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MFN 07-212, Supplement 1

Docket No. 52-010

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U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555-0001

Subject: Response to Portion of NRC Request for Additional Information Letter No. 40
Related to ESBWR Design Certification Application, RAI Number 19.2-29S01.

The purpose of this letter is to supplement the GE-Hitachi Nuclear Energy Americas LLC (GEH) response to the U.S. Nuclear Regulatory Commission (NRC) Request for Additional Information (RAI) sent by NRC letter dated July 5, 2006 (Reference 1) and responded to in Reference 2 on May 16, 2007. The information in that letter was provided to support NRC review of the GEH application for final design approval and standard design certification of the Economic Simplified Boiling Water Reactor (ESBWR) standard plant design pursuant to 10 CFR Part 52. This letter provides further discussion as requested by NRC via email. The responses to that question is addressed in Enclosure 1 as RAI Number 19.2-29.

Sincerely,



James C. Kinsey
Project Manager, ESBWR Licensing

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Reference:

1. MFN 06-222, Letter from U.S. Nuclear Regulatory Commission to David Hinds, Request for Additional Information Letter No. 40 Related to ESBWR Design Certification Application, July 5, 2006.
2. MFN 07-212, Response to Portion of NRC Request for Additional Information Letter No. 40 Related to ESBWR Design Certification Application ESBWR Probabilistic Risk Assessment RAI Numbers 19.2-8, 19.2-9, 19.2-14, 19.2-29 and 19.2-30. May 16, 2007.

Enclosure:

1. MFN 07-212, Supplement 1 Response to Portion of NRC Request for Additional Information Letter No. 40 Related to ESBWR Design Certification Application ESBWR Probabilistic Risk Assessment RAI Number 19.2-29 S01.

cc:	AE Cabbage	USNRC (with enclosure)
	GB Stramback	GEH/San Jose (with enclosure)
	RE Brown	GEH/Wilmington (with enclosure)
	eDRF Section	0000-0072-5376

Enclosure 1 to MFN 07-212, Supplement 1

**Response to Portion of NRC Request for
Additional Information Letter No. 40 Related to
ESBWR Design Certification Application
ESBWR Probabilistic Risk Assessment
RAI Number 19.2-29 S01**

NRC RAI 19.2-29 S01

Received by e-mail from Tom KeVERN.

PRA Table 7.2-5, as modified in response to RAI 19.0.0-3, and Table A.8-1 provide only a qualitative description of the LDW water level, and not the computed water level. Provide the computed water level in the LDW at the time of reactor vessel breach for those sequences where supporting MAAP analyses are available, and for the MAAP cases discussed in PRA Section 8.3.2. Address the impact of the non-safety containment spray system, since the spray system may be operated after the operators have entered the severe accident guidelines, provided the water level in the LDW is less than 0.7m

GEH Response

- (a) See Response to RAI 19.0.0-3 S01 transmitted via MFN 06-059, Supplement 1, dated August 10, 2007.

DCD /NEDO-33201 Impact

No DCD changes will be made in response to this RAI.

No changes to the subject NEDO-33201 will be made in response to this RAI.