

July 6, 2007

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513	GRANT, LEAH	LRQ TRAINING	#48
518	DOC CONTROL DESK	NRC	OFFSITE

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	<b>IPEC SITE MANAGEMENT MANUAL</b>	<b>QUALITY RELATED ADMINISTRATIVE PROCEDURE</b>	<b>IP-SMM-AD-103    Revision 0</b>
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**ATTACHMENT 10.1**

**SMM CONTROLLED DOCUMENT TRANSMITTAL FORM**

**SITE MANAGEMENT MANUAL CONTROLLED DOCUMENT TRANSMITTAL FORM - PROCEDURES**

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		<b>CONTROLLED DOCUMENT TRANSMITTAL FORM - PROCEDURES</b>	
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**Distribution of IP3 Technical Specification Amendment 234**

(Approved by NRC August 16, 2007)

Pages are to be inserted into your controlled copy of the IP3 Improved Technical Specifications following the instructions listed below. The **TAB** notation indicates which section the pages are located.

<b>REMOVE PAGE</b>	<b>INSERT PAGE</b>
<b>TAB – Facility Operating License</b>	
FOL Page 3, Amd 233	FOL Page 3, Amd 234
<b>TAB – List of Effective Pages</b>	
Pages 1 through 3, (Amd 233)	Pages 1 through 3, (Amd 234)
<b>TAB – List of Amendments</b>	
Page 14	Page 14
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<b>TAB 3.3 – Instrumentation</b>	
3.3.1-19, Amd 225	3.3.1-19, Amd 234
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<b>TAB 3.7 – Plant Systems</b>	
3.7.11-1, Amd 223	3.7.11-1, Amd 234
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<b>TAB 5.0 – Administrative Controls</b>	
5.0-5, Amd 232	5.0-5, Amd 234
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5.0-24, Amd 224	5.0-24, Amd 234
5.0-30, Amd 206	5.0-30, Amd 234

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; Amdt. 203 11/27/00
- (5) ENO pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Amdt. 203 11/27/00

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 3216 megawatts thermal (100% of rated power).

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 234 are hereby incorporated in the License. ENO shall operate the facility in accordance with the Technical Specifications.

(3) (DELETED) Amdt. 205 2-27-01

(4) (DELETED) Amdt. 205 2-27-01

D. (DELETED) Amdt.46 2-16-83

E. (DELETED) Amdt.37 5-14-81

F. This amended license is also subject to appropriate conditions by the New York State Department of Environmental Conservation in its letter of May 2, 1975, to Consolidated Edison Company of New York, Inc., granting a Section 401 certification under the Federal Water Pollution Control Act Amendments of 1972.

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Indian Point 3 Nuclear Power Plant  
License Amendments Page 14

AMENDMENT	SUBJECT	LETTER DATE
232	Adoption of TSTF-258, TSTF-308, and Related Administrative Control Changes Based on NUREG-1431	12/13/2006
233	Adoption of TSTF-449 - Steam Generator Tube Integrity	02/13/2007
234	Adoption of TSTF-485 and Miscellaneous Editorial Changes	08/16/2007

1.4 Frequency

EXAMPLES  
(continued)

EXAMPLE 1.4-1

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
Perform CHANNEL CHECK.	12 hours

Example 1.4-1 contains the type of SR most often encountered in the Technical Specifications (TS). The Frequency specifies an interval (12 hours) during which the associated Surveillance must be performed at least one time. Performance of the Surveillance initiates the subsequent interval. Although the Frequency is stated as 12 hours, an extension of the time interval to 1.25 times the stated Frequency is allowed by SR 3.0.2 for operational flexibility.

The measurement of this interval continues at all times, even when the SR is not required to be met per SR 3.0.1 (such as when the equipment is inoperable, a variable is outside specified limits, or the unit is outside the Applicability of the LCO). If the interval specified by SR 3.0.2 is exceeded while the unit is in a MODE or other specified condition in the Applicability of the LCO, and the performance of the Surveillance is not otherwise modified (refer to Example 1.4-3), then SR 3.0.3 becomes applicable.

If the interval as specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, then SR 3.0.4 becomes applicable. The Surveillance must be performed within the Frequency requirements of SR 3.0.2, as modified by SR 3.0.3, prior to entry into the MODE or other specified condition or the LCO is considered not met (in accordance with SR 3.0.1) and LCO 3.0.4 becomes applicable.

(continued)

Table 3.3.1-1 (page 7 of 8)  
Reactor Protection System Instrumentation

Note 1: Overtemperature  $\Delta T$

The Overtemperature  $\Delta T$  Function Allowable Value shall not exceed the following Trip Setpoint by more than 2.8% of  $\Delta T$  span:

$$\Delta T \leq \Delta T_0 \left\{ K_1 - K_2 \frac{(1 + \tau_1 s)}{(1 + \tau_2 s)} [T - T'] + K_3 (P - P') - f_1(\Delta I) \right\}$$

- Where:  $\Delta T$  is measured RCS  $\Delta T$ , °F.  
 $\Delta T_0$  is the indicated  $\Delta T$  at RTP, °F.  
 $s$  is the Laplace transform operator,  $\text{sec}^{-1}$ .  
 $T$  is the measured RCS average temperature, °F.  
 $T'$  is the nominal  $T_{\text{avg}}$  at RTP,  $\leq [ * ]^\circ\text{F}$ .  
 $P$  is the measured pressurizer pressure, psig  
 $P'$  is the nominal RCS operating pressure,  $\geq [ * ]$  psig  
 $K_1 \leq [ * ]$        $K_2 \geq [ * ]/^\circ\text{F}$        $K_3 \geq [ * ]/\text{psig}$   
 $\tau_1 \geq [ * ] \text{ sec}$        $\tau_2 \leq [ * ] \text{ sec}$

$$f_1(\Delta I) = \begin{cases} [ * ] \{ [ * ] - (q_t - q_b) \} & \text{when } q_t - q_b \leq [ * ]\% \text{ RTP} \\ 0\% \text{ of RTP} & \text{when } [ * ]\% \text{ RTP} < q_t - q_b \leq [ * ]\% \text{ RTP} \\ [ * ] \{ (q_t - q_b) - [ * ] \} & \text{when } q_t - q_b > [ * ]\% \text{ RTP} \end{cases}$$

Where  $q_t$  and  $q_b$  are percent RTP in the upper and lower halves of the core, respectively, and  $q_t + q_b$  is the total THERMAL POWER in percent RTP.

The values denoted with [ \* ] are specified in the COLR.



3.7 PLANT SYSTEMS

3.7.11 Control Room Ventilation System (CRVS)

LOO 3.7.11 Two CRVS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One CRVS train inoperable.	A.1 Restore CRVS train to OPERABLE status.	7 days
B. Two CRVS trains inoperable.	B.1 Restore one CRVS train to OPERABLE status.	72 hours
C. Required Action and associated Completion Time of Condition A or B not met.	C.1 Be in MODE 3.	6 hours
	<u>AND</u> C.2 Be in MODE 5.	36 hours

**SURVEILLANCE REQUIREMENTS**

SURVEILLANCE	FREQUENCY
SR 3.8.7.1 Verify correct inverter voltage, frequency, and alignment to required 120V AC vital instrument buses.	7 days
SR 3.8.7.2 Verify manual transfer of the AC power source for VIB 34 from inverter 34 to each required CVT.	24 months

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

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- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the following:
- a. The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975; and
  - b. The operations manager shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except for the SRO license requirement which shall be in accordance with Technical Specification 5.2.2.e.
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
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5.0 ADMINISTRATIVE CONTROLS

5.4 Procedures

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- 5.4.1 Written procedures shall be established, implemented, and maintained covering the following activities:
- a. The applicable procedures recommended in Regulatory Guide 1.33, Revision 2, Appendix A, February 1978;
  - b. The emergency operating procedures required to implement the requirements of NUREG-0737 and to NUREG-0737, Supplement 1, as stated in Generic Letter 82-33;
  - c. Quality assurance for effluent and environmental monitoring;
  - d. Fire Protection Program implementation; and
  - e. All programs specified in Specification 5.5.
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5.5 Programs and Manuals

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5.5.10 Ventilation Filter Testing Program (VFTP) (continued)

- c. Demonstrate for each system that a laboratory test of a sample of the charcoal adsorber shows the methyl iodide removal efficiency specified below when tested in accordance with ASTM D3803-1989, subject to clarification below, at a temperature of 86°F and a relative humidity of 95%.

Ventilation System	Methyl iodide removal efficiency (%):	ASTM D3803-1989 Clarification
Control Room Ventilation System	≥ 95.5	78 ft/min face velocity
Containment Fan Cooler Units	≥ 85	59 ft/min face velocity

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(continued)

5.5 Programs and Manuals

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5.5.15 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak Test Program, dated September 1995" as modified by the following exception:

ANS 56.8-1994, Section 3.3.1: WCCPPS isolation valves are not Type C tested.

The maximum allowable primary containment leakage rate,  $L_a$ , at a minimum test pressure equal to  $P_a$ , shall be 0.1% of primary containment air weight per day.  $P_a$  is the peak calculated containment internal pressure related to the design basis accident.

Leakage acceptance criteria are:

- a. Containment leakage rate acceptance criterion is  $\leq 1.0 L_a$ . During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are  $\leq 0.60 L_a$  for the Type B and C tests and  $\leq 0.75 L_a$  for Type A tests;
- b. Air lock testing acceptance criteria are:
  - 1) Overall air lock leakage rate is  $\leq 0.05 L_a$  when tested at  $\geq P_a$ ,
  - 2) For each door, leakage rate is  $\leq 0.01 L_a$  when pressurized to  $\geq P_a$ ,
- c. Isolation Valve Seal Water System leakage rate acceptance criterion is  $\leq 14,700$  cc/hr at  $\geq 1.1 P_a$ .
- d. Acceptance criterion for leakage into containment from isolation valves sealed with the service water system is  $\leq 0.36$  gpm per fan

(continued)