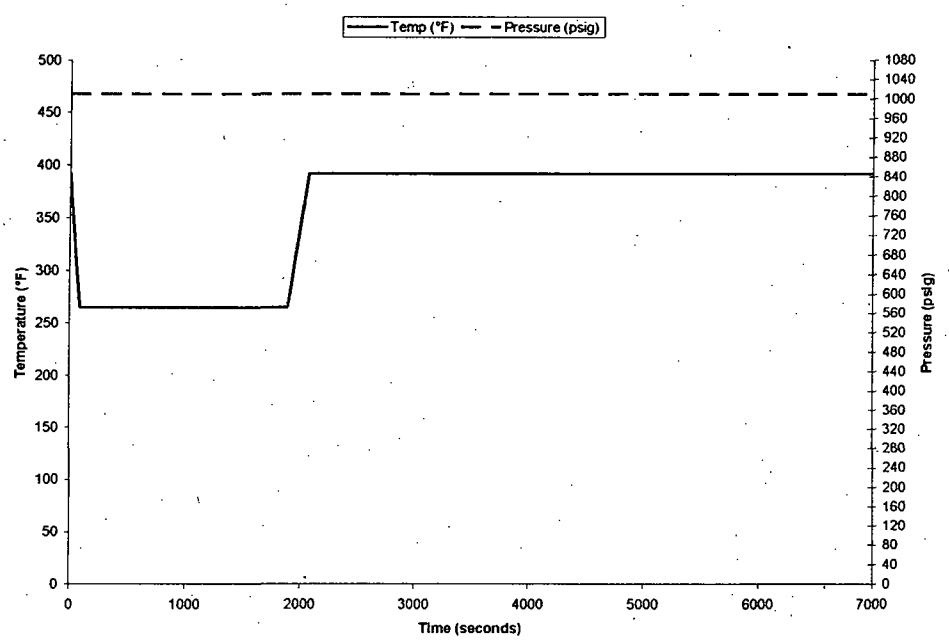


Figure 3-26: Transient 10, Feedwater Bypass

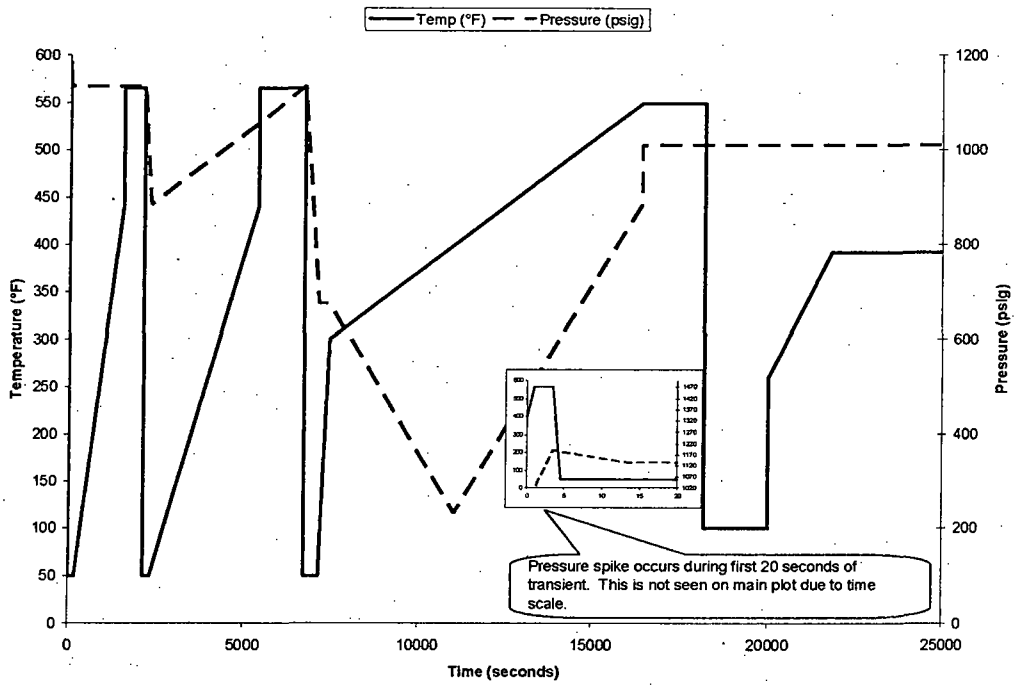


Figure 3-27: Transient 11, Loss of Feedwater Pumps

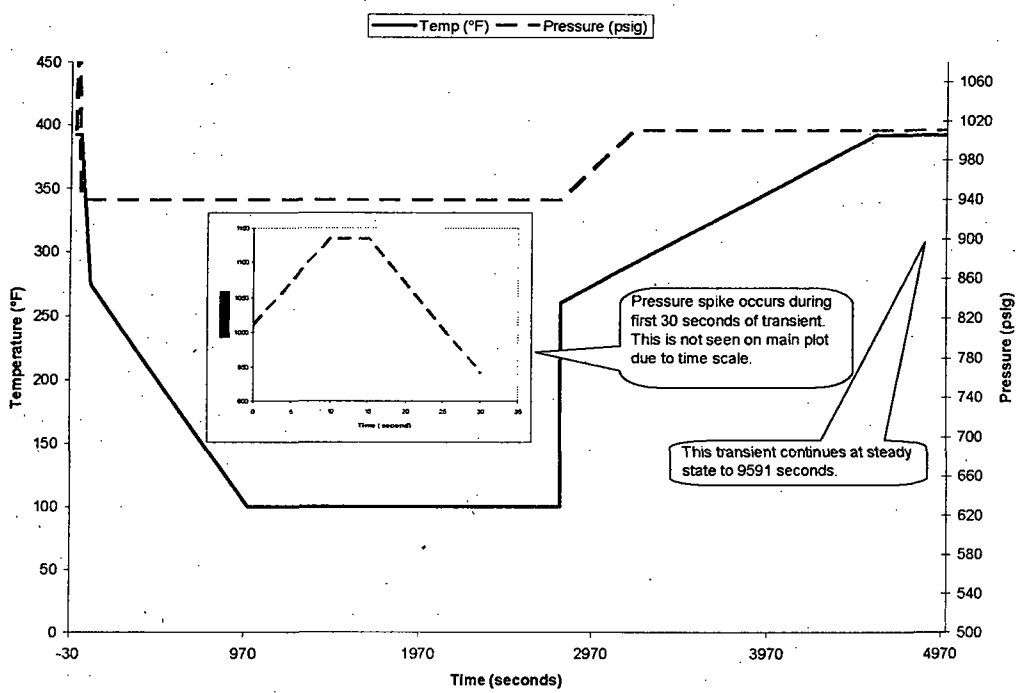


Figure 3-28: Transient 12, Turbine Generator Trip

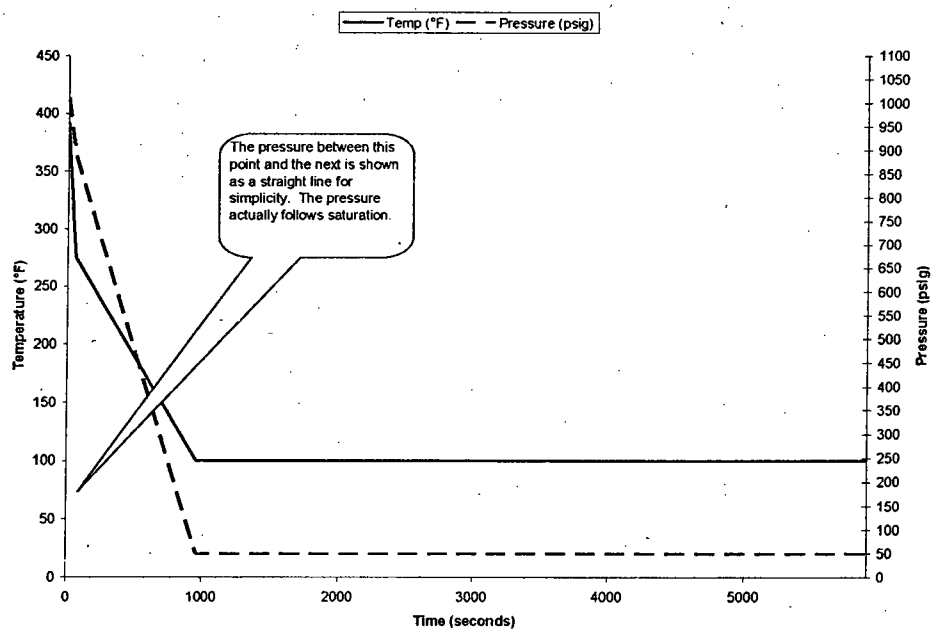


Figure 3-29: Transient 14, SRV Blowdown

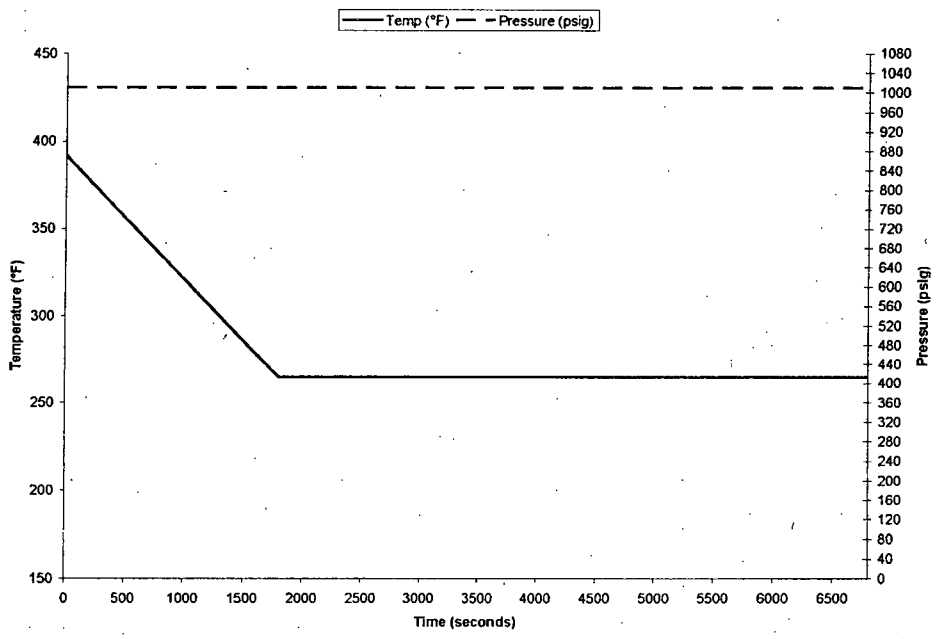


Figure 3-30: Transient 19, Reduction to 0% Power

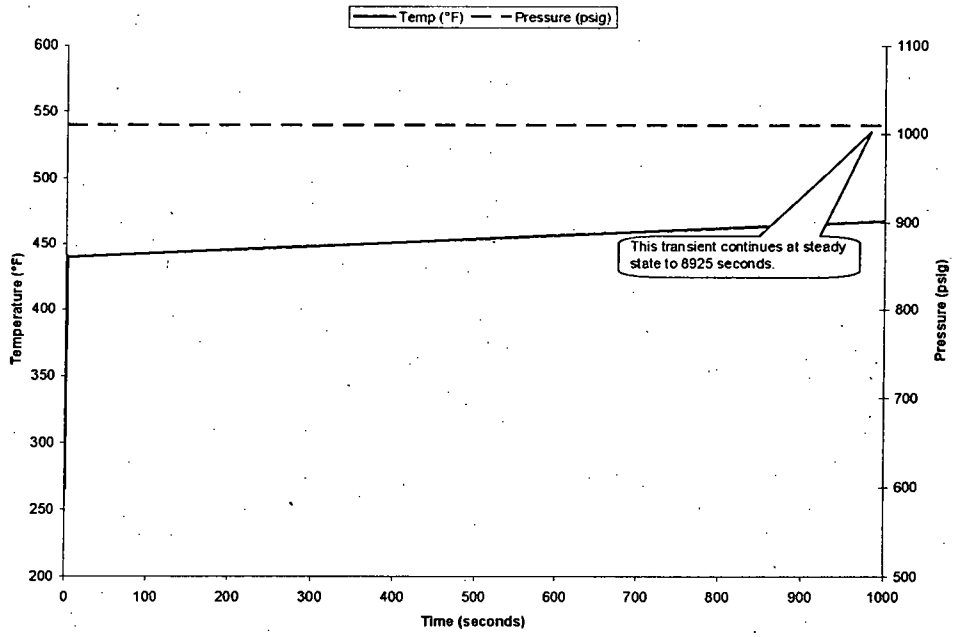


Figure 3-31: Transient 20, Hot Standby (Heatup Portion)

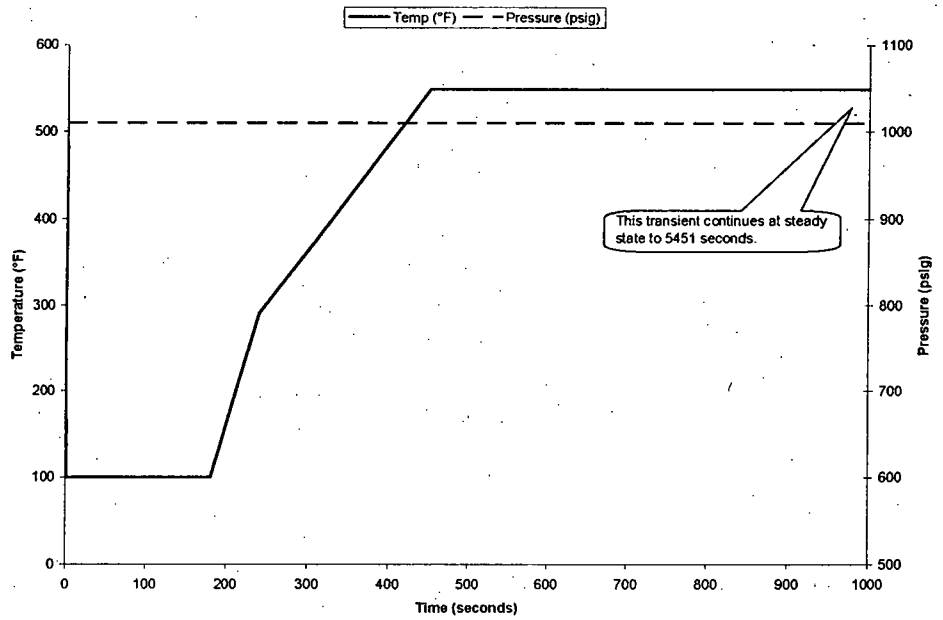


Figure 3-32: Transient 20A, Hot Standby (Feedwater Injection Portion)

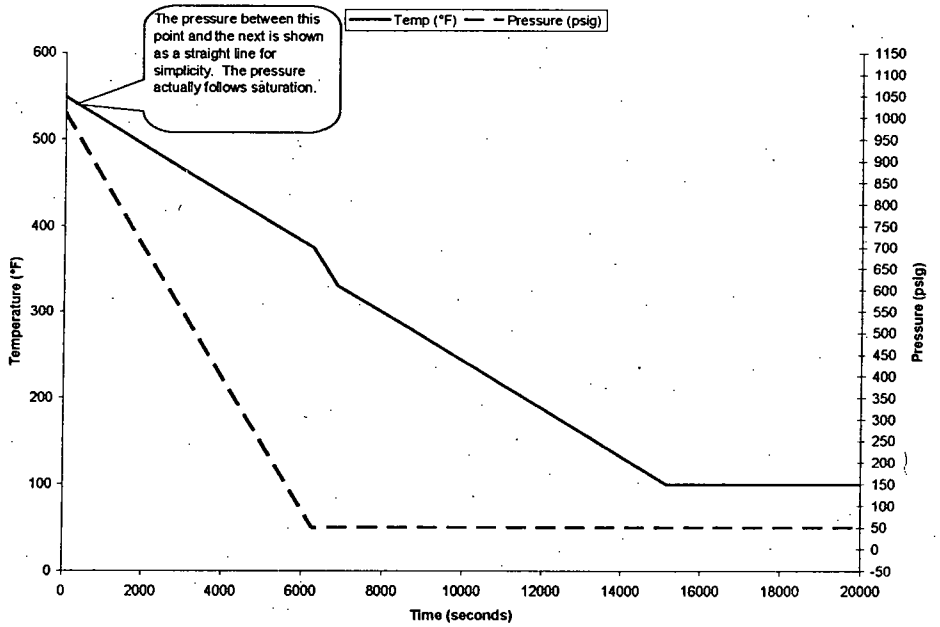


Figure 3-33: Transient 21-23, Shutdown

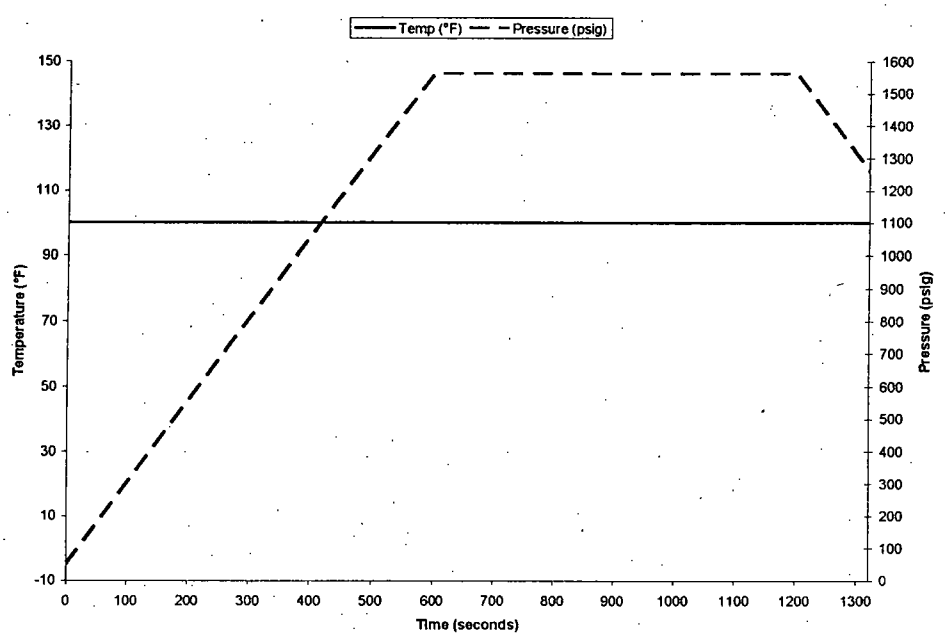


Figure 3-34: Transient 24, Hydrostatic Test

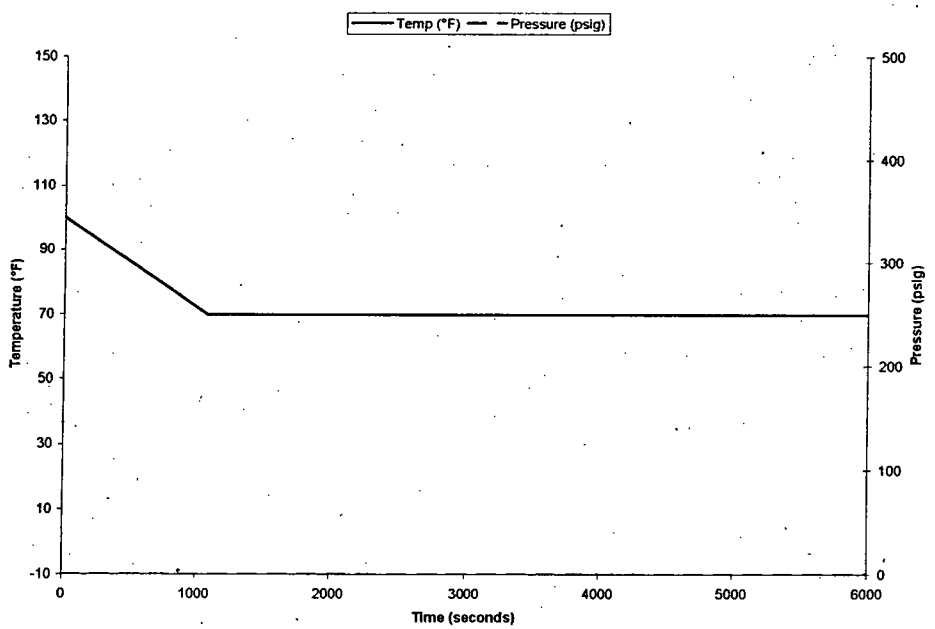


Figure 3-35: Transient 25, Unbolt

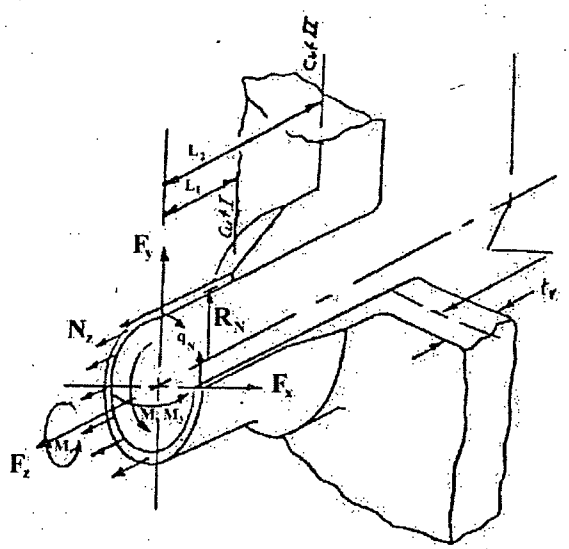


Figure 3-36: External Forces and Moments on the Feedwater Nozzle

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY FW nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [15]. Fatigue analysis was performed in the Reference [23] calculation for the two locations identified in Section 3.1.2 using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [19].

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the blend radius) and Table 4-2 (for the safe end) show the final thermal peak and valley output. The pressure values for Column 6 in each table were determined from the transient pressures specified in Tables 3-4 and 3-5. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in Columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-4 and 3-5.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The results of the fatigue analysis are presented in Tables 4-4 and 4-5 for the safe end and blend radius for 60 years, respectively. The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years.

Table 4-1: Feedwater Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
2	0	0	0	70	0	0	0	0	0	0.00	0.00	120
	1680	0	0	100	1100	41506.3	40318.3	15.77042	15.77042	41522.07	40334.07	120
	10880	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	120
3	0	29166	23676	100	50	1886.65	1832.65	15.77042	15.77042	31068.42	25524.42	300
	16782.8	-3577	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34281.53	33629.73	300
	21164	-3532	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34326.53	33629.73	300
4	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	1801.9	29465	22266	244.004	1010	38110.33	37019.53	91.47053	91.47053	67666.80	59377.00	300
	8602	7720	6749	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43937.80	300
5	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
	2229.8	13598	11941	311.002	1010	38110.33	37019.53	126.6901	126.6901	51835.02	49087.22	10000
	8600	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
6	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
	2820.3	15742	13892	280.691	1010	38110.33	37019.53	110.7562	110.7562	53963.09	51022.29	2000
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
9	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	2524	29006	23417	118.311	1010	38110.33	37019.53	25.39616	25.39616	67141.73	60461.93	10
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
10	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
	1632.4	16828	14701	267.399	1010	38110.33	37019.53	103.7688	103.7688	55042.10	51824.30	70
	7070	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
11	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	3.5	6620	6632	565	1190	44902.27	43617.07	260.2119	260.2119	51782.48	50509.28	10
	4.5	6190	6608	50	1185	44713.61	43433.81	10.51361	10.51361	50914.12	50052.32	10
	194.5	31720	21067	109.348	1135	42826.96	41601.16	20.68448	20.68448	74567.64	62688.84	10
	2166.3	-4761	-1859	513.483	972	36676.48	35626.72	-233.1304	-233.1304	31682.35	33534.59	10
	2362.5	31268	22070	102.255	1010	38110.33	37019.53	16.95583	16.95583	69395.29	59106.49	10
	6728.3	-4913	-3149	513.448	1010	38110.33	37019.53	-233.112	-233.112	32964.22	33637.42	10
	7149.9	32114	21472	83.333	1010	38110.33	37019.53	7.0089	7.0089	70231.34	58498.54	10
	18213.3	-3565	-3162	503.978	1010	38110.33	37019.53	-228.1338	-228.1338	34317.20	33629.40	10
	19122.6	29156	23083	100.048	1010	38110.33	37019.53	15.79565	15.79565	67282.13	60118.33	10
	26814.5	7720	6410	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43598.80	10
12	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	60
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	60
	2033.7	28648	25301	132.007	940	35469.02	34453.82	32.59588	32.59588	64149.62	59787.42	60
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
13	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	10	7720	6752	392	1375	51882.88	50397.88	169.2692	169.2692	59772.14	57319.14	1
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	1
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	1
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
14	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	5960	28487	25650	100	50	1886.65	1832.65	15.77042	15.77042	30389.42	27498.42	1
15	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	228
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	228
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	228
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
19	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	300
	6800	16752	14971	265	1010	38110.33	37019.53	102.5077	102.5077	54964.84	52093.04	300
20	0	17151	13815	265	1010	38110.33	37019.53	102.5077	102.5077	55363.84	50937.04	300
	8925	-3531	-3146	549	1010	38110.33	37019.53	-251.801	-251.801	34327.53	33621.73	300
20A	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	183	28102	12153	233	1010	38110.33	37019.53	85.68595	85.68595	66298.02	49258.22	300
	5451	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
21-23	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	20144	29168	23656	100	50	1886.65	1832.65	15.77042	15.77042	31070.42	25504.42	300
24	0	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
	600	0	0	100	1563	58976.68	57288.64	15.77042	15.77042	58992.45	57304.41	1
	2400	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
25	0	0	0	100	0	0	0	15.77042	15.77042	15.77	15.77	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

For notes, see last page of table...

Table 4-1: Feedwater Nozzle Blend Radius Stress Summary (continued)

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 3-4.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 37733)/1000 [Table3, 10].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 36653)/1000 [Table 3, 10].
Column 9: Total external stress from calculation in Table 3-2, 265.47 psi*(Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 4-2: Feedwater Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
2	0	0	0	70	0	0	0	0	0	0.00	0.00	120
	1680	0	0	100	1100	9780.1	9562.3	339.0875	339.0875	10119.19	9901.39	120
	6960	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	120
3	0	-170	-165	100	50	444.55	434.65	-339.0875	-339.0875	-64.54	-69.44	300
	153.2	-235	-212	104.256	50	444.55	434.65	-387.1927	-387.1927	-177.64	-164.54	300
	16328.2	2	3	549	1010	8979.91	8779.93	5414.097	5414.097	14396.01	14197.03	300
	16664	-1	0	549	1010	8979.91	8779.93	-5414.097	5414.097	3564.81	14194.03	300
4	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	3.6	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	1804.6	-15889	-11224	260.286	1010	8979.91	8779.93	-2150.787	-2150.787	-9059.88	-4594.86	300
	4102	21	23	392	1010	8979.91	8779.93	3639.539	3639.539	12640.45	12442.47	300
5	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
	900.1	244	189	310	1010	8979.91	8779.93	2712.7	2712.7	11936.61	11681.63	10000
	3600	-169	-110	392	1010	8979.91	8779.93	-3639.539	-3639.539	5171.37	5030.39	10000
	3684.4	33	35	392	1010	8979.91	8779.93	3639.539	3639.539	12652.45	12454.47	10000
	4100	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
6	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
	1800.1	196	159	280	1010	8979.91	8779.93	2373.612	2373.612	11549.52	11312.54	2000
	5400.2	-108	-68	392	1010	8979.91	8779.93	-3639.539	-3639.539	5232.37	5072.39	2000
	5496.6	29	31	392	1010	8979.91	8779.93	3639.539	3639.539	12648.45	12450.47	2000
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
9	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
	97.3	180	137	385.135	1010	8979.91	8779.93	3561.945	3561.945	12721.85	12478.87	10
	1884.1	63	65	265	1010	8979.91	8779.93	2204.069	2204.069	11246.98	11049.00	10
	2059.2	1161	859	226.597	1010	8979.91	8779.93	1770.003	1770.003	11910.91	11408.93	10
	3420.1	-334	-211	265	1010	8979.91	8779.93	-2204.069	-2204.069	6441.84	6364.86	10
	3490.2	97	98	265	1010	8979.91	8779.93	2204.069	2204.069	11280.98	11082.00	10
	5400.1	-126	-80	392	1010	8979.91	8779.93	-3639.539	-3639.539	5214.37	5060.39	10
	5470.6	31	32	392	1010	8979.91	8779.93	3639.539	3639.539	12650.45	12451.47	10
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
10	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
	77.1	2308	3188	285.461	1010	8979.91	8779.93	2435.338	2435.338	13723.25	14403.27	70
	169.4	-12	-13	265	1010	8979.91	8779.93	-2204.069	-2204.069	6763.84	6562.86	70
	1890	74	72	265	1010	8979.91	8779.93	2204.069	2204.069	11257.98	11056.00	70
	1968.2	-1069	-1511	322.362	1010	8979.91	8779.93	-2852.427	-2852.427	5058.48	4416.50	70
	2147.2	91	90	392	1010	8979.91	8779.93	3639.539	3639.539	12710.45	12509.47	70
	2570	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
11	0	-29	-27	392	1010	8979.91	8779.93	-3639.539	-3639.539	5311.37	5113.39	10
	2.9	-20317	-13859	565	1147	10197.98	9970.871	-5594.944	-5594.944	-15713.97	-9483.07	10
	6.8	42852	29563	565	1172	10420.25	10188.2	5594.944	5594.944	58867.20	45346.14	10
	1567.4	-15216	-10526	565	1135	10091.29	9866.555	-5594.944	-5594.944	-10719.66	-6254.39	10
	2168.4	60377	41773	50	1134	10082.39	9857.862	-226.0583	-226.0583	70233.34	51404.80	10
	5409.4	-14924	-10329	565	1054	9371.114	9162.422	-5594.944	-5594.944	-11147.83	-6761.52	10
	6730.4	60377	41773	50	1133	10073.5	9849.169	-226.0583	-226.0583	70224.44	51396.11	10
	7243.2	-1965	-1434	128.917	675	6001.425	5867.775	-665.9339	-665.9339	3370.49	3767.84	10
	18215.4	52636	36417	100	1010	8979.91	8779.93	339.0875	339.0875	61955.00	45536.02	10
	20015.5	-24511	-16189	260.183	1010	8979.91	8779.93	-2149.623	-2149.623	-17680.71	-9558.69	10
	22314.5	22	23	392	937	8330.867	8145.341	3639.539	3639.539	11992.41	11807.88	10
12	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	60
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	60
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	60
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	60
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
13	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1
	10	23	22	392	1375	12225.13	11952.88	3639.539	3639.539	15887.66	15614.41	1
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	1
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	1
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	1
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1

For notes, see last page of table...

Table 4-2: Feedwater Nozzle Safe End Stress Summary (continued)

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
14	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	1
	60	4383	3174	275	885	7868.535	7693.305	2317.098	2317.098	14568.63	13184.40	1
	148	420	300	258.492	803	7139.473	6980.479	2130.509	2130.509	9689.98	9410.99	1
	960	544	424	100	50	444.55	434.65	339.0875	339.0875	1327.64	1197.74	1
	1460	137	139	100	50	444.55	434.65	339.0875	339.0875	920.64	912.74	1
15	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	228
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	228
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	228
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	228
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
19	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	300
	1800	219	177	265	1010	8979.91	8779.93	2204.069	2204.069	11402.98	11161.00	300
	2300	72	74	265	1010	8979.91	8779.93	2204.069	2204.069	11255.98	11058.00	300
20	0	-109	-105	265	1010	8979.91	8779.93	-2204.069	-2204.069	6666.84	6470.86	300
	4	-17288	-12189	440.106	1010	8979.91	8779.93	-4183.277	-4183.277	-12491.37	-7592.35	300
	4425	-2	-1	549	1010	8979.91	8779.93	-5414.097	-5414.097	3563.81	3364.83	300
20A	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	4	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	241	-7461	-5525	290.247	1010	8979.91	8779.93	-2489.433	-2489.433	-970.52	765.50	300
	572	128	132	549	1010	8979.91	8779.93	5414.097	5414.097	14522.01	14326.03	300
	951	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
21-23	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	138	62	45	545.167	989	8793.199	8597.377	5370.773	5370.773	14225.97	14013.15	300
	6264	-5	-20	374.97	50	444.55	434.65	-3447.05	-3447.05	-3007.50	-3032.40	300
	6390	104	59	366.172	50	444.55	434.65	3347.607	3347.607	3896.16	3841.26	300
	15644	-173	-167	100	50	444.55	434.65	-339.0875	-339.0875	-67.54	-71.44	300
24	0	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
	600	0	0	100	1563	13896.63	13587.16	339.0875	339.0875	14235.72	13926.25	1
	2400	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
25	0	0	0	100	0	0	0	339.0875	339.0875	339.09	339.09	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 3-5.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 8891)/1000 [Table 3, 10].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 8693)/1000 [Table3, 10].
Column 9: Total external stress from calculation in Table 3-2, 5707.97 psi*(Column 5-70°F)/(575°F -70°F).
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Feedwater Nozzle Fatigue Analysis

	Blend Radius	Safe End
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel) [15]	3.0 & 0.2 (carbon steel) [15]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17800 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [15]	30.0×10^6 psi [15]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [10]	28.1×10^6 psi [10]
The Geometric Stress Concentration Factor K_t	1.0	1.34 [2, page 35 of S4]

Table 4-4: Fatigue Results for Feedwater Nozzle Blend Radius

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 2.0
n = .2
Sm = 26700. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.670E+07 psi
Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
74568.	0.	74568.	62689.	1.000	41892.	1.000E+01	7.488E+03	.0013
70231.	0.	70231.	58499.	1.000	39456.	1.000E+01	8.944E+03	.0011
69395.	0.	69395.	59106.	1.000	38986.	1.000E+01	9.268E+03	.0011
67667.	0.	67667.	59377.	1.000	38015.	9.300E+01	9.988E+03	.0093
67667.	0.	67667.	59377.	1.000	38015.	1.200E+02	9.988E+03	.0120
67667.	0.	67667.	59377.	1.000	38015.	8.700E+01	9.988E+03	.0087
67282.	0.	67282.	60118.	1.000	37799.	1.000E+01	1.018E+04	.0010
67142.	0.	67142.	60462.	1.000	37720.	1.000E+01	1.025E+04	.0010
66791.	0.	66791.	62353.	1.000	37523.	1.000E+00	1.044E+04	.0001
66791.	0.	66791.	62353.	1.000	37523.	1.500E+01	1.044E+04	.0014
66791.	16.	66775.	62337.	1.000	37514.	1.230E+02	1.045E+04	.0118
66791.	1902.	64889.	60505.	1.000	36454.	9.000E+01	1.152E+04	.0078
66298.	1902.	64396.	47410.	1.000	36177.	3.000E+01	1.182E+04	.0025
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	30389.	35909.	21760.	1.000	20173.	1.000E+00	9.581E+04	.0000
66298.	31068.	35230.	23734.	1.000	19792.	2.670E+02	1.038E+05	.0026
64150.	31068.	33081.	34263.	1.000	18585.	3.300E+01	1.303E+05	.0003
64150.	31070.	33079.	34283.	1.000	18584.	2.700E+01	1.303E+05	.0002
59772.	31070.	28702.	31815.	1.000	16125.	1.000E+00	2.222E+05	.0000
58992.	31070.	27922.	31800.	1.000	15687.	1.000E+00	2.519E+05	.0000
55364.	31070.	24293.	25433.	1.000	13648.	2.710E+02	4.757E+05	.0006
55364.	31682.	23681.	17402.	1.000	13304.	1.000E+01	5.703E+05	.0000
55364.	32964.	22400.	17300.	1.000	12584.	1.000E+01	9.414E+05	.0000
55364.	34282.	21082.	17307.	1.000	11844.	9.000E+00	1.912E+06	.0000
55042.	34282.	20761.	18195.	1.000	11663.	7.000E+01	2.231E+06	.0000
54965.	34282.	20683.	18463.	1.000	11620.	2.210E+02	2.310E+06	.0001
54965.	34317.	20648.	18464.	1.000	11600.	1.000E+01	2.348E+06	.0000
54965.	34327.	20638.	18463.	1.000	11595.	6.900E+01	2.358E+06	.0000
53963.	34327.	19637.	17393.	1.000	11032.	2.310E+02	3.757E+06	.0001
53963.	34328.	19636.	17401.	1.000	11031.	3.000E+02	3.758E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	41522.	12441.	10688.	1.000	6989.	1.200E+02	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	6.000E+01	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	1.000E+00	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	8.800E+01	1.000E+20	.0000
51835.	43358.	8477.	7712.	1.000	4762.	1.400E+02	1.000E+20	.0000
51835.	46000.	5835.	5149.	1.000	3278.	3.000E+02	1.000E+20	.0000

51835.	46000.	5835.	5146.	1.000	3278.	9.560E+03	1.000E+20	.0000
51782.	46000.	5783.	6568.	1.000	3249.	1.000E+01	1.000E+20	.0000
50914.	46000.	4915.	6112.	1.000	2761.	1.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	6.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.320E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+04	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
								=======
							TOTAL USAGE FACTOR =	.0636

Table 4-5: Fatigue Results for the Feedwater Nozzle Safe End

LOCATION = LOCATION NO. 1 -- SAFE END
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 3.0
n = .2
Sm = 17800. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.810E+07 psi
Kt = 1.34

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
70233.	-17681.	87914.	60963.	1.283	74422.	1.000E+01	1.338E+03	.0075
70224.	-15714.	85938.	60879.	1.280	72869.	1.000E+01	1.415E+03	.0071
61955.	-12491.	74446.	53128.	1.000	49383.	1.000E+01	4.568E+03	.0022
58867.	-12491.	71359.	52938.	1.000	47700.	1.000E+01	5.094E+03	.0020
53379.	-12491.	65870.	47699.	1.000	43819.	2.800E+02	6.552E+03	.0427
53379.	-11148.	64527.	46869.	1.000	42951.	1.000E+01	6.953E+03	.0014
53379.	-10720.	64099.	46361.	1.000	42631.	1.000E+01	7.109E+03	.0014
53379.	-9972.	63351.	58588.	1.194	53087.	6.000E+01	3.628E+03	.0165
53379.	-9972.	63351.	58588.	1.194	53087.	1.000E+00	3.628E+03	.0003
53379.	-9972.	63351.	58588.	1.194	53087.	2.280E+02	3.628E+03	.0628
53379.	-9060.	62439.	44702.	1.000	41444.	1.100E+01	7.731E+03	.0014
15888.	-9060.	24948.	20209.	1.000	16985.	1.000E+00	1.802E+05	.0000
14569.	-9060.	23629.	17779.	1.000	15840.	1.000E+00	2.410E+05	.0000
14522.	-9060.	23582.	18921.	1.000	16022.	2.870E+02	2.287E+05	.0013
14522.	-3008.	17530.	17358.	1.000	12508.	1.300E+01	9.944E+05	.0000
14396.	-3008.	17404.	17229.	1.000	12417.	2.870E+02	1.083E+06	.0003
14396.	-971.	15367.	13432.	1.000	10641.	1.300E+01	5.165E+06	.0000
14236.	-971.	15206.	13161.	1.000	10506.	1.000E+00	5.563E+06	.0000
14226.	-971.	15196.	13248.	1.000	10516.	2.860E+02	5.531E+06	.0001
14226.	-178.	14404.	14178.	1.000	10262.	1.400E+01	6.379E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	6.000E+01	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	1.000E+00	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	2.250E+02	6.547E+06	.0000
13849.	-68.	13916.	14943.	1.000	10141.	3.000E+00	6.837E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	6.000E+01	8.117E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	2.280E+02	8.117E+06	.0000
13723.	-68.	13791.	14475.	1.000	9989.	9.000E+00	7.465E+06	.0000
13723.	-65.	13788.	14473.	1.000	9987.	6.100E+01	7.474E+06	.0000
12722.	-65.	12786.	12548.	1.000	9103.	1.000E+01	1.729E+07	.0000
12710.	-65.	12775.	12579.	1.000	9102.	7.000E+01	1.730E+07	.0000
12652.	-65.	12717.	12524.	1.000	9061.	1.590E+02	1.833E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.200E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	339.	12313.	12115.	1.000	8772.	1.230E+02	2.905E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.200E+02	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	921.	11732.	11542.	1.000	8357.	1.000E+00	5.462E+07	.0000
12652.	1328.	11325.	11257.	1.000	8088.	1.000E+00	7.100E+07	.0000
12652.	3370.	9282.	8687.	1.000	6531.	1.000E+01	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000

12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3564.	9089.	9090.	1.000	6501.	3.000E+02	1.000E+20	.0000
12652.	3565.	9088.	-1740.	1.000	4535.	3.000E+02	1.000E+20	.0000
12652.	3896.	8756.	8613.	1.000	6237.	3.000E+02	1.000E+20	.0000
12652.	5058.	7594.	8038.	1.000	5513.	7.000E+01	1.000E+20	.0000
12652.	5171.	7481.	7424.	1.000	5341.	7.048E+03	1.000E+20	.0000
12650.	5171.	7479.	7421.	1.000	5339.	1.000E+01	1.000E+20	.0000
12648.	5171.	7477.	7420.	1.000	5338.	2.000E+03	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12641.	5171.	7470.	7412.	1.000	5333.	2.240E+02	1.000E+20	.0000
12641.	5214.	7427.	7382.	1.000	5304.	1.000E+01	1.000E+20	.0000
12641.	5232.	7409.	7370.	1.000	5293.	2.000E+03	1.000E+20	.0000
12641.	5311.	7330.	7329.	1.000	5243.	1.000E+01	1.000E+20	.0000
12641.	6442.	6200.	6078.	1.000	4412.	1.000E+01	1.000E+20	.0000
12641.	6667.	5975.	5972.	1.000	4273.	3.000E+02	1.000E+20	.0000
12641.	6764.	5878.	5880.	1.000	4205.	7.000E+01	1.000E+20	.0000
12641.	9690.	2951.	3031.	1.000	2126.	1.000E+00	1.000E+20	.0000
12641.	10119.	2522.	2541.	1.000	1808.	1.200E+02	1.000E+20	.0000
12641.	11247.	1394.	1393.	1.000	997.	1.000E+01	1.000E+20	.0000
12641.	11256.	1385.	1384.	1.000	991.	3.000E+02	1.000E+20	.0000
12641.	11258.	1383.	1386.	1.000	990.	7.000E+01	1.000E+20	.0000
12641.	11281.	1360.	1360.	1.000	973.	1.000E+01	1.000E+20	.0000
12641.	11403.	1238.	1281.	1.000	894.	3.000E+02	1.000E+20	.0000
12641.	11550.	1092:	1130.	1.000	788.	2.000E+03	1.000E+20	.0000
12641.	11911.	731.	1034.	1.000	578.	1.000E+01	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	4.555E+03	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	5.445E+03	1.000E+20	.0000
12641.	11992.	649.	635.	1.000	462.	1.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	6.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	1.000E+00	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	2.280E+02	1.000E+20	.0000
12641.	12640.	1.	0.	1.000	1.	3.000E+02	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	3.956E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
								=======
							TOTAL USAGE FACTOR =	.1471

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

In the response to NRC request for additional information (RAI) 4.3-H-02 [19], VYNPS states that they have conservatively assumed that fatigue cracks may be present in the clad. VYNPS manages this cracking by performing periodic inspections that were implemented in response to Generic Letters 80-095 and 81-11, and NUREG-0619. The inspection frequency is based on the calculated fatigue crack growth of a postulated flaw in the nozzle inner blend radius. The VYNPS fatigue crack growth calculation uses methods in compliance with GE BWR Owners Group Topical Report "Alternate BWR Feedwater Nozzle Inspection Requirements", GE-NE-523-A71-0594, Revision 1, August 1999 and the associated NRC Final Safety Evaluation (TAC No. MA6787) dated March 10, 2000. The NRC has reviewed and approved this approach to handling FW nozzle inner blend radius cracking (Letter D.H. Dorman (USNRC) to D.A. Reid (VYNPC), Subject: Evaluation of Request for Relief from NUREG-0619 for VYNPS dated 2/6/95, (TAC No. M88803)).

The analysis performed for the feedwater nozzle calculated fatigue in the blend radius base metal, not the clad. This is consistent with the VYNPS position stated in the response to RAI 4.3-H-02, and is also consistent with ASME Code methodology since cladding is structurally neglected in fatigue analyses, per ASME Code, Section III, NB-3122.3 [15].

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [21] for various regions of the VY RPV and attached piping. Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the FW nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV upper region chemistry was assumed for the FW nozzle blend radius location, since this location experiences reactor conditions for all times. FW line chemistry was assumed for the FW nozzle safe end location, since this location experiences feedwater conditions for all times.

For the safe end location, the environmental fatigue factors for pre-HWC and post-HWC are both 1.74 from Table 3 of Reference [21] for the RPV FW line. This results in an EAF adjusted CUF as follows:

60-Year CUF, U_{60} = 0.1470 (from Table 4-5)
Overall EAF multiplier, F_{en} = 1.74
60-Year EAF CUF, $U_{60\text{-}env}$ = 0.14709 x 1.74 = 0.2560

The EAF CUF value of 0.2560 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [15]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [16]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82, respectively, from Table 4 of Reference [21] for the RPV upper region. This results in an EAF adjusted CUF as follows:

60-Year CUF, U_{60} = 0.0636 (from Table 4-4)
Overall EAF multiplier, F_{en} = (11.14 x 53% + 8.82 x 47%) = 10.05
60-Year EAF CUF, $U_{60\text{-}env}$ = 0.0636 x 10.05 = 0.6392

The EAF CUF value of 0.6392 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY FW nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the FW nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [16] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY FW nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [15]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.1470, and the CUF for the limiting blend radius location is 0.0636. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY FW nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.2560, and the EAF CUF for the limiting blend radius location is 0.6392. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

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EXHIBIT M

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Project No.: VY-16Q
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Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Reactor Recirculation Outlet Nozzle

Prepared for:
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Contract Number: 10150394

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Structural Integrity Associates, Inc.

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Document Number: SIR-07-141-NPS

Title: Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Reactor Recirculation Outlet Nozzle

Client: Entergy Nuclear Operations, Inc.

SI Project Number: VY-16Q

Section	Pages	Revision	Date	Comments
1.0	1-1 – 1-5	0	7/26/07	Initial Issue
2.0	2-1 – 2-3			
3.0	3-1 – 3-34			
4.0	4-1 – 4-11			
5.0	5-1 – 5-2			
6.0	6-1			
7.0	7-1 – 7-2			

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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) value for the reactor pressure vessel (RPV) reactor recirculation outlet nozzle is reported as 0.810. Application of environmentally assisted fatigue (EAF) multipliers, as required for the license renewal period, resulted in an unacceptable EAF CUF value of 1.98. Therefore, further refined analysis is necessary to show acceptable EAF CUF results for this component.

This report documents a refined fatigue evaluation for the VY reactor recirculation outlet nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the reactor recirculation outlet nozzle are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The resulting fatigue results will be used as a replacement to the value previously reported in the VY LRA.

The refined evaluation summarized in this report included development of a detailed finite element model of the reactor recirculation outlet nozzle, including relevant portions of the safe end, the nozzle forging, a portion of the vessel shell, and cladding as shown in the applicable drawings [2, 3]. Thermal and pressure stress histories were developed for relevant transients affecting the reactor recirculation outlet nozzle, including any effects of Extended Power Uprate (EPU), as specified by the VY RPV Design Specification [4], the VY EPU Design Specification [5] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress intensities and primary plus secondary stress intensities for use in a subsequent fatigue evaluation [11]. Stress intensities were also included due to loads from the attached piping for application in the stress/fatigue analysis based on the bounding reaction loads obtained from the relevant design document. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code [17], and was performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

For the reactor recirculation outlet nozzle evaluated as a part of this work, stress intensity histories were computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [6]. A general outline is provided in this section.

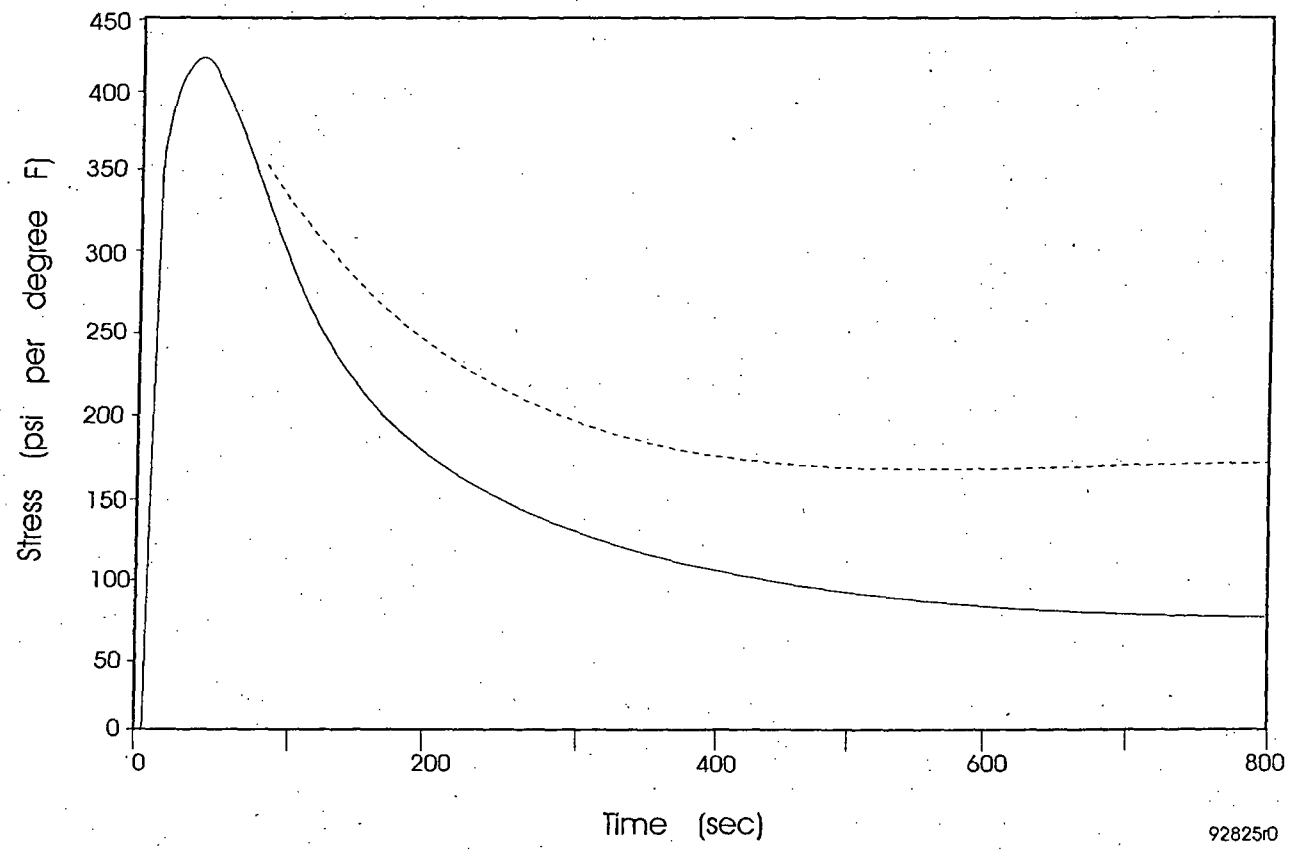
A Green's Function is derived by using finite element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress intensity, and the thermal stress intensity response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound

the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor recirculation outlet nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements [17].



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1. Typical Green's Functions for Thermal Transient Stress

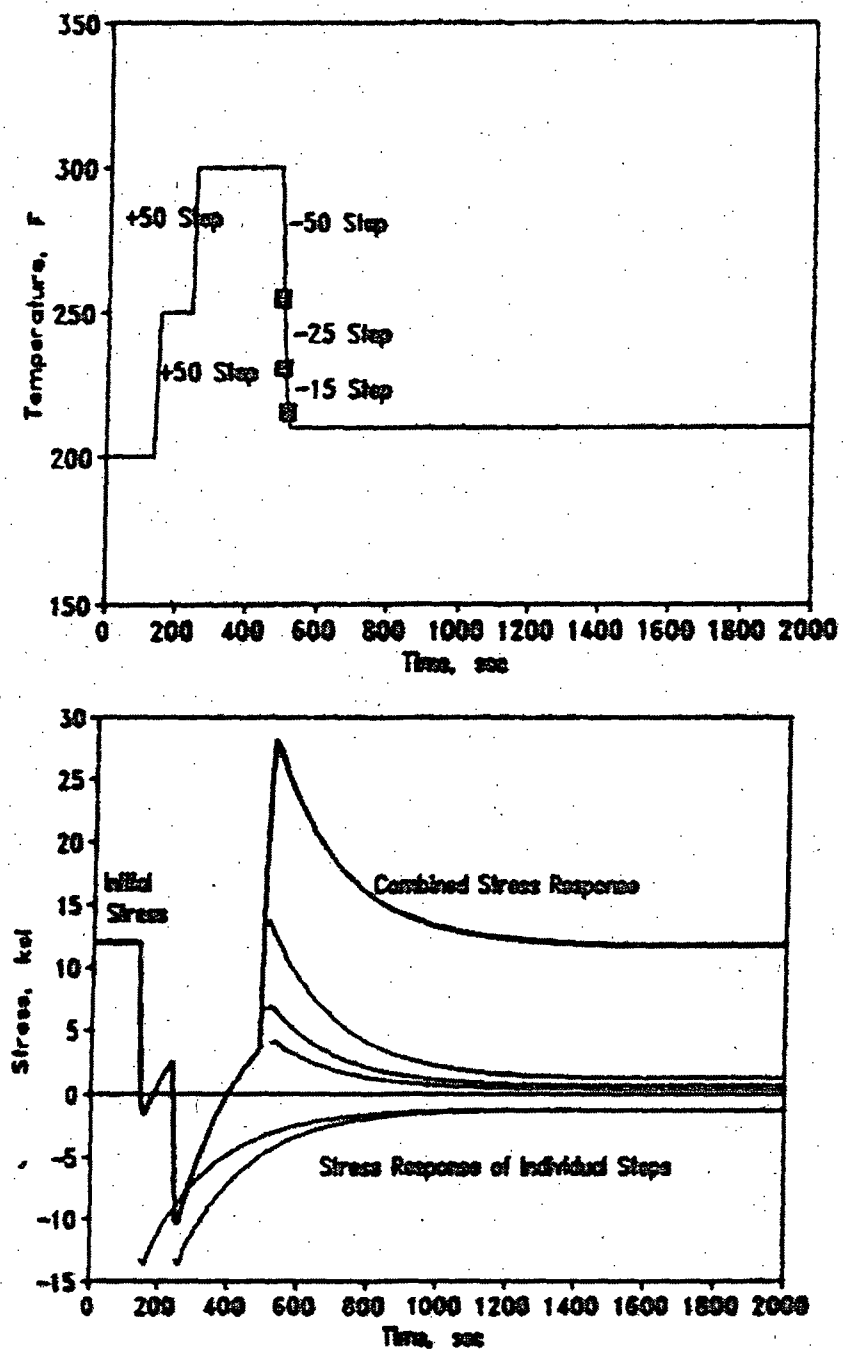


Figure 1-2. Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

An ANSYS [7] finite element model (FEM) of the VY reactor recirculation outlet nozzle configuration was developed and used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [8] calculation.

The materials of the various components of the model are listed below:

- Safe End – SA182 F316 (16Cr-12Ni-2Mo)
- Piping – SA376 TP316 (16Cr-12Ni-2Mo)
- Nozzle Forging – SA508 Class 2 (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 Type 304 (18Cr-8Ni)

The radius of the RPV was increased by a factor of two to account for the fact that the vessel portion of the two dimensional axisymmetric finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], and are shown in Table 2-1. The properties for the Green's Functions were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which was applied to the FEM model for Green's Function development. Due to the thermal shocks having a different temperature range at the nozzle blend radius and safe end for the Improper Start transient (Transient 9), this transient was run separately. Material properties were evaluated at 400°F for all locations except for the safe end and piping, which were evaluated using material properties at 300°F since the temperature is lower at this location and the properties at 300°F are closest to the average temperature of 330°F.

The finite element model is shown in Figure 2-1.

Table 2-1. Material Properties

For the Green's Functions (evaluated at 300°F):

Material	SA533 Gr. B Mn-1/2Mo-1/2Ni	SA508 Cl. 2 3/4Ni-1/2Mo-1/3Cr-V	SA240 TP 304 18Cr-8Ni	SA18 SA37 16Cr-
Modulus of Elasticity, e+6 psi	28.0	26.7	27.0	2
Coefficient of Thermal Expansion, e-6, in/in/°F	7.7	7.3	9.8	
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	
Thermal Diffusivity, ft^2/hr	0.401	0.401	0.160	0
Specific Heat, Btu/lb-°F	0.119	0.119	0.125	0
Density, lb/in³	0.283	0.283	0.283	0
Poisson's Ratio	0.3	0.3	0.3	

For Transient 9 (evaluated at 400°F – except for the safe end and piping, which are evaluated at 300°F):

Property	SA533 Gr. B Mn-1/2Mo-1/2Ni	SA508 Cl. 2 3/4Ni-1/2Mo-1/3Cr-V	SA240 TP 304 18Cr-8Ni	SA1 SA3 16Cr- @
Modulus of Elasticity, e+6 psi	27.4	26.1	26.5	
Coefficient of Thermal Expansion, e-6, in/in/°F	8.0	7.7	10.2	
Thermal Conductivity, Btu/hr-ft-°F	23.1	23.1	10.4	
Thermal Diffusivity, ft^2/hr	0.378	0.378	0.165	(
Specific Heat, Btu/lb-°F	0.125	0.125	0.129	(
Density, lb/in³	0.283	0.283	0.283	(
Poisson's Ratio	0.3	0.3	0.3	

Note: 1. Material properties are taken from the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], except for de which are assumed typical values and specific heat calculated as $[k/\rho d]/12^3$.

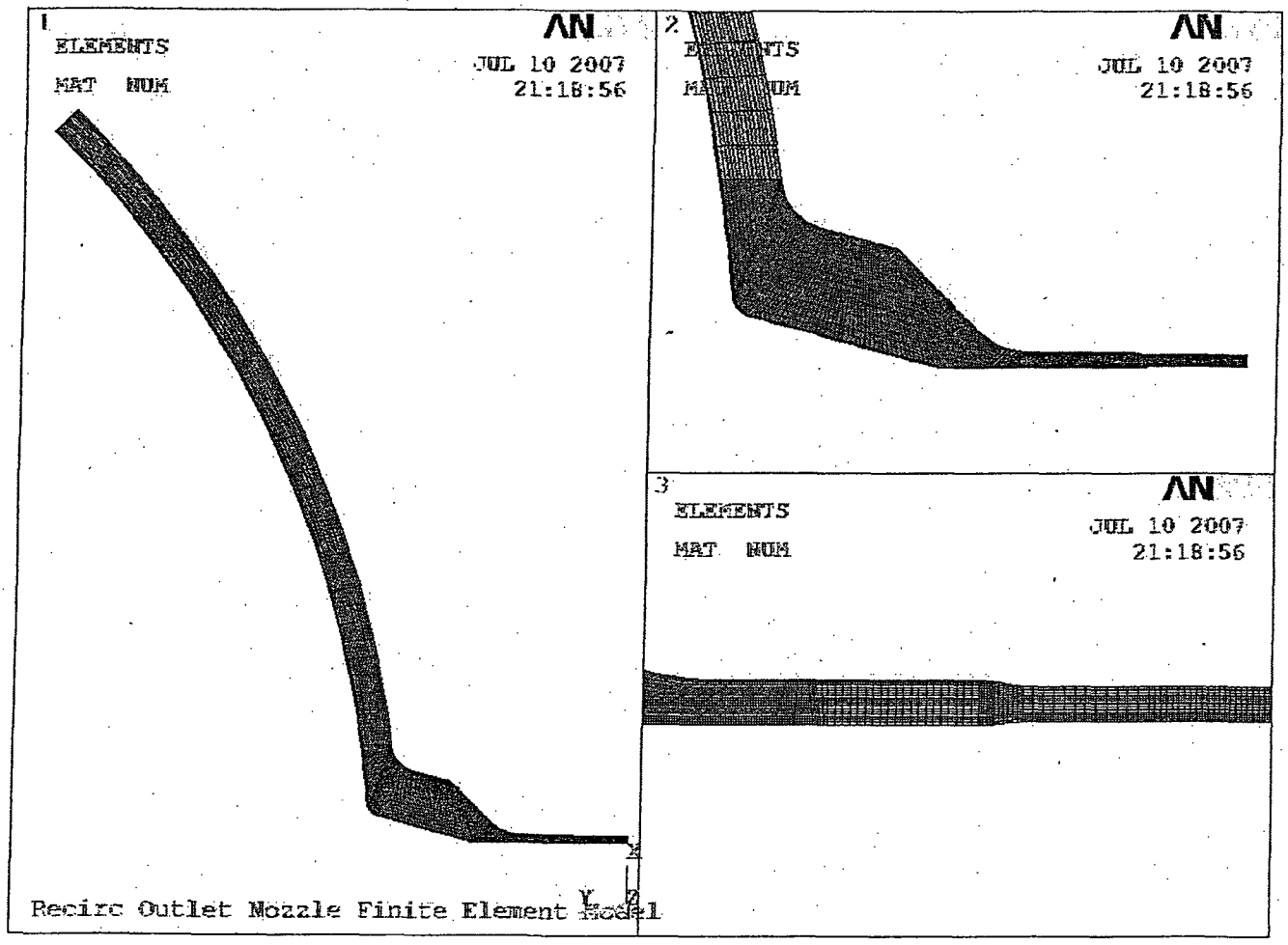


Figure 2-1. VY Reactor Recirculation Outlet Nozzle FEM

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the reactor recirculation outlet nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model described in Section 2.0 of this report. The details of the Green's Function development and associated stress and fatigue evaluations are documented in the Reference [10] and [11] calculations.

3.1 Thermal Loading

Thermal loads were applied to the recirculation outlet nozzle model to generate the Green's Functions. As a first step in the Green's Function process, heat transfer coefficients were determined for various regions of the reactor recirculation outlet nozzle FEM for three different flow cases: (1) 100% reactor recirculation outlet nozzle flow, (2) 50% reactor recirculation outlet nozzle flow and (3) 0% reactor recirculation outlet nozzle flow.

The 100% flow case simulates the operational condition of the reactor recirculation outlet nozzle (i.e., normal recirculation system flow). The heat transfer coefficients for the high flow case are for forced convection. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle due to normal operating conditions.

The 50% flow case simulates a reduced flow condition of the reactor recirculation outlet nozzle (i.e., during power ascension). The heat transfer coefficients for the reduced flow case are also for forced convection. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle under reduced flow conditions.

The 0% flow case simulates a stagnant condition of the reactor recirculation outlet nozzle when recirculation flow is stopped and the entire reactor recirculation outlet nozzle is at the same temperature as the RPV fluid. The heat transfer coefficients for the 0% flow case are for free

convection (stagnant) conditions. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle in the stagnant condition.

The temperature on the exterior of the reactor, nozzle, safe end and pipe was assumed to be 120°F (ambient). Figure 3-1 shows the heat transfer coefficient regions assumed for the reactor recirculation outlet nozzle FEM. The applied heat transfer coefficients and the fluid temperatures are summarized in the sections that follow.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-1, heat transfer coefficients for the Green's Functions were applied as follows:

For Green's Functions:

Region 1

The heat transfer coefficient, h, for 100% flow is 3,577.8 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 50% flow is 2,054.9 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 0% flow is 112.34 BTU/hr-ft^2-°F at 300°F.

Region 2

The heat transfer coefficient for Region 2 was linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used in Region 3.

Region 3

The heat transfer coefficient, h, for 100% flow is 3,361 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 50% flow is 1,930.9 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 0% flow is 112.34 BTU/hr-ft^2-°F at 300°F.

Region 4

The heat transfer coefficient for Region 4 (nozzle blend radius) was linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h, for 100% flow is 1,788.9 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 50% flow is 1,027.4 BTU/hr-ft^2-°F at 300°F.

The heat transfer coefficient, h, for 0% flow is 101 BTU/hr-ft^2-°F at 300°F.

Region 6

The heat transfer coefficient, h, is 0.4 BTU/hr-ft^2-°F.

For all three Green's functions, a 500°F to 100°F thermal shock was evaluated in Regions 1 through 5 to determine the stress response. For Region 6, a constant temperature of 120°F was used.

Referring to Figure 3-2, heat transfer coefficients for Transient 9 were applied as follows:

For Transient 9:

Region 1

The heat transfer coefficient, h, for 12% flow is 672.8 BTU/hr-ft^2-°F at 500°F.

The heat transfer coefficient, h, for 12% flow is 308.2 BTU/hr-ft^2-°F at 100°F.

The fluid temperature shock is: T = 526°F - 130°F - 526°F.

Region 2

The heat transfer coefficient, h, for 12% flow is 632.2 BTU/hr-ft^2-°F at 500°F.

The heat transfer coefficient, h, for 12% flow is 616.6 BTU/hr-ft^2-°F at 300°F.

The fluid temperature shock is: T = 526°F - 268°F - 526°F.

Region 3

The heat transfer coefficient, h, for 12% flow is 336.4 BTU/hr-ft^2-°F at 500°F.

The heat transfer coefficient, h, for 12% flow is 328.0 BTU/hr-ft^2-°F at 300°F.

The fluid temperature shock is:

Case 1: T = 526°F - 268°F - 526°F

Case 2: T = 526°F

Region 4

The heat transfer coefficient, h, is 0.4 BTU/hr-ft^2-°F.

The temperature, T, is 120°F.

Transition Regions

The heat transfer coefficient was linearly transitioned from the value of the heat transfer coefficient used in adjacent regions.

3.1.2 Green's Functions

The three flow-dependent thermal load cases outlined in the previous section were run on the reactor recirculation outlet nozzle FEM with the heat transfer coefficients and the fluid temperature conditions listed in Section 3.1.1. Two locations were selected for analysis (see Figures 3-3 and 3-4):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under high flow conditions. The highest stress intensity due to thermal loading occurred at Node 6395 (see Figure 3-3), on the inside diameter of the nozzle safe end. Therefore, this node was selected for analysis.
2. The critical blend radius location was chosen based upon the highest pressure stress intensity, neglecting the cladding. The critical location is selected as Node 3829, as shown in Figure 3-4.

Twelve stress intensity Green's Functions were developed (i.e., total stress intensity and membrane plus bending stress intensity for each location and each flow case). The total stress intensity Green's Functions for the safe end location at 100%, 50% and 0% flow are shown in Figures 3-5, 3-6 and 3-7, respectively. The total stress intensity Green's Function for the blend radius location at 100%, 50% and 0% flow are shown in Figures 3-8, 3-9 and 3-10, respectively.

3.1.3 Thermal Transients

The transients analyzed for the reactor recirculation outlet nozzle were developed based on the definitions in the original RPV Design Specification [4], as modified for EPU [5], as well as more recent definitions based on BWR operating experience. For BWR operating experience, the transients described in the thermal cycle diagrams for a BWR-4 plant similar in design and vintage to VY were obtained, and plant-specific data from VY was applied to each transient. The resulting thermal cycle diagrams are shown in References [12 and 13]. The final transients evaluated in the stress and fatigue analyses are shown in Figures 3-11 through 3-20.

The number of cycles projected for the 60-year operating life was used for each transient [14]. Tables 3-1 and 3-2 summarize the thermal transients for the safe end and blend radius locations, respectively.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the reactor recirculation outlet nozzle and the RPV wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the FEM. This cap load was calculated as follows:

$$P_{cap} = P \cdot \frac{\left(R_i^2\right)}{\left(R_o^2 - R_i^2\right)}$$

where:

P_{cap}	=	end cap pressure load (psi)
P	=	unit pressure load = 1,000 psi
R_i	=	Inner Radius = 12.969 in
R_o	=	Outer Radius = 14.188 in

The calculated pressure cap load of 5,081.7 psi was applied as a negative value so that it would exert tension on the end of the model. The nodes on the end of the FEM were coupled in the axial direction to ensure mutual displacement of the end of the FEM due to the attached piping. Figures 3-21, 3-22, and 3-23 show the internal pressure distribution, cap load, and symmetry condition applied to the RPV end of the model, respectively.

The internal pressure load case for the blend radius resulted in a total stress intensity of 31,300 psi, and for the safe-end resulted in a total stress intensity of 11,490 psi. The membrane plus bending stress intensity at the blend radius is 33,640 psi and at the safe end is 11,350 psi, respectively.

3.3 Piping Loading

The piping stress intensities (stress caused by the attached piping) were determined for the two evaluated reactor recirculation outlet nozzle locations. The design piping reactions that were used in the stress and fatigue evaluation are defined on the Reference [15] drawing. These loads represent shear and moment loadings on the nozzle resulting from thermal expansion of the attached piping and seismic loads. The loads are as shown in Figure 3-24. The stresses resulting from these loads were calculated by hand using classical structural mechanics formulas, as documented in Reference [11], and are shown in Tables 3-3 and 3-4 for the safe end and blend radius locations, respectively.

Table 3-1: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0 (50%)'
	16164	549	16164	1010	
	16864	549	700	1010	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294 (100%)'
	1	542	1	1010	
	601	542	600	1010	
	602	526	1	1010	
	1302	526	700	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294 (100%)'
	1800	542	1800	1010	
	2100	542	300	1010	
	2460	526	360	1010	
	3060	526	600	1010	
	3960	542	900	1010	
	4260	542	300	1010	
	6060	526	1800	1010	
	6760	526	700	1010	
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0 (0%)'
	3	526	3	1190	
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6773	500	4380	1135	
	7193	300	420	675	14147 (50%)'
	7493	300	300	675	
	11093	400	3600	240	
	16457	549	5364	1010	
	16517	549	60	1010	
	16518	542	1	1010	28294 (100%)'
	17118	542	600	1010	
	17119	526	1	1010	
	17819	526	700	1010	
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294 (100%)'
	10	526	10	1135	
	15	526	5	1135	
	30	526	15	940	
	1830	526	1800	940	
	2250	549	420	1010	
	2310	549	60	1010	
	2311	542	1	1010	
	2911	542	600	1010	
	2912	526	1	1010	
	3612	526	700	1010	
6. Reactor Overpressure 1 Cycle	0	526		1010	28294 (100%)'
	2	526	2	1375	
	32	526	30	940	
	1832	526	1800	940	
	2252	549	420	1010	
	2312	549	60	1010	
	2313	542	1	1010	
	2913	542	600	1010	
	2914	526	1	1010	
	3614	526	700	1010	
7. SRV Blowdown 1 Cycle	0	526		1010	28294 (100%)'
	600	375	600	170	
	11580	70	10980	50	
	12280	70	700	50	
8. SCRAM Other 228 Cycles	0	526		1010	28294 (100%)'
	15	526	15	940	
	1815	526	1800	940	
	2235	549	420	1010	
	2295	549	60	1010	
	2296	542	1	1010	
	2356	542	60	1010	
	2357	526	1	1010	
	3057	526	700	1010	
9. Improper Startup 1 Cycle	0	526		1010	3395 (12%)'
	1	130 (3)	1	1010	
	27	130 (3)	26	1010	
	28	526	1	1010	
	728	526	700	1010	
10. Shutdown 300 Cycles	0	549		1010	14147 (50%)'
	6264	375	6264	170	
	6864	330	600	88	
	16224	70	9360	50	
	16924	70	700	50	
11. Design Hydrostatic Test 120 Cycles	—	100	—	0	1981 (7%)'
				1100	
				50	
12. Hydrostatic Test 1 Cycle	—	100	—	50	1981 (7%)'
				1563	
				50	

Notes:
1. The instant temperature change is assumed as 1 second time step.
2. The number of cycles is for 60 years [14].
3. 130°F is the safe end temperature for this transient. The blend radius has a different temperature for Transient 9. [13]

Note: These transients are the same as in Table 3-2 with the exception of the 700 second steady state time increment that is used. The transients in Table 2 are plotted using a 6000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend radius.

Table 3-2: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0 (50%)'
	16164	549	16164	1010	
	22164	549	6000	1010	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294 (100%)'
	1	542	1	1010	
	601	542	600	1010	
	602	526	1	1010	
	6602	526	6000	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294 (100%)'
	1800	542	1800	1010	
	2100	542	300	1010	
	2460	526	360	1010	
	3060	526	600	1010	
	3960	542	900	1010	
	4260	542	300	1010	
	6060	526	1800	1010	
	12060	526	6000	1010	
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0 (0%)'
	3	526	3	1190	
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1136	
	2393	300	180	885	
	6773	500	4380	1135	
	7193	300	420	675	
	7493	300	300	675	
	10793	400	3600	240	
	12862	549	5369	1010	
	12922	549	60	1010	
	12923	542	1	1010	
	13523	542	600	1010	
	13524	526	1	1010	
	19524	526	6000	1010	
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294 (100%)'
	10	526	10	1135	
	15	526	5	1135	
	30	526	15	940	
	1830	526	1800	940	
	2250	549	420	1010	
	2310	549	60	1010	
	2311	542	1	1010	
	2911	542	600	1010	
	2912	526	1	1010	
	8912	526	6000	1010	
6. Reactor Overpressure 1 Cycle	0	526		1010	28294 (100%)'
	2	526	2	1375	
	32	526	30	940	
	1832	526	1800	940	
	2252	549	420	1010	
	2312	549	60	1010	
	2313	542	1	1010	
	2913	542	600	1010	
	2914	526	1	1010	
	8914	526	6000	1010	
7. SRV Blowdown 1 Cycle	0	526		1010	28294 (100%)'
	600	375	600	170	
	11580	70	10980	50	
	17580	70	6000	50	
8. SCRAM Other 228 Cycles	0	526		1010	28294 (100%)'
	15	526	15	940	
	1815	526	1800	940	
	2235	549	420	1010	
	2295	549	60	1010	
	2296	542	1	1010	
	2356	542	60	1010	
	2357	526	1	1010	
	8357	526	6000	1010	
9. Improper Startup 1 Cycle	0	526		1010	3395 (12%)'
	1	268 (3)	1	1010	
	27	268 (3)	26	1010	
	28	526	1	1010	
	6028	526	6000	1010	
10. Shutdown 300 Cycles	0	549		1010	14147 (50%)'
	6264	375	6264	170	
	6864	330	600	88	
	16224	70	9360	50	
	22224	70	6000	50	
11. Design Hydrostatic Test 120 cycles	—	100	—	50	1981 (7%)
				1563	
				50	
12. Hydrostatic Test 1 Cycle	—	100	—	0	1981 (7%)
				1100	
				50	

Notes:
1. The instant temperature change is assumed as 1 second time step.
2. The number of cycles is for 60 years [14].
3. 268°F is the blend radius temperature for this transient. The safe end has a different temperature for Transient 9. [13]

Table 3-3: Stresses Due to Piping Loads for Safe End Location

Safe End External Piping Loads		
Parameters		
F_x =	20.00	kips
F_y =	20.00	kips
F_Z =	30.00	kips
M_x =	2004.00	in-kips
M_y =	3000.00	in-kips
M_Z =	2004.00	in-kips
OD=	28.38	in
ID=	25.938	in
R_N =	13.58	in
L =	4.25	in
t_N =	1.22	in
$(M_x)_2$ =	1919.00	in-kips
$(M_y)_2$ =	3085.00	in-kips
M_{xy} =	3633.15	in-kips
F_{xy} =	28.28	kips
N_Z =	6.62	kips/in
q_N =	-1.07	kips/in
Primary Membrane Stress Intensity		
PM_Z =	5.43	ksi
τ =	-0.88	ksi
SI_{max} =	5.71	ksi
SI_{max} =	5708.89	psi

Table 3-4: Stresses Due to Piping Loads for Blend Radius Location

Blend Radius External Piping Loads		
Parameters		
F_x =	20.00	kips
F_y =	20.00	kips
F_Z =	30.00	kips
M_x =	2004.00	in-kips
M_y =	3000.00	in-kips
M_Z =	2004.00	in-kips
OD=	55.88	in
ID=	37.368	in
R_N =	23.31	in
L =	42.77	in
t_N =	9.25	in
$(M_x)_2$ =	1148.54	in-kips
$(M_y)_2$ =	3855.46	in-kips
M_{xy} =	4022.90	in-kips
F_{xy} =	28.28	kips
N_Z =	2.56	kips/in
q_N =	-0.20	kips/in
Primary Membrane Stress Intensity		
PM_Z =	0.28	ksi
τ =	-0.02	ksi
SI_{max} =	0.28	ksi
SI_{max} =	280.16	psi

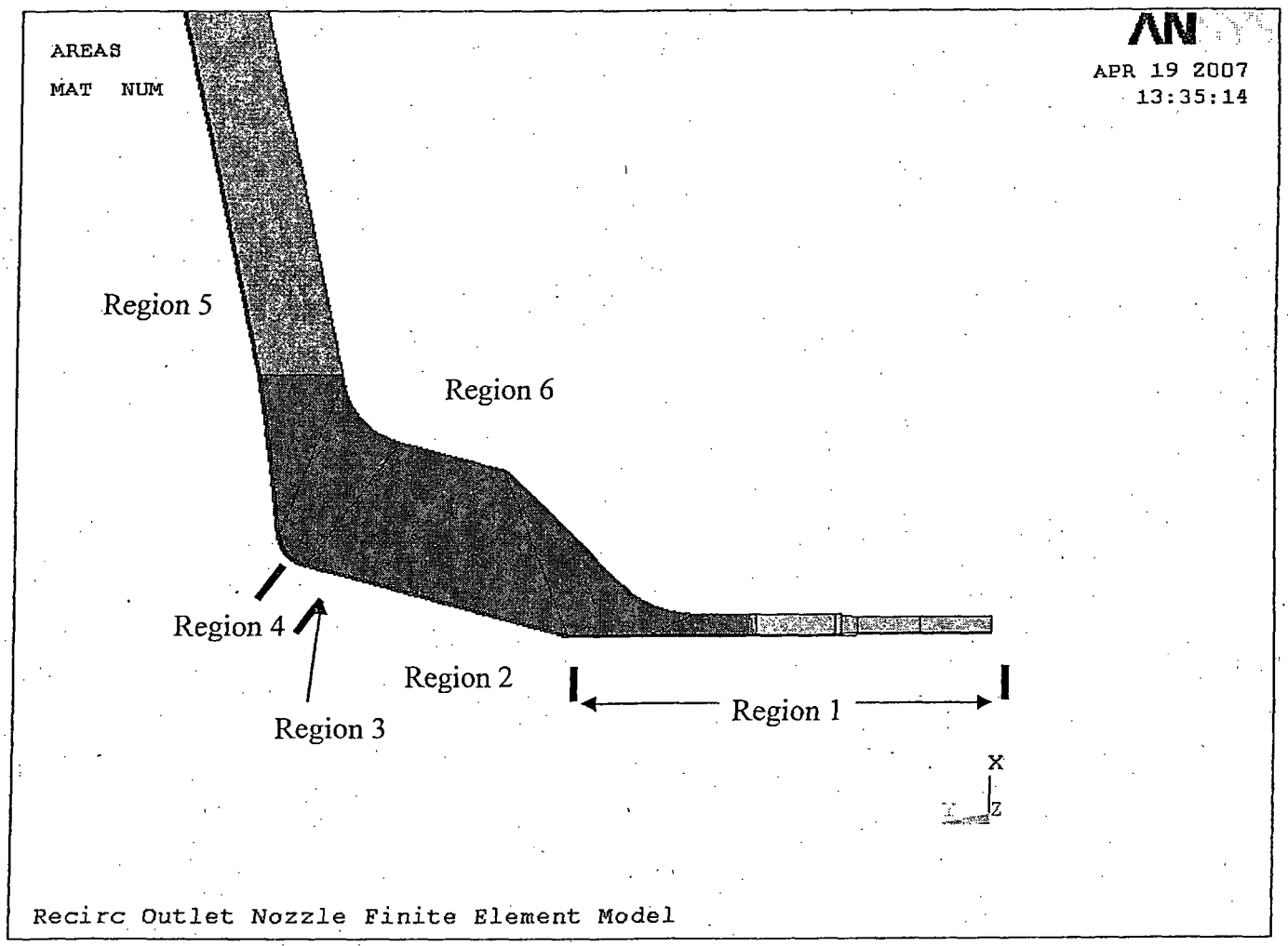


Figure 3-1: Thermal Regions for Green's Functions

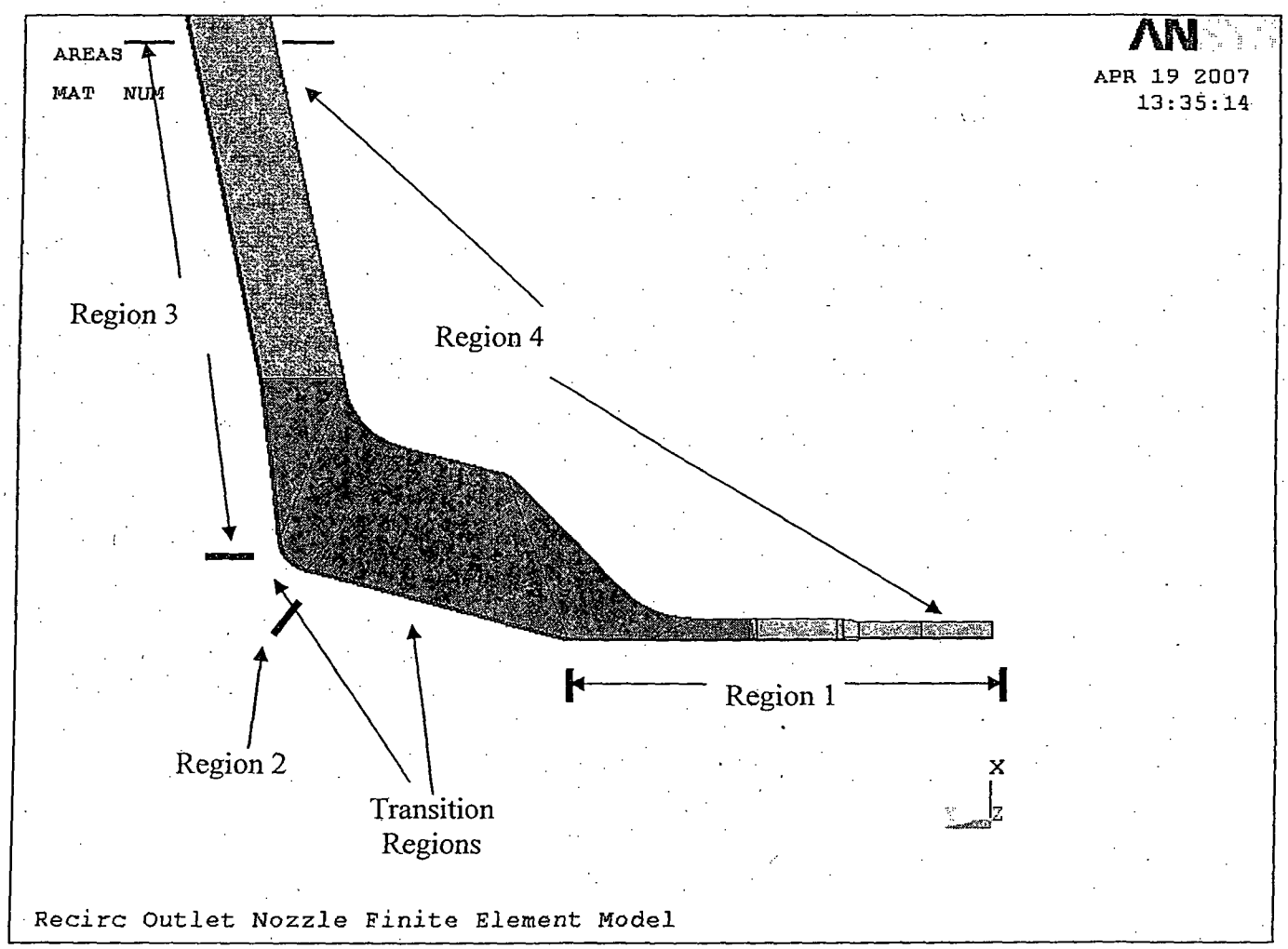


Figure 3-2: Thermal Regions for Transient 9

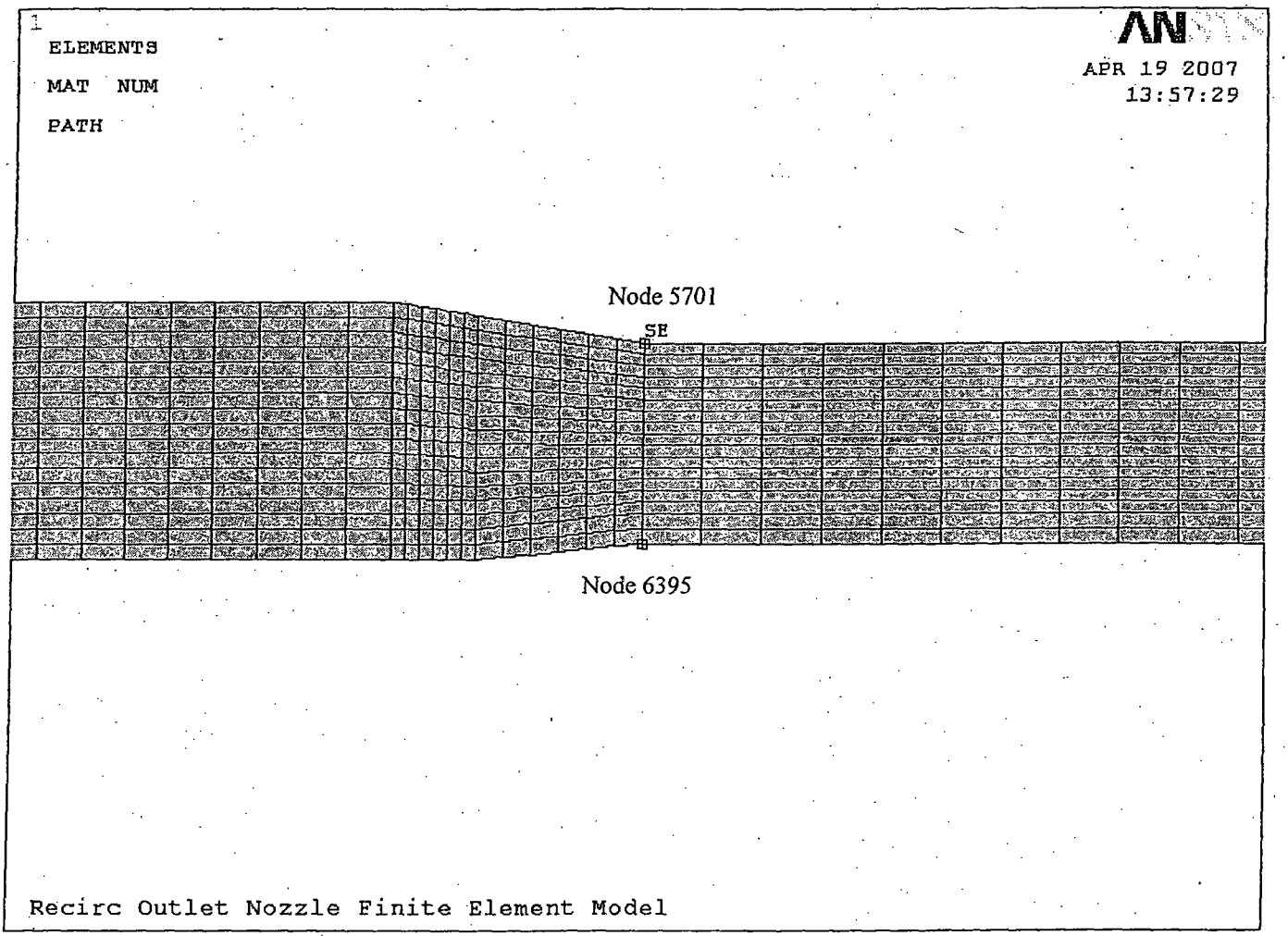


Figure 3-3: Safe End Critical Thermal Stress Location

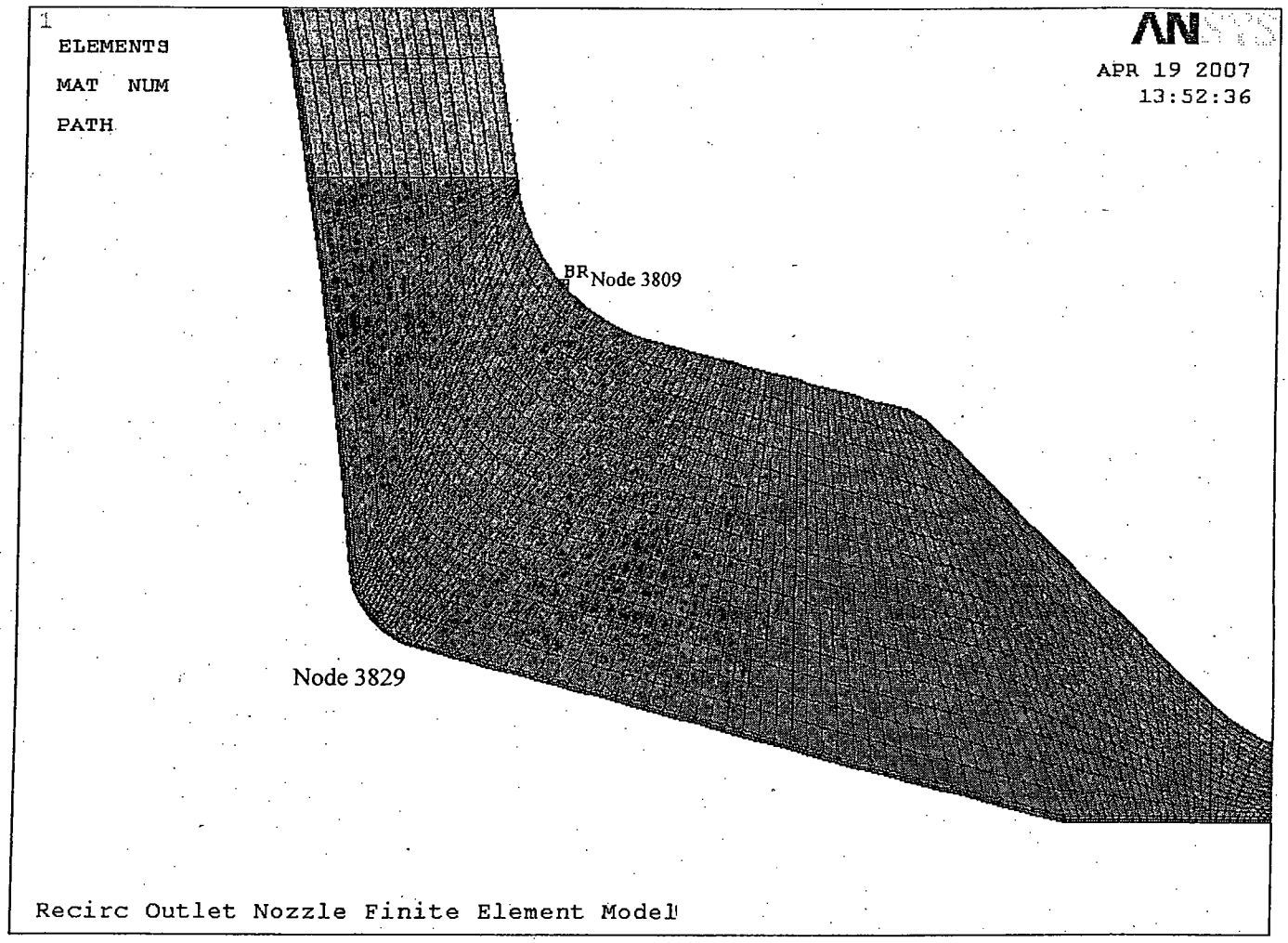


Figure 3-4: Blend Radius Critical Thermal Stress Location

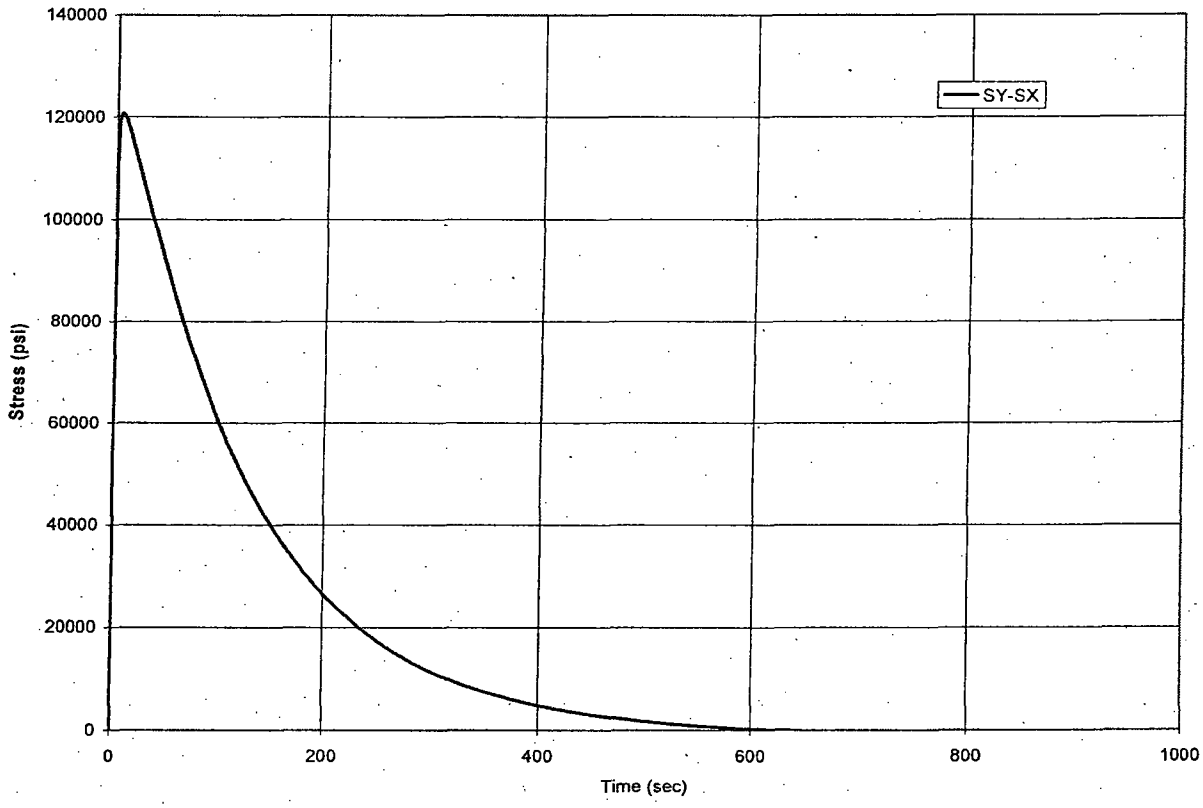


Figure 3-5: Safe End Total Stress Intensity Green's Function for 100% Flow

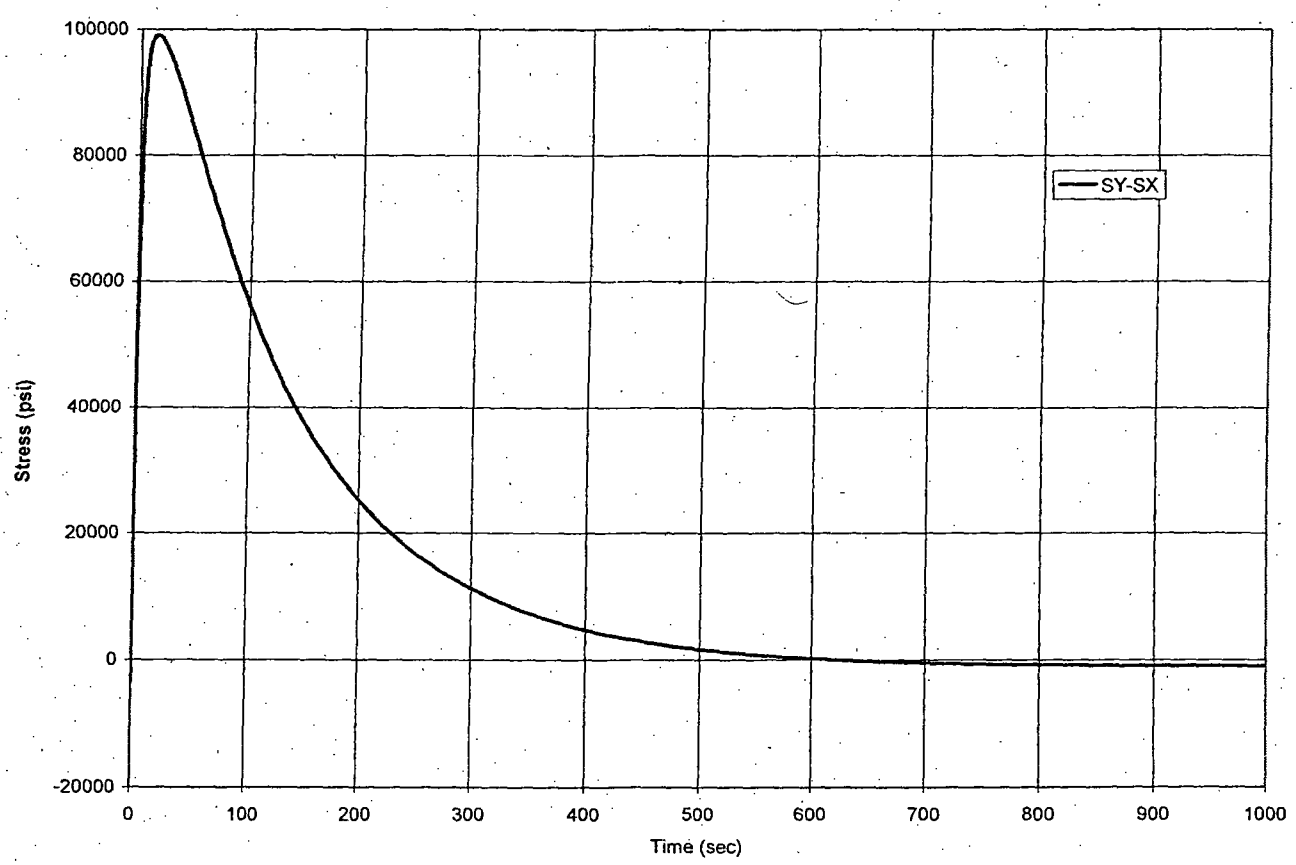


Figure 3-6: Safe End Total Stress Intensity Green's Function for 50% Flow

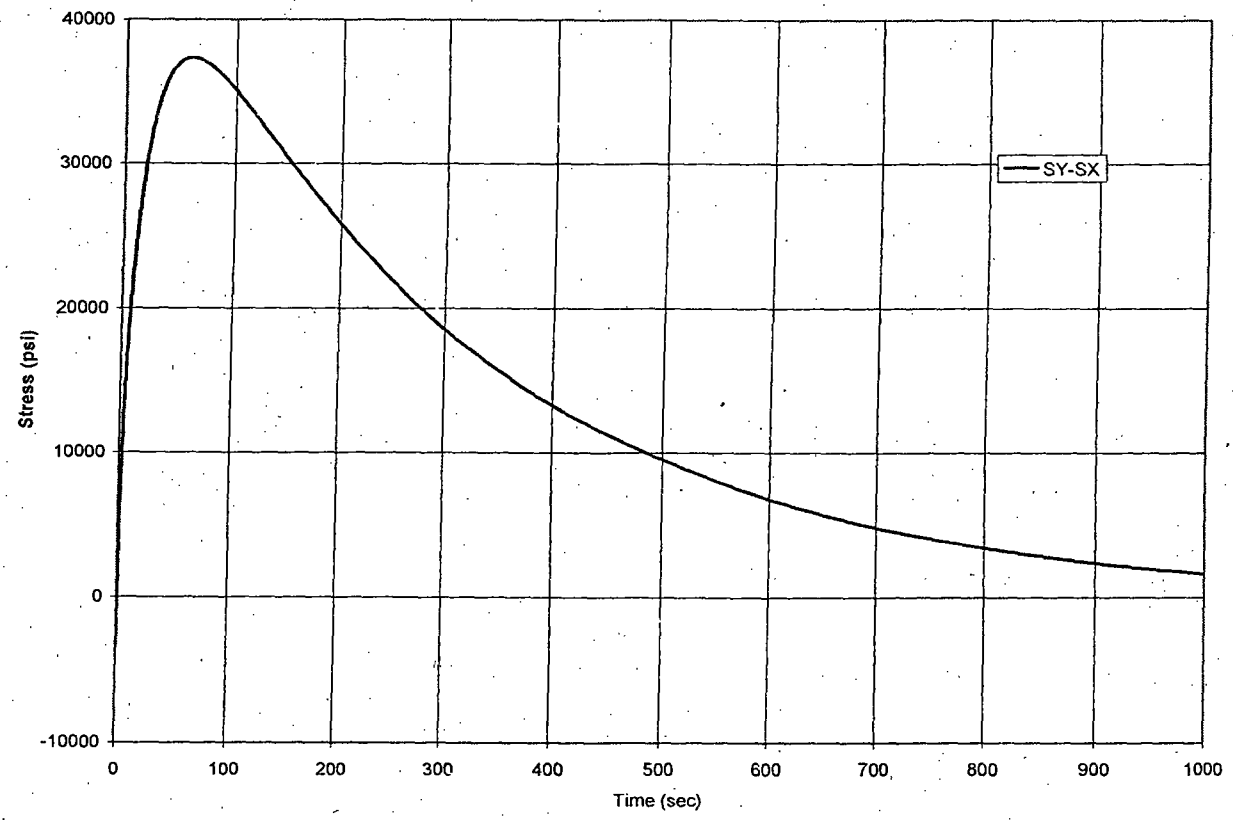


Figure 3-7: Safe End Total Stress Intensity Green's Function for 0% Flow

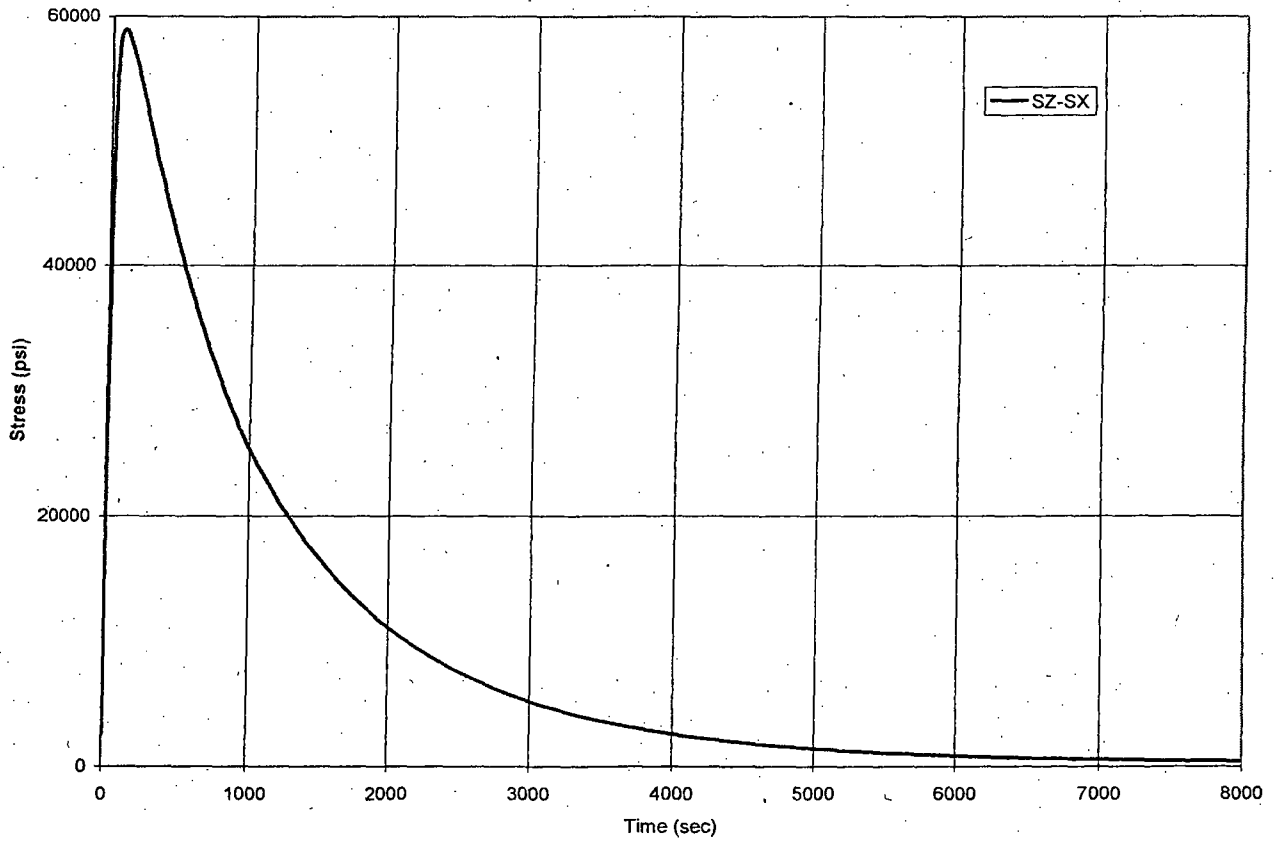


Figure 3-8: Blend Radius Total Stress Intensity Green's Function for 100% Flow

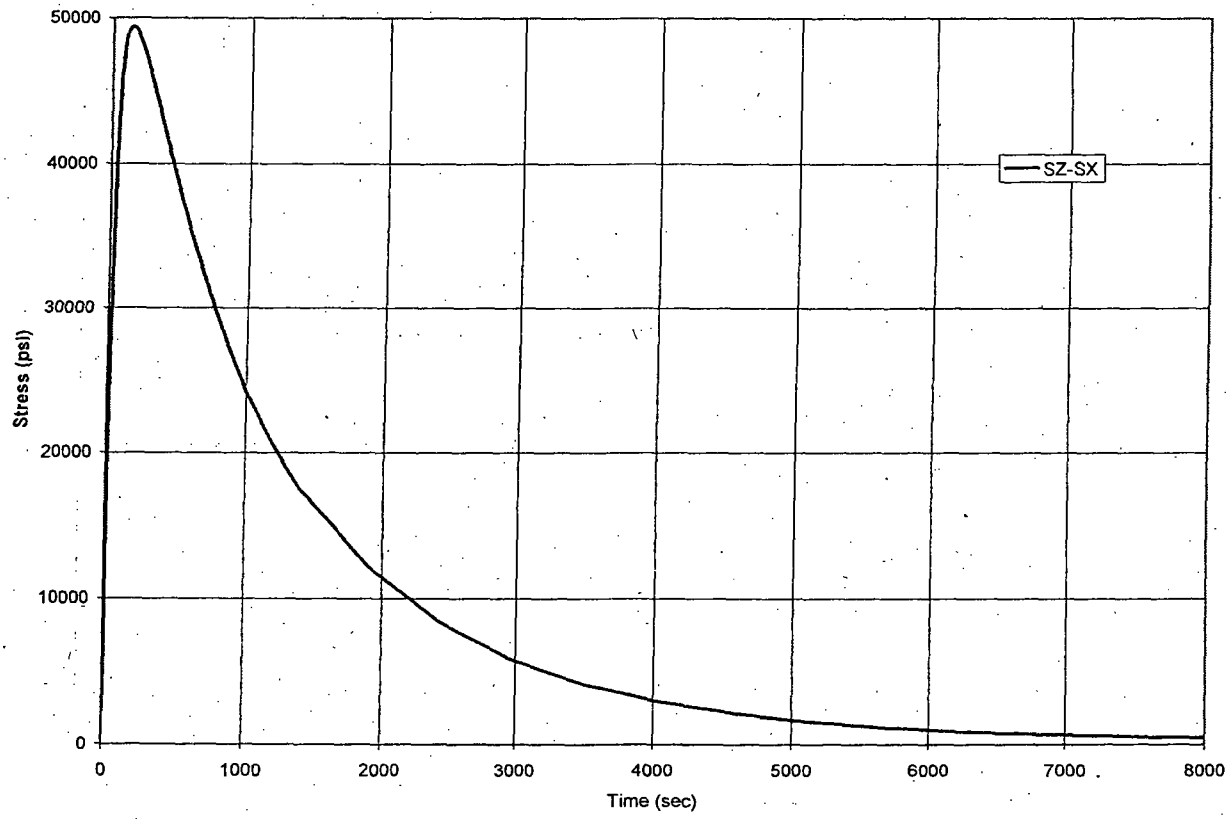


Figure 3-9: Blend Radius Total Stress Intensity Green's Function for 50% Flow

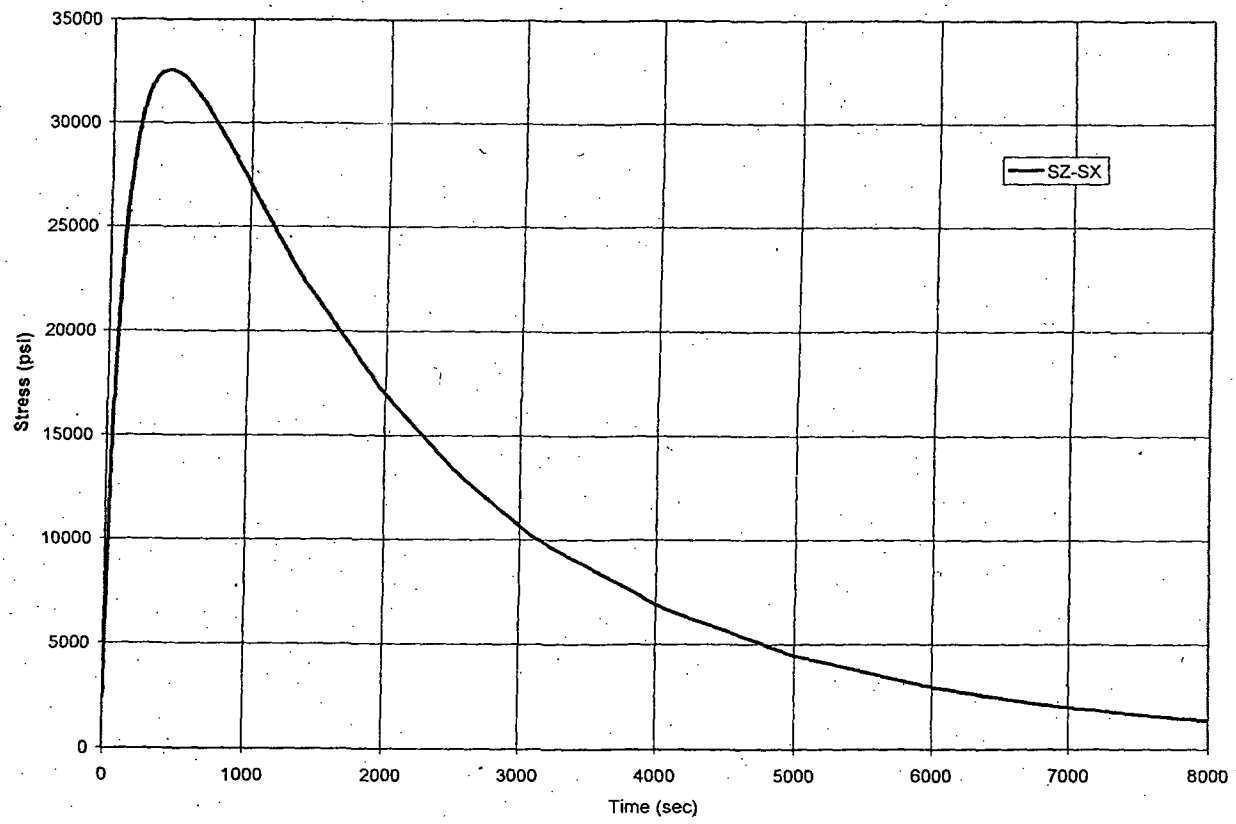


Figure 3-10: Blend Radius Total Stress Intensity Green's Function for 0% Flow

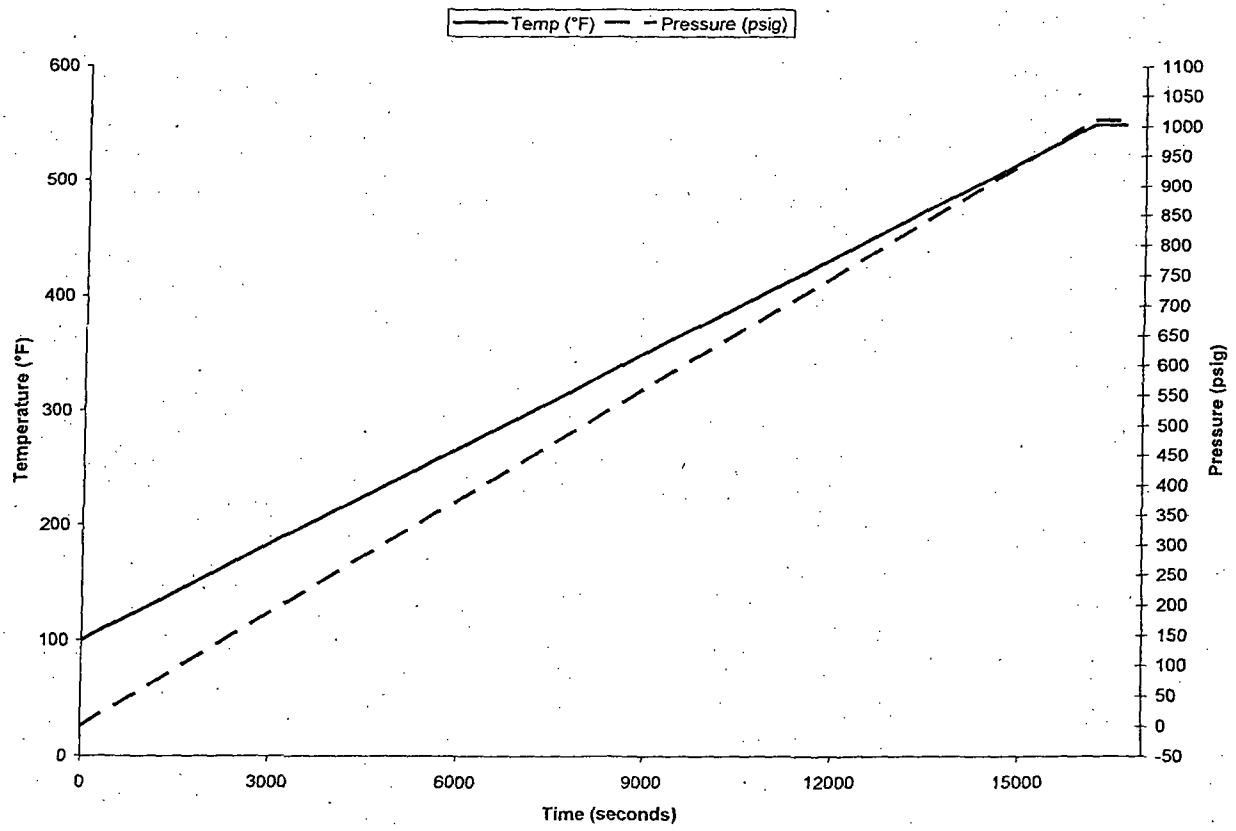


Figure 3-11: Transient 1: Normal Startup at 100°F/hr

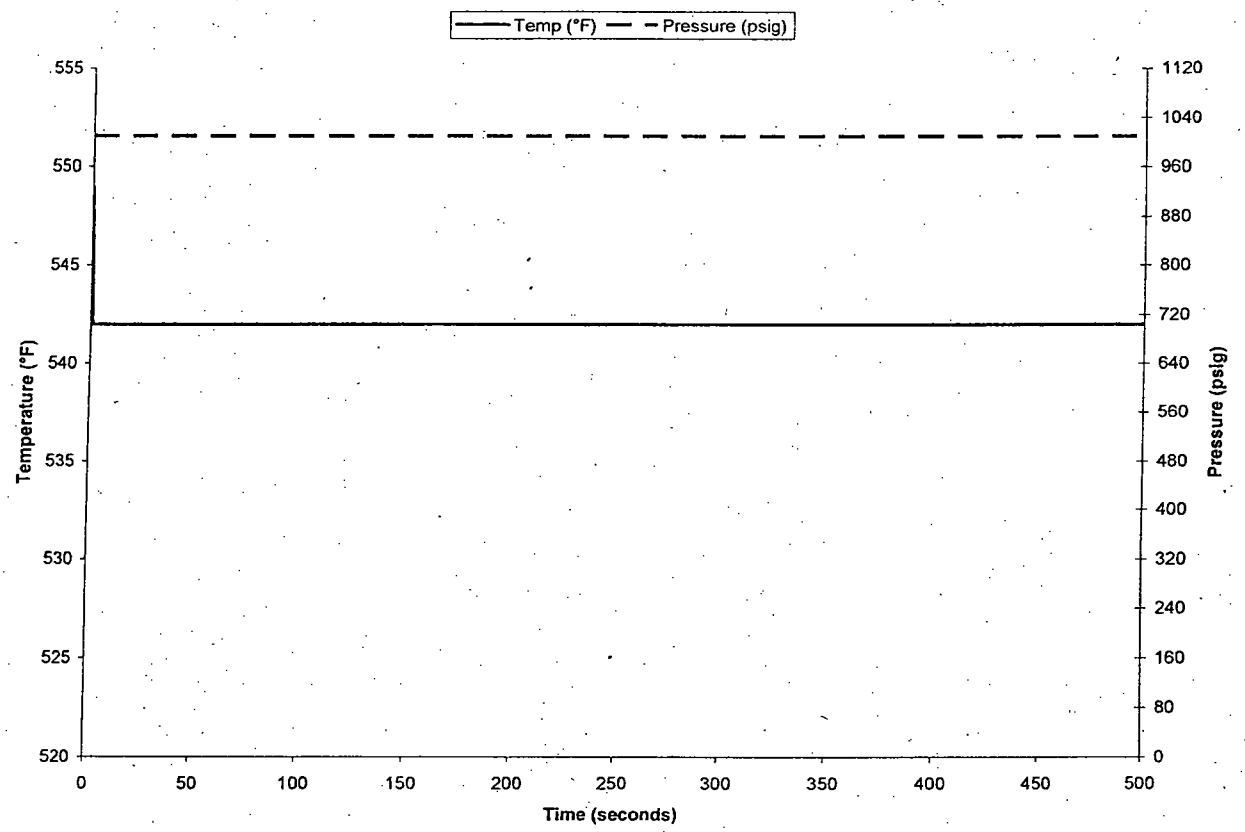


Figure 3-12: Transient 2: Turbine Roll and Increase to Rated Power

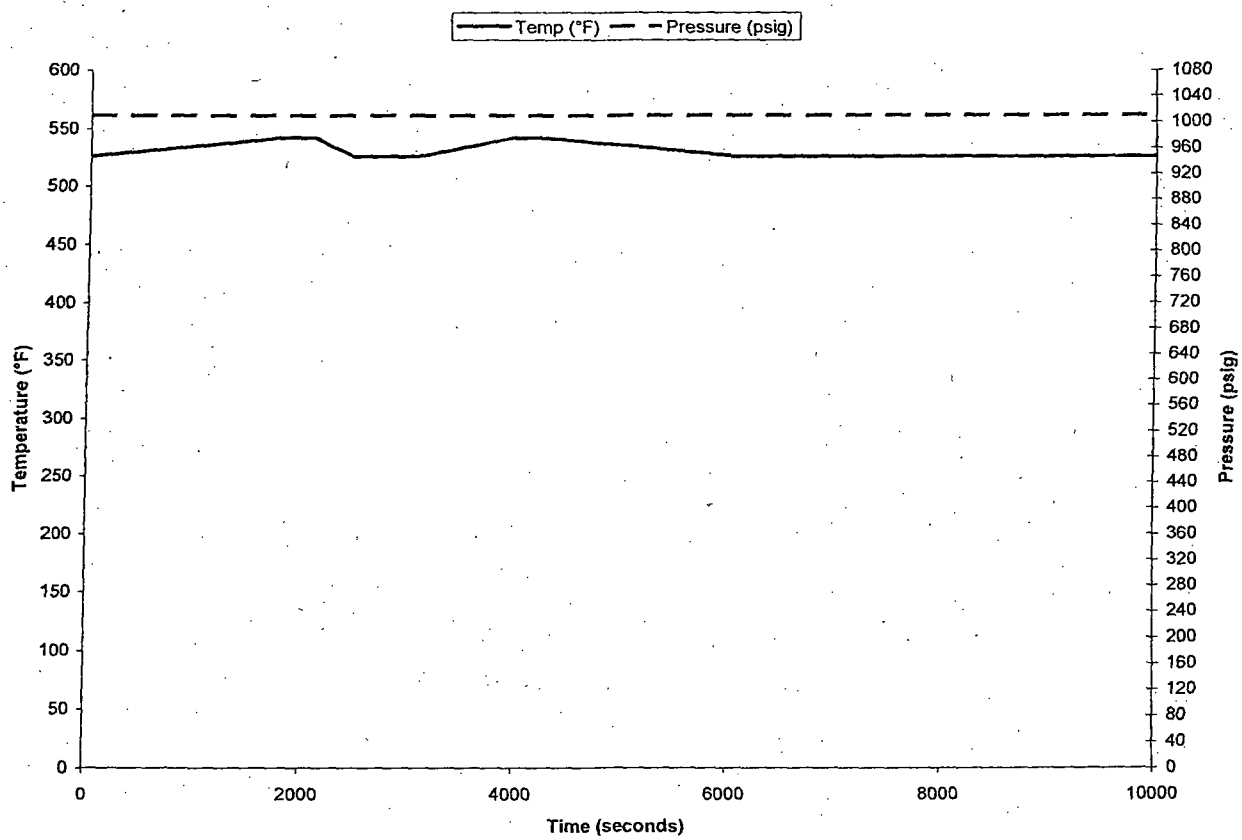


Figure 3-13: Transient 3: Loss of Feedwater Heaters and Turbine Trip at 25% Power

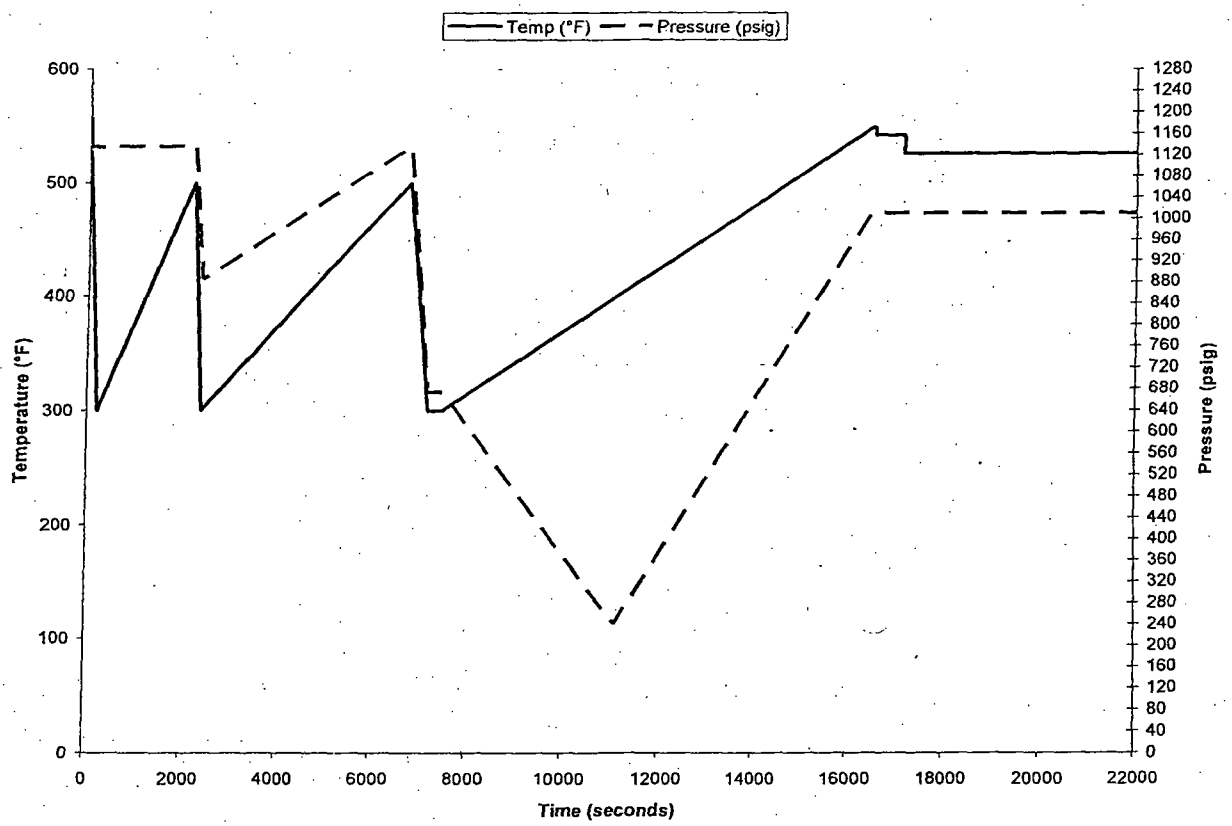


Figure 3-14: Transient 4: Loss of Feedwater Pumps

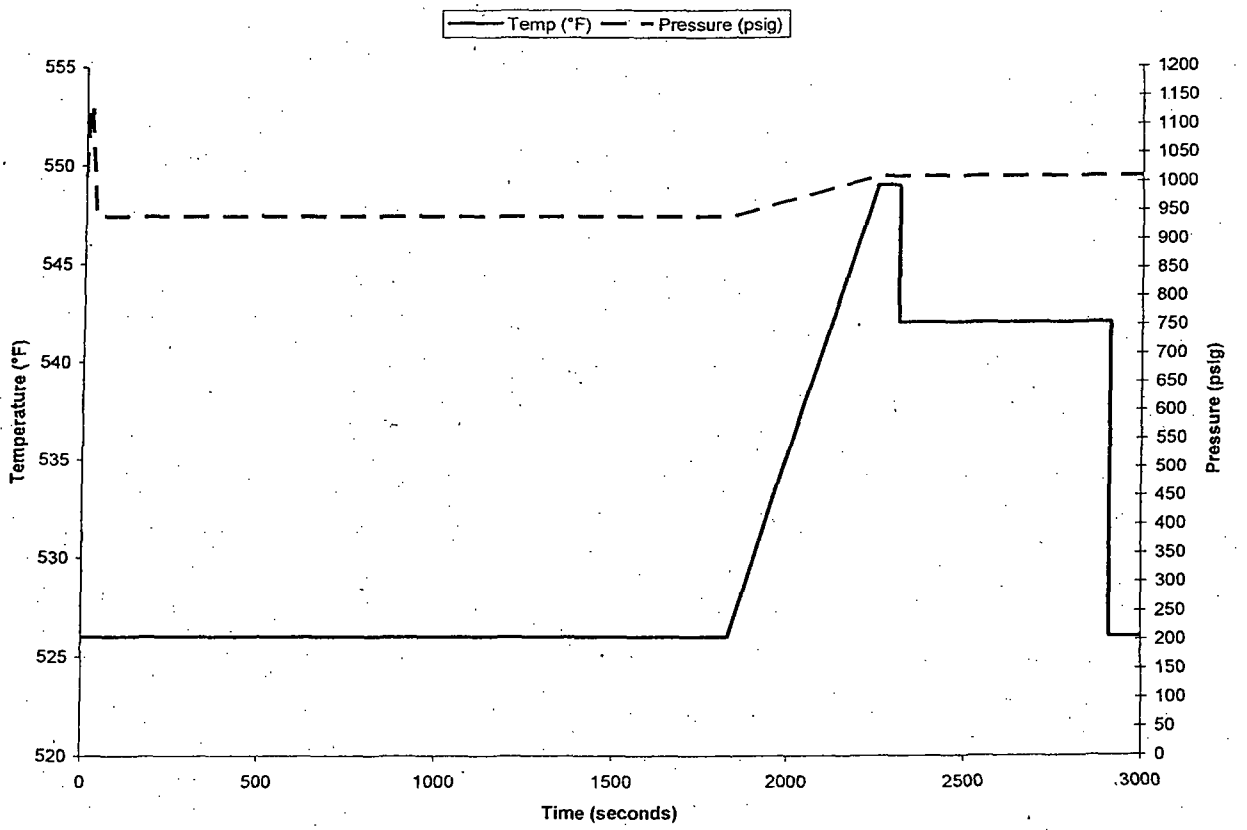


Figure 3-15: Transient 5: Turbine Generator Trip

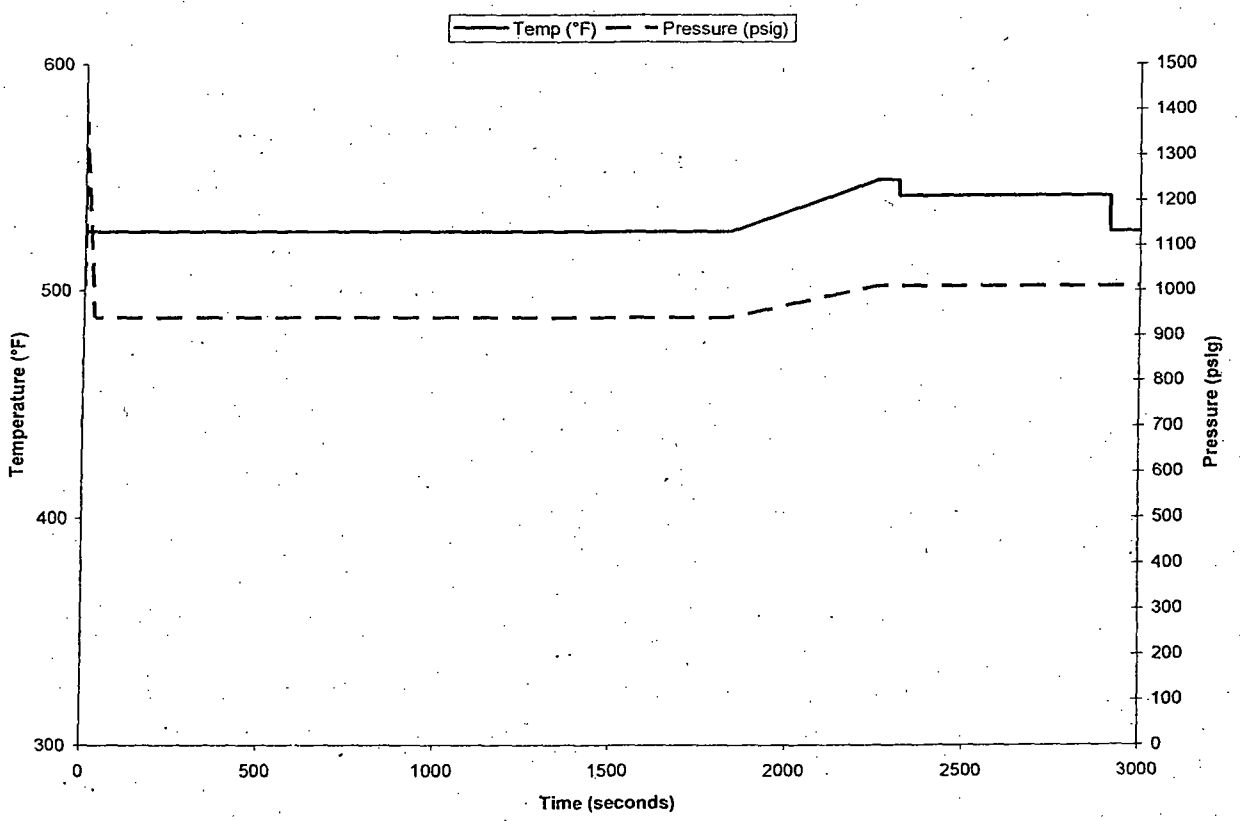


Figure 3-16: Transient 6: Reactor Overpressure

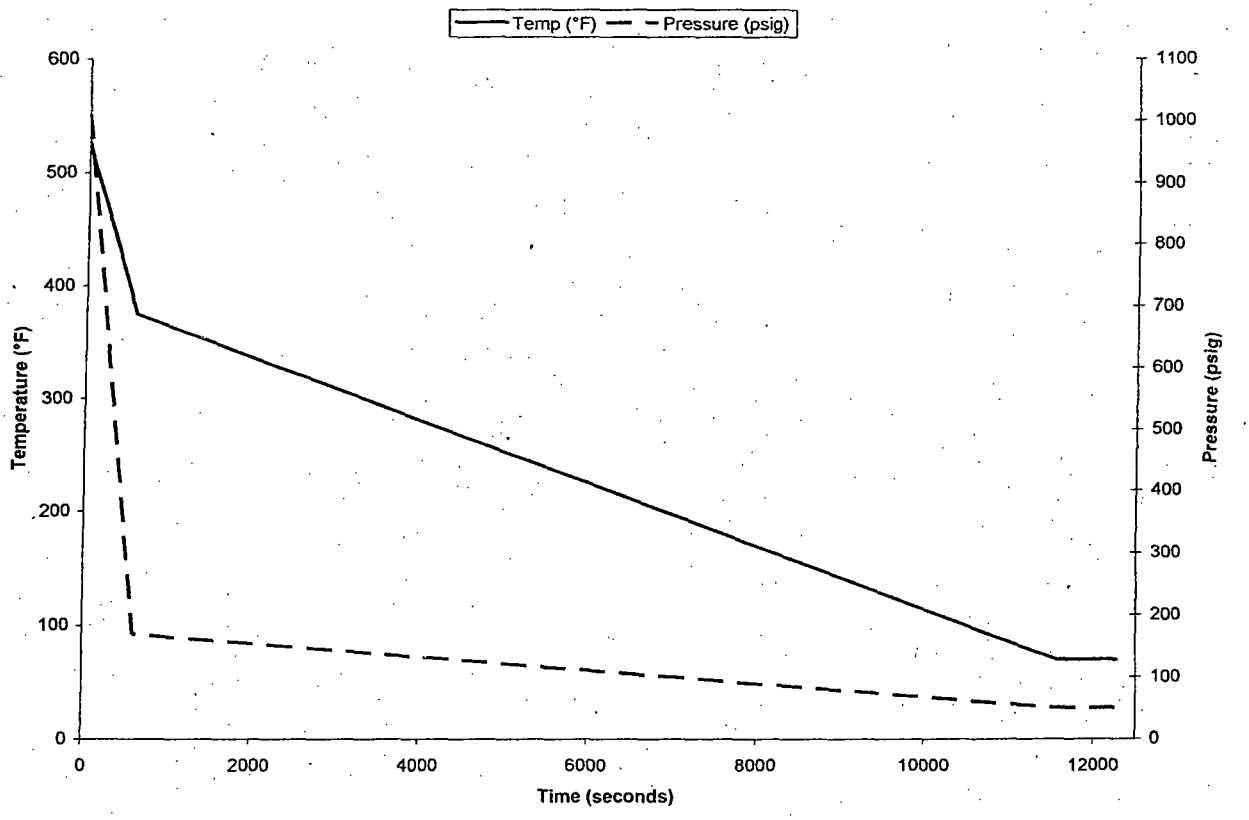


Figure 3-17: Transient 7: SRV Blowdown

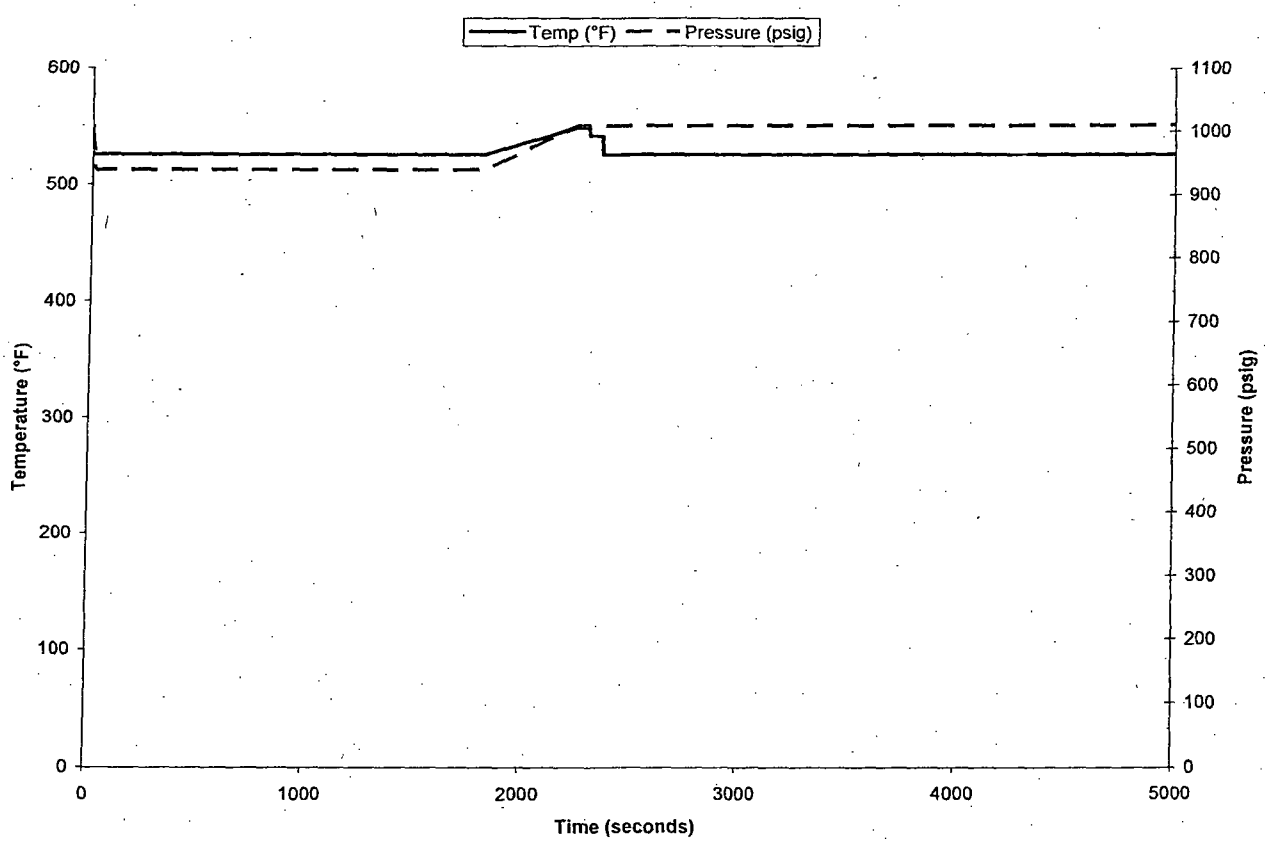


Figure 3-18: Transient 8: Scram - Other

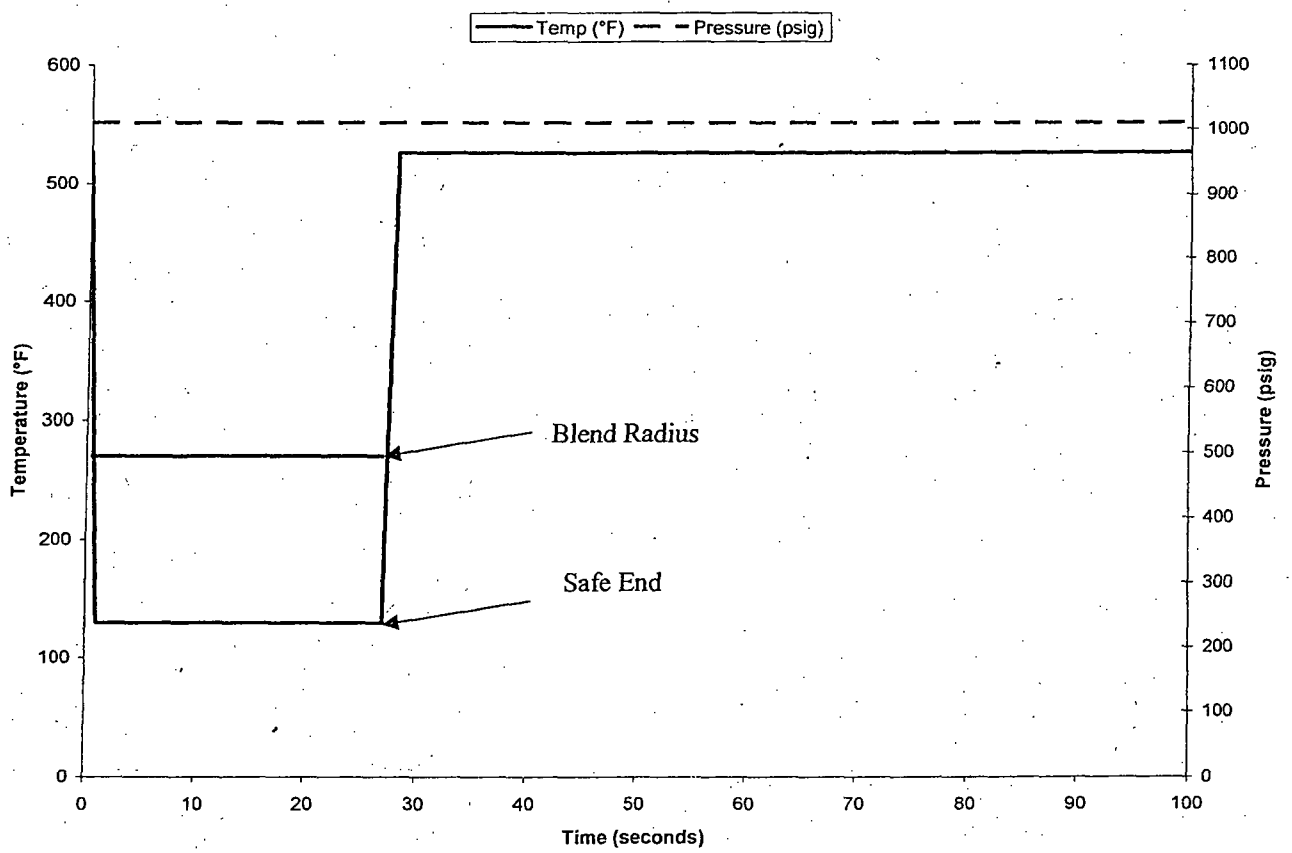


Figure 3-19: Transient 9: Improper Startup

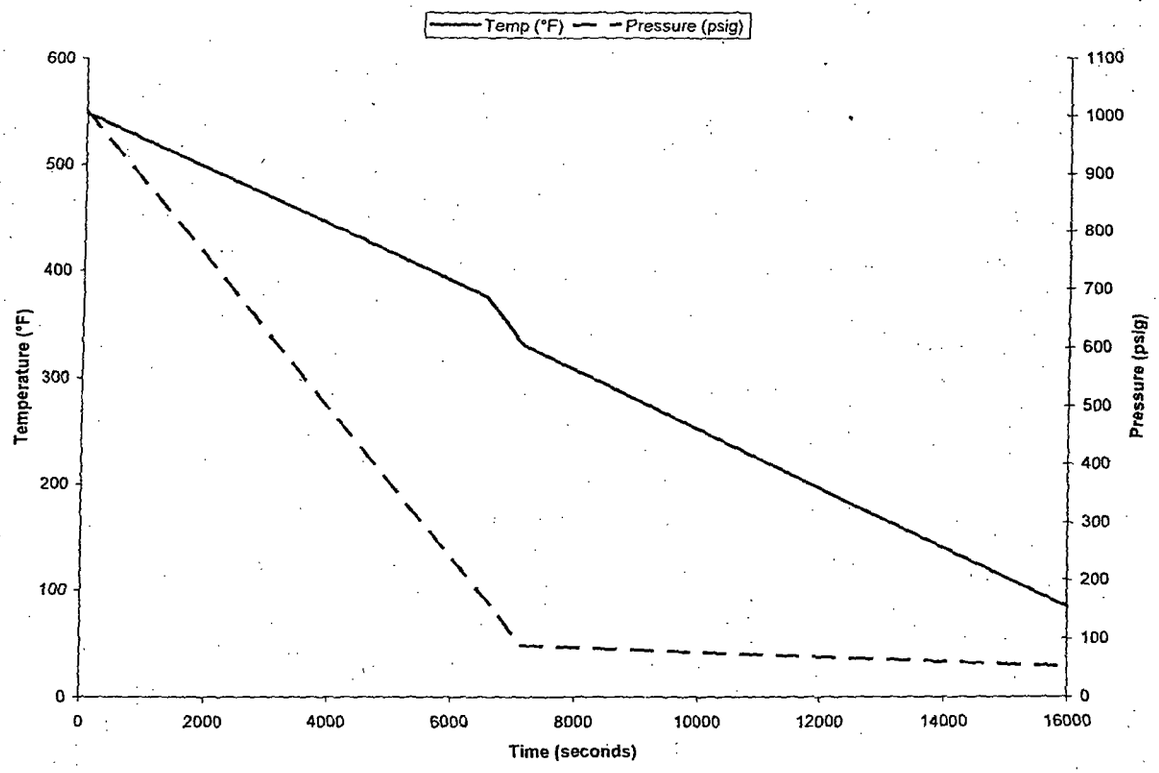


Figure 3-20: Transient 10: Shutdown

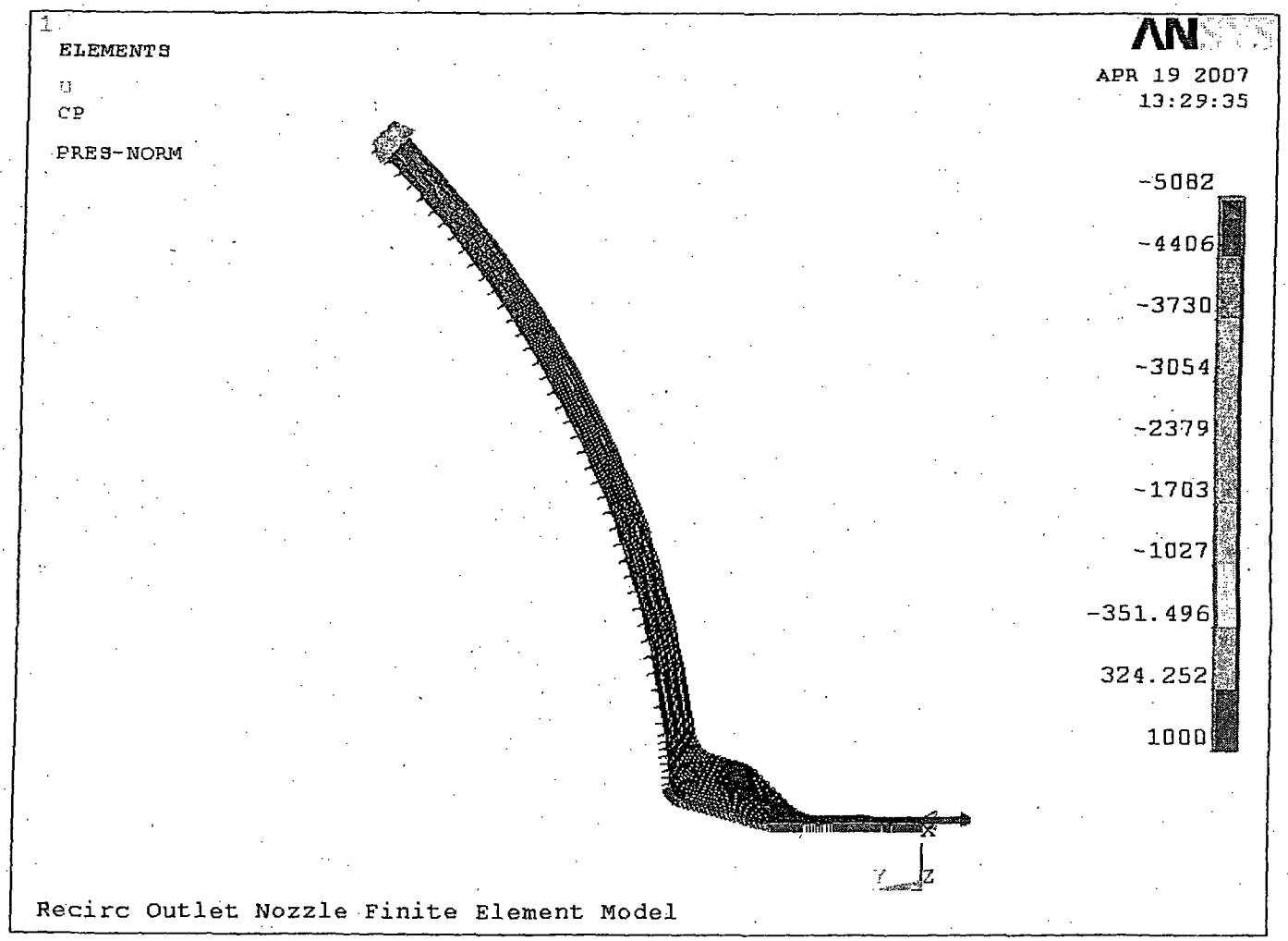


Figure 3-21: Reactor Recirculation Outlet Nozzle Internal Pressure Distribution

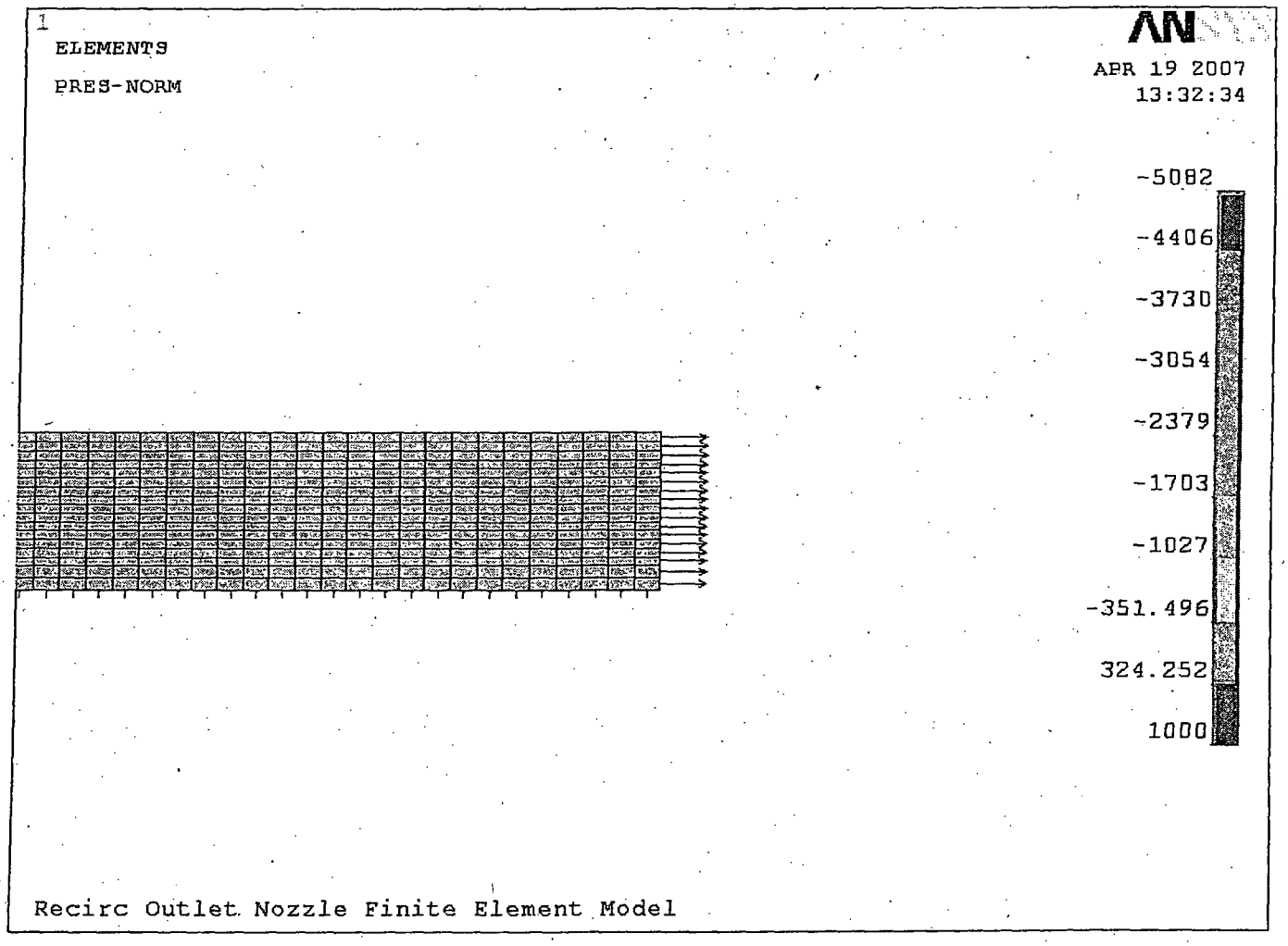


Figure 3-22: Reactor Recirculation Outlet Nozzle Pressure Cap Load

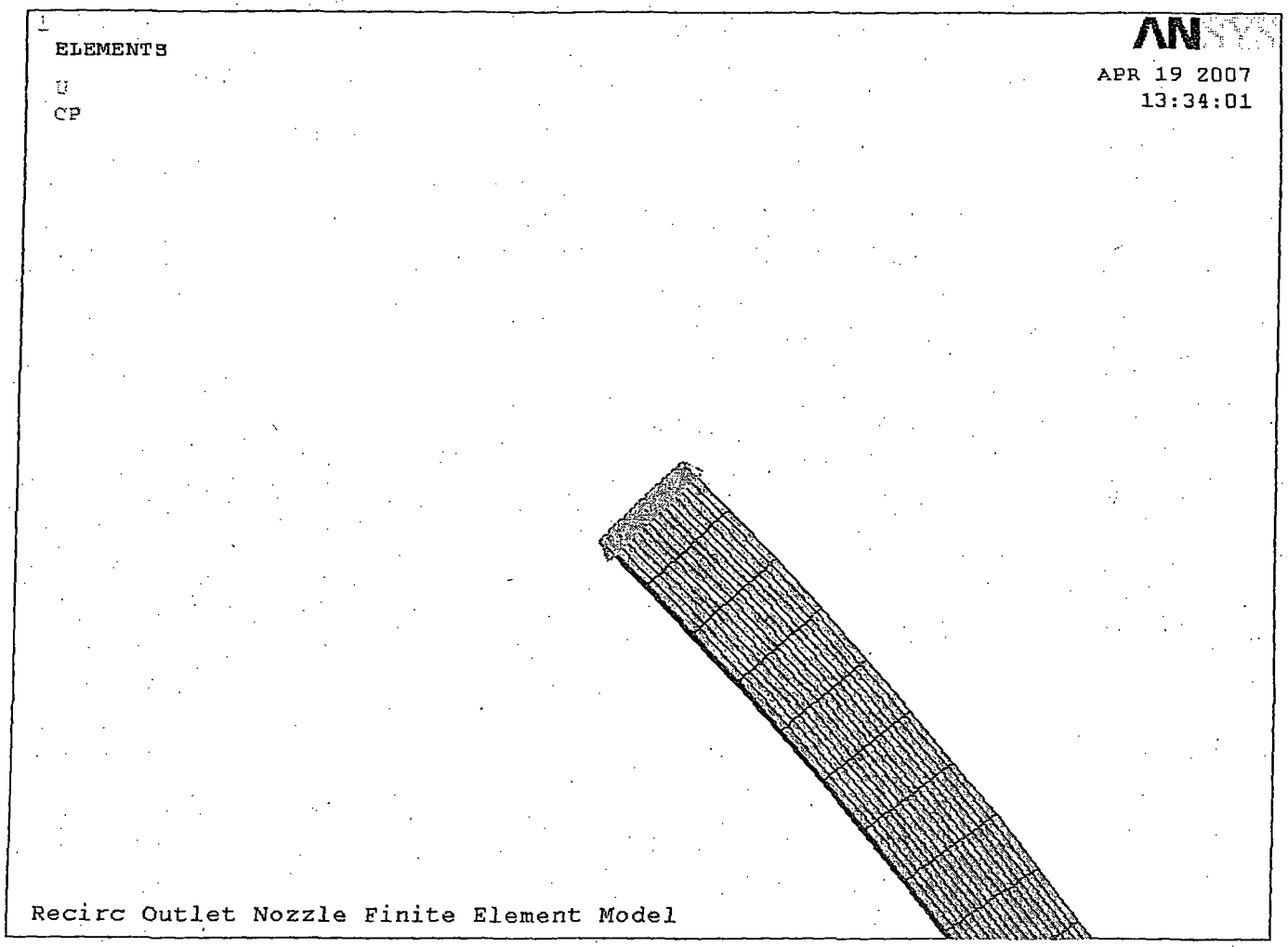


Figure 3-23: Reactor Recirculation Outlet Nozzle Vessel Boundary Condition

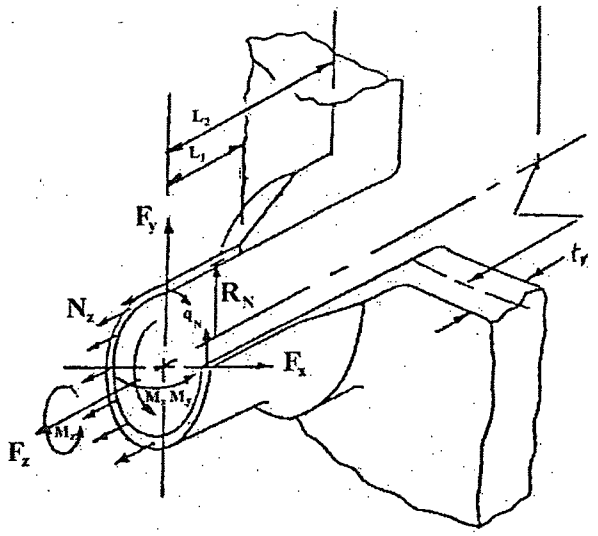


Figure 3-24: Pipe Reactions

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY reactor recirculation outlet nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [17]. Fatigue analysis was performed in the Reference [11] calculation for the two locations identified in Section 3.1.2 using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [14].

Three computer programs were used to facilitate the fatigue analysis process: STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III methodology. All three programs are explained in detail and were independently verified for use in the Reference [16] calculation.

In order to perform the fatigue analysis, input files with the necessary data were prepared and the three analysis programs were run. The program STRESS.EXE required the following three input files:

- **Green.dat**: This file contains the Green's Function. As discussed above, the reactor recirculation outlet nozzle analyses utilize twelve Green's Functions: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function for both the safe end and blend radius locations for each of three flow conditions.
- **Green.cfg**: A configuration file containing parameters that describe the Green's Function.
- **Transnt.inp:** This file contains the input transient history defined in Tables 3-1 and 3-2.

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the safe end) and Table 4-2 (for the blend radius) show the final peak and valley output after stress history reduction. The pressure values for Columns 6 through 8 in each table were determined from the transient pressures specified in Tables 3-1 and 3-2. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in Columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-1 and 3-2.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The results of the fatigue analysis are presented in Tables 4-4 and 4-5 for the safe end and the blend radius for 60 years, respectively.

Table 4-1: Reactor Recirculation Outlet Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	-925	-949	100.00	0	0	0	-339.1419	-339.1419	-1264.14	-1288.14	300
	16164	-4814	-4433	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	1375.93	1615.53	300
	16864	-3749	-3705	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	2440.93	2343.53	300
2	0	-3838	-3665	549	1010	11604.9	11463.5	-5414.966	-5414.966	2351.93	2383.53	300
	6	-1664	-2263	542	1010	11604.9	11463.5	-5335.833	-5335.833	4605.07	3864.67	300
	601	-3773	-3607	542	1010	11604.9	11463.5	-5335.833	-5335.833	2496.07	2520.67	300
	606.6	1196	-403	526	1010	11604.9	11463.5	5154.958	-5154.958	17955.86	5905.54	300
	1302	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	300
3	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	10
	1800.1	-4165	-3904	542	1010	11604.9	11463.5	-5335.833	-5335.833	2104.07	2223.67	10
	2460.2	-1932	-2200	526	1010	11604.9	11463.5	-5154.958	-5154.958	4517.94	4108.54	10
	3960.2	-4537	-4185	542	1010	11604.9	11463.5	-5335.833	-5335.833	1732.07	1942.67	10
	6060.2	-3315	-3241	526	1010	11604.9	11463.5	-5154.958	-5154.958	3134.94	3067.54	10
	6760	-3687	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2762.94	2786.54	10
4	0.00	-3756	-3716	526	1020	11719.8	11577	-5154.958	-5154.958	2808.84	2706.04	10
	3.00	-3756	-3716	526	1190	13673.1	13506.5	-5154.958	-5154.958	4762.14	4635.54	10
	13.00	-3756	-3716	526	1135	13041.15	12882.25	-5154.958	-5154.958	4130.19	4011.29	10
	242.30	15878	10049	302.374	1135	13041.15	12882.25	2626.926	2626.926	31546.08	25558.18	10
	2213.10	-6388	-5428	499.889	1135	13041.15	12882.25	-4859.78	-4859.78	1793.37	2594.47	10
	2408.60	13203	8265	301.443	885	10168.65	10044.75	2616.401	2616.401	25988.05	20926.15	10
	6773.40	-4763	-4312	499.809	1135	13041.15	12882.25	-4858.875	-4858.875	3419.27	3711.37	10
	7193.10	15374	9801	300	675	7755.75	7661.25	2600.088	2600.088	25729.84	20062.34	10
	16457.50	-4812	-5032	549	240	2757.6	2724	-5414.966	-5414.966	-7469.37	-7722.97	10
	16524.70	-2358	-2725	542	1010	11604.9	11463.5	-5335.833	-5335.833	3911.07	3402.67	10
	17118.00	-3778	-3610	541.996	1010	11604.9	11463.5	-5335.788	-5335.788	2491.11	2517.71	10
	17123.60	1192	-406	526	1010	11604.9	11463.5	5154.958	-5154.958	17951.86	5902.54	10
	17819.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	10
5	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	60
	10.00	-3688	-3522	526	1135	13041.15	12882.25	-5154.958	-5154.958	4198.19	4205.29	60
	30.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	60
	2250.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	60
	2319.90	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	60
	2911.00	-3782	-3613	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	60
	2916.70	1188	-408	526	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	60
	3612.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	60
6	0.00	-3688	-3522	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	2.00	-3688	-3522	5.26E+02	1375	15798.75	15606.25	-5154.958	-5154.958	6955.79	6929.29	1
	32.00	-3688	-3522	5.26E+02	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	1
	2252.10	-6054	-5337	5.49E+02	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	1
	2322.20	-2977	-3123	5.42E+02	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	1
	2913.00	-3782	-3613	5.42E+02	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	1
	2918.70	1188	-408	5.26E+02	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	1
	3614.00	-3670	-3509	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	1
7	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	600	7773	5336	375	170	1953.3	1929.5	3447.943	3447.943	13174.24	10713.44	1
	1367.9	-1390	-1567	354.172	162	1861.38	1838.7	-3212.488	-3212.488	-2741.11	-2940.79	1
	11580.1	454	190	70	50	574.5	567.5	0	0	1028.50	757.50	1
	12280	-707	-689	70	50	574.5	567.5	0	0	-132.50	-121.50	1
8	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	228
	15.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	228
	2235.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	228
	2305.20	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	228
	2356.00	-3183	-3151	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	3086.08	2976.68	228
	2361.50	1761	-28	526	1010	11604.9	11463.5	5154.958	-5154.958	18520.86	6280.54	228
	3057.00	-3667	-3506	526	1010	11604.9	11463.5	-5154.958	-5154.958	2782.94	2802.54	228
9	0	-2968	-2837	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3485.22	3474.82	1
	27	68473	45303	291.3	1010	11604.9	11463.5	2501.737	2501.737	82579.74	59268.34	1
	80.7	-11546	-8877	518.4	1010	11604.9	11463.5	-5069.042	-5069.042	-5010.04	-2482.14	1
	5200	-2967	-2832	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3486.21	3479.78	1
10	0	-3745	-3709	549	1010	11604.9	11463.5	-5414.966	-5414.966	2444.93	2339.53	300
	6864.2	501	-405	329.994	170	1953.3	1929.5	2939.162	-2939.162	5393.46	-1414.66	300
	7455.5	-1183	-1528	314.325	88	1011.12	998.8	-2762.029	-2762.029	-2933.91	-3291.23	300
	16224.1	334	-35	70	50	574.5	567.5	0	0	908.50	532.50	300
	16924	-731	-763	70	50	574.5	567.5	0	0	-156.50	-195.50	300
11		0	0	100	0	0	0	339.1419	339.1419	339.14	339.14	120
		0	0	100	1100	12639	12485	339.1419	339.1419	12978.14	12824.14	120
		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	120
12		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1
		0	0	100	1563	17958.87	17740.05	339.1419	339.1419	18298.01	18079.19	1
		0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1

For notes, see next page.

Table 4-1: Reactor Recirculation Outlet Nozzle Safe End Stress Summary (concluded)

NOTES:

Column 1:	Transient number identification.
Column 2:	Time during transient where maximum or minimum stress intensity occurs from P-V.OUT output file.
Column 3:	Maximum or minimum total stress intensity from P-V.OUT output file.
Column 4:	Maximum or minimum membrane plus bending stress intensity from P-V.OUT output file.
Column 5:	Temperature per total stress intensity.
Column 6:	Pressure per Table 3-1.
Column 7:	Total pressure stress intensity from the quantity (Column 6 x 11490)/1000.
Column 8:	Membrane plus bending pressure stress intensity from the quantity (Column 6 x 11350)/1000.
Column 9:	Total external stress, 5707.89 psi*(Column 5-70°F)/(549°F -70°F).
Column 10:	Same as Column 9, but for M+B stress.
Column 11:	Sum of total stresses (Columns 3, 7, and 9).
Column 12:	Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13:	Number of cycles for the transient (60 years).

Table 4-2: Reactor Recirculation Outlet Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	459	388	100.00	0	0	0	16.64312	16.64312	475.64	404.64	300
	4303	-3417	-1594	219.53	1010	31613	33976.4	-82.95209	-82.95209	28113.05	32299.45	300
	22164	2713	2306	549.00	1010	31613	33976.4	265.7352	265.7352	34591.74	36548.14	300
2	0.00	3094	1934	549	1010	31613	33976.4	265.7352	265.7352	34972.74	36176.14	300
	94.30	4079	2481	542	1010	31613	33976.4	261.8518	261.8518	35953.85	36719.25	300
	601.70	3683	2435	538.8	1010	31613	33976.4	260.0765	260.0765	35556.08	36671.48	300
	680.10	5891	3489	526	1010	31613	33976.4	252.9754	252.9754	37756.98	37718.38	300
	6602.00	2977	1859	526	1010	31613	33976.4	252.9754	252.9754	34842.98	36088.38	300
3	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	10
	1807.20	1834	1043	542	1010	31613	33976.4	261.8518	261.8518	33708.85	35281.25	10
	2491.50	4425	2667	526	1010	31613	33976.4	252.9754	252.9754	36290.98	36896.38	10
	3974.40	1706	1060	542	1010	31613	33976.4	261.8518	261.8518	33580.85	35298.25	10
	6070.80	3971	2551	526	1010	31613	33976.4	252.9754	252.9754	35836.98	36780.38	10
	12060.00	2965	1852	526	1010	31613	33976.4	252.9754	252.9754	34830.98	36081.38	10
4	0	2465	-703	526.00	1010	31613	33976.4	252.9754	-252.9754	34330.98	33020.42	10
	3	2465	-703	526.00	1190	37247	40031.6	252.9754	-252.9754	39964.98	39075.62	10
	13	2465	-703	526.00	1135	35525.5	38181.4	252.9754	-252.9754	38243.48	37225.42	10
	435.6	18138	9690	356.38	1135	35525.5	38181.4	158.8774	158.8774	53822.38	48030.28	10
	2222.5	-1169	-2598	489.44	1135	35525.5	38181.4	-232.6952	-232.6952	34123.80	35350.70	10
	2665.5	12763	6695	328.40	885	27700.5	29771.4	143.3539	143.3539	40606.85	36609.75	10
	6779.2	-4008	-2829	497.05	1010	31613	33976.4	-236.9137	-236.9137	27368.09	30910.49	10
	7243.8	19275	9965	302.91	1010	31613	33976.4	129.2122	129.2122	51017.21	44070.61	10
	13996	-2135	34	542.00	1010	31613	33976.4	-261.8518	261.8518	29216.15	34272.25	10
	17247	3413	2074	526.00	1010	31613	33976.4	252.9754	252.9754	35278.98	36303.38	10
	23119	2971	1855	526.00	1010	31613	33976.4	252.9754	252.9754	34836.98	36084.38	10
5	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	60
	10.00	2959	1849	526	1135	35525.5	38181.4	252.9754	252.9754	38737.48	40283.38	60
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	60
	2269.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	60
	3010.10	4407	2579	526	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	60
	8912.00	2968	1854	526	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	60
6	0.00	2959	1849	526.00	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	2.00	2959	1849	526.00	1375	43037.5	46255	252.9754	252.9754	46249.48	48356.98	1
	32.00	2959	1849	526.00	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	1
	2271.50	111	295	549.00	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	1
	3022.00	4407	2579	526.00	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	1
	8914.00	2968	1854	526.00	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	1
7	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	615.10	20260	12980	374.581	170	5321	5718.8	168.9726	168.9726	25749.97	18867.77	1
	17580.00	279	179	70	50	1565	1682	0	0	1844.00	1861.00	1
8	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	228
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	228
	2254.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	228
	2491.20	3792	2234	526	1010	31613	33976.4	252.9754	252.9754	35657.98	36463.38	228
	8357.00	2963	1851	526	1010	31613	33976.4	252.9754	252.9754	34828.98	36080.38	228
9	0	2058	961	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35190.26	1
	0.52	1956	734	525.6	1010	31613	33976.4	252.7535	252.7535	33821.75	34963.15	1
	28	23747	3188	504.5	1010	31613	33976.4	241.0479	241.0479	55601.05	37405.45	1
	425	1520	611	525.5	1010	31613	33976.4	252.698	252.698	33385.70	34840.10	1
	12400	2058	879	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35108.26	1
10	0	2767	2176	549	1010	31613	33976.4	265.7352	265.7352	34645.74	36418.14	300
	4240.8	6643	4158	445.775	441	13803.3	14835.24	208.469	208.469	20654.77	19201.71	300
	6268	6498	3675	374.7	170	5321	5718.8	169.0386	169.0386	11988.04	9562.84	300
	6891.8	9282	5241	329.228	88	2754.4	2960.32	143.8121	143.8121	12180.21	8345.13	300
	22224	361	120	70	50	1565	1682	0	0	1926.00	1802.00	300
11		0	0	100	0	0	0	16.64312	16.64312	16.64	16.64	120
		0	0	100	1100	34430	37004	16.64312	16.64312	34446.64	37020.64	120
		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	120
12		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1
		0	0	100	1563	48921.9	52579.32	16.64312	16.64312	48938.54	52595.96	1
		0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1

For notes, see next page.

Table 4-2: Reactor Recirculation Outlet Nozzle Blend Radius Stress Summary (concluded)

NOTES:

Column 1:	Transient number identification.
Column 2:	Time during transient where maximum or minimum stress intensity occurs from P-V.OUT output file.
Column 3:	Maximum or minimum total stress intensity from P-V.OUT output file.
Column 4:	Maximum or minimum membrane plus bending stress intensity from P-V.OUT output file.
Column 5:	Temperature per total stress intensity.
Column 6:	Pressure per Table 3-2.
Column 7:	Total pressure stress intensity from the quantity (Column 6 x 31300)/1000.
Column 8:	Membrane plus bending pressure stress intensity from the quantity (Column 6 x 33640)/1000.
Column 9:	Total external stress, 280.16 psi*(Column 5-70°F)/(549°F -70°F).
Column 10:	Same as Column 9, but for M+B stress.
Column 11:	Sum of total stresses (Columns 3, 7, and 9).
Column 12:	Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13:	Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Recirculation Outlet Nozzle Fatigue Analysis

Parameter	Blend Radius	Safe End
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel) [17]	1.7 & 0.3 (stainless steel) [17]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17000 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [17]	28.3×10^6 psi [17]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [10]	27.0×10^6 psi [10]
The Geometric Stress Concentration Factor K_t	1.0	1.53 [20]

Table 4-4: Fatigue Results for Reactor Recirculation Outlet Nozzle Safe End

```
      LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
             m = 1.7
             n =  .3
            Sm = 17000. psi
        Ecurve = 2.830E+07 psi
     Eanalysis = 2.700E+07 psi
            Kt = 1.53
```

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
82580.	-7469.	90049.	66991.	2.045	134573.	1.000E+00	6.765E+02	.0015
31546.	-7469.	39015.	33281.	1.000	29691.	9.000E+00	6.857E+05	.0000
31546.	-5010.	36556.	28040.	1.000	26947.	1.000E+00	1.160E+06	.0000
25988.	-2934.	28922.	24217.	1.000	21884.	1.000E+01	2.383E+06	.0000
25730.	-2934.	28664.	23354.	1.000	21509.	1.000E+01	2.566E+06	.0000
18521.	-2934.	21455.	9572.	1.000	13903.	2.280E+02	9.710E+08	.0000
18298.	-2934.	21232.	21370.	1.000	17063.	1.000E+00	7.876E+06	.0000
17956.	-2934.	20890.	9197.	1.000	13502.	5.100E+01	1.000E+20	.0000
17956.	-2741.	20697.	8846.	1.000	13304.	1.000E+00	1.000E+20	.0000
17956.	-1264.	19220.	7194.	1.000	12071.	2.480E+02	1.000E+20	.0000
17952.	-1264.	19216.	7191.	1.000	12068.	1.000E+01	1.000E+20	.0000
17948.	-1264.	19212.	7189.	1.000	12065.	4.200E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.800E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.000E+00	1.000E+20	.0000
13174.	-157.	13331.	10909.	1.000	10016.	1.000E+00	1.000E+20	.0000
12978.	-157.	13135.	13020.	1.000	10500.	1.200E+02	1.000E+20	.0000
6956.	-157.	7112.	7125.	1.000	5706.	1.000E+00	1.000E+20	.0000
5393.	-157.	5550.	-1219.	1.000	2570.	1.590E+02	1.000E+20	.0000
5393.	-133.	5526.	-1293.	1.000	2537.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	6.000E+01	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	7.900E+01	1.000E+20	.0000
4762.	136.	4626.	3924.	1.000	3514.	1.000E+01	1.000E+20	.0000
4605.	136.	4469.	3153.	1.000	3218.	1.390E+02	1.000E+20	.0000
4605.	339.	4266.	3526.	1.000	3215.	1.200E+02	1.000E+20	.0000
4605.	909.	3697.	3332.	1.000	2863.	4.100E+01	1.000E+20	.0000
4518.	909.	3609.	3576.	1.000	2885.	1.000E+01	1.000E+20	.0000
4198.	909.	3290.	3673.	1.000	2744.	6.000E+01	1.000E+20	.0000
4130.	909.	3222.	3479.	1.000	2655.	1.000E+01	1.000E+20	.0000
3911.	909.	3003.	2870.	1.000	2371.	1.000E+01	1.000E+20	.0000
3486.	909.	2578.	2947.	1.000	2170.	1.000E+00	1.000E+20	.0000
3485.	909.	2577.	2942.	1.000	2168.	1.000E+00	1.000E+20	.0000
3419.	909.	2511.	3179.	1.000	2199.	1.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	6.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	1.000E+00	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	9.600E+01	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.200E+02	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000

Table 4-4: Fatigue Results for Reactor Recirculation Outlet Nozzle Safe End (concluded)

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
3292.	1029.	2264.	2247.	1.000	1810.	1.000E+00	1.000E+20	.0000
3292.	1376.	1916.	1389.	1.000	1390.	9.000E+00	1.000E+20	.0000
3135.	1376.	1759.	1452.	1.000	1325.	1.000E+01	1.000E+20	.0000
3086.	1376.	1710.	1361.	1.000	1274.	2.280E+02	1.000E+20	.0000
2809.	1376.	1433.	1091.	1.000	1054.	1.000E+01	1.000E+20	.0000
2783.	1376.	1407.	1187.	1.000	1067.	4.300E+01	1.000E+20	.0000
2783.	1732.	1051.	860.	1.000	790.	1.000E+01	1.000E+20	.0000
2783.	1793.	990.	208.	1.000	576.	1.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	6.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.000E+00	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.040E+02	1.000E+20	.0000
2780.	1958.	822.	808.	1.000	655.	1.240E+02	1.000E+20	.0000
2780.	2104.	676.	576.	1.000	514.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.660E+02	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	6.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+00	1.000E+20	.0000
2763.	2352.	411.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	4.300E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.700E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	2.280E+02	1.000E+20	.0000
2496.	2441.	55.	177.	1.000	78.	5.300E+01	1.000E+20	.0000
2496.	2445.	51.	181.	1.000	77.	2.470E+02	1.000E+20	.0000
2491.	2445.	46.	178.	1.000	74.	1.000E+01	1.000E+20	.0000
2487.	2445.	42.	175.	1.000	71.	4.300E+01	1.000E+20	.0000
2487.	2487.	0.	0.	1.000	0.	1.700E+01	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0015

Table 4-5: Fatigue Results for the Reactor Recirculation Outlet Nozzle Blend Radius

```
       LOCATION = LOCATION NO. 2 -- BLEND RADIUS
  FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
              m = 2.0
              n =  .2
             Sm = 26700. psi
         Ecurve = 3.000E+07 psi
      Eanalysis = 2.670E+07 psi
             Kt = 1.00
```

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
55601.	17.	55584.	37389.	1.000	31227.	1.000E+00	1.951E+04	.0001
53822.	17.	53806.	48014.	1.000	30228.	1.000E+01	2.161E+04	.0005
51017.	17.	51001.	44054.	1.000	28652.	1.000E+01	2.547E+04	.0004
48939.	17.	48922.	52579.	1.000	27484.	1.000E+00	2.894E+04	.0000
46249.	17.	46233.	48340.	1.000	25974.	1.000E+00	3.443E+04	.0000
40607.	17.	40590.	36593.	1.000	22803.	1.000E+01	5.217E+04	.0002
39965.	17.	39948.	39059.	1.000	22443.	1.000E+01	5.647E+04	.0002
38737.	17.	38721.	40267.	1.000	21753.	6.000E+01	6.592E+04	.0009
38243.	17.	38227.	37209.	1.000	21476.	1.000E+01	7.025E+04	.0001
37757.	17.	37740.	37702.	1.000	21202.	7.000E+00	7.486E+04	.0001
37757.	476.	37281.	37314.	1.000	20945.	2.930E+02	7.954E+04	.0037
36291.	476.	35815.	36492.	1.000	20121.	7.000E+00	9.705E+04	.0001
36291.	1582.	34709.	35198.	1.000	19500.	3.000E+00	1.096E+05	.0000
36273.	1582.	34691.	35110.	1.000	19490.	6.000E+01	1.098E+05	.0005
36273.	1582.	34691.	35110.	1.000	19490.	1.000E+00	1.098E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	5.600E+01	1.135E+05	.0005
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1844.	34110.	34858.	1.000	19163.	1.000E+00	1.167E+05	.0000
35954.	1926.	34028.	34917.	1.000	19117.	2.410E+02	1.177E+05	.0020
35837.	1926.	33911.	34978.	1.000	19051.	1.000E+01	1.191E+05	.0001
35658.	1926.	33732.	34661.	1.000	18951.	4.900E+01	1.214E+05	.0004
35658.	11988.	23670.	26901.	1.000	13298.	1.790E+02	5.728E+05	.0003
35556.	11988.	23568.	27109.	1.000	13240.	1.210E+02	5.955E+05	.0002
35556.	12180.	23376.	28326.	1.000	13133.	1.790E+02	6.411E+05	.0003
35279.	12180.	23099.	27958.	1.000	12977.	1.000E+01	7.138E+05	.0000
34973.	12180.	22793.	27831.	1.000	12805.	1.110E+02	8.050E+05	.0001
34973.	20655.	14318.	16974.	1.000	8044.	1.890E+02	7.421E+07	.0000
34843.	20655.	14188.	16887.	1.000	7971.	1.110E+02	7.983E+07	.0000
34843.	25750.	9093.	17221.	1.000	5108.	1.000E+00	1.000E+20	.0000
34843.	27368.	7475.	5178.	1.000	4199.	1.000E+01	1.000E+20	.0000
34843.	28113.	6730.	3789.	1.000	3781.	1.780E+02	1.000E+20	.0000
34837.	28113.	6724.	3785.	1.000	3777.	1.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	6.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	1.000E+00	1.000E+20	.0000
34831.	28113.	6718.	3782.	1.000	3774.	1.000E+01	1.000E+20	.0000
34829.	28113.	6716.	3781.	1.000	3773.	4.100E+01	1.000E+20	.0000
34829.	29216.	5613.	1808.	1.000	3153.	1.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	6.000E+01	1.000E+20	.0000

Table 4-5: Fatigue Results for Reactor Recirculation Outlet Nozzle Blend Radius (concluded)

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
34829.	31990.	2839.	1543.	1.000	1595.	1.000E+00	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.160E+02	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	6.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	4.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	6.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.000E+00	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.270E+02	1.000E+20	.0000
34646.	32634.	2012.	2695.	1.000	1130.	1.010E+02	1.000E+20	.0000
34646.	33386.	1260.	1578.	1.000	708.	1.000E+00	1.000E+20	.0000
34646.	33581.	1065.	1120.	1.000	598.	1.000E+01	1.000E+20	.0000
34646.	33709.	937.	1137.	1.000	526.	1.000E+01	1.000E+20	.0000
34646.	33822.	824.	1455.	1.000	463.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1228.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1310.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	34124.	522.	1067.	1.000	293.	1.000E+01	1.000E+20	.0000
34646.	34331.	315.	3398.	1.000	177.	1.000E+01	1.000E+20	.0000
34646.	34447.	199.	-603.	1.000	112.	1.200E+02	1.000E+20	.0000
34646.	34592.	54.	-130.	1.000	30.	3.500E+01	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0108

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [18] for various regions of the VY RPV and attached piping.

The Recirculation Outlet nozzle has three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified; locations in the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the FEM analysis occur in the stainless steel safe end. Stainless steel F_{en} multipliers are significantly higher than Ni-Cr-Fe multipliers (F_{en} values are 2.55 or higher for stainless steel [18] vs. a constant value of 1.49 for Ni-Cr-Fe [19]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.

- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel F_{en} multipliers are higher than Ni-Cr-Fe multipliers (F_{en} values are 2.45 or higher for low alloy steel [18] vs. a constant value of 1.49 for Ni-Cr-Fe [19]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the reactor recirculation outlet nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV beltline region chemistry was assumed for both the reactor recirculation outlet nozzle safe end and blend radius locations, since both locations experience reactor conditions at all times.

For the safe end location, the environmental fatigue factors for NWC and HWC are 8.36 and 15.35, respectively, from Table 5 of Reference [18] for the RPV beltline region. This results in an EAF adjusted CUF as follows:

60-Year CUF, $U_{60} = 0.0015$

Overall EAF multiplier, $F_{en} = (8.36 \times 53\% + 15.35 \times 47\%) = 11.65$

60-Year EAF CUF, $U_{60\text{-}env} = 0.0015 \times 11.65 = 0.0175$

The EAF CUF value of 0.0175 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [17]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [1]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for NWC and HWC are 12.43 and 2.45, respectively, from Table 5 of Reference [18] for the RPV beltline region. This results in an EAF adjusted CUF as follows:

60-Year CUF, $U_{60} = 0.0108$

Overall EAF multiplier, $F_{en} = (12.43 \times 53\% + 2.45 \times 47\%) = 7.74$

60-Year EAF CUF, $U_{60\text{-}env} = 0.0108 \times 7.74 = 0.0836$

The EAF CUF value of 0.0836 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY reactor recirculation outlet nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the reactor recirculation outlet nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY reactor recirculation outlet nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [17]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.0015, and the CUF for the limiting blend radius location is 0.0108. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY reactor recirculation outlet nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.0175, and the EAF CUF for the limiting blend radius location is 0.0836. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

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13. Nozzle Thermal Cycles (Recirculation Outlet), Attachment 1, page 4 of Entergy Design Input Record (DIR) EC No. 1773, Rev. 1, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
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Environmental Fatigue Analysis for the Vermont Yankee Reactor Pressure Vessel Core Spray Nozzle

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Section	Pages	Revision	Date	Comments
1.0	1-1 – 1-5	0	07/26/07	Initial issue.
2.0	2-1 – 2-3			
3.0	3-1 – 3-26			
4.0	4-1 – 4-8			
5.0	5-1 – 5-2			
6.0	6-1			
7.0	7-1 – 7-3			

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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) values for the reactor pressure vessel (RPV) core spray nozzle are reported as 0.625 (nozzle) and 0.182 (safe end). The safe end value was reported as a generic value, since no plant-specific value was determined. Application of environmentally assisted fatigue (EAF) multipliers, as required for the license renewal period, resulted in unacceptable EAF CUF values of 1.53 and 2.79 for the nozzle and safe end, respectively. Therefore, further refined analysis was necessitated to show acceptable EAF CUF results for this component.

This report documents a refined fatigue evaluation for the VY core spray nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the core spray nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The EAF effects for the core spray piping, which is also a NUREG/CR-6260 location, are considered to be covered by the nozzle and safe end calculations because the nozzle region bounds the piping[1]. The resulting fatigue results will be used as a replacement to the values previously reported in the VY LRA.

The refined evaluation summarized in this report included the development of a detailed finite element model of the core spray nozzle, including relevant portions of the safe end, thermal sleeve, the RPV wall, and the weld overlay repair documented in Reference [2]. Thermal and pressure stress histories were developed for relevant transients affecting the core spray nozzle, including any effects of Extended Power Uprate (EPU), as specified by the VY RPV Design Specification [3], the VY EPU Design Specification [4] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress and primary plus secondary stress for use in a subsequent fatigue evaluation. Stresses were also included due to loads from the attached piping for application in the stress/fatigue analysis, based on the bounding reaction loads obtained from the

[1] The nozzle stresses are more severe due to the nozzle discontinuity, and the nozzle thermal transients are more severe due to interaction with the hot RPV.

relevant design documents. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code, and were performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

For the core spray nozzle evaluated as a part of this work, stress histories were computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [5]. A general outline is provided in this section.

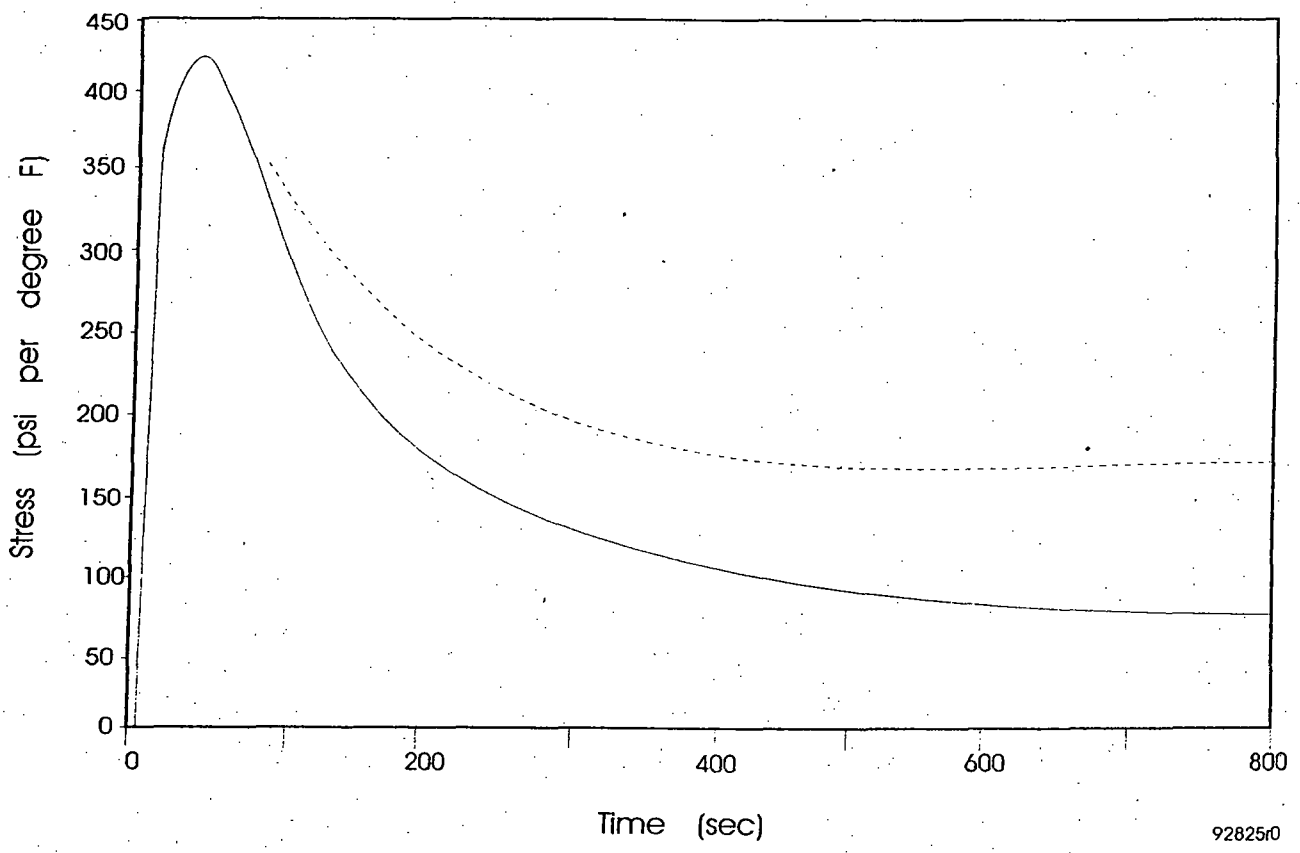
A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual

transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY core spray nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1: Typical Green's Functions for Thermal Transient Stress

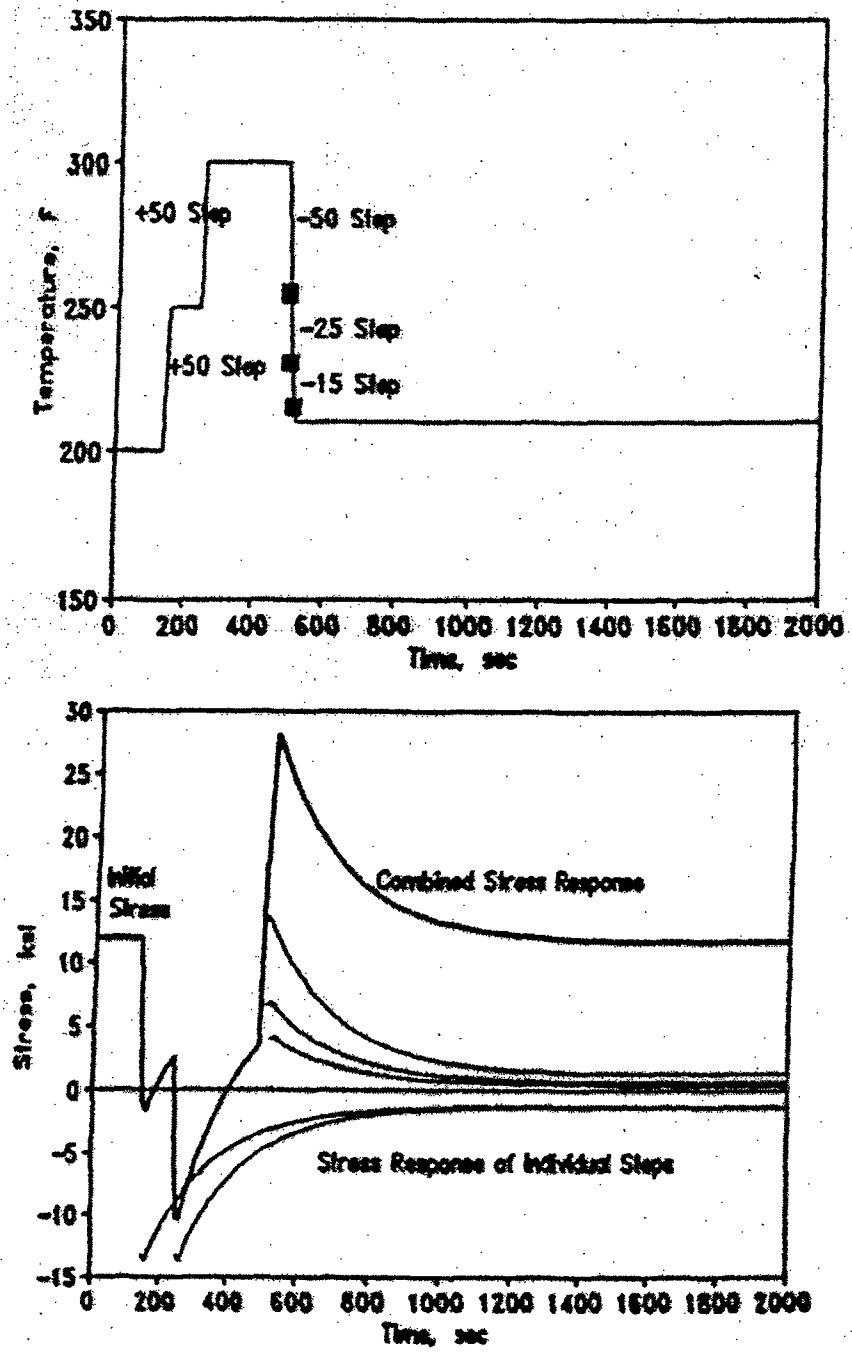


Figure 1-2: Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

An ANSYS [6] finite element model (FEM) of the VY core spray nozzle and safe end was developed and used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [7] calculation.

The materials of the various components of the model are listed below:

- Safe End – SB 166 (72Ni-15Cr-8Fe, N06600)
- 8Ø x 10Ø Conc. Reduction – SA312 TP304 (18Cr-8Ni)
- Nozzle Forging – SA508 Class II (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 TP 304 (18Cr-8Ni)

In the FEM model, the radius of RPV was increased by a factor of two to account for the fact that the vessel portion of the finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8], and are shown in Table 2-1. The properties were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which was applied to the FEM model for Green's Function development.

The finite element model, which includes the weld overlay, is shown in Figure 2-1.

Table 2-1: Material Properties @ 300°F [1]

Material ID	Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft^2/hr	Specific H Btu/lb-° [C] [3]
2	Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
2	Weld Overlay	INCONEL 82	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
1	Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193
3	Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193
4	3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252
4	8Ø x 10Ø Conc. Reduction[2]	SA312 [2] TP304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252

Note:
1. Material properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8 Poisson's ratio, which are assumed typical values and specific heat is calculated as $[k/\rho d]/12^3$.
2. The 8Ø x 10Ø concentric reduction was modeled as a straight pipe with the material properties of the original de was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity properties.

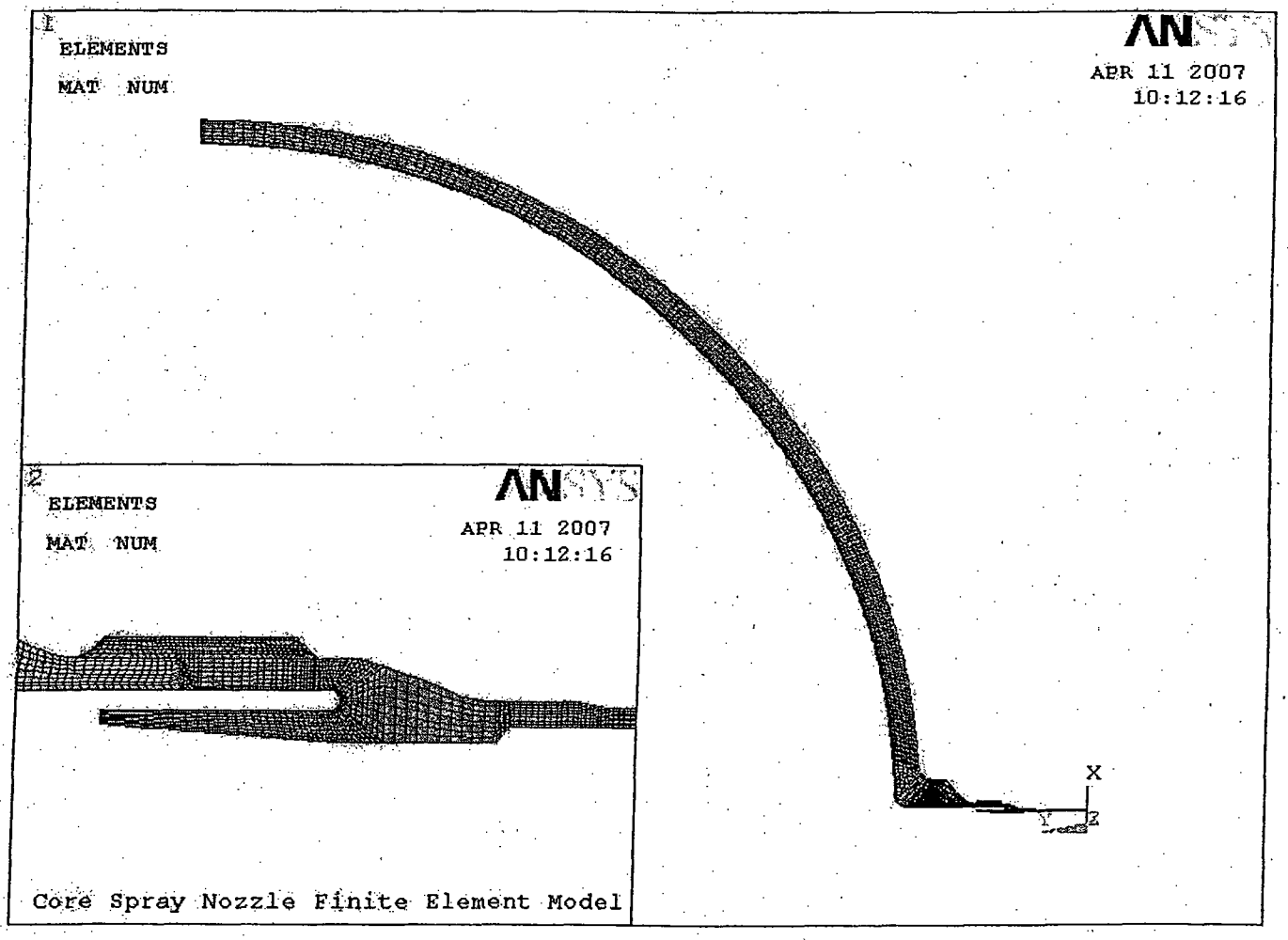


Figure 2-1: VY Core Spray Nozzle FEM

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the core spray nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model, described in Section 2.0 of this report. The details of the Green's function development and associated stress evaluation are documented in the Reference [9] and [10] calculations.

3.1 Thermal Loading

Thermal loads were applied to the core spray nozzle model to generate the Green's Function. As a first step in the Green's Function process, heat transfer coefficients were determined for various regions of the core spray FEM for two different flow cases: (1) 0% core spray flow, and (2) 100% core spray flow through the nozzle.

The 0% flow case simulates a stagnant condition of the core spray nozzle when not in operation and the entire core spray nozzle is at the same temperature as the RPV fluid. The heat transfer coefficients for the 0% flow case are for free convection (stagnant) conditions. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle in the stagnant condition.

The 100% flow case simulates the operational condition of the core spray nozzle (i.e., the entire core spray nozzle experiences 100°F water due to injection). The heat transfer coefficients for the high flow case are for forced and free convection depending on the region of the FEM. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle due to injection.

The temperature on the exterior of the reactor, nozzle, safe end and pipe was assumed to be 120°F (ambient). Figure 3-4 shows the heat transfer coefficient regions assumed for the core spray nozzle FEM. The applied heat transfer coefficients and the fluid temperatures are summarized in the sections that follow.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-4, heat transfer coefficients were applied as follows:

- The heat transfer coefficient for the outside surfaces of the FEM (Region 12) was a constant value of 0.2 BTU/hr-ft^2-°F (3.858×10^{-7} BTU/sec-in^2-°F).
- Table 3-1 shows the results of the heat transfer coefficient calculations for all of the thermal regions identified in Figure 3-4. The detailed heat transfer calculations for Regions 1, 3, 5, 7, 9, and 11 are contained in the Reference [9] calculation.
- In Regions 2, 4, 6, 8, and 10, the heat transfer coefficients are interpolated.

For both Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response. For the 0% flow case, the entire inside surface of the FEM was shocked. For the 100% flow case, only the nozzle flow path was shocked.

3.1.2 Green's Functions

The two flow-dependent thermal load cases outlined in previous section were run on the core spray nozzle FEM with the heat transfer coefficients and the fluid temperature conditions listed in Table 3-1. Two locations were selected for analysis (see Figures 3-5 and 3-6):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under nozzle flow conditions. The highest stress intensity due to thermal loading occurred at Node 3719 (see Figure 3-5), on the inside diameter of the nozzle safe end. Therefore, this node was selected for analysis.
2. The critical blend radius location was chosen based upon the highest pressure stress intensity. The critical location was selected as Node 2166, as shown in Figure 3-6.

Two stress intensity Green's Functions were developed for each location and each flow case: (1) total stress intensity, and (2) membrane plus bending stress intensity. The total stress intensity

Green's Functions for the safe end location are shown in Figures 3-7 and 3-8. The total stress intensity Green's Functions for the blend radius location are shown in Figures 3-9 and 3-10.

3.1.3 Thermal Transients

The transients analyzed for the core spray nozzle were developed based on the definitions in the original RPV Design Specification [3], as modified for EPU [4], as well as more recent definitions based on BWR operating experience. For BWR operating experience, the transients described in the thermal cycle diagrams for a BWR-4 plant similar in design and vintage to VY were obtained, and plant data from VY applied to each transient. The resulting thermal cycle diagrams are shown in References [11 and 12]. The final transients evaluated in the stress and fatigue analyses are shown in Figures 3-11 through 3-16.

The number of cycles projected for the 60-year operating life is used for each transient [13]. Tables 3-2 and 3-3 summarize the thermal transients for the safe end and blend radius locations, respectively.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the core spray nozzle and the RPV wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load of 4,774 psi was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{cap} = \frac{PD_i^2}{\left(D_o^2 - D_i^2\right)}$$

where:

	P_{cap}	=	end cap pressure load (psi)
	P	=	unit pressure load = 1,000 psi
	D_i	=	inside diameter of end of FEM = 9.834"

D_o = outside diameter of end of FEM = 10.815"

The calculated pressure was applied as a negative value so that it would exert tension on the end of the model. The nodes on the end of the FEM were coupled in the axial direction to ensure mutual displacement of the end of the nozzle due to attached piping. Figures 3-1, 3-2, and 3-3 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

The internal pressure load case for Node 2166 (blend radius) resulted in a total stress intensity of 35,860 psi, and for Node 3719 (safe end) resulted in a total stress intensity of 12,030 psi. The membrane plus bending stress intensity at Node 2166 and Node 3719 are 34,970 psi and 12,020 psi, respectively.

3.3 Piping Loading

The piping stress intensities (stress caused by the attached piping) were determined for the two evaluated core spray nozzle locations. The design piping reactions that were used in the stress and fatigue evaluation are defined on the Reference [14] drawing. These loads represent shear and moment loadings on the nozzle resulting from thermal expansion of the attached piping and seismic loads. The loads are applicable at the piping end of the safe end, as shown in Figure 3-17. The stresses resulting from these loads were calculated by hand using classical structural mechanics formulas, as documented in Reference [10], and are shown in Tables 3-4 and 3-5 for the safe end and blend radius locations, respectively.

Table 3-1: Summary of Heat Transfer Coefficients

Regions	0% Flow		100% Flow	
	Initial Temperature °F	HTC Btu/hr-ft^2-°F	Initial Temperature °F	HTC Btu/hr-ft^2-°F
R1	500	143	500	2693
R2	500	Interpolated	500	Interpolated
R3 (1)	500	39	500	52
R4	500	Interpolated	500	Interpolated
R5 (1)	500	47	500	66
R6	500	Interpolated	500	Interpolated
R6B	500	97	500	97
R7A (1)	500	38	500	50
R7B (1)	500	20	500	23
R8	500	Interpolated	500	Interpolated
R9 (1)	500	33	500	41
R10	500	Interpolated	500	Interpolated
R11	500	500	500	500
R12	120	0.20	120	0.2

Table 3-2: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test 120 Cycles	---	100	---	0 1100 50	
3. Startup 300 Cycles	0 16164 17164	100 549 549	 16164 1000	0 1010 1010	0 (0%)
11. Loss of Feedwater Pumps 10 Cycles	0 3 13 233 2213 2393 6893 7313 7613 11213 16577 16637 16638 16698 16699 17699	526 526 526 300 500 300 500 300 300 400 549 549 542 542 526 526	 3 10 220 1980 180 4500 420 300 3600 5364 60 1 60 1 1000	1010 1190 1135 1135 1135 885 1135 675 675 240 1010 1010 1010 1010 1010 1010	0 (0%)
14. SRV Blowdown 1 Cycle	0 600 11580 12580	526 375 70 70	 600 10980 1000	1010 400 50 50	0 (0%)
21-23. Shutdown 300 Cycles	0 6264 6864 16224 17224	549 375 330 100 100	 6264 600 9360 1000	1010 50 50 50 50	0 (0%)
12. Hydrostatic Test 1 cycle	---	100	---	50 1563 50	
30. Emergency Shut Down 1 Cycle	0 10 11 1011	549 406 70 70	 10 1 1000	1010 250 250 0	3200 (100%)

Table 3-3: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test	---	100	---	0	
				1100	
120 Cycles				50	
3. Startup	0	100		0	0
300 Cycles	16164	549	16164	1010	(0%)
	24164	549	8000	1010	
11. Loss of Feedwater	0	526		1010	0
Pumps	3	526	3	1190	(0%)
10 Cycles	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
	16698	542	60	1010	
	16699	526	1	1010	
	24699	526	8000	1010	
14. SRV Blowdown	0	526		1010	0
1 Cycle	600	375	600	400	(0%)
	11580	70	10980	50	
	19580	70	8000	50	
21-23. Shutdown	0	549		1010	0
300 Cycles	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
	24224	100	8000	50	
24. Hydrostatic Test	---	100	---	50	
1 Cycle				1563	
				50	
30. Emergency Shut Down	0	549		1010	3200
1 Cycle	10	406	10	250	(100%)
	11	70	1	250	
	8011	70	8000	0	

Table 3-4: Stresses Due to Piping Loads for Safe End Location

Safe End External Piping Loads		
Parameters		
F_x =	2.50	kips
F_y =	4.60	kips
F_Z =	1.70	kips
M_x =	264.00	in-kips
M_y =	85.20	in-kips
M_Z =	105.60	in-kips
OD=	10.82	in
ID=	9.834	in
R_N =	5.16	in
L =	0.30	in
t_N =	0.49	in
$(M_x)_1$ =	262.60	in-kips
$(M_y)_1$ =	85.96	in-kips
M_{xy} =	276.31	in-kips
F_{xy} =	5.24	kips
N_Z =	3.35	kips/in
q_N =	-0.31	kips/in
Primary Membrane Stress Intensity		
PM_Z =	6.84	ksi
τ =	-0.63	ksi
SI_{max} =	6.95	ksi
SI_{max} =	6949.94	psi

Table 3-5: Stresses Due to Piping Loads for Blend Radius Location

Blend Radius External Piping Loads		
Parameters		
F_x =	2.50	kips
F_y =	4.60	kips
F_z =	1.70	kips
M_x =	264.00	in-kips
M_y =	85.20	in-kips
M_z =	105.60	in-kips
OD=	18.87	in
ID=	11.750	in
R_N =	7.65	in
L =	30.82	in
t_N =	3.56	in
$(M_x)_2$ =	122.24	in-kips
$(M_y)_2$ =	162.24	in-kips
M_{xy} =	203.14	in-kips
F_{xy} =	5.24	kips
N_z =	1.14	kips/in
q_N =	-0.07	kips/in
Primary Membrane Stress Intensity		
PM_z =	0.32	ksi
τ =	-0.02	ksi
SI_{max} =	0.32	ksi
SI_{max} =	322.52	psi

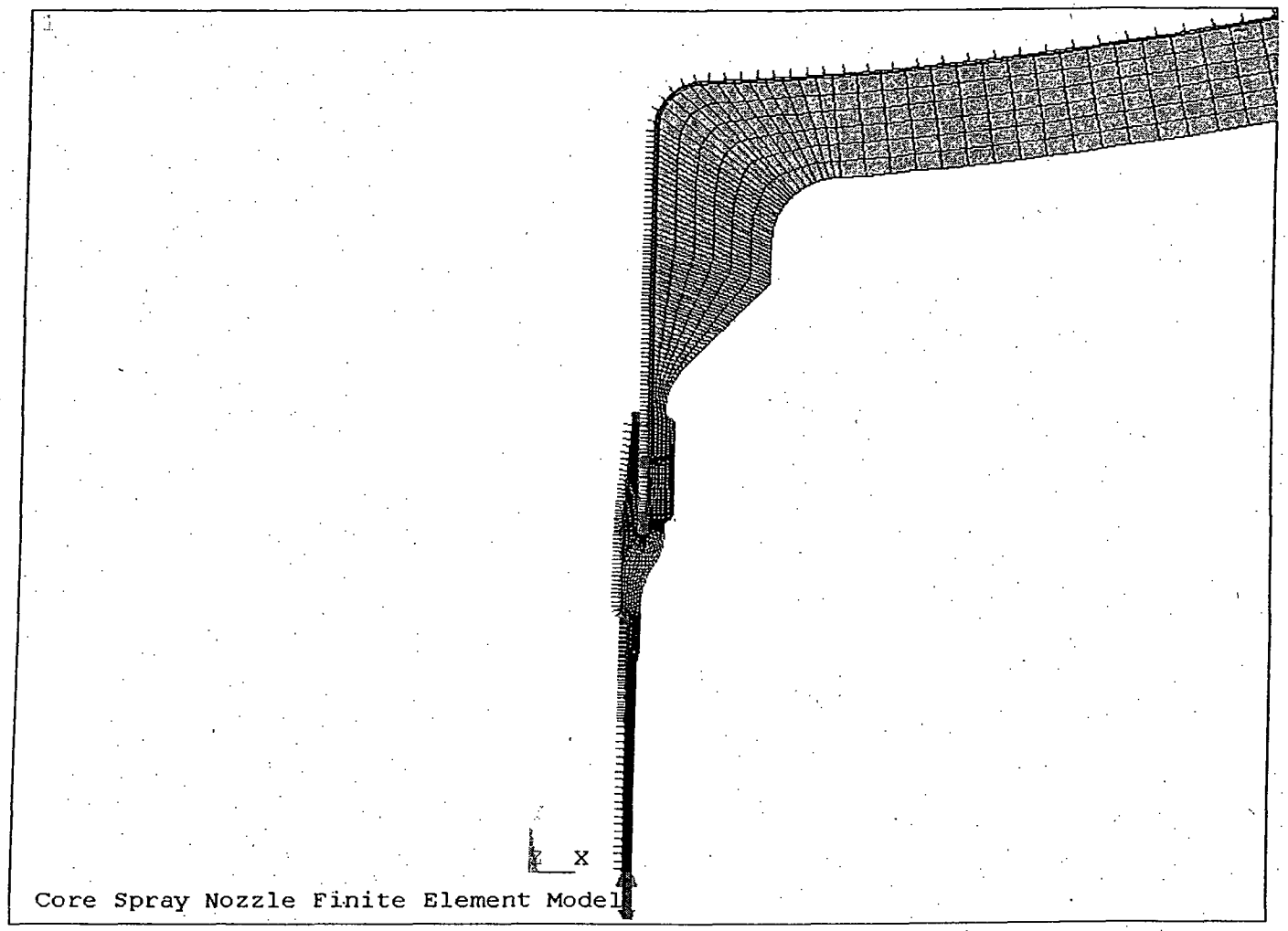


Figure 3-1: Core Spray Nozzle Internal Pressure Distribution

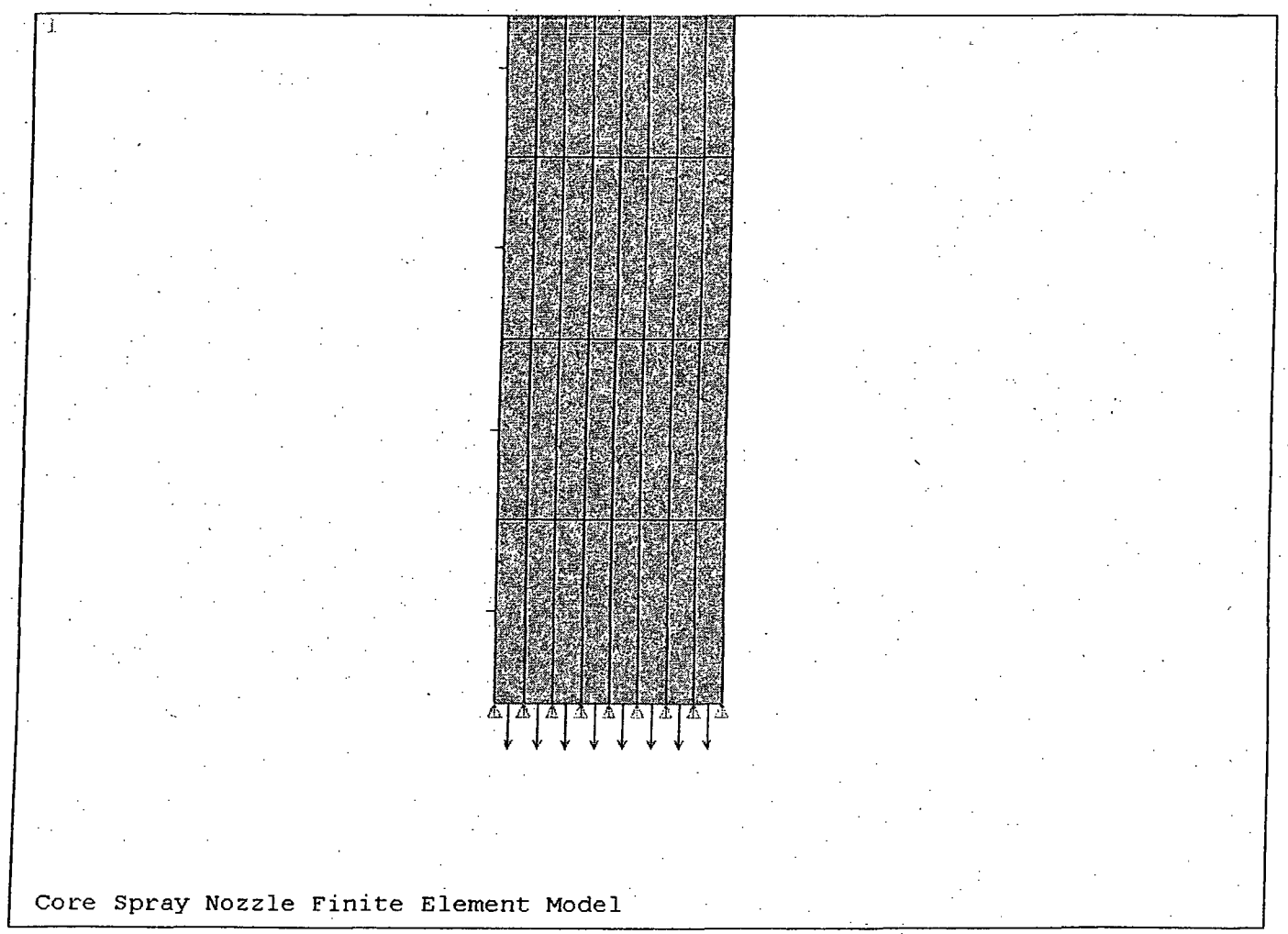


Figure 3-2: Core Spray Nozzle Pressure Cap Load

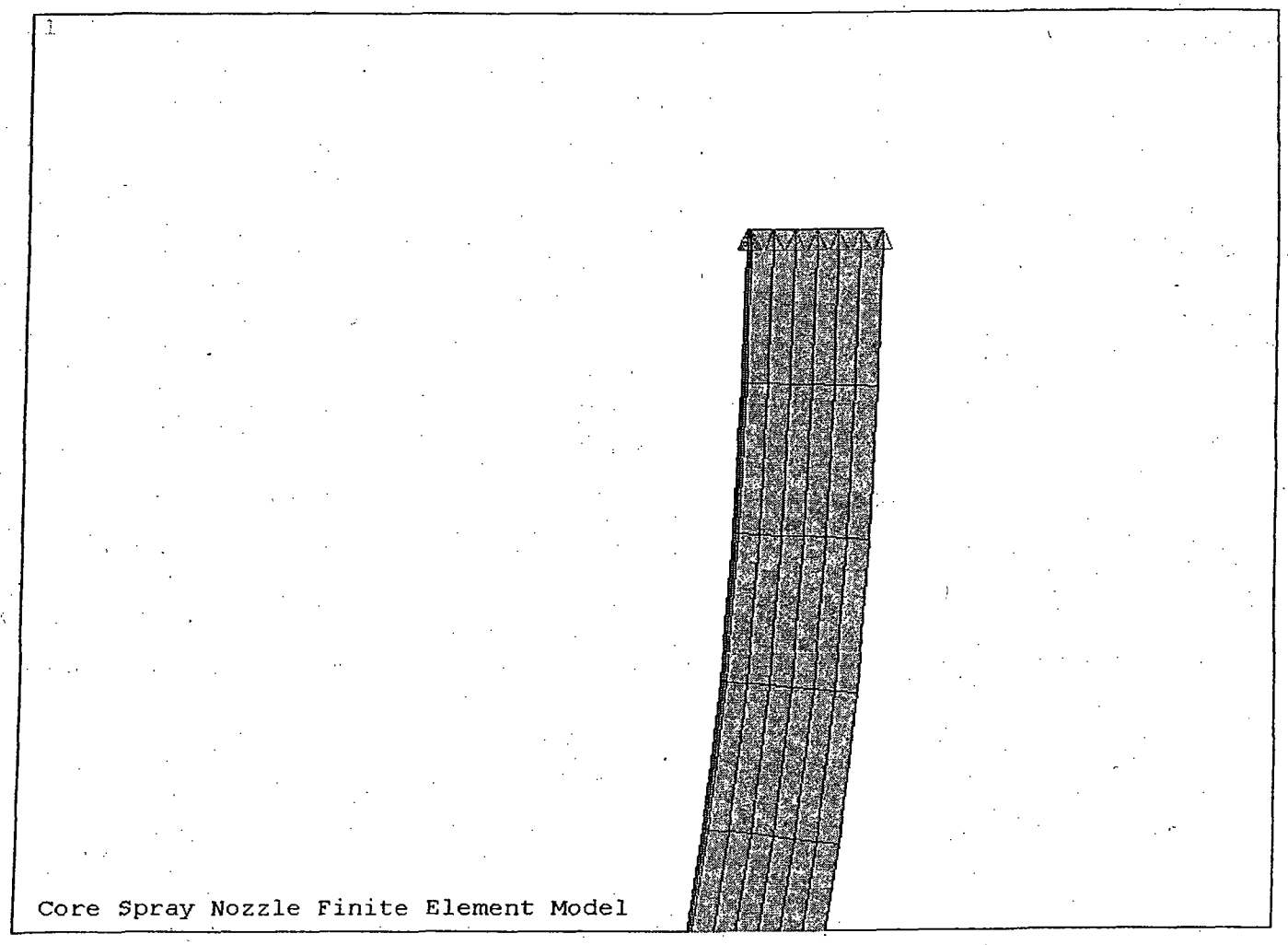


Figure 3-3: Core Spray Nozzle Vessel Boundary Condition

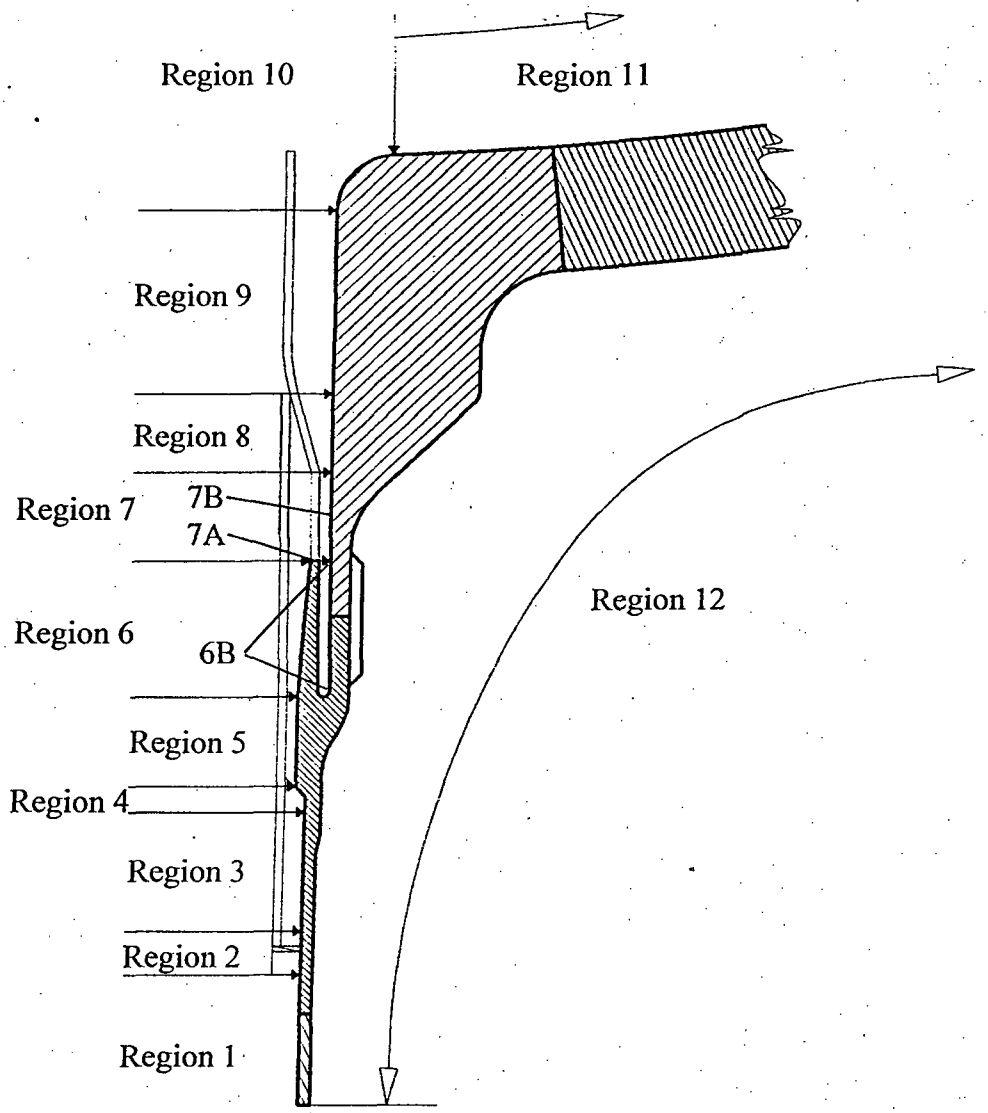


Figure 3-4: Thermal Regions

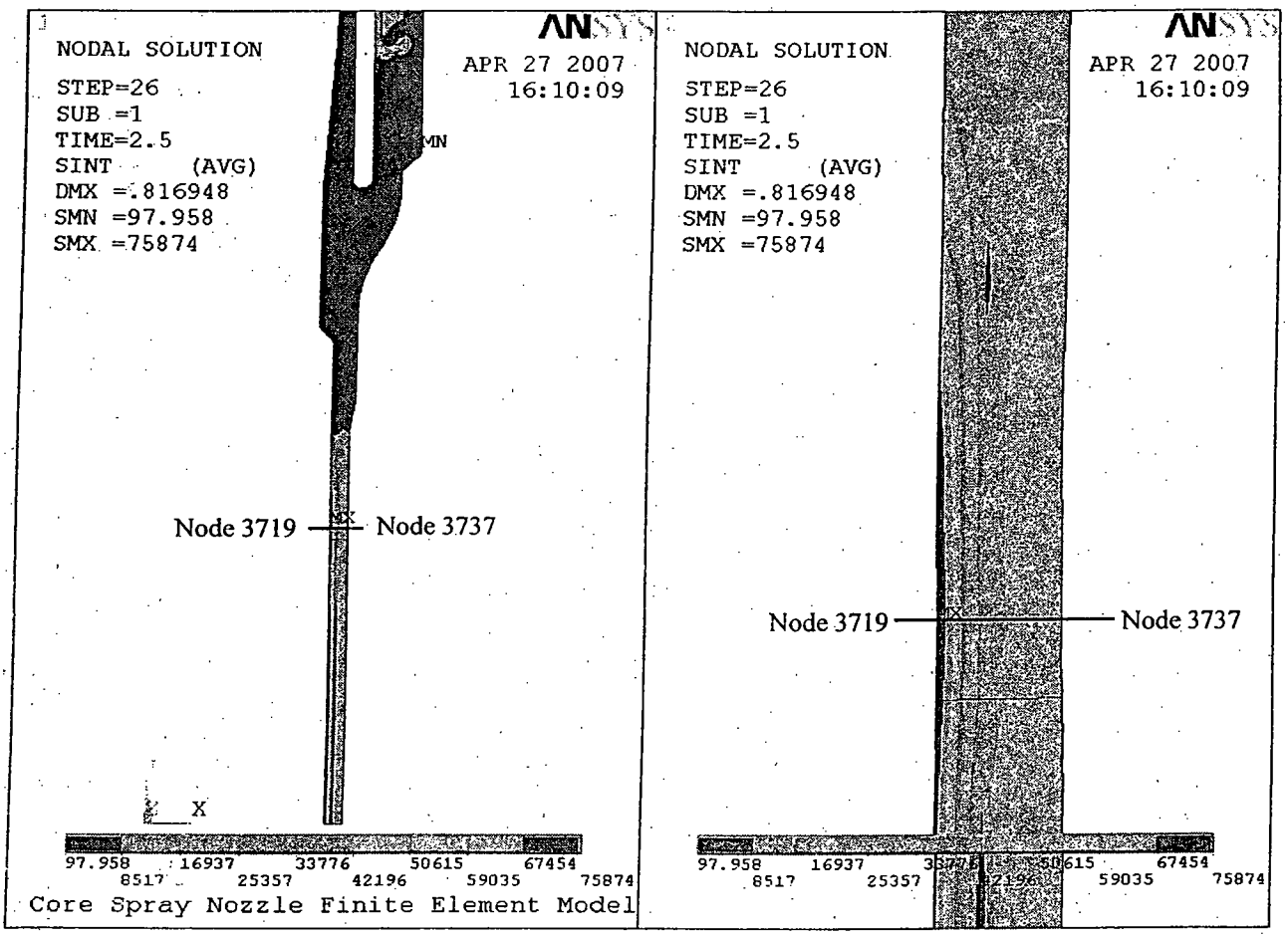


Figure 3-5: Safe End Critical Thermal Stress Location

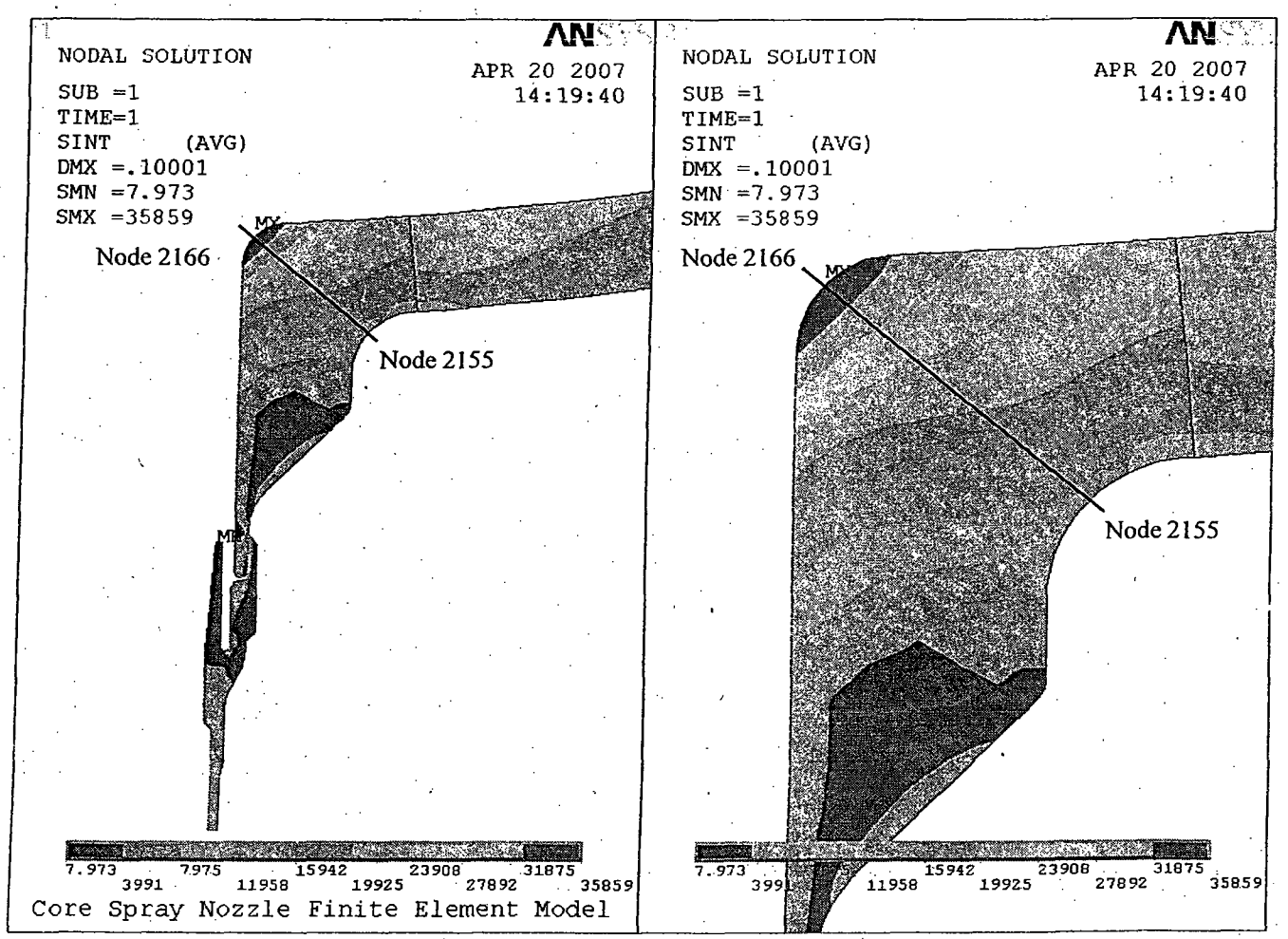


Figure 3-6: Blend Radius Critical Thermal Stress Location

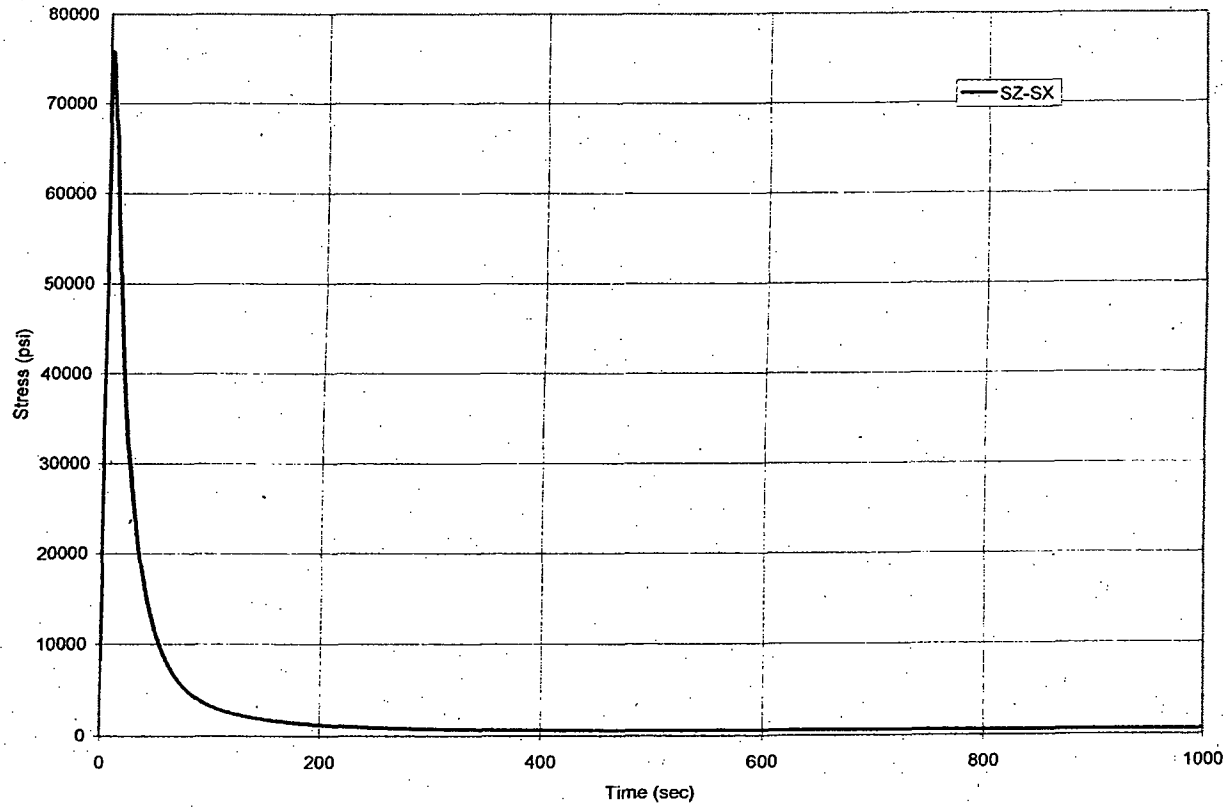


Figure 3-7: Safe End Green's Function for 100% Flow

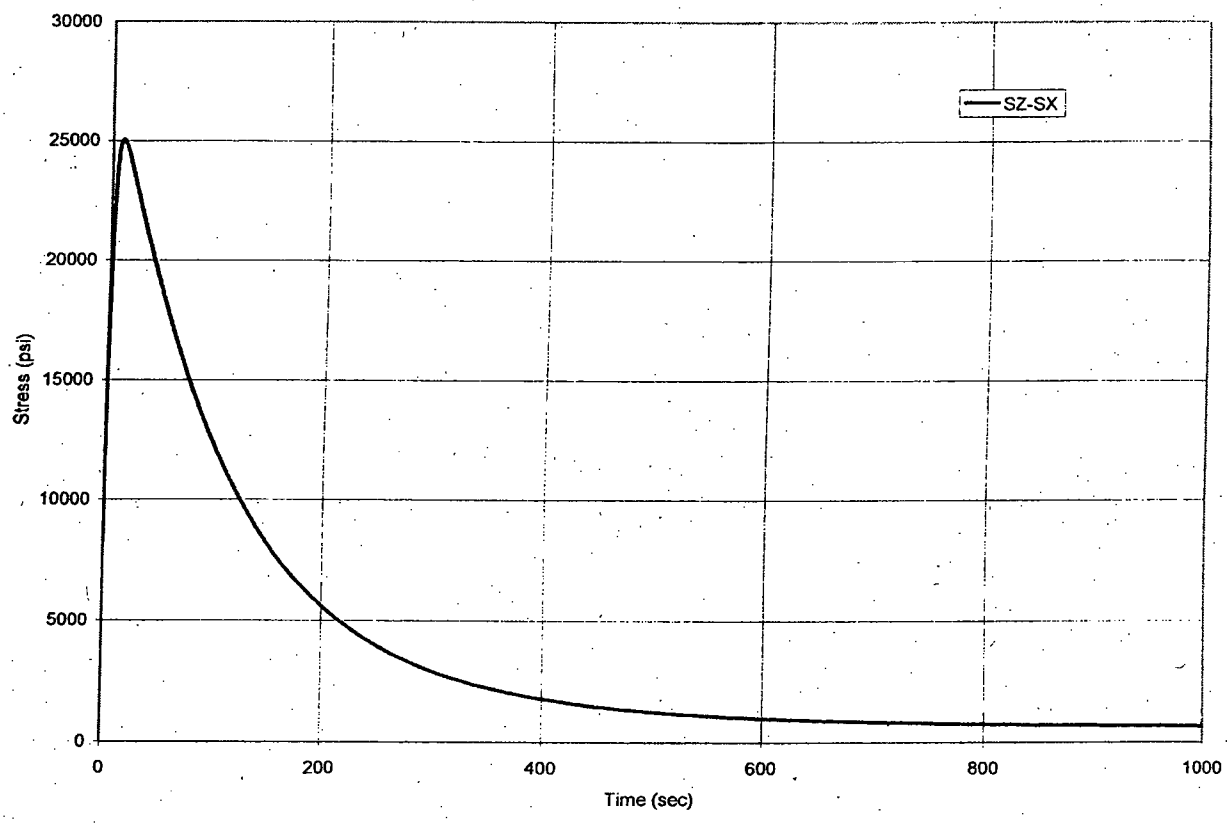


Figure 3-8: Safe End Green's Function for 0% Flow

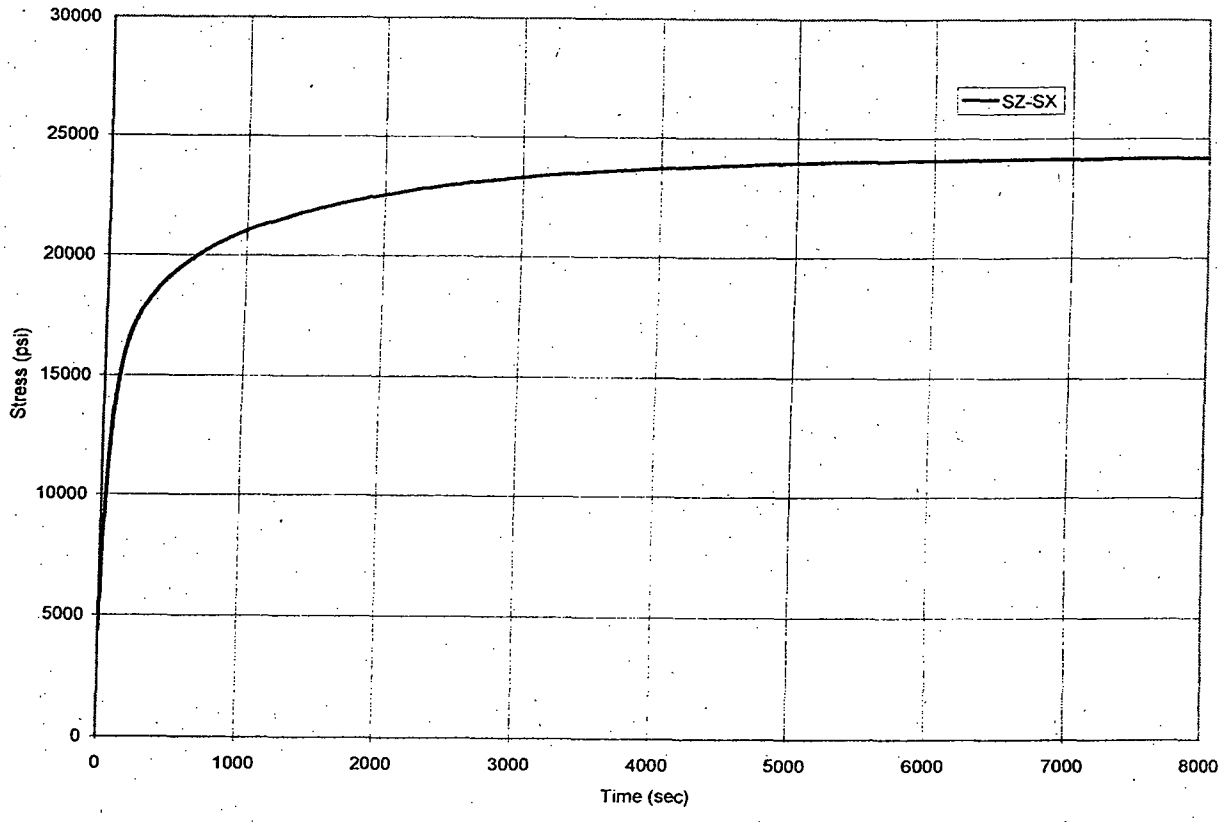


Figure 3-9: Blend Radius Green's Function for 100% Flow

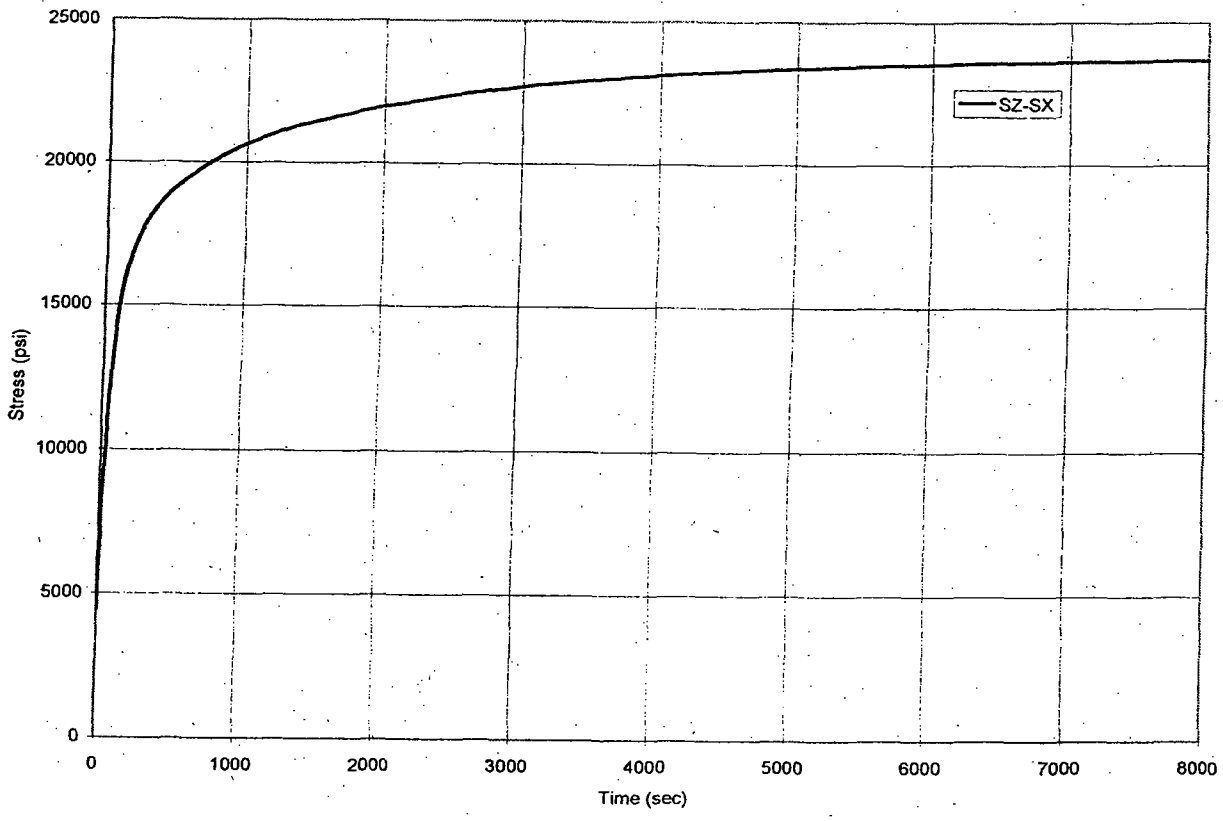


Figure 3-10: Blend Radius Green's Function for 0% Flow

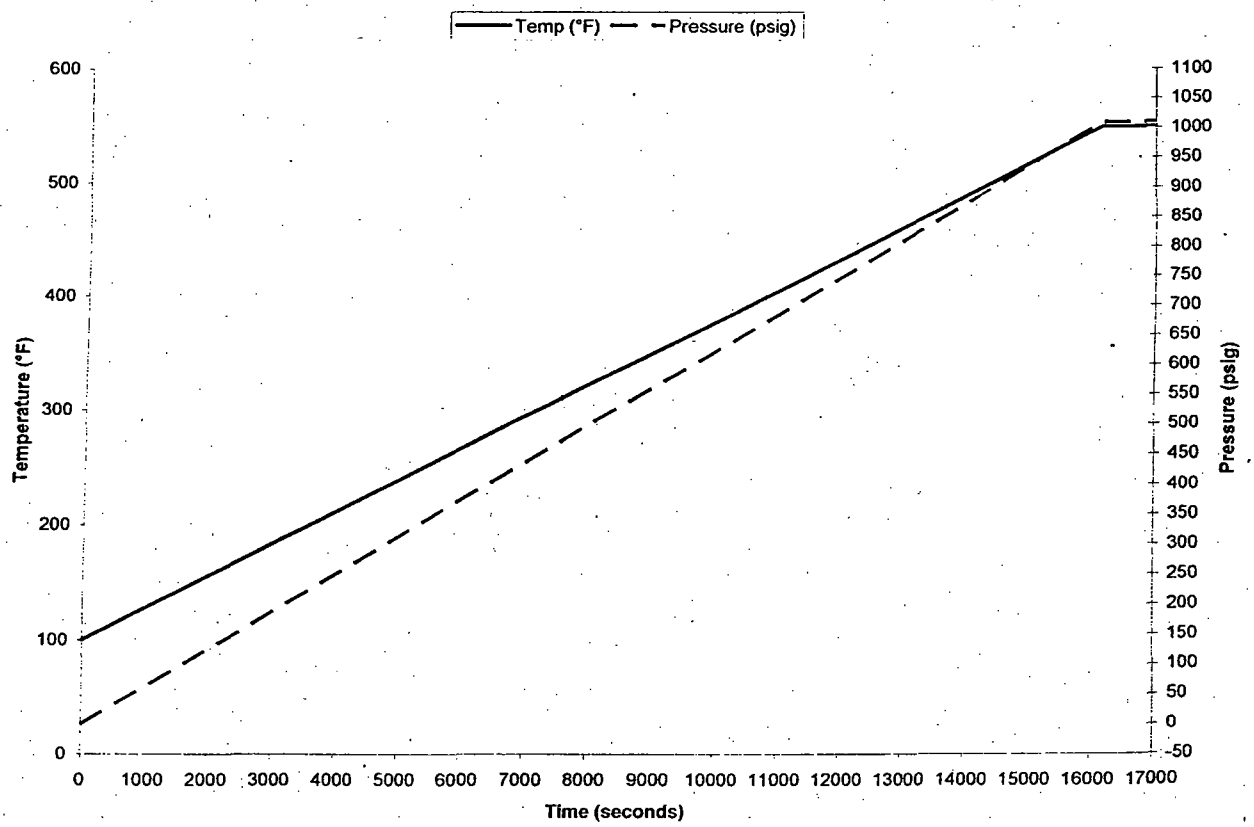


Figure 3-11: Transient 03: Startup

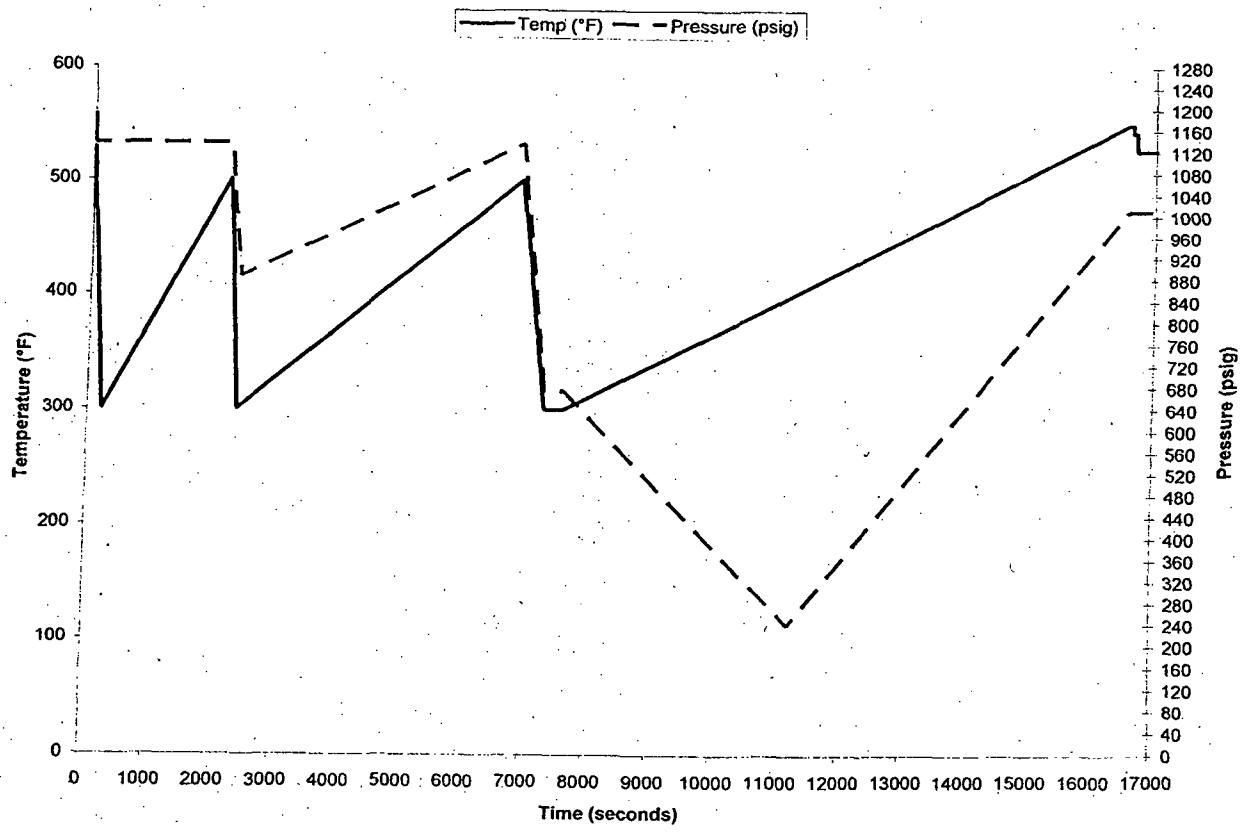


Figure 3-12: Transient 11: Loss of Feedwater Pumps, Isolation Valves Close

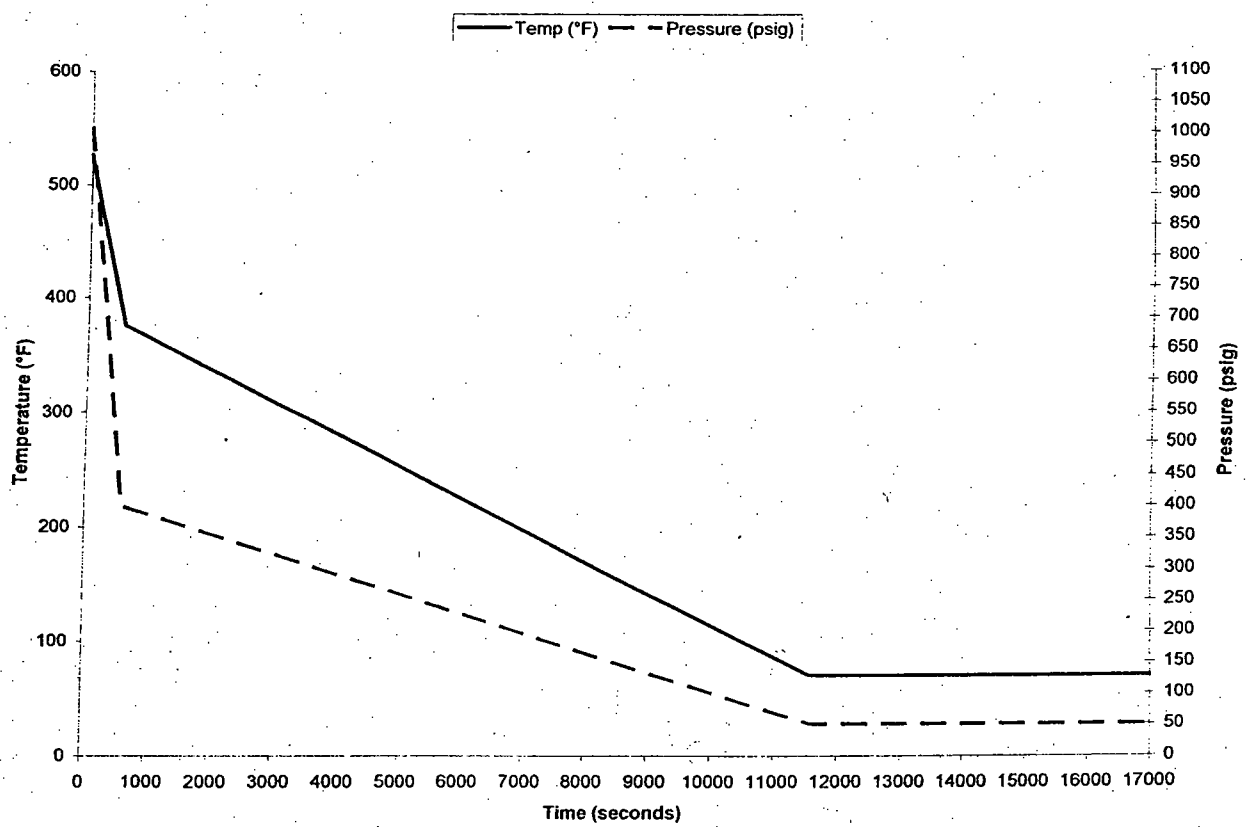


Figure 3-13: Transient 14: Single Relief of Safety Valve Blow Down

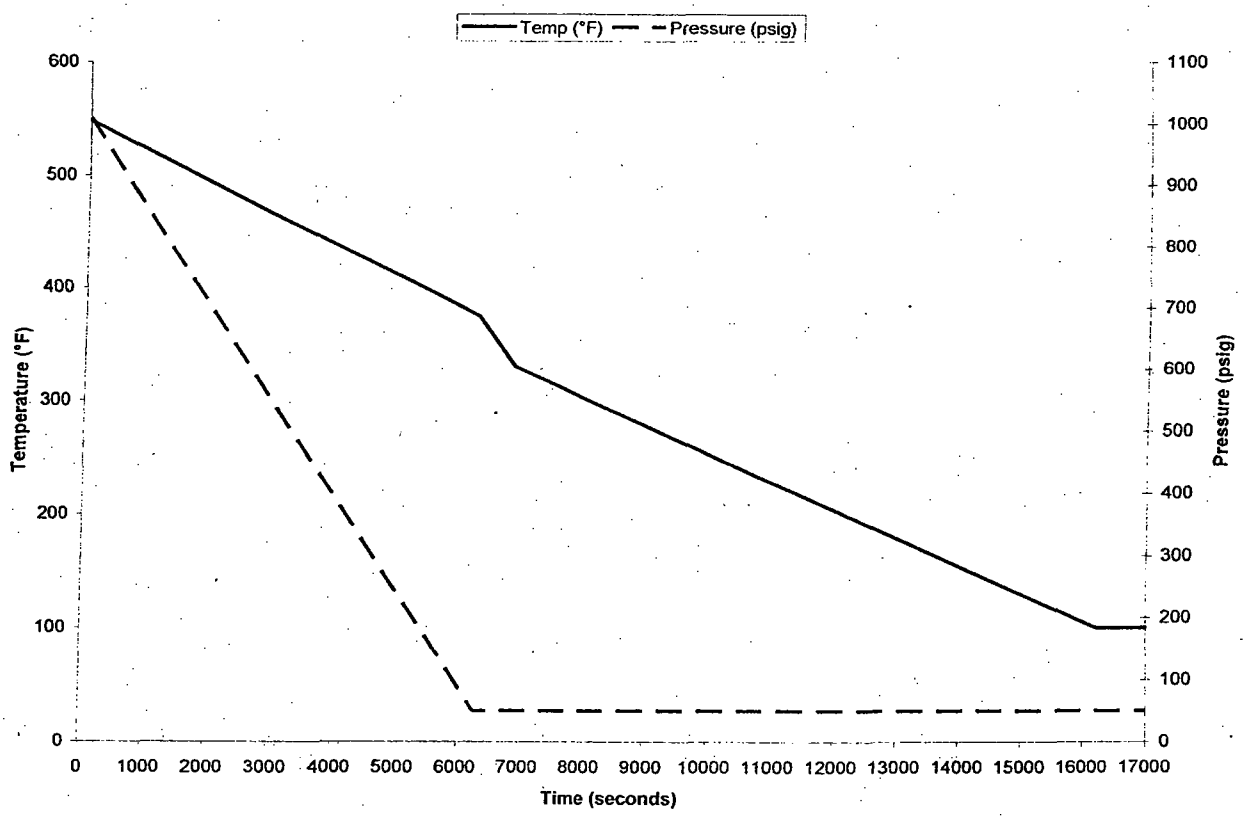


Figure 3-14: Transient 21-23: Shutdown Vessel Flooding

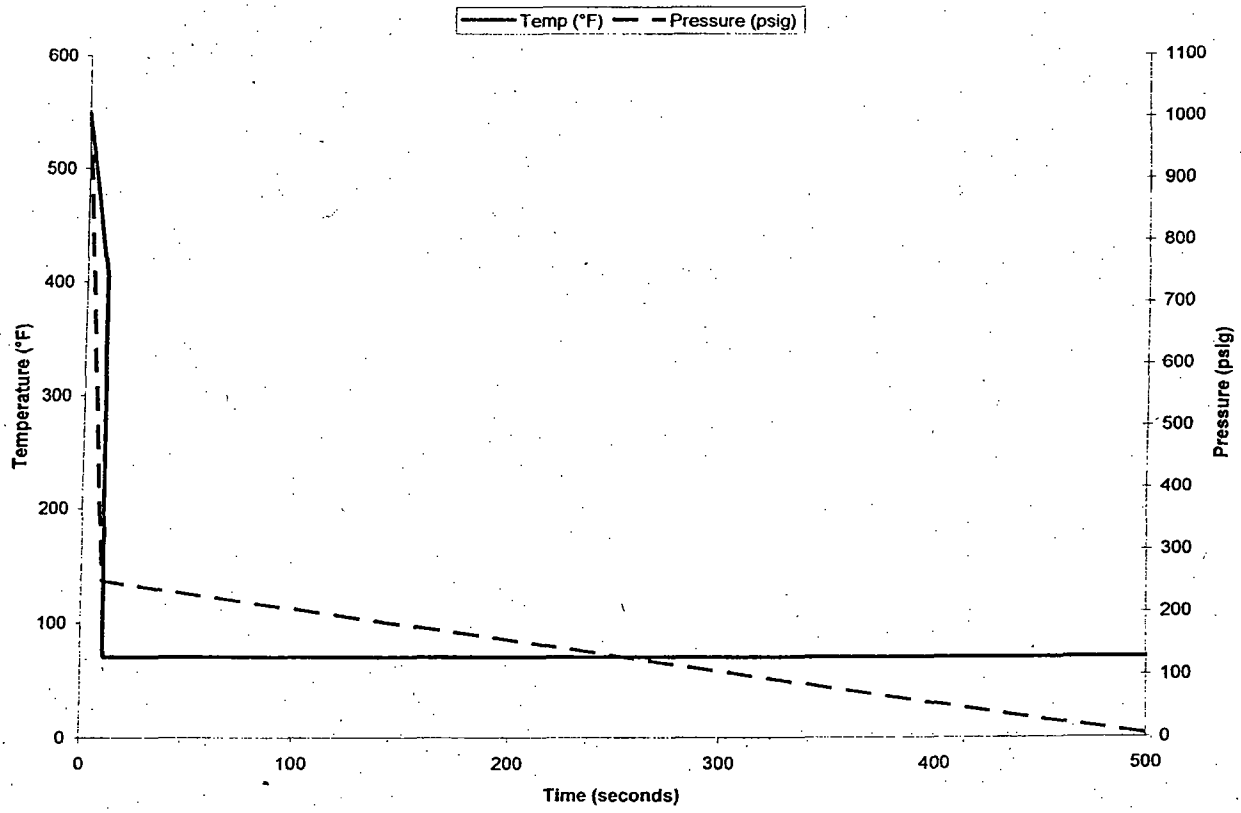


Figure 3-15: Transient 30: Emergency Shutdown 100% Flow (Safe End)

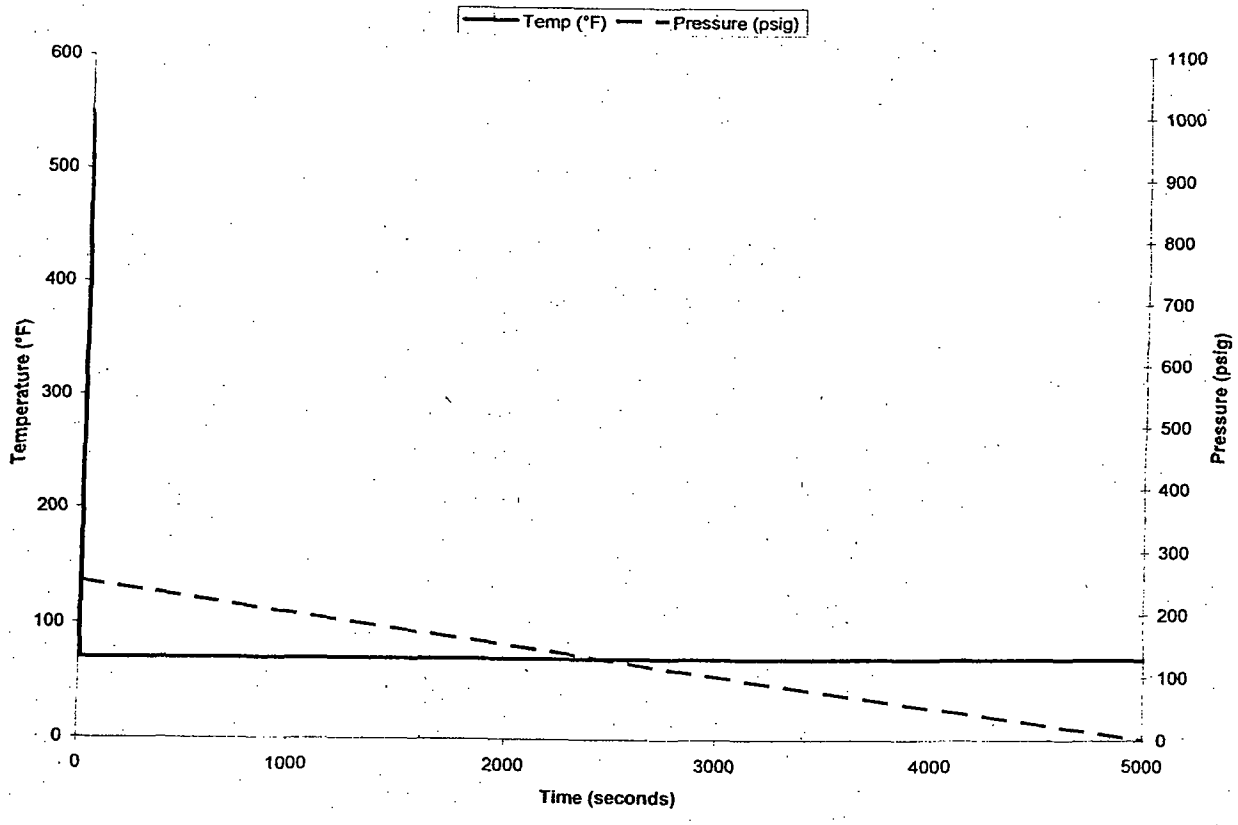


Figure 3-16: Transient 30: Emergency Shut Down 100% Flow (Blend Radius)

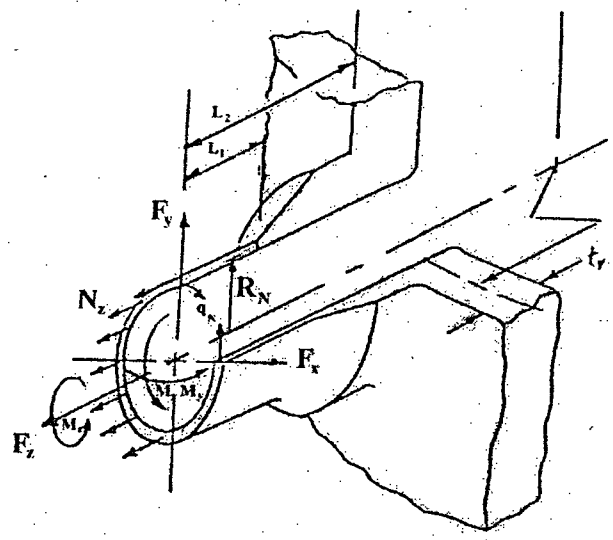


Figure 3-17: Pipe Reactions

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY core spray nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [16]. Fatigue analysis was performed in the Reference [10] calculation for the two locations identified in Section 3.1.2, using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [13].

Three computer programs were used to facilitate the fatigue analysis process: STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III methodology. All three programs are explained in detail and were independently verified for use in the Reference [15] calculation.

In order to perform the fatigue analysis, input files with the necessary data were prepared and the three analysis programs were run. The program STRESS.EXE required the following three input files:

- **Green.dat**: This file contains the Green's Function. As discussed above, the core spray nozzle analyses utilize four Green's Functions: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function for both the safe end and blend radius locations.
- **Green.cfg**: A configuration file containing parameters that describe the Green's Function.
- **Transnt.inp**: This file contains the input transient history defined in Tables 3-2 and 3-3.

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the safe end) and Table 4-2 (for the blend radius) show

the final peak and valley output. The pressure values for Column 6 in each table were determined from the transient pressures, specified in Tables 3-2 and 3-3. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-2 and 3-3.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The core spray piping adjacent to the safe end was also analyzed because of its proximity to the maximum safe end thermal stress location. For this fatigue analysis, the stress results of the safe end were used with stainless steel material properties and a value of 1.8 was selected for K_t at the weld location, based on the maximum value given in ASME Code, Section III, Table NB-3681(a)-1 [16].

The results of the fatigue analysis are presented in Tables 4-4, 4-5 and 4-6 for the safe end, blend radius, and piping for 60 years, respectively.

Table 4-1: Core Spray Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	413	413	413	413	120
				100	1100	13233	13222	413	413	13646	13635	120
				100	50	602	601	413	413	1014	1014	120
3	0	661	759	100	0	0	0	413	413	1074	1172	300
	17164	9240	10700	549	1010	12150	12140	6592	6592	27982	29432	300
11	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
	3	8802	10236	526	1190	14316	14304	6276	6276	29393	30815	10
	13	8802	10236	526	1135	13654	13643	6276	6276	28732	30154	10
	164	11645	11598	408	1135	13654	13643	4657	4657	29956	29898	10
	672	4808	5791	344	1135	13654	13643	3775	3775	22237	23209	10
	2374	11140	10841	361	912	10971	10962	4005	4005	26116	25808	10
	2955	4722	5577	325	916	11019	11010	3509	3509	19250	20096	10
	7054	9518	10162	441	959	11537	11527	5100	5100	26155	26789	10
	7930	4491	5276	309	637	7663	7657	3287	3287	15441	16219	10
	16709	9960	11116	526	1010	12150	12140	6276	6276	28386	29532	10
	17699	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
14	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	1
	152	9499	10570	497	855	10286	10277	5880	5880	25664	26727	1
	12580	91	95	70	50	602	601	0	0	693	696	1
21-23	0	9242	0	549	1010	12150	12140	6592	6592	27984	18732	300
	17224	664	0	100	50	602	601	413	413	1678	1014	300
24				100	50	602	601	413	413	1014	1014	1
				100	1563	18803	18787	413	413	19216	19200	1
				100	50	602	601	413	413	1014	1014	1
30	0	9280	10800	549	1010	12150	12140	6592	6592	28022	29532	1
	13	85600	44694	162	250	3002	3000	1260	1260	89862	48953	1
	1011	12	10	70	0	0	0	0	0	12	10	1

NOTES:
- Column 1: Transient number identification.
- Column 2: Time during transient where a maximum or minimum stress intensity occurs from P-V.OUT output file.
- Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
- Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
- Column 5: Temperature per total stress intensity.
- Column 6: Pressure per Table 3-4.
- Column 7: Total pressure stress intensity from the quantity (Column 6 x 12,030)/1000.
- Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 12,020)/1000.
- Column 9: Calculated using the total external stress from Table 3-4 as 6949.94 psi*(Column 5-70°F)/(575°F -70°F).
- Column 10: Same as Column 9, but for M+B stress.
- Column 11: Sum of total stresses (Columns 3, 7, and 9).
- Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
- Column 13: Number of cycles for the transient (60 years).

Table 4-2: Core Spray Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	19	19	19	19	120
				100	1100	39446	38467	19	19	39465	38486	120
				100	50	1793	1749	19	19	1812	1768	120
3	0	23700	12600	100	0	0	0	19	19	23719	12619	300
	24164	2100	3180	549	1010	36219	35320	306	306	38625	38806	300
11	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
	3	3209	3644	526	1190	42673	41614	291	291	46174	45550	10
	526	10458	5374	330	1135	40701	39691	166	166	51325	45231	10
	2222	5488	1664	490	1122	40235	39236	268	268	45991	41168	10
	2860	11776	7444	321	911	32668	31858	160	160	44605	39462	10
	6903	5435	3621	495	1124	40307	39306	272	272	46013	43199	10
	8012	12577	6791	390	627	22484	21926	204	204	35265	28921	10
	16640	2772	4370	542	1010	36219	35320	301	301	39292	39991	10
	16991	3389	4115	526	1010	36219	35320	291	291	39899	39726	10
	24699	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
14	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	1
	19580	25122	13197	70	50	1793	1749	0	0	26915	14946	1
21-23	0	2103	3161	549	1010	36219	35320	306	306	38628	38787	300
	24224	23680	12568	100	50	1793	1749	19	19	25492	14336	300
24				100	50	1793	1749	19	19	1812	1768	1
				100	1563	56049	54658	19	19	56068	54677	1
				100	50	1793	1749	19	19	1812	1768	1
30	0	2040	2950	549	1010	36219	35320	306	306	38565	38576	1
	8011	25700	14900	70	0	0	0	0	0	25700	14900	1

NOTES:
- Column 1: Transient number identification.
- Column 2: Time during transient where a maximum or minimum stress intensity occurs from P-V.OUT output file.
- Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
- Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
- Column 5: Temperature per total stress intensity.
- Column 6: Pressure per Table 3-5.
- Column 7: Total pressure stress intensity from the quantity (Column 6 x 35,860)/1000.
- Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 34,970)/1000.
- Column 9: Calculated using the total external stress from Table 3-5 as 322.52 psi*(Column 5-70°F)/(575°F -70°F).
- Column 10: Same as Column 9, but for M+B stress.
- Column 11: Sum of total stresses (Columns 3, 7, and 9).
- Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
- Column 13: Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Core Spray Nozzle Fatigue Analysis

	Blend Radius (SA508 Class II)	Safe End (N06600)	Piping (Stainless Steel)
Parameters *m* and *n* for Computing K_e	2.0 & 0.2 (low alloy steel)	1.7 & 0.3	1.7 & 0.3
Design Stress Intensity Values, S_m	26,700 psi @ 600°F	23,300 psi @ 600°F	17,000 psi @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi	28.3×10^6 psi	28.3×10^6 psi
Elastic Modulus Used in Finite Element Model (300°F)	26.7×10^6 psi	29.8×10^6 psi	27.0×10^6 psi
The Geometric Stress Concentration Factor K_t	1.0	4.0 See Note 1	1.8

Note: 1. Conservative bounding value per ASME Code, Subsection NB-3600 to conservatively cover adjacent thread and weld regions.

Table 4-4: Fatigue Results for Core Spray Nozzle Safe End

```
      LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
             m = 1.7
             n =  .3
            Sm = 23300. psi
        Ecurve = 2.830E+07 psi
     Eanalysis = 2.980E+07 psi
            Kt = 4.00
```

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	112423.	1.000E+00	1.213E+03	.0008
29956.	413.	29543.	29485.	1.000	56029.	1.000E+01	1.910E+04	.0005
29393.	413.	28980.	30402.	1.000	57068.	1.000E+01	1.746E+04	.0006
28732.	413.	28319.	29741.	1.000	55813.	1.000E+01	1.946E+04	.0005
28386.	413.	27973.	29119.	1.000	54762.	1.000E+01	2.140E+04	.0005
28022.	413.	27609.	29119.	1.000	54590.	1.000E+00	2.174E+04	.0000
27984.	413.	27571.	18319.	1.000	39187.	7.900E+01	1.244E+05	.0006
27984.	693.	27291.	18036.	1.000	38651.	1.000E+00	1.341E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.200E+02	1.460E+05	.0008
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1074.	26910.	17560.	1.000	37792.	9.800E+01	1.514E+05	.0006
27982.	1074.	26908.	28260.	1.000	53033.	2.020E+02	2.517E+04	.0080
27982.	1678.	26304.	28418.	1.000	52971.	9.800E+01	2.532E+04	.0039
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+00	2.919E+04	.0000
26155.	1678.	24477.	25775.	1.000	48339.	1.000E+01	4.021E+04	.0002
26116.	1678.	24438.	24794.	1.000	46923.	1.000E+01	4.673E+04	.0002
25664.	1678.	23986.	25713.	1.000	48017.	1.000E+00	4.159E+04	.0000
22237.	1678.	20559.	22195.	1.000	41379.	1.000E+01	9.257E+04	.0001
19250.	1678.	17572.	19082.	1.000	35526.	1.000E+01	2.135E+05	.0000
19216.	1678.	17538.	18186.	1.000	34234.	1.000E+00	2.691E+05	.0000
15441.	1678.	13763.	15205.	1.000	28195.	1.000E+01	1.001E+06	.0000
13646.	1678.	11968.	12621.	1.000	23661.	1.200E+02	1.772E+06	.0001

TOTAL USAGE FACTOR = .0184

Table 4-5: Fatigue Results for the Core Spray Nozzle Blend Radius

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 2.0
n = .2
Sm = 26700. psi
Ecurve = 3.000E+07 psi
Eanalysis = 2.670E+07 psi
Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
56068.	19.	56049.	54658.	1.000	31488.	1.000E+00	1.896E+04	.0001
51325.	19.	51306.	45212.	1.000	28824.	1.000E+01	2.501E+04	.0004
46174.	19.	46155.	45531.	1.000	25930.	1.000E+01	3.460E+04	.0003
46013.	19.	45994.	43180.	1.000	25839.	1.000E+01	3.498E+04	.0003
45991.	19.	45972.	41149.	1.000	25827.	1.000E+01	3.503E+04	.0003
44605.	19.	44586.	39443.	1.000	25048.	1.000E+01	3.848E+04	.0003
39899.	19.	39880.	39707.	1.000	22404.	1.000E+01	5.695E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+00	5.824E+04	.0000
39465.	19.	39446.	38467.	1.000	22161.	3.800E+01	6.012E+04	.0006
39465.	1812.	37653.	36718.	1.000	21153.	8.200E+01	7.572E+04	.0011
39292.	1812.	37480.	38223.	1.000	21056.	1.000E+01	7.747E+04	.0001
38628.	1812.	36816.	37019.	1.000	20683.	2.800E+01	8.466E+04	.0003
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	23719.	14909.	26168.	1.000	8376.	2.700E+02	5.366E+07	.0000
38625.	23719.	14906.	26187.	1.000	8374.	3.000E+01	5.375E+07	.0000
38625.	25492.	13133.	24470.	1.000	7378.	2.700E+02	3.042E+08	.0000
38565.	25492.	13073.	24240.	1.000	7344.	1.000E+00	3.374E+08	.0000
35265.	25492.	9773.	14585.	1.000	5490.	1.000E+01	1.000E+20	.0000
26915.	25492.	1423.	610.	1.000	799.	1.000E+00	1.000E+20	.0000
25700.	25492.	208.	564.	1.000	117.	1.000E+00	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0043

Table 4-6: Fatigue Results for the Core Spray Stainless Steel Piping

```
      LOCATION = LOCATION NO. 1 -- SS Piping
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
             m = 1.7
             n =  .3
            Sm = 17000. psi
        Ecurve = 2.830E+07 psi
     Eanalysis = 2.700E+07 psi
            Kt = 1.80
```

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	67629.	1.000E+00	8.006E+03	.0001
29956.	413.	29543.	29485.	1.000	27845.	1.000E+01	1.042E+06	.0000
29393.	413.	28980.	30402.	1.000	27934.	1.000E+01	1.031E+06	.0000
28732.	413.	28319.	29741.	1.000	27310.	1.000E+01	1.110E+06	.0000
28386.	413.	27973.	29119.	1.000	26868.	1.000E+01	1.171E+06	.0000
28022.	413.	27609.	29119.	1.000	26678.	1.000E+00	1.198E+06	.0000
27984.	413.	27571.	18319.	1.000	22130.	7.900E+01	2.272E+06	.0000
27984.	693.	27291.	18036.	1.000	21864.	1.000E+00	2.392E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.200E+02	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1074.	26910.	17560.	1.000	21465.	9.800E+01	2.588E+06	.0000
27982.	1074.	26908.	28260.	1.000	25950.	2.020E+02	1.311E+06	.0002
27982.	1678.	26304.	28418.	1.000	25700.	9.800E+01	1.354E+06	.0001
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+00	1.485E+06	.0000
26155.	1678.	24477.	25775.	1.000	23634.	1.000E+01	1.779E+06	.0000
26116.	1678.	24438.	24794.	1.000	23202.	1.000E+01	1.889E+06	.0000
25664.	1678.	23986.	25713.	1.000	23351.	1.000E+00	1.850E+06	.0000
22237.	1678.	20559.	22195.	1.000	20080.	1.000E+01	3.442E+06	.0000
19250.	1678.	17572.	19082.	1.000	17209.	1.000E+01	7.481E+06	.0000
19216.	1678.	17538.	18186.	1.000	16816.	1.000E+00	8.600E+06	.0000
15441.	1678.	13763.	15205.	1.000	13588.	1.000E+01	1.000E+20	.0000
13646.	1678.	11968.	12621.	1.000	11564.	1.200E+02	1.000E+20	.0000
							TOTAL USAGE FACTOR =	.0005

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [17] for various regions of the VY RPV and attached piping. Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the core spray nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV upper region chemistry was assumed for both the core spray nozzle safe end and blend radius locations, since both locations experience reactor conditions for all times except during core spray injections, (which are rare occurrences).

For the safe end location, the environmental fatigue factors for pre-HWC and post-HWC are both 1.49 from Reference [18]. This results in an EAF adjusted CUF as follows:

60-Year CUF, U_{60} = 0.0184 (from Table 4-4)
Overall EAF multiplier, F_{en} = 1.49
60-Year EAF CUF, $U_{60\text{-}evn}$ = 0.0184 x 1.49 = 0.0274

The EAF CUF value of 0.0274 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

For the stainless steel piping, the environmental fatigue factors for pre-HWC and post-HWC are both 8.36 from Table 4 of Reference [17] for the RPV upper region. This results in an EAF adjusted CUF as follows:

60-Year CUF, U_{60} = 0.0005 (from Table 4-6)
Overall EAF multiplier, F_{en} = 8.36
60-Year EAF CUF, $U_{60\text{-}evn}$ = 0.0005 x 8.36 = 0.0042

The EAF CUF value of 0.0042 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material, since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [16]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [1]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82, respectively, from Table 4 of Reference [17] for the RPV upper region. This results in an EAF adjusted CUF as follows:

60-Year CUF, U_{60} = 0.0043 (from Table 4-5)
Overall EAF multiplier, F_{en} = (11.14 x 53% + 8.82 x 47%) = 10.05
60-Year EAF CUF, $U_{60\text{-}evn}$ = 0.0043 x 10.05 = 0.0432

The EAF CUF value of 0.0432 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY core spray nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the core spray nozzle, safe end, and piping are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY core spray nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [16]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.0184, the CUF for the limiting blend radius location is 0.0043, and the CUF for the stainless steel piping is 0.0005. All of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY core spray nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.0274, the EAF CUF for the limiting blend radius location is 0.0432, and the EAF CUF for the stainless steel piping is 0.0042. All of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

7.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.

2. Structural Integrity Associates Report No. SIR-86-037, Revision 0, "Evaluation of Modifications to Vermont Yankee Core Spray Nozzle and Safe End," January 1987, SI Project No. YAEC-08.

3. GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.

4. GE Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," August 29, 2003, SI File No. VY-05Q-236.

5. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.

6. ANSYS, Release 8.1A1 (w/Service Pack 1), ANSYS, Inc., June 2004.

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8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Materials, Part D, "Properties (Customary)," 1998 Edition, 2000 Addenda.

9. Structural Integrity Associates Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions."

10. Structural Integrity Associates Calculation No. VY-16Q-310, Revision 0, "Fatigue Analysis of Core Spray Nozzle."

11. Reactor Thermal Cycles, Attachment 1 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.

12. Nozzle Thermal Cycles (Drain, Core Spray & Head Spray), Attachment 1 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.

13. Reactor Thermal Cycles for 60 Years of Operation, Attachment 1 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.

14. Vermont Yankee Drawing 5920-00024, Rev. 11, (GE Drawing No. 919D294, Sheet No. 7), "Reactor Vessel," SI File No. VY-05Q-241.

15. Structural Integrity Associates Calculation No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."

16. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, 1998 Edition, 2000 Addenda.

17. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."

18. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.

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EXHIBIT O

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Revision No.: 0
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July 2007

Summary Report of Plant-Specific Environmental Fatigue Analyses for the Vermont Yankee Nuclear Power Station

NOTE

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1.0	1-1	0	7/27/07	Initial issue.
2.0	2-1 – 2-2			
3.0	3-1 – 3-18			
4.0	4-1			
5.0	5-1 – 5-2			

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1.0 INTRODUCTION

This report provides the results of plant-specific environmental fatigue calculations for the Vermont Yankee Nuclear Power Station (VYNPS). These calculations are performed to satisfy Nuclear Regulatory Commission (NRC) requirements for Entergy Nuclear Vermont Yankee's (ENVY's) License Renewal Application for VYNPS, submitted to the NRC in 2006.

Generic Safety Issue (GSI) 166 [1], later renumbered as GSI-190 [2], was identified by the NRC staff because of concerns about the effects of reactor water environments on fatigue life during the period of extended operation [3]. GSI-190 was closed in December 1999, based on a memorandum from NRC-RES to NRC-NRR [4]. Timing of issue closure required the first two license renewal applicants – Baltimore Gas & Electric Company for the Calvert Cliffs Nuclear Power Plant and Duke Energy for the Oconee Nuclear Station – to address GSI-190 in their applications prior to issue closure. Each of the applicants developed responses to the NRC staff without the benefit of information from GSI-190 closure. Subsequent license renewal applicants have had the benefit of this information that could be used to guide the resolution of the fatigue design basis and time limited aging analyses (TLAA) issues.

This report addresses VYNPS reactor water environmental effects on the fatigue life of selected fatigue-sensitive reactor coolant system (RCS) components, in accordance with the resolution of GSI-190, as required by Chapter X, "Time Limited Aging Analyses Evaluation of Aging Management Programs Under 10CFR54.21(c)(1)(iii), Section X.M1 "Metal Fatigue of Reactor Coolant Pressure Boundary", of the Generic Aging Lessons Learned (GALL) Report [5]. Consistent with the requirements of the GALL report, the method chosen for this environmentally-assisted fatigue (EAF) evaluation is based on evaluation of the locations identified in NUREG/CR-6260 [6] and the NRC-accepted EAF relationships generated from laboratory data, as documented in References [7] and [8].

2.0 BACKGROUND

As a part of the NRC's Fatigue Action Plan [3], incorporation of environmental fatigue effects originally involved a reduced set of fatigue design curves, such as those proposed by Argonne National Laboratory (ANL) in NUREG/CR-5999 [9]. As a part of the effort to close GSI-166 (later GSI-190) for operating nuclear power plants during the current 40-year licensing term, Idaho National Engineering Laboratory (INEL) evaluated fatigue-sensitive component locations at plants designed by all four U. S. nuclear steam supply system (NSSS) vendors. The ANL fatigue curves were used by INEL to recalculate the cumulative usage factors (CUFs) for fatigue-sensitive component locations in early and late vintage Combustion Engineering (CE) pressurized water reactors (PWRs), early and late vintage Westinghouse PWRs, early and late vintage General Electric (GE) boiling water reactors (BWRs), and Babcock & Wilcox Company (B&W) PWRs. The results of the INEL calculations were published in NUREG/CR-6260 [6]. The INEL calculations took advantage of conservatisms present in governing ASME Code fatigue calculations, including the numbers of actual plant transients relative to the numbers of design-basis transients, but did not recalculate stress ranges based on actual plant transient profiles. The BWR calculations, especially the early-vintage GE BWR calculations, are directly relevant to VYNPS.

The fatigue-sensitive component locations chosen for the older-vintage GE BWR plant were: (1) the reactor vessel shell and lower head, (2) the reactor vessel feedwater nozzle, (3) the reactor recirculation piping (including the reactor inlet and outlet nozzles), (4) the core spray line reactor vessel nozzle and associated Class 1 piping, (5) the residual heat removal (RHR) return line Class 1 piping, and (6) the feedwater line Class 1 piping. For the recirculation, RHR, and feedwater piping locations, INEL performed representative design-basis fatigue calculations. This is because no CUF calculations had originally been performed since the piping systems for the selected BWR plant were initially designed and analyzed in accordance with the criteria of USAS B31.1-1967 [10].

The six RCS component locations described above are evaluated for EAF effects for VYNPS in this report through separate plant-specific analyses of nine VY component locations (with report section numbers indicated): the reactor pressure vessel (RPV) shell and lower head (3.1); the RPV shell at the shroud support junction (3.1); the feedwater nozzle (3.2); the recirculation / residual heat removal Class 1 piping (3.3.1 and 3.5); the recirculation inlet nozzle forging (3.3.2); the recirculation inlet nozzle safe end (3.3.2); the recirculation outlet nozzle forging (3.3.3); the core spray nozzle, safe end, and Class 1 piping (3.4); and the feedwater Class 1 piping (3.6).

The calculations reported in NUREG/CR-6260 were based on the interim reduced fatigue design curves given in NUREG/CR-5999 [9]. Such an approach penalizes the component location fatigue analysis unnecessarily, because research has shown that a combination of environmental conditions is required before reactor water environmental effects become pronounced. The strain rate must be sufficiently low and the strain range must be sufficiently high to cause continuing rupture of the passivation layer that protects the exposed surface area. Temperature, dissolved oxygen content, metal sulfur content, and water flow rate are additional variables to be considered. In order to take these parameters into consideration, EPRI and GE jointly developed a method, called the F_{en} approach [11], which permits reactor water environmental effects to be applied selectively, as justified by parameter combinations.

In 1999, the NRC staff raised a number of issues relative to the use of the EPRI/GE methodology in various industry applications. Those issues, coupled with more recent laboratory fatigue data in simulated LWR reactor water environments generated by ANL for carbon and low-alloy steels and stainless steels, resulted in a revised F_{en} methodology, as published in NUREG/CR-6583 [7] for carbon and low alloy steels, and NUREG/CR-5704 [8] for stainless steels. The methodology documented in these reports was used to evaluate environmental effects for VYNPS components, as described in Section 3.0 of this report.

3.0 ENVIRONMENTAL FATIGUE CALCULATIONS

Section 2.0 identifies the locations evaluated in NUREG/CR-6260 for the older vintage GE plant, which corresponds to VYNPS. NUREG/CR-6260 provided an assessment of these six selected component locations with respect to environmental fatigue using the older reduced environmental fatigue curves. Potential reactor water environmental effects are evaluated using the updated F_{en} methodology on a plant-specific basis in this subsection, in order to address the associated effects on fatigue as required by the GALL Report [5].

For each of the components identified in Section 2.0, environmental fatigue calculations were performed. The details of these calculations are documented in the Reference [12, 17, 18, 21, 22 and 24] calculations. The calculations were carried out using the appropriate methodology contained in NUREG/CR-6583 for carbon/low alloy steel material, and in NUREG/CR-5704 for stainless steel material. This methodology is as follows:

For Carbon Steel [7]: $F_{en} = \exp(0.585 - 0.00124T' - 0.101\, S^*\, T^*\, O^*\, \dot{\varepsilon}^*)$
$= \exp(0.554 - 0.101\, S^*\, T^*\, O^*\, \dot{\varepsilon}^*)$

For Low Alloy Steel [7]: $F_{en} = \exp(0.929 - 0.00124T' - 0.101\, S^*\, T^*\, O^*\, \dot{\varepsilon}^*)$
$= \exp(0.898 - 0.101\, S^*\, T^*\, O^*\, \dot{\varepsilon}^*)$

Note that the above expressions have been corrected as summarized in Reference [23].

where:

F_{en}	=	fatigue life correction factor
T'	=	25°C (NUREG/CR-6583, Section 6, F_{en} relative to air)
S^*	=	S for 0 < sulfur content, $S \leq 0.015$ wt. %
	=	0.015 for $S > 0.015$ wt. %
T^*	=	0 for $T < 150$°C
	=	$(T - 150)$ for $150 \leq T \leq 350$°C
T	=	fluid service temperature (°C)
O^*	=	0 for dissolved oxygen, DO < 0.05 parts per million (ppm)
	=	$\ln(DO/0.04)$ for 0.05 ppm $\leq DO \leq$ 0.5 ppm
	=	$\ln(12.5)$ for DO > 0.5 ppm

$\dot{\varepsilon}^*$ = 0 for strain rate, $\dot{\varepsilon} > 1\%/sec$

= $\ln(\dot{\varepsilon})$ for $0.001 \leq \dot{\varepsilon} \leq 1\%/sec$

= $\ln(0.001)$ for $\dot{\varepsilon} < 0.001\%/sec$

For Types 304 and 316 Stainless Steel [8]: $F_{en} = \exp(0.935 - T^* \dot{\varepsilon}^* O^*)$

where: F_{en} = fatigue life correction factor

T = fluid service temperature (°C)

T^* = 0 for $T < 200°C$

= 1 for $T \geq 200°C$

$\dot{\varepsilon}^*$ = 0 for strain rate, $\dot{\varepsilon} > 0.4\%/sec$

= $\ln(\dot{\varepsilon}/0.4)$ for $0.0004 \leq \dot{\varepsilon} \leq 0.4\%/sec$

= $\ln(0.0004/0.4)$ for $\dot{\varepsilon} < 0.0004\%/sec$

O^* = 0.260 for dissolved oxygen, DO < 0.05 parts per million (ppm)

= 0.172 for DO ≥ 0.05 ppm

Bounding F_{en} values are determined or, where necessary, computed for each load pair in a detailed fatigue calculation. The environmental fatigue is then determined as $U_{env} = (U)(F_{en})$, where U is the original fatigue usage, and U_{env} is the EAF usage factor.

REDACTED

Since implementation of HWC in 2003, VYNPS's availability has exceeded 98.5% and the objective for future HWC system availability is a minimum of 99% [12]. With these considerations, the overall availability for HWC since implementation at VYNPS until the end of the 60-year operating period was estimated at 98.5%.

This Page Contains Reference to Vendor Proprietary Information
(such information is marked with a "bar" in the right-hand margin)

Some nozzles, (e.g., recirculation outlet nozzle) have three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified, locations in both the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the finite-element model (FEM) analysis occur in the stainless steel safe end. Stainless steel F_{en} multipliers at VYNPS are significantly higher than Ni-Cr-Fe multipliers (F_{en} values are 2.55 or higher for stainless steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [11]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.
- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel F_{en} multipliers at VYNPS are higher than Ni-Cr-Fe multipliers (F_{en} values are 2.45 or higher for low alloy steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [11]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

The number of cycles for forty years was adjusted based on the number of cycles actually experienced by the plant, projected out to 60 years of operation [14]. In addition, VYNPS has implemented extended power uprate (EPU). These effects have been incorporated into the evaluations documented in this report. With the use of this information, the CUF values documented in this report are applicable for 60 years of operation.

The environmental fatigue calculations are shown in Tables 3-1 through 3-9 and summarized in Table 3-10. Component-specific details are provided in the subsections that follow.

3.1 Reactor Vessel Shell and Lower Head

The environmental fatigue calculations for the reactor vessel shell and lower head location are shown in Table 3-1. The limiting CUF value reported in the VY LRA for the RPV shell/bottom

head location corresponds to a point located on the outside surface of the RPV bottom head at the junction with the support skirt. Therefore, this location is not exposed to the reactor coolant, and EAF effects do not apply. Based on this, evaluation of the limiting location along the inside surface of the RPV bottom head was performed.

The calculations shown in Table 3-1 are for the RPV lower head at the area with the highest alternating stress, which represents the limiting RPV bottom head location [12]. Reference [15] is the governing stress report for this low alloy steel location. The design fatigue calculation for the limiting RPV lower head location is reproduced in Table 3-1. The effects of EPU as well as conservative cycle counts for 60 years of plant operation are incorporated in this table. The final results in Table 3-1 show an EAF adjusted CUF of 0.0809 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The calculations shown in Table 3-2 are for the RPV shell at the RPV shell junction to the shroud support plate, which represents the limiting RPV shell location exposed to the reactor coolant [12]. Reference [16] is the governing stress report for this low alloy steel location. The design fatigue calculation for the limiting RPV shell location is reproduced in Table 3-2, which considers the effects of EPU and conservative cycle counts were used for 60 years of plant operation. The final results in Table 3-2 show an EAF adjusted CUF of 0.7364 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.2 Reactor Vessel Feedwater Nozzle

The environmental fatigue calculations for the reactor vessel feedwater nozzle location are summarized in Table 3-3. The calculations summarized in Table 3-3 show both the blend radius, which represents the limiting feedwater nozzle location, and the safe end. Reference [17] contains the governing fatigue calculation for this location. Upper RPV region chemistry was assumed for the feedwater nozzle blend radius location, since this location is exposed to the reactor water chemistry in this region, whereas feedwater line chemistry was assumed for the safe end location.

The governing fatigue calculation for the limiting feedwater nozzle locations includes the effects of EPU and cycle counts for 60 years of operation obtained from Attachment 1 of Reference [14]. The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years. Although the carbon steel safe end has a higher CUF prior to considering environmental effects, the environmental multiplier from Table 3-3 results in a higher CUF at the low alloy steel blend radius. For the safe end location, the EAF adjusted CUF is 0.2560 for 60 years. For the blend radius location, EAF adjusted CUF is 0.6392 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.3 Reactor Recirculation Piping (Including the Reactor Inlet and Outlet Nozzles)

Three locations were identified for the reactor recirculation piping in NUREG/CR-6260: the reactor vessel nozzle (includes both the inlet and outlet nozzles), and the recirculation piping. The evaluations for each of these components are described in the following subsections.

3.3.1 Reactor Recirculation Piping

Two locations were identified for the reactor recirculation/RHR piping in NUREG/CR-6260 (both stainless steel): the RHR return tee connection to the recirculation piping, and a tapered transition on the RHR line just upstream of the RHR return tee. Reference [18] contains the governing fatigue calculations for these locations. These analyses determined the limiting location to be at the RHR return tee.

The environmental fatigue calculations for the limiting recirculation/RHR piping location is summarized in Table 3-4, which includes the effects of EPU and cycle counts for 60 years of plant operation.

A review of the shutdown cooling mode of operation since the time of recirculation piping replacement in 1986 was performed by VYNPS, and the number of cycles per loop was conservatively estimated to be 150 through Year 60 [14]. Based on this, the cycle counts for the

recirculation piping were reduced by a factor of 150/300 (50%) for all transients with the exception of transients that have fewer than 10 transient cycles. To ensure this cycle reduction adequately considered the potential impact on the RHR piping, which has not been replaced, the full number of transient cycles listed in Attachment 1 of Reference [14] was initially applied to the PIPESTRESS model and the highest CUF for the RHR piping was lower than the value obtained for the recirculation piping with reduced cycles.

Due to replacement of the recirculation piping, HWC conditions exist for 39% of the time, and NWC conditions exist for 61% of the time. This is based on 17.5 years of operation with NWC between March 1986 when the piping was replaced and November 2003 when HWC was implemented, and 46 years of operation from March 1986 to the end of the period of extended operation in March 2032. Using the bounding EAF multipliers (8.36 for HWC and 15.35 for NWC) [12], the overall multiplier is 12.62. Applying this to the 60-Year CUF of 0.0590 results in a total environmentally assisted CUF of 0.7446.

3.3.2 Reactor Recirculation Inlet Nozzle

References [15, 19 and 20] are the applicable stress reports for this location. An evaluation was performed for both the inlet nozzle forging (low alloy steel) and the safe end (stainless steel).

The environmental fatigue calculations for the recirculation inlet nozzle forging location are shown in Table 3-5. The governing fatigue calculation for the recirculation inlet nozzle location is reproduced in Table 3-5 [12], which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results show an EAF adjusted CUF of 0.5034 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The environmental fatigue calculations for the recirculation inlet nozzle safe end location are shown in Table 3-6. The governing fatigue calculation for the recirculation inlet nozzle location is reproduced in Table 3-6 [12], which includes the effects of EPU and cycle counts for 60 years

of plant operation from Attachment 1 of Reference [14]. The final results show an EAF adjusted CUF of 0.0199 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.3.3 Reactor Recirculation Outlet Nozzle

The recirculation outlet nozzle was evaluated for environmental fatigue effects. Reference [24] is the fatigue calculation for this location. An evaluation was performed for both the outlet nozzle safe end (stainless steel) and the nozzle inner corner blend radius (low alloy steel). The results for the limiting nozzle forging location are reported here.

The environmental fatigue calculations for the limiting recirculation outlet nozzle forging blend radius location are shown in Table 3-7 [24], which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results in Table 3-7 show an EAF adjusted CUF of 0.0836 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.4 Core Spray Line Reactor Vessel Nozzle and Associated Class 1 Piping

Locations that were evaluated in NUREG/CR-6260 included the reactor vessel nozzle blend radius (low alloy steel), the reactor vessel nozzle safe end (Alloy 600) and the core spray piping (stainless steel).

Reference [21] is the applicable fatigue calculation for these locations, which shows the nozzle limiting location to be the blend radius. The design fatigue calculations for the limiting location at the core spray nozzle, safe end, and piping are summarized in Table 3-8 [21], which include the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The cumulative fatigue usage, prior to considering environmental effects for the blend radius, is 0.0043. Factoring in the environmental multiplier from Table 3-8 [12], the EAF adjusted CUF is 0.0432 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.5 RHR Return Line Class 1 Piping

The environmental fatigue calculations for the RHR return line Class 1 piping are covered by the calculations in Subsection 3.3.1 above.

3.6 Feedwater Line Class 1 Piping

The environmental fatigue calculation for the limiting feedwater Class 1 piping location (carbon steel) is summarized in Table 3-9. The calculations shown in Table 3-9 are for the limiting feedwater Class 1 piping location. Per Reference [22], the limiting total fatigue usage for the analyzed feedwater/high pressure coolant injection (HPCI) piping system occurs on the riser to the RPV feedwater nozzle N4B. The limiting fatigue usage value for the feedwater Class 1 piping location is 0.1661, which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results in Table 3-9 show the EAF adjusted CUF of 0.2890 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.7 Summary of Results

The results of the calculations contained in Tables 3-1 through 3-9 are summarized in Table 3-10.

It is noteworthy that the CUF results presented in this section include uniformly applied environmental effects without consideration of threshold criteria that might indicate an absence of conditions that would lead to environmental fatigue effects. Furthermore, conservative values were applied for temperature, strain rate and metal sulfur content in calculating environmental multipliers. Therefore, the environmental adjustments to the CUF results are considered to be conservative.

Table 3-1. Environmental Fatigue Evaluation for the Reactor Vessel Shell

Component: RPV Shell/Bottom Head
NUREG/CR-6260 CUF: 0.032 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-102
Stress Report CUF: 0.0057 *(for Point 14, see below)*
Material: Low Alloy Steel *(Material = A-533 Gr. B)*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.149	*Conservatively used minimum E of 26.1 from Section S2 Appendix of RPV Stress Report.*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1*
K_t =	1.000	*stress concentration factor*
m =	2.0	*NB-3228.5 of ASME Code, Section III*
n =	0.2	*NB-3228.5 of ASME Code, Section III*
S_m =	26,700	psi *(ASME Code, Section II, Part D)*

P_L+P_B+Q *(see Note 1)*	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 4)*	N *(see Note 5)*	U
44,526	1.00	25,762	200	35,300	0.0057
				Total, U_{40} =	**0.0057**

Notes:
1. *P_L+P_B+Q is obtained for Point 14 from p. A52 of VYC-378, Rev. 0.*
2. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
3. *$S_{alt} = 0.5 * K_e * K_t * E_{fatigue\ curve}/E_{analysis} *$ Power Uprate $* (P_L+P_B+Q)$.*
4. *n for 40 years is the number of Heatup-Cooldown cycles, per p. B8 of VYC-378, Rev. 0.*
5. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
6. *n for 60 years is the projected number of Heatup-Cooldown cycles.*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 6)*	N *(see Note 4)*	U
44,526	1.00	25,762	300	35,300	0.0085
				Total, U_{60} =	**0.0085**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions = 5.39
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions = 13.17

$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47 =$ 0.0809
Overall Multiplier = $U_{env\text{-}60}/U_{60} =$ 9.51

Table 3-2. Environmental Fatigue Evaluation for the Reactor Vessel Shell at Shroud Support

Component:	RPV Shell at Shroud Support	
NUREG/CR-6260 CUF:	0.032	*(for reference only)*
Reference:	NUREG/CR-6260, p. 5-102	
Stress Report CUF:	0.0549	*(for Point 9, see below)*
Material:	Low Alloy Steel	*(Material = A-533 Gr. B)*

Design Basis CUF Calculation for 40 years:

Hydrotest H_H =	26,240	psi *(p. S3-97 of RPV Stress Report)*
Hydrotest H_r =	-1,250	psi *(p. S3-97 of RPV Stress Report)*
Stress Concentration Factor, K_t =	2.40	*(p. S3-99d of RPV Stress Report)*
Hydrotest K_tH_H =	62,976	psi *(p. S3-97 of RPV Stress Report)*
Improper Startup H_H =	28,060	psi *(p. S3-98 of RPV Stress Report)*
Improper Startup H_r =	-1,025	psi *(p. S3-98 of RPV Stress Report)*
Improper Startup Skin Stress =	156,099	psi *(p. S3-98 of RPV Stress Report)*
Improper Startup K_tH_H + Skin Stress =	223,443	psi *(p. S3-98 of RPV Stress Report)*
Warmup H_H =	-5,707	psi *(p. S3-99a of RPV Stress Report)*
Warmup H_r =	-102	psi *(p. S3-99a of RPV Stress Report)*
Warmup K_tH_H =	-13,696	psi *(p. S3-99a of RPV Stress Report)*
$E_{fatigue\ curve}/E_{analysis}$ =	1.0417	*30.0 / 28.8 per S3-99f of RPV Stress Report and ASME Code fatigue curve*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1*
m =	2.0	*NB-3228.5 of ASME Code, Section III*
n =	0.2	*NB-3228.5 of ASME Code, Section III*
S_m =	26,700	psi *(ASME Code, Section II, Part D)*

P_L+P_B+Q *(see Note 1)*	Events	K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 4)*	N *(see Note 5)*	U
34,690	Improper Startup - Warmup	1.00	124,825	5	332	0.0151
33,095	Hydrotest - Warmup	1.00	40,804	322	8,095	0.0398
					Total, U_{40} =	**0.0549**

Notes:

1. *P_L+P_B+Q is computed for Point 9 based on the $[(H_H - H_r)_{Event\ 1} - (H_H - H_r)_{Event\ 2}]$ stress intensity.*
2. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
3. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis} *$ Power Uprate $* [(K_tH_H - H_r)_{Event\ 1} - (K_tH_H - H_r)_{Event\ 2}]$.*
4. *n for 40 years is the number of cycles as follows per p. S3-99e and S3-99f of the RPV Stress Report:*

Improper Startup =	*5*	*cycles*
Hydrotest =	*2*	*cycles*
Isothermal at 70°F and 1,000 psi =	*120*	*cycles (same as number of Startup events)*
Warmup-Cooldown =	*199*	*cycles*
Warmup-Blowdown =	*1*	*cycle*
TOTAL =	*327*	*cycles*

5. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
6. *n for 60 years is the projected number of cycles as follows:*

Improper Startup =	*1*	*cycles*
Hydrotest =	*1*	*cycles*
Isothermal at 70°F and 1,000 psi =	*300*	*cycles (same as number of Startup events)*
Warmup-Cooldown =	*300*	*cycles*
Warmup-Blowdown =	*1*	*cycle*
TOTAL =	*603*	*cycles*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*		K_e *(see Note 2)*	S_{alt} *(see Note 3)*	n *(see Note 6)*	N *(see Note 4)*	U
34,690	Improper Startup - Warmup	1.00	124,825	1	332	0.0030
33,095	Hydrotest - Warmup	1.00	40,804	602	8,095	0.0744
					Total, U_{60} =	**0.0774**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions =	5.39
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions =	13.17
$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47$ =	**0.7364**
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ =	**9.51**

Table 3-3. Environmental Fatigue Evaluation for the Reactor Vessel Feedwater Nozzle Forging Blend Radius

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\square)$

Assume $S^* = 0.015$ (maximum)
Assume $\square = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	3.90
250	482	6.20
288	550	8.82

Thus, maximum F_{en} = 8.82

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.25
250	482	7.35
288	550	11.14

Thus, maximum F_{en} = 11.14

[$T^* = (T-150)$ for $T > 150°C$]

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\square)$

Assume $S^* = 0.015$ (maximum)
Assume $\square = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

For a BWR with NWC environment (pre-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

[$T^* = (T-150)$ for $T > 150°C$]

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Feedwater Nozzle Forging Blend Radius	Low Alloy Steel	0.0636	10.05	0.6392
2	Feedwater Nozzle Forging Safe End	Carbon Steel	0.1471	1.74	0.2560

Notes:
1. An Fen Multiplier was used for each respective component with the following conditions: + 47% HWC conditions and 53% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-4. Environmental Fatigue Evaluation for the Recirculation/RHR Piping Tee

Stainless Steel:		$F_{en} = \exp(0.935 - T^*\varepsilon^*O^*)$		
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm < 0.050 ppm, so O* = 0.260 Conservatively use T* = 1 for T > 200°C Thus:			**For a BWR with NWC environment (pre-HWC implementation):** DO = 123 ppb = 0.123 ppm > 0.05 ppm, so O* = 0.172 Conservatively use T* = 1 for T > 200°C Thus:	
$\varepsilon^* = 0$ for $\varepsilon > 0.4$%/sec	so F_{en} =	2.55	so F_{en} =	2.55
$\varepsilon^* = \ln(\varepsilon/0.4)$ for $0.0004 <= \varepsilon <= 0.4$%/sec	so F_{en} ranges from	2.55	so F_{en} ranges from	2.55
	to	15.35	to	8.36
$\varepsilon^* = \ln(0.0004/0.4)$ for $\varepsilon < 0.0004$%/sec	so F_{en} =	15.35	so F_{en} =	8.36
	Thus, maximum F_{en} =	15.35	Thus, maximum F_{en} =	8.36

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Recirculation /RHR Piping Return Tee	Stainless Steel	0.0590	12.62	0.7446

Notes: 1. An F_{en} multiplier was used for each respective component with the following conditions:
+ 39% HWC conditions and 61% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-5. Environmental Fatigue Evaluation for the Reactor Recirculation Inlet Nozzle Forging

Component: Recirculation Inlet Nozzle Forging
NUREG/CR-6260 CUF: 0.310 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-105
Stress Report CUF: 0.0433 *(updated for Point 12, see below)*
Material: Low Alloy Steel *(Material = A-508 Cl. II per p. I-S8-4 of CBIN Stress Report Section S8)*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.1278	*= 30.0 / 26.6 (per p. I-S8-24 of CBIN Stress Report Section S8 and ASME Code fatigue curve)*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1*
K_t =	1.660	*stress concentration factor (p. A270 of VYC-378, Rev. 0)*
m =	2.0	*NB-3228.5 of ASME Code, Section III*
n =	0.2	*NB-3228.5 of ASME Code, Section III*
S_m =	26,700	psi *(ASME Code, Section II, Part D)*

P_L+P_B+Q *(see Note 1)*	Skin Stress *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 6)*	U
43,110	15,145	1.00	49,224	200	4,614	0.0433
					Total, U_{40} =	**0.0433**

Notes:
1. *P_L+P_B+Q is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.*
2. *Skin Stress is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.*
3. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
4. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis}$ * Power Uprate * [$(P_L+P_B+Q)\ K_t$ + Skin Stress].*
5. *n for 40 years is the number of Heatup-Cooldown cycles, per p. B28 of VYC-378, Rev. 0.*
6. *N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.*
7. *n for 60 years is the projected number of Heatup-Cooldown cycles.*

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	Skin Stress *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 7)*	U
43,110	15,145	1.00	49,224	300	4,614	0.0650
					Total, U_{60} =	**0.0650**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions = 2.45
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions = 12.43

$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47$ = 0.5034
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ = 7.74

Table 3-6. Environmental Fatigue Evaluation for Reactor Recirculation Inlet Nozzle Safe End

Component: Recirculation Inlet Nozzle Safe End
NUREG/CR-6260 CUF: 0.310 *(for reference only)*
Reference: NUREG/CR-6260, p. 5-105
Stress Report CUF: 0.0017 *(updated for Location 6-I, see below)*
Material: Stainless Steel *(316L per p. 8 of 23A4292, Rev. 4)*

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis}$ =	1.1076	*= 28.3 / 25.55 (per p. 62 of Reference [18] and ASME Code fatigue curve)*
Power Uprate =	1.0067	*=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1*
K_t =	1.280	*stress concentration factor (p. B27 of VYC-378, Rev. 0)*
m =	1.7	*NB-3228.5 of ASME Code, Section III*
n =	0.3	*NB-3228.5 of ASME Code, Section III*
S_m =	16,600	psi *(ASME Code, Section II, Part D)*

P_L+P_B+Q *(see Note 1)*	P+Q+F *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 6)*	U
47,183	36,972	1.00	26,385	2,076	1,242,266	0.0017
					Total, U_{40} =	**0.0017**

Notes:
1. *P_L+P_B+Q is obtained for Surface I (after weld overlay) from p. 117 of Reference [18].*
2. *P+Q+F is obtained for Point 6-I from p. 118 of Reference [18] (BEFORE weld overlay).*
3. *K_e computed in accordance with NB-3228.5 of ASME Code, Section III.*
4. *$S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis} *$ Power Uprate $* [(P+Q+F) K_t]$.*
5. *n for 40 years is the number of cycles as follows per p. B26 of VYC-378, Rev. 0:*

Design Hydrotest =	*130*	
Loss of Feedpumps Composite:		
Startup/Shutdown =	*290*	
SRV Blowdown =	*8*	
Loss of Feedwater Pumps	*30*	*10 events x 3 up/down cycles per event*
SCRAM =	*270*	
Normal +/- Seismic =	*11*	*10 cycles of upset seismic, plus 1 Level C seismic event*
Normal =	*739*	*= Sum of all of above events*
Zeroload =	*598*	*= Startup/Shutdown + SRV Blowdown + Scram + LOFP*
Total number of cycles =	*2,076*	

6. *N obtained from Figure I-9.2 of Appendix I of ASME Code, Section III.*
7. *n for 60 years is the projected number of cycles as follows:*

Design Hydrotest =	*120*	
Loss of Feedpumps Composite:		
Startup/Shutdown =	*300*	
SRV Blowdown =	*1*	
Loss of Feedwater Pumps	*30*	*10 events x 3 up/down cycles per event*
SCRAM =	*289*	*All remaining scrams*
Normal +/- Seismic =	*11*	*Assume the same*
Normal =	*751*	*= Sum of all of above events*
Zeroload =	*620*	*= Startup/Shutdown + SRV Blowdown + Scram + LOFP*
Total number of cycles =	*2,122*	

Revised CUF Calculation for 60 Years:

P_L+P_B+Q *(see Note 1)*	P+Q+F *(see Note 2)*	K_e *(see Note 3)*	S_{alt} *(see Note 4)*	n *(see Note 5)*	N *(see Note 7)*	U
47,183	36,972	1.00	26,385	2,122	1,242,266	0.0017
					Total, U_{60} =	**0.0017**

Environmental CUF Calculation for 60 Years:

Maximum $F_{en\text{-}HWC}$ Multiplier for HWC Conditions =	15.35
Maximum $F_{en\text{-}NWC}$ Multiplier for NWC Conditions =	8.36
$U_{env\text{-}60} = U_{60} \times F_{en\text{-}NWC} \times 0.53 + U_{60} \times F_{en\text{-}HWC} \times 0.47$ =	**0.0199**
Overall Multiplier = $U_{env\text{-}60}/U_{60}$ =	**11.64**

Table 3-7. Environmental Fatigue Evaluation for Recirculation Outlet Nozzle Forging

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 46 ppb = 0.046 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
269.45	517.01	2.45
288	550	2.45

Thus, maximum F_{en} = 2.45

For a BWR with NWC environment (pre-HWC implementation):
DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.42
269.45	517.01	10.00
288	550	12.43

Thus, maximum F_{en} = 12.43

[$T^* = (T-150)$ for $T > 150°C$]

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Recirculation Outlet Nozzle Forging Blend Radius	Low Alloy Steel	0.0108	7.74	0.0836

Notes: 1. An F_{en} multiplier was used for each respective component with the following conditions:
+ 47% HWC conditions and 53% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-8. Environmental Fatigue Evaluation for Core Spray Reactor Vessel Nozzle Forging Blend Radius, Safe End, and Piping

Low Alloy Steel:

$F_{en} = \exp(0.898 - 0.101 S^* T^* O^* \dot{\varepsilon}^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\dot{\varepsilon}^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	3.90
250	482	6.20
288	550	8.82

Thus, maximum F_{en} = 8.82

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$

Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.25
250	482	7.35
288	550	11.14

Thus, maximum F_{en} = 11.14

[$T^* = (T-150)$ for $T > 150°C$]

Stainless Steel:

$F_{en} = \exp(0.935 - T^* \dot{\varepsilon}^* O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 97 ppb = 0.097 ppm > 0.050 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

$\dot{\varepsilon}^* = 0$ for $\dot{\varepsilon} > 0.4$%/sec	so F_{en} =	2.55
$\dot{\varepsilon}^* = \ln(\dot{\varepsilon}/0.4)$ for $0.0004 <= \dot{\varepsilon} <= 0.4$%/sec	so F_{en} ranges from	2.55
	to	8.36
$\dot{\varepsilon}^* = \ln(0.0004/0.4)$ for $\dot{\varepsilon} < 0.0004$%/sec	so F_{en} =	8.36
	Thus, maximum F_{en} =	8.36

For a BWR with NWC environment (pre-HWC implementation):
DO = 114 ppb = 0.114 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200°C$
Thus:

so F_{en} =	2.55
so F_{en} ranges from	2.55
to	8.36
so F_{en} =	8.36
Thus, maximum F_{en} =	8.36

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Core Spray Nozzle Forging Blend Radius	Low Alloy Steel	0.0043	10.05	0.0432
2	Core Spray Nozzle Safe End	Ni-Cr-Fe	0.0184	1.49	0.0274
3	Core Spray Piping	Stainless Steel	0.0005	8.36	0.0042

Notes:
1. An Fen Multiplier was used for each respective component with the following conditions: + 47% HWC conditions and 53% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-9. Environmental Fatigue Evaluation for the Feedwater Line Class 1 Piping

Carbon Steel:

$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\varepsilon^*)$

Assume $S^* = 0.015$ (maximum)
Assume $\varepsilon^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

For a BWR with NWC environment (pre-HWC implementation):
DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum F_{en} = 1.74

[$T^* = (T-150)$ for $T > 150°C$]

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Feedwater Piping Riser to RPV Nozzle N4B	Carbon Steel	0.1661	1.74	0.2890

Notes: 1. An F_{en} multiplier was used for each respective component with the following conditions:
+ 47% HWC conditions and 53% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-10. Summary of Environmental Fatigue Calculations for VYNPS

No.	Component	Material	40-Year CUF (1)	60-Year CUF (2)	Overall Environmental Multiplier (3)	60-Year Environmental CUF
1	RPV Shell/Bottom Head	Low Alloy Steel	0.0057	0.0085	9.51	0.0809
2	RPV Shell at Shroud Support	Low Alloy Steel	0.0549	0.0774	9.51	0.7364
3	Feedwater Nozzle Forging Blend Radius	Low Alloy Steel	(4)	0.0636	10.05	0.6392
4	Recirculation/RHR Class 1 Piping (Return Tee)	Stainless Steel	(4)	0.0590	12.62	0.7446
5	Recirculation Inlet Nozzle Forging	Low Alloy Steel	0.0433	0.0650	7.74	0.5034
6	Recirculation Inlet Nozzle Safe End	Stainless Steel	0.0017	0.0017	11.64	0.0199
7	Recirculation Outlet Nozzle Forging	Low Alloy Steel	(4)	0.0108	7.74	0.0836
8	Core Spray Nozzle Forging Blend Radius (5)	Low Alloy Steel	(4)	0.0043	10.05	0.0432
9	Feedwater Piping Riser to RPV Nozzle N4B	Carbon Steel	(4)	0.1661	1.74	0.2890

Notes:
1. Updated 40-year CUF calculation based on recent ASME Code methodology and design basis cycles.
2. CUF results using updated ASME Code methodology and actual cycles accumulated to-date and projected to 60 years.
3. An F_{en} multiplier was used for each respective component with the following conditions:
   + 47% HWC conditions and 53% NWC conditions (with the exception of Recirculation piping that uses 61% HWC conditions and 39% NWC conditions).
4. 40 year CUF values were not calculated for these locations.
5. Only the highest CUF from Table 3-8 is shown.

4.0 SUMMARY AND CONCLUSIONS

The results of Tables 3-1 through 3-9, as summarized in Table 3-10, demonstrate that the fatigue usage factor, including environmental effects, remains within the allowable value of 1.0 for 60 years of operation for the following component locations:

- ✓ Reactor vessel shell, bottom head and shroud support
- ✓ Reactor vessel feedwater nozzle
- ✓ Reactor recirculation piping (including the reactor inlet and outlet nozzles)
- ✓ Core spray line reactor vessel nozzle and associated Class 1 piping
- ✓ Feedwater line Class 1 piping

Therefore, the environmental fatigue assessment results for all of the NUREG/CR-6260 locations associated with the older vintage BWR plant are acceptable for 60 years of operation for VYNPS.

5.0 REFERENCES

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16. Chicago Bridge & Iron RPV Stress Report, Section S3, Revision 4, "Stress Analysis, Shroud Support, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2/3/70, SI File No. VY-16Q-203.

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18. Structural Integrity Associates Calculation No. VY-16Q-307, Revision 0, "Recirculation Class 1 Piping Fatigue and EAF Analysis."

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20. GE Nuclear Energy Certified Stress Report No. 23A4292, Revision 0, "Reactor Vessel – Recirculation Inlet Safe End Nozzle," January 21, 1985, SI File No. VY-16Q-203.

21. Structural Integrity Associates Calculation No. VY-16Q-310, Revision 0, "Fatigue Analysis of Core Spray Nozzle."

22. Structural Integrity Associates Calculation No. VY-16Q-311, Revision 0, "Feedwater Class 1 Piping Fatigue Analysis."

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24. Structural Integrity Associates Calculation No. VY-16Q-306, Revision 0, "Fatigue Analysis of Recirculation Outlet Nozzle."

This Page Contains Reference to Vendor Proprietary Information

(such information is marked with a "bar" in the right-hand margin)