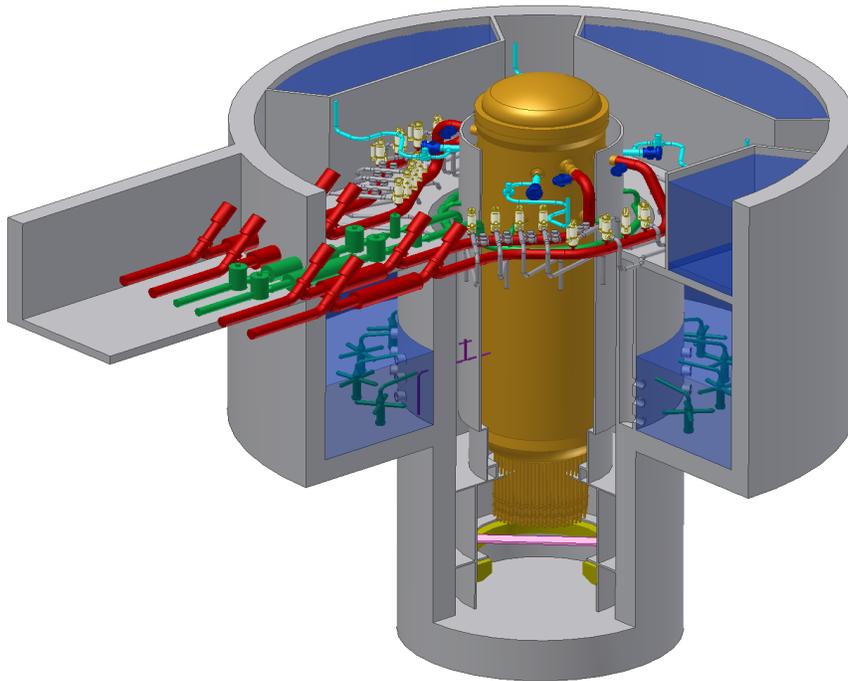




**GE Energy Nuclear**

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# **ESBWR Design Control Document**

**Tier 2**

**Chapter 9**

*Auxiliary Systems*



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## Abbreviations And Acronyms List

<b><u>Term</u></b>	<b><u>Definition</u></b>
10 CFR	Title 10, Code of Federal Regulations
ABS	Auxiliary Boiler System
ADA	American Disability Act
AHS	Auxiliary Heat Sink
AHU	Air handling unit
AISC	American Institute of Steel Construction
AISI	American Iron and Steel Institute
ALARA	As Low As Reasonably Achievable
ALWR	Advanced Light Water Reactor
ANS	American Nuclear Society
ANSI	American National Standards Institute
AOO	Anticipated Operational Occurrence
APRM	Average Power Range Monitor
ARI	Alternate Rod Insertion
ARMS	Area Radiation Monitoring System
ASHRAE	American Society of Heating, Refrigerating, and Air Conditioning Engineers
ASME	American Society of Mechanical Engineers
ATWS	Anticipated Transients Without Scram
AWWA	American Water Works Association
B&PV	Boiler and Pressure Vessel
BOP	Balance of Plant
BRE	Battery Room Exhaust
BTP	NRC Branch Technical Position
BWR	Boiling Water Reactor
C&FS	Condensate and Feedwater System
C&I	Control and Instrumentation
CB	Control Building
CBGAVS	Control Building General Area HVAC Subsystem
CBVS	Control Building HVAC System
CIRC	Circulating Water System
CIS	Containment Inerting System
CLAVS	Reactor Building Clean Area HVAC Subsystem
CM	Cold Machine Shop
CMS	Containment Monitoring System
CMAA	(CMAA70) Specifications for Electric Overhead Traveling Cranes
CMU	Control Room Multiplexing Unit
COL	Combined Operating License

### Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
COLR	Core Operating Limits Report
CONAVS	Reactor Building Contaminated Area HVAC Subsystem
CR	Control Rod
CRD	Control Rod Drive
CRHA	Control Room Habitability Area
CRHAVS	Control Room Habitability Area HVAC Subsystem
CSDM	Cold Shutdown Margin
CS / CST	Condensate Storage Tank
CT	Main Cooling Tower
CWS	Chilled Water System
D-RAP	Design Reliability Assurance Program
D/SP	Dryer /Separator Pool
DAC	Design Acceptance Criteria
DCS	Drywell Cooling System
DCIS	Distributed Control and Information System
DG	Diesel-Generator
DGVS	Electrical Building Diesel Generators HVAC Subsystem
dPT	Differential Pressure Transmitter
DPS	Diverse Protection System
DPV	Depressurization Valve
DW	Drywell
EB	Electrical Building
EBVS	Electrical Building HVAC System
ECCS	Emergency Core Cooling System
EER	Electrical Building Electric and Electronic Rooms
EERVS	Electrical Building Electric and Electronic Rooms HVAC Subsystem
EFU	Emergency Filter Unit
EMI	Electromagnetic Interference
ENS	Emergency Notification System
EOF	Emergency Operations Facility
EPRI	Electric Power Research Institute
EQ	Environmental Qualification
ERICP	Emergency Rod Insertion Control Panel
ERIP	Emergency Rod Insertion Panel
ESF	Engineered Safety Feature
ETS	Emergency Trip System
FAPCS	Fuel and Auxiliary Pools Cooling System
FATT	Fracture Appearance Transition Temperature

### Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
FB	Fuel Building
FBFPVS	Fuel Building Fuel Pool Area HVAC Subsystem
FBGAVS	Fuel Building General Area HVAC Subsystem
FBVS	Fuel Building HVAC System
FCU	Fan Cooling Unit
FHA	Fire Hazards Analysis
FM	Factory Mutual
FHM	Fuel Handling Machine
FMCRD	Fine Motion Control Rod Drive
FPE	Fire Pump Enclosure
FPM	Fuel Preparation Machine
FPS	Fire Protection System
FSGT	Fuel Support and Guide Tube
FSP	Fuel Storage Pool
FTS	Fuel Transfer System
FW	Feedwater
FWS	Firewater Storage Tank
GCS	Generator Cooling System
GDC	General Design Criteria
GDCS	Gravity-Driven Cooling System
GEEN	GE Energy Nuclear
GEN	Main Generator System
GETAB	General Electric Thermal Analysis Basis
G/F	Ground Floor
FM	Factory Mutual
FHM	Fuel Handling Machine
FMCRD	Fine Motion Control Rod Drive
FPE	Fire Pump Enclosure
FPM	Fuel Preparation Machine
FPS	Fire Protection System
FSGT	Fuel Support and Guide Tube
FSP	Fuel Storage Pool
FTS	Fuel Transfer System
FW	Feedwater
FWS	Firewater Storage Tank
HWC	Hydrogen Water Chemistry
HWCS	Hydrogen Water Chemistry System
HWS	Hot Water System

### Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
HX	Heat Exchanger
IC/PCCS	Isolation Condenser and Passive Containment Cooling System
I&C	Instrumentation and Control
IAS	Instrument Air System
IBC	International Building Code
IC	Isolation Condenser
ICD	Interface Control Diagram
IEEE	Institute of Electrical and Electronic Engineers
IESNA	Illuminating Engineering Society of North America
IFTS	Inclined Fuel Transfer System
IGSCC	Intergranular Stress Corrosion Cracking
ILRT	Integrated Leak Rate Test
IRV	Inside Reactor Vessel
ISA	Instrument Society of America
ISI	In-Service Inspection
ISLT	In-Service Leak Test
LCW	Low Conductivity Waste
LD	Logic Diagram
LDA	Lay down Area
LD&IS	Leak Detection and Isolation System
LLRT	Local Leak Rate Test
LO	Dirty/Clean Lube Oil Storage Tank
LOCA	Loss-of-Coolant-Accident
LOOP	Loss of Offsite Power
LOPP	Loss of Preferred Power
LP	Low Pressure
LPCI	Low Pressure Coolant Injection
LWMS	Liquid Waste Management System
MCC	Motor Control Center
MCR	Main Control Room
MOV	Motor-Operated Valve
MS	Main Steam
MSIV	Main Steam Isolation Valve
MSS	Main Steam System
MST	Main Steam Tunnel
MWS	Makeup Water System
NBS	Nuclear Boiler System
N-DCIS	Non safety-Related Distributed Control and Information System

## Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
NDE	Nondestructive Examination
NFI	New Fuel Inspection
NFPA	National Fire Protection Association
NFS	New Fuel Storage
NICWS	Nuclear Island Chilled Water Subsystem
NPHS	Normal Power Heat Sink
NPSH	Net Positive Suction Head
NRC	Nuclear Regulatory Commission
NRHX	Non-Regenerative Heat Exchanger
NS	Non-seismic
NSSS	Nuclear Steam Supply System
NT	Nitrogen Storage Tank
NTSP	Nominal Trip Setpoint
O&M	Operation and Maintenance
Q-DCIS	Safety-Related DCIS (Distributed Control and Information System)
OGS	Offgas System
OHLHS	Overhead Heavy Load Handling System
OIS	Oxygen Injection System
OOS	Out-of-service
OSHA	Occupational Safety and Health Administration
OSI	Open Systems Interconnect
P&ID	Piping and Instrumentation Diagram
PA/PL	Page/Party-Line
PABX	Private Automatic Branch (Telephone) Exchange
PCC	Passive Containment Cooling
PCCS	Passive Containment Cooling System
PL	Parking Lot
PM	Preventive Maintenance
PMF	Probable Maximum Flood
PRNM	Power Range Neutron Monitoring
PS	Plant Stack
PSS	Process Sampling System
PT	Pressure Transmitter
QA	Quality Assurance
RAM	Reliability, Availability and Maintainability
RB	Reactor Building
RBVS	Reactor Building HVAC System
RBV	Reactor Building Vibration

## Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
RC&IS	Rod Control and Information System
RCC	Remote Communication Cabinet
RCCV	Reinforced Concrete Containment Vessel
RCCWS	Reactor Component Cooling Water System
RCPB	Reactor Coolant Pressure Boundary
RCS	Reactor Coolant System
REPAVS	Reactor Building Refueling and Pool Area HVAC Subsystem
RF	Refueling Floor
RFI	Radio Frequency Interference
RG	Regulatory Guide
RH	Relative Humidity
RMS	Root Mean Square
RMS	Radiation Monitoring Subsystem
RMU	Remote Multiplexer Unit
RO	Reverse Osmosis
RPV	Reactor Pressure Vessel
RV	Reactor Vessel
RVC	Reactor Vessel Cavity
R/W	Reactor Well
RWCR	Radwaste Building Control Room
RWCRVS	Radwaste Building Control Room HVAC Subsystem
RWGA	Radwaste Building General Area
RWGAVS	Radwaste Building General Area HVAC Subsystem
RWVS	Radwaste Building HVAC System
SA	Severe Accident
S/N	Signal-to-Noise
S/P	Suppression Pool
SDC	Shutdown Cooling
SDS	System Design Specification
SFP	Spent fuel pool
SLC	Standby Liquid Control
SLC	Standby Liquid Control System
SP	Setpoint
SPC	Suppression Pool Cooling
SRM	Source Range Monitor
SRNM	Startup Range Neutron Monitor
SRO	Senior Reactor Operator
SRP	Standard Review Plan

## Abbreviations And Acronyms List

<u>Term</u>	<u>Definition</u>
SRV	Safety Relief Valve
SSC(s)	Structure, System and Component(s)
SSE	Safe Shutdown Earthquake
SSLC	Safety System Logic and Control
SV	Safety Valve
TAF	Top of Active Fuel
TB	Turbine Building
TBCE	Turbine Building Compartment Exhaust
TBE	Turbine Building Exhaust
TBLOE	Turbine Building Lube Oil Area Exhaust
TEMA	Tubular Exchanger Manufacturers' Association
TMI	Three Mile Island
TSC	Technical Support Center
TSCVS	Electrical Building Technical Support Center HVAC Subsystem
TSI	Turbine Supervisory Instrument
UDS	Upper Drywell / Steam tunnel
UDW	Upper Drywell
UHS	Ultimate heat sink
UL	Underwriter's Laboratories Inc.
UPS	Uninterruptible Power Supply
WS	Water Storage
WT	Water Treatment

## 9. AUXILIARY SYSTEMS

### 9.1 FUEL STORAGE AND HANDLING

Upon receipt of the new fuel bundles at the reactor site, each fuel bundle containers is uncrated from its shipping crate and the fuel bundle container is raised to the refueling floor in the fuel building. The fuel bundles are removed from the container and moved to the new fuel inspection stand where the fuel bundles are inspected and the fuel channels are installed to create fuel assemblies.

There are three areas where new and spent fuel is stored. In the fuel building there are spent fuel storage racks for storing new or spent fuel. In the reactor building there are new fuel storage racks for storing new fuel and a small array of spent fuel racks in a deep pit in the buffer pool for temporary storage of spent fuel.

The fuel bundle assemblies are then placed in the Spent Fuel Pool or in the Inclined Fuel Transfer System (IFTS) for transfer to the Reactor Building buffer pool. The newly channeled fuel assemblies are then moved to the new fuel storage racks in the Reactor Building buffer pool until time to move them into the reactor.

The new fuel storage racks in the buffer pool can store 60% of the total core load of new fuel assemblies. The fuel assemblies are stored in underwater storage racks located adjacent to the reactor well. The racks are side loading and are accessed using the refueling bridge.

Spent fuel removed from the reactor vessel must be stored underwater. There are two locations containing spent fuel storage racks. Buffer racks in the deep pit area of the Reactor Building buffer pool are used to temporarily store discharged fuel or fuel to be returned to the reactor during fuel shuffles. Spent fuel racks for long-term storage of spent fuel are located in the fuel building. These spent fuel storage racks are located at the bottom of the storage pools at a depth sufficient to provide adequate radiological shielding. The spent fuel storage pool water is processed by the Fuel and Auxiliary Pools Cooling System (FAPCS), which provides cooling to the spent fuel and maintains the spent fuel storage pool water quality. The buffer pool deep pit storage area in the Reactor Building can store minimum of 154 fuel assemblies. The fuel building spent fuel storage pool can store the fuel assemblies resulting from ten calendar years of operation plus one full core offload of fuel.

#### 9.1.1 New Fuel Storage

##### 9.1.1.1 Design Bases

###### Nuclear Design

The new fuel storage racks in the buffer pool are designed to assure that the fully loaded array is sub critical by at least 5%  $\Delta k/k$ .

Monte Carlo techniques are employed in the calculations performed to assure that  $k_{\text{eff}}$  does not exceed 0.95 under all normal and abnormal conditions.

The assumption is made that the storage array is infinite in all directions. Because no credit is taken for neutron leakage, the values reported as effective neutron multiplication factors are, in reality, infinite neutron multiplication factors.

The biases between the calculated results, experimental results, and the uncertainty in the calculation, are taken into account as part of the calculation procedure to assure that the specific  $k_{\text{eff}}$  limit is met.

### **Storage Design**

The new fuel storage racks in the buffer pool can store up to 60% of one full core fuel load.

### **Mechanical and Structural Design**

The new fuel storage racks contain storage space in the Reactor Building buffer pool in the reactor for 60% of the RPV core capacity of fuel assemblies (with channels) or bundles (without channels). They are designed to withstand all credible static and seismic loadings.

The racks are designed to protect the fuel assemblies and fuel bundles from excessive physical damage which may cause the release of radioactive materials in excess of 10 CFR 20 and 10 CFR 100 requirements, under normal and abnormal conditions caused by being struck by fuel assemblies, fuel bundles or other equipment.

The racks are constructed in accordance with the Quality Assurance Requirements of 10 CFR 50, Appendix B.

The racks are classified as Nonsafety-Related and Seismic Category I

### **Thermal-Hydraulic Design**

The fuel storage racks will be designed to provide sufficient natural convection coolant flow through the rack and fuel to remove any heat present without reaching excessive water temperatures. A thermal-hydraulic analysis to evaluate the rate of naturally circulated flow and the maximum rack exit temperature will be performed, see Subsection 9.1.6, COL Information.

### **Material Considerations**

Structural material used in the fabrication of the fuel storage racks is in accordance with the latest issue of the applicable ASTM specification at the time of equipment order. Materials are chosen for their corrosion resistance and their ability to be formed and welded with consistent quality.

### **Dynamic and Impact Analysis**

A standard dynamic analysis, using the appropriate response spectra, shall be performed by the COL licensee to demonstrate compliance to design requirements. The input excitation for these analyses utilizes the horizontal and vertical response spectra provided in Section 3.7.

Vertical impact analysis shall be performed by the COL licensee because the fuel assembly is held in the storage rack by its own weight without any mechanical hold-down devices. See Subsection 9.1.6 for COL information requirements for Dynamic and Impact Analysis.

#### ***9.1.1.2 Facilities Description (New Fuel Storage)***

##### **Pool Storage**

The new fuel storage racks in the Reactor Building buffer pool hold up to 60% of the RPV core capacity of channeled fuel assemblies. The racks have double rows of storage positions for assemblies that are side loaded into the storage racks. Because the racks are open on the side to

allow side loading, the weight of the fuel assemblies placed in the storage position actuates a mechanism that restrains the assemblies in position. The racks are floor mounted.

### **9.1.1.3 Safety Evaluation**

#### **Criticality Control**

The design of the new fuel storage racks provides for an effective multiplication factor ( $k_{\text{eff}}$ ) for either storage conditions equal to or less than 0.95. To ensure that design criteria are met, the normal or abnormal following normal and abnormal new fuel storage conditions were analyzed:

- Normal positioning in the new fuel array; and
- Eccentric positioning in the new fuel array.

The new fuel storage area accommodates fuel ( $k_{\text{inf}} \leq 1.35$  at 20°C in standard core geometry) with no safety implications.

#### **Structural Design**

The new fuel storage racks are designed to meet Seismic Category I requirements. Stresses in a fully loaded rack do not exceed stresses specified by the applicable American Society Mechanical Engineers (ASME) Codes and Standards when subjected to seismic loads.

The storage rack structure is designed to withstand the impact resulting from a falling fuel assembly.

Procedural fuel handling requirements and equipment design dictate that no more than one bundle at a time can be handled over the storage racks. The structural arrangement is such that no lateral displacement of the fuel occurs; therefore, sub-critical spacing is maintained. The COL licensee shall develop fuel handling procedures.

The racks are fabricated from ASTM material.

#### **Protection Features of the New Fuel Storage Facilities**

The new fuel storage racks are housed in the Reactor Building. The Reactor Building is Seismic Category I and designed for natural phenomena such as tornadoes, tornado missiles, floods and high winds.

The refueling machine is used to move fuel in the Reactor Building. It contains interlocks to prevent overloads from being applied to the fuel bail of the fuel assembly. Thus, the transfer devices used for new fuel handling to the new fuel rack cannot impose excessive uplift loads on the rack.

Should it become necessary to move major loads along or over the pools, administrative controls require that the load be moved over the empty portion of the buffer pool and avoid the area of the new fuel racks. Procedures are written for fuel handling.

## 9.1.2 Spent Fuel Storage

### 9.1.2.1 Design Bases

#### 9.1.2.2 Nuclear Design

A full array in the loaded spent fuel rack is designed to be sub-critical by at least 5%  $\Delta k/k$ . Neutron-absorbing material, as an integral part of the design, is employed to assure that the calculated  $k_{\text{eff}}$ , including biases and uncertainties, does not exceed 0.95 under all normal and abnormal conditions.

Monte Carlo techniques are employed in the calculations performed to assure that  $k_{\text{eff}}$  does not exceed 0.95 under all normal and abnormal conditions.

The storage array is assumed to be infinite in all directions. No credit is taken for neutron leakage, therefore, the values reported as effective neutron multiplication factors are, in reality, infinite neutron multiplication factors.

The biases between the calculated results and experimental results, as well as the uncertainty involved in the calculations, are taken into account as part of the calculative procedure to assure that the specific  $k_{\text{eff}}$  limit is met.

#### 9.1.2.3 Storage Design

The fuel storage racks provided in the Spent Fuel Pool in the Fuel Building provide for storage of irradiated fuel assemblies resulting from 10 calendar years of plant operation plus one full core off load. The fuel storage racks in the Reactor Building buffer pool deep pit can hold a minimum of 154 spent fuel assemblies.

#### 9.1.2.4 Mechanical and Structural Design

The spent fuel storage racks in the Reactor Building buffer pool and in the Spent Fuel Pool in the Fuel Building contain storage space for fuel assemblies (with channels) or bundles (without channels). A standard dynamic analysis using the appropriate response spectra is performed to demonstrate compliance to design requirements. They are designed to withstand all credible static and seismic loadings. The racks are designed to protect the fuel assemblies and bundles from excessive physical damage which may cause the release of radioactive materials in excess of 10 CFR 20 and 10 CFR 100 requirements, under normal and abnormal conditions caused by impact from fuel assemblies, bundles or other equipment.

The Spent Fuel Pool and buffer pool are reinforced concrete structures with a stainless steel liner. Fuel storage racks and pool liner embedments are designed to meet Seismic Category I requirements. Pool liner and anchorage are designed to the same loads and load combinations as the pool concrete structure in accordance with Table 3.8-15, except that load factors for all cases are equal to 1.0, and the acceptance criteria follow ASME Section III, Division 2, CC-3700. Pool liners will be evaluated to ensure structural integrity under fuel handling accidents. The bottoms of the pool gates are higher than the minimum water level required over the spent fuel storage racks to provide adequate shielding and cooling. Pool fill and drain lines enter the pool above the safe shielding water level. Redundant anti-siphon vacuum breakers are located at the

high point of the pool circulation lines to preclude a pipe break from siphoning the water from the pool and jeopardizing the safe water level.

The racks include individual solid tube storage compartments, which provide lateral restraints over the entire length of the fuel assembly or bundle. The weight of the fuel assembly or bundle is supported axially by the rack fuel support. Lead-in guides at the top of the storage spaces provide guidance of the fuel during insertion.

Materials used for construction are specified in accordance with the latest issue of applicable ASTM specifications. The racks are constructed in accordance with a quality assurance program that ensures the design, construction, and testing requirements are met.

The structural integrity of the rack is demonstrated for the loads and load combinations described below using linear elastic design methods.

The applied loads to the rack are as follows:

- Dead loads—weight of rack and fuel assemblies plus the hydrostatic loads;
- Live loads—effect of lifting an empty rack during installation;
- Thermal loads—effects caused by pool temperature changes occurring as a result of normal operating or abnormal conditions, as applicable;
- Seismic loads;
- Fuel drop load—effect of an accidental drop of the heaviest fuel assembly or bundle from the maximum possible height; and
- Stuck fuel load—upward force on the rack caused by a postulated stuck fuel assembly.

The load combinations considered in the rack design are as follows:

- Dead plus live loads;
- Dead plus live plus thermal loads
- Dead plus live plus thermal plus stuck fuel loads;
- Dead plus live plus thermal plus seismic loads; and
- Dead plus live plus fuel drop loads;

Stress analyses are performed by classical methods based upon shears and moments developed by the dynamic method. Using the given loads, load conditions and analytical methods, stresses are calculated at critical sections of the rack and compared to acceptance criteria referenced in ASME Code Section III, Subsection NF. In addition, the design of the spent fuel storage racks and associated support structures meet the requirements of Appendix D to SRP 3.8.4.

#### ***9.1.2.5 Thermal-Hydraulic Design***

During normal operation the fuel storage racks are designed to provide sufficient natural convection coolant flow through the rack and fuel to remove decay heat without reaching excessive water temperatures (100°C; 212°F).

In the spent fuel storage pool, the bundle decay heat is removed by FAPCS recirculation flow to maintain the pool temperature below 48.9°C (120°F) during normal conditions.

A thermal-hydraulic analysis to evaluate the rate of naturally circulated flow and the maximum rack exit temperature will be performed. See Subsection 9.1.6 for COL information.

In the event of loss of FAPCS cooling trains boiling can occur, see Subsection 9.1.3.2. The structural acceptance criteria for the fuel storage racks is that the storage rack design shall not exceed the allowable stress levels given in the ASME B&PV Code, Section III, Subsection NF during boiling.

#### ***9.1.2.6 Material Considerations***

Structural material used in the fabrication of the fuel storage racks is in accordance with the latest issue of the applicable ASTM specification at the time of equipment order. Materials are chosen for their corrosion resistance and their ability to be formed and welded with consistent quality.

The storage tube material is permanently marked with identification traceable to the material certifications. The fuel storage tube assembly is compatible with the environment of treated water and provides a design life of 60 years.

#### ***9.1.2.7 Facilities Description (Spent Fuel Storage)***

There are two separate areas for storage of spent fuel assemblies. These are in a separate deep pit area in the buffer pool in the Reactor Building and in the Spent Fuel Pool in the Fuel Building.

Spent fuel storage racks in the buffer pool area provide storage in the Reactor Building Spent Fuel Pool for spent fuel received from the reactor vessel during the refueling operation. The amount to be stored is equal to the amount of fuel that may be removed and subsequently returned to the RPV during a typical fuel shuffling activity. The deep pit for the storage of spent fuel in the rack is designed such that the depth of the cavity allows the fuel to be placed in the rack with sufficient margin below the rack for natural convection cooling to occur and that the top of the active fuel remains below the top of the cavity. The spent fuel storage racks are top entry racks designed to preclude the possibility of criticality under normal and abnormal conditions.

Spent fuel storage racks in the Spent Fuel Pool area provide storage in the fuel building Spent Fuel Pool for spent fuel received from the reactor vessel resulting from ten calendar years of operation plus one full core offload of fuel. The cavity for the storage of spent fuel in the rack is designed such that the depth of the cavity allows the fuel to be placed in the rack with sufficient margin below the rack for natural convection cooling to occur and that the top of the active fuel remains below the top of the cavity. The spent fuel storage racks are top entry racks designed to preclude the possibility of criticality under normal and abnormal conditions.

On a complete loss of the FAPCS active cooling capability and under the condition of maximum heat load associated with 20 years of fuel storage and a full core off load, sufficient quantity of water is available in the Spent Fuel Pool above the top of active fuel (TAF) level to allow boiling for 72 hours and still have the TAF submerged under water.

### 9.1.2.8 Safety Evaluation

#### Criticality Control

The spent fuel storage racks are designed to assure that the fully loaded array is sub critical by at least 5%  $\Delta k/k$ .

Monte Carlo techniques are employed in the calculations performed to assure that  $k_{eff}$  does not exceed 0.95 under all normal and abnormal conditions.

The assumption is made that the storage array is infinite in all directions. Because no credit is taken for neutron leakage, the values reported as effective neutron multiplication factors are, in reality, infinite neutron multiplication factors.

The biases between the calculated results, experimental results, and the uncertainty in the calculation, are taken into account as part of the calculative procedure to assure that the specific  $k_{eff}$  limit is met.

Structural Design and Material Compatibility Requirements:

- The support structure allows sufficient pool water flow for natural convection cooling of the stored fuel and allows the rack material temperatures to stay within limits.
- The racks include individual solid tube storage compartments, which provide lateral restraints over the entire length of the fuel assembly or bundle.
- The racks are fabricated from materials used for construction and are specified in accordance with the latest issue of applicable ASTM specifications at the time of equipment order.
- The racks are designed to withstand the impact force generated by the vertical free-fall drop of a fuel assembly and its handling tool from the maximum height expected during normal fuel handling.
- The rack is designed to withstand a pull-up force in the event a fuel assembly should jam.
- The fuel storage pools have adequate water shielding for the stored spent fuel. See Subsection 9.1.3.

NRC Regulatory Guide 1.13 is applicable to spent fuel storage facilities. The Reactor Building, which contains the fuel storage facilities, including the storage racks and pool, is designed to protect the fuel from damage caused by:

- Natural events such as earthquake, high winds and flooding; and
- Mechanical damage caused by dropping of fuel assemblies, bundles, or other objects onto stored fuel.

#### Summary of Radiological Considerations

By adequate design and careful operational procedures, the design bases of the spent fuel storage arrangement are satisfied. Thus, the exposure of plant personnel to radiation is maintained well below published guideline values. Further details of radiological considerations, including those for the spent fuel storage arrangement, are presented in Chapter 12.

### 9.1.3 Fuel and Auxiliary Pools Cooling System

#### 9.1.3.1 Design Bases

##### Safety Design Basis

Fuel and Auxiliary Pools Cooling System (FAPCS) is a Nonsafety-Related system, except for the containment isolation valves, the high-pressure interface with the Reactor Water Cleanup / Shutdown Cooling System, and emergency water supply flow paths.

##### Power Generation Design Basis

FAPCS provides continuous cooling and cleaning of the spent fuel storage pool during normal plant operation. It also provides occasional cooling and cleaning of various pools located inside the containment during normal plant operation and refueling outage.

#### 9.1.3.2 System Description

##### System Description Summary

The FAPCS consists of two physically separated cooling and cleanup (C/C) trains, each with 100% capacity during normal operation. Each train contains a pump, a heat exchanger and a water treatment unit for cooling and cleanup of various cooling and storage pools except for the Isolation Condenser and Passive Containment Cooling System (IC/PCCS) pools (refer to Figure 9.1-1). A separate subsystem with its own pump, heat exchanger and water treatment unit is dedicated for cooling and cleaning of the IC/PCCS pools independent of the FAPCS C/C train operation during normal plant operation (refer to Figure 9.1-1).

The primary design function of FAPCS is to cool and clean pools located in the containment, Reactor Building and Fuel Building (refer to Table 9.1-1) during normal plant operation. FAPCS provides flow paths for filling and makeup of these pools during normal plant operation and during post accident conditions, as necessary.

FAPCS is also designed to provide the following accident recovery functions in addition to the Spent Fuel Pool cooling function:

- Suppression pool cooling (SPC);
- Drywell spray;
- Low pressure coolant injection (LPCI) of suppression pool water into the RPV; and
- Alternate Shutdown Cooling.

In addition to its accident recovery function, suppression pool cooling (SPC) mode is also designed to automatically initiate during normal operation in response to a high temperature signal from the suppression pool.

Redundancy and physical separation will be provided in accordance with SECY 03-087 for active components in lines dedicated to LPCI and SPC modes.

During normal plant operation, at least one FAPCS C/C train is available for continuous operation to cool and clean the water of the Spent Fuel Pool, while the other train can be placed in standby or other mode for cooling the Gravity Driven Cooling System (GDSCS) pools and

suppression pool. If necessary during refueling outage, both trains may be used to provide maximum cooling capacity for cooling the Spent Fuel Pool. The water treatment units can be bypassed when necessary, and will be bypassed automatically on a high temperature signal downstream of the heat exchangers.

Each FAPCS C/C train has sufficient flow and cooling capacity to maintain Spent Fuel Pool bulk water temperature below 48.9°C (120°F) under normal Spent Fuel Pool heat load conditions. During the maximum Spent Fuel Pool heat load conditions of a full core off-load plus irradiated fuel in the Spent Fuel Pool resulting from 20 years of plant operations, both FAPCS C/C trains are needed to maintain the bulk temperature below 60°C (140°F).

During a loss of the FAPCS cooling trains, the cooling to the Spent Fuel Pool and IC/PCCS pools is accomplished by allowing the water to heat and boil. Sufficient pool capacity exists for pool boiling to continue for at least 72 hours post-accident, at which point emergency makeup water can be provided through safety-related connections to the Fire Protection System (FPS) or another onsite or offsite water source.

All operating modes (refer to Table 9.1-2) are manually initiated and controlled from the Main Control Room (MCR), except the SPC mode, which is initiated either manually, or automatically on high suppression pool water temperature signal. Instruments are provided for indication of operating conditions to aid the operator during the initiation and control of system operation. Provisions are provided to prevent inadvertent draining of the pools during FAPCS operation by including anti-siphon holes on all FAPCS piping that is normally submerged.

The FAPCS is designed to provide for the collection, monitoring, and drainage of pool liner leaks from the spent fuel pools, auxiliary pools, and IC/PCCS pools (refer to Table 9.1-1) to the Liquid Waste Management System.

Containment isolation valves are provided on the lines that penetrate the primary containment. Containment isolation valves are powered from independent safety-related sources. Air-operated valves with containment isolation function are designed to close upon loss of its electric power or air supply, except for containment isolation valves on the suppression pool supply and return lines, which fail as-is.

The containment isolation valves are automatically closed upon receipt of a containment isolation signal from the Leakage Detection and Isolation System (LD&IS). The containment isolation valves needed to perform an accident recovery function described above, will require a manual operator action to override the isolation signal.

The FAPCS is a Nonsafety-Related system with the exception of piping and components required for:

- Containment isolation;
- Refilling of the IC/PCCS pools and the Spent Fuel Pool with emergency water supplies from the Fire Protection System or another onsite or offsite source.
- The high-pressure interface with the Reactor Water Cleanup/Shutdown Cooling system used for low pressure coolant injection.

The FAPCS piping and components that are required to support safety-related and/or accident recovery function have Quality Group B or C and Seismic I classification (Table 9.1-3). A

Seismic I classification is required for all safety-related functions listed above. A Seismic II classification is sufficient for the remaining Nonsafety-Related piping and components that support accident recovery functions. This classification satisfies the requirements of SRP 9.1.3 Section I.1.

### Detailed System Description

The FAPCS is provided with two cooling and cleanup (C/C) trains with 100% capacity during normal operation. Each FAPCS train is physically separated and has one pump, one heat exchanger and one water treatment unit consisting of a prefilter and a demineralizer.

A manifold of four motor operated valves is attached to each end of the FAPCS C/C trains [refer to Figure 9.1-1]. These manifolds are used to connect the FAPCS C/C train with one of the two pairs of suction and discharge piping loops to establish the desired flow path during FAPCS operation. One loop is used for the fuel pools and auxiliary pools, and the other loop for the GDSC pools and suppression pool and for injecting water to drywell spray sparger and reactor vessel via RWCU/SDC and feedwater pipes.

The use of manifolds with proper valve alignment and separate suction-discharge piping loops 1) allows operating of one train independent of the other train to permit on-line maintenance or dual mode operation using separate trains if necessary, 2) prevents inadvertent draining of the pool and mixing of contaminated water in the Spent Fuel Pool with cleaner water in other pools.

Each water treatment unit is equipped with a prefilter, a demineralizer and a post strainer. A bypass line is provided to permit bypass of the water treatment unit, when necessary. The prefilter and demineralizers of the water treatment units are located in shielding cells so that radiation exposure of plant personnel is within acceptable limits.

Proper physical separation is provided between the active components of the two redundant trains to assure operation of one train in the event of failure of the other train.

A reactor makeup water discharge line is provided for injecting suppression pool water or water from the Fire Protection System to the reactor vessel via Reactor Water Cleanup/Shutdown Cooling System (RWCU/SDC) Loop B and Feedwater Loop A discharge pipes. This injection line shall be provided with two pairs of isolation valves such that the flow path branches to include two parallel air-operated gate valves and two parallel testable check valves in series (refer to Figure 9.1-1). This line is safety-related up to the second pair of isolation valve upstream of the RWCU/SDC interface. These isolation valves fail as-is, are normally closed, and prevented from opening by a high reactor pressure signal from the Nuclear Boiler System to protect the low pressure portion of FAPCS piping and components. Redundant valves are contained in separate fire zones for improved reliability.

A drywell spray discharge line and a ring header with spray nozzles mounted on the header are provided for spraying water inside the drywell to reduce the drywell pressure 72 hours following a LOCA to assist in post accident recovery. In order to prevent excessive negative pressure the drywell spray flow rate must be less than 227 m<sup>3</sup>/hr (1000 gpm). The drywell spray flow rate is maintained below this value by a flow-restricting orifice. The ring header equipped with spray nozzles is located in the drywell.

A separate cooling and cleanup subsystem completely independent of FAPCS C/C trains and their piping loop is provided for cooling and cleanup of the IC and PCCS pools to prevent

radioactive contamination of these pools. The subsystem consists of one pump, one heat exchanger, and one water treatment unit .

FAPCS contains two containment isolation valves on the lines that penetrate the primary containment. One isolation valve is located inside the containment and the other isolation valve is outside the containment. For added reliability, the suppression pool suction and return lines shall branch to form parallel flow paths as they penetrate containment. The suppression pool return line shall branch outside containment, and both parallel flow paths shall have one isolation valve outside and one isolation valve inside containment. The suppression pool supply line shall branch inside containment and each parallel flow path shall have two isolation valves in series outside containment. It is acceptable to locate both valves outside of containment because

An isolation valve inside the drywell would be submerged under water during severe accident conditions.

All outboard valves shall be located as close as possible to the containment boundary. This arrangement with two outboard valves in series is in accordance with the criteria in SRP 6.2.4 section II.6.d (see Table 1.9-20).

Pipes equipped with normally closed manual valves are provided for establishing flow paths from off-site emergency water supplies or the Fire Protection System to refill the IC/PCCS pools and Spent Fuel Pool following a design basis loss of coolant accident.

Anti-siphoning devices are used on all submerged FAPCS piping to prevent unintended drainage of the pools. The anti-siphoning holes for all FAPCS discharge lines are located at the elevation of normal water level to prevent significant draining of the pool in case of a suction line break at a lower elevation. The anti-siphoning holes in the suction piping of the Suppression Pool, GDSC Pools, and IC/PCCS C/C subsystem are located at the elevation of minimum water level to prevent significant draining of the pool in case of a suction line break at a lower elevation..

The spent fuel pool is equipped with drainage paths behind the liner welds. These paths are designed to:

- Prevent stagnant water buildup behind the liner plate;
- Prevent the uncontrolled loss of contaminated pool water; and
- Provide liner leak detection and measurement.

The reactor well, dryer/separator pool, buffer pool, upper and lower fuel transfer pools, cask pool, and IC/PCCS pools are also equipped with stainless steel liners, and shall be equipped with leak detection drains as part of the FAPCS. All leak detection drains are designed to permit free gravity drainage to the Liquid Waste Management System.

The containment isolation valves and other equipment required for the post-accident recovery function are provided with electric power from reliable power supplies. In the event of loss of off-site power, these electric power supplies are automatically connected to the on-site power sources. The electrical power supplies, control and instrumentation of the two FAPCS trains and their supporting systems are divisionally separated. Pneumatic power assisted containment isolation valves on the suppression pool supply and return lines are designed to fail as-is upon loss of its electric power or pneumatic (air or nitrogen) supply. All other containment isolation valves are designed to fail closed.

Provisions are provided to protect FAPCS components from fire, missile generating event, plant internal flooding, or seismic event of intensity up to and including a Safe Shutdown Earthquake (SSE) so that sufficient capability is retained for the fuel pool cooling function.

The FAPCS is designed to permit surveillance testing and in-service inspection of the safety-related components in accordance with ASME Section XI. Additionally, the FAPCS is designed to permit leak rate testing of its components required to perform containment isolation function, in accordance with 10 CFR 50 Appendix J.

Piping and components completely separate from FAPCS pool cooling piping provide flow paths for post-accident make-up water transfer, from off-site water supply sources to the IC/PCCS pool and spent fuel pool. Active FAPCS valves located inside the Reactor Building are not required to operate to accomplish this makeup. This piping and components are designed to meet Quality Group C and Seismic Category II requirements.

The Dryer/Separator Pool and Reactor Well contains valves that, when opened, create a connection between the two IC/PCCS expansion pools through the Dryer/Separator Pool. These valves are designed to open on receiving a low level signal from either of the IC/PCCS expansion pools, and will allow the IC/PCCS Pools to utilize the inventory in the Dryer/Separator Pool and Reactor Well.

Branch connections are provided on the suppression pool suction line and return line, which serve as attachments for portable external cooling equipment that bypasses the FAPCS C/C trains.

FAPCS piping and components, relied upon for containment integrity, are designed to Quality Group B and Seismic Category I requirements.

### **System Operation**

FAPCS C/C trains operate continuously to cool and clean the water in the Spent Fuel Pool during normal plant operation and refueling outage. Operation of only one FAPCS C/C train is sufficient to handle the cooling requirements under the normal heat load condition in the Spent Fuel Pool. Operation with up to two FAPCS C/C trains is sufficient to handle the cooling requirement under the maximum heat load condition. At least one FAPCS C/C train is available for cooling the Spent Fuel Pool, except for a short period as long as the water temperature in the pool remains below the maximum temperature limit for normal operation.

During a refueling outage, FAPCS can be operated in the Fuel and Auxiliary Pool Cooling and Cleanup mode with both C/C trains under the maximum heat load condition in the Spent Fuel Pool.

If necessary the FAPCS can operate in a dual mode using two separated FAPCS C/C trains with separate suction and discharge piping loops. However, dual mode operation using a single train is prohibited, because it could result in redistribution of water between pools containing contaminated water and pools containing clean water.

As necessary during normal plant operation, the standby FAPCS C/C train is placed in operation to cool and clean the water in the Suppression Pool and GDSCS pools. The IC/PCCS pools C/C subsystem operates as necessary to cool and clean the water in the IC/PCCS pools during normal plant operation.

The FAPCS may be operated in the following modes for post-accident recovery following an accident:

- Spent Fuel Pool Cooling;
- Low Pressure Coolant Injection (LPCI);
- Suppression Pool Cooling (SPC);
- Drywell Spray; and
- Alternate Shutdown Cooling (ASDC).

All FAPCS lines penetrating the containment that do not have a post-accident recovery function are automatically isolated upon receipt of a containment isolation signal from Leak Detection and Isolation System (LD&IS).

The FAPCS piping provides flow paths for delivery of makeup water to IC/PCCS pools and Spent Fuel Pool from offsite emergency water supply or Fire Protection system as needed 72 hours after a design basis accident.

### **System Operating Modes**

FAPCS is designed to operate in the modes listed in Table 9.1-2. The following paragraphs describe the major operating modes of FAPCS:

**Spent Fuel Pool Cooling and Cleanup Mode** – One of the FAPCS C/C trains is continuously operated in this mode to cool and clean the water in the Spent Fuel Pool during normal plant operation and refueling outage. This mode may be initiated following an accident to cool the Spent Fuel Pool for accident recovery. During this mode of operation, water is drawn from the skimmer surge tanks, pumped through the heat exchanger and water treatment unit to be cooled and cleaned and then returned to the Spent Fuel Pool (SFP). As the SFP level rises, water will spill into the weir and flow back to the skimmer surge tanks. When necessary, a portion or all of the water may bypass the water treatment unit.

**Fuel and Auxiliary Pool Cooling and Cleanup Mode** - During a refueling outage, one or both FAPCS C/C trains are placed in this mode of operation to cool and clean the water in the Spent Fuel Pool and pools listed below depending on the heat load condition in these pools.

- Upper fuel transfer pool;
- Buffer pool;
- Reactor well; and
- Dryer and separator storage pool.

During this mode of operation, water is drawn from the skimmer surge tanks, pumped through the heat exchanger and water treatment unit to be cooled and cleaned and then returned to these pools. When necessary, portion or all of the water may bypass the water treatment unit.

**IC/PCCS Pool Cooling and Cleanup Mode** –The FAPCS-IC/PCCS pool C/C subsystem is placed in this mode as necessary during normal plant operation. During this mode of operation, water is drawn via a common suction header from each IC/PCCS subcompartment. Water is

cooled and cleaned by the IC/PCCS pool C/C subsystem and is then returned to the two expansion pools through a common line that branches and discharges deep into each pool.

**GDCS Pool Cooling and Cleanup Mode** – One train of the FAPCS C/C that is not operating in Spent Fuel Pool cooling mode is placed in this mode as necessary during normal plant operation. Water is drawn from GDCS pools A and D in this mode of operation. The water is cooled and cleaned and is then returned to GDCS pool B/C. The water level in GDCS pool B/C rises and the water is cascaded and discharged at a submerged location in the adjacent GDCS pools A and D during this mode of operation.

**Suppression Pool Cooling and Cleanup Mode** – One of the FAPCS C/C trains that is not operating in Spent Fuel Pool cooling mode is placed in this mode as necessary during normal plant operation. Water is drawn from the suppression pool and is cooled and cleaned and then returned to the suppression pool in this mode of operation. This mode may be manually initiated following an accident to cool the suppression pool for accident recovery. This mode may also be automatically initiated during normal operation in response to a high temperature signal from the suppression pool.

**Low Pressure Coolant Injection (LPCI) Mode** - This mode may be initiated following an accident after the reactor has been depressurized to provide reactor makeup water for accident recovery. In this mode the FAPCS pump takes suction from the suppression pool and pumps it into the reactor vessel via RWCU/SDC loop B and then Feedwater loop A.

**Alternate Shutdown Cooling Mode** – This mode may be initiated following an accident for accident recovery. In this mode, FAPCS operates in conjunction with other systems to provide reactor shutdown cooling in the event of loss of other shutdown cooling methods. FAPCS flow path is similar to that of LPCI mode during this mode of operation. Water is drawn from the suppression pool, cooled and then discharged back to the reactor vessel via the LPCI injection flow path. The warmer water in the reactor vessel rises and then overflows into the suppression pool via two opened safety-relief valves on the main steam lines, completing a closed loop for this mode operation.

**Drywell Spray Mode** - This mode may be initiated following an accident for accident recovery. During this mode of operation, FAPCS draws water from the suppression pool, cools and then sprays the cooled water to drywell air space to reduce the containment pressure.

### **9.1.3.3 Safety Evaluation**

The FAPCS is a Nonsafety-Related system except for the portions of the system that establish flow paths necessary for

- The interface with safety-related RWCU/SDC piping;
- The supply of emergency makeup water to the Spent Fuel Pool and IC/PCCS pools following an accident; and
- The system's containment isolation function.

The SFP is designed to dissipate fuel decay heat through heat up and boiling of the pool water. The pool water performs the safety-related heat removal function stipulated in GDC 44. Upon loss of power, the Fuel Building HVAC isolates the fuel building as described in Subsection 9.4.2.5. Steam generated by boiling of the SFP is released to the atmosphere through a relief

panel in the Fuel Building. Water inventory in the SFP is adequate to keep the fuel covered through 72 hours, thereby avoiding heat up of the fuel and the potential for fission product release. Engineered safety feature atmosphere cleanup systems and associated guidance described in RG 1.52 are not provided in the ESBWR design as indicated in Subsection 15.4.1.4.1. The Fuel Building does not house any safety-related equipment, subject to flooding, as stated in Subsection 3.4.1.4.3. Sufficient reserve capacity is maintained on-site to extend the safe shutdown state from 72 hours through 7 days ensuring compliance with GDC 61. Post 72-hour inventory makeup is provided via safety-related connections to the Fire Protection System and to offsite water sources.

The FAPCS piping and components that provide the flow paths for the emergency makeup water supply are designed to meet the requirements contained in Table 9.1-3, Item 3. No active valves are required to operate for establishing emergency makeup water supply flow paths.

Two containment isolation valves are provided on the FAPCS lines that penetrate the containment.

The use of two containment isolation valves meets NRC GDC 56 requirement. The use of two air-operated outboard containment isolation valves also satisfies GDC 56 based on the allowable exception (Section II.6.d of SRP 6.2.4).

Containment isolation provisions that differ from the explicit requirements of GDC 56 are acceptable if the basis for the difference is justified. The exception for the suppression pool suction line is quoted in the SRP 6.2.4 Section II.6.d and is acceptable because an inboard valve would be submerged following a severe accident. In order to take credit for this alternate arrangement, the valve closest to containment shall be designed in accordance with SRP Section 3.6.2:

The containment isolation valves on the suppression pool supply and return lines are fail as-is on loss of electric power or the air supply. All other containment isolation valves are fail closed.

The Reactor Building and the Fuel Building provide adequate protection against natural phenomena for the safety-related components of the FAPCS as required by GDC 2 and GDC 4.

Safety-related level instrumentation is provided in the spent fuel pool and IC/PCC pools to detect a low water level that would indicate a loss of decay heat removal ability in accordance with GDC 63.

#### ***9.1.3.4 Testing and Inspection Requirements***

The FAPCS is designed to permit surveillance test and in-service inspection of its safety-related components and components required to perform the post-accident recovery functions, in accordance with GDC 45 and ASME Section XI. The FAPCS is designed to permit leak rate testing of its components required to perform containment isolation function in accordance with 10 CFR 50 Appendix J.

#### ***9.1.3.5 Instrumentation and Control***

##### **System Instrumentation**

**Water Levels** - The skimmer surge tank level is monitored by a level transmitter mounted on a local panel. The skimmer surge tank level is displayed in the MCR. In addition to level indication, this signal is used to initiate low and high water-level alarms and to operate the Condensate Storage and Transfer System makeup water control valve for the skimmer surge tank.

The IC/PCCS pool has two local panel-mounted, safety-related level transmitters for both expansion pools. Both transmitter signals are indicated on the safety-related displays and sent through the gateways for Nonsafety-Related display and alarms. Both signals are validated and used to control the valve in the makeup water supply line to the IC/PCCS pool.

The Spent Fuel Pool has two wide-range safety-related level transmitters that transmit signals to the MCR. These signals are used for water level indication and to initiate high/low-level alarms.

All other pools (upper transfer pool, lower fuel transfer pool, cask pool, buffer pool, reactor well, dryer and separator storage pool) have local, Nonsafety-Related, panel-mounted level transmitters to provide signals for high/low-level alarms in the MCR.

Level instruments for the suppression pool and GDCS pools are provided by other systems.

**Water Temperatures** – Water temperatures are monitored in the Fuel and Auxiliary pools (listed in Table 9.1-1) with temperature transmitters that send signals to the MCR for water temperature indication and high-temperature alarms. In the IC/PCCS pool, each condenser subcompartment also has temperature transmitters that send signals to the MCR for water temperature indication and high-temperature alarms. The upstream and downstream piping of the two heat exchangers in the cooling/cleanup trains have temperature transmitters that send signals to the MCR.

**Flow and Pressure** - Panel-mounted pressure transmitters for the pump suction and discharge pressure are provided locally. A pump trip signal is generated on low suction pressure to provide for pump protection. The pressure transmitters send signals to pressure indicators in the MCR. An orifice type flow element is located on the downstream side of each pump discharge check valve. A local panel-mounted flow transmitter sends the signals from these transmitters to flow indicators in the MCR.

### **System Controls**

All FAPCS operating modes are manually initiated, except Suppression Pool Cooling (SPC) mode. The SPC mode is initiated either manually, or automatically upon receipt of a high suppression pool temperature signal from the Containment Monitoring System. The automatic SPC mode initiation selects the FAPCS standby train (if available) to be initiated.

Upon receipt of a containment isolation signal from the Leak Detection and Isolation System, all containment isolation valves are signaled to close, with the exception of the containment isolation valves on the suppression pool supply and return lines as well as the drywell spray line.

FAPCS C/C train pumps are automatically tripped under the following operating conditions:

- Low water levels in skimmer surge tank
- Low water level in Suppression Pool
- Low water level in GDCS pools

- High water level in GDCS pools
- Low pump suction pressure
- Low pump discharge flow
- High pump discharge flow

The FAPCS IC/PCCS pools C/C subsystem pump is automatically tripped on low water level in IC/PCCS pools. Water level in the skimmer surge tanks is maintained by automatic open/closure of the makeup water supply isolation valve. Water level in the IC/PCCS pools is maintained by automatic open/closure of the makeup water supply isolation valve.

#### **9.1.4 Light Load Handling System (Related to Refueling)**

The reactor and fuel servicing system associated with the handling of light loads includes the fuel storage arrangements and the necessary facilities, special tools, and equipment required to accomplish normal fueling and refueling outage tasks.

The system is integrated with other customer provided equipment and supporting services to enhance and implement the fuel handling procedure in a safe and efficient manner.

##### **9.1.4.1 Design Bases**

The fuel handling system is designed to provide a safe and effective means for transporting and handling fuel from the time it reaches the plant until it leaves the plant after post-irradiation cooling. Safe handling of fuel includes design considerations for maintaining occupational radiation exposures as low as reasonably achievable (ALARA).

The following subsections briefly describe the integrated fuel transfer and reactor vessel servicing system that ensures that the design bases of the fuel handling system and the requirements of Regulatory Guide 1.13 are satisfied.

Table 9.1-4 provides the design criteria for major fuel handling system equipment and lists the safety class, quality group and seismic category. Where applicable, Table 9.1-5 identifies applicable ASME, American National Standards Institute (ANSI), Industrial and Electrical Codes. Additional design criteria are described below and expanded further discussed in Subsection 9.1.4.2.

Fuel transfer from the point of receipt up to inspection, storage, and placement in the reactor core is accomplished with fuel grapples. A general purpose fuel grapple is used when fuel movement is performed by the fuel building crane on the fuel building floor prior to placement in the fuel preparation machine and transfer to the Spent Fuel Pool or buffer pool. During refueling operations, however, fuel movement is performed in the fuel building by the fuel handling machine and in the Reactor Building by the refueling machine telescoping grapples.

The refueling machine and the fuel handling machine are classified as Nonsafety-Related Seismic Category II, and are constructed in accordance with a supplemental quality assurance program to ensure compliance with applicable design, construction and test requirements.

Working loads of the refueling machine and fuel handling machines structures are in accordance with the American Institute of Steel and Construction (AISC) Manual of Steel Construction. All parts of the hoist systems are designed to have a safety factor of at least 10, based on the ultimate strength of the material. A redundant load path is incorporated in the fuel grapples.

Both the refueling machine and the fuel handling machine have telescoping masts with integral grapples mounted from a trolley structure. They are also equipped with auxiliary hoists and jib cranes to which other grapples are attached when required. Both have redundant safety features and indicators that ensure positive engagement with fuel bundles. The fuel masts have interlocks that provide the necessary position control boundaries during deployment and limit travel during transfer of irradiated fuel. These safety provisions ensure that a safe water shielding depth is always maintained over the active fuel, while preventing physical damage to the mast.

The spent fuel cask pit is intentionally located outside the areas normally confined to fuel movement. The cask and other heavy loads are not permitted to encroach within any part of any spent fuel, spent fuel storage pool, or safety-related structure.

Inadvertent cask movement by the main crane over the fuel storage pools is prevented by travel limit controls. See Subsection 9.1.5.

#### ***9.1.4.2 System Description***

Table 9.1-6 is a listing of typical tools and servicing equipment. The following paragraphs describe the use of some of the major tools and equipment and address safety aspects of the design where applicable for light loads.

Subsection 9.1.4 provides the data that verifies the ESBWR Standard Plant light load handling systems and satisfies the guidelines of NUREG-0612.

#### ***9.1.4.3 Spent Fuel Cask***

Not in ESBWR Standard Plant Scope

#### ***9.1.4.4 Overhead Bridge Cranes***

##### **Fuel Building Crane**

The Fuel Building crane is classified as Seismic Category II to prevent structural failure with SSE loads, leading to degradation of the functioning of a Seismic Category I structure, system, or equipment to an unacceptable safety level. The crane is used for lifting large heavy components and tools up to and over the Fuel Building floor. The crane is also used during plant maintenance activities to move light loads such as inspection equipment consoles on the fuel building floor. The crane consists of two parallel girders along which the trolley traverses across its span. It is supported on its track on the fuel building wall structural columns.

Among its required light load lifting tasks during fuel handling is to lift the fuel bundle from the shipping container and place it in the new fuel inspections stand. It is also used to remove the channeled fuel assembly from the fuel inspection stand and place it in the fuel preparation machine.

The Fuel Building crane is required for lifting heavy components, fuel containers, fuel assemblies during inspection, and handling the fuel shipping cask. It also provides extensive support to general construction and operation activities in the fuel building. The principal design criteria for the fuel building crane is the same as the Reactor Building crane as contained in Subsection 9.1.5.

### **Reactor Building Crane**

The Reactor Building crane is classified as Seismic Category II to prevent structural failure with SSE loads, leading to degradation of the functioning of a Seismic Category I structure, system, or equipment to an unacceptable safety level. The crane is used for lifting large heavy components and tools up to and over the refueling floor. The crane is also used during plant maintenance activities to move light loads such as inspection equipment consoles on the fuel building floor. The crane consists of two parallel girders along which the trolley traverses across its span.

Among its required light load lifting tasks during plant operation is to handle small tools and equipment normally used during inspection and servicing activities.

During fuel transport, the main crane is also called upon to move and store pool gates. The principal design criteria for the Reactor Building crane are contained in Subsection 9.1.5.

#### ***9.1.4.5 Refueling Equipment***

### **Refueling Machine**

The refueling machine is located in the Reactor Building and is similar to a gantry style crane and is used to transport fuel and reactor components to and from buffer pool storage, the inclined fuel transfer system, and the reactor vessel. The machine spans the buffer pool on embedded tracks in the refueling floor. A telescoping mast and grapple suspended from a trolley system is used to lift and orient fuel assemblies for placement in the core or storage rack. Control of the machine is from an operator station on the refueling machine.

A position indicating system and travel limit computer is provided to locate the grapple over the vessel core and prevent collisions with pool obstacles. Two auxiliary hoists are provided for in-core servicing. The grapple in its retracted position provides sufficient water shielding over the active fuel during transit. The fuel grapple hoist has a redundant load path so that no single component failure will result in a fuel bundle drop. Interlocks on the machine;

- Prevent hoisting a fuel assembly over the vessel with a control rod removed;
- Prevent collision with fuel pool walls or other structures;
- Limit travel of the fuel grapple;
- Interlock grapple hook engagement with hoist load and hoist up power; and
- Ensure correct sequencing of the transfer operation in the automatic or manual mode.

The refueling machine is Seismic Class II. The refueling machine is designed to withstand the SSE. A standard dynamic analysis using the appropriate response spectra is performed to demonstrate compliance to design requirements. Except for hoisting speed, the fuel hoist is designed to meet the requirements of NUREG-0554, Single Failure Proof Cranes. An auxiliary hoist is designed to meet the requirements of NUREG-0612, Control of Heavy Loads at Nuclear

Power Plants to allow simultaneous handling of the control blade and fuel support casting with the dual function grapple. A second auxiliary hoist is provided for handling smaller lightweight tools.

### **Fuel Handling Machine**

The fuel handling machine is similar to a gantry style crane, and is used to transport fuel and reactor components to and from the inclined fuel transfer system and the spent fuel storage and equipment storage racks. It is also used to move spent fuel to the shipping cask. The machine spans the Spent Fuel Pool on embedded tracks in the refueling floor. A telescoping mast and grapple suspended from a trolley system is used to lift and orient fuel assemblies for placement in the cask or storage rack. Control of the machine is from an operator station on the fuel handling machine.

A position indicating system and travel limit computer is provided to locate the grapple over the spent fuel racks, IFTS, and prevent collisions with pool obstacles. An auxiliary hoist is provided for additional servicing. The grapple in its retracted position provides sufficient water shielding over the active fuel during transit. The fuel grapple hoist has a redundant load path so that no single component failure will result in a fuel bundle drop. Interlocks on the machine:

- Prevent collision with fuel pool walls or other structures;
- Limit travel of the fuel grapple;
- Interlock grapple hook engagement with hoist load and hoist up power; and
- Ensure correct sequencing of the transfer operation in the automatic or manual mode.

The fuel handling machine is Seismic Class II. The fuel handling machine is designed to withstand the SSE. A standard dynamic analysis using the appropriate response spectra is performed to demonstrate compliance to design requirements. Except for hoisting speed, the fuel hoist is design to meet the requirements of NUREG-0554, Single Failure Proof Cranes.

#### ***9.1.4.6 Fuel Servicing Equipment***

### **Fuel Prep Machine**

Two fuel preparation machines are mounted on the wall of the Spent Fuel Pool and are used to assist in the loading of new fuel into the spent fuel storage pool racks and for channeling and re-channeling of new and spent fuel assemblies. The machines are also used with fuel inspection fixtures to provide an underwater inspection capability.

Each fuel preparation machine consists of a work platform, a frame, and a movable carriage. The frame and movable carriage are located below the normal water level in the Spent Fuel Pool, thus providing a water shield for the fuel assemblies being handled. The fuel preparation machine carriage has an up-travel-stop to prevent raising fuel above the safe water shield level. The operator places assembled new fuel in the fuel preparation machine, the carriage is lowered and the fuel removed from the fuel preparation machine using the fuel handling machine.

### **New Fuel Inspection Stand**

The new fuel inspection stand is a vertical frame mounted in a pit that supports two fuel bundles contained in a mechanically driven inspection carriage. In the carriage the lower tie plate of each

fuel bundle rests on a bearing seat and at the top each fuel assembly is supported in a separate bearing assembly. The fuel assemblies can be individually rotated about their longitudinal axis to permit viewing all sides. The fuel channel is placed on the fuel bundle in the new fuel inspection stand.

To facilitate fuel inspection, the stand is set into an inspection pit designed to allow the carriage to be lowered and raised permitting eye level viewing by inspecting personnel on the refueling floor.

### **Channel Bolt Wrench**

The channel bolt wrench is a manually operated device approximately 3.76 m (12 ft) in overall length. The wrench is used for removing or installing the channel fastener assembly that attaches the channel to the fuel assembly while the fuel assembly is held in the fuel preparation machine. The channel bolt wrench socket mates and captures the channel fastener cap-screw.

### **Channel Handling Tool**

The channel handling tool is used in conjunction with the fuel preparation machine to install and transport fuel channels.

The tool is composed of a handling bail, lock/release knob, extension shaft, angle guides, and clamp arms that engage the fuel channel. The clamps are actuated (extended or retracted) by manually rotating a lock/release knob.

The channel handling tool is suspended by its bail from a spring balancer on the channel handling boom located on the Spent Fuel Pool periphery.

### **General Purpose Grapple**

The general purpose grapple performs many tasks and is primarily used on the auxiliary hoist of either the refueling or fuel handling machines. It is designed to fit a standard fuel bail, which is replicated on certain tooling for handling purposes. One example is handling the underwater vacuum cleaner.

The general purpose grapple, when utilizing an extension cable, is also attached to the auxiliary hook of the fuel building crane as the need arises for handling new fuel.

#### ***9.1.4.7 Servicing Aids***

General area underwater lights are provided with a suitable reflector for illumination.

Support brackets are furnished to support the lights in the reactor vessel to allow the light to be positioned over the area being serviced independent of the platform. Local area underwater lights are small diameter lights for additional illumination. Drop lights are used for illumination where needed.

Other servicing aids that are available include the following:

- A radiation-hardened portable underwater television camera for in-vessel inspection or observation of servicing activity.

- General purpose viewing aid in the form of a floatable plastic box that has brightly colored side panels for easy identification. Its purpose is to prevent surface motion of the water and alleviate image distortion of submerged objects.
- A portable, submersible underwater vacuum cleaner to help remove crud or miscellaneous loose matter from the reactor vessel or storage pools. The required pump and filter is also submersible for extended periods. The filter can be changed remotely and disposed of in standard containers after use for off-site burial.

#### **9.1.4.8 Reactor Vessel Servicing Equipment**

Table 9.1-4 lists the safety designation, QA requirements, and the seismic category. Lifting tools for lifting heavy loads are designed using a dual load path or designed for a safety factor of 10 or better with respect to the ultimate strength of the material used. NUREG-0612, Section 5.1.6 (3) Control of Heavy Loads at Nuclear Power Plants, applies to the head lifting load path. The following subparagraphs describe the equipment contained in the table.

#### **Reactor Vessel Service Tools**

These tools are used when the reactor is shut down and the reactor vessel head is being removed or reinstalled. Tools in this group are as follows:

- RPV Stud Tensioning System and RPV Head Strongback;
- Stud handling tool;
- Seal Surface Protector;
- Stud thread protector;
- Stud elongation measuring rod;
- Dial indicator elongation measuring device; and
- Head guide cap;

#### **Steam and DPV/IC Line Plugs**

Prior to refueling when the reactor water is at the refueling level, the main steam and DPV/IC line nozzles must be plugged to prevent outflow of water from the reactor.

The plugs are housed on the end of a spider structure and are lowered into the vessel and aligned with their respective nozzles, actuated into position, and then their pressurized seals are activated. The plug structure is constructed of corrosion resistant materials and has a factor of safety of 5 or greater.

#### **Head Support Pedestal**

Head support pedestals are mounted on the refueling floor for supporting the reactor vessel head and strongback/carousel during periods of reactor servicing.

The pedestals have dowel pins that engage three evenly spaced stud holes in the head flange. The flange surface rests on replaceable wear pads made of aluminum.

When resting on the pedestals, the head flange is sufficiently high above the floor to allow access to the seal surface for inspection and O-ring replacement.

The pedestal structure is a carbon steel weldment coated with an approved paint. It has a base with bolt holes for mounting it to the concrete floor.

A seismic analysis is used to determine the input forces imposed onto the pedestals and floor anchors. The structure is designed to withstand these calculated forces and meet the requirements of AISC.

### **Dryer and Chimney Head / Separator Strongback**

The dryer and chimney head / separator strongback is a lifting device used for transporting the steam dryer or the chimney head with the steam separators between the reactor vessel and the storage pools. The strongback structure has a hook box with two hook pins in the center for engagement with the RB crane sister hook. The strongback has a socket with a remotely operated pin on the end of each arm for engaging it to the four lift eyes on the steam dryer or shroud head. This strongback or a similar device is also used to remove the chimney partition assembly if this option is implemented to facilitate refueling operations.

The strongback has been designed such that one hook pin and one main beam of the cruciform is capable of carrying the total load (160-ton) and no single component failure could cause the load to drop or swing uncontrollably out of the Safety-Related level attitude. The strongback meets the requirements of NUREG-0612, Control of Heavy Loads at Nuclear Power Plants and ANSI-14.6, Standard for Special Lifting Devices.

### **Head Strongback/Tensioner**

The RPV head strongback stud tensioning system is an integrated piece of equipment consisting of a strongback, a four station rotating frame with stud tensioners, nut and washer handling tools, stud cleaning tools, a nut and washer rack, and service platform.

The strongback structure has a hook box with two hook pins in the center for engagement with the reactor service crane sister hook. Extending from the center section are arms to connect to the circular monorail. The four arms have a lift rod for engagement to the four lift lugs on the RPV head. The rotating frame is connected to the strongback arms and four additional arms equally spaced between the strongback arms. The rotating frame positions the four stations of the stud tensioning and nut and washer handling tools above the stud circle of the reactor vessel and serves to suspend stud tensioners and nut and washer handling devices. The nut and washer rack is attached to the strongback and surrounds the RPV flange. The head strongback rotating frame serves the following functions:

- Lifting of Vessel Head—the strongback, when suspended from the RB crane main hook, will transport the RPV head plus the rotating frame with all its attachments between the reactor vessel and storage on the pedestals.
- Tensioning of Vessel Head Closure—the strongback with rotating frame, when supported on the RPV head on the vessel, will carry four stations of stud Tensioners, nut and washer handling tools, its own weight, the strongback, storage of nuts, washers, and associated tools and equipment.
- Storage with RPV Head—the strongback with rotating frame, when stored with the RPV head holding pedestals, carries the same load as outlined in the second bullet above.

- Storage without RPV Head—during reactor operation, the strongback and rotating frame is stored on four separate pedestals.

The strongback, with its lifting components, is designed to meet NUREG-0612, Control of Heavy Loads at Nuclear Power Plants and ANSI-14.6 Standard for Special Lifting Devices. After completion of welding and before painting, the lifting assembly is proof load tested and all load-affected welds and lift pins are magnetic-particle inspected.

The steel structure is designed in accordance with the Manual of Steel Construction by AISC. Aluminum structures are designed in accordance with the Aluminum Construction Manual by the Aluminum Association.

The strongback is tested in accordance with American National Standard for Overhead Hoists ANSI B30.16, Paragraph 16-1.2.2.2, such that one hook pin and one main beam of the structure is capable of carrying the total load, and so that no single component failure will cause the load to drop or swing uncontrollably out of the Safety-Related level attitude. The ASME Boiler and Pressure Vessel Code, Section IX (Welder Qualification) is applied to all welded structures.

#### ***9.1.4.9 In-Vessel Servicing Equipment***

##### **Instrument Strongback**

The instrument strongback, when attached to the Reactor Building auxiliary hoist, is used for servicing the Local Power Range Monitor (LPRM), Startup Range Neutron Monitors (SRNM), and dry tubes when they require replacement. The strongback initially supports the dry tube into the vessel. The in-core dry tube is then decoupled from the strongback and is guided into place while being supported by the instrument handling tool.

##### **Instrument Handling Tool**

The instrument handling tool is attached to the refueling platform auxiliary hoist and is used for SRNM removing and installing PRNM fixed in-core dry tubes, as well as handling the SRNM dry tubes. Dry tubes are stored in the spent fuel storage pool.

#### ***9.1.4.10 Storage Equipment***

Specially designed equipment storage racks are provided. Additional storage equipment is listed in Table 9.1-6. For fuel storage racks description and fuel arrangement, refer to Subsections 9.1.1 and 9.1.2.

Defective fuel assemblies can be placed in special fuel storage containers and can be stored in the equipment storage rack; both are designed for the defective fuel. These may be used to isolate leaking or defective fuel while in the fuel pool and during shipping.

#### ***9.1.4.11 Under-Vessel Servicing Equipment***

The primary functions of the under-vessel servicing equipment are as follows:

- Servicing tools for maintenance of control rod drive motors packing and fine motion control rod drives (FMCRDs); and
- Work platform for installation during the construction phase.

Table 9.1-6 lists the equipment and tools required for servicing. Of the equipment listed, the equipment handling platform and the FMCRD handling equipment are powered electrically and pneumatically.

The FMCRD handling equipment is designed for the removal and installation of the fine motion control rod drives' major components from their housings. This equipment is used in conjunction with the equipment-handling platform. It is designed in accordance with OSHA-1910.179, and American Institute of Steel Construction, AISC standards.

The under vessel platform provides a working surface for equipment and personnel performing work in the under vessel area. It is a polar platform capable of rotating 360°.

The under vessel servicing equipment is used in conjunction with a rail system and various carts to transport FMCRD components from outside the containment to the under vessel area.

The in-core monitoring seal flushing equipment is designed to prevent leakage of primary coolant from in-core detector housings during detector replacement. It is designed to industrial codes and manufactured from corrosion-resistant material.

#### ***9.1.4.12 Fuel Transfer System***

The ESBWR is equipped with an Inclined Fuel Transfer System (IFTS). In general the arrangement of the IFTS (refer to Figure 9.1-2) consists of a terminus at the upper end in the Reactor Building buffer pool that allows the fuel to be tilted from a vertical position to an inclined position prior to transport to the Spent Fuel Pool. There is a means to lower the transport device (i.e., a carriage), means to seal off the top end of the transfer tube, and a control system to effect transfer. It has a lower terminus in the fuel building storage pool, and a means to tilt the fuel into a vertical position allowing it to be removed from the transport cart. There are controls contained in local control panels to effect transfer. There is a means to seal off the upper and lower end of the tube while allowing filling and venting of the tube.

There is sufficient redundancy and diversity in equipment and controls to prevent loss of load [carriage with fuel is released in an uncontrolled manner] and there are no modes of operation that allow simultaneous opening of any set of valves that could cause draining of water from the upper pool in an uncontrolled manner. The carriage and valves may be manually operated in the event of a power failure, to allow completion of the fuel transfer process.

The IFTS has sufficient cooling such that a freshly removed fuel assembly can remain in the IFTS until it is removed without damage to the fuel or excessive overheating.

The IFTS tubes and supporting structure can withstand an SSE without failure of the basic structure or compromising the integrity of adjacent equipment and structures. Therefore, the portion of the IFTS transfer tube assembly from where it interfaces with the upper fuel pool, the portion of the tube assembly extending through the building, the drain line connection, and the lower Spent Fuel Pool terminus equipment (tube, valve, support structure, and bellows) are designated as Nonsafety-Related and Seismic Category I. The remaining equipment is designated as Nonsafety-Related and Seismic Category NS.

The IFTS is anchored to the bottom of the Inclined Fuel Transfer pool in the buffer pool floor in the Reactor Building. The IFTS penetrates the Reactor Building at an angle down to the IFTS pit in the fuel storage pool in the Fuel Storage Building.

The IFTS terminates in a separate pit in the fuel storage pool. The lower terminus of the IFTS allows for thermal expansion (axial movement relative to the anchor point in the Reactor Building). The lower terminus allows for differential movement between the anchor point in the Reactor Building and the fuel pool terminus, and allows it to have rotational movement at the end of the tube relative to the anchor point in the Reactor Building. The lower end interfaces with the fuel storage pool with a bellows to seal between the transfer tube and the Spent Fuel Pool wall.

The IFTS carriage primarily handles nuclear fuel using a removable insert and control blades in a separate insert in the transfer cart. Other contaminated items may be moved in the carriage utilizing a suitable insert.

For radiation protection, personnel access into areas of high radiation or areas immediately adjacent to the IFTS is controlled. Access to any area adjacent to the transfer tube is controlled through a system of physical controls, interlocks and an annunciator. Specifically:

- Controls prevent personnel from inadvertently or unintentionally being left in those areas at the time the access doors are closed;
- During IFTS operation or shutdown, personnel are prevented from (a) either reactivating the IFTS while personnel are in a controlled maintenance area, or (b) entering a controlled IFTS maintenance area while irradiated fuel or components are in any part of the IFTS;
- Both an audible alarm and flashing red lights are provided both inside and outside any maintenance room to indicate IFTS operation;
- Radiation monitors with alarms are provided both inside and outside any maintenance area; and
- A key-lock system of key-locks in both the IFTS main operation panel and in the control room is provided to allow access to any IFTS maintenance area.

#### ***9.1.4.13 Refueling Operations***

The fuel servicing system outlined in this section includes the requirements for a refueling program and the provision for tools and equipment. A general plant refueling and servicing sequence diagram is shown on Figure 9.1-3. It depicts, in chronological order, each event and elapsed time estimates based on historical BWR fleet experience.

#### ***9.1.4.14 Arrival of Fuel at Reactor Site***

Fuel bundles are packed in metal containers and then are shipped in wooden crates. Prior to moving the fuel to the Reactor Building the inner metal shipping container is removed from the outer wooden shipping box. The inner metal shipping container is moved to the floor of the fuel building using the fuel building crane. The containers must be in a vertical position before removal of the fuel, which is usually accomplished by the combined use of the fuel building crane auxiliary hook and the new fuel inspection stand. The fuel is inspected and channels placed over the fuel bundle. The fuel assembly is then placed in the spent storage racks.

## **Channeling New Fuel**

Channeling of new fuel is performed on the new fuel inspection stand. With the aid of a channel handling tool, the channel is lowered over the fuel after inspection and bolted in place using a channel bolt wrench. The fuel is then transported using the fuel building crane auxiliary hook to the spent fuel storage pool, in preparation for eventual transfer to the Reactor Building for fueling the reactor.

### ***9.1.4.15 Reactor Preparation for Refueling***

The reactor is shut down and cooled down to reduce the reactor water temperature to under 50°C (120°F), so that open vessel work may take place.

## **Drywell Head Removal**

Following drain down of the reactor well and the completion of wall cleaning, the drywell head quick disconnect pins are removed and stored for reinsertion after replacement of the drywell head.

The drywell head is lifted by the Reactor Building crane and transported to its storage area on the refueling floor. The drywell seal protection is then installed before any other activity can proceed.

## **Reactor Bulkhead Servicing**

Following drywell head removal, penetration openings in the reactor bulkhead are sealed to prevent leakage from well flooding. The RPV head insulation frame is removed and placed in its storage area on the refueling floor.

## **Vessel Head Removal**

Removal of the reactor vessel head is performed with the Reactor Building crane, utilizing the RPV strongback. The RB crane and RPV strongback with the RPV head stud tensioning system is used to handle the RPV head and attachments. The strongback is designed so that no single component failure will cause the load to drop or swing uncontrollably out of the Safety-Related horizontal attitude. The RPV strongback and stud tensioning equipment is detached from the Reactor Building crane during stud de-tensioning or tensioning operations. Following stud de-tensioning operations, the Reactor Building crane is used to lift the RPV head using the previously mounted strongback with the tensioning system, nuts, and washers.

The maximum potential drop height is at the point where the head is lifted vertically from the vessel and before moving it horizontally to the head storage pedestals.

## **Dryer and Chimney Head / Separator Removal**

### **Fuel Pool Gate Removal**

In preparation for spent fuel movement out of the reactor core, the fuel pool and reactor well gates are removed after equalization of water levels between the fuel pool and reactor well.

### **Dryer Removal**

The dryer-separator strongback is lowered by the RB crane and attached to the dryer lifting lugs. The dryer is lifted from the reactor vessel and transported underwater to its storage location in the D/S pit adjacent to the reactor well.

### **Nozzle Plugs Installation**

Nozzle plug installation comprises plugging both the main steam and IC line nozzles. The installation is performed with the vessel flooded to just below the refueling floor level. Once installed, other outage tasks such as local leak rate testing, SRV or DPV replacement and main steam isolation valve (MSIV) maintenance activities can be initiated.

### **Chimney Head / Separator Removal**

Disengagement of all chimney head bolts from the chimney head/separator flanged joint is completed and verified prior to chimney head/separator removal. The chimney head/separator is removed together as an assembly with the lifting rods attached to the separator flange.

The reactor well is then flooded and the water level is raised to the same level as the equipment storage pool. This allows the removal of the pool gate to permit passage of the chimney head/separator assembly into the storage pool.

Lifting of the chimney head/separator assembly is affected by the chimney head/separator strongback suspended from the main crane. The lifting pins at the ends of four cruciform beams are pneumatically actuated into the lifting eyes on the separator lifting rods.

#### ***9.1.4.16 Refueling***

Fuel movement during the refueling process is carried out by the refueling machine, the IFTS, and the fuel handling machine. The move by the refueling and fuel handling machines may be in a direct path as the resultant of both x and y vectors. Accuracy of position in either horizontal direction can be achieved at each RPV core position and at each fuel storage rack cell location.

### **Core Verification and Final Core Audit**

The grapple camera is used to review the core location of each fuel bundle, its orientation and seating condition during each fuel move.

#### ***9.1.4.17 Vessel Closure***

Following refueling, all other reactor vessel related outage activities are completed before reassembly of the reactor can begin. These may include the following:

- FMCRD maintenance and tests;
- Neutron Monitoring System servicing;
- SV, SRV and DPV replacement; and
- MSIV maintenance.

The sequence of reassembly is Safety-Related in reverse order to that of disassembly but may have some steps performed in parallel. The sequence is as follows:

- Install fuel pool gate;
- Remove nozzle plugs;
- Remove and store equipment storage pool gate;
- Install steam dryer and chimney head / separator assemblies;

- Install equipment storage pool gate;
- Drain reactor cavity;
- Install and tighten reactor vessel head;
- Install reactor vessel insulation;
- Perform in-service leak test (ISLT - Equipment is tagged out and inoperable during this test, which is a critical path item);
- Remove tags and restore valve lineups;
- Install drywell head;
- Flood reactor cavity;
- Perform startup operations check; and
- Check final drywell closeout.

#### ***9.1.4.18 Safety Evaluation of Fuel Handling System***

Fuel servicing equipment is discussed in Subsection 9.1.4.6 and refueling equipment is discussed in Subsection 9.1.4.5. In addition, the summary safety evaluation of the fuel handling system is described in the following paragraphs.

The refueling machine and fuel handling machine are designed to prevent them from becoming unstable and toppling into pools during a SSE, and interlocks, as well as limit switches, are provided to prevent accidental movement of the grapple mast into pool walls.

The grapple on both the refueling machine and fuel handling machine is hoisted to its retracted position by redundant cables inside the mast and is lowered to full extension by gravity. The retracted position is controlled by both interlocks and physical stops to prevent raising the fuel assembly above the normal stop position required for safe handling of the fuel. The operator can observe the exact grapple position over the core by a display screen at the operator console.

These racks requires the submittal of information pertaining to load drop analysis. See Subsection 9.1.6 for COL license information requirements under “Spent Fuel Racks Load Drop Analysis.”

The fuel handling system complies with General Design Criterion 61 of 10 CFR 50 as described in Subsection 3.1.6.2.

#### ***9.1.4.19 Inspection and Testing Requirements***

##### **Inspection**

The fuel storage racks and refueling machine have additional “quality requirements” that identify features that require specific QA verification of compliance to drawing requirements.

##### **Testing**

Functional tests are performed on refueling and servicing in the shop prior to the shipment of most production units and generally include electrical tests, leak tests, and sequence of operations tests.

When the unit is received at the site, it is inspected to ensure that no damage has occurred during transit or storage. Prior to use and at periodic intervals, each piece of equipment is again tested to ensure that the electrical and/or mechanical functions are operational.

Passive components, such as the fuel storage racks, are inspected periodically. Procedures for interim verification of the presence of neutron absorbing material during the life of the plant are provided.

Fuel handling and vessel servicing equipment preoperational tests are described in Subsection 14.2.8.

#### ***9.1.4.20 Instrumentation Requirements***

##### **Refueling and Fuel Handling Machines**

The refueling and fuel handling machines have a position indicator system to indicate to the operator which core fuel cell the fuel grapple is accessing. Interlocks and a monitor are provided to prevent the fuel grapple from operating in a fuel cell where the control rod is not in the proper orientation for refueling.

There is also a series of mechanically activated switches and relays that provides monitor indications on the operator's console for grapple limits, hoist and cable load conditions, and confirmation that the grapple's hook is either engaged or released.

A series of load cells is installed to provide automatic shutdown whenever threshold limits are exceeded for either the fuel grapple or the auxiliary hoist units.

##### **Fuel Grapple**

Although the fuel grapple is not safety-related, it is equipped with a mounted TV camera, lighting system, and instrumentation system consisting of mechanical switches and indicator lights. This system provides the operator with a positive indication that the grapple is properly aligned and oriented and that the grappling mechanism is either extended or retracted.

#### **9.1.5 Overhead Heavy Load Handling Systems (OHLHS)**

##### ***9.1.5.1 Design Bases***

The objective for the OHLHS is to control the movement of heavy loads in order to ensure the safe handling of heavy loads, to reduce the potential for uncontrolled movement of heavy loads or load drops, and to limit the consequences of dropping a heavy load.

##### ***9.1.5.2 General***

The equipment described in this subsection covers items considered as heavy loads that are handled under conditions that mandate critical handling compliance.

Critical load handling conditions relate to the moving of loads, the use of equipment and the performance of operations, which, by inadvertent operation or equipment malfunctions, either separately or in combination, could cause:

- A release of radioactivity;
- A criticality accident;
- The inability to cool fuel within the reactor vessel or within the Spent Fuel Pool; and

- Prevent a safe shutdown of the reactor.

This includes risk assessments of spent fuel and of storage pool levels, cooling of fuel pool water, or new fuel criticality. Critical load handling therefore includes all components and equipment used for moving loads weighing more than one fuel assembly with its associated handling devices.

The Reactor and Fuel Building cranes provide a safe and effective means for transporting heavy loads including the handling of new and spent fuel, plant equipment, service tools and fuel casks. Safe handling includes design considerations for maintaining occupational radiation exposure as low as practicable during transportation and handling.

Where applicable, the appropriate seismic category, safety classification, ASME, ANSI, industrial and electrical codes have been identified (refer to Tables 9.1-4 and 9.1-5). The designs conform to the relevant requirements of General Design Criteria 2, 4, and 61 of 10 CFR 50, Appendix A.

The lifting capacity of each crane or hoist is designed to at least the maximum actual or anticipated weight of equipment and handling devices in a given area serviced. The hoists, cranes, or other lifting devices comply with the requirements of NRC Bulletin 96-02, NUREG-0554, ANSI N14.6, ANSI B30.9, ANSI B30.10 and NUREG-0612 Subsection 5.1.1(4) or 5.1.1(5). Cranes and hoists are also designed to criteria and guidelines of NUREG-0612 Subsection 5.1.1(7), ANSI B30.2 and CMAA-70 specifications for electrical overhead traveling cranes, including ANSI B30.11, and ANSI B30.16 as applicable.

#### ***9.1.5.3 Applicable Design Criteria for All OHLH Equipment***

All handling equipment subject to heavy loads handling criteria has ratings consistent with lifts required and the design loading will be visibly marked. Cranes/hoists or monorail hoists pass over the centers of gravity of heavy equipment that is to be lifted. In locations where a single monorail or crane handles several pieces of equipment, the routing is such that each transported piece passes clear of other parts.

Pendant control is required for the bridge, trolley, and auxiliary hoist to provide efficient handling of fuel shipping containers during receipt and also to handle fuel during new fuel inspection. The crane control system is selected considering the long lift required through the equipment hatch as well as the precise positioning requirements when handling the RPV and drywell heads, the RPV internals, and the RPV head stud tensioner assembly. The control system provides stepless regulated variable speed capability with high empty-hook speeds. Efficient handling of the drywell and RPV heads and stud tensioner assembly require that the control system provide spotting control. Because fuel shipping cask handling involves a long duration lift, low speed, and spotting control, thermal protection features are incorporated.

Heavy load equipment is also used to handle light loads and related fuel handling tasks. Therefore, much of the handling systems and related design, descriptions, operations, and service task information of Subsection 9.1.4 is applicable here. The cross-reference for the handling operations/equipment and Subsection 9.1.4 is provided in Table 9.1-6.

Transportation routing drawings are made covering the transportation route of every piece of heavy load removable equipment from its installed location to the appropriate service shop or

building exit. Routes will be arranged to prevent congestion and to assure safety while permitting a free flow of equipment being serviced. The frequency of transportation and usage of route are documented based on the predicted number of times usage either per year and/or per refueling or service outage.

#### ***9.1.5.4 System Description***

#### ***9.1.5.5 Fuel Building and Reactor Building Cranes***

##### **Fuel Building Crane**

The Fuel Building (FB) is a reinforced concrete structure enclosing the Spent Fuel Pool, cask handling and cleaning facility, and other equipment. The FB crane provides heavy load lifting capability for the Fuel Building floor. The main hook (160-ton capacity) is used to lift new fuel shipping containers and the spent fuel shipping cask (refer to Table 9.1-6). The orderly placement and movement paths of these components by the FB crane preclude transport of these heavy loads over the Spent Fuel Pool.

The FB crane is used during refueling/servicing as well as when the plant is on-line. Minimum crane coverage includes the FB floor laydown areas, cask wash down area, and the FB equipment hatch. During normal plant operation, the crane is used to handle new fuel shipping containers and the spent fuel shipping cask. The FB crane is interlocked to prevent movement of heavy loads over the Spent Fuel Pool.

##### **Reactor Building Crane**

The Reactor Building (RB) is a reinforced concrete structure enclosing the Reinforced Concrete Containment Vessel (RCCV), the refueling floor, the new fuel storage buffer pool, buffer pool deep pit pool for spent fuel storage, the dryer and chimney head/separator, and other equipment. The Reactor Building crane provides heavy load lifting capability for the refueling floor. The main hook (150-ton capacity) is used to lift the drywell head, RPV head insulation, RPV head, dryer, chimney head / separator strongback, and RPV head stud tensioning equipment (refer to Table 9.1-7). The orderly placement and movement paths of these components by the RB crane preclude transport of these heavy loads over the spent fuel racks in the deep pit buffer pool or over the new fuel rack.

The RB crane is used during refueling/servicing as well as when the plant is on-line. Minimum crane coverage includes the RPV for shield block removal and vessel servicing RB refueling floor lay down areas, RB equipment storage, refueling floor and the equipment hatches. The RB crane is interlocked to prevent movement of heavy loads over the fuel pools.

#### ***9.1.5.6 Other Overhead Load Handling System***

##### **Upper Drywell Servicing Equipment**

The upper drywell arrangement provides servicing access for the main steam isolation valves (MSIVs), feedwater isolation valves; safety/relief valves (SRVs), depressurization valves (DPVs), Passive Containment Cooling System (PCCS) valves, Gravity-Driven Cooling System (GDCCS) valves, and drywell cooling coils, fans and motors. Access to the space is from the RB through either the upper drywell personnel lock or equipment hatch. Equipment is removed through the upper drywell equipment hatch. Platforms are provided for servicing the feedwater

and main steam isolation valves, safety/relief valves, and drywell cooling equipment with the objective of reducing maintenance time and operator exposure. Items such as MSIVs, SRVs, DPVs, and feedwater isolation valves weigh in excess of a fuel assembly and its handling device and therefore are considered heavy loads.

Drywell maintenance activities are only performed during a plant outage, therefore, only the PCCS and GDCS piping and valves need to be protected from inadvertent load drops. Because the PCCS and GDCS piping and valves are spatially separated, an inadvertent load drop that breaks more than one pipe or valve of the PCCS or GDCS is not credible. In addition, a piping support structure and equipment platform separates and shields the PCCS and GDCS piping from heavy load transport paths.

This protection is such that no credible load drop can cause (a) a release of radioactivity, (b) a criticality accident, or (c) the inability to cool fuel within the reactor vessel or Spent Fuel Pool. Therefore, the upper drywell servicing equipment is not subject to the requirements of Subsections 9.1.5.2 and 9.1.5.3.

### **Lower Drywell Servicing Equipment**

The lower drywell arrangement provides for servicing, handling and transportation operations for FMCRDs. The lower drywell OHLHS consists of a rotating equipment service platform, chain hoists, FMCRD removal equipment, and other special purpose tools.

The rotating equipment platform provides a work surface under the reactor vessel to support the weight of personnel, tools, and equipment and to facilitate transportation moves and heavy load handling operations. The platform rotates 360° in either direction from its stored or “idle” position. The platform is designed to accommodate the maximum weight of the accumulation of tools and equipment plus a maximum sized crew. The weight of tools and equipment are specified in the interface control drawings for the equipment used in the lower drywell. Special hoists are provided in the lower drywell and Reactor Building to facilitate handling of these loads.

There are FMCRDs in the lower drywell that require servicing. There are two types of servicing operations: (1) replacement of the FMCRD drive mechanism and (2) motor and seal replacement. Separate servicing equipment is provided for each of these operations.

The FMCRD drive servicing equipment has its own mechanisms for rotating and raising FMCRD drive assemblies from a carrier on the equipment platform to their installed position. This servicing equipment interfaces with the lower drywell equipment platform and permits positioning under any one of the FMCRDs.

Separate equipment and a cart are provided for servicing FMCRD motors and seal assemblies and transporting them to the service shop located immediately outside the lower drywell equipment hatch.

There is no safety-related equipment below either component. Inadvertent load drops by the FMCRD servicing equipment cannot cause (1) a release of radioactivity, (2) a criticality accident, or (3) the inability to cool fuel within the reactor vessel or Spent Fuel Pool. Therefore, the FMCRD servicing equipment is not subject to the requirements of Subsections 9.1.5.2 and 9.1.5.3.

## **Main Steam Tunnel Servicing Equipment**

The main steam tunnel is a reinforced concrete structure surrounding the main steam lines and feedwater lines. The safety-related valve area of the main steam tunnel is located inside the Reactor Building. Access to the main steam tunnel is during a refueling/servicing outage. At this time, MSIVs or feedwater isolation valves and/or feedwater check valves may be removed using permanent overhead monorail type hoists. They are transported by monorail out of the steam tunnel and placed on the floor below a ceiling removal hatch. Valves are then lifted through the ceiling hatch by the valve service shop monorail. During shutdown, all of the piping and valves are not required to operate. Any load drop can only damage the other valves or piping within the main steam tunnel. Inadvertent load drops by the main steam tunnel servicing equipment cannot cause (a) a release of radioactivity, (b) a criticality accident, or (c) the inability to cool fuel within reactor vessel or Spent Fuel Pool. Therefore, the main steam tunnel servicing equipment is not subject to the requirements of Subsections 9.1.5.2 and 9.1.5.3.

## **Other Servicing Equipment**

Outside of the containment, the main steam tunnel, or the refueling floor there are no safety-related components of one division routed over any portion of a safety-related portion of another division at locations susceptible to heavy load drops capable of causing the loss of a safety-related component required to maintain the plant in a safe condition. Therefore, inadvertent load drops cannot cause (a) a release of radioactivity, (b) a criticality accident, (c) the inability to cool fuel within reactor vessel or Spent Fuel Pool, or (d) prevent the safe shutdown of the reactor. Therefore, the servicing equipment located outside the containment, the main steam tunnel, or the refueling floor is not subject to the requirements of Subsections 9.1.5.2 and 9.1.5.3.

### ***9.1.5.7 Equipment Operating Procedures Maintenance and Service***

Each item of equipment requiring servicing is described on an interface control diagram (ICD) delineating the space around the equipment required for servicing. This includes pull space for internal parts, access for tools, handling equipment, and alignment requirements. The ICD specifies the weights of large removable parts, shows the location of their centers of gravity, and describes installed lifting accommodations such as eyes and trunnions. An instruction manual describes maintenance procedures for each piece of equipment to be handled for servicing. Each manual contains suggestions for rigging and lifting of heavy parts, and identifies any special lifting or handling tools required.

Operating instruction and maintenance manuals are provided for reference and use by operations personnel for all major handling equipment components (cranes, hoist, etc.).

### ***9.1.5.8 Operational Responsibilities***

Critical heavy load handling in the plant includes the following key elements for the administration and implementation of heavy load handling systems:

- Heavy Load Handling System and Equipment Operating Procedure;s
- Heavy Load Handling Equipment Maintenance Procedures and/or Manuals;
- Heavy Load Handling Equipment Inspection and Test Plans; NDE, Visual, etc.;
- Heavy Load Handling Safe Load Paths and Routing Plans;

- QA Program to Monitor and Assure Implementation and Compliance of Heavy Load Handling Operations and Controls; and
- Personnel Qualifications, Training and Control Program.

#### **9.1.5.9 Safety Evaluations**

The arrangement of the refueling floor precludes transporting heavy loads, other than spent fuel handled by the refueling machine or fuel handling machine, over spent fuel stored in the spent fuel storage pool.

The separation (arrangement, equipment interlocks, and routing) of redundant safety-related components in relation to heavy load paths minimizes the potential to cause failure of safety-related components. Administrative procedures further minimize the potential hazard from heavy loads.

#### **9.1.5.10 Inspection and Testing**

Qualification load and performance testing, including nondestructive examination (NDE) and dimensional inspection on heavy load handling equipment, is performed. Tests may include load capacity, safety overloads, life cycle, sequence of operations, and functional performance.

When load handling equipment is received at the site, it is inspected to ensure no damage has occurred during transit or storage. Prior to use and at periodic intervals, each piece of equipment is tested again to ensure the electrical and/or mechanical functions are operational including visual inspection and, if required, NDE inspection.

Crane inspections and testing comply with ANSI B30.2.

#### **9.1.5.11 Instrumentation Requirements**

The majority of the heavy load handling equipment is manually operated and controlled by the operator based on visual observations. This type of operation does not necessitate the need for a dynamic instrumentation system.

Load cells may be installed to provide automatic shutdown whenever threshold limits are exceeded for critical load handling operations to prevent overloading.

### **9.1.6 COL Information**

#### **Dynamic and Impact Analyses of Fuel Storage Racks**

The COL Holder shall provide the NRC confirmatory dynamic and impact analyses of the fuel storage racks. Refer to Subsections 9.1.1.1, and 9.1.2.4, under subheading Dynamic and Impact Analysis.

The COL Holder shall confirm the fuel storage racks are designed to provide sufficient natural convection coolant flow through the rack and fuel to remove decay heat without reaching excessive water temperatures (100°C; 212°F), refer to Subsections 9.1.1.1 and 9.1.2.5.

#### **Fuel Storage Racks Criticality Analysis**

The COL Holder shall provide the NRC confirmatory criticality analysis as required by *Criticality Control* refer to Subsections 9.1.1.1 and 9.1.1.2).

### **Fuel Racks Load Drop Analysis**

The COL Holder shall provide the NRC confirmatory load drop analysis as required by Subsection 9.1.2.4.

### **Handling of Light Loads (Fuel Handling)**

The COL Holder shall provide the NRC the following for confirmatory review:

- Fuel handling procedures.
- Maintenance manuals and procedures for equipment used to move fuel.
- Equipment inspection and test plans for equipment used to move fuel.
- Personnel qualifications, training, and control programs for fuel handling personnel.
- QA programs to monitor, implement, and assure compliance to fuel handling operations.

### **Handling of Heavy Loads**

The COL Holder shall provide the NRC the following for confirmatory review:

- A listing of all heavy loads, heavy load handling equipment, and their associated heavy load attributes;
- Heavy load handling safe load paths and routing plans including descriptions of automatic and manual interlocks and safety devices and procedures to assure safe load path compliance;
- Heavy load handling equipment maintenance manuals and procedures;
- Heavy load handling equipment inspection and test plans;
- Heavy load personnel qualifications, training, and control programs; and
- QA programs to monitor, implement, and assure compliance to heavy load handling operations.

### **9.1.7 References**

None.

**Table 9.1-1**  
**Pools Served by FAPCS and IC/PCC**

<b>Pools Served by FAPCS C/C Subsystem</b>	<b>Location</b>
Fuel Pools <ul style="list-style-type: none"> <li>• Spent Fuel Pool</li> <li>• Lower Fuel Transfer Pool</li> <li>• Cask Pool</li> </ul>	Fuel Building
Auxiliary Pools <ul style="list-style-type: none"> <li>• Steam Dryer and Separator Storage Pool</li> <li>• Reactor Well</li> <li>• Buffer Pool</li> <li>• Upper Fuel Transfer Pool</li> </ul>	Reactor Building
GDCS Pools <ul style="list-style-type: none"> <li>• GDCS A pool</li> <li>• GDCS B/C pool</li> <li>• GDCS D pool</li> </ul>	Containment
Suppression Pool	Containment
<b>Pools Served by IC/PCCS Pool C/C Subsystem</b>	
IC/PCCS Pools <ul style="list-style-type: none"> <li>• IC Pools</li> <li>• PCCS Pools</li> <li>• Expansion (Outer) Pools</li> </ul>	Reactor Building

**Table 9.1-2**  
**FAPCS Operating Modes**

Mode	Description	Plant Condition
1	Spent Fuel Pool Cooling and Cleanup	a. Normal* b. Refueling c. Post accident**
2	Fuel and Auxiliary Pool Cooling and Cleanup	Refueling
3	IC/PCCS Pool Cooling and Cleanup	Normal*
4	GDCS Pool Cooling and Cleanup	Normal*
5	Suppression Pool Cooling and Cleanup	a. Normal* b. Post accident**
6	LPCI	Post accident**
7	Drywell Spray	Post accident**
8	IC/PCCS pool automatic makeup	Normal*
9	Reactor Well fill up	Refueling
10	Reactor Well Draining	Refueling
11	Overboarding of Suppression Pool Water	Refueling
12	Post-LOCA fill-up of Spent Fuel Pool and IC/PCC pools	Post accident**
13	Alternate Shutdown Cooling (SDC)	Post accident**

\* Normal plant operation including reactor power operation and plant shutdown.

\*\* Post accident recovery period starting at 72 hours after the accident. Cleanup function is not required during this period.

**Table 9.1-3  
Safety Classification, Quality Group and Seismic Category**

<b>Component</b>	<b>Safety Class</b>	<b>Quality Group</b>	<b>Seismic Category</b>
1. Piping between inboard and outboard containment isolation valves for Suppression pool return line GDCS pool suction line GDCS pool return line Drywell Spray discharge line	2	B	I
2. Piping between inboard manual valve and 2 <sup>nd</sup> outboard containment isolation valve on suppression pool suction line, as well as the LPCI piping between the RWCU/SDC interface and the second isolation valve	2	B	I
3. Piping and components providing dedicated emergency make-up water to the IC/PCCS pools and Spent Fuel Pool from piping connections located at grade level in the reactor yard and Fire Protection System	3	C	I
4. Interconnecting pipes between GDCS pools	3	C	I
5. Piping and components outside containment required for SFP Cooling, SPC, LPCI and Drywell Spray modes of operation including skimmer lines and all components of the cooling and cleanup trains.	N	B	II
6. Suppression pool suction line inside containment between the inboard manual valve and its termination point (including suction strainers)	N	C	I
7. Piping inside containment between the inboard containment isolation valve and its termination point for Suppression pool return line Drywell spray discharge line	N	C	I
8. Piping inside containment between inboard containment isolation valves and their termination points inside containment for: GDCS pool suction line GDCS pool return line	N	D	II
9. IC/PCCS Pool C/C subsystem piping	N	D	II

**Table 9.1-3**  
**Safety Classification, Quality Group and Seismic Category (continued)**

<b>Component</b>	<b>Safety Class</b>	<b>Quality Group</b>	<b>Seismic Category</b>
10. Auxiliary pools skimmer lines and auxiliary pools return lines between the isolation valves and terminating points, and all piping and components associated with pool liner leak detection	N	D	NS
11. Instrument Sensing lines for the following parameters: IC/PCCS pool water level Spent Fuel Pool water level.	3	C/D*	I
12. Electrical Module and cables with safety-related function (containment isolation, LPCI isolation)	3	NA	I
13. Electrical Module and cables with Nonsafety-Related function	N	NA	II
14. Control and Instrumentation required for fuel pool cooling. SPC and Containment Spray modes of operation	N	NA	I
15. All other FAPCS instrumentation and controls are classified as non-Class 1E	N	NA	II

\* - applicable per instrument sensing line size

**Table 9.1-4  
Classification of Equipment**

<b>Principal Component</b>	<b>Safety Class</b>	<b>Location</b>	<b>QA Requirement</b>	<b>Seismic Category</b>	<b>Notes</b>
Fuel Servicing Equipment 1. Fuel Preparation machine 2. All Other Equipment	N N	FB/RB FB/RB	E E	II NS	
Miscellaneous Servicing Equipment	N	FB/RB	E	NS	
RPV Servicing Equipment	N	RB	E	NS/I	
RPV Internal Servicing Equipment	N	RB	E	NS	
Refueling Equipment 1. Fuel Handling machine 2. Refueling Machine 3. Refueling Bellows	N N N	FB RB CV	E E E	II II NS	

**Table 9.1-4  
Classification of Equipment (continued)**

<b>Principal Component</b>	<b>Safety Class</b>	<b>Location</b>	<b>QA Requirement</b>	<b>Seismic Category</b>	<b>Notes</b>
Fuel Storage Facility 1. Fuel Storage Racks (new and spent)	N	FB/RB	E	I	
Under RPV Servicing Equipment	N	CV	E	NS	
CRD Maintenance Facility	N	RB	E	NS	
Fuel Cask Cleaning Facility	N	FB	E	NS	
Fuel Transfer System	N	RB	E	I/NS	

**Table 9.1-5  
Reference Codes and Standards**

<b>Number</b>	<b>Title</b>	<b>Device</b>
ANS-N14.6	Standard for Special Lifting Devices for Shipping Containers Weighing (5 tons) or More for Nuclear Materials	Applicable to any item carrying a heavy load such as the Reactor Building (RB) and Fuel Building (FB) overhead cranes and the refueling and fuel handling machine
ANSI B30.9	Slings	Applicable to the RPV dryer strongback slings.
ANSI B30.10	Hooks	Applicable to the RB and FB overhead cranes.
ANSI B30.2	Performance Standards for Overhead Electric Overhead Traveling Cranes	Applicable to the RB and FB overhead cranes.
ANSI B30.16	Performance Standards for Air Wire Rope Hoists	Applicable to the RB and FB overhead cranes.
ANSI B30.11	Overhead and Gantry Crane	Applicable to the RB and FB overhead cranes.
CMAA70	Specifications for Electric Overhead Traveling Cranes	Applicable to the RB and FB overhead cranes and the refueling and fuel handling machines.
NUREG-0612	Control of Heavy Loads at Nuclear Power Plants	Applicable to the RB and FB overhead cranes. A portion of the NUREG is applicable to the RPV strongback or dryer strongback interface with the lifting device. Applicable to the hoist on the refueling and fuel handling machines that handles the combined fuel support and control blade grapple. Typically the combined weight of the equipment and tooling exceeds the weight of a fuel assembly.
NUREG-0554	Single Failure Proof Cranes for Nuclear Power Plants	Applicable to the RB and FB overhead cranes. Applicable to the hoist on the refueling and fuel handling machines that handles the combined fuel support and control blade grapple. Typically the combined weight of the equipment and tooling exceeds the weight of a fuel assembly.

Table 9.1-6

**Heavy Load Equipment Used to Handle Light Loads and Related Refueling Handling Tasks**

<b>Handling Operations/Equipment</b>	<b>Applicable Light Load Handling Subsection</b>
Reactor Building Crane	9.1.4.2.2
Fuel Building Crane	9.1.4.2.3
Reactor Vessel Servicing Equipment	9.1.4.2.5
Steam Line Plugs	
Dryer/Separator Strongback	
Head Strongback/Carousel	
In Vessel Servicing Equipment	9.1.4.2.6
Refueling Equipment	9.1.4.2.7- 9.1.4.2.10
Refueling Machine	
Fuel Handling Service Tasks	
Reactor Shutdown Handling Tasks	
Drywell Head Removal	
Reactor Well Servicing	
Reactor Vessel Head Removal	
Dryer and Separator Removal	
Fuel Handling Machine	
Vessel Closure	

**Table 9.1-7**  
**Summary of Heavy Load Operations**

<b>Hardware Handling Tasks</b>	<b>Handling System</b>	<b>Handling Equipment</b>	<b>Location</b>
<b>RPV OPENING/CLOSING OPERATIONS</b>			
Pool Seal Gates Removal, reinstallation and storage	RB	RB Crane Main or Auxiliary Hoist, Slings and Strongbacks	RF D/SP FSP
Drywell Head Removal, storage and reinstallation	RB	RB Crane Main Hoist, Drywell Head Strongbacks	RF R/W
Reactor Vessel Head Insulation Removal, storage and reinstallation	RB	RB Crane Main Hoist Lifting Sling	RF R/W
Reactor Vessel Head Removal, storage and reinstallation, includes handling stud tensioners studs, nuts, Automatic Tensioning Machine	RB	RB Crane Main Hoist Auxiliary Hoist Automatic Tensioning Machine RPV Head support Pedestal	RF RW
Steam Dryer/Separator Removal, storage and reinstallation. Includes unbolting Separator/Chimney flange joint bolts from Refueling Machine	RB	RB Crane Main Hoist Dryer/Separator Strongback	RW D/SP IRV
Steam & DPV Nozzle Plugs Remote Machine Installation and removal	RB	RB Crane Auxiliary Hoist	RF IRV

**Table 9.1-7**  
**Summary of Heavy Load Operations (continued)**

<b>Hardware Handling Tasks</b>	<b>Handling System</b>	<b>Handling Equipment</b>	<b>Location</b>
<b>REFUELING OPERATIONS:</b>			
New Fuel:			
Receive at Grade and receipt inspect. Lift to FB floor after removing outer container.	FB	FB Crane Auxiliary Hoist	FB G/F
Remove from inner container and move fuel to new fuel inspection stand. Handle channel and place channel on new fuel bundle.	FB	FB Crane Auxiliary Hoist	NFS NFI
Move new fuel assembly from inspection stand and move load to storage pool and place in spent fuel storage racks.	FB	FB Crane Auxiliary Hoist Fuel Preparation Machine Fuel Handling Machine Auxiliary Hoists Fuel Grapples	NFS FPM FHM RF RVC
Move from pool racks and place in IFTS. Transfer fuel to Reactor Building.	FB/RB	Inclined Fuel Transfer Machine	
Spent Fuel:			
Store new fuel, refuel reactor, and remove spent fuel.	RB	Refueling Machine Auxiliary Hoists Fuel Grapple	RW FSP FCF
Fuel Cask:			
Receive, inspect, lift to fuel building floor. Lower into cask pit. Remove closure head. Move spent fuel to cask pit. Replace closure head and wash cask. Move cask to G/F for shipment.	FB	FB Crane Main Hoist Auxiliary Hoist Fuel Handling Machine Auxiliary Hoists Fuel Grapple	G/F RF FLP

**Table 9.1-7**  
**Summary of Heavy Load Operations (continued)**

<b>Hardware Handling Tasks</b>	<b>Handling System</b>	<b>Handling Equipment</b>	<b>Location</b>
<b>REACTOR SERVICE OPERATIONS:</b>			
Control Rod Blades Replacement including moving adjacent fuel bundles, and blade guide removal and installation.	RB	Refueling Machine Auxiliary Hoists Fuel Grapple Control Rod Grapple	RVC RV
Fuel Support and Guide Tube (FSGT) (Non-routine) Removal and Replacement.	RB	Refueling Platform Auxiliary Hoists	RVC
<b>UPPER DRYWELL AND MAIN STEAM TUNNEL SERVICING</b>			
MSIVs, DPVs, and SRVs Servicing: SRV removal, installation, and transportation for repair and calibrations from installed location to RCCV entrance and up to special service room area and return.	UDS SRM	Monorail for servicing MSIVs, DPVs and SRVs Monorail Hoist Transportation Cart Hatchway Hoist Wall Mount	UDW RB RB SRM
	MSS	Steam Tunnel Crane Hoist Transportation Cart Hatchway Hoist Wall Mount	MST  SRM

- LEGEND**
- LOW PRESSURE COOLANT INJECTION
  - HIGH PRESSURE COOLING SYSTEM
  - REACTOR COMPONENT COOLING
  - WATER SYSTEM
  - IC --- ISOLATION CONTAINMENT COOLING
  - PCCS --- SYSTEM CONTAINMENT COOLING
  - MWS --- MAKE UP WATER SYSTEM
  - CS&TS --- CONDENSATE STORAGE AND SHUTDOWN COOLING
  - RWCUISDC --- REACTOR WATER CLEANUP/SHUTDOWN COOLING
  - RPV --- REACTOR PRESSURE VESSEL
  - REACTOR PRESSURE VESSEL
  - REACTOR PRESSURE VESSEL
  - LWS --- LIQUID WASTE SYSTEM
  - B, C, D --- QUALITY GROUP B, C, D
  - NS --- NON-SEISMIC

- --- MOTOR OPERATOR
- --- NITROGEN OPERATOR
- --- AIR OPERATOR
- --- FLOW ELEMENT

- NOTES:**
1. EQUIPMENT AND INSTRUMENT IDENTIFICATIONS ARE PREFIXED BY ACD UNLESS OTHERWISE NOTED.
  2. VALVES OTHERWISE NOTED, ALL VALVES AND EQUIPMENT OF THE FAPCS ARE NON-SEISMIC UNLESS OTHERWISE NOTED. VALVES AND EQUIPMENT ARE NON-SEISMIC UNLESS OTHERWISE NOTED. VALVES AND EQUIPMENT ARE NON-SEISMIC UNLESS OTHERWISE NOTED. VALVES AND EQUIPMENT ARE NON-SEISMIC UNLESS OTHERWISE NOTED.
  3. OUTBOARD CONTAINMENT ISOLATION VALVES SHALL BE LOCATED AS CLOSE AS PRACTICAL TO THE CONTAINMENT PENETRATION.
  4. ALL MOTOR OPERATED VALVES ARE AC UNLESS OTHERWISE NOTED.
  5. PIPING HIGH POINT VENTS AND LOW POINT DRAINS ARE TO BE ADDED AT ALL SUCH HIGH OR LOW POINTS NOT SERVED BY EQUIPMENT VENTS AND DRAINS.
  6. VACUUM BREAKERS TO BE 1" NOMINAL SIZE AT LOCATIONS TO BE DETERMINED DURING DETAILED ENG PHASE.
  7. ANTI-SIPHONING PROVISION SHALL BE LOCATED AT THE DRAW DOWN WATER LEVEL OF THE SUPPRESSION POOL.
  8. PIPING ARRANGEMENT SHALL BE SUCH THAT ADEQUATE MIXING OF SUPPLEMENTARY WATER SHALL BE ASSURED. SUPPLEMENTARY WATER SHALL BE LOCATED 180 DEGREES APART. INTAKE STRAINERS SHALL BE CONSTRUCTED IN SUCH A WAY THAT INTAKE FROM THE UPPER REGION AS WELL AS THE LOWER REGION OF THE SP WATER TAKES PLACE.
  9. THE CONTAINMENT PENETRATION OF THE GDCCS SUCTION LINE SHALL BE LOCATED BELOW THE SUCTION INLET FROM THE POOL.

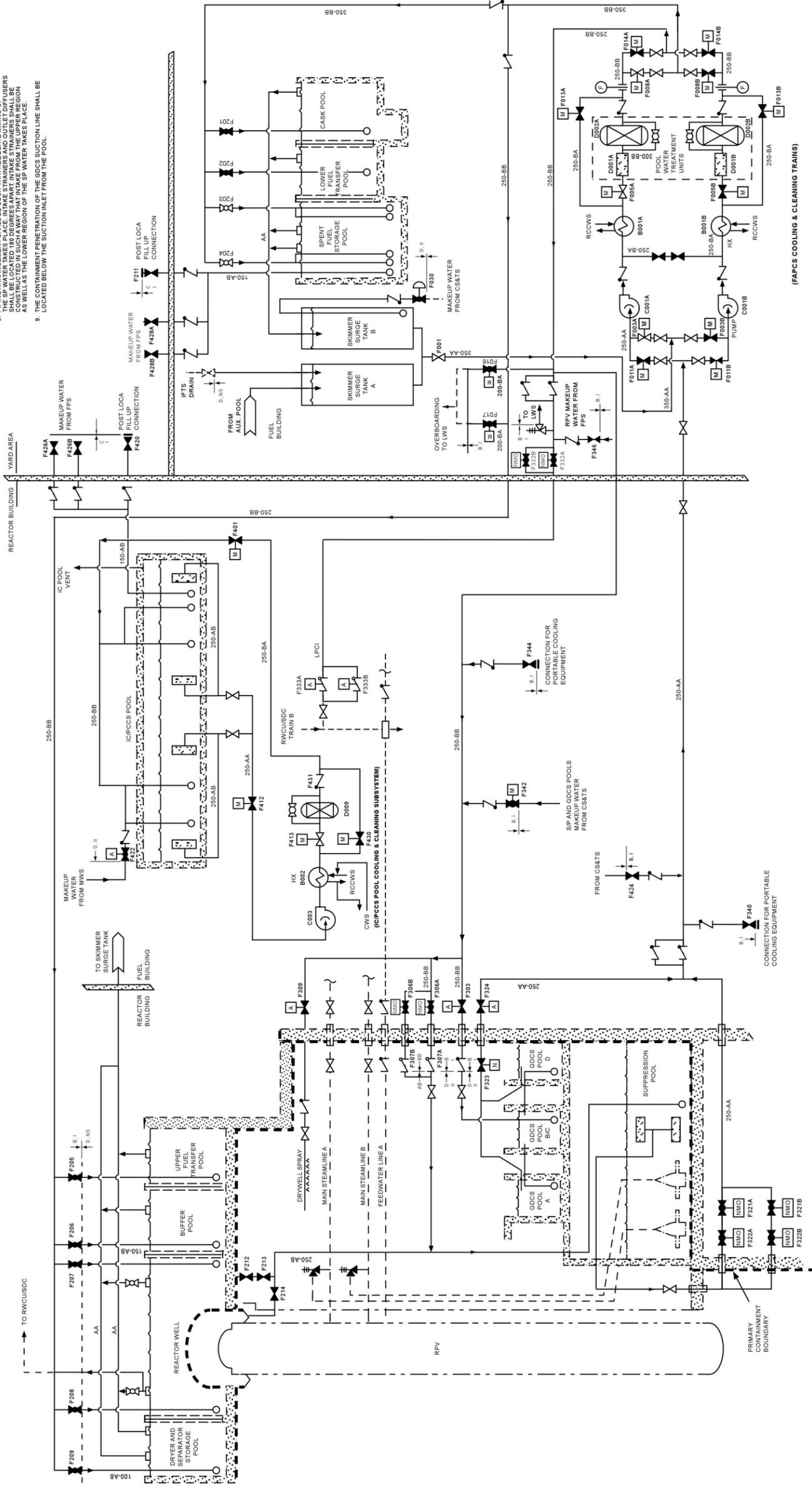


Figure 9.1-1. FAPCS Schematic Diagram

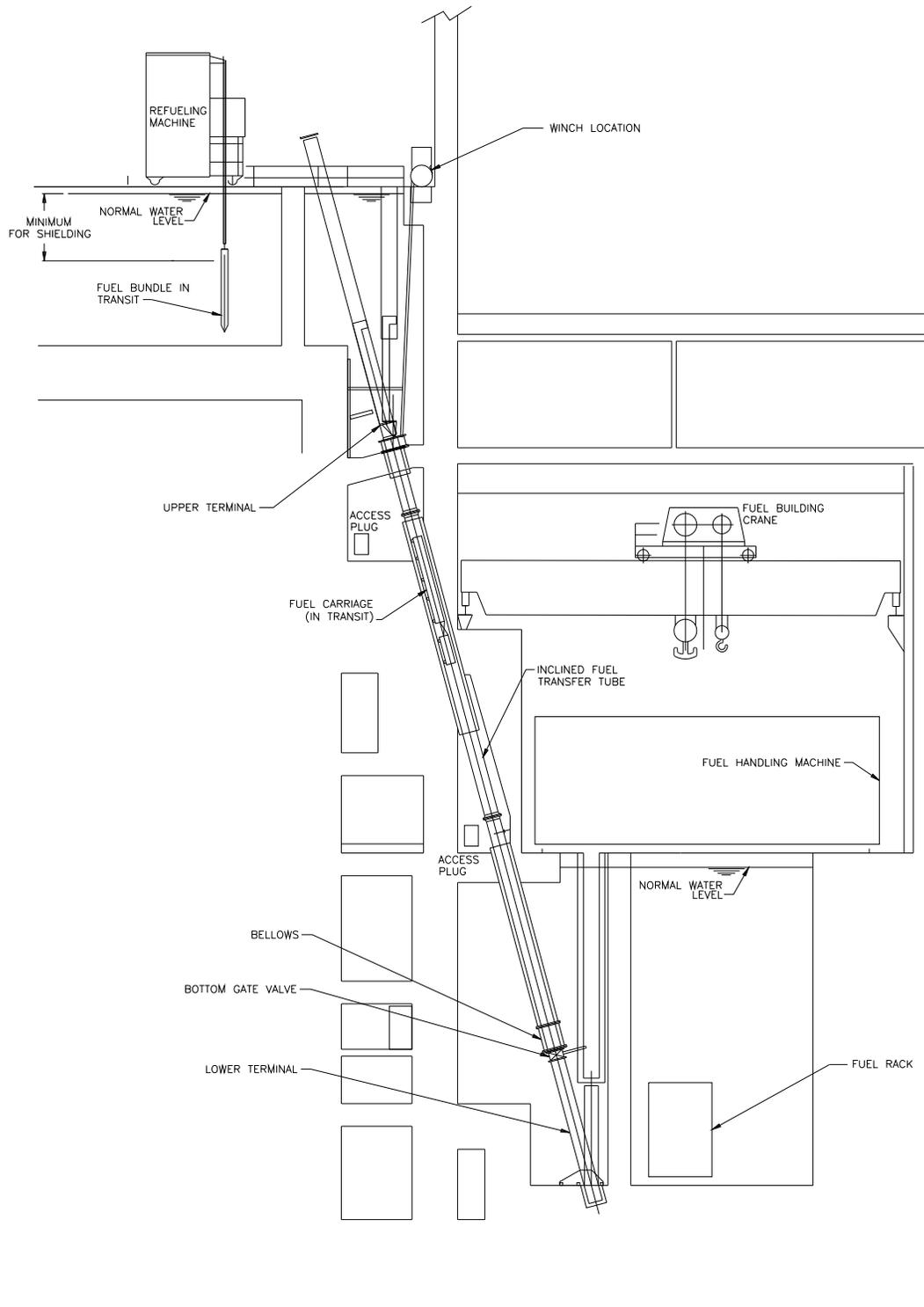


Figure 9.1-2. Inclined Fuel Transfer System



## 9.2 WATER SYSTEMS

### 9.2.1 Plant Service Water System

#### 9.2.1.1 *Design Bases*

##### **Safety (10 CFR 50.2) Design Bases**

The Plant Service Water System (PSWS) does not perform any safety-related function. There is no interface with any safety-related component.

The PSWS has Regulatory Treatment of Non-Safety Systems (RTNSS) functions to provide post 72-hour cooling to the RCCWS and TCCWS. Performance of RTNSS functions is assured by applying requirements for redundant trains, physical and electrical separation of trains, seismic requirements, and ability to withstand category 5 hurricane missiles and flood protection. Appendix 19A provides the level of oversight and additional requirements to meet the RTNSS functions.

The PSWS meets the requirements of GDC 2 as it pertains to Position C.2 of Reg. Guide 1.29. The PSWS also meets the intent of GDC 2 as it pertains to Position C.1 of Reg. Guide 1.29.

The PSWS meets GDC 4 with respect to dynamic effects associated with water hammer. The PSWS is vented at components and high points vents and operation and maintenance procedures are used to assure sufficient measures are taken to avoid water hammer.

The PSWS meets GDC 5 for shared systems and components important to safety. The PSWS Standard Plant design does not share any SSC with any other unit.

The ESBWR PSWS meets the acceptance criteria of GDC's 44, 45, and 46 by providing the following design considerations:

- Capable of transferring heat loads from SSC's to a heat sink under normal and accident conditions;
- Component redundancy so the system will remain functional assuming a single failure coincident with a loss of offsite power;
- Capability to isolate components or piping so system function will not be compromised; and
- Design provisions to permit inspection and operational testing of components and equipment.

##### **Power Generation Design Bases**

The PSWS provides 100% of the required cooling to the Reactor Component Cooling Water System (RCCWS) and Turbine Component Cooling Water System (TCCWS) heat exchangers.

The PSWS is designed so that neither a single active nor single passive component failure results in a complete loss of nuclear island cooling and/or plant dependence on any safety-related system. This is achieved by redundant components, automatic valves and piping cross-connects for increased reliability.

The PSWS is designed for ease of restoration of its function after a component failure without plant operating mode or power level change.

The PSWS is designed to operate during a Loss of Preferred Power (LOPP).

The PSWS is designed for remote operation from the main control room (MCR).

The RCCWS utilizes plate type heat exchangers to mitigate cross-contamination of either RCCWS or PSWS.

### ***9.2.1.2 System Description***

#### **Summary Description**

The PSWS rejects heat from Nonsafety-Related components in the reactor and turbine buildings to the environment. The source of cooling water to the PSWS is from either the normal power heat sink (NPHS) or the auxiliary heat sink (AHS), while the heat removed is rejected to either the NPHS or to the AHS. The portions of PSWS that are not a part of the ESBWR Standard Plant consist of the heat rejection facilities (NPHS and AHS), which are dependent on actual site conditions. The reference design utilizes a natural draft cooling tower for the NPHS and mechanical draft cooling towers for the AHS.

A simplified diagram of the PSWS is shown in Figure 9.2-1.

#### **Detailed System Description**

The PSWS consists of two independent and 100% redundant trains that continuously circulate water through the RCCWS and TCCWS heat exchangers.

Each PSWS train consists of two 50% capacity vertical pumps taking suction in parallel from a plant service water (PSW) basin. Discharge is through a check valve, a self-cleaning duplex strainer, and a motorized discharge valve at each pump to a common header. Each common header supplies PSW to each RCCWS and TCCWS heat exchanger train arranged in parallel. The PSW is returned via a common header to the mechanical draft cooling tower (AHS) or to the natural draft cooling tower (NPHS) in each train. Remotely-operated isolation valves and a crosstie line permit routing of the PSW to either heat sink. The RCCWS and TCCWS heat exchangers are provided with isolation valves for remote operation. Manual balancing valves are provided at each heat exchanger outlet.

The PSWS pumps are located at the plant service water basins. Each pump is sized for 50% of the train flow requirement for normal operation. The pumps are low speed, vertical wet-pit designs with allowance for increase in system friction loss and impeller wear. Basin water level is monitored to ensure sufficient NPSH at design flow is provided to the PSWS pumps.

Normally, the pumps in each train are powered from redundant electrical buses. During a LOPP, the pumps are powered from the two Nonsafety-Related standby diesel-generators.

Valves are provided with hard seats to withstand erosion. The valves are arranged for ease of maintenance, repair, and in-service inspection. During a LOPP, the motor-operated valves are powered from the two Nonsafety-Related standby diesel-generators.

The AHS provided for each PSWS train is a separate, multi-celled mechanical draft cooling tower with 50% of the cell fans supplied by one of the redundant electrical buses. During a LOPP, the fans are powered from the two Nonsafety-Related standby diesel-generators. The adjustable-speed, reversible motor fan units can be controlled for cold weather conditions to prevent freezing in the basin. A full flow bypass is provided to return water directly to the PSWS basin to allow ease of cold weather startup. The mechanical and electrical isolation of the cooling towers allows maintenance, including complete disassembly, during full power operation. The Station Water System provides makeup for blowdown, drift, and evaporation losses from the basin. Refer to Subsection 9.2.10 for Station Water System discussion. The COL Applicant will determine material selection and provide provisions to preclude long-term corrosion and fouling of the PSWS based on site water quality analysis.

In the event of a LOPP, the PSWS supports the RCCWS in bringing the plant to cold shutdown condition in 36 hours assuming the most limiting single active failure.

The ESBWR Standard Plant PSWS design heat loads are shown in Table 9.2-1. The PSWS component design characteristics are shown in Table 9.2-2.

The PSWS design permits detection and alarming in the MCR of any potential gross leakage and permits the isolation of any such leak in a sufficiently short period of time to preclude extensive plant damage.

Means are provided to detect leakage into the PSWS from the RCCWS, which may contain low levels of radioactivity.

The potential for water hammer is mitigated through the use of various system design and layout features, such as automatic air release/vacuum valves installed at high points in system piping and at the pump discharge, proper valve actuation times to minimize water hammer, procedural requirements ensuring proper line filling prior to system operation and after maintenance operations, and the use of check valves at pump discharge to prevent backflow into the pump.

### **Operation**

The PSWS operates during startup, normal power operation, hot standby, cooldown, shutdown/refueling, and LOPP.

During normal power operation, two of the four 50% capacity PSWS pumps are in operation to pump water from the plant service water basin through the PSWS. Heat removed from the RCCWS and TCCWS can be rejected to the main cooling tower or to the PSWS cooling tower.

Operation of any two of the four PSWS pumps is sufficient for the design heat load removal in any normal operating mode with the exception of the normal cooldown mode, when three pumps are initially required.

During a LOPP, the running PSWS pumps restart automatically using power supplied by the Nonsafety-Related standby diesel-generators.

#### **9.2.1.3 Safety Evaluation**

The PSWS has no safety-related function. Failure of the system does not compromise any safety-related system or component, nor does it prevent safe shutdown of the plant.

#### **9.2.1.4 Testing and Inspection Requirements**

Initial testing of the system includes performance testing of the cooling towers and pumps for conformance with design heat loads, water flows, and heat transfer capabilities. An integrity test is performed on the system upon completion. This initial testing of the system will include demonstrating that PSWS supplies adequate cooling water flow rate to the RCCWS and TCCWS heat exchangers.

Additional testing details of PSWS are described in Subsection 14.2.8.1.51.

Provision is made for periodic inspection of components to ensure the capability and integrity of the system. The pumps are tested in accordance with standards of the Hydraulic Institute ANSI/HI 2.6 (M108). Testing is performed to simulate all normal modes of operation to the greatest extent practical. Transfer between normal and standby power source is included in the periodic tests.

Motor-operated valves are in-service tested and inspected to improve plant availability.

#### **9.2.1.5 Instrumentation Requirements**

The PSWS is operated and monitored from the MCR. The PSWS can also be operated from the remote shutdown panels.

With one PSWS pump operating, the respective standby pump shall start automatically on detection of low system pressure signal in that train, loss of electric power to the operating pump, or operating pump trip signal.

Starting a PSWS pump automatically opens a flow path through the RCCWS and TCCWS heat exchangers..

The pump discharge strainers have remote manual override features for their automatic cleaning cycle. Pressure drop across the strainer is locally indicated and a high-pressure drop is annunciated in the control room. Flow elements and transmitters are used in the return headers to monitor PSWS flow in the MCR and can be used to assist in leak detection.

Supply and return header temperatures and supply header pressure are indicated in the MCR.

Each TCCWS and RCCWS heat exchanger will have a pressure differential transmitter to indicate the pressure drop across the heat exchangers. In addition, a discharge flow transmitter shall be placed after each RCCWS and TCCWS heat exchanger.

This PSWS instrumentation conforms with GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.2.1.6 COL Unit-Specific Information**

The COL Applicant will determine material selection and provide provisions to preclude long-term corrosion and fouling of the PSWS based on site water quality analysis.

#### **9.2.1.7 References**

9.2.1-1 ANSI/HI 2.6 (M108) American National Standard for Vertical Pump Tests, (see Table 1.9-22)

## 9.2.2 Reactor Component Cooling Water System

### 9.2.2.1 Design Bases

#### Safety (10 CFR 50.2) Design Bases

The Reactor Component Cooling Water System (RCCWS) does not perform any safety-related function. Therefore, the RCCWS has no safety design basis.

The RCCWS has Regulatory Treatment of Non-Safety Systems (RTNSS) functions to provide post 72-hour cooling to the nuclear island chillers and diesel generators. Performance of RTNSS functions is assured by applying requirements for redundant trains, physical and electrical separation of trains, seismic requirements, and ability to withstand Category 5 hurricane missiles and flood protection. Appendix 19A provides the level of oversight and additional requirements to meet the RTNSS functions.

The RCCWS meets the requirements of GDC 2 as it pertains to Position C.2 of Reg. Guide 1.29. The RCCWS also meets the intent of GDC 2 as it pertains to Position C.1 of Reg. Guide 1.29.

The RCCWS meets GDC 4 with respect to dynamic effects associated with water hammer. The RCCWS has high point vents and operation and maintenance procedures assure sufficient measures are taken to avoid water hammer.

The RCCWS meets GDC 5 for shared systems and components important to safety. The RCCWS Standard Plant design does not share any SSC with any other unit.

The ESBWR RCCWS meets the acceptance criteria of GDC's 44, 45, and 46 by providing the following design considerations:

- Capable of transferring heat loads from SSC's to a heat sink under normal and accident conditions;
- Component redundancy so the system will remain functional assuming a single failure coincident with a loss of offsite power;
- Capability to isolate components or piping so system function will not be compromised;
- Design provisions to permit inspection and operational testing of components and equipment.

#### Power Generation Design Bases

The RCCWS provides cooling water to plant auxiliary equipment during start-up, normal power operation, hot standby, and plant cooldown. The temperature limits for the cooling water cold leg supplied for equipment cooling are shown in Table 9.2-4. The RCCWS is also designed to operate during a Loss of Preferred Power (LOPP).

The RCCWS is designed so that neither a single active nor credible single passive component failure results in a complete loss of active nuclear island cooling and/or plant dependence on any safety-related system.

The RCCWS is designed for ease of restoration after a single component failure without plant operating mode or power level change.

The RCCWS is designed to limit leakage of radioactive or chemical contamination to the environment.

### ***9.2.2.2 System Description***

#### **Summary Description**

The RCCWS provides cooling water to Nonsafety-Related components in the Reactor Building and provides a barrier against radioactive contamination of the PSWS.

A simplified diagram of the closed loop RCCWS is shown in Figure 9.2-2.

#### **Detailed System Description**

The RCCWS consists of two 100% capacity independent and redundant trains.

RCCWS cooling water is continuously circulated through various auxiliary equipment heat exchangers and rejects the heat to the PSWS. The ESBWR Standard Plant RCCWS heat loads are shown on Table 9.2-3. The RCCWS component design characteristics are shown on Table 9.2-4.

In the event of LOPP, the RCCWS supports the FAPCS and the RWCU/SDC in bringing the plant to cold shutdown condition in 36 hours if necessary assuming the most limiting single active failure. In addition, RCCWS provides cooling water to the Standby On Site Power System Diesel Generators.

Each RCCWS train consists of parallel pumps, parallel heat exchangers, one surge tank, connecting piping, and instrumentation. Both trains share a chemical addition tank. The two trains are normally connected by crosstie piping during operation for flexibility, but may be isolated for individual train operation or maintenance of either train.

The pumps in each train discharge through check valves and butterfly valves to a common header leading to the RCCWS heat exchangers' header. Crosstie lines between each train are provided upstream and downstream of the heat exchangers; at the pump suction and discharge headers; and downstream of the Diesel Generators' cooling water supplies. The heat exchanger outlet isolation valves are automatic. The heat exchanger flow control valves, bypass temperature control valves, and cross-tie isolation valves are pneumatically operated.

RCCWS cooling water is supplied to the following major users:

- Chilled Water System (CWS) Nuclear Island chiller-condenser (Subsection 9.2.7)
- RWCU/SDC non-regenerative heat exchanger (Subsection 5.4.8)
- FAPCS heat exchanger (Subsection 9.1.3)
- Standby On Site AC Power Supply Diesel Generators (Subsection 9.5.5) [1325]

The flow paths to heat exchangers and coolers are provided with flow balancing features that may be fixed orifice plates and/or control or manual valves, which can also be used for isolation. The major heat exchangers and coolers have motor-operated isolation valves for operator convenience.

The RCCWS pumps and heat exchangers are located in the Turbine Building.

RCCWS pumps' and heat exchangers' sizes, capacities, and types are shown in Table 9.2-4. Normally, the pumps in each train are powered from independent buses. During a LOPP, the pumps in either train can be powered from the Standby On Site AC Power System.

The RCCWS utilizes plate type heat exchangers. Leakage through holes or cracks in the plates is not considered credible based on industry experience with plate type heat exchangers. In addition, the heat exchangers are designed such that any gasket leakage from either RCCWS or PSWS drains to the Equipment and Floor Drain System. This design mitigates cross-contamination of either RCCWS or PSWS. Pressure and air relief valves are provided as necessary.

Surge tanks provide a constant pump suction head and allow for thermal expansion of the RCCWS inventory. The tanks are located above the highest point in the system. Makeup to the RCCWS inventory is from the Makeup Water System (MWS) through an automatic level control valve. A manual valve provides a backup source of makeup from the Fire Protection System.

### **System Operation**

The RCCWS operates during startup, normal power operation, hot standby, normal cooldown, shutdown/refueling, and LOPP.

RCCWS pump and heat exchanger configuration during each operational mode is shown in Table 9.2-5.

If any of the redundant users requires cooling in addition to the primary users, additional pumps may need to be started.

RCCWS heat exchanger operation is coordinated with PSWS flow. RCCWS cooling water flow through a RCCWS heat exchanger is only allowed if there is a corresponding PSWS water flow to absorb the heat load.

#### ***9.2.2.3 Safety Evaluation***

The RCCWS has no safety-related function. Failure of the system does not compromise any safety-related system or component, nor does it prevent a safe shutdown of the plant.

There is no interface with the Class 1E electrical system.

#### ***9.2.2.4 Testing and Inspection Requirements***

All major components are tested and inspected as separate components prior to installation and as an integrated system after installation to ensure design performance. Additional testing details of RCCWS are described in Subsection 14.2.8.1.21.

Provision is made for periodic inspection of major components to ensure the continued capability and integrity of the system. Indicators are provided for vital parameters required for testing and inspection. Provisions for grab sampling of RCCWS cooling water are provided for chemical and radiological analyses.

### **9.2.2.5 Instrumentation Requirements**

The RCCWS is operated and monitored from the Main Control Room (MCR). Major system parameters (loop flow rate, heat exchanger outlet temperature and pressure) are indicated in the MCR. Low pump discharge header pressure, high or low head tank level, and excessive makeup valve opening time are alarmed/annunciated in the MCR.

Local temperature, pressure and level indicators provide additional component performance information.

The RCCWS heat exchanger isolation valves open automatically upon start of the corresponding PSWS flow. Failure of a RCCWS pump automatically starts the standby pump. Failure of one of the electrical buses automatically starts the standby pump(s) in the unaffected train.

RCCWS radiation monitors are provided for monitoring radiation levels and alerting the plant operator of abnormal radiation levels.

This RCCWS instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

### **9.2.2.6 COL Unit-Specific Information**

None

### **9.2.2.7 References**

None.

## **9.2.3 Makeup Water System**

### **9.2.3.1 Design Bases**

#### **Safety (10 CFR 50.2) Design Bases**

The Makeup Water System (MWS) is a Nonsafety-Related system, and has no safety design basis other than provision for safety-related containment penetrations and isolation valves. The MWS meets GDC 2, as it relates to meeting the guidance of Regulatory Guide (RG) 1.29. The applicable sections of RG 1.29 include Position C.1 for safety-related portions and Position C.2 for Nonsafety-Related portions. The seismic and quality group classifications are identified in Table 3.2-1. The MWS meets GDC 5 for shared systems and components important to safety. The MWS Standard Plant design does not share any SSC with any other unit.

As discussed below, if available, the MWS can be used to provide makeup water to the Isolation Condenser / Passive Containment Cooling (IC/PCC) pools following an anticipated operational occurrence (AOO) or any abnormal event. However, this MWS function is not assumed or modeled in any safety analysis.

#### **Power Generation Design Bases**

The MWS is designed to supply demineralized water to the equipment and components shown in Table 9.2-6, and meets the water quality requirements shown in Tables 9.2-7 and 9.2-8. The MWS equipment sizes are shown in Table 9.2-9.

The system provides the following design functions:

- Remove dissolved minerals, organics, and other impurities;
- Transfer Makeup Water throughout the entire plant; and
- Provide 7 day capacity of makeup water demand during normal power generation.

#### ***9.2.3.2 System Description***

The MWS consists of two subsystems: (1) the demineralization subsystem and (2) the storage and transfer subsystem. The demineralization subsystem is a reference design that is dependent on the site-specific water quality of the available source water. The storage and transfer subsystem is a standard design applicable to any site. The makeup water transfer pumps and the demineralization subsystem are sized to meet the demineralized water needs of all operation conditions except for shutdown/refueling. During the shutdown/refueling mode, the increases in plant water consumption require use of a temporary demineralization subsystem and temporary makeup water transfer pumps to be used as a supplemental water source.

In the reference design, the MWS major equipment is housed entirely in the Service Water/Water Treatment Building except for the demineralized water storage tank (which is outdoors and adjacent to this building) and the distribution piping to the interface systems. Based on local weather conditions, freeze protection is provided for the demineralized water storage tank and piping exposed to freezing conditions.

The MWS equipment and associated piping in contact with demineralized water are fabricated from corrosion resistant materials such as stainless steel to prevent contamination of the makeup water.

Table 9.2-9 lists the major MWS components.

#### **Demineralization Subsystem**

Feedwater for the demineralization subsystem is drawn from the Station Water storage tank. Production of demineralized water by the demineralization subsystem can be initiated and shut down either manually or automatically based on the demineralized water storage tank level. A chemical addition system is located upstream of the cartridge filters to provide treatment to the pretreated water supplied by the Station Water System. One of the Reverse Osmosis (RO) high-pressure pumps provides the pressure required for flow through the RO unit membranes. The second high-pressure pump is a backup. The RO unit reject flow is sent to the cooling tower blowdown facility. Because of the pressure drop across the RO membranes, product water is temporarily stored in the catch tank before being pumped by one of the forwarding pumps to the mixed bed demineralizer unit. Operation of the RO high-pressure pumps is interlocked with that of the forwarding pumps. The mixed bed demineralizer consists of both strong cation and anion resins in the same vessel that polishes the RO product water. The mixed bed unit effluent is monitored for water quality. This effluent is automatically recirculated to the pretreated source water storage tank until the water quality requirements are met. Makeup water is then delivered to the MWS demineralized water storage tank. The modular design of the RO unit and the mixed bed unit allows continuous demineralized water production. Cleaning, back flushing, or

module removal are manual operations based on elevated differential pressure across the module or total flow through the system. No regeneration of mixed bed modules is performed on-site.

### **Storage and Transfer Subsystem**

The flow path of the storage and transfer subsystem of the MWS is from the MWS demineralized water storage tank, through a MWS transfer pump, to the interface systems. One pump operates continuously to maintain the system pressure. Increased demand or primary transfer pump failure automatically starts the second transfer pump.

#### **9.2.3.3 Safety Evaluation**

The MWS does not have any safety-related functions except for containment isolation. Failure of the MWS does not compromise any safety-related system or component nor does it prevent a safe shutdown. If available, MWS can be used to provide makeup water to the IC/PCC pools following an AOO or any abnormal event. However, this MWS function is not assumed or modeled in any safety analysis.

#### **9.2.3.4 Testing and Inspection Requirements**

Initial preoperational acceptance testing of the MWS is performed to demonstrate proper system and component functioning. MWS operability is continuously demonstrated during normal plant operation. MWS containment isolation components are designed to meet the in-service inspection requirements of ASME Section XI.

#### **9.2.3.5 Instrumentation Requirements**

Instrumentation provided for the MWS includes pressure, flow, level, conductivity, silica, chloride, and sodium. These parameters are monitored and recorded at the appropriate locations in the system. Monitoring and control of the MWS operation are provided on a local panel. Monitoring of storage tank water level is provided in the MCR.

Controls and interlocks are provided for the maintenance of MWS water quality and system equipment protection. Demineralizer effluent water quality and quantity is monitored and recorded. Out-of-specification water is automatically recirculated back to the source water storage tank and alarmed on the local control panel. Pumps are protected by a low tank level alarm on the suction side and minimum flow recirculation piping on the discharge side. Automatic controls are provided with manual backup.

This MWS instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.2.3.6 COL Unit-Specific Information**

None

#### **9.2.3.7 References**

9.2.3-1 Reg Guide 1.29 “Seismic Design Classification” (see Table 1.9-21)

9.2.3-2 ASME Boiler and Pressure Vessel Code (B&PVC), Section XI – Rules for Inservice Inspection of Nuclear Power Plant Components (see Table 1.9-22)

#### **9.2.4 Potable and Sanitary Water Systems**

The Potable and Sanitary Water Systems meets GDC 60 for provisions provided to control the release of liquid effluents containing radioactive material. The Potable and Sanitary Water Systems have no interconnections to systems with the potential for containing radioactive material.

The COL applicant will provide the design of the potable and sanitary water system that provides sufficient supply and is designed to provide a minimum of 12.6 l/s (200 gpm) of potable water during peak demand periods.

The COL applicant will provide design of wastewater effluent systems that properly disposes of sanitation wastes.

#### **9.2.5 Ultimate Heat Sink**

In the event of an accident, the Ultimate Heat Sink (UHS) is provided by the Isolation Condenser / Passive Containment Cooling System (IC/PCCS) pools, which provide the heat transfer mechanism for the reactor and containment to the atmosphere. Subsection 5.4.6 provides a discussion of the Isolation Condenser System (ICS). Subsection 6.2.2 provides a discussion of the Passive Containment Cooling System.

The IC/PCCS meets GDC 2, by compliance with Regulatory Guide (RG) 1.29. The applicable sections of RG 1.29 include Position C.1 and C.2. The seismic and quality group classifications are identified in Table 3.2-1. The IC/PCCS meets GDC 2, by compliance with Regulatory Guide (RG) 1.27 Positions C.2 and C.3 by providing a highly reliable source of decay heat. A separate reservoir is not required for the ESBWR Standard Plant.

The IC/PCCS meets GDC 5 for shared systems and components important to safety. The IC/PCCS Standard Plant design does not share any SSC with any other unit.

The requirements of Criterion 44 for heat transfer to the ultimate heat sink are met. The ESBWR ultimate heat sink is the IC/PCC pool. In the event of a design basis accident, heat is transferred to the IC/PCC pool(s) through the Passive Containment Cooling System (PCCS). The water in the IC/PCC pool(s) is allowed to boil and the resulting steam is vented to the environment. The PCCS has no active components and requires no electrical motive power or control and instrumentation functions to perform its safety-related function of transferring heat to the ultimate heat sink. Therefore, no credible single failure can prevent the PCCS from performing its safety-related function.

The IC/PCC pool is located outside containment and is accessible for periodic inspections. During outages, the IC/PCC pool compartments can be drained to permit inspection of the condensers. PCCS piping inside containment can be inspected during outages. The features of the PCCS meet the requirements of Criterion 45.

The design of the PCCS meets the requirements of Criterion 46. Redundancy and isolation are provided to allow periodic pressure testing of the PCCS. As discussed in the evaluation of

Criterion 44, the PCCS contains no active components; therefore, functional testing is not necessary. The periodic inspections described in the response to Criterion 45 verify system integrity.

The Fire Protection System (FPS) provides post accident makeup to the IC/PCCS pools through safety-related Fuel and Auxiliary Pool Cooling System (FAPCS) piping. The FPS also provides post-accident makeup to the Spent Fuel Pool (SFP). Subsection 9.5.1.1 discusses the FPS as a backup emergency makeup water source through the FAPCS. Table 9.5-2 provides IC/PCCS pools and SFP minimum total makeup flow rate at 72 hours into an event. Subsection 9.5.1 states that the FPS provides on-site makeup water capability from 72 hours to 7 days, after which time offsite makeup sources can be provided via safety-related external FAPCS connections outside the Reactor and Fuel Buildings or on-site makeup sources, if available, can be used. Table 9.5-2 provides flow rate requirements for the makeup water sources. Normally, the makeup water quality is required to meet demineralized water chemistry requirements. However, during accident conditions, makeup water quality can meet fire protection water chemistry requirements.

The principle heat source is decay heat from the fuel. The decay heat input rate decreases with time as shown in the Figure 6.2-10c series of decay heat curves. The evaporation and, therefore, makeup water demand would not exceed the Table 9.5-2 rate beyond 72 hours. Subsection 9.1.3.2 discusses the use of the FAPCS to provide water after 72 hours post-accident. The requirement for 30-day water makeup capability during an accident is identified in Table 1.9-9, SRP Section 9.2.5.

#### ***9.2.5.1 COL Unit-Specific Information***

The COL applicant will develop procedures to use an external makeup water supply through the FAPCS to the IC/PCCS pools and SFP beyond 7 days following an accident. The external makeup water supply will, as a minimum, meet the flow requirements of the Table 9.5-2 section entitled, "Required minimum total makeup flow rate to IC/PCC and spent fuel pools at 72 hours into an event," and, as a minimum, the fire protection water chemistry requirements.

#### ***9.2.5.2 References***

None.

### **9.2.6 Condensate Storage and Transfer System**

#### ***9.2.6.1 Design Bases***

##### **Safety (10 CFR 50.2) Design Bases**

The Condensate Storage and Transfer System (CS&TS) does not perform or ensure any safety-related function, and thus, has no safety design basis.

The CS&TS meets GDC 2 by compliance with Regulatory Guide (RG) 1.29. The applicable section of RG 1.29 includes Position C.2 for Nonsafety-Related portions. The seismic and quality group classifications are identified in Table 3.2-1.

The CS&TS meets GDC 5 for shared systems and components important to safety. The CS&TS Standard Plant design does not share any SSC with any other unit.

The requirements of GDC 44 are not applicable to the CS&TS Standard Design for the ESBWR. The CS&TS is Nonsafety-Related and is not required to meet GDC 44 requirements.

The CS&TS meets the requirements of GDC 44 and GDC 45 by providing design provisions to permit inspection and operational testing of components and equipment.

The CS&TS meets GDC 60 by compliance with RG 1.143 Position C.1.2 for provisions provided to control the release of liquid effluents containing radioactive material.

The requirements of RG 1.155 are met by providing a design to shut down safely without reliance on offsite or diesel-generator-derived AC power for 72 hours, which exceeds station blackout requirements.

### **Power Generation Design Bases**

The Condensate Storage and Transfer System is designed to:

- Operate during plant startup, power operation, and normal shutdown. The system is not required to operate following loss of power or during any design basis event.
- Provide managed storage capacities in the condensate storage tank;
- Provide a distribution system to supply condensate quality water to equipment;
- Provide one transfer pump sized to meet the requirements for filling the Condensate and Feedwater System in approximately 16 hours plus recirculation of a minimum of 25% of the pump capacity;
- Provide a 100% redundant backup transfer pump;
- Provide the capability to maintain the water quality requirements in the Condensate Storage Tank (CST) by pumping tank contents to the Liquid Radwaste System when the Condensate Purification System is not operating;
- Provide an enclosed area to retain any tank overflow or leakage until an appropriate disposal action is taken;
- Provide sampling of the retention area sump prior to disposal to determine if the activity of the sump contents is within 10 CFR 20 limits and
- The CS&TS is designed to Seismic Category II criteria when located in Seismic Category I buildings to preclude damage to safety-related equipment should a seismic event occur.

#### **9.2.6.2 System Description**

The CS&TS consists of two independent and 100% redundant transfer pumps that take suction from the CST and provide water to interface systems as required. The CST provides storage capacity for condensate rejected from the Condensate and Feedwater System, for condensate quality Liquid Waste Management System effluent during normal operation, and for Condensate and Feedwater System and condenser hotwell inventory during system maintenance outages. The CST also provides a minimum storage capacity for the Control Rod Drive (CRD) System as

a reserve water source for Reactor Pressure Vessel (RPV) makeup following a Nuclear Steam Supply System (NSSS) isolation event. The CS&TS equipment and associated piping are fabricated from stainless steel to prevent contamination of the system water.

### **Condensate Storage Tank**

The CST is the normal source of water for make up to selected plant systems. The condensate transfer pumps take their suction from the CST and provide makeup water for various services in the Reactor Building, the Turbine Building, Fuel Building, and the Radwaste Building.

The makeup water transfer pumps supply makeup to the CST. The CST also acts as surge capacity for the condensate system by receiving rejected condensate from and making up water to the steam makeup-feedwater cycle. The tank overflows to the enclosed retention area. The tank nozzle location for the CRD pumps suction is located at a lower elevation than the nozzles for the condensate transfer pumps and the hotwell makeup line to ensure availability of the water volume reserved for the CRD System. Other water volumes generated during normal, refueling, and maintenance operations, shown in Table 9.2-10, are accommodated through management of the CST water level.

A basin surrounding the tank is designed to prevent uncontrolled runoff in the event of a tank failure. The enclosed space is sized to contain the total tank capacity. Tank overflow is also collected in this space. A sump is provided inside the retention area with provisions for sampling collected liquids prior to routing them to the Liquid Waste Management System or the storm sewer as per sampling and release requirements.

### **Condensate Transfer Pumps**

There are two 100% redundant condensate transfer pumps. One of the two transfer pumps runs continuously to provide condensate quality water as required. Minimum flow recirculation is provided for pump protection.

#### ***9.2.6.3 Safety Evaluation***

The CS&TS does not perform or ensure any safety-related function. Failure of the system does not compromise any safety-related system or component, nor does it prevent safe shutdown of the plant.

#### ***9.2.6.4 Testing and Inspection Requirements***

Initial preoperational acceptance testing of the CS&TS is performed to demonstrate proper system and equipment functioning. CS&TS operability is demonstrated by continuous use during normal plant operation. CST grab samples are periodically taken and analyzed to monitor water quality.

#### ***9.2.6.5 Instrumentation Requirements***

The redundant CS&TS pumps are interlocked to automatically start the standby pump on low discharge header pressure and the trip of an active pump is annunciated in the MCR. High discharge pressure stops the standby pump. Insufficient discharge pressure or low tank level initiates an alarm in the MCR. Level transmitters actuate a MCR alarm and shut off the condensate transfer pumps when the tank level reaches a minimum volume reserved for the CRD

system shown on Table 9.2-10. The makeup water control valve level transmitters control the CST water level. An alarm is initiated if the CST level decreases below the level that opens the makeup water valve. An alarm actuates in the MCR if the CST water level increases above the level that isolates makeup water to the tank. This alarm point is lower than the overflow level. CST level indication is provided in the MCR.

The operating status of the CS&TS components, including CST level and pump header discharge pressure is monitored and indicated locally.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.2.6.6 COL Information**

None.

#### **9.2.6.7 References**

None.

### **9.2.7 Chilled Water System**

The Chilled Water System (CWS) consists of the Nuclear Island Chilled Water Subsystem (NICWS) and the Balance of Plant Chilled Water Subsystem (BOPCWS).

#### **9.2.7.1 Design Bases**

##### **Safety (10 CFR 50.2) Design Bases**

The Chilled Water System (CWS) does not perform or ensure any safety-related function, and thus, has no safety design basis, except for the containment isolation valves.

The CWS meets GDC 2, by compliance with Regulatory Guide (RG) 1.29. The applicable sections of RG 1.29 include Position C.1 for safety-related portions and Position C.2 for Nonsafety-Related portions. The seismic and quality group classifications are identified in Table 3.2-1.

The Nonsafety-Related portions of CWS has Regulatory Treatment of Non-Safety Systems (RTNSS) functions to provide post 72-hour cooling for HVAC. Appendix 19A provides the level of oversight and additional requirements to meet the RTNSS functions.

##### **Power Generation Design Bases**

The CWS is designed to provide chilled water to the plant facilities equipment at maximum demand.

The NICWS is designed to operate during a Loss of Preferred Power (LOPP) and can receive power from the on-site diesel-generators.

The heat exchangers associated with the Offgas System (OGS) handle potentially radioactive material at an operating pressure lower than the pressure of the water that cools it. Any tube leakage, therefore, results in a flow from the CWS to the OGS.

The CWS is designed to Seismic Category II criteria when located in Seismic Category I buildings to preclude damage to safety-related equipment in a seismic event, except for the containment penetrations and isolation valves that are designed to Seismic Category I.

### **9.2.7.2 System Description**

#### **Summary Description**

The CWS consists of two independent subsystems: the NICWS and the BOPCWS. The CWS provides chilled water to the cooling coils of air conditioning units and other coolers in the Reactor Building, Control Building, Turbine Building, Radwaste Building, Service Building, Electrical Building, Fuel Building, Technical Support Center and Hot Machine Shop. The chilled water absorbs the rejected heat from these coolers and is pumped through the chiller-evaporators where the heat is transferred to RCCWS and TCCWS.

The CWS component design characteristics are listed in Table 9.2-11. The CWS simplified diagram is shown in Figure 9.2-3.

#### **Detailed NICWS Description**

The NICWS consists of two 100% capacity redundant and independent loops (Train A and Train B loops) with crossties between their chilled water piping. The isolation valves in the crosstie lines upstream and downstream of the evaporators are normally open. Each NICWS loop consists of parallel pumps, parallel chillers, one surge tank, an air separator, startup strainer, piping, valves, and instrumentation. A bypass line with a flow control valve is provided parallel to the loads. A chemical feed tank for each loop is installed in parallel with the loads for corrosion inhibitor addition to the chilled water. Each loop is powered from separate buses.

The following units are cooled by the NICWS:

- Fuel Building HVAC air handling units;
- Control Building HVAC air handling units;
- Reactor Building HVAC air handling units and building supply air units;
- Drywell Air Coolers;
- RCCWS equipment room HVAC fan coil units in Turbine Building;
- Instrument air compressors room HVAC fan coil units in Turbine Building;
- NICWS chillers room HVAC fan coil units in Turbine Building;
- Diesel Generator Switchgear Rooms;
- Diesel Generator Control Room;
- Technical Support Center in Electrical Building; and
- Electrical Building HVAC air handling units.

Each NICWS loop consists of a dual centrifugal compressor chillers with pump out unit, oil pump and oil heater. A temperature sensor in the chilled water outlet regulates the output capacity of the chiller by adjusting the compressor inlet guide vanes and/or by turning one of two

compressors on or off. The NICWS bypasses chilled water through a flow control valve in response to reduced demand by the air conditioning units. The NICWS condensers are cooled by the RCCWS.

### Detailed BOPCWS Description

The BOPCWS subsystem consists of one 100% capacity independent loop with crossties between the BOPCWS chilled water piping. BOPCWS consists of parallel water chiller units, including (one per chiller) local control panels, parallel chilled water circulating pumps (one per chiller), a surge tank, an air separator, startup strainer, piping, valves, instrumentation, and separate chemical feed tank. The chemical feed tank is installed in parallel to the loads for corrosion inhibitor addition to the chilled water. BOPCWS water chiller units are powered from separate buses than the NICWS water chiller units.

The following units are cooled by the BOPCWS:

- Turbine Building HVAC air handling units, fan coil units;
- Offgas cooler condenser and building supply air units;
- Radwaste Building HVAC air handling units; and
- Hot Machine Shop HVAC air handling units.

The BOPCWS loop consists of dual centrifugal compressor chillers with pump out unit, oil pump and oil heater. A temperature sensor in the chilled water outlet regulates the output capacity of each chiller by adjusting the compressor inlet guide vanes and/or by turning one of two compressors on or off. The BOPCWS bypasses chilled water through two flow control valves, one per pump/chiller set, in response to reduced demand by the air conditioning units. The BOPCWS condensers are cooled by the TCCWS.

To facilitate control of the BOPCWS, a closed system cooled by the BOPCWS, the Turbine Building Intermediate Chilled Water Subsystem, is installed. This subsystem is comprised of intermediate heat exchangers (plate type), intermediate circulating pumps and one surge tank. The Turbine Building Intermediate Chilled Water Subsystem provides chilled water to several Turbine Building HVAC fan coil units.

The components of both chilled water subsystems have the same design features. The following applies to both subsystems:

- The chiller units are packaged designs, including compressor, condenser, evaporator, refrigerant piping, relief valve, instrumentation, controls, and control panel; and
- The chiller units are capable of operating at partial capacity; varying from less than 25% to 100%.

The surge tanks provide a constant pump suction head and allow for thermal expansion/contraction of the chilled water inventory. The tanks are located a minimum of three feet above the highest system point. Makeup to the chilled water inventory is from the Makeup Water System through an automatic level control valve to the surge tanks.

The BOPCWS loop has an air separator in the chilled water recirculation pump suction with a vent to the surge tank.

## System Operation

The CWS remains functional during startup, normal, and shutdown operations. At least one chiller unit is in operation with the others on standby.

The four chiller units come into operation in a staggered manner based on the actual chilled water flow required for the plant as a whole.

The CWS is designed so that failure or malfunction of one loop does not affect system operability. In case of failure the system automatically generates loop isolation signal.

The following actions are required in case of loop isolation signal:

- Closing cross-tie isolation valves;
- Startup the chillers and pumps on standby;
- Startup air conditioning units of NICWS scope;
- Startup the second fans in the Drywell Cooling System;
- The following events require the automatic loop isolation signal;
- Chilled water leakage exceeding makeup capacity; system leakages are detected by low level signals in surge tanks;
- Loss of Preferred Power (LOPP). LOPP signal generates isolation between NICWS loops; and
- During LOPP, the NICWS is automatically powered from two Nonsafety-Related on-site diesel generators.

### 9.2.7.3 Safety Evaluation

The CWS is classified as a Nonsafety-Related system except for its containment isolation functions. Refer to Subsection 6.2.4 for containment isolation valves and to Subsection 7.3.3 for containment isolation instrumentation.

### 9.2.7.4 Testing and Inspection Requirements

Initial testing of the system includes performance testing of the chillers, pumps and coils for conformance with design heat loads, water flows, and heat transfer capabilities. An integrity test is performed on the system upon completion.

Provision is made for periodic inspection of major components to ensure the capability and integrity of the system. Local display devices are provided to indicate all vital parameters during testing and inspections.

The chillers are tested in accordance with American Society of Heating, Refrigerating, and Air Conditioning Engineers (ASHRAE) Standard 30 (Methods of Testing for Rating Liquid Chilling Packages). The pumps are tested in accordance with standards of the Hydraulic Institute ANSI/HI 1.6 (M104).

The functional capabilities of the containment isolation valves are testable in-place in accordance with the inservice inspection requirements. Such leak test connections are isolatable by two

valves in series. Periodic leak testing of the containment isolation valves is prescribed in the Technical Specifications (refer to DCD Chapter 16) and described in Subsection 6.2.6.

Samples of chilled water may be obtained for chemical analyses. The chilled water is not expected to become radioactive.

#### **9.2.7.5 Instrumentation Requirements**

The CWS status indications, control instrumentation, alarms and annunciators are located in the MCR to provide the operator sufficient data for remote operation of standby units. The plant-wide multiplexing system provides data communication and control.

The chillers and pumps automatically startup and shutdown according to chilled water flow required by the plant. They can also be manually started from the MCR or from the local chiller control panels. The local control panels display the active component operating status and system parameters including flows, temperatures, and pressures.

Chiller package protective controls and monitoring instruments indicate high and low oil pressure, condenser pressure, high and low chilled water temperature and flow, high and low condenser water temperature, and unit diagnostics.

A CWS standby chiller unit starts automatically upon failure of an operating unit. Loss of chilled water or RCCWS/TCCWS cooling water flow automatically stops the chiller unit and associated chilled water recirculating pump.

The chilled water temperature is automatically controlled.

Protective interlocks prevent chiller start if there is no flow through the evaporator or if the RCCWS/TCCWS flow through the NICWS/BOPCWS condenser is out of range. An anti-recycle timer prevents successive compressor starts.

CWS system containment penetration line isolation valves automatically close on a LOCA signal to control the NICWS flow into and out of the containment (refer to Subsection 6.2.4).

The surge tanks are provided with level controlled demineralized water makeup valves and high/low level alarms in the MCR.

This CWS instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.2.7.6 COL Unit-Specific Information**

None

#### **9.2.7.7 References**

- 9.2.7-1 American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE) Standard 30, Methods of Testing Liquid-Chilling Packages, (see Table 1.9-22)
- 9.2.7-2 ANSI/HI 1.6 (M104), Centrifugal Tests, (see Table 1.9-22)

## 9.2.8 Turbine Component Cooling Water System

### 9.2.8.1 Design Bases

#### Safety (10 CFR 50.2) Design Bases

The Turbine Component Cooling Water System (TCCWS) performs or ensures no safety-related function, and thus, has no safety design basis.

There are no connections between the TCCWS and any safety-related systems.

The TCCWS has Regulatory Treatment of Non-Safety Systems (RTNSS) functions to provide post 72-hour cooling to the TB HVAC. Performance of RTNSS functions is assured by applying requirements for redundant trains, physical and electrical separation of trains, seismic requirements, and ability to withstand Category 5 hurricane missiles and flood protection. Appendix 19A provides the level of oversight and additional requirements to meet the RTNSS functions.

The TCCWS meets the requirements of GDC 2 as it pertains to Position C.2 of Reg. Guide 1.29. The TCCWS also meets the intent of GDC 2 as it pertains to Position C.1 of Reg. Guide 1.29.

The ESBWR TCCWS meets the acceptance criteria of GDC's 44, 45, and 46 by providing the following design considerations:

- Capable of transferring heat loads from SSC's to a heat sink under normal and accident conditions;
- Component redundancy so the system will remain functional assuming a single failure coincident with a loss of offsite power;
- Capability to isolate components or piping so system function will not be compromised; and
- Design provisions to permit inspection and operational testing of components and equipment.

#### Power Generation Design Bases

The TCCWS provides cooling water to all turbine island auxiliary equipment listed in Table 9.2-12.

During power operation, the TCCWS operates to provide a continuous supply of cooling water to the turbine island auxiliary equipment.

The TCCWS is designed to permit the maintenance of any single active component without interruption of the cooling function.

Makeup to the TCCWS is designed to permit continuous system operation with design failure leakage and to permit expeditious post-maintenance system refill.

The TCCWS includes an atmospheric surge tank located such that the water level in the tank is above any other component in the system.

The TCCWS utilizes plate and frame type heat exchangers. This design mitigates cross-contamination of either RCCWS or PSWS.

### **9.2.8.2 System Description**

#### **Summary Description**

The TCCWS is a single loop system and consists of one surge tank, one chemical addition tank, pumps, heat exchangers connected in parallel, associated coolers, piping, valves, controls and instrumentation. System parameters are shown in Table 9.2-12 and the system configuration is shown in the Figure 9.2-4. Heat is removed from the TCCWS and transferred to the Nonsafety-Related Plant Service Water System (PSWS) (Subsection 9.2.1).

A TCCWS sample is periodically taken for analysis to assure that the water quality meets chemical specifications.

#### **Detailed Description**

The system is designed in accordance with Quality Group D specifications.

The chemical addition tank is located in the turbine building in close proximity to the TCCWS pumps.

The TCCWS pumps are constant speed electric motor driven, horizontal centrifugal pumps. The pumps are connected in parallel with common suction and discharge lines.

The TCCWS heat exchanger capacity is based on the normal heat loads.

The surge tank is an atmospheric carbon steel tank located at the highest point in the TCCWS. The surge tank is provided with level transmitters and a level control valve to control makeup water addition. The surge tank is located at the highest point in the system and connected to the TCCWS pumps suction header.

#### **System Operation**

During normal power operation, the TCCWS pumps circulate water through one side of the TCCWS heat exchangers in service. The heat from the TCCWS is rejected to the PSWS that circulates water on the other side of the parallel plate TCCWS heat exchangers.

The standby TCCWS pump is automatically started on detection of low TCCWS pump discharge pressure. The standby TCCWS heat exchangers are placed in service manually.

Flow control valves regulate the cooling water flow to the turbine lube oil coolers and the generator hydrogen cooler. Control valves in the cooling water side of these components are throttled in response to temperature signals from the fluid being cooled.

TCCWS flow through heat exchangers and coolers are provided with fixed orifice plates, control valves, and/or manual valves for system balancing. Major heat exchangers and coolers have motor-operated isolation valves for operator convenience.

The surge tank provides a reservoir for leakage from the system and for the expansion and contraction of the cooling fluid with changes in the system temperature. The surge tank is connected to the system in the pump suction lines to ensure that adequate net positive suction head for the TCCWS pumps is available.

A control valve that is actuated by level transmitters sensing the surge tank level automatically controls demineralized makeup water to the TCCWS. A corrosion inhibitor is manually added to the system by means of the chemical addition tank.

#### **9.2.8.3 Safety Evaluation**

The TCCWS has no safety design basis and does not perform any safety-related function. Failure of the TCCWS does not affect any safety-related systems or components.

#### **9.2.8.4 Tests and Inspections**

All major components are tested and inspected as separate components prior to installation and as an integrated system after installation to ensure design performance.

The components of the TCCWS and associated instrumentation are accessible during plant operation for visual examination. Periodic inspections during normal operation are made to ensure operability and integrity of the system. Inspections include measurements of cooling water flows, temperatures, pressures, water quality, corrosion-erosion rate, control positions, and setpoints to verify the system condition.

Additional testing details of TCCWS are described in Subsection 14.2.8.1.52.

#### **9.2.8.5 Instrumentation Requirements**

Pressure and temperature transmitters are provided as required for testing and balancing the system. Flow indicator taps are provided at strategic points in the system for initial balancing of the flows and verifying flows during plant operation.

Surge tank high and low level and TCCWS discharge pressure alarms are transmitted to the MCR.

Makeup flow to the TCCWS surge tank is initiated automatically on low surge tank level and is continued until the normal level is reestablished.

Provisions for taking TCCWS water samples are included.

The TCCWS instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.2.8.6 COL Unit-Specific Information**

None

#### **9.2.8.7 References**

None

### **9.2.9 Hot Water System**

The Hot Water System (HWS) is a closed system that provides heating water to hot water coils in the Air Handling Units (AHU) and Fan Coil Units (FCU) of the HVAC Systems for building heating during cold weather operation or during cold weather plant outages.

### **9.2.9.1 Design Basis**

#### **Safety (10 CFR 50.2) Design Bases**

The HWS serves no safety-related function and therefore has no safety design basis.

#### **Power Generation Design Bases**

During cold weather plant operation or during cold weather plant outages, the HWS maintains acceptable design ambient air temperatures in various areas throughout the ESBWR.

### **9.2.9.2 System Description**

#### **General Description**

The design of the Hot Water System for the ESBWR Standard Plant is based on bounding ambient environmental conditions as defined in Chapter 2. The HWS consists of two 100% capacity heat exchangers, two 100% capacity pumps, an expansion tank, a chemical addition tank, piping, valves and instrumentation. Water is heated in the heat exchangers using steam from the Auxiliary Boiler System (ABS). The heated water is pumped in a closed loop system to hot water coils in the HVAC systems.

During cold weather plant operation or during cold weather plant outages, the HWS supplies hot water throughout the plant to protect equipment from freezing and for personnel comfort.

#### **Component Description**

##### **Heat Exchanger**

There are two 100% capacity heat exchangers. Each heat exchanger is a horizontal shell-and-tube type, with an integral drain cooler, and uses steam from the ABS to heat HWS water. The heat exchanger is located downstream from the system pumps in the turbine building and supplies hot water at a maximum temperature lower than 90°C (194°F).

##### **Pumps**

Two 100% capacity motor driven centrifugal pumps distribute hot water to the various HVAC systems and FCUs.

##### **Surge Tank**

The surge tank maintains system pressure by allowing the water to expand when the water temperature increases and provides a volume to accept makeup water to the HWS.

The tank is an atmospheric, carbon steel, welded, pressure vessel, with tank recirculation, and instrument connections.

##### **Chemical Feed Tank**

The chemical feed tank provides a means of chemical mixing in the system. Addition of chemicals provides control of corrosion.

##### **System Operation**

As the system is filled with demineralized water, samples are taken and the closed loop water chemistry adjusted with chemicals recirculated through the chemical mixing tank with the use of

a single pump. A pump is started and steam is admitted to an HWS heat exchanger and the system is gradually heated.

Condensate level is maintained in each heat exchanger by throttling the heat exchanger discharge flow to the ABS condensate receiving tank.

Hot water flowing to individual heating coils is controlled either by flow balancing fixed orifices or by temperature control valves, according to the requirements of the heating system.

#### ***9.2.9.3 Safety Evaluation***

The HWS has no safety design basis and does not perform any safety-related function. Failure of the HWS does not affect any safety-related systems or components.

#### ***9.2.9.4 Testing and Inspection Requirements***

All major components are tested and inspected as separate components prior to installation and as an integrated system after installation to ensure design performance.

The components of the HWS and associated instrumentation are accessible during plant operation for visual examination. Periodic inspections during normal operation are made to ensure operability and integrity of the system. Inspections include measurements of cooling water flows, temperatures, pressures, water quality, control positions, and setpoints to verify the system condition.

#### ***9.2.9.5 Instrumentation***

Instruments are provided for monitoring system parameters. The Safety-Related system parameters are monitored in the Main Control Room (MCR).

Pressure and temperature indicators are provided as required for testing and balancing the system. Flow indicator taps are provided at strategic points in the system for initial balancing of the flows and verifying flows during plant operation.

Surge tank high and low level and HWS pump discharge pressure alarms are transmitted to the MCR. Makeup flow to the HWS surge tank is initiated automatically on low surge tank level. Level instrumentation and controls are provided for automatically regulating the heat exchanger drain flow rate to maintain the proper level in the heat exchanger shell.

Provisions for taking HWS water samples are included.

#### ***9.2.9.6 COL Information***

None.

#### ***9.2.9.7 References***

None.

## 9.2.10 Station Water System

### 9.2.10.1 Design Basis

#### Safety (10 CFR 50.2) Design Bases

The Station Water System does not perform any safety-related function. Therefore, the Station Water System has no safety design basis.

#### Power Generation Design Bases

The Station Water System is designed to provide a supply of water for the following services:

- Makeup water to the Circulating Water System (CIRC) cooling tower basin
- Makeup water to the Plant Service Water System (PSWS) cooling tower basins
- Feedwater to the Makeup Water System
- Fill water to the Fire Protection System

### 9.2.10.2 System Description

#### Summary Description

The Station Water System provides makeup water and feedwater to Nonsafety-Related systems.

#### Detailed System Description

The station water system consists of the following subsystems:

- Plant Cooling Tower Makeup System
- Pretreated Water Supply System

The Plant Cooling Tower Makeup System provides makeup water to the cooling tower basins for both the Plant Service Water System (Subsection 9.2.1) and Circulating Water System (Subsection 10.4.5). The supply of water makes up for losses resulting from evaporation, drift, and blowdown from the cooling towers. In addition, the Plant Cooling Tower Makeup System provides makeup water to replace water used for PSWS strainer backwash. The Plant Cooling Tower Makeup System consists of a water source, pumps, filters, connecting piping, valves and instrumentation.

The Pretreated Water Supply System filters and chemically pretreats water supplied to the Makeup System (Subsection 9.2.3) for further treatment for use as demineralized water. The Pretreated Water Supply System also supplies water to the Fire Protection System (Subsection 9.5.1) for filling the primary firewater tanks and for maintaining pressure in the yard loop. In addition, the Pretreated Water Supply System provides Plant Service Water System cooling tower makeup as an alternate to the Plant Cooling Tower Makeup System. The Pretreated Water Supply System consists of a water source, pumps, filters, chemical treatment equipment, a pretreated water storage tank, connecting piping, valves and instrumentation.

## **System Operation**

The Plant Cooling Tower Makeup System is not required to operate to support any safety mode of plant operation.

The Pretreated Water Supply System is not required to operate to support any safety mode of plant operation.

### **9.2.10.3 Safety Evaluation**

The Station Water System has no safety-related function. Failure of the system does not compromise any safety-related system or component, nor does it prevent a safe shutdown of the plant.

There is no interface with any safety-related component.

No interconnections exist between the Station Water System and any potentially radioactive system.

### **9.2.10.4 Testing and Inspection Requirements**

The performance, structural, and leaktight integrity of system components is demonstrated by the continuous operation of the system.

### **9.2.10.5 Instrumentation Requirements**

Instruments are provided for monitoring system parameters. The Safety-Related system parameters are monitored in the Main Control Room (MCR).

Pressure and temperature indicators are provided as required for testing and balancing the system. Flow indicator taps are provided at strategic points in the system for initial balancing of the flows and verifying flows during plant operation.

Pretreated station water storage tank high and low level, and low suction pressure for each pump taking suction from the storage tank are alarmed to the MCR.

Provisions for taking water samples are included.

### **9.2.10.6 COL Information**

The COL applicant will provide the design of the Station Water System including:

- Components and design capacities for each subsystem; and
- A simplified diagram of the Station Water System showing the Plant Cooling Tower Makeup System and the Pretreated Water Supply System.

**Table 9.2-1**  
**PSWS Heat Loads**

<b>Normal Operation</b>		
Trains A and B		
RCCW System:	30.9 MW	$1.05 \times 10^8$ Btu/hr
TCCW System:	54.0 MW	$1.84 \times 10^8$ Btu/hr
Total Trains A and B:	84.9 MW	$2.90 \times 10^8$ Btu/hr
<b>Normal Cooldown</b>		
Trains A and B		
RCCW System:	90.9 MW	$3.10 \times 10^8$ Btu/hr
TCCW System:	26.0 MW	$8.88 \times 10^7$ Btu/hr
Total Trains A and B:	116.9 MW	$3.99 \times 10^8$ Btu/hr
<b>LOPP Operation</b>		
Total for Trains A and B		
Single Train, 2 PSWS Pumps	90.4MW	$3.09 \times 10^8$ Btu/hr
Both Trains, 4 PSWS Pumps	145.5MW	$4.97 \times 10^8$ Btu/hr

**Table 9.2-2**  
**PSWS Component Design Characteristics**

<b>PSWS Pumps</b>	
Type	Vertical, wet-pit, centrifugal turbine
Quantity	4
Capacity Each	1.262 m <sup>3</sup> /s (20000 gpm)
Total Discharge Head	Established during detail design
Motor Size	Established during detail design
<b>Plant Service Water System</b>	
Flow (NPHS or AHS)	2.524 m <sup>3</sup> /s (40000 gpm)
<b>PSWS Cooling Towers and Basins</b> (Conceptual Design Only – Site Specific Portion of PSWS)	
Type	Mechanical draft, multi-cell, redundant dual speed, reversible fans
Quantity	2
Heat Load Each*	90.4 MW (3.09 x 10 <sup>8</sup> BTU/hr)
Flow Rate (Water) Each	2.524 m <sup>3</sup> /s (40000 gpm)
Ambient Wet Bulb Temperature	27.2°C (81°F)
Approach Temperature	3.9°C (7°F)
Cold Leg Temperature	31.1°C (88°F)
<b>Strainers</b>	
Type	Automatic cleaning, duplex, basket
Quantity	4

\* - This worst-case heat load occurs during a Single Train Failure with a Loss of Preferred Power.

**Table 9.2-3  
RCCWS Nominal Heat Loads**

<b>Normal Operation</b>		
Nominal Heat Load Contributions		
RWCU/SDC:	9.6 MW	32.8 MBtu/hr
FAPCS:	9.6 MW	32.8 MBtu/hr
CWS:	12.3 MW	41.9 MBtu/hr
Diesel Generator	0 MW	0 MBtu/hr
Other:	1.7 MW	5.7 MBtu/hr
Total Trains A & B:	33.2 MW	113.2 MBtu/hr
<b>Normal Cooldown</b>		
Nominal Heat Load Contributions (Train A)		
RWCU/SDCS:	28 MW	95.6 MBtu/hr
FAPCS:	9.6 MW	32.8 MBtu/hr
CWS:	12.3 MW	41.9 MBtu/hr
Other	1.7 MW	5.7 MBtu/hr
Diesel Generator A	0 MW	0 MBtu/hr
Total Train A:	51.6 MW	176 MBtu/hr
Nominal Heat Load Contributions (Train B)		
RWCU/SDC:	28 MW	95.6 MBtu/hr
FAPCS	9.6 MW	32.8 MBtu/hr
Diesel Generator B	0 MW	0 MBtu/hr
Other:	1.7 MW	5.7 MBtu/hr
Total Train B:	39.3 MW	134 MBtu/hr
Total Train A&B:	90.9 MW	310 MBtu/hr

**Table 9.2-3  
RCCWS Nominal Heat Loads (continued)**

<b>Cooldown with LOPP</b>		
Nominal Heat Load Contributions (Train A)		
RWCU/SDC:	28 MW	95.6 MBtu/hr
FAPCS:	9.6 MW	32.8 MBtu/hr
CWS:	12.3 MW	41.9 MBtu/hr
Other	1.7 MW	5.7 MBtu/hr
Diesel Generator A	14.3 MW	48.8 MBtu/hr
Total Train A:	65.9 MW	225 MBtu/hr
Nominal Heat Load Contributions (Train B)		
RWCU/SDC:	28 MW	95.6 MBtu/hr
FAPCS	9.6 MW	32.8 MBtu/hr
Diesel Generator B	14.3 MW	48.8 MBtu/hr
Other:	1.7 MW	5.7 MBtu/hr
Total Train B:	53.6 MW	183 MBtu/hr
Total Train A&B:	120 MW	408 MBtu/hr
<b>Single Failure Cooldown (Extended Cooldown)</b>		
Nominal Heat Load Contributions (Train A or B)		
RWCU/SDC:	36 MW	123 MBtu/hr
FAPCS:	9.6 MW	32.8 MBtu/hr
CWS:	12.3 MW	41.9 MBtu/hr
Diesel Generator	0 MW	0 MBtu/hr
Other:	1.7 MW	5.7 MBtu/hr
Total Train A or B:	59.6 MW	203 MBtu/hr

**Table 9.2-3**  
**RCCWS Nominal Heat Loads (continued)**

<b>Single Failure Cooldown w/ LOPP</b>		
Nominal Heat Load Contributions (Train A or B)		
RWCU/SDC:	36 MW	123 MBtu/hr
FAPCS:	9.6 MW	32.8 MBtu/hr
CWS:	12.3 MW	41.9 MBtu/hr
Diesel Generator	14.3 MW	48.8 MBtu/hr
Other:	1.7 MW	5.7 MBtu/hr
<b>Total Train A or B:</b>	<b>73.9 MW</b>	<b>252 MBtu/hr</b>

**Table 9.2-4**  
**RCCWS Component Design Characteristics**

<b>Pumps</b>	
Type	Horizontal, centrifugal
Quantity	6
Capacity each	1,350 m <sup>3</sup> /h (5,944 gpm)
Total Discharge Head	Established during detail design
Motor Size	Established by Total Discharge Head
<b>Heat Exchangers</b>	
Type	Horizontal Plate
Quantity	6
Cooling Capacity (Nominal)	30.56 MW (104.3 MBtu/h)
RCCWS Flow per Unit	1,350 m <sup>3</sup> /h (5,944 gpm)
Maximum Operating RCCWS Cold Leg Supply Temperature	35°C (95°F)
Minimum Operating RCCWS Cold Leg Supply Temperature	15°C (59°F)
Maximum Operating PSWS Cold Leg Temperature	31.1°C (88°F)
Material (Plates)	Titanium or Stainless Steel*

\*Site specific water quality is used to determine the material portion of Heat Exchangers

**Table 9.2-5**  
**RCCWS Configuration by Mode**

Operation Mode	No. Train A Pumps Used	No. Train B Pumps Used	No. Train A HX Used	No. Train B HX Used	Total No. Pumps Used	Total No. HX Used	Total Cooling Water Flow
Normal Operation	1	1	1	1	2	2	2,700 m <sup>3</sup> /hr (11,888 gpm)
Normal Cooldown	2	2	2	2	4	4	5,400 m <sup>3</sup> /hr (23,775 gpm)
Cooldown w/ Single Failure of Train A	0	2	0	2	2	2	2,700 m <sup>3</sup> /hr (11,888gpm)
Cooldown w/ Single Failure of Train B	2	0	2	0	2	2	2,700 m <sup>3</sup> /hr (11,888gpm)
Cooldown w/ Single Failure of Train A w/ LOPP	0	3	0	3	3	3	4,050 m <sup>3</sup> /hr (17,832 gpm)
Cooldown w/ Single Failure of Train B w/ LOPP	3	0	3	0	3	3	4,050 m <sup>3</sup> /hr (17,832 gpm)
Normal Cooldown w/ LOPP	3	3	3	3	6	6	8,100 m <sup>3</sup> /hr (35,663 gpm)

**Table 9.2-6  
Makeup Water System Supplied Equipment**

The Makeup Water System provides water for the following:	
1	CST makeup
2	Standby Liquid Control System makeup
3	Liquid Waste System chemical addition and line flushing
4	Solid Waste System for line flushing
5	Reactor Component Cooling Water System makeup
6	Turbine Component Cooling Water System makeup
7	Chilled Water System makeup
8	Process Sampling System process use
9	Auxiliary Boiler System makeup
10	Post Accident Sampling Station flushing
11	HVAC makeup
12	Miscellaneous uses
13	IC/PCC Pool normal makeup water

**Table 9.2-7**  
**Makeup Water System Demineralized Water Storage Tank Nominal Water Quality Requirements**

<b>Water Quality Parameter</b>	<b>Operating Target</b>	<b>System Design</b>	<b>Maximum Value</b>
Chloride (PPB as Cl)	2.5	5.0	25.0
Sulfate (PPB as SO <sub>4</sub> )	2.5	5.0	25.0
Conductivity* at 25°C (μS/cm)	0.080	0.3	1.0
Silica (PPB as SiO <sub>2</sub> )	5.0	10.0	50.0
Corrosion Product Metals			
Iron (PPB as Fe)	8.0	16.0	80.0
Copper (PPB as Cu)	1.0	2.0	10.0
All Other Metals (PPB as the metal)	1.0	2.0	10.0
Total Metals (PPB as the metal)	10.0	20.0	100.0
Organic Impurities** (equivalent ΔK (μS/cm))	0.2	0.4	2.0

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\* Does not include an incremental conductivity value of 0.8 μS/cm at 25°C caused by carbon dioxide from air in water stored in tanks open to the atmosphere.

\*\* Organic impurity values apply to fresh makeup water stored in the makeup water storage tank.

Table 9.2-8

**Makeup Water System Demineralizer Effluent Nominal Water Quality Requirements**

<b>Water Quality Parameter</b>	<b>Operating Target</b>	<b>System Design</b>	<b>Maximum Value</b>
<b>Demineralizer Effluent</b>			
Chloride (PPB as Cl)	1.0	2.0	10.0
Sulfate (PPB as SO <sub>4</sub> )	1.0	2.0	10.0
Conductivity at 25°C (μS/cm)	0.065	0.075	0.20
Silica (PPB as SiO <sub>2</sub> )	1.0	2.0	10.0
<b>Corrosion Product Metals</b>			
Iron (PPB as Fe)	4.0	8.0	40.0
Copper (PPB as Cu)	0.5	1.0	5.0
All Other Metals (PPB as the metal)	0.5	1.0	5.0
Total Metals (PPB as the metal)	5.0	10.0	50.0
Organic Impurities (Equivalent ΔK (μS/cm))	0.2	0.4	2.0

**Table 9.2-9**  
**Major Makeup Water System Components**

<ul style="list-style-type: none"> <li>• Two 5-micron cartridge filters, each rated 1,999 l/min (528 gpm).</li> <li>• Two reverse osmosis (RO) high-pressure pumps, each rated at 1,999 l/min (528 gpm).</li> <li>• One RO unit rated at 1,499 l/min (396 gpm). This unit consists of multiple modules.</li> <li>• One chemical treatment system that provides pretreatment and membrane cleaning specific to the RO membrane type.</li> <li>• One RO product water catch tank with two 1,499 l/min (396 gpm) product forwarding pumps.</li> <li>• One 1,499 l/min (396 gpm) mixed bed demineralizer unit consisting of multiple modules.</li> <li>• One 950 m<sup>3</sup> (250,963 gal) demineralized water storage tank.</li> <li>• Two 1,249 l/min (330 gpm) makeup water transfer pumps.</li> </ul>
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**Table 9.2-10**  
**Capacity Requirements for the Condensate Storage Tank**

<b>Function</b>	<b>Capacity Required</b>	<b>Time Period</b>
Storage capacity to drain the Condensate and Feedwater System and the condenser	2,960 m <sup>3</sup> (781,950 gal)	Maintenance outages
Condensate quality water reclaimed from LWS	270 m <sup>3</sup> /day (71,326 gal per day)	Normal operations; Maintenance outages
Dedicated volume for Control Rod Drive System following a NSSS isolation event*	948 m <sup>3</sup> (250,435 gal)	Normal operations
Total CST capacity	4885 m <sup>3</sup> (1,290,480 gal)	N/A

\* The alarm level volume includes the volume required for transients and for the Control Rod Drive System following a NSSS isolation event.

**Table 9.2-11**  
**Chilled Water System Component Design Characteristics**

<b><u>CWS Chiller Units</u></b>	
Quantity	7
Cooling capacity (each)	4.85 x 10 <sup>6</sup> W (16.55 x 10 <sup>6</sup> Btu/hr) *
<b>Condensers</b>	
Flow Rate (each)	383 m <sup>3</sup> /hr (1686 gpm)
Inlet Water Temperature	35°C (95°F)
Outlet Water Temperature	48.9°C (120°F)
<b>Evaporators</b>	
Flow Rate	600 m <sup>3</sup> /hr (2640 gpm)
Inlet Water Temperature (CW)	14°C (57.2°F)
Outlet Water Temperature (CW)	7°C (44.6°F)
<b>Compressors</b>	
Type	Centrifugal, dual, electric motor driven
Power	1350 kW (1810 hp)
<b>Circulating Pumps</b>	
Quantity	7
Type	Centrifugal
Flow	600 m <sup>3</sup> /hr (2640 gpm)
<b>Surge tanks</b>	
Quantity	3
Type	Vertical, cylindrical
<b>Piping</b>	
Material (NICWS)	Carbon Steel
Material (BOPCWS)	Carbon Steel

**Table 9.2-11  
Chilled Water System Component Design Characteristics (continued)**

<b><u>Turbine Building Intermediate Chilled Water Subsystem</u></b>	
<b>Intermediate Circulating Pumps</b>	
Quantity	2
Type	Centrifugal
<b>Intermediate Heat Exchangers</b>	
Quantity	2
Type	Plate and frame
<b>Intermediate Surge Tank</b>	
Quantity	1
Type	Vertical, cylindrical
<b>Intermediate Subsystem Piping</b>	
Material	Carbon Steel

\* The chiller unit cooling capacity and other CWS component design characteristics correspond to CWS chilled water load of 19,110 kW ( $6.5 \times 10^7$  Btu/hr).

**Table 9.2-12  
Turbine Component Cooling Water System Heat Loads**

The TCCWS removes heat from the following components:	
<ul style="list-style-type: none"> <li>– Balance of Plant chiller condensers</li> <li>– Generator stator cooling water heat exchangers, Generator hydrogen coolers, Hydrogen seal oil coolers and Exciter coolers</li> <li>– Main turbine lube oil coolers</li> <li>– Mechanical vacuum pump cooler</li> <li>– Iso-phase bus cooler</li> <li>– Electro-hydraulic control system coolers</li> <li>– Reactor feed pump motor coolers and Auxiliary Steam Drain feed pump</li> <li>– Condensate pump motor coolers</li> <li>– Auxiliary Steam Drain Cooler</li> <li>– Service air compressor intercoolers and aftercoolers</li> </ul>	
System parameters	
Heat removal capacity	54 MW (184.3E6 BTU/hr)
System flow	Later
Maximum cold leg temperature	35°C (95°F)
<b>Pumps</b>	
Type	Horizontal, split case, single stage, centrifugal
Quantity	3
Capacity each	<b>Later</b>
Total Discharge Head	Established during detail design
Motor Size	<b>Later</b>

**Table 9.2-12**  
**Turbine Component Cooling Water System Heat Loads (continued)**

<b>Heat Exchangers</b>	
Type	Horizontal Plate
Quantity	4
Maximum Operating TCCWS Cold Leg Supply Temperature	35°C (95°F)
Maximum Operating PSWS Cold Leg Temperature	31.1°C (88°F)
Material (Plates)	Titanium or Stainless Steel*

\* Site specific water quality is used to determine the material portion of Heat Exchangers

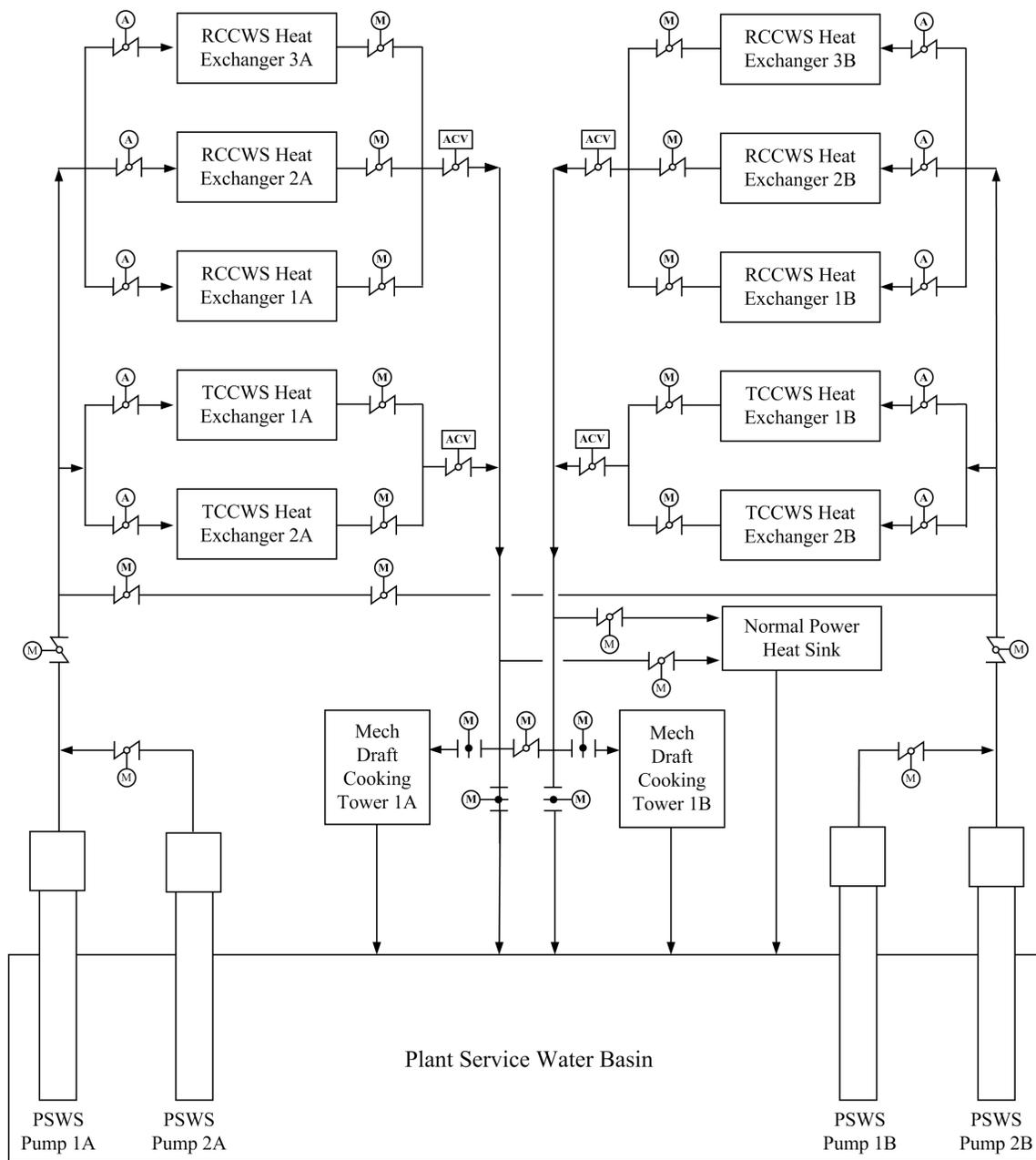


Figure 9.2-1. Plant Service Water System Simplified Diagram

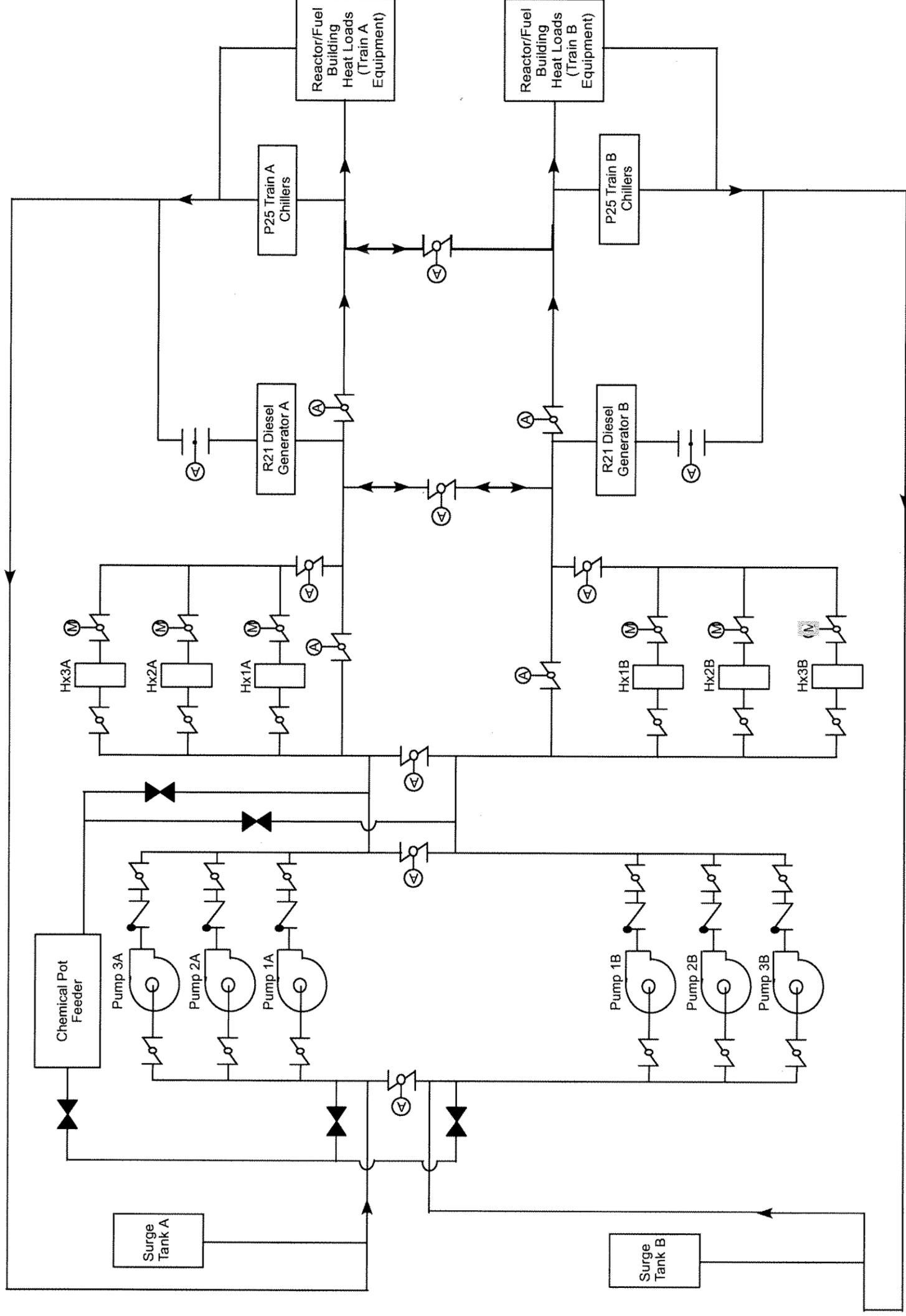


Figure 9.2-2. Reactor Component Cooling Water System

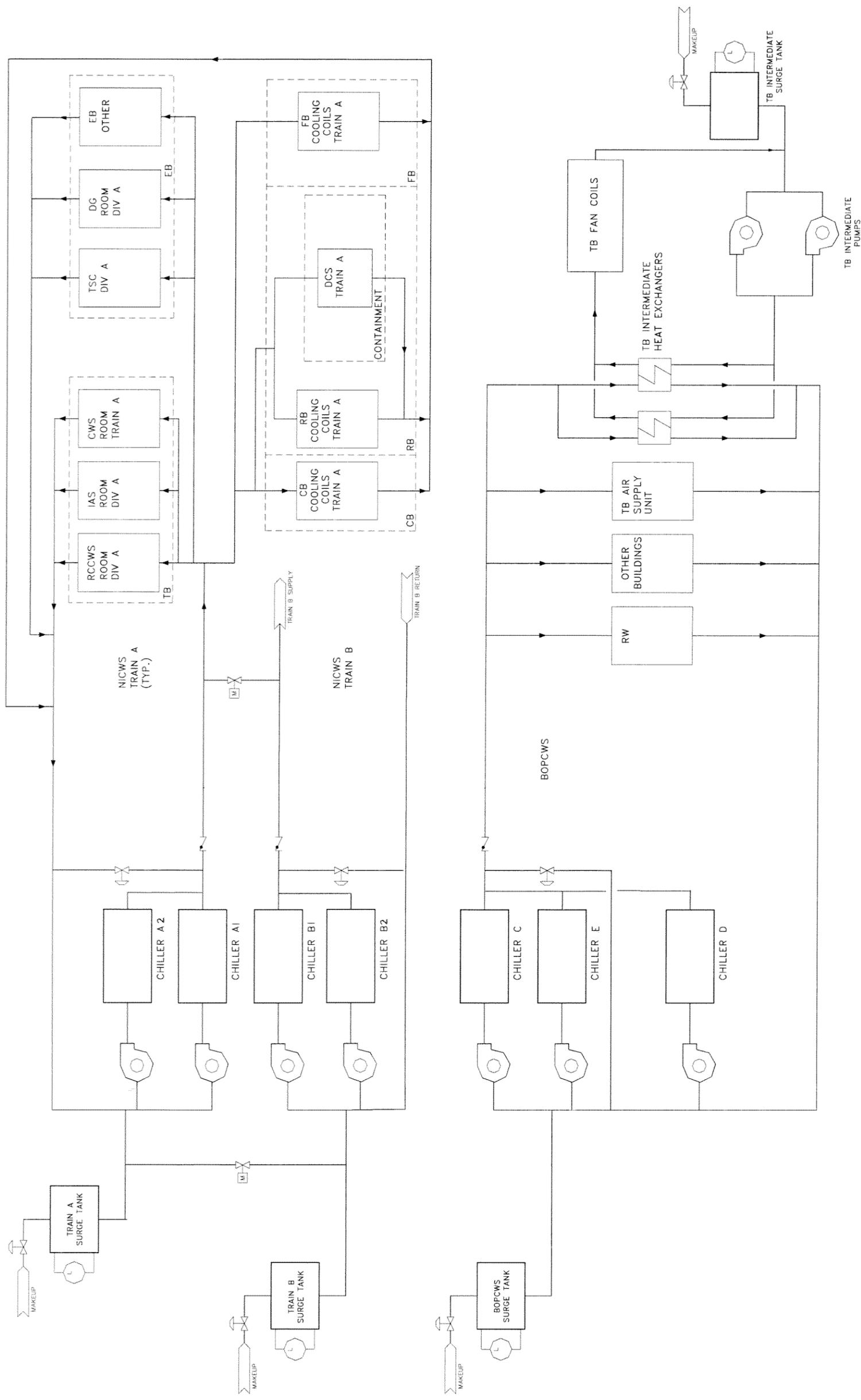


Figure 9.2-3. Chilled Water System Simplified Diagram

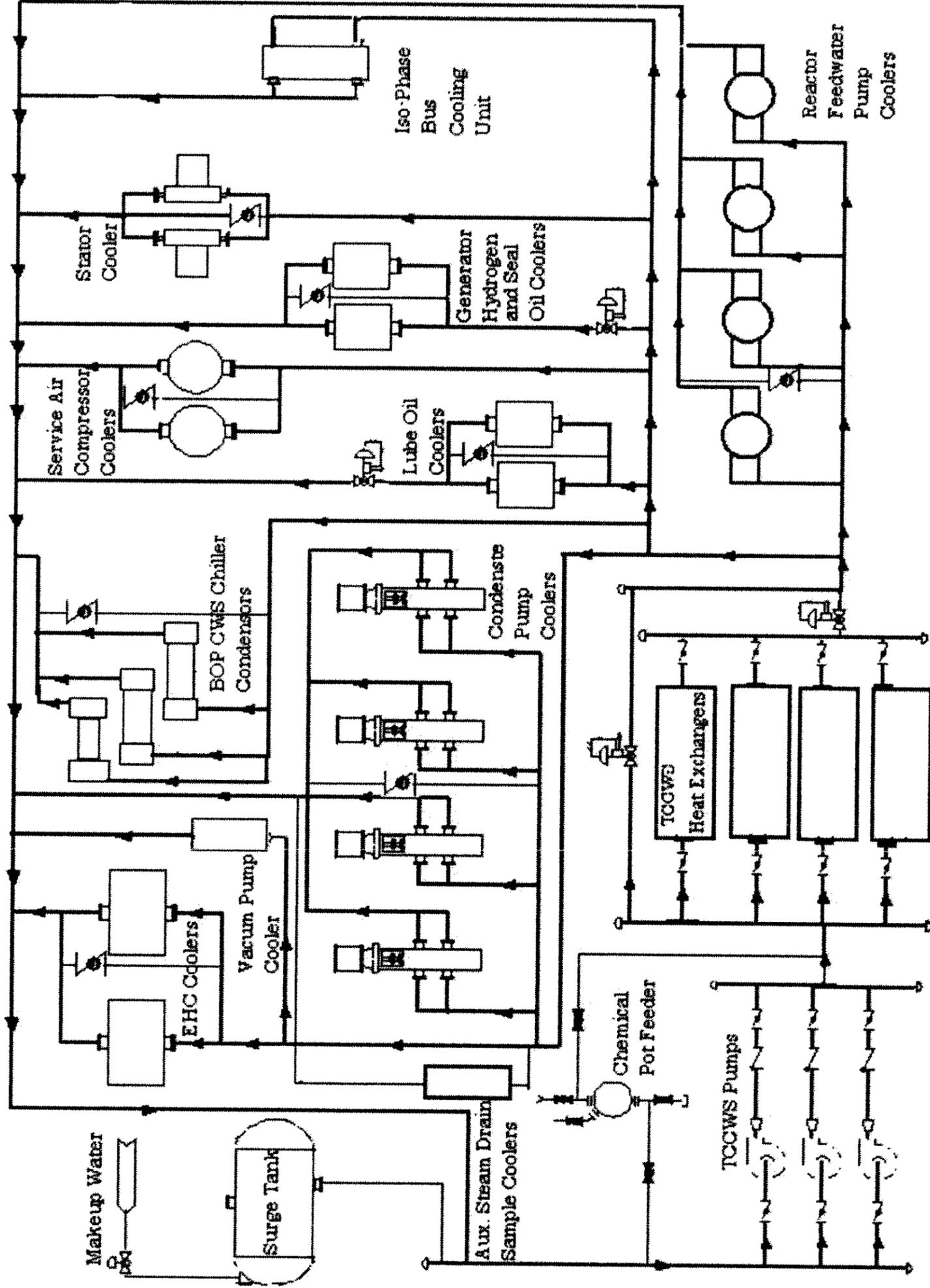


Figure 9.2-4. Turbine Component Cooling Water System Configuration

## 9.3 PROCESS AUXILIARIES

### 9.3.1 Compressed Air Systems

Compressed air systems include the Instrument Air System (IAS), the Service Air System (SAS), the Containment Inerting System (CIS) and the High Pressure Nitrogen Supply System (HPNSS). The IAS is discussed in Subsection 9.3.6; the SAS is discussed in Subsection 9.3.7. The Containment Inerting System and the HPNSS are described in Subsections 9.4.9 and 9.3.8, respectively. The CIS and the HPNSS provide nitrogen gas for instruments and valve operators within the inerted containment.

### 9.3.2 Process Sampling System

#### 9.3.2.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The Process Sampling System (PSS) does not perform or ensure any safety-related function. Therefore, this system has no safety-related design basis. The post-accident monitoring program uses sample point parameters and key sample locations as described in this Subsection and Subsections 7.5.1 and 7.5.2. Relative to the Process Sampling System (PSS), this subsection addresses applicable requirements of General Design Criteria as are discussed in Standard Review Plan Section 9.3.2.II.

PSS is in conformance with the relevant requirements and criteria that are stipulated in the codes and standards that are identified below:

- 10 CFR 20.20 & 20.1101(b);
- 10 CFR 50, Appendix A, GDC 1, 2, 13, 14, 26, 41, 60, 63, and 64;
- 10 CFR 50.34(f)(2)(viii) and 50.34(2)(xxvi);
- Regulatory Guides (RG) 1.21, 1.26, 1.29, 1.33, 1.56, 1.97, and 8.8;
- NUREG-0737, Item II.B.3; and
- ANSI/HPS N13.1.

##### Power Generation Design Bases

The Process Sampling System (PSS) collects representative liquid samples for analysis and provides the analytical information required to monitor plant and equipment performance and changes to operating parameters.

The PSS is designed to function during all plant operational modes under individual system requirements. Design guidelines related to PSS capabilities, the attainment of representative samples, and safety are described in the following paragraphs and in Table 9.3-1.

### 9.3.2.2 System Description

#### Summary Description

The PSS provides sampling of all principle fluid process streams associated with plant operation. The PSS consists of:

- Permanently installed sample lines;
- Sampling panels with analyzers and associated sampling equipment;
- Provisions for local grab sampling; and
- Permanent shielding.

Table 9.3-1 provides a list of the sample points and analyzed parameters and identifies parameters required to be assessed as part of the post accident sampling program per SRP 9.3.2.

#### Provisions for Obtaining Representative Samples

Where practical, sample connections are located in turbulent flow zones to ensure adequate mixing.

Connection is made on the side of horizontal process pipe runs.

Sampling lines are sized to maintain turbulent flow and to minimize purge time. Routing is as short and as straight as possible. Large radius bends are used to avoid traps and dead legs.

Sampling lines and associated valves and fittings are fabricated from stainless steel.

Heat tracing of sampling lines is provided as necessary to prevent crystallization or solidification of contents.

Cooling capabilities are provided for temperature control of the samples as required.

Sampling equipment is designed with flushing and blowdown capability in order to remove sediment deposits, air and gas pockets. Provisions are made to purge sample lines in the stations. All flushing fluids are either returned to appropriate process streams or sent to the radwaste system, except as noted.

#### Detailed System Description

Continuous sample flows are routed from select locations in the process streams to the sampling stations. The sample flows enter the sample stations where pressure, temperature, and flow adjustments are made as necessary. Facilities for grab sampling and special monitoring are provided. Continuous samples are diverted to continuous monitoring equipment. The continuous monitoring equipment transmits signals to the plant computer located in the Main Control Room (MCR). Alarms are provided for indicating off-normal conditions. The parameters monitored for each process stream are described in Table 9.3-1. The sample station's effluents are returned to an appropriate process stream or to the radwaste drain headers through a common return line. ALARA is considered in station layout and design.

#### Reactor Building Sample Station

The Reactor Building Sample Station processes samples from the following systems for analysis:

- Reactor Water Cleanup/Shutdown Cooling System (RWCU/SDC);

- Control Rod Drive System;
- Fuel and Auxiliary Pools Cooling System (FAPCS);

Auxiliary pools, including the Suppression Pool, Gravity-driven Cooling System pools and Isolation Condenser pools, are monitored during the cooling and cleanup modes of FAPCS.

Typical process stream sample parameters for the Reactor Building Sample Station are outlined in Table 9.3-1. Grab sample facilities are also provided at the Reactor Building Sample Station.

### **Turbine Building Sample Station**

The Turbine Building Sample Station is located in the Turbine Building. Process samples from the following systems are routed to this panel for monitoring:

- Condensate and Feedwater System;
- Moisture Separator Reheater System;
- Heater Drain and Vent System;
- Generator Cooling System; and
- Turbine Main Steam System.

Typical process stream sample parameters for the Turbine Building Sample Station are outlined in Table 9.3-1. Grab sample facilities are also provided at the Turbine Building Sample Station.

### **Condensate Polishing Sample Station**

The Condensate Polishing Sample Station is located in the Turbine Building. Process samples from the following systems are routed to this panel for monitoring:

- Condensate and Feedwater System
- Condensate Purification System

Typical process stream sample parameters for the Condensate Polishing Sample Station are outlined in Table 9.3-1. Grab sample facilities are also provided at the Condensate Polishing Sample Station.

### **Condenser Sample Station**

The Condenser Sample Station is located in the Turbine Building. Process samples from the Main Condenser and Auxiliaries are routed to this panel for analysis.

Typical process stream sample parameters for the Condenser Sample Station are outlined in Table 9.3-1. Grab sample facilities are also provided at the Condenser Sample Station.

### **Radwaste Building Sample Station**

The Radwaste Building Sample Station is located in the Radwaste Building. This sample station maintains continuous conductivity monitoring of the equipment and floor drain inputs. Additional grab samples are drawn from selected locations in the Radwaste System. Fume hoods and ventilation exhausts are provided for grab samples as necessary.

### **Auxiliary Boiler Building Sample Station**

The Auxiliary Boiler Building Sample Station is located in the Auxiliary Boiler Building. This station is used to monitor the water quality of the Auxiliary Boiler System. Process samples from the Auxiliary Boiler System are routed to this panel for analysis.

Typical process stream sample parameters for the Auxiliary Boiler Building Sample Station are outlined in Table 9.3-1. Grab sample facilities are also provided at the Auxiliary Boiler Building Sample Station.

### **Local Grab Sampling Stations**

Local grab sampling points are located throughout the plant to monitor process streams requiring intermittent sampling. Local grab sampling points are provided for the following systems:

- Reactor Component Cooling Water System
- Turbine Component Cooling Water System
- Plant Service Water System
- Chilled Water System
- Circulating Water System
- Standby Liquid Control System

Grab sample stations are used when the operating conditions of the process stream being sampled are suitable for operator handling without any conditioning.

### **Sample Piping Design**

Sample lines are routed to be as short as possible, avoiding traps, dead legs and dips upstream of the sample stations. Lines are sized to maintain turbulent flow at the minimum required flow for each sample line.

The seismic design and quality group classifications of sample lines and their components conform to the classification of the system to which they are connected, up to and including the block valve (or valves). Sample lines downstream of the block valves are in conformance with ASME B31.1, Power Piping Code.

All sampling lines have the process isolation block valves located as close as practical to the process taps. These valves may be closed if sample line rupture occurs downstream of the valves.

#### **9.3.2.3 Safety Evaluation**

The operation of the PSS is not required for any of the following:

- Integrity of the reactor coolant pressure boundary;
- Capability to shut down the reactor and maintain it in a safe shutdown condition; and
- Ability to prevent or mitigate the consequences of accidents that can result in potential off-site exposures comparable to the guideline exposures of 10 CFR 100.

However, the system incorporates features that improve operator safety. The sampling stations are closed systems and the grab samples taken at the sampling stations have a chemical fume

hood to preclude the exposure of operating personnel to contamination hazards. A constant air velocity is maintained through the working face of the hoods to ensure that airborne contamination does not escape to the room under operating conditions.

In the event of a loss of cooling water to the sample cooler or sample flow in excess of sample cooler capacity, the sampling system valves are interlocked to prevent high-temperature water flow through the lines.

Safety/relief valves, vented to the drain headers, are provided in the stations for high-pressure process streams.

#### **9.3.2.4 Tests and Inspections**

The sample stations are in continuous use during normal plant operation, therefore PSS operability is continuously demonstrated during normal plant operation. The sample stations are tested and calibrated at frequencies in accordance with the sample station supplier's operation and maintenance requirements.

#### **9.3.2.5 Instrumentation Application**

PSS instrumentation is provided in each sample station for the following:

- Provisions are made to stop sample flow upon detection of high-temperature sample flow leaving the sample cooler;
- Pressure and temperature indication is provided for all high-pressure and high-temperature samples, respectively;
- Process conditions are measured and recorded for each sample flow;
- Alarms are provided for necessary measurement indications; and
- Provisions are made for sample flow to be indicated.

Additional monitoring equipment is provided within the panels to meet the process stream monitoring conditions listed in Table 9.3-1.

#### **9.3.2.6 COL Information**

The COL applicant shall develop a post accident sampling program to monitor, as a minimum, the parameters delineated in Table 9.3-1 and SRP 9.3.2.

#### **9.3.2.7 References**

- 9.3.2-1 NUREG 0800, Standard Review Plan, Section 9.3.2.II, "Process and Post-Accident Sampling Systems."
- 9.3.2-2 Regulatory Guide 1.21, "Measuring and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants." (see Table 1.9-21)
- 9.3.2-3 Regulatory Guide 1.26, "Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants." (see Table 1.9-21)

- 9.3.2-4 Regulatory Guide 1.29, "Seismic Design Classification." (see Table 1.9-21)
- 9.3.2-5 Regulatory Guide 1.33, "Quality Assurance Program Requirements." (see Table 1.9-21)
- 9.3.2-6 Regulatory Guide 1.56, "Maintenance of Water Purity in Boiling Water Reactors." (see Table 1.9-21)
- 9.3.2-7 Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident." (see Table 1.9-21)
- 9.3.2-8 Regulatory Guide 8.8, "Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable." (see Table 1.9-21)
- 9.3.2-9 ANSI/HPS N13.1, "Sampling and Monitoring Releases of Airborne Radioactive Substances from the Stacks and Ducts of Nuclear Facilities." (see Table 1.9-22)
- 9.3.2-10 ASME B31.1, Power Piping (see Table 1.9-22).

### 9.3.3 Equipment and Floor Drain System

The Equipment and Floor Drain System (EFDS) consists of the Clean (nonradioactive) Drain Subsystem and the following five potentially contaminated subsystems:

- Low Conductivity Waste (LCW) Drain Subsystem;
- High Conductivity Waste (HCW) Drain Subsystem;
- Detergent Drain Subsystem;
- Chemical Waste Drain Subsystem; and
- Reactor Component Cooling Water System (RCCWS) Drain Subsystem.

These five subsystems collect liquid drainage from various plant areas and transfer them to the radwaste system.

#### 9.3.3.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The EFDS does not perform any safety-related function. Therefore, EFDS has no safety design basis, other than provision for safety-related containment penetrations and containment isolation valves, as described in Subsection 6.2.4.

The EFDS meets GDC 2, by compliance with Regulatory Guide 1.29. The applicable sections of RG 1.29 include Position C.1 for safety-related portions and Position C.2 for Nonsafety-Related portions. The seismic and quality group classifications are identified in Table 3.2-1.

The EFDS meets GDC 4 related to environmental conditions. The EFDS design is not susceptible to external flooding that could result in adverse effects to safety-related SSC's.

The EFDS meets requirements of GDC 60 by providing a design to avoid the transfer of contaminated fluids to a non-contaminated drainage system for disposal.

## Power Generation Design Bases

The EFDS collects waste liquids from their point of origin and transfers them to a suitable processing or disposal system.

Drainage systems are designed to accommodate the maximum anticipated normal volumes of liquid without overflowing including such inputs as the anticipated water flow from a fire hose and other fire suppression water discharges to the area floor drains without impacting the safety function of any safety-related component or system. However, as delineated in Subsection 3.4.1.3, no credit is taken for the drainage system in the flooding analysis.

To preclude inadvertent transfer of radioactive liquids to non-radioactive systems, the radioactively contaminated or potentially contaminated liquids are collected by completely separate systems (e.g. no cross connections) from those that collect non-radioactive liquids.

Redundant sump pumps are included to increase the reliability, availability, and maintainability of the EFDS.

Systems are designed and arranged to minimize flooding of multiple compartments.

### 9.3.3.2 System Description

#### Summary Description

The EFDS includes sumps, motor-driven pumps, isolation valves, and instrumentation for pump operation, and interconnecting piping. Separate subsystems collect clean (nonradioactive) drains, low conductivity waste (LCW) drains, high conductivity waste (HCW) drains, detergent drains, chemical, and RCCWS drain wastes.

The Clean Drain Subsystem collects and transfers liquid wastes by gravity from the clean nonradioactive equipment and floor drains to sumps and pumps these wastes to an appropriate disposal system.

The LCW Drain Subsystem collects liquid wastes from equipment drains in potentially contaminated systems. These liquids gravity drain to sumps located in the drywell and other areas. The drywell LCW drain, which is monitored for activity, is pumped to the LCW collection tank. The drywell LCW sump pump discharge line is provided with redundant containment isolation valves. The liquid wastes collected in the LCW sumps are also pumped to the LCW collection tank.

The HCW Drain Subsystem collects liquid wastes from floor drains in potentially contaminated areas. These liquids gravity drain to sumps located in the drywell and other areas. The drywell HCW drain, which is monitored for activity, is pumped to the HCW collection tank. The drywell HCW sump pump discharge line is provided with redundant containment isolation valves. Liquids collected in the HCW sumps are also pumped to the HCW collection tank.

The Detergent Drain Subsystem collects potentially contaminated wastes from the personnel decontamination stations, laundry, and shower facility drains and transfers them to the detergent drain collection tank.

The Chemical Waste Drain Subsystem collects liquid wastes containing potentially contaminated chemicals and corrosive substances from washdown areas, laboratory drains, hot maintenance

shop, and other miscellaneous sources in the plant. These liquid wastes are transferred to the chemical drain collection tank.

Dedicated sumps in the EFDS shall collect vent and drain water from the closed loop RCCWS and shall be directed to the Reactor Building Cooling Water Drain Subsystem. The size of this subsystem shall accommodate the draining of the largest isolable cooling water pipe segment in the Reactor Building. The sump contents shall be evaluated for radioactivity and water quality. If the cooling water is radioactively contaminated, it shall be directed to the LWMS, where it can be processed. If not, the cooling water may be recycled through a line tied back to the cooling water system.

Safety divisions are provided with a separate drain line connecting to the main drainage piping and leading to an appropriate sump in both the Reactor Building and Control Building. Each drain line is provided with a normally closed manual valve, closed to prevent flooding of multiple safety divisions due to backflow. Watertight walls, floors, and doors on safety-related compartments also prevent flooding of multiple safety-related compartments.

The results of the flooding analysis are acceptable assuming the floodwater is retained in localized areas or zones, which is conservative in the determination of the resulting water level of these specific areas. The flooding analysis shows that the watertight doors and drain line isolation valves are not required to protect the safety-related equipment. They are included to provide additional protection and to minimize the spread of floodwater.

### **Detailed System Description**

The system includes the major equipment listed in Table 9.3-2.

The EFDS equipment is primarily located in the areas where the drains are collected. Capability is provided to sample the liquids collected in each sump.

Containment isolation valves and piping are classified as safety-related. All other EFDS equipment and components are Nonsafety-Related.

The EFDS interfaces with numerous systems from which drains are collected. The collected liquids are discharged to the clean waste system or the Liquid Waste Management System, as appropriate.

### **System Operation**

Liquid wastes from various floors and equipment drains gravity drain to the appropriate sumps. Each sump has two pumps. One pump operates as required and the other is in standby. The lead sump pump starts automatically when the liquid reaches a predetermined level in the sump and stops at a predetermined low level. Both pumps operate simultaneously if one pump cannot accommodate the rate of accumulation of liquids in the sump. Contaminated or potentially contaminated liquids are transferred to the Liquid Waste Management System for processing. Drywell sumps transfer collected liquids by pumping to the LCW or HCW collection tanks in the Radwaste Building.

#### **9.3.3.3 Safety Evaluation**

The EFDS does not have any safety-related function, other than containment isolation functions.

Failure of the EFDS does not prevent safety-related equipment from performing their safety-related functions. Section 3.4, Water Level (Flood) Design, presents analyses demonstrating that safety-related equipment in areas drained by the EFDS is not affected by drain or flood water backing up in the drainage system because of malfunction of active components, blockage, or the probable maximum flood.

#### ***9.3.3.4 Testing and Inspection Requirements***

The EFDS is designed to permit periodic inspection and testing of important components, such as valves, motor operators, and piping, to verify their integrity and capability. Equipment layout provides easy access for inspection and maintenance.

Drainage piping is hydrostatically tested prior to embedment in concrete. Potentially radioactive drainage piping is pressure tested in accordance with ASME B31.1. The EFDS components are tested periodically by normal operation of the equipment.

#### ***9.3.3.5 Instrumentation Requirements***

Instrumentation and controls for the EFDS are located at local panels, and appropriate signals are duplicated and sent to the MCR. These indications and controls include the following:

- On/off switches for pumps;
- Sump level alarms; and
- Valve position indicators and controls.

The EFDS can be controlled manually or can be automatically operated as described below.

High and low level transmitters on each sump start and stop the sump pump automatically. A separate high-high level switch starts the second sump pump and actuates an alarm in the MCR.

Leaks in the drywell are detected by monitoring the rate of increase of the sump level. Leak detection in other areas is accomplished by monitoring the frequency and duration of sump pump operation.

#### ***9.3.3.6 COL Information***

None.

#### ***9.3.3.7 References***

9.3.3-1 ASME B31.1, Power Piping (see Table 1.9-22).

### **9.3.4 Chemical and Volume Control System**

Not applicable to ESBWR.

### 9.3.5 Standby Liquid Control System

#### 9.3.5.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The Standby Liquid Control (SLC) system performs safety-related functions, therefore it is classified as safety-related and is designed as a Seismic Category I system. The SLC system meets the following safety design bases:

The SLC system provides a diverse backup capability for reactor shutdown that is independent of normal reactor shutdown provisions.

The SLC system provides makeup water to the RPV to mitigate the consequences of a Loss-of-Coolant-Accident (LOCA).

The SLC system has full capacity for reducing core reactivity between the steady-state rated operating condition of the reactor with voids and the reactor cold shutdown condition, including shutdown margin, to ensure complete shutdown from the most reactive conditions at any time in core life.

The time required for actuation and effectiveness of the SLC system is consistent with the nuclear reactivity rate-of-change corresponding to the normal reactor rate of cooldown and depressurization and the accompanying influence of xenon decay. A fast scram of the reactor or operational control of fast reactivity transients is not specified for the SLC system.

Means are provided to ensure the capability of the SLC system to respond to an initiation demand by confirming adequate accumulator pressure and solution level, and initiating circuit continuity and proper valve positions. Additional measures are capable of being taken during plant shutdown to confirm injection valve operability through in-position or in-laboratory firing tests. Provisions are made to confirm an absence of obstructions in the flow path, and to confirm adequacy of the solution concentration during plant operation.

The SLC system injects a boron solution, which performs as a neutron absorber, at multiple locations into the core bypass region at high velocity. This ensures adequate mixing and total injection of the solution to accomplish reactor shutdown. The injection geometry ensures balancing of reaction forces.

The SLC system is reliable to a degree consistent with its role. The potential for unintentional or accidental shutdown of the reactor by this system is minimized.

##### Power Generation Design Bases

None.

#### 9.3.5.2 System Description

##### Summary Description

The SLC system (Figure 9.3-1) is manually initiated for its shutdown function described in Subsection 9.3.5.1. In addition, the system is automatically initiated for Anticipated Transients Without Scram (ATWS) and LOCA events. The SLC system process conditions are shown in Figure 9.3-1a.

### Detailed System Description

The SLC system contains two identical and separate trains. Each train provides 50% injection capacity. All components of the SLC system in contact with the boron solution are constructed of, or lined with, stainless steel. The safety-related portion of each train of the SLC is comprised of the components and groups of components listed in Table 9.3-3.

The SLC system requires support from the safety-related systems listed in Table 9.3-4.

The SLC system also includes a Nonsafety-Related Nitrogen charging subsystem that includes a liquid nitrogen tank, vaporizer, and high-pressure pump for initial accumulator charging and makeup for the normal system losses during normal plant operations.

Control of the equipment compartment temperature and humidity conditions avoids solute precipitation in the accumulator or injection line, thereby ensuring proper system operation. This system readiness function is Nonsafety-Related.

The major components of the SLC system, which are necessary for injection of Sodium Pentaborate Solution (neutron absorber) into the reactor, are located within the Reactor Building. The Nonsafety-Related high-pressure cryogenic nitrogen equipment is located outside the Reactor Building at grade elevation.

The sparger system is located within the reactor vessel and those portions of the SLC injection line downstream of the squib valve contain only cold stagnant reactor water.

Electrical heating of the accumulator tank and the injection line is not necessary because the saturation temperature of the boron solution is less than 15.5°C (60°F) and the equipment room temperature is maintained above that value at all times when SLC is required to be operable. Adiabatic expansion of the cover gas can lead to temperatures less than the saturation temperature. The low rate of heat transfer between the cover gas and the solution and the rapid rate of injection of the solution ensure that neither precipitation nor freezing affect the solution injection process or its effectiveness.

### System Operation

The SLC system can be manually initiated from the MCR to inject a boron solution into the reactor if the operator determines the reactor cannot be shutdown or kept shutdown using the control rods. Once the operator decision for initiation of the SLC system is made, the design intent is to simplify the manual process by providing dual, protected switches. This prevents inadvertent injection of the boron solution.

The dual manual initiation switches in the MCR are of a recessed, spring-loaded, rotate and push type and have a protective switch cover. Actuation requires simultaneous depression of the switches, and the switches are located such that a single operator can readily initiate the system. Procedural controls govern operation of these initiation switches.

The SLC system is required to shutdown the reactor and keeps the reactor from going critical again during cooldown. The SLC system is needed in the improbable event that insufficient control rods can be inserted in the reactor core to accomplish shutdown and cooldown in the normal manner.

The SLC system is also designed to provide makeup water to the RPV during a LOCA event by injecting the boron solution from both accumulators. The Emergency Core Cooling System

(ECCS) and the SLC system are designed to flood the core during a LOCA event to provide the required core cooling. By providing core cooling following a LOCA, the ECCS and the SLC system, in conjunction with the containment, limits the release of radioactive materials to the environment.

The shutdown functional performance requirements of the SLC system are bounded by the ATWS event performance requirements shown in Table 9.3-5. For ATWS events, the failure of control rods to insert in response to a valid trip demand is assumed. The SLC system automatically initiates by the Average Power Range Monitor (APRM) not downscale ( $\geq 6\%$ ) and one of the following conditions persisting for at least 3 minutes:

- High reactor dome gauge pressure of  $\geq 7.76$  MPa ( $\geq 1125$  psig); and
- Low reactor vessel water level ( $\leq$  Level 2).

The three-minute delay provides time for completion of the FMCRD motor-driven run-function before initiation of the SLC system, thereby preventing the unnecessary injection of the boron solution into the reactor.

Sodium Pentaborate Solution injection ensures a timely accomplishment of hot shutdown. Subsequent injections as the reactor depressurizes ensure that cold shutdown can be achieved with no further occurrence of critical conditions. Refer to Section 15.5 for SLC system performance in the evaluation of ATWS events.

During a LOCA or an ATWS event, at the completion of the boron solution injection, redundant accumulator level measurement instrumentation using 2-out-of-4 logic close the two redundant injection line shut-off valves. The closing of these valves prevent the injection of nitrogen from the accumulator into the reactor vessel. In the event of a single shut-off valve failure, the remaining valve is sufficient to prevent the injection of nitrogen into the reactor vessel. An accumulator vent is also provided, which can be used to quickly reduce the pressure in the accumulator.

### ***9.3.5.3 Safety Evaluation***

The SLC system is mainly a reactivity control system that is maintained in an operable status whenever the reactor is critical. A large number of independent control rods are available to shutdown the reactor including redundant and diverse methods to insert the control rods, at any time during the core life.

Availability of the SLC system is ensured by redundancy in the injection valves. Adequate functioning of the system is ensured if one of the two injection valves open in each train. No other function is required for proper system operation. Addition of nitrogen to recover gas pressure after initial injection is not necessary for adequate functioning of the system.

The system is designed to bring the reactor from rated power to a cold shutdown condition at any time in core life. The reactivity compensation provided reduces reactor power from rated to zero and allows cooling of the nuclear system to less than the cold shutdown temperature with the control rods remaining withdrawn in the rated power pattern. These conditions (hot shutdown and cold shutdown) include, where applicable, the reactivity gains that result from complete decay of the rated power xenon. They include the positive reactivity effects from eliminating steam voids, changing water density from hot to cold, reduced Doppler effect in uranium,

reducing neutron leakage from boiling to cold, and decreasing control rod worth as the moderator cools.

The minimum uniformly mixed equivalent concentration of natural boron required in the reactor core to provide adequate cold shutdown margin after operation of the SLC system is 760 ppm. Calculation of the minimum quantity of isotopically enriched sodium pentaborate to be injected into the reactor is based on the required 760 ppm equivalent natural boron concentration in the reactor coolant at 20°C (68°F) and reactor water level conservatively taken at the elevation of the bottom edge of the main steam lines. This result is then increased by a factor of 1.25 to provide a 25% general margin to discount potential non-uniformities of the mixing process within the reactor. This result is then increased by a factor of 1.15 to provide a further margin of 15% to discount potential dilution by the RWCU/SDC System in the shutdown cooling mode.

Cooldown of the nuclear system requires a minimum of several hours to remove the thermal energy stored in the reactor, reactor water, and associated equipment. The limit for the reactor vessel cooldown is 55.6°C/hr (100°F/hr) and the normal operating temperature is approximately 288°C (550°F). Use of the main condenser and the shutdown cooling systems normally requires 10 to 24 hours to decrease the reactor vessel temperature to 20°C (68°F). Although hot shutdown is the condition of maximum reactivity, cold shutdown condition is associated with the largest total water mass in which the particular shutdown concentration must be established and therefore, this condition determines the total mass of boron solution to be injected.

The extremely rapid initial rate of isotopically enriched boron injection ensures that hot shutdown boron concentration is achieved within several minutes of SLC system initiation based on initial reactor water inventory. Maintaining normal water level with the voids collapsed causes some dilution of this concentration but does not cause hot shutdown concentrations to be violated. As the reactor cools and begins to depressurize, completion of all boron solution injection occurs long before cold shutdown conditions are reached. The high injection velocity of the injection spargers and the natural circulation flow within the reactor vessel ensures efficient mixing and distribution of the boron throughout the reactor vessel.

The SLC system is designed to conform with the requirements for equivalent reactivity control capacity specified in 10 CFR 50.62(c)(4).

The SLC system equipment is Safety-Related for injection of boron solution into the reactor is designed as Seismic Category I for withstanding the specified earthquake loadings (refer to Section 3.7). The system piping and equipment are designed, installed, and tested in accordance with the requirements stated in Section 3.9.

The safety functions of the SLC system are powered from the Class 120 VAC electrical systems. Environmental conditions to prevent precipitation of solute do not require operation of the Reactor Building HVAC systems during the time that SLC operation is required.

The initial accumulator tank inventory of compressed nitrogen is adequate to ensure full injection of the boron solution inventory at a reactor pressure of 6.9 MPa (1000 psia). After the boron injection is complete, the redundant shut-off valves close to prevent the injection of nitrogen into the reactor vessel. A single shut-off valve is sufficient to prevent nitrogen from entering the reactor vessel. Protection against inadvertent premature operation of the shut-off valve is ensured by use of redundancy in the initiation signal for this function.

The SLC system is evaluated against the applicable General Design Criteria (GDC) as follows:

The SLC system is designed to conform to the GDC of 10 CFR 50. The overall requirements (Section I) of the GDC are applicable to the system, and the system equipment has been designed and installed in conformance with the presentations in Chapter 3. For its function to provide makeup water to the RPV during a LOCA, the SLC system is designed to meet the requirements of GDC 17, 35, 36 and 37 and 10 CFR 50.46 in conjunction with the other ECC systems. Conformance to these criteria is discussed in Section 6.3, Emergency Core Cooling Systems. Other related GDCs are presented individually below:

**Criterion 26, Reactivity Control System Redundancy and Capability:** The FMCRD system is the intended means for normal operational control of reactivity. Anticipated operational occurrences initiate the hydraulic scram function and rapidly insert all rods to ensure a safe limit on reactivity changes. These systems are capable of safely accommodating all reactivity changes from the normal power range (including xenon influences) to cold shutdown conditions. The SLC system is redundant to these functions and represents a diverse means for reactor shutdown to ensure the ability to bring and maintain the core to such critical conditions for the full range of reactor conditions from hot shutdown to cold shutdown.

**Criterion 27, Combined Reactivity Control Systems Capability:** Operation of the SLC system is not required to meet the intent of this criterion. The system could, for many conditions exceeding the design basis of the plant, provide additional reactor shutdown capability to ensure achieving shutdown conditions.

**Criterion 28, Reactivity Limits:** This criterion applies only to the reactivity control systems intended for normal operation.

The SLC system is evaluated against the applicable regulatory guides as follows:

**Regulatory Guide 1.26, Quality Group Classifications and Standards:** Because the SLC system is an ECC system, all mechanical components required for boron injection are at least Quality Group B. Those portions that are part of the reactor coolant pressure boundary are Quality Group A.

**Regulatory Guide 1.29, Seismic Design Classification:** All components of the SLC system necessary for injection of the boron solution into the reactor are Seismic Category I.

**SRP Branch Technical Positions SPLB 3-1 and EMEB 3-1:** Because the SLC system is located within its own compartment inside the Reactor Building, it is adequately protected from flooding, tornadoes, and internally/externally generated missiles. The SLC system equipment is protected from pipe break by providing adequate distance between the Seismic Category I and non-seismic SLC equipment, where such protection is necessary. In addition, appropriate distance is provided between the SLC system and other high energy piping systems. This system is used in ATWS events presented in Section 15.5. The ATWS events are extremely low probability postulated events.

### **Testing and Inspection Requirements**

The SLC system requires firing of only one of two squib valves in each train and no dynamic equipment for operation under design conditions. Assurance that design conditions are maintained is accomplished by critical parameter alarms and periodic surveillances.

Critical parameters (accumulator level and pressure) are alarmed and recorded in the MCR.

Valves, including the injection valves, shut-off valves, vent valves, and relief valves, are periodically tested to ensure operability.

The SLC system preoperational test to demonstrate adequate system performance is described in Subsection 14.2.8.

During scheduled plant shutdowns, the pyrotechnic charges are replaced in the injection line squib valves. Replacement is done without any opening of the reactor coolant pressure boundary (RCPB). Subsequently in the laboratory, the removed charges are tested to confirm end of life capability to function upon demand. Access for charging each accumulator with boron solution, draining the accumulator, and sampling from each accumulator is provided. Fill-and-drain and makeup operations are intended to be very infrequent operations (several year intervals). Periodic samples are taken to ensure acceptable solution characteristics. Charging the solution inventory is performed with the accumulator depressurized. Provision is made for a minimum of 8 sample withdrawals without a requirement for makeup. Sampling and makeup of the solution inventory can be performed with the accumulator at full pressure.

#### **9.3.5.4 Instrumentation Requirements**

The instrumentation and control system for the SLC system is designed to allow the injection of liquid poison into the reactor and to ensure that the liquid poison solution and its cover gas are maintained within the allowable range of initial conditions to achieve full solution injection upon demand.

Because of the passive nature of the system and the short operating time for the system, no provision is made for flow measurement. Injection status is provided by MCR indication of accumulator pressure and accumulator solution level for each train. These parameters are direct and independent indicators of injection quantity given normal operation of the system. Because of the high rate of injection by this system, verification of injection is almost immediately available (< 30 sec) by observation of accumulator level. The verification can be confirmed by observation of accumulator pressure, which is also a direct measure of injection under conditions of normal, unfaulted system operation.

Closure of the redundant injection shut-off valves is automatically initiated by the accumulator level instrumentation using 2-out-of-4 logic. Closure, or override, of the automatic closure can also be initiated manually from the MCR. Operation of the accumulator vent system is manual from the MCR. Operation of the accumulator nitrogen charging, and makeup to accommodate small losses is manual. MCR alarms are provided for high, low, and low-low conditions of accumulator pressure and low and low-low conditions of accumulator solution level. At low-level conditions, the nitrogen and poison solution makeup systems are manually started. Accumulator temperature, solution level, and accumulator pressure instrumentation is provided locally inside the accumulator room.

The status of injection and shut-off valves vital to the operation of the system is provided in the MCR.

A further discussion of the SLC system instrumentation may be found in Section 7.4.

#### **9.3.5.5 COL Information**

None.

#### **9.3.5.6 References**

None.

### **9.3.6 Instrument Air System**

#### **9.3.6.1 Design Bases**

##### **Safety (10 CFR 50.2) Design Bases**

The Instrument Air System (IAS) is a Nonsafety-Related system and has no safety design basis.

The IAS is a Nonsafety-Related system and meets the requirements of GDC 1 as it pertains to minimum instrument air quality standards by meeting Instrument Society of America (ISA) 7.0.01. The IAS also meets the requirements of GDC 1 as it pertains to the guidance of Regulatory Guide (RG) 1.68.3, related to operational testing of instrument air system.

The IAS meets the requirements of GDC 2 as it pertains to Position C.2 of Reg. Guide 1.29 for Nonsafety-Related systems. The failure of the IAS does not impact any safety-related structures and will not degrade a safety-related system to an unacceptable safety level.

The IAS meets GDC 5 for shared systems and components important to safety. The IAS Standard Plant design does not share any SSC with any other unit.

The requirements of RG 1.155 are met by providing a design to shut down safely without reliance on offsite or diesel-generator-derived AC power for 72 hours, which exceeds station blackout requirements.

##### **Power Generation Design Bases**

The function of the IAS is to provide dry, oil free, filtered compressed air to pneumatically operated valve operators, instrumentation, equipment and components.

The IAS is designed with sufficient capacity to operate during normal plant operation, transients, plant startup, and plant shutdown.

#### **9.3.6.2 System Description**

The IAS provides filtered, dry and oil-free compressed air for plant instrumentation, control systems and pneumatic valve/damper actuators located outside of the containment during normal operation. During refueling operations, the IAS supplies compressed air to the High Pressure Nitrogen Supply System (HPNSS) loads inside containment via the HPNSS piping. The IAS is shown in Figure 9.3-2.

Design of the system ensures that failure of the IAS does not compromise any safety-related system or component nor does it prevent a safe shutdown. Pneumatically operated devices are

designed fail-safe and do not require continuous air supply under emergency or abnormal conditions.

The IAS consists of two identical 100% capacity trains in parallel, one normally operating and the other in standby. Components of the IAS are designed to meet ASME Boiler and Pressure Vessel Code, Section VIII, Division 1, ASME Power Piping Code B31.1, or ASME Process Piping Code B31.3, as applicable.

During normal operation, one compressor is selected for continuous operation, while the other serves as standby and starts automatically if the continuously operating compressor cannot meet system demand. The operating compressor automatically loads or unloads in response to the IAS demand as determined by pressure changes in the air receivers.

The IAS is automatically switched to the standby AC power source during a LOPP and is backed up by the Service Air System (SAS) upon low system pressure. The instrument air compressor units are powered from separate electrical buses.

Each of the compressors takes suction through an air intake filter/silencer. Each compressor train is equipped with an after-cooler. An air receiver services each air compressor. Air, leaving the air receivers, passes through one of two parallel instrument air filtering and drying trains, and a receiver/surge tank before being distributed to the instrument air piping system. Each train is equipped with redundant prefilters, a regenerative desiccant dryer, and redundant after-filter. Both air compressor trains are connected to a common header, which distributes to the Radwaste Building, Turbine Building and Reactor Building.

The cooling water for compressors and after-coolers is supplied from the Reactor Component Cooling Water System.

Performance Requirements for the IAS are provided in Table 9.3-6.

#### ***9.3.6.3 Safety Evaluation***

The IAS is not a safety-related system, however, the IAS incorporates features that ensure its operation over the full range of normal plant operations. If IAS pressure falls below a desired limit, air from the Service Air System (SAS) is automatically added from a tie line. Air receivers and a receiver/surge tank are provided to maintain air supply pressure if all of the IAS and SAS compressors fail. Pneumatically operated devices are designed to fail-safe and do not require continuous air supply under emergency or abnormal conditions.

#### ***9.3.6.4 Inspection and Testing Requirements***

IAS operability is demonstrated by use during normal plant operation. Compressors, after-coolers, air receivers, prefilters, air dryers, after-filters, and the control panel are shop inspected and tested. System operational tests for components normally closed to airflow are performed periodically to ensure system capability and integrity. Air filters are periodically inspected for cleanliness, and the desiccant in the air dryers is periodically sampled to verify its useful life. Periodic testing of air quality is performed to ensure compliance with ISA Standard 7.0.01. Pneumatic isolation valves are capable of operational integrity testing. A remote manual switch and open/closed position lights are provided in the Main Control Room (MCR) for verification of proper valve operation.

Additional testing details of IAS are described in Subsection 14.2.8.1.19.

### **9.3.6.5 Instrumentation Application**

Instrumentation for the IAS is primarily local, consisting of pressure, differential pressure and temperature and/or control. Pressure transmitters and pressure switches provide MCR pressure indications and alarms. The system is maintained at constant pressure, with local pressure reduction provided as required.

Pressure-reducing valves are used for services requiring less pressure than the respective receiver tank pressure.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

### **9.3.6.6 COL Information**

None.

### **9.3.6.7 References**

- 9.3.6-1 ISA Standard 7.0.01, Quality Standard for Instrument Air (see Table 1.9-22)
- 9.3.6-2 ASME Boiler and Pressure Vessel Code (B&PVC), Section VIII – Pressure Vessels (see Table 1.9-22)
- 9.3.6-3 ASME B31.1, Power Piping (see Table 1.9-22)
- 9.3.6-4 ASME B31.3, Process Piping (see Table 1.9-22)

## **9.3.7 Service Air System**

### **9.3.7.1 Design Bases**

#### **Safety (10 CFR 50.2) Design Bases**

The Service Air System (SAS) is a Nonsafety-Related system and has no safety design basis, other than provisions for safety-related containment penetrations and isolation valves, as described in Subsection 6.2.4.

The SAS is a Nonsafety-Related system and meets the requirements of GDC 1 as it relates to air quality standards or operational testing of instrument air system by providing an interconnection to IAS. See Subsection 9.3.6.1.

The SAS meets GDC 2, by compliance with Regulatory Guide 1.29. The applicable sections of RG 1.29 include Position C.1 for safety-related portions and Position C.2 for Nonsafety-Related portions. The seismic and quality group classifications are identified in Table 3.2-1.

The SAS meets GDC 5 for shared systems and components important to safety. The SAS Standard Plant design does not share any SSC with any other unit.

The requirements of RG 1.155 are met by providing a design to shut down safely without reliance on offsite or diesel-generator-derived AC power for 72 hours, which exceeds station blackout requirements.

### **Power Generation Design Bases**

The SAS is designed with redundant active components.

The SAS has sufficient capacity to support plant operations during normal, transients, startup, and shutdown conditions.

#### **9.3.7.2 System Description**

The SAS provides filtered compressed air for general plant use and service air outlets located outside of the containment. Breathing air is also provided by the SAS. During Instrument Air System (IAS) failure, the SAS can supply compressed air to the IAS loads.

The SAS is shown in Figure 9.3-3.

The SAS is designed to provide compressed air of suitable quality for Nonsafety-Related functions.

The design ensures that failure of the SAS does not compromise the safety-related function of any safety-related system or component or prevent a safe shutdown.

The SAS provides compressed air for tank sparging, filter/demineralizer backwashing, air-operated tools and other services requiring air of lower quality than that provided by the IAS.

The SAS consists of two identical trains in parallel, one normally operating, and the other in standby. Components of the SAS are designed to meet ASME Boiler and Pressure Vessel Code, Sections III and VIII, Division 1, and ASME Power Piping Code B31.1, and ASME Process Piping Code B31.3, as applicable.

During normal operation, one compressor train is selected for continuous operation, while the other serves as standby and starts automatically if the continuously operating compressor cannot meet system demand. The operating compressor automatically loads or unloads in response to the SAS demand as determined by pressure changes in the air receivers.

Each of the compressors takes suction through an air intake filter/silencer. Each compressor train is equipped with an inter-cooler, after-cooler, moisture separator, and air receiver. Both air compressor trains are connected to a common header that distributes air to the breathing air purifiers, Radwaste Building, Turbine Building, and Reactor Building. The SAS also serves as a backup to the IAS.

The breathing air purifiers consist of two identical 100% trains in parallel, one normally operating, and the other in standby. The major components of the breathing air purifiers are designed to meet ASME Boiler and Pressure Vessel Code, Section VIII, Division I, ASME Piping Code B31.3, and Occupational Safety and Health Administration (OSHA) breathing air requirements.

The SAS supplies service air to the breathing air purifiers. Each breathing air purifier is equipped with prefilter, twin desiccant towers, catalyst, and activated charcoal filter. One air receiver with a bypass line serves both purifiers. Both breathing air purifiers are connected to a

common header that supplies the Reactor Building, Turbine Building, Radwaste Building, and Hot Machine Shop.

The cooling water for after-coolers and compressors is supplied from the Turbine Component Cooling Water System.

The service air compressor units are powered from separate electrical buses of the permanent Nonsafety-Related power distribution system.

The SAS includes system connections for connecting additional compressor capacity for high-usage periods, if required.

Performance requirements of the SAS are provided in Table 9.3-7.

#### ***9.3.7.3 Safety Evaluation***

The SAS does not have any safety-related functions, except for containment isolation. The SAS incorporates features to ensure its operation over the full range of normal plant operations. Pneumatic-operated devices are designed to fail-safe and do not require continuous air supply under emergency or abnormal conditions.

#### ***9.3.7.4 Inspection and Testing Requirements***

The SAS operability is demonstrated by use during normal plant operation. Compressors, inter-coolers, after-coolers, moisture separators, air receivers, breathing air purifiers, and the control panels are shop inspected and tested. System operational tests for components normally closed to airflow are performed periodically to ensure system capability and integrity. Filters are periodically inspected for cleanliness, and the desiccant in the breathing air purifiers is periodically sampled to ensure its performance. Periodic testing of breathing air quality is performed to ensure compliance with OSHA standards (29 CFR 1910).

Additional testing details of SAS are described in Subsection 14.2.8.1.19.

#### ***9.3.7.5 Instrumentation Application***

Instrumentation for the SAS is primarily local, consisting of pressure, differential pressure and temperature indication and/or control. Pressure transmitters and pressure switches provide indications and alarms in the Main Control Room (MCR). The system is maintained at constant pressure, with local pressure reduction provided as required.

Pressure-reducing valves are used for services requiring less pressure than the respective receiver tank pressure.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### ***9.3.7.6 COL Information***

None.

### **9.3.7.7 References**

- 9.3.7-1 ASME Boiler and Pressure Vessel Code (B&PVC), Section III – Rules for Construction of Nuclear Power Plant Components (see Table 1.9-22)
- 9.3.7-2 ASME Boiler and Pressure Vessel Code (B&PVC), Section VIII – Pressure Vessels (see Table 1.9-22)
- 9.3.7-3 ASME B31.1, Power Piping (see Table 1.9-22)
- 9.3.7-4 ASME B31.3, Process Piping (see Table 1.9-22)

## **9.3.8 High Pressure Nitrogen Supply System**

### **9.3.8.1 Design Bases**

#### **Safety (10 CFR 50.2) Design Bases**

The High Pressure Nitrogen Supply System (HPNSS) does not perform any safety-related function. Therefore, the HPNSS has no safety design basis other than provision for safety-related containment penetrations and isolation valves, as described in Subsection 6.2.4.

#### **Power Generation Design Bases**

The HPNSS distributes clean, dry, oil-free nitrogen gas to containment nitrogen users from the Containment Inerting System (CIS). Nitrogen loads include the Automatic Depressurization Subsystem (ADS) Safety Relief Valve (SRV) accumulators, the Isolation Condenser (IC) steam and condensate line isolation valve accumulators, the Main Steamline Isolation Valve (MSIV) accumulators, and other pneumatically operated valves.

The HPNSS provides a means for switchover from CIS to nitrogen bottle backup during low CIS supply pressure.

The HPNSS provides a means for manual switchover from the HPNSS supply to the IAS supply to provide an operating gas for HPNSS loads during refueling outages.

### **9.3.8.2 System Description**

#### **Summary Description**

The HPNSS consists of the distribution piping between the CIS and the containment nitrogen users. The HPNSS also provides bottled high-pressure nitrogen gas that is clean, dry, and oil-free to the NBS ADS SRV accumulators, IC line isolation valves' accumulators, MSIVs, and other nitrogen loads if the CIS fails to maintain the required nitrogen supply pressure.

All containment nitrogen loads are normally supplied by the CIS. The CIS nitrogen supply line branches into two HPNSS distribution lines. One branch line supplies the low-pressure nitrogen loads (i.e., MSIVs, instruments, and pneumatic-operated valves) while the second branch supplies the high-pressure nitrogen loads (i.e., NBS ADS SRV accumulators and the IC piping isolation valve accumulators).

#### **Detailed System Description**

The HPNSS is shown in Figure 9.3-4.

The HPNSS provides the distribution piping from the CIS to the nitrogen loads in the containment. To meet the requirements of the accumulators, a pressure reducing station initially reduces the CIS nitrogen gas pressure. One of the two branch lines distributes high-pressure nitrogen to the accumulators. The remaining branch line incorporates a pressure reducing station to lower the CIS nitrogen supply to the required pressure.

The HPNSS incorporates containment isolation valves where the HPNSS supply lines penetrate the containment. An air-operated isolation valve is located outboard of the containment penetration while a check valve, which also functions as an isolation valve, is provided inboard. The check valve prevents backflow while the isolation valve closes. The containment isolation can be initiated manually from the MCR by closing the outboard isolation valve.

The HPNSS backup consists of two bottle-rack trains in parallel. Each bottle-rack train consists of 8 or more bottles, each bottle with a pigtail and station valve, which are piped to a manifold. Each bottle rack manifold interfaces with the nitrogen supply distribution header through an isolation valve. The supply header incorporates a pressure reducing station to depressurize the bottled nitrogen gas to the required supply distribution pressure. The nitrogen bottle racks each have about 8-hour supply for the containment nitrogen loads upon loss of normal CIS supply.

The piping and valves of the HPNSS are designed to meet ASME Piping Code B31.1. Its safety class is Nonsafety-Related.

HPNSS requirements are tabulated in Table 9.3-8.

### **System Operation**

The HPNSS distributes CIS nitrogen supply to the loads in the containment during normal operation. An upstream pressure control valve on each branch modulates the CIS nitrogen supply to provide the required nitrogen supply pressure to the nitrogen loads.

If the CIS fails to maintain the required nitrogen supply pressure, the HPNSS provides uninterrupted nitrogen gas supply from the nitrogen storage bottles. When the nitrogen gas pressure in the main header drops below the set pressure, the manifold isolation valve automatically opens to provide nitrogen gas from the storage bottles to all nitrogen loads.

One bottle rack train and one pressure-reducing station are utilized to maintain design nitrogen supply as required. The nitrogen bottle station valves and manifold isolation valve on one train are kept open, while the standby train bottle station valves and manifold isolation valves are kept closed. Switchover from one bottle rack train to the other is a manual operation.

The HPNSS bottled nitrogen normally remains on standby, through an isolation valve located upstream of the pressure reducing station. During low nitrogen supply pressure in the main supply header, the isolation valve automatically opens to allow nitrogen gas supply from the HPNSS nitrogen bottles to all system loads. The normal CIS supply line is isolated following a set time delay.

Restoration of the HPNSS to the CIS is by manual operation of the isolation valves from the MCR.

### ***9.3.8.3 Safety Evaluation***

The HPNSS is not safety-related, however, the system incorporates features that ensure reliable operation over the full range of normal plant operation and with earthquake intensities up to an SSE. Pneumatic-operated devices are designed for a fail-safe mode and do not require continuous air supply under emergency or abnormal conditions.

### ***9.3.8.4 Inspection and Testing Requirements***

The HPNSS is designed such that it can be tested at any time.

Periodic testing of nitrogen quality is performed to ensure compliance with ISA 7.0.01. Nitrogen isolation valve integrity testing is performed by manual actuation by MCR operators using associated valve position indicating lights.

### ***9.3.8.5 Instrumentation Requirements***

The flow of nitrogen gas from the HPNSS bottled nitrogen is automatically initiated when a pressure sensor in the main supply line indicates low nitrogen supply pressure. The pressure switch also alarms in the MCR upon either low or high pressure. The CIS supply isolation valve closes following a time delay.

### ***9.3.8.6 COL Information***

None.

### ***9.3.8.7 References***

9.3.8-1 ISA Standard 7.0.01, Quality Standard for Instrument Air (see Table 1.9-22)

## **9.3.9 Hydrogen Water Chemistry System**

The ESBWR Standard Plant design includes the capability to connect a Hydrogen Water Chemistry System (HWCS), but the system itself is not part of the ESBWR Standard Plant design.

### ***9.3.9.1 Design Bases***

#### **Safety (10 CFR 50.2) Design Bases**

The HWCS is Nonsafety-Related. Therefore, the HWCS has no safety design basis. However, it is required to be safe and reliable, consistent with the requirements for using hydrogen gas.

#### **Power Generation Design Basis**

Provisions are made to permit installation of a system for adding hydrogen to the feedwater at the suction of the feedwater pumps. If experience shows it necessary, a hydrogen water injection system can be added later in plant life.

The HWCS utilizes the guidelines in EPRI report "BWR Hydrogen Water Chemistry Guidelines".

### **9.3.9.2 System Description**

The HWCS, illustrated in Figure 9.3-5, is composed of hydrogen and oxygen supply systems to inject hydrogen in the feedwater and oxygen in the offgas and several monitoring systems to track the effectiveness of the HWCS. The hydrogen and oxygen demand requirements and supply systems are site dependent and shall be defined by the COL Applicant/Holder at the time of deployment, if implemented. The hydrogen supply system may be integrated with the generator hydrogen supply system (as described in Subsection 10.2.2.2.8).

### **9.3.9.3 Safety Evaluation**

The HWCS is not safety-related, however, the HWCS is used, along with other measures, to reduce the likelihood of corrosion failures that would adversely affect plant availability. The means for storing and handling hydrogen utilize the guidelines in EPRI report "Guidelines for Permanent BWR Hydrogen Water Chemistry Installations".

### **9.3.9.4 Inspection and Testing Requirements**

The HWCS, if necessary, can be installed at the connection points provided. The COL Applicant/Holder shall provide the inspection and testing requirements at the time of deployment.

### **9.3.9.5 Instrumentation and Controls**

The necessary instrumentation can be provided to control the injection of hydrogen and augment the injection of oxygen. Automatic control features in the HWCS minimize the need for operator attention and improve performance.

### **9.3.9.6 COL Information**

COL Holder will determine Oxygen and Hydrogen demand requirements and supply system, if HWC is implemented.

COL Holder will furnish necessary information Test and Inspection when vendor information becomes available.

### **9.3.9.7 References**

- 9.3.9-1 Electric Power Research Institute, "BWR Hydrogen Water Chemistry Guidelines," EPRI Report NP-4947-SR (see Table 1.9-22).
- 9.3.9-2 Electric Power Research Institute, "Guidelines for Permanent BWR Hydrogen Water Chemistry Installations," EPRI Report NP-5283-SR-A (see Table 1.9-22).

## **9.3.10 Oxygen Injection System**

### **9.3.10.1 Design Bases**

#### **Safety (10 CFR 50.2) Design Bases**

The Oxygen Injection System (OIS) does not perform any safety-related function. Therefore, the OIS has no safety design basis.

## **Power Generation Design Bases**

The OIS is designed to add sufficient oxygen to the condensate system to suppress corrosion and corrosion product release in the Condensate and Feedwater System. Experience has shown that the preferred feedwater oxygen concentration is 30 to 200 ppb. During startup and shutdown operation, the feedwater oxygen concentration is usually greater than the 30 to 200 ppb range. However, during power operation, deaeration in the main condenser can reduce the condensate oxygen concentration below 30 ppb, requiring that oxygen be added. The main function of the Oxygen Injection System is to maintain the oxygen concentration in the condensate and feedwater between 30 and 200 ppb with a target of less than 100 ppb during reactor operation.

### ***9.3.10.2 System Description***

The condensate oxygen injection module and the feedwater injection module are provided with pressure regulators and associated piping, valves, and controls to depressurize the gaseous oxygen and route it to the condensate and feedwater injection modules. There are check valves and isolation valves between the oxygen injection modules and the condensate lines downstream of the condensate demineralizers and the feedwater downstream of the direct contact feedwater heater.

The flow regulating valves in this system are operated from the MCR. Analyzers in the Process Sampling System (Subsection 9.3.2) monitor the oxygen concentration in the condensate / feedwater system. An operator can make changes in the oxygen injection rate in response to changes in the condensate / feedwater concentration. An automatic control system is not required because instantaneous changes in oxygen injection rate are not required.

### ***9.3.10.3 Safety Evaluation***

The injection system itself is not safety-related. The oxygen storage facility is located in an area where the amount of combustible material is limited through design (permanent combustible material) and administrative controls (temporary combustibles). Normal safe practices for handling high-pressure gases are followed.

### ***9.3.10.4 Testing and Inspection Requirements***

The oxygen injection system is demonstrated operable by its use during normal operation. The system valves can be tested to ensure operability from the MCR.

### ***9.3.10.5 Instrumentation***

The oxygen supply system has monitors that indicate to the operators when re-supply is required. Two flow elements (one for each injection point) indicate the oxygen gas flow rate at all times. The gas flow regulating valves have position indication in the MCR.

### ***9.3.10.6 COL Information***

The COL applicant shall define site storage requirements. .

### ***9.3.10.7 References***

None.

### **9.3.11 Zinc Injection System**

The ESBWR Standard Plant design includes the capability to connect a Zinc Injection System (ZIS), but the system itself is not part of the ESBWR Standard Plant design.

#### ***9.3.11.1 Design Bases***

##### **Safety (10 CFR 50.2) Design Bases**

The ZIS does not perform any safety-related function. Therefore, the ZIS has no safety design basis.

##### **Power Generation Design Bases**

Provisions are made to permit installation of a system for adding a zinc solution to the feedwater. Piping connections for a bypass loop around the feedwater pumps and space for the zinc addition equipment are provided.

#### ***9.3.11.2 System Description***

See Subsection 9.3.11.6.

#### ***9.3.11.3 Safety Evaluation***

The Zinc Injection System is not safety-related, and does not affect the safety function of any other safety-related system.

#### ***9.3.11.4 Test and Inspections***

The Zinc Injection System, if necessary, can be installed at the connection points provided. Zinc injection is not performed when the plant is in cold shutdown. See Subsection 9.3.11.6.

#### ***9.3.11.5 Instrumentation and Controls***

Instrumentation can be provided to automatically stop the injection of zinc solution if feedwater flow stops. The zinc injection rate can be manually adjusted based on the zinc concentration in the reactor water.

#### ***9.3.11.6 COL Information***

The COL Applicant/Holder shall determine if a Zinc Injection System is required based on the site-specific water quality requirements.

COL Applicant/Holder will furnish necessary information on System Description, Test and Inspection when vendor information becomes available.

#### ***9.3.11.7 References***

None.

### 9.3.12 Auxiliary Boiler System

#### 9.3.12.1 Design Basis

##### Safety (10 CFR 50.2) Design Bases

The Auxiliary Boiler System (ABS) is a nonsafety-related system and has no safety design basis.

##### Power Generation Design Bases

The ABS supplies steam required by the unit for plant use during startup and shutdown and at any other time when the main steam or gland steam evaporator steam (refer to Subsection 10.4.3) are unavailable.

#### 9.3.12.2 System Description

##### General Description

During plant startup and shutdown and also at normal operation (if required), the ABS provides the necessary non-radioactive steam at the required pressure to the following equipment:

- Steam Jet Air Ejectors to maintain the motive power required to perform a continuous evacuation of the non-condensable gasses from the Main Condenser and to the Off-gas System
- Turbine Gland Sealing System to allow the Main Condenser to reach vacuum
- Main Condenser to deaerate the condensate in the hotwell during the startup of the plant.
- Hot water generator of the Hot Water System (if required)
- Preoperational testing of off-gas system equipment
- Evaporation of liquid nitrogen for inerting of the Containment

##### Component Description

The ABS components are located in the Auxiliary Boiler Building. The following components are part of the ABS:

- One (1) 100% packaged firetube Auxiliary Boiler composed of two (2) 50% fuel oil boilers;
- Two (2) complete firing systems including fuel-oil burners and fans;
- Two (2) 100% fuel-oil pumps;
- One (1) 100% Deaerator with integral storage tank;
- Three (3) 50% Auxiliary Boiler Feedwater Pumps;
- One (1) 100% Auxiliary Boiler Drain Tank;
- Two (2) 50% Continuous Blowdown Flash Tank;
- Two (2) 50% Steam Separators; and
- Instrumentation and controls.

The Auxiliary Boilers burn fuel oil to boil demineralized water to produce steam during plant startup, shutdown and off-line operation when main steam or gland steam evaporator steam is unavailable. Auxiliary Boilers Fuel Oil Pumps are provided in the Auxiliary Boiler System to transfer fuel oil from the fuel oil tank to the Auxiliary Boiler. The fuel oil piping shall be connected to the fuel oil tank by means of a fuel tank nozzle situated above the minimum level required for the Diesel Generator System.

The Makeup Water System provides makeup feedwater to the ABS. A pressure control valve maintains the steam supply pressure while a level control valve regulates Auxiliary Boiler feedwater flow to the boiler. The steam generated by the Auxiliary Boiler passes through Steam Separators and is then distributed via two supply lines to the auxiliary steam header and to the gland steam supply line in the Turbine Gland Seal System. Each supply line contains an air-operated block valve and check valve for isolation.

The condensate from the Continuous Blowdown Flash Tanks is directed to the Auxiliary Boiler Drain Tank. The non-radioactive area of the Equipment and Floor Drain System collects drains from the ABS.

The ABS has sample connections to monitor pH, dissolved oxygen and conductivity.

#### ***9.3.12.3 Safety Evaluation***

The ABS does not perform or ensure any safety-related function and therefore requires no safety evaluation. High-energy pipe rupture analysis is not required for the ABS because none of the lines pass through areas where safety-related equipment is located.

#### ***9.3.12.4 Testing and Inspection Requirements***

Testing of the ABS is performed prior to initial plant operation. Components of the system are monitored during operation to verify satisfactory performance.

#### ***9.3.12.5 Instrumentation***

A boiler control system is provided with the Auxiliary Boiler package for automatic control of the Auxiliary Boiler. Features of the control system include automatic shutdown of the Auxiliary Boiler and the Auxiliary Boiler feedwater pumps on an abnormal condition.

The ABS is provided with the necessary controls and indicators for local or remote monitoring and control of the operation of the system.

#### ***9.3.12.6 COL Information***

None.

#### ***9.3.12.7 References***

None.

**Table 9.3-1**  
**Process Sampling System Sampling Measurements**

<b>Sampled System</b>	<b>Typical Process Measurements</b>
Reactor Water Cleanup/Shutdown Cooling System	Conductivity, Dissolved Oxygen**, pH**, Chloride**, Silica, Corrosion Product Metals, Gross Activity**, Corrosion Product Activity, Fission Product Activity, Sulfate, I-131**, Total Anions, Organic Impurities
Control Rod Drive System	Conductivity, Dissolved Oxygen
Fuel and Auxiliary Pools Cooling System	Conductivity**, pH**, Chloride**, Silica, Corrosion Product Metals, Gross Activity, Corrosion Product Activity, Fission Product Activity, I-131, Turbidity, Sulfate, Total Anions, Organic Impurities
Main Steam System*	Gaseous Fission Products (Xe, Kr)
Condensate and Feedwater System	Conductivity, Dissolved Oxygen, Corrosion Product Metals, Iron, Copper, Chloride, Isotopics, Sulfate
Moisture Separator Reheater System	Iron, Copper
Heater Drain and Vent System	Iron, Copper
Generator Cooling System	Sodium, Chloride, Silica, Iron, Copper
Condensate Purification System	Chloride, Conductivity, Dissolved Oxygen, Isotopics, Corrosion Product Metals, Iron, Copper, Sulfate
Main Condenser and Auxiliaries	Conductivity or Sodium
Auxiliary Boiler System	pH, Cation Conductivity, Silica, Sodium, Ammonia, Morpholine
Liquid Radwaste System Effluent Sample Tank	Gross Radioactivity and identification and Concentration of Principal Radionuclide and Alpha Emitters, State and Federal Environmental Discharge Requirement such as pH, Suspended Solids, Oil and Grease, Iron, Copper, Sodium Nitrate

\* The main steam sampling is listed as part of the Turbine Building Sample Station, but due to the high pressure, temperature, and radiation involved with a main steam sample this sample may be routed to a separate sample panel.

\*\* These sample parameters are required to be monitored as part of the post accident sampling program per SRP 9.3.2.

**Table 9.3-2**  
**Major Equipment for EFDS**

<b>Clean Drain Subsystem</b>		
Reactor Building clean area floor drains (including nonsafety-related electrical areas)		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Control Building clean area floor drains		
Tanks or sumps	Quantity:	2
	Usable Capacity:	Volume – $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	4 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Turbine Building clean area floor drains		
Tanks or sumps	Quantity:	3
	Usable Capacity:	Volume – $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ ) each
Pumps	Quantity:	6 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	

**Table 9.3-2**  
**Major Equipment for EFDS (continued)**

<b>Low Conductivity Waste (LCW) Drain Subsystem</b>		
Drywell equipment drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Reactor Building equipment drains		
Tanks or sumps	Quantity:	2
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ ) each
Pumps	Quantity:	4 - 100% capacity (2 per sump)
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Fuel Building equipment drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity (2 per sump)
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	

**Table 9.3-2**  
**Major Equipment for EFDS (continued)**

Turbine Building equipment drains		
Tanks or sumps	Quantity:	3
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal) each
Pumps	Quantity:	6 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	
Radwaste Building equipment drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal)
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 18 m (60 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	
<b>High Conductivity Waste (HCW) Drain Subsystem</b>		
Drywell floor drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal)
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	

**Table 9.3-2**  
**Major Equipment for EFDS (continued)**

Reactor Building floor drains		
Tanks or sumps	Quantity:	2
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal) each
Pumps	Quantity:	4 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	
Fuel Building floor drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal) each
Pumps	Quantity:	2 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	
Turbine Building floor drains		
Tanks or sumps	Quantity:	3
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859$ gal) each
Pumps	Quantity:	6 - 100% capacity (2 per sump)
	Capacity:	Flow – $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head – 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30$ hp)	

**Table 9.3-2**  
**Major Equipment for EFDS (continued)**

Radwaste Building floor drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 18 m (60 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
<b>Detergent Drain Subsystem</b>		
Turbine Building detergent drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Radwaste Building detergent drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 18 m (60 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	

**Table 9.3-2**  
**Major Equipment for EFDS (continued)**

<b>Chemical Waste Drain Subsystem</b>		
Turbine Building Chemical waste drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
Radwaste Building Chemical drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 3.25 \text{ m}^3$ ( $\geq 859 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 18 m (60 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	
<b>Reactor Component Cooling Water Waste Drain Subsystem</b>		
Reactor Component Cooling Water System waste drains		
Tanks or sumps	Quantity:	1
	Usable Capacity:	Volume - $\geq 13 \text{ m}^3$ ( $\geq 3434 \text{ gal}$ )
Pumps	Quantity:	2 - 100% capacity
	Capacity:	Flow - $30.66 \text{ m}^3/\text{hr}$ (135 gpm) per pump
		Head - 61 m (200 ft) at discharge
Type:	Sump pump, $\leq 22.37 \text{ kW}$ ( $\leq 30 \text{ hp}$ )	

**Table 9.3-3****Safety-Related Portions of the SLC System**

- An accumulator tank for each SLC system train at ambient temperature containing a volume of a 12.5 weight % solution of sodium pentaborate with the boron content enriched to 94% of the B10 isotope, sufficient to ensure injection of at least 7.8 m<sup>3</sup> (2061 gal) of solution.
- A piping system for nitrogen charging and sparging of the solution with valves, controls, and logic necessary to maintain 14.8 m<sup>3</sup> (523 ft<sup>3</sup>) of nitrogen cover gas at a minimum absolute pressure of 14.82 MPa (2150 psia) in the accumulator for each SLC system train.
- A pressure relief line and valve to prevent nitrogen pressure from exceeding the design pressure for each accumulator.
- A vent system to permit depressurization of each accumulator for access or after completion of solution injection.
- An injection line with valves, controls, and logic to ensure manual or automatic injection, and post injection closure of the line for each SLC system train. The injection line is a 3-in. stainless steel line designed for 17.24 MPa (2500 psia) internal pressure with corresponding ratings for the valves.
- Redundant level instrumentation for each accumulator to ensure adequate solution inventory and to initiate closure of the injection line on completion of solution injection.
- A poison solution line used for initial charging and any necessary periodic makeup to each accumulator.

**Table 9.3-4**  
**Safety-Related Support Systems for the SLC System**

Power from the Class 1E

- 120 VAC power systems
- Initiation signals for automatic initiation
- A sparger system consisting of a two reactor vessel penetrations to serve two trains, two internal injection lines, two distribution manifolds, and in each quadrant four injection nozzles located in the external core bypass volume to ensure uniform solution injection, each nozzle has two jet-openings angled outwardly from each other.
- Instrumentation and alarms to ensure conformance to equipment environmental qualification constraints

**Table 9.3-5**  
**SLC ATWS Mitigation Function Parameters**

<b>Parameter</b>	<b>Value</b>
Initial reactor absolute pressure	8.61 MPa (1250 psia)
Approximate initial injection flow rate per train	18.4 l/s (292 gpm)
Approximate average velocity for the first 5.4 m <sup>3</sup> of the injection	30.5 m/s (100 ft/s)
Approximate average velocity for the second 5.4 m <sup>3</sup> of the injection	18.4 m/s (60 ft/s)
Total solution injection (per each train) at the initial reactor absolute pressure	≥5.4 m <sup>3</sup> (1427 gal)
Equivalent natural boron concentration for the total solution injection volume, based on a hot shutdown liquid inventory *	≥ 1600 ppm
With the water level at the main steam line, the total injection solution inventory (per each train), and equivalent natural boron concentration at cold shutdown conditions	≥ 7.8 m <sup>3</sup> (2061 gal) > 1100 ppm **

\* Liquid inventory in RPV calculated based on reactor coolant level extending up to main steam line nozzle.

\*\* This concentration ensures maintaining shutdown reactivity even after initiation and operation of the reactor shutdown cooling system.

**Table 9.3-6  
Instrument Air System Requirements**

Capacity	
Flow rate:	22.7 m <sup>3</sup> /min (800 scfm), minimum
Pressure	
Air compressor intake, absolute:	0.1 MPa (14.7 psi)
Air compressor discharge, gage:	0.86 MPa (125 psi)
System operating, gage:	0.62 to 0.86 MPa (90 to 125 psi)
Maximum system operating, gage:	0.86 MPa (125 psi)
Temperature	
Air compressor intake:	-23 to 38°C (-10 to 100°F)
After-cooler outlet:	46°C (115°F) maximum
Pressure Dewpoint	
Air dryer outlet:	-40°C (-40°F) maximum
Filtration	
Particle size:	3 microns maximum

**Table 9.3-7**  
**Service Air System Requirements**

Capacity	
Normal flow rate:	28 m <sup>3</sup> /min (1000 scfm)
Maximum flow rate:	56 m <sup>3</sup> /min (2000 scfm)
Pressure	
Air compressor intake, absolute:	0.1 MPa (14.7 psi)
Air compressor discharge, gage:	0.7 MPa (100 psi)
System operating, gage:	0.65 to 0.72 MPa (94 to 105 psi)
Maximum system operating, gage:	0.86 MPa (125 psi)
Temperature	
Air compressor intake:	-23 to 38°C (-10 to 100°F)
After-cooler outlet:	38°C (100°F) maximum
Filtration	
Particle size:	10 micron maximum
Breathing air particle size:	0.5 micron maximum

Table 9.3-8

**High Pressure Nitrogen Supply Requirements**

Nitrogen Demand	
MSIVs and containment instrumentation (intermittent):	9627 l/min (340 scfm)
SRV and IC Isolation Valve accumulator recharging (intermittent):	12743 l/min (450 scfm)
SRV and IC Isolation Valve leakage:	0.472 l/min (1/60 scfm) each
Bottles (intermittent):	10307 l/min (364 scfm)
Normal nitrogen supply from CIS: (Intermittent):	12743 l/min (450 scfm) maximum
Pressure	
MSIV and other containment instruments supply:	0.86 MPa <sub>g</sub> (125 psig)
SRV and IC Isolation Valve accumulators supply (maximum):	1.38 MPa <sub>g</sub> (200 psig) maximum
SRV and IC Isolation Valve accumulators:	1.13 MPa <sub>g</sub> (164 psig) minimum
Bottle initial fill:	24.92 MPa abs (3615 psia)

LEGEND:  
 A, B, C, D = QUALITY GROUP A, B, C, D  
 I = SEISMIC CATEGORY I  
 NS = NON-SEISMIC

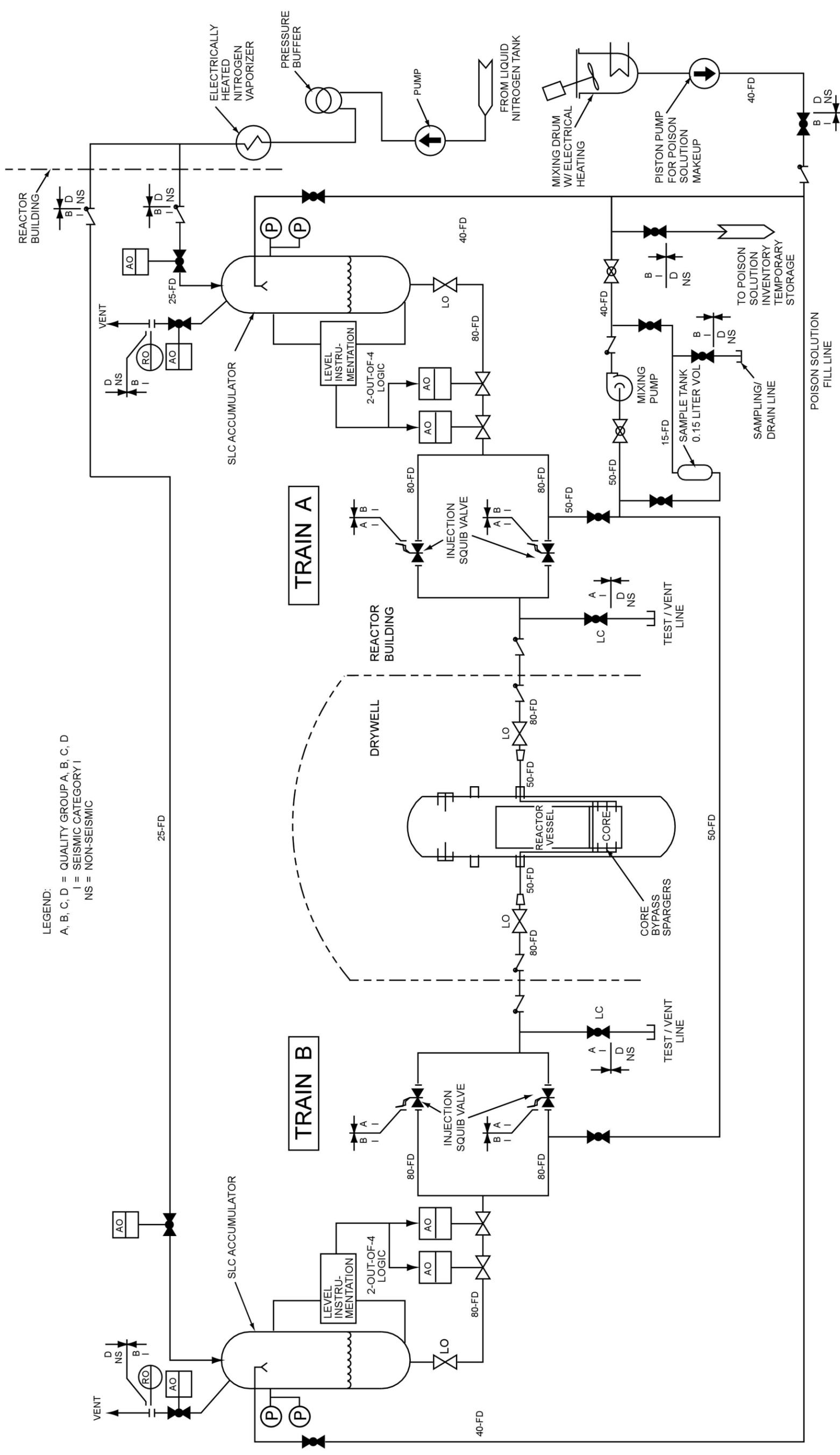
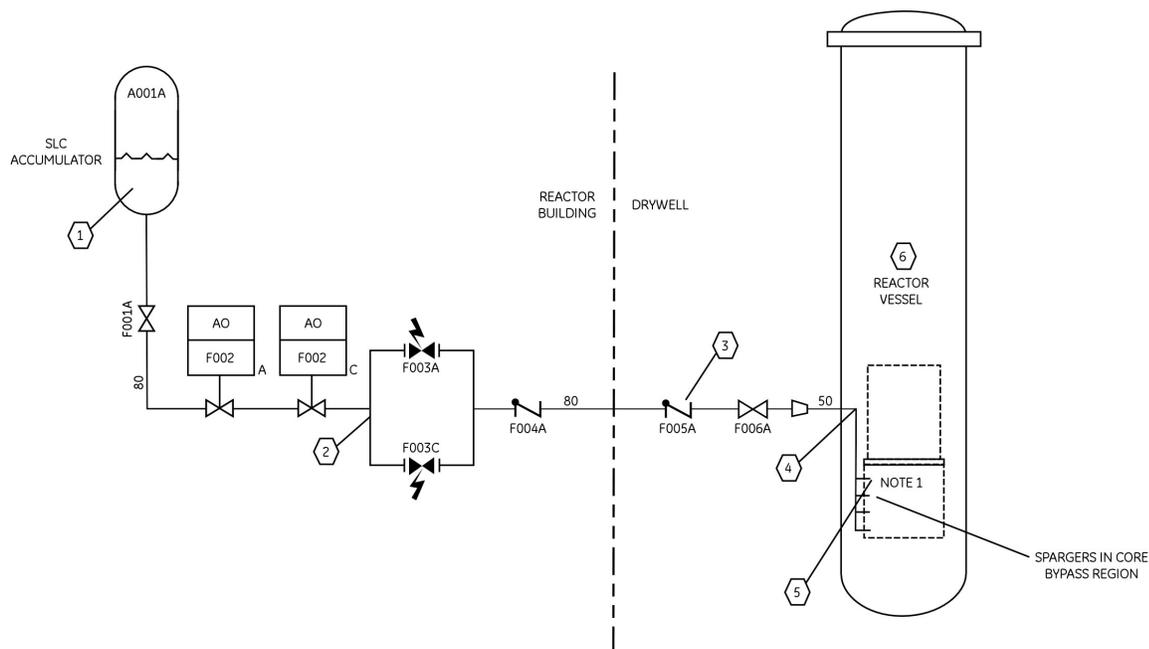


Figure 9.3-1. Standby Liquid Control System Simplified Diagram

**TRAIN A**  
(Typical For Train B)



**SYSTEM STANDBY MODE**

LOCATION		1	2	3	4	5	6
PRESSURE	MPa Gauge (PSIG)	15.17 (2200)	15.20 (2204)	7.10 (1030)	7.10 (1030)	7.10 (1030)	7.10 (1030)
TEMPERATURE	°C (°F)	18 (65)	18 (65)	18 (65)	278 (533)	278 (533)	278 (533)

**INJECTION MODE (RPV-HOT STANDBY)**

LOCATION		1	2	3	4	5 NOTE 4	6
PRESSURE NOTE 3	MPa Gauge (PSIG)	12.1 (1755)	12.0 (1746)	11.9 (1719)	11.5 (1670)	9.6 (1390)	8.63 (1251)
TEMPERATURE	°C (°F)	-2.3 (+27.9)	18 (65)	18 (65)	18 (65)	18 (65)	298 (571)
AVERAGE VELOCITY NOTE 2	M/SEC (FT/SEC)	-	8.9 (29.3)	8.9 (29.3)	21.5 (70.7)	34.2 (112.5)	-

**NOTES:**

1. EACH SLC INJECTION LINE CONNECTS TO A HEADER INSIDE THE REACTOR VESSEL THAT DIRECTS FLOW TO TWO DOWNCOMERS. EACH DOWNCOMER CONNECTS TO FOUR NOZZLES THAT DIRECT FLOW THROUGH THE SHROUD INTO THE CORE BYPASS REGION. THE NOZZLES ARE LOCATED AT FOUR DIFFERENT ELEVATIONS FOR A TOTAL OF EIGHT NOZZLES PER EACH SLC Train.
2. AVERAGE VELOCITY DURING INJECTION OF THE FIRST HALF OF REQUIRED SOLUTION VOLUME.
3. PRESSURE CONDITIONS CORRESPOND TO TIME AT WHICH HALF THE SOLUTION VOLUME HAS BEEN INJECTED.
4. VELOCITY THROUGH A SINGLE NOZZLE HOLE. EACH NOZZLE HAS TWO HOLES.
5. THE INJECTION LINE FROM THE ACCUMULATOR TO THE REACTOR HAS A DESIGN PRESSURE OF 17.24 MPa G (2500 PSIG) AND DESIGN TEMPERATURE OF 60°C (140°F).

**Figure 9.3-1a. Standby Liquid Control System Simplified Process Flow Diagram**

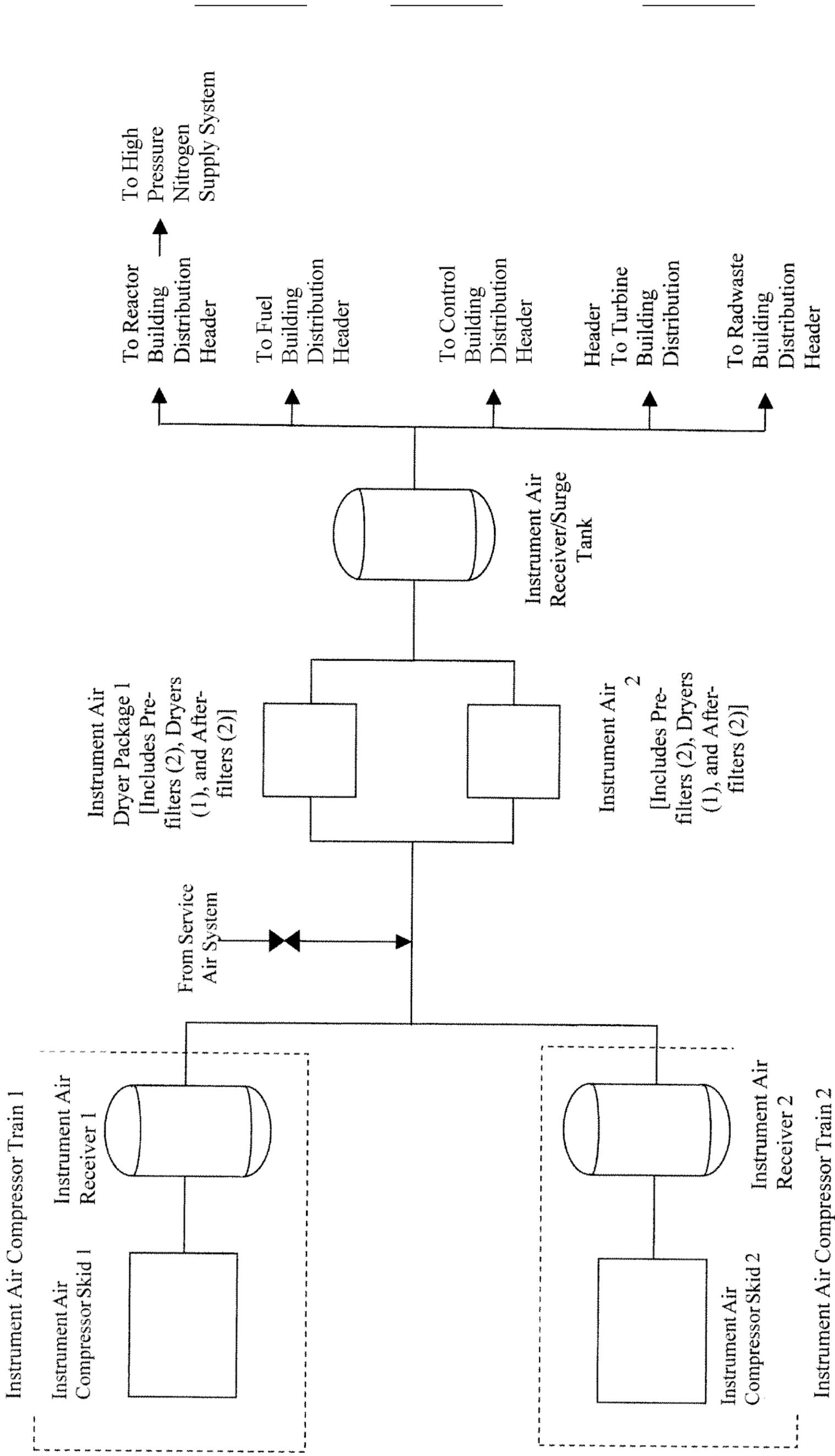


Figure 9.3-2. Instrument Air System Simplified Diagram

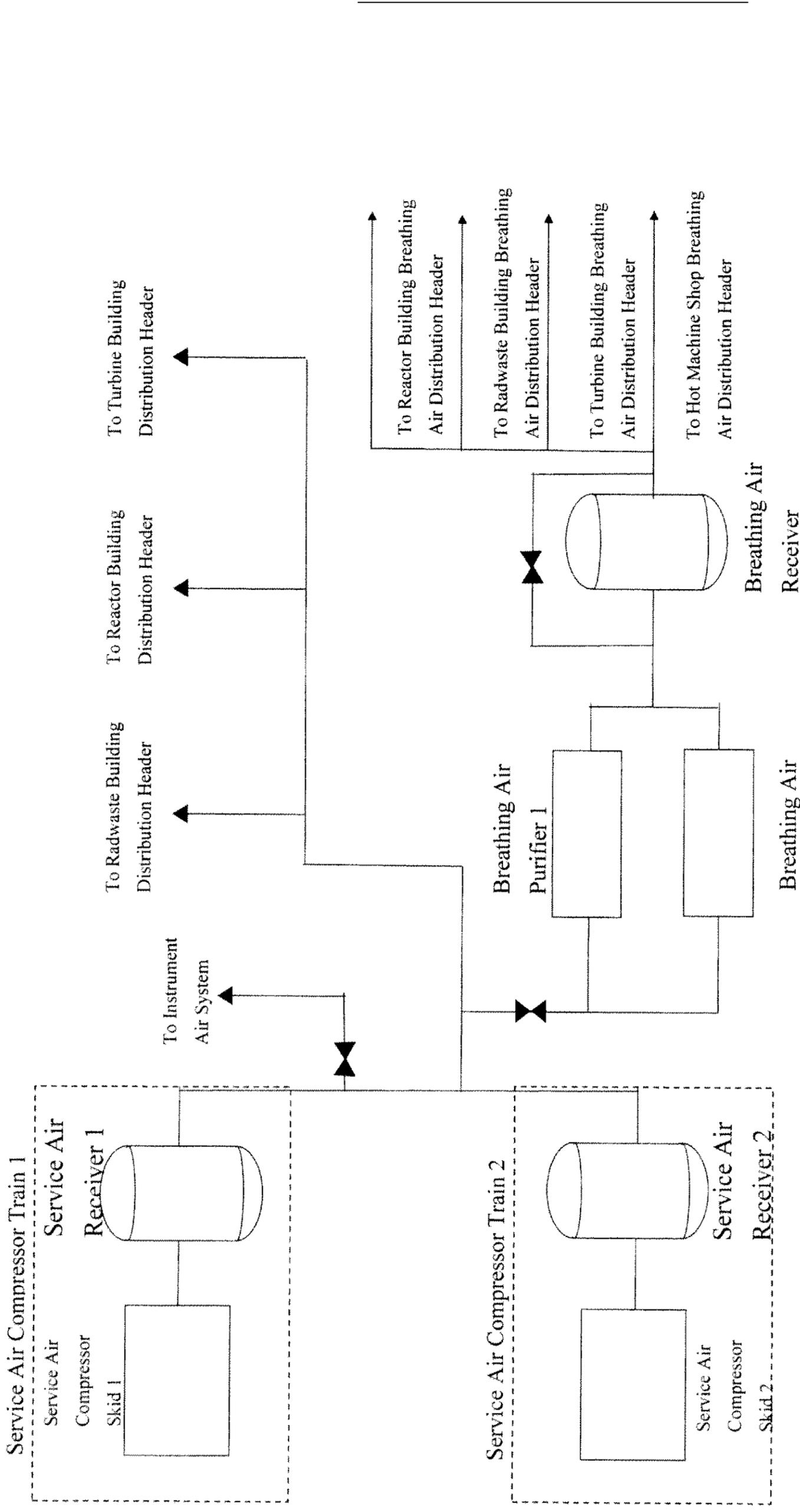


Figure 9.3-3. Service Air System Simplified Diagram

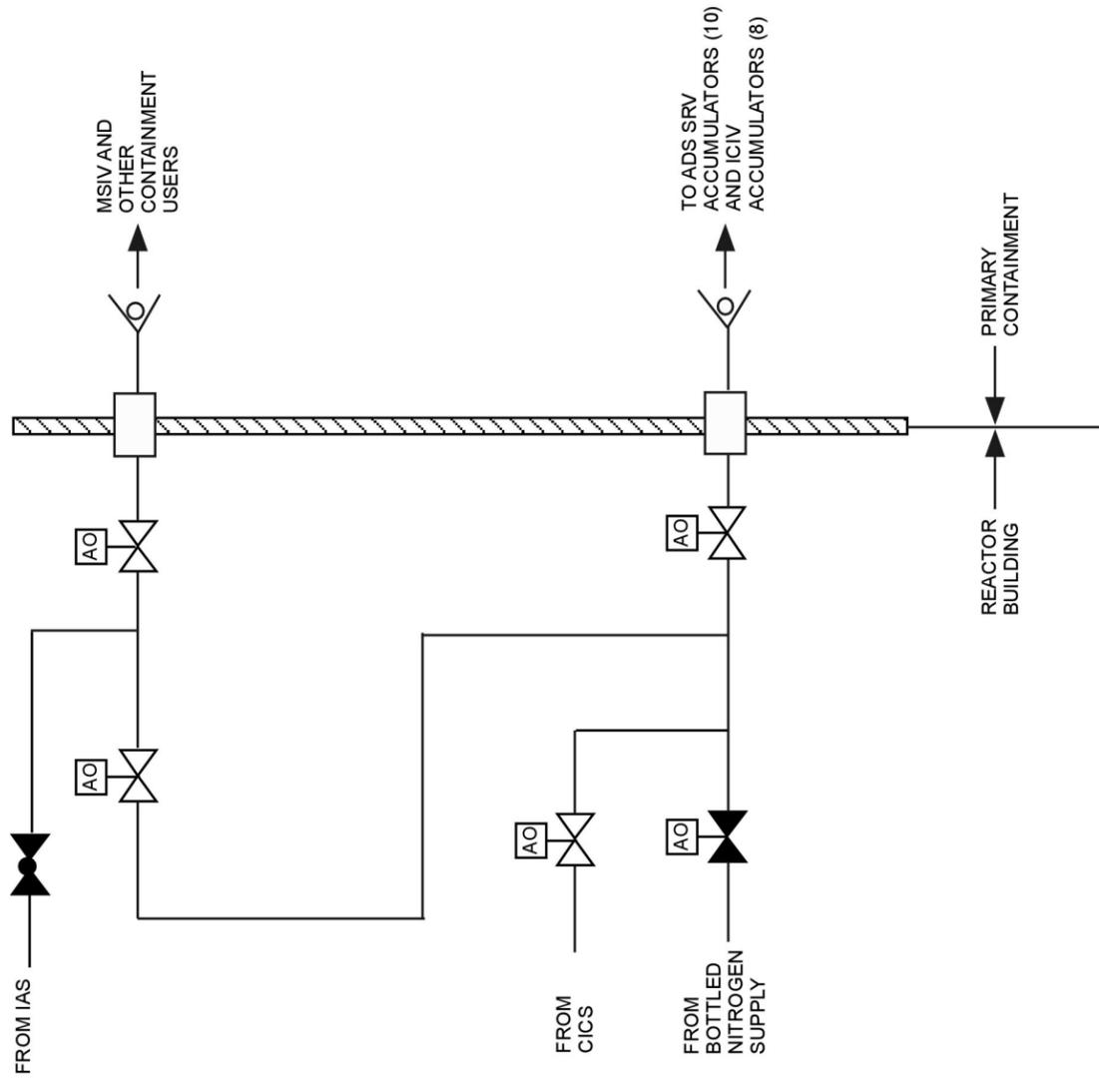


Figure 9.3-4. HPNSS Simplified Diagram

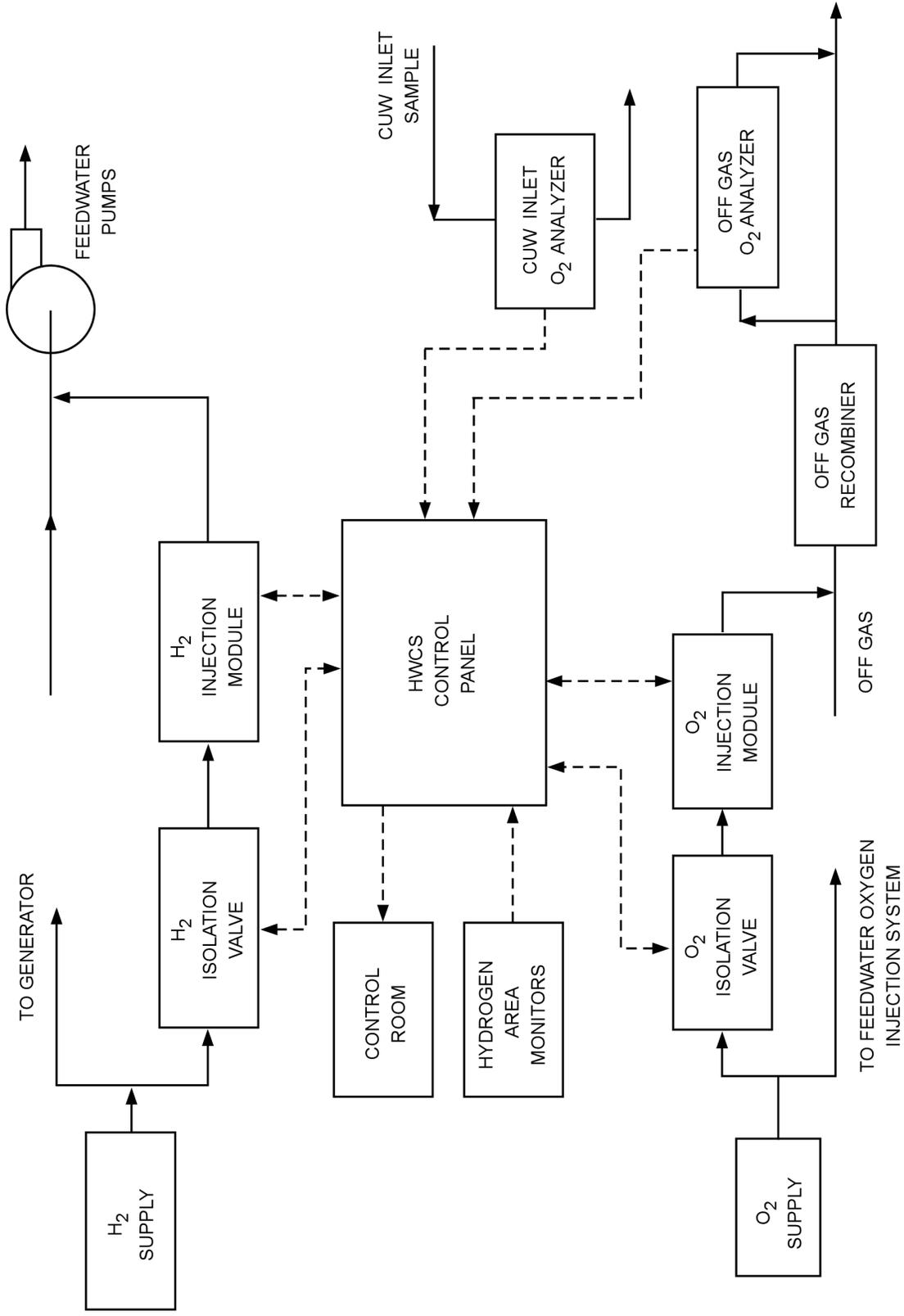


Figure 9.3-5. Hydrogen Water Chemistry System Simplified Diagram

## 9.4 HEATING, VENTILATION, AND AIR CONDITIONING

### 9.4.1 Control Building HVAC System

The Control Building Heating, Ventilating and Air Conditioning System (CBVS) serves all areas of the Control Building during normal operation. The CBVS maintains space design temperatures, quality of air and pressurization. It provides a controlled environment for personnel safety and comfort, and for the proper operation and integrity of equipment located in the control building.

The CBVS consists of two subsystems:

- The Control Room Habitability Area HVAC Subsystem (CRHAVS) serves the Main Control Room (MCR) and associated support areas. These areas comprise the Control Room Habitability Area (CRHA). The CRHA envelope can be isolated and protected as described herein during emergency modes of operation. When AC power is available, the CRHAVS provides HVAC functions for the CRHA via two nonsafety-related redundant fresh air supply fans and two redundant nonsafety-related internal floor mounted air handling units (AHU). Radiological protection is provided from a redundant set of safety-related Emergency Filter Units (EFU). The EFUs are 100% capacity redundant units designed and tested to meet the requirements of RG 1.52. The EFUs supply a 99% credited efficiency for particulates and a 99% credited efficiency for elemental iodine and organic iodide as described in Regulatory Guide 1.52.

When AC power is not available, the CRHA is cooled passively via heat transfer to the CRHA passive heat sink, and radiological protection continues to be provided from the safety-related EFUs which are powered from the safety-related 1E battery power source. Portions of the CRHAVS that are safety-related are the EFUs and their associated fans, ductwork, instrumentation and controls, the CRHA boundary envelope, and the CRHA isolation dampers and associated ductwork. All remaining CRHAVS equipment is nonsafety-related. The CRHA isolation dampers automatically close to isolate the CRHA envelope and an EFU is automatically actuated in the event of a loss of electrical power (SBO) or during a radiological event.

- The Control Building General Area HVAC Subsystem (CBGAVS) serves the area outside the CRHA. The CBGAVS is nonsafety-related. The subsystem is made up of two subsets, Set A and Set B, each of which contain two redundant 100% capacity AHUs and associated fans and ductwork. Set A serves its respective HVAC equipment room, the A Non-1E Distributed Control and Instrumentation System (DCIS) Room, and the Division I and IV DCIS Rooms. Set B serves its respective HVAC equipment room, the B Non-1E DCIS Room, the Division II and III DCIS Rooms, and the corridor area around the CRHA.

The following General Design Criteria have been evaluated against the CRHAVS:

- The CRHAVS meets the acceptance criteria of GDC 2. The CRHA envelope is comprised of Seismic Category I structures and components that are protected from postulated tornados, hurricanes, tsunamis, seiches, and seismic events. The CRHAVS components are designated as Seismic Category II with the exception of the safety-

related CRHA envelope, isolation dampers, the EFUs and associated fans, dampers, ductwork, and instrumentation and controls, which are Seismic Category I. The CB structure is a Seismic Category I structure.

- The CRHAVS meets the acceptance criteria of GDC 4. The CRHAVS design complies with GDC 4 by ensuring that personnel in the MCR are protected from the effects of postulated accidents. The safety-related CRHA envelope, isolation dampers, EFUs and associated fans, dampers, ductwork, instrumentation and controls are designed to be protected from all postulated environmental and dynamic effects.
- The CRHAVS meets the acceptance criteria of GDC 5. The ESBWR CRHAVS supports a single unit, including one CRHA envelope. This meets the intent of GDC 5 with the following design considerations:
  - Each ESBWR unit at a multi-unit site has a separate control room;
  - An accident in one unit does not impair the ability to perform safety functions of the remaining units; and
  - An orderly shutdown and cool down of remaining unit(s) would not be impaired.
- The CRHAVS meets the acceptance criteria of GDC 19. The CRHA is isolated during SBO or LOCA conditions, and the safety-related EFUs provide pressurization and radiologically filtered air. Radiation detectors in the intake air supply to the CRHA provide warning and initiate actions to protect control room personnel under accident conditions. The CRHAVS design maintains a habitable control room under accident conditions by providing adequate radiation protection and breathing air. When power is available, the Air Handling Units (AHU) maintain the space temperature. Upon a loss of power, the remaining nonsafety-related heat loads are dissipated for 2 hrs using battery power, and the remaining safety-related heat loads are passively dissipated by the walls, floor, ceiling and interior walls for the remainder of the 72 hr passive duration. ESBWR is committed to maintaining the MCR at a temperature no greater than 8.3°C (15°F) above the initial MCR temperature for any loss of cooling event, including SBO or LOCA conditions. This temperature rise limit applies both during the 72 hr passive period, and the duration of any postulated accident. The radiation detection range is selected to cover normal operation, with sufficient sensitivity to initiate isolation of the MCR prior to exceeding the 10CFR50 Appendix A GDC 19 guidelines.
- The CBVS, including the CRHAVS meets the acceptance criteria of GDC 60. The CRHAVS EFUs meet the guidance of Regulatory Guide 1.52 as related to design, testing and maintenance criteria for atmosphere cleanup system and normal ventilation system air filtration and adsorption units. The Control Building does not house any portion of the nuclear steam supply process or other equipment that can act as a source of radioactive material; and therefore has no postulated sources of radioactive materials in either particulate or gaseous form. Therefore, the CB exhaust systems require no filtration or adsorption.

### 9.4.1.1 Design Bases

#### Safety (10 CFR 50.2) Design Bases

Only the CRHAVS performs safety-related design basis functions in support of CRHA habitability. The habitability requirements for the CRHA are discussed and described in Section 6.4.

The CRHAVS provides the following safety-related design basis functions:

- Monitors the CRHA air supply for smoke and radioactive particulate and/or iodine concentrations;
- Isolates the normal CRHA air supply and restroom exhaust, starts an EFU fan, and aligns the air supply through an EFU, upon a high radiation detection signal in the CRHA normal air supply, or upon an extended loss of AC power to support operation of a CRHA normal air supply fan; and
- Isolates the normal CRHA air supply and restroom exhaust upon detection of smoke in the CRHA normal air supply.

The portions of the CRHAVS which penetrate the CRHA envelope are nonsafety-related and designed as Seismic Category I to provide isolation of the CRHA envelope from the outside and surrounding areas in the event of a design basis accident (DBA). The EFU portion of the subsystem is safety-related and designed and supported as Seismic Category I including the air intakes, ductwork, dampers, fans, instrumentation and controls. The remaining CRHAVS functions are nonsafety-related. The penetrations contain safety-related isolation dampers or valves that fail closed upon a loss of control signal, power, or instrument air. EFUs are automatically actuated upon radiological isolation of the CRHA envelope or an extended loss of AC power.

CBVS equipment and ductwork whose failure could affect the operability of safety-related systems or components are designed as Seismic Category II. The remaining portion of the system is nonsafety-related and nonseismic..

The following CRHA components are safety-related and Seismic Category I:

- CRHA Boundary envelope including structures, doors, and components;
- EFUs and related components;
- Ductwork from the CRHA boundary envelope up to the CRHA isolation dampers and the CRHA isolation dampers; and

The CBVS outside air intake structures are nonsafety-related and do not require tornado dampers and tornado missile shields.

The CBVS provides a safety-related means to passively maintain habitable conditions in the CRHA following a design basis accident (radiological event concurrent with SBO).

A redundant set of safety-related Emergency Filter Units (EFU) including HEPA and carbon filters serves the CRHA envelope.

Radiation detected in the CRHA outside air inlet causes the following actions:

- A normally closed EFU outside air inlet to open;
- The normal outside air inlet and restroom exhaust dampers to close; and
- An EFU fan to automatically start.

The CRHA is isolated during SBO conditions and a safety-related EFU provides pressurization and breathing quality air. An EFU is powered from the safety-related battery supply for a 72 hour duration.

The CBVS provides the capability to maintain the integrity of the CRHA with redundant safety-related isolation dampers in all ductwork penetrating the CRHA envelope. The active safety-related components (CRHA isolation dampers and EFUs), that ensure habitability in the CRHA envelope, are redundant. Therefore a single active failure cannot result in a loss of the system design function.

During normal modes of operation and emergency modes with electrical power available, the CRHA is maintained within the temperature and relative humidity ranges noted in Table 9.4-1 by the nonsafety-related CRHAVS AHU. During emergency operation, with an SBO, a nonsafety-related CRHA air handling unit (AHU), powered from the nonsafety-related Uninterruptible AC Power Supply System, maintains the CRHA within the normal operating temperature range for two hours. This allows the continued operation of certain high heat producing nonsafety-related MCR DCIS electric loads. These nonsafety-related MCR DCIS electrical loads automatically de-energize should the redundant CRHA AHUs not be available. In the event the SBO duration extends beyond two hours, the reduced CRHA heat load is passively cooled by the CRHA heat sink, which consists of the internal and external concrete walls, floor, and ceiling, such that the CRHA temperature rise is no greater than 8.3°C (15°F).

### **Power Generation Design Bases**

The CBVS:

- Provides a controlled environment for personnel comfort and safety. Sufficient outside air is provided to meet the ventilation requirements for acceptable indoor air quality per ASHRAE 62-2001, Table 2 (Table 1.9-22). See Table 9.4-1 for area design temperature and humidity design parameters.
- Provides a controlled environment for the proper operation and integrity of equipment in the Control Building during normal, startup and shutdown operations;
- Provides redundant active components to increase reliability, availability and maintainability of the ventilation system;
- Provides shutoff dampers on the inlet and outlet of fans and AHUs if necessary to allow for maintenance;
- Provides shutoff valves at the inlet and outlet of cooling and heating coils if necessary to allow for maintenance;
- Provides access for AHU fans, filter sections and duct mounted dampers to allow for maintenance;

- Provides the capability for manual control of system fans to facilitate maintenance and testing;
- Minimizes exposure to personnel during inspection and maintenance by locating equipment and instrumentation as far as practical from potential sources of high radiation;
- Maintains higher than atmospheric (positive) pressure to minimize the infiltration of outside air. Construction materials and processes ensure the Control Building structure maintains low leakage or leak tight conditions above and below grade. The CRHA envelope penetrations are sealed and access doors are designed with self-closing devices that close and latch the doors following use. There are double door airlocks in the CRHA envelope for access and egress during emergencies when the CRHA is isolated and an EFU is operating.
- Reduces the potential spread of airborne contamination by maintaining airflow from areas of lower potential for contamination to areas of greater potential for contamination. The CRHA is maintained at a higher pressure than surrounding areas except during the isolation and smoke exhaust modes.
- Detects and limits the introduction of airborne hazardous materials (radioactivity or smoke) into the CRHA;
- Provides the capability to exhaust smoke, heat and gaseous combustion products from inside the Control Building to the outside atmosphere in the event of a fire. Construction materials and processes ensure that materials of construction are non-combustible and heat and flame resistant wherever possible. Materials that produce toxic or noxious vapors when subjected to a fire are avoided.
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.
- Is designed such that failure of nonsafety-related equipment does not compromise or otherwise damage safety-related equipment; and
- Is provided with bullet-resistant exterior HVAC openings that penetrate the MCR vital envelope in accordance with 10 CFR 73.

### 9.4.1.2 System Description

#### Summary Description

The CBVS subsystems, the CRHAVS and the CBGAVS, are recirculating ventilation systems that provide filtered, conditioned air to serve all areas of the Control Building. The EFUs provide breathing air and pressurization to the CRHA when the CRHA envelope is radiologically isolated. The CBVS maintains space design temperatures and air quality. Outside air is normally supplied to augment the return air to maintain the Control Building under a slightly positive pressure. The CBGAVS return/exhaust fans normally direct most of the system airflow back to the system return flow with a portion of the flow exhausted to the atmosphere. The CBVS provides a controlled environment for personnel safety and comfort, and for the proper operation and integrity of equipment located in the Control Building.

CBVS equipment, including fans, AHUs, EFUs, and the CRHA are located within the Control Building Seismic Category I structural areas.

#### Detailed System Description

Figures 9.4-1, 9.4-2, 9.4-3 and 9.4-4 show the simplified system diagrams for the CRHAVS and the CBGAVS Set A and B, respectively. Table 9.4-2 lists the major equipment that comprises the CBVS. The layout of the CRHA envelope is shown in Figure 1.2-3.

The CRHA envelope includes the following areas:

- Main Control Room (Room 3275)
- Shift Supervisor Office (Room 3272)
- Shift Supervisor Conference Room (Room 3273)
- Operator's Area (Room 3270)
- Shift Technical Advisor Office (Room 3271)
- Main Control Room Storage Room (Room 3204)
- Electrical Panel Board Room (Room 3205)
- Restroom A (Room 3201)
- Restroom B (Room 3202)

These areas constitute the operation control area, which can be isolated and remain habitable for 72 hours, if required, by the existence of a LOCA, Station Blackout (SBO) or a high radiation condition with or without AC power. Also, potential sources of danger such as steam lines, pressurized piping, pressure vessels, CO<sub>2</sub> fire fighting containers, etc. are located outside of the CRHA.

The CRHAVS consists of two trains. Each train consists of:

- One 100% capacity AHU;
- One 100% capacity outside air supply fan;
- One 100% capacity safety-related EFU;

- One 100% capacity safety-related EFU fan; and
- Redundant set of safety-related CRHA isolation dampers.

The CRHAVS is configured as a recirculation system that contains the entire supply and return AHU air flow inside the CRHA, and incorporates a common supply duct for introducing outside air to the CRHA. The normal and emergency (EFU) outside air intake flows are adjusted as required to maintain a 31 Pa (1/8" wg) minimum positive pressure in the CRHA.

The intake design and location are in accordance with Regulatory Guide 1.194. Intake design, location and control also include considerations that minimize the introduction of radiological material, toxic gases, hazardous chemicals, smoke, dust and other foreign material. Ductwork, housings, access openings, etc. are constructed in such a manner as to minimize inleakage of potentially contaminated air into the CRHAVS air stream.

During normal operation, air is conditioned and distributed by an AHU and particulates are removed from the air by medium efficiency filters. Heat is transferred between the air and the heating and cooling coils inside the AHU. Moisture is added to the air stream, if required, to maintain minimum humidity levels in the CRHA by the automatically controlled humidifier. The heating and cooling processes inherently remove moisture from the air stream and maintain the humidity below the maximum specified level. The supply AHU distributes conditioned air beneath the CRHA raised floor to the CRHA rooms via registers in the raised floor. The AHU intake is ducted to a location above the suspended ceiling and return air is returned to the AHU via registers in the suspended ceiling.

Each EFU consists of a medium efficiency filter (40% minimum), a high efficiency particulate air (HEPA) filter (99.97% DOP minimum), a carbon adsorption filter (99% credited efficiency), and a post-filter downstream of the carbon filter (95% DOP minimum). The EFUs operate only during a radiological emergency and are able to function while powered from an offsite AC source, an onsite AC source, or an onsite safety-related DC source. The EFUs are monitored by instrumentation that detects a loss of airflow and detects radiation downstream of the EFU filters. Upon such detection, the operating EFU is isolated and the standby EFU is automatically placed in service.

Exhaust air from the restroom is ducted to the exhaust fan and exhausted to the outside atmosphere.

The CRHA AHUs provide cooling to the CRHA whenever offsite or onsite AC power is available. Furthermore, an AHU fan and a chilled water recirculation pump are battery powered during the first two hours of an SBO from the nonsafety-related battery supply. Cooling of the CRHA for the two hour duration during which the nonsafety-related heat loads in the CRHA are powered from the nonsafety-related battery supply is achieved via circulation of chilled water from the Chilled Water Storage Tank through an AHU. With AC power available, an AHU can operate indefinitely during a CRHA isolation event. If the AHUs are not available during the SBO, the nonsafety-related heat loads are automatically de-energized. The nonsafety-related Uninterruptible AC Power Supply System provides power for the CRHA AHUs.

An EFU provides sufficient breathing quality air to maintain positive pressure in the CRHA when the CRHA envelope is radiologically isolated. An EFU is automatically actuated when the

CRHA envelope is radiologically isolated. Controls to manually isolate the CRHA envelope and to manually actuate the EFUs are also provided.

The CBGAVS consists of:

- Two sets of two 100% capacity supply AHUs;
- Two sets of two 100% capacity return/exhaust fans; and
- Supplemental electric duct heaters.

The CBGAVS serves non-divisional equipment rooms, corridors and other miscellaneous rooms in the Control Building general areas. Set A serves Division I and IV areas. Set B serves Division II and III areas. Each set is configured as a recirculation system that incorporates a common supply and return duct system for the distribution of conditioned air. During normal operation air travels through the AHU stages. Particulates are removed from the air by low and high efficiency filters. Heat is transferred between the air and the heating and cooling coils. The outside air intake and exhaust are adjusted to maintain a slightly positive pressure in the Control Building general areas.

### **System Operation**

The CBVS operates during all modes of normal power plant operation, including startup and shutdown. The CBVS is not required to operate during an SBO except the EFUs and the CRHAVS isolation dampers. The CRHA isolation dampers fail closed on a loss of power or instrument air. During an SBO or an event that causes isolation of the CRHA envelope, the CRHA isolation dampers automatically close and an EFU is automatically actuated. Upon an isolation of the CRHA envelope an EFU operates indefinitely from an AC power source, or up to 72 hours from the safety-related battery supply. Also, an AHU continues operation to control the CRHA environment upon the initiation of a CRHA isolation. An AHU operates for the first two hours of an SBO; otherwise certain nonsafety-related heat loads are automatically de-energized. Rooms containing safety-related equipment are passively cooled by heat transfer to the CRHA heat sink for the first 72 hours of an SBO to limit the temperature rise to the maximum temperature limits listed in Table 9.4-1.

Normal Operating Mode:

- Each subsystem of the CBVS is fully operable with one train of its redundant equipment inoperable.
- The CRHAVS and CBGAVS operate with one AHU and one exhaust/return fan (CBGAVS only) in operation to maintain design conditions in the respective areas served. The outside air intakes and exhausts (CBGAVS only) are adjusted to maintain a slightly positive pressure in the CRHA and CBGA.
- The CRHAVS maintains normal design conditions in the CRHA. One AHU and one outside air intake fan are manually selected to start and cause the associated dampers to open to the required positions. The EFUs and EFU fans do not operate during normal operation.

- The CBGAVS maintains normal design conditions in the general areas of the Control Building. One supply AHU and its corresponding return/exhaust fan is manually selected to start and cause its return air and outside dampers to open to the required positions
- The CBGAVS includes electric unit heaters to provide heat when required to maintain minimum space temperature. Thermostats control the unit heaters to maintain the space temperatures. Electric heating coils are provided in the AHUs when required to preheat the outside air.
- The CRHAVS AHUs include electric heating coils to heat the CRHA recirculated air as necessary to maintain the normal CRHA temperature range.
- Cooling coils are provided in the AHUs and provide cooling to the CRHA and Control Building general areas as required to maintain design conditions.

Standby Operating Mode, in the event of low airflow in an AHU, supply duct, or exhaust duct:

- The operating AHU and return/exhaust fan (CBGAVS only) trips and the associated isolation dampers close.
- The standby AHU and return/exhaust fan (CBGAVS only) start and the associated dampers automatically open.

#### Fire/Smoke Operating Modes

The following is a brief description of the Fire/Smoke Operating Modes. For a more complete description of the interface between the HVAC systems and the Fire Protection System, refer to Subsection 9.5.1.

- Upon detection of smoke in the CRHAVS or the CBGAVS outside air intake, the normal outside air inlet dampers and the CBGAVS exhaust dampers close. Return air is recirculated through the CBGAVS AHU, and the CRHAVS AHU continues to operate normally. The CRHAVS restroom exhaust air dampers also close. With the exhaust flow path closed, the CRHAVS restroom exhaust fan is stopped manually.
- Upon detection of smoke in the CRHAVS or the CBGAVS supply or return air plenum or duct, the subsystem shuts down and the isolation dampers close. In the CBGAVS, smoke dampers are provided to prevent smoke and hot gases from migrating into other fire areas. Upon detection of smoke, the smoke dampers automatically close and isolate the fire area in the general areas of the Control Building.
- MCR operators manually initiate the smoke purge mode of operation of the CBGAVS. Fully opening the normal outside air inlet dampers and exhaust air dampers and closing the recirculation dampers, accomplishes the smoke purge. This provides 100% outside airflow. Any closed dampers in the fire area that are required to be open for purging smoke are reopened as part of the smoke purge mode. Area smoke detectors are provided in the CRHA and general areas of the Control Building.
- The smoke purge mode of operation of the CRHAVS is manually initiated by MCR operators. Smoke purge is accomplished by opening the smoke removal outside air intake dampers and the Smoke Removal Fan exhaust dampers, and starting the Smoke Removal Fan. The CRHAVS AHUs, EFUs, or normal outside air intake fans can be

operated or secured during smoke purge at the discretion of the MCR operators. Any closed fire dampers in the fire area that are required to be open for smoke purging mode are reopened as part of the smoke purge mode.

Radiological Event Operation:

- When AC power is available, an outside air high radiation signal automatically starts an EFU fan and opens the normally closed outside air inlet dampers to the EFU. The signal also closes the normal outside air inlet dampers, stops the normal outside air intake fan, and closes the restroom exhaust dampers. All outside air is drawn through an EFU. During this mode of operation the EFU is monitored for low airflow and for radiation downstream of the EFU. Should either of these conditions be detected, the standby EFU is automatically started and the operating EFU is isolated. Also, due to the CRHA isolation, the restroom fan is manually stopped.
- When offsite and onsite AC power is lost (SBO), the CRHA assumes the radiological protection mode of operation. Detection of an outside air high radiation condition during an SBO initiates redundant signals to lineup and operate an EFU, isolate the normal outside air intake, and isolate the restroom exhaust. Due to the SBO event alone, the CRHA envelope is automatically isolated as the normal outside air intake and restroom exhaust dampers fail closed, the restrooms exhaust fan stops, and an EFU is automatically started to supply outside air and pressurization to the CRHA. The EFU fans and dampers are powered from the safety-related battery supply. The CRHA AHUs and chilled water storage tank recirculation pumps remain in operation for two hours powered from the nonsafety-related battery supply.

#### ***9.4.1.3 Safety Evaluation***

The CBVS is nonsafety-related except for the CRHA envelope, the CRHA heat sink, the EFUs and associated dampers, ductwork, instrumentation and controls, which are safety-related. The CRHA envelope includes structures, doors, penetrations, valves, isolation dampers, the piping, tubing, and ductwork up to the valves and isolation dampers, and the EFU flow path. The redundant isolation dampers fail closed upon a loss of control signal, power, or instrument air.

#### ***9.4.1.4 Testing and Inspection Requirements***

Routine testing of components of the CBVS is conducted in accordance with routine power plant requirements for demonstrating system and component operability and integrity.

Periodic surveillance testing of safety-related CRHA isolation dampers and the EFU components and associated instruments and controls are performed per IEEE-338. Safety-related CRHA isolation dampers and the EFUs are operational during plant normal and abnormal operating modes.

The CRHA VS EFU components are periodically tested in accordance with the applicable portions of ASME AG-1 Code On Nuclear Air And Gas Treatment and/or ASME N510, Testing of Nuclear Air Treatment Systems, to meet RG 1.52 requirements. The EFUs have a credited HEPA efficiency of 99% and a credited carbon efficiency of 99%.

#### **9.4.1.5 Instrumentation Requirements**

The CBVS component operating status and system parameters are monitored, indicated and controlled in the MCR and locally when required.

CRHA Isolation Damper position indicators (open/closed) are provided and are located in the MCR.

Flow instrumentation is provided for fans and AHUs to indicate status and to trip malfunctioning units on low flow and to start standby units.

Differential pressure transmitters are used to indicate filter pressure drop and alarm high filter pressure drop.

CRHAVS differential pressure transmitters are provided to monitor CRHA pressure with respect to adjacent areas. Multiple transmitters are used with low select logic to compensate for wind effects. The CRHAVS EFUs have safety-related instrumentation and controls that detect low air flow and detect radiation downstream of the EFUs. Such detection will initiate the standby EFU.

CRHA airlock door position instrumentation is provided and an audible alarm sounds if airlock doors are open during a CRHA isolation.

Room temperature sensors are provided to:

- Monitor Room Temperature;
- Provide control signals to heating and cooling coils; and
- Provide alarms on high or low space temperatures.

Duct temperature sensors are provided to indicate outside air, AHU coil entering and leaving temperatures as required by system logic control.

Instrument sensing elements are located and mounted such that the accuracy of the measured parameter is representative and optimized.

Smoke detectors are provided as required by NFPA 90A to detect smoke in the system ductwork and Control Building areas.

The Q-DCIS provides a control and instrumentation data communication network to support the CBVS safety-related functions.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.4.1.6 COL Information**

None

#### **9.4.1.7 References**

The applicable HVAC codes and standards are shown in Table 9.4-17.

### 9.4.2 Fuel Building HVAC System (FBVS)

The Fuel Building Heating Ventilating and Air Conditioning System (FBVS) serves the following areas of the Fuel Building:

- General areas;
- Spent Fuel Pool; and
- Equipment areas.

Relative to the FBVS, this subsection addresses applicable requirements of General Design Criteria (GDC) 2, 5, 60 and 61. These GDCs are discussed in Standard Review Plan (SRP) 9.4.2 Rev.2, July 1981.

The ESBWR:

- Meets GDC 2, by compliance to Regulatory Guide (RG) 1.29, Position C.1 for safety-related portions and Position C.2 for nonsafety-related portions. The FB is a Seismic Category I structure except for the penthouse that houses HVAC equipment that is Seismic Category II. All FBVS components are designed as Seismic Category II with the exception of the safety-related isolation dampers and associated controls. The FBVS maintains its structural integrity after a Safe Shutdown Earthquake (SSE).
- Meets GDC 5, for shared systems and components important to safety for the Fuel Building isolation dampers. There are no safety-related shared (multiple reactor units) structures, systems or components in the FBVS.
- Meets GDC 60, by suitably controlling the release of gaseous radioactive effluents to the environment. The system may direct its exhaust air to the Reactor Building HVAC Purge Exhaust Filter Unit during periods of high radioactivity. The Reactor Building HVAC Purge Exhaust Filter Unit is designed, tested, and maintained in accordance with Regulatory Guide 1.140. DCD Tier 2, Section 11.5, Process Radiation Monitoring System, describes the Radiation Monitoring and control system interfaces with the FBVS.
- Meets GDC 61, by providing containment, confinement, and filtration to limit releases of airborne radioactivity as stated in Regulatory Guide 1.13, Position C.4. The system may direct its exhaust air to the Reactor Building HVAC Purge Exhaust Filter Unit during periods of high radioactivity, which provides filtration prior to discharge to the plant stack. The Reactor Building HVAC Purge Exhaust Filter Unit is designed, tested, and maintained in accordance with Regulatory Guide 1.140.

#### 9.4.2.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The FBVS is nonsafety-related except for the isolation dampers and ducting penetrating the Fuel Building boundary. The Fuel Building boundary is automatically isolated in the event of fuel handling accident or other radiological accidents. With the above exception, the FBVS performs no safety-related functions.

## Power Generation Design Bases

### The Fuel Building HVAC System:

- Provides a controlled environment for personnel comfort, and proper operation and integrity of equipment located in the building. See Table 9.4-3 for area temperatures.
- Maintains a negative pressure in the building to minimize exfiltration of potentially contaminated air. See Table 9.4-3 for system design pressures. The term “Slightly Negative Pressure” is applied hereafter and represents a pressure range from less than zero to -0.75” wg.
- Maintains airflow from areas of lower contamination potential to areas of greater contamination potential by maintaining a slightly negative pressure in potentially contaminated areas.
- Is provided with redundant active components to increase the reliability, availability, and maintainability of the system.
- Is capable of exhausting smoke, heat and gaseous combustion products in the event of a fire.
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.
- Shuts down during radiological accidents and isolates the Fuel Building boundary to prevent uncontrolled releases to the outside atmosphere.
- Provides capability to divert exhaust air to the Reactor Building HVAC Purge Exhaust Filter Units.
- Provides pool sweep ventilation air over the Spent Fuel Pool surface.
- Is designed such that failure of the system does not compromise or otherwise damage safety-related equipment.
- Is provided with shutoff dampers on the inlet and outlet of fans and AHUs to allow for maintenance as applicable.
- Is provided with shutoff valves at the inlet and outlet of cooling and heating coils to allow for maintenance as applicable.
- Is provided with access doors for AHUs, fans, filter sections, and duct-mounted dampers to allow for maintenance as applicable.

- Is provided with capability for manual control of system fans to facilitate testing and maintenance.
- Maintains its structural integrity after a Safe Shutdown Earthquake.

#### **9.4.2.2 System Description**

##### **Summary Description**

The FBVS maintains space design temperatures, quality of air, and pressurization in the Fuel Building. The system consists of two subsystems: the Fuel Building General Area HVAC Subsystem (FBGAVS) and the Fuel Building Fuel Pool Area HVAC Subsystem (FBFPVS). The FBGAVS serves the general areas of the Fuel Building. The FBFPVS serves the Spent Fuel Pool and equipment areas of the Fuel Building. Recirculation air handling units provide supplementary cooling for selected rooms in the Fuel Building as required.

##### **Detailed System Description**

###### **FBGAVS**

Figure 9.4-5 shows the simplified system diagram for the FBGAVS. Table 9.4-4 shows the major equipment for FBGAVS and Subsection 9.4.10 describes system components.

The FBGAVS is a once-through air conditioning and ventilation system with redundant air handling units (AHUs), exhaust fans and Fuel Building boundary isolation dampers. Each AHU includes filters, heating and cooling coils, and an AHU supply fan. Outside air is filtered and heated or cooled prior to being distributed by the AHU in service. A common supply duct system is incorporated to distribute conditioned air to the general areas of the Fuel Building. The exhaust fan discharges the air to the outside atmosphere through the monitored vent stack where the exhaust air is monitored for radioactivity. The exhaust air may be manually diverted to the Reactor Building HVAC Purge Exhaust Filter Unit. Electric unit heaters provide supplementary heating as required. A recirculation AHU provides supplementary cooling for the fine motion control rod drive (FMCRD) room. The hot water and chilled water systems provide heating and cooling water for the FBGAVS AHUs. The instrument air system provides instrument air for the pneumatic actuators.

The FBGAVS AHUs are located in the Fuel Building HVAC Equipment Area.. The FBGAVS exhaust fans are located in the Reactor Building. The FMCRD recirculation AHU is located in the Fuel Building.

###### **FBFPVS**

Figure 9.4-6 shows the system diagram for the FBFPVS. Table 9.4-5 shows the major equipment for the FBFPVS.

The FBFPVS is a once-through air conditioning and ventilation system with redundant AHUs and exhaust fans. Each AHU includes filters, heating and cooling coils, and an AHU supply fan. Outside air is filtered, heated or cooled, and distributed across the Spent Fuel Pool surface and to the equipment areas. Air is exhausted from the Spent Fuel Pool, through redundant Fuel Building boundary isolation dampers, to the outside atmosphere through the vent stack. During high radiation conditions, the exhaust air may be manually diverted to the Reactor Building HVAC Purge Exhaust Filter Unit. The exhaust fans are also used for smoke removal. Electric

unit heaters provide supplementary heating as required. The hot water and chilled water systems provide heating and cooling water for the FBFVPS AHUs. Instrument air is provided for the pneumatic actuators.

The FBFVPS AHUs are located in the Fuel Building HVAC Equipment Area.. The FBFVPS exhaust fans are located in the Reactor Building.

During high radiation conditions, the Fuel Building boundary isolation dampers close automatically and the supply AHU and exhaust fan shut down automatically in both subsystems.

### **System Operation**

The FBVS operates during all normal, startup and shutdown modes of plant operation.

During normal operation, both the FBGAVS and FBFVPS are fully operable. Each subsystem operates with one supply AHU and one exhaust fan in service. The redundant AHU and fans are maintained on standby. In the event of low airflow in an exhaust duct, the standby exhaust fan starts. Simultaneously, due to a loss of negative pressure in the area, the AHU supply fan serving the area stops. The AHU supply fan restarts upon reestablishment of the required negative pressure. In the event of a fan failure, the failed fan automatically shuts down and the standby fan automatically starts.

On detection of high radiation, the Process Radiation Monitoring System provides a signal that trips the FBGAVS and FBFVPS. Each subsystem's supply AHU and exhaust fan shuts down and their associated dampers close. Exhaust air from either subsystem may be manually diverted to the Reactor Building HVAC Purge Exhaust Filter Unit. It is then exhausted to the plant vent stack by the Reactor Building HVAC Purge Exhaust Filter Unit exhaust fan. Normal ventilation for the area is resumed once the area is decontaminated or the source of radioactivity is removed.

If smoke is detected in a supply air duct, the affected subsystem is shut down. In the event of a fire, fire dampers close to isolate the fire area. Following a recovery from a fire, a subsystem exhaust fan removes smoke from the area by exhausting to the outside.

The FMCRD room AHU fan is started and stopped locally. A room thermostat modulates the chilled water valve in response to the room temperature.

An individual local thermostat controls each electric unit heater.

#### **9.4.2.3 Safety Evaluation**

The FBVS is not required to operate during a Station Blackout (SBO).

The Fuel Building boundary isolation dampers automatically close in the event of a fuel handling accident or other radiological accident. The safety-related isolation dampers fail closed upon a loss of control signal, power, or instrument air.

The FBVS components are designed as Seismic Category II, except for the safety-related isolation dampers and associated controls. The isolation dampers and associated controls are designed as Seismic Category I.

The FBVS does not have any safety-related functions, except for boundary isolation dampers closing in the event of radiological accidents. Redundant dampers and controls are provided so the refueling area is isolated upon demand even if a damper or control fails.

#### ***9.4.2.4 Testing and Inspection Requirements***

The FBVS is designed to permit periodic inspection and testing of major components, such as fans, motors, dampers, coils, filters, ducts, piping, and valves to verify their integrity and capability. Equipment layout provides easy access for inspection and maintenance.

Routine testing of the FBVS is conducted in accordance with normal power plant requirements. This testing demonstrates system and component operability and integrity.

Surveillance testing of safety-related isolation dampers is carried out in accordance with IEEE-338.

The Reactor Building HVAC Purge Exhaust Filter Unit is tested in accordance with Regulatory Guide 1.140.

#### ***9.4.2.5 Instrumentation Requirements***

The FBVS is operated from the Main Control Room. The FBVS is manually controlled, except for certain automatic operations described below:

- Fuel Building boundary isolation dampers close on receipt of a high radiation signal or on a loss of AC power, control power, or instrument air;
- For supply and exhaust fans, the lead fan is selected manually. The standby fan automatically starts upon indication of low flow in the associated discharge duct.
- Fan operation is allowed only when the corresponding fan shutoff dampers (one upstream and one downstream) are open;
- Supply fans auto start after the exhaust fans have started and a negative pressure has been established in the ventilated spaces; and
- Differential pressures between the ventilated spaces and the outside are transmitted to a pressure controller. The pressure controller adjusts the exhaust fan's flow with variable speed motor drives or with variable inlet vanes. This exhaust airflow control maintains a negative pressure.

The FBVS component operating status and system parameters are monitored and indicated in the MCR and locally where required.

Indications and alarms include the following:

- Indicators for system operating parameters, including flow rates, temperatures, damper position, filter pressure drop and building pressure with respect to atmospheric; and
- Alarms for high or low conditions, including airflow rates, temperatures, filter pressure drop, building differential pressure and smoke detection.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### ***9.4.2.6 COL Information***

None

### 9.4.2.7 References

The applicable HVAC codes and standards are shown in Table 9.4-17.

### 9.4.3 Radwaste Building Heating, Ventilating and Air Conditioning System

The Radwaste Building HVAC System (RWVS) consists of the following subsystems:

- Radwaste Building Control Room (RWCR) HVAC Subsystem (RWCRVS); and
- Radwaste Building General Area (RWGA) HVAC Subsystem (RWGAVS).

For the nonsafety-related portions of the RWVS, this subsection addresses or refers to other DCD locations that address the applicable requirements of General Design Criteria (GDC) 2, 5 and 60 discussed in Standard Review Plan (SRP) 9.4.3 (see Table 1.9-20). The ESBWR:

- Meets GDC 2, by compliance to Regulatory Guide 1.29, Position C.2 for nonsafety-related portions. The Radioactive Waste (RW) Building structure meets RG 1.143 category RW-IIa.
- Meets GDC 5, compliance for safety-related shared systems and components. There are no safety-related shared systems and/or components in the RWVS.
- Meets GDC 60, by suitably controlling the release of gaseous radioactive effluents to the environment. The system directs exhaust air to the RWGA exhaust filtration units. The RWGA exhaust filtration units are designed, tested and maintained in accordance with Regulatory Guide 1.140. The RWGAVS contains 3-50% capacity exhaust filtration units. These units exhaust air from the building, which is maintained under negative pressure, to the vent stack.

#### 9.4.3.1 Design Bases

The RWVS provides a controlled environment for personnel comfort and for proper operation and integrity of equipment.

#### RWCRVS

- The RWCRVS maintains the Radwaste Building control room area temperature.
- The RWCRVS maintains the control room areas at a slightly positive pressure relative to adjacent areas to minimize infiltration of air. The term “Slightly Positive Pressure” used hereafter is a range from greater than zero to + 0.75” wg.
- Redundant components are provided to increase system reliability, availability and maintainability.

#### RWGAVS

- The RWGAVS maintains the conditions in the RWGA as provided in Table 9.4-6.
- The RWGAVS maintains the Radwaste Building general area at a slight negative pressure relative to adjacent areas and outside atmosphere to prevent the exfiltration of air to adjacent areas. The term “Slightly Negative Pressure” is applied hereafter and represents a pressure range from less than zero to -0.75” wg. Adequate exhaust from the

trailer bays is provided to maintain inflow of air from the outside when the truck doors are open.

- The RWGAVS is comprised of supply and exhaust subsystems to maintain direction of air flow from personnel occupancy areas towards areas of increasing potential contamination. Exhaust hoods are provided at locations where, under normal operation, contaminants could escape to the surrounding areas.
- The RWGAVS provides the capability to exhaust air from the mobile radwaste processing systems.
- All exhaust air from the RWGA is discharged to the plant vent stack.
- Redundant components are provided as necessary to increase system reliability, availability and maintainability.
- Filtered outdoor air is provided at a minimum air exchange rate of two volume changes per hour.
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.
- The RWGAVS limits the release of airborne radioactive particulates to the atmosphere by HEPA filtration of the exhaust air from the building prior to discharge to the atmosphere.
- The exhaust air is monitored for radiation prior to discharge to atmosphere.

#### **9.4.3.2 System Description**

##### **Summary Description**

##### **RWCRVS**

The RWCRVS is a recirculating air conditioning system to provide filtered, heated or cooled, and humidified air to the RWCR area to maintain the required design ambient conditions and pressurization. The RWCR consists of two 100% capacity AHUs and a common outside air intake louver. Each AHU contains filters, a humidifier, a chilled water cooling coil, a hot water heating coil, and a supply fan. Conditioned air is supplied to the control room area through ducts, dampers and registers.

The RWCRVS is capable of once-through operation for smoke removal using a separate 100% capacity exhaust fan.

##### **RWGAVS**

The RWGAVS is a once-through air conditioning and ventilation system to provide filtered and heated or cooled, and humidified air to the RWGA. The RWGAVS supply consists of two 100% capacity AHUs connected to a common supply distribution ductwork and a common outside air intake louver. Isolation dampers are provided at the inlet and outlet of each unit. Each AHU contains filters, cooling and heating coils, a humidifier, a fan, and dampers.

The RWGAVS exhaust consists of three 50% capacity filtration trains, each with prefilters and HEPA filters, a 50% capacity exhaust fan, and a check valve/backdraft damper. Exhaust capacity is greater than the supply capacity in order to maintain the required RWGA negative pressure. Each filtration train is connected to a common exhaust collection duct and a common exhaust duct discharging to the plant vent stack.

The RWGAVS exhaust subsystem is capable of once-through operation for smoke removal. The HEPA filters are bypassed in this mode.

### **Detailed System Description**

#### **RWVS**

The RWVS simplified system diagram is provided in Figure 9.4-7. The system has Design Conditions in Table 9.4-6, Major Equipment listed in Table 9.4-7, and Component Information described in Subsection 9.4.10.

The RWVS equipment is located in the Radwaste Building HVAC Fan and Filter room.

All RWVS equipment is classified as nonsafety-related, seismic Category NS, and is powered from a nonsafety-related bus distribution system.

The Hot Water System and the Chilled Water System provides the heating and cooling for the AHUs. Makeup water is provided to the humidifiers and instrument air is provided to the pneumatic controls.

Hot water from the Hot Water System is piped to and from heating coils in the AHUs. Flow controlled valves adjust the hot water flow through the heating coils by using signals from temperature sensors in the Radwaste Building.

#### **System Operations**

The RWCRVS operates by mixing return air and outside air. The air mixture is filtered, cooled or heated, humidified and distributed to the RWCR area. One 100% capacity AHU normally operates.

Following a fire, the system can be placed in the purge mode by manually starting the smoke removal fan and operating the supply system in the once-through mode.

The RWGAVS operates in the once-through mode. One 100% supply AHU and two 50% exhaust filtration trains operate during normal plant operation. The second AHU and the third 50% exhaust filtration train are in standby. The supply fans are interlocked with the exhaust fans. Starting the RWGAVS starts the lead filtration train exhaust fan. Once negative pressure in RWGA is established, the outside air intake damper opens and the lead AHU supply fan starts. Outside air is filtered, heated or cooled, humidified, and distributed to the RWGA.

Failure of one operating exhaust fan that results in the loss of negative pressure in the RWGA automatically starts the redundant standby exhaust fan (and filtration train). The supply fan

continues to operate. The negative pressure in the building can be lost momentarily without damage to the building structure. If the standby exhaust fan does not start, the supply fan and the operating exhaust fan are stopped.

Following a fire, the RWGA Exhaust Subsystem is placed in the smoke removal mode by closing the HEPA filtration inlet and outlet dampers, opening the bypass duct dampers, and operating the exhaust fan(s).

#### **9.4.3.3 Safety Evaluation**

The RWVS has no safety-related function. Operational failure of any single unit of the RWVS does not prevent safety-related equipment from performing any safety-related function.

#### **9.4.3.4 Testing and Inspection Requirements**

The RWVS is designed to permit periodic inspection and testing of major components, such as fans, motors, dampers, coils, filters, ducts, piping and valves to verify their integrity and capability. Equipment layout provides easy access for inspection and maintenance.

RWVS components are tested periodically to verify system availability. Filtration units are periodically tested in accordance with Regulatory Guide 1.140, Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants.

#### **9.4.3.5 Instrumentation Requirements**

Instrumentation and controls for the RWVS are located at a local panel in the Radwaste Building. Selected HVAC alarm signals are transmitted to the general trouble alarm located in the RWCR. These indications and controls include the following:

- Start-Stop-Auto control switches for system and operating equipment;
- System operating parameters (including flow rate, temperature, humidity and differential pressure);
- Damper position indication and controls;
- Failure alarms for major components; and
- High radiation in the exhaust air duct.

The RWVS is manually controlled, except for certain automatic operations described below.

For the RWGA exhaust fans, the lead fans are started manually and the standby fan automatically starts upon indication of low flow in either lead fan discharge duct.

Interlocks only allow fan operation when the corresponding fan isolation dampers (one upstream and one downstream) are open. For the RWGAVS the interlock only allows the lead supply fan to run when negative pressure is established in the associated spaces. Differential pressures between these areas and the outside are transmitted to a pressure controller (located on the HVAC board in the RWCR) that modulates airflow by controlling fan speed or adjusting the exhaust fan inlet vanes. This exhaust airflow modulating maintains negative air pressure in the ventilated spaces.

A temperature controller modulates the temperature control valves in the water supply lines to the cooling and heating coils from signals from temperature transmitters in the RWCR and general areas. A relative humidity controller adjusts the RWCRVS humidifier to maintain the humidity within the desired range in the RWCR.

A low differential pressure across the operating fan sounds an alarm in the RWCR, de-energizes the failed fan, and starts the corresponding standby fan. The fan discharge damper closes when the fan is de-energized.

A general interlock between exhaust fans and supply fans stops all the RWGAVS fans when the RWGA air pressure becomes higher (or lower) than specified to avoid damage to the building structure.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.4.3.6 COL Unit-Specific Information**

None

#### **9.4.3.7 References**

The applicable HVAC codes and standards are shown in Table 9.4-17.

### **9.4.4 Turbine Building HVAC System**

The Turbine Building HVAC System (TBVS) includes the Turbine Building supply air fans and associated AHUs, and the Turbine Building exhaust fans and associated filter trains. The various fan-coil units for local area heating and cooling within the Turbine Building are included in the TBVS.

Regarding the ESBWR nonsafety-related TBVS, this subsection addresses or refers to other locations that address the applicable requirements of the General Design Criteria (GDC) 2, 5 and 60, discussed in Standard Review Plan (SRP) 9.4.4 (Table 1.9-20).

The ESBWR:

- Meets GDC 2, as it relates to meeting the guidance of Regulatory Guide 1.29 (Table 1.9-21), Position C.2 for nonsafety-related portions. The TB is a Seismic Category II nonsafety-related structure.
- Meets GDC 5, for shared systems and components. There are no safety-related shared (multiple reactor units) structures, systems or components in the TBVS.
- Meets GDC 60 by suitably controlling the release of gaseous radioactive effluents to the environment. The system directs building exhaust air to the TBVS system filtration units. The TBVS exhaust system (TBE) filtration units contain medium and high efficiency filters, which remove particulate from the air prior to discharge to the plant vent stack. This system uses high efficiency filters in the TB, which is generally a low contamination area, which meets the intent of GDC 60. The high efficiency filters assure radioactive material entrained in gaseous effluent will not exceed the limits specified in 10 CFR Part 20, for normal operations and anticipated operational occurrences. TBVS subsystems are equipped with high efficiency and HEPA filtration units for localized air

cleanup prior to mixing with the main ventilation exhaust (TBE). The local HEPA units are designed, tested and maintained in accordance with Regulatory Guide 1.140. The TBE combined ventilation exhaust is monitored for halogens, particulates and noble gas releases. The TB Compartment area and normal ventilation HVAC RMS subsystems monitor air for gross radiation levels and alarm functions. The TB is maintained at a slight negative pressure to minimize exfiltration. TB Equipment rooms are maintained at a negative pressure to minimize potential airborne radioactivity escaping to adjacent areas or to the outside atmosphere during normal operation by exhausting air through filters from the areas in which a significant potential for contamination exists.

#### **9.4.4.1 Design Bases**

##### **Safety (10 CFR 50.2) Design Bases**

The TBVS does not perform or ensure any safety-related function, and thus, has no safety design basis.

##### **Power Generation Design Bases**

The TBVS:

- Provides temperature control and air movement control for personnel comfort;
- Optimizes equipment performance by the removal of heat dissipated from plant equipment;
- Provides a sufficient quantity of filtered fresh air for personnel;
- Provides for air movement from areas of lesser potential airborne radioactivity to areas of greater potential airborne radioactivity prior to final exhaust;
- Minimizes the possibility of exhaust air recirculation into the air intake;
- Minimizes the escape of potential airborne radioactivity to the outside atmosphere during normal operation;
- Provides capability to exhaust the smoke, heat and gaseous combustion products in event of a fire;
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.
- Maintains the Turbine Building minimum required exhaust capability, as well as the fan coil units of the Reactor Component Cooling Water System (RCCWS), Nuclear Island

subsystem of the Chilled Water System (CWS) and Instrument Air System (IAS) rooms in operation during Loss of Preferred Power (LOPP);

- Provides an adequate level of standby cooling capacity in areas where loss of cooling could cause degraded equipment performance; and
- Chiller mechanical equipment rooms meet ASHRAE-15, Safety Standard for Refrigeration Systems. They are equipped with a dedicated ventilation system and leak detectors with alarms.

#### ***9.4.4.2 System Description***

##### **Summary Description**

Figure 9.4-8 shows the TBVS Simplified System Diagram, Table 9.4-15 lists TBVS Design Parameters and Subsection 9.4.10 describes Component Information.

The TBVS supplies air from the outside to the Turbine Building non-contaminated areas and removes it from potentially contaminated rooms.

The main stairwells are pressurized to prevent infiltration of smoke from other Turbine Building areas in the event of a fire.

The TBVS is designed to minimize exfiltration by maintaining a slightly negative pressure in the Turbine Building by exhausting more air than is supplied to the Turbine Building. The pressure hereafter called “Slightly Negative Pressure” is a range from less than zero to -0.75” wg.

Exhaust air from potentially high airborne contamination Turbine Building areas or component vents is collected, filtered, and discharged to the atmosphere through the Turbine Building Compartment Exhaust (TBCE) system.

Exhaust air from other (low potential airborne contamination) Turbine Building areas and component vents, except lube oil areas, is exhausted to the atmosphere through the Turbine Building Exhaust (TBE) system.

Turbine Building exhaust air is directed to the plant vent stack where it is monitored for radiation prior to being discharged to the atmosphere.

The TBVS is designed to provide for local air recirculation and cooling in high heat load areas using local unit coolers. A minimum of 50% standby cooling capacity is provided in areas where a loss of cooling could cause degraded equipment performance.

Turbine Building ventilation systems and subsystems required for normal plant operation are provided with redundant fans with automatic start logic.

The TBVS equipment is located in the Turbine Building.

The chiller rooms, located in the turbine building, meet ASHRAE-15 (Table 1.9-22), Safety Standard for Refrigeration Systems. They are equipped with a dedicated ventilation system and leak detectors with alarms.

## Detailed System Description

### Turbine Building Air Supply (TBAS) Subsystem

The TBAS consists of outside air intake louvers, dampers, filters, hot water heating coils, chilled water cooling coils, and three 50% capacity constant volume supply fans.

Two of the three fans are normally operating to supply filtered, temperature-controlled air to all levels of the Turbine Building. The third fan is a standby unit that starts automatically upon failure of either operating fan. On the outlet common duct there is one pneumatically operated damper to maintain the desired negative pressure in the Turbine Building. Each supply fan is provided with a pneumatically operated isolation damper. The TBAS uses 100% outside air during normal plant operation.

The TBAS fans are started manually from the MCR. The supply fans are interlocked with the TBVS exhaust fans and TBVS compartment exhaust fans to ensure that the exhaust fans are running before a supply fan is started.

Temperature controllers located at the heating and cooling coils air outlet modulate the TBAS air handling heating and cooling coil operation.

### Turbine Building Exhaust (TBE) Subsystem

The TBE fans exhaust air from the building clean and low potential contamination areas. The exhaust air is filtered through filters and exhausted through the monitored vent stack.

The TBE subsystem is provided with three 50% capacity fans downstream of the filter train. Two fans are normally in operation and one is in automatic standby.

A filter bypass is provided to allow purging smoke from Turbine Building in the event of a fire. All three TBE fans can be operated simultaneously to provide maximum smoke removal, if necessary.

Each TBE fan is provided with pneumatically operated inlet vanes or variable speed drives and isolation dampers. A flow controller automatically adjusts the blade pitch or speed of the operating fans to vary the system airflow rate. Failure of one operating exhaust fan automatically starts the standby fan. The TBVS exhaust fans are interlocked with the TBAS fans.

### Turbine Building Compartment Exhaust (TBCE) Subsystem

The TBCE subsystem consists of two 100% capacity exhaust fans, one filter unit and associated controls. One fan is normally in operation with the other one in automatic standby. The subsystem includes a 100% capacity filter bypass duct for purging smoke in the event of a fire.

The air exhausted from the Turbine Building high potential airborne contamination compartments and equipment vents is filtered through a filter before it is released to the atmosphere through the plant vent stack, except during smoke removal.

The TBCE subsystem has radiation detectors in the exhaust duct to monitor the air for radioactivity prior to its being discharged to the plant vent stack.

The two exhaust fans are provided with inlet vanes or variable speed drives and isolation dampers. An airflow controller automatically adjusts the inlet vanes or speed of the operating

fan to vary the system exhaust flowrate. In the automatic mode, loss of flow from the operating fan starts the standby fan.

### **Turbine Building Lube Oil Area Exhaust (TBLOE) Subsystem**

The TBLOE subsystem includes two 100% capacity fans, isolation dampers and exhaust ductwork. The TBLOE fans discharge the exhaust air directly to the atmosphere through the plant vent stack, passing first through the air filtration unit of TBE subsystem. One fan is designed to continuously exhaust at a constant volumetric flow rate from the Turbine Lube Oil Tank Room. A bypass duct is provided around the lube oil exhaust fans for purging high temperature combustion products and limiting room pressurization in the event of a fire in one of the rooms.

### **Turbine Building Decontamination Room Exhaust (TBDRE) Subsystem**

The TBDRE subsystem consists of one air filtration unit, which includes one 100% capacity exhaust fan, a filter, an isolation damper and associated controls.

The air exhausted from the TBDRE, once filtered, passes through the air filtration unit of TBE subsystem and is finally released to the atmosphere through the plant vent stack, except during smoke removal.

The subsystem includes a 100% capacity filter bypass duct for purging smoke in the event of a fire.

### **TBVS Unit Coolers and Unit Heaters**

Localized unit coolers and unit heaters are provided as required in various locations within the Turbine Building. The unit coolers are supplied with chilled water from the balance of plant subsystem of the Chilled Water System (CWS) and the unit heaters are supplied with heated water from the hot water system. The system provides redundant unit coolers to allow operation of associated equipment with a unit cooler out of service, or to maintain cooling upon the failure of one unit cooler.

Temperature controls for the unit coolers and unit heaters are located in the unit inlet air path or are installed locally.

The cooling coils of the Reactor Components Cooling Water System (RCCWS), Nuclear Island subsystem of the CWS and Instrument Air System rooms are fed from the corresponding Nuclear Island subsystem of the CWS train.

### **System Operation**

The TBVS is designed to operate during all modes of normal power plant operation, including start-up and shutdown.

#### **Normal Operation**

The TBVS fans are started manually and operate automatically thereafter. Standby fans start automatically if one of the running fans trip due to low flow or equipment trip.

#### **Smoke Purge Mode**

Upon detection of smoke in the Turbine Building, the TBAS outside air supply fans and the TBE subsystem exhaust fans stop automatically.

Upon detection of smoke in the TBCE or the TBLOE or the TBDRE subsystems MCR operators bypass the subsystem filters manually.

MCR operators normally initiate the smoke purge mode of operation of the Turbine Building. Smoke purge is accomplished by starting two supply fans in the TBAS and two exhaust fans in the TBE subsystem. This provides 100% outside air. All the three fans in the TBAS and in the TBE subsystem can be started to provide maximum smoke removal.

#### **Loss of Preferred Power (LOPP)**

Upon a LOPP, at least one of the fans of the TBE subsystem remains in operation because they are powered from the nonsafety-related diesel generators.

The fan coil units of the RCCWS, Nuclear Island subsystem of the CWS and Instrument Air System rooms also remain in operation.

#### **9.4.4.3 Safety Evaluation**

The TBVS does not perform or ensure safety-related function.

Where a system is provided with a redundant fan, failure of an operating fan automatically starts the standby fan to maintain continuity of ventilation.

The exhaust air from the TBVS is monitored for radioactivity prior to discharge to the plant vent. Alarms annunciate in the MCR upon detection of high radiation. Section 11.5 describes the radiological monitoring system.

The TBVS components are designed as Seismic Category NS.

#### **9.4.4.4 Tests and Inspections**

All major components are tested and inspected as separate components prior to installation and as integrated systems after installation to ensure design performance. Ductwork system airflows are measured and adjusted to meet design requirements and all instruments are calibrated to the design setpoints. The systems are preoperational tested in accordance with the requirements of Chapter 14.

Periodic inspections and measurements including air flows, water flows, air and water temperatures, filter pressure drops, controls positions, are taken to verify the systems operating conditions and to ensure the integrity of the systems for normal plant operation.

The TBVS filtration components are periodically tested in accordance with Regulatory Guide 1.140, Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants.

#### **9.4.4.5 Instrumentation Requirements**

All control actuations, indicators, and alarms for normal plant operation are located in the MCR. Controls and instrumentation for the TBVS include:

- Heating and cooling temperature indicators, controls and alarms for the entering air;
- Low and high temperature switches and alarms for heated and cooled air supply;
- Differential pressure indicators, differential pressure switches and alarm for the air filters;

- Supply airflow indicator and controls, alarms and trips for supply fans; and
- Airflow failure switch, alarm and trip for each exhaust fan

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.4.4.6 COL Unit Specific Information**

None

#### **9.4.4.7 References**

The applicable HVAC codes and standards are shown in Table 9.4-17.

### **9.4.5 Engineered Safety Feature Ventilation System**

The EFU portion of the CRHAVS supplies the engineered safety feature for CRHA radiological protection as described in Section 6.4 and Subsection 9.4.1.

### **9.4.6 Reactor Building HVAC System**

The Reactor Building HVAC System (RBVS) serves the following areas of the Reactor Building:

- The potentially contaminated areas;
- The refueling area;
- The non-radiologically controlled areas; and
- The RBVS does not serve the primary containment except during inerting operations.

Relative to the RBVS, this subsection addresses applicable requirements of General Design Criteria (GDC) 2, 5 and 60. These GDCs are discussed in Standard Review Plan (SRP) 9.4.3 (see Table 1.9-20) The ESBWR:

- Meets GDC 2 via compliance to the guidance of Regulatory Guide 1.29, Position C.2 for nonsafety-related portions. The RBVS is nonsafety-related except for the building isolation dampers. The RBVS components are designed as Seismic Category II except for the safety-related building isolation dampers and associated controls that are Seismic Category I. The RB is a Seismic Category I structure. The Fuel Building penthouse that houses the RBVS equipment is Seismic Category II.
- Meets GDC 5 for shared systems and components important to safety for the Reactor Building isolation dampers. The RBVS is not shared among other operating units.
- Meets GDC 60 by suitably controlling the release of gaseous radioactive effluents to the environment. The system may direct its exhaust air to the Reactor Building HVAC Purge Exhaust Filter Unit during periods of high radioactivity. The Reactor Building HVAC Purge Exhaust Filter Unit is designed, tested and maintained in accordance with Regulatory Guide 1.140. The RBVS exhaust system is equipped with a control system to automatically isolate the effluent on indication of a high radiation level. The RB boundary isolation dampers close on receipt of a high radiation signal, or on a loss of AC power.

### 9.4.6.1 Design Bases

#### Safety (10 CFR 50.2) Design Bases

With the following exception, the RBVS is nonsafety-related. The isolation dampers and ducting penetrating the Reactor Building boundary and associated controls that provide the isolation signal are safety-related. The RBVS performs no safety-related function except for automatic isolation of the Reactor Building boundary during accidents. The RBVS has nonsafety-related Reactor Building Purge Exhaust Filter Units for mitigating and controlling gaseous effluents from the Reactor or Fuel buildings.

#### Power Generation Design Bases

The RBVS:

- Provides a controlled environment for personnel comfort and safety, and for proper operation and integrity of equipment. See Table 9.4-8 for area temperatures maintained.
- Maintains potentially contaminated areas at a negative pressure to minimize exfiltration of potentially contaminated air. See Table 9.4-8 for area pressurization;
- Maintains clean areas of the building, except for the battery rooms, at a positive pressure to minimize infiltration of outside air. See Table 9.4-8 for area pressurization.
- Maintains airflow from areas of lower potential for contamination to areas of greater potential for contamination. The pressure in these areas hereafter called “Slightly Negative Pressure” is a range from less than zero to  $-0.75$ ” wg.
- Is provided with redundant active components to increase the reliability, availability, and maintainability of the systems.
- Is capable of exhausting smoke, heat and gaseous combustion products in the event of a fire.
- Prevents smoke and hot gases from migrating into other fire areas by automatically closing smoke dampers upon detection of smoke.
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.
- Shuts down during radiological events and isolates the Reactor Building boundary to prevent uncontrolled releases to the outside atmosphere.

- Provides the capability to manually divert exhaust air for processing through the Reactor Building HVAC Purge Exhaust Filter Units.
- Provides pool sweep ventilation air over the refueling area pool surface.
- Maintains its structural integrity after a safe shutdown earthquake.
- Is designed such that failure of the system does not compromise or otherwise damage safety-related equipment.
- Is provided with shutoff dampers on the inlet and outlet of fans and AHUs to allow for maintenance as required.
- Is provided with shutoff valves at the inlet and outlet of cooling and heating coils to allow for maintenance as required.
- Is provided with access doors for AHUs, fans, filter sections, and duct mounted dampers to allow for maintenance as required.
- Is provided with capability for manual control of system fans to facilitate testing and maintenance.
- Maintains the hydrogen concentration levels in the battery rooms below 2% by volume in accordance with RG 1.128.
- Replaces the primary containment inerted atmosphere with conditioned air during a refueling operation.
- Provides filtered exhaust capability for the Fuel Building through the Reactor Building HVAC Purge Exhaust Filter Unit.

#### ***9.4.6.2 System Description***

##### **Summary Description**

The RBVS maintains space design temperature, quality of air, and pressurization in the Reactor Building. The system consists of three subsystems. The Reactor Building Contaminated Area HVAC Subsystem (CONAVS) serves the potentially contaminated areas of the Reactor Building. The Refueling and Pool Area HVAC Subsystem (REPAVS) serves the refueling area of the Reactor Building. The Reactor Building Clean Area HVAC Subsystem (CLAVS) serves the clean (non-radiological controlled) areas of the Reactor Building.

##### **Detailed System Description**

###### **CONAVS**

Figure 9.4-10 shows a simplified system diagram for the CONAVS. Table 9.4-11 shows the major equipment for the CONAVS and Subsection 9.4.10 describes component information.

The CONAVS is a once-through ventilation system and consists of redundant AHUs; exhaust fans and building isolation dampers. It includes a primary containment purge exhaust fan, recirculation AHUs and unit heaters. Each AHU includes filters, heating and cooling coils and a supply fan. Outside air is filtered and heated or cooled prior to distribution by the AHU in service. The chilled water system provides cooling water for the CONAVS AHUs. The Instrument Air System provides instrument air for the pneumatic actuators. A common supply

air duct distributes conditioned air to the potentially contaminated areas of the Reactor Building. Air is exhausted from the potentially contaminated areas of the Reactor Building by the operating exhaust fan and discharged to the plant vent stack. During primary containment de-inerting operations the supply airflow rate of the AHU supply fan is increased. At the same time the airflow rate of the exhaust fan is increased an equal amount. In the event of a fire, fire dampers close to isolate the fire area. In the event smoke is detected in the supply air duct, the affected subsystem is shut down. The exhaust fans are then used for smoke removal with the exhaust air being monitored for radiological contamination. If contaminated, temporary portable filters may be used to exhaust the contaminated air. The building isolation dampers close and the supply and exhaust fans stop due to high radiation in the exhaust ducts. CONAVS also includes redundant Reactor Building HVAC Purge Exhaust Filter Units and exhaust fans. During radiological events, exhaust air from contaminated areas may be manually diverted through the Reactor Building HVAC Purge Exhaust Filter Units. The RB purge exhaust filter units are equipped with pre-filters, HEPA filters and carbon filters for mitigating and controlling gaseous effluents from the Reactor or Fuel buildings. The supply AHUs and normal exhaust fans are shut down during filtered purge exhaust. Recirculation AHUs provide supplementary cooling for selected rooms. Electric unit heaters provide supplementary heating.

The CONAVS AHUs are located in the Fuel Building HVAC Equipment Area. The CONAVS exhaust fans are located in the Reactor Building. The Reactor Building HVAC Purge Exhaust Filter Units and exhaust fans are located in the Reactor Building. The steam tunnel recirculating AHUs are located in the Turbine Building.

The refueling machine control room recirculating AHU is located in the Reactor Building. Electric unit heaters are located in or near the areas they serve.

## **REPAVS**

Figure 9.4-11 shows a simplified system diagram for the REPAVS. Table 9.4-10 shows the major equipment for the REPAVS.

The REPAVS is a once-through ventilation system and consists of redundant AHUs, exhaust fans and building isolation dampers. Each AHU includes filters, heating and cooling coils and a supply fan. Outside air is filtered and heated or cooled prior to distribution by the AHU in service. The conditioned air is distributed to the refueling area and across the pool surface. Exhaust air is ducted to the exhaust fans and exhausted to the outside atmosphere through the plant vent stack. During a radiological event, exhaust air from the refueling area may be manually diverted through the Reactor Building HVAC Purge Exhaust Filter Units. The chilled water system provides cooling water for the REPAVS AHUs. The instrument air system provides instrument air for the pneumatic actuators. In the event of a fire, fire dampers close to isolate the fire area. In the event smoke is detected in the supply air duct, the affected subsystem is shut down. The exhaust fans are then used for smoke removal with the exhaust air being monitored for radiological contamination. If contaminated, temporary portable filters are used to exhaust the contaminated air. The building isolation dampers close and the supply and exhaust fans stop due to high radiation in the exhaust ducts.

The REPAVS AHUs are located in the Fuel Building HVAC Equipment Area. The REPAVS exhaust fans are located in the Reactor Building. Electric unit heaters are located in or near the areas they serve.

## CLAVS

Figure 9.4-9 shows a simplified system diagram for the CLAVS. Table 9.4-9 shows the major equipment for the CLAVS.

The CLAVS is a recirculating ventilation system with redundant AHUs, return/exhaust fans and smoke exhaust fans. Each AHU includes filters, heating and cooling coils and a supply fan. A mixture of outside and return air is filtered and heated/cooled prior to distribution by the AHU in service. A common supply and return/exhaust air duct system distributes conditioned air to and from the Reactor Building clean areas. Return air not directed back to the AHU is exhausted directly outdoors. An economizer cycle is used, when outside air conditions are suitable, to reduce mechanical cooling operating hours. The economizer cycle provides all outside air, or a mixture of outside air and return air, to Reactor Building clean areas. The temperature of the air provided is at or below the supply air design temperature. In the event of a fire, fire dampers close to isolate the fire area. In the event smoke is detected in the supply air duct, the affected subsystem is shut down. The CLAVS smoke exhaust fans are then used for smoke removal. The chilled water system provides cooling water for the CLAVS AHUs. The instrument air system provides instrument air for the pneumatic actuators. Electric unit heaters provide supplementary heating. The CLAVS AHU supplies air to the battery rooms. The air is then exhausted from the battery rooms by the battery room exhaust fans which discharge directly to the plant vent stack.

The CLAVS AHUs and return/exhaust fans are located in the Fuel Building HVAC Equipment Area. The CLAVS smoke exhaust fans are located in the Reactor Building. The electric unit heaters are located in or near the areas they serve.

### System Operation

The RBVS operates during normal power plant operation, plant startup, and plant shutdown. It is not required to operate during a Station Blackout.

## CONAVS

During normal operation, the CONAVS operates with one AHU and one exhaust fan in service. The exhaust fan starts first to establish negative pressure in the areas served. Then the AHU supply fan starts. Failure of an operating exhaust fan automatically energizes the standby exhaust fan. Simultaneously, the CONAVS AHU supply fan is de-energized due to a loss in room negative pressure. The AHU supply fan is re-energized upon reestablishment of room negative pressure.

Before and during personnel entry into the primary containment area, the CONAVS is used to de-inert the primary containment. The CONAVS AHU supply fan provides purge supply air to primary containment while the primary containment purge exhaust fan exhausts air from primary containment. On detection of high radiation in the exhaust air by PRMS, supply and exhaust dampers to primary containment are automatically closed. During inerting operation, the CONAVS exhausts air from primary containment while the Containment Inerting System supplies nitrogen to the primary containment.

## REPAVS

During normal operation, the REPAVS operates with one AHU and one exhaust fan in service. The exhaust fan starts first to establish negative pressure in the areas served. Then the AHU supply fan starts. Failure of an operating exhaust fan automatically energizes the standby exhaust fan. Simultaneously, the REPAVS AHU supply fan is de-energized due to a loss in room negative pressure. The AHU supply fan is re-energized upon reestablishment of room negative pressure.

### **CLAVS**

During normal operation, the CLAVS operates with one AHU and one return/exhaust fan in service. When outside air conditions are suitable, the CLAVS incorporates an economizer cycle to reduce operating hours for mechanical cooling equipment. Failure of an operating AHU supply fan automatically energizes the standby AHU supply fan and de-energizes the failed fan. Simultaneously, the return/exhaust fan is de-energized due to a loss in room pressurization. The return/exhaust fan is re-energized upon reestablishment of room positive pressure.

Following a fire recovery, exhaust fans or smoke exhaust fans are used to remove smoke from the area by exhausting to the outdoors.

An individual local thermostat controls each RBVS electric unit heater.

#### ***9.4.6.3 Safety Evaluation***

The RBVS is nonsafety-related, except for the building isolation dampers. The safety-related isolation dampers fail closed upon a loss of control signal, power, or instrument air.

The RBVS components are designed as Seismic Category II, except for the safety-related building isolation dampers and associated controls. The building isolation dampers and associated controls are designed as Seismic Category I.

The RBVS does not perform any safety-related functions, except for boundary isolation dampers closing in the event of radiological events. Redundant dampers and controls are provided so the refueling area can be isolated even if one of the dampers or controls fail.

Rooms containing safety-related equipment have passive cooling features designed to limit the room temperature to the equipment's environmental qualification temperature.

#### ***9.4.6.4 Testing and Inspection Requirements***

Routine testing of the RBVS is conducted in accordance with normal power plant requirements for demonstrating system and component operability. Periodic surveillance testing of safety-related building isolation dampers is carried out per IEEE-338.

The Reactor Building HVAC Purge Exhaust Filter components are periodically tested in accordance with Regulatory Guide 1.140, Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants.

#### ***9.4.6.5 Instrumentation Requirements***

The RBVS is operated from the MCR. A local run/stop control switch is provided for each fan for maintenance and testing purposes. This meets the URD requirement for local control of the

RBVS. The RBVS is manually controlled, except for certain automatic operations described below:

- Reactor Building boundary isolation dampers close on receipt a high radiation signal or on a loss of AC power;
- For systems with redundant fans, the lead fan is selected manually. The standby fan automatically starts upon indication of low flow in the associated discharge duct;
- Fan operation is allowed only when the corresponding fan shutoff dampers (one upstream and one downstream) are open;
- The CLAVS return/exhaust fan auto starts after the supply fan starts and the ventilated spaces are at a positive pressure;
- Differential pressures between the ventilated spaces and the outside are transmitted to a pressure controller. The controller adjusts the CLAVS AHU supply fan's variable inlet vanes or fan speed that modulates airflow to maintain the ventilated at a positive pressure.
- A temperature controller modulates the CLAVS outside, return and exhaust air dampers when outside air temperatures are below design supply air temperatures. Damper modulation provides a mixture of outside and return air at or below design supply air temperatures to the ventilated spaces.
- The CONAVS supply fan auto starts after the exhaust fan starts and a negative pressure has been established in the ventilated spaces;
- Differential pressures between the ventilated spaces and the outside are transmitted to a pressure controller. The pressure controller adjusts the CONAVS exhaust fan's variable inlet vanes or fan speed that modulates exhaust airflow to maintain a negative pressure.
- When a recirculating AHU is started, the fan runs continuously. A room thermostat automatically modulates the chilled water supplied to the cooling coil to maintain the room temperature.
- Local thermostats automatically start the unit heaters are located in rooms served by CLAVS and CONAVS; and
- The RBVS component operating status and system parameters are monitored and indicated in the MCR and locally where required;

Indications and alarms include the following:

- Indicators for system operating parameters, including flow rates, damper position, filter pressure drop, building pressure with respect to atmospheric and temperatures; and
- Alarms for high or low conditions, including airflow rates, temperatures, filter pressure drop, building differential pressure and smoke detection.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of GDC 13.

#### 9.4.6.6 COL Information

None.

#### 9.4.6.7 References

The applicable HVAC codes and standards are shown in Table 9.4-17.

### 9.4.7 Electrical Building HVAC System

The Electrical Building HVAC System (EBVS) consists of the following subsystems:

- Electric and Electronic Rooms (EER) HVAC Subsystem (EERVS)
- Technical Support Center (TSC) HVAC Subsystem (TSCVS)
- Diesel Generators (DG) HVAC Subsystem (DGVS)

Regarding the ESBWR nonsafety-related EBVS, this subsection addresses the applicable requirements of the General Design Criteria (GDC) 2, 4, 5, 19 and 60 discussed in Standard Review Plans (SRP) 9.4.1 and 9.4.3. The ESBWR:

- Meets GDC 2 via compliance with the guidance of Regulatory Guide 1.29, Position C.2 for nonsafety-related portions. The EBVS does not perform any safety-related function. The EBVS components are designated as Seismic Category NS. The Electrical Building is nonsafety-related and Seismic Category NS.
- Meets GDC 4. While the TSC ventilation system is not specified in SRP 9.4.1, the ESBWR design is committed to providing a TSC that maintains environmental conditions in the TSC compatible with the design limits of safety-related equipment located therein.
- Meets GDC 5 for shared safety-related systems and components. The ESBWR does not share any safety-related structure, system or component with any other unit.
- Meets GDC 19. The ESBWR design is committed to providing a TSC that meets the requirements of Utility Requirement Document (URD) Section 4.6.6, which states that a TSC shall be provided that complies with the requirements of NUREG-0696. NUREG-0696 requires the TSC to supply the same level of radiological protection as that supplied to the MCR under GDC 19; however, the TSC is not specifically committed to providing a safety-related environment in full compliance with GDC 19 that defines the Control Room habitability acceptance criteria.
- Meets GDC 60 because the EER, TSC and Diesel Building HVAC Systems have no source of radioactive materials in either particulate or gaseous form. The exhaust systems have no provision for filtration or adsorption because these areas are clean.

#### 9.4.7.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The EBVS does not perform or ensure any safety-related function, and thus, has no safety design basis.

## Power Generation Design Bases

The EERVS:

- Provides conditioned air to maintain acceptable temperatures for equipment and personnel comfort and habitability;
- Provides a sufficient quantity of filtered fresh air for personnel;
- Prevents the buildup of hydrogen in the nonsafety-related battery rooms to less than 2 percent hydrogen by volume;
- Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.
- Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. Manual smoke control is provided in accordance with NFPA 804 guidelines for all plant areas by manual actions of the site fire brigade.

The TSCVS:

- Provides a controlled environment for personal comfort and safety and for the proper operation and integrity of equipment in the TSC.
- Maintains the TSC at a slightly positive pressure with respect to the adjacent rooms and outside environment to minimize the infiltration of air. The pressure hereafter called "Slightly Positive Pressure" is a range from greater than zero to +0.75" wg. The TSCVS automatically switches to the recirculation mode if smoke is detected in the outside intake air. In this case, there may be no differential pressure between the TSC and the surrounding areas.
- Detects and limits the introduction of airborne hazardous materials (radioactivity or smoke) into the TSC.
- Contains nonsafety-related filter units. The TSCVS filter units are defense-in-depth components and provide the function of filtration for the TSC during conditions of abnormal airborne radioactivity when power is available. Since RG 1.140 applies specifically to normal atmosphere cleanup, and since the filter units are not credited ESF units per RG 1.52, the Codes and Standards that dictate the testing requirements of a filtration system designed for habitability are described. The specific tested and credited filtration efficiencies meet or exceed the guidance in RG 1.140.
- Removes vitiated air from the kitchen and restrooms.
- Redundant components are included to increase the reliability, availability and maintainability of the ventilation system.

- The on-site diesel generators provide electrical power to the TSCVS in case of Loss of Preferred Power (LOPP).

The DGVS:

- Provides ventilation air to maintain acceptable temperatures within the generator rooms for equipment operation and reliability during periods of diesel generator operation.
- Provides adequate heating and ventilation for suitable environmental conditions for maintenance personnel working in the diesel generator room when the generators are not in operation.
- Provides suitable environmental conditions for equipment operation in each diesel generator electrical and electronic equipment area under the various modes of diesel generator operation.
- Prevents the accumulation of combustible vapors and dissipate their concentration in the fuel oil day tank room.

#### ***9.4.7.2 System Description***

##### **Summary Description**

Figure 9.4-12 shows the EBVS Simplified System Diagram, Table 9.4-16 lists the EBVS major equipment and Subsection 9.4.10 describes component information.

##### **EERVS**

The EERVS provides a controlled environment for the Electric Equipment Building switchgear, cable spreading, electronic and nonsafety-related battery rooms.

The EERVS consists of two independent HVAC trains. One train services the rooms where the train A electric and electronic equipment is located and the other EERVS train services the rooms where the train B electric and electronic equipment is located. Each EERVS train is a recirculation ventilation system to provide heated or cooled air to the EER. The recirculating system includes an AHU with filter, heating and cooling coils and two redundant fans. Building air is returned/exhausted by two redundant fans. Dedicated exhaust fans are provided for the battery rooms.

##### **TSCVS**

The TSCVS is a recirculating ventilation system to provide filtered conditioned air to the TSC. Two supply fans and filtration units with high efficiency particulate air (HEPA) filters and charcoal filters remove radioactive materials when required. The supply fans provide fresh air to the TSC to augment the return air to maintain the TSC under slight positive pressure. The recirculating system includes redundant air conditioning units (with air mixing plenum, filters, heating and cooling coils, and humidifier) and recirculating fans to provide conditioned air through ducts, dampers, and registers to the TSC. The exhaust system includes redundant fans to direct the air from the kitchen and toilet areas into the atmosphere.

##### **DGVS**

The normal heating and ventilating system of the DGVS serves the diesel generator area. Each diesel generator train is provided with independent ventilation and heating equipment for the

building areas serving that diesel generator train. Each normal heating and ventilating subsystem for a diesel generator train consists of one 100 percent capacity engine room AHU which ventilates the diesel generator room and the diesel fuel oil day tank room.

The supplementary ventilation system of the DGVS consists of roof-mounted exhaust fans and motor-operated air intake dampers mounted on the exterior walls of each diesel generator area. The DGVS electronic equipment area cooling system consists of an AHU with a filter, a cooling coil and the associated fan. Independent AHUs serve each of the diesel generator areas.

All the EBVS equipment is located in the Electrical Building.

## **System Operation**

### **Normal Operating Mode**

#### **EERVS**

One of the two air supply fans of each train is normally operating to supply filtered, temperature-controlled air to the Electrical Building. The second supply fan is a standby unit that starts automatically upon failure of the operating fan. One of the two air return/exhaust fans of each train is also operating. The second return/exhaust fan is a standby unit that starts automatically upon failure of the operating fan. Each fan is provided with backdraft isolation dampers. The EERVS uses 100% outside air when the outside air temperature is moderate to maintain the minimum operating cost. The EERVS modulates the intake, return and exhaust air dampers to minimize the energy used for either cooling by the chilled water system or heating by the hot water system. In extreme outside air temperature conditions (either high or low), the outside air intake dampers are at their minimum position.

A minimum exhaust air is continuously extracted from battery rooms in order to keep hydrogen concentration below 2% by volume in accordance with RG 1.128.

Temperature controllers located at the heating and cooling coils air outlet modulate the EERVS air handling heating and cooling coil operation.

#### **TSCVS**

During normal operation, outside air, filtered and conditioned in the air-handling unit is propelled by fans and supplied to the TSC. General rooms and areas exhaust air is recirculated to the AHU. Ventilation air is exhausted from the kitchen and restrooms by roof fans.

The filtration unit fans do not operate during normal operation. The TSCVS automatically transfers from its normal operation mode to its radiological mode upon detection of radioactivity at the outside air intakes. The TSCVS automatically switches to the recirculation mode if smoke is detected in the outside intake air.

#### **DGVS**

During normal operation, the diesel generators are on stand-by and electronic equipment is energized. Outside air, filtered and heated (when required) in the all-fresh-air handling unit, is propelled by a fan and supplied to general areas, electronic equipment areas and the diesel oil day tank. Exhaust air from general areas is discharged outdoors by means of automatically operated control dampers.

The electronic equipment AHU recirculates and cools the air of the electronic equipment areas to control the temperature.

The supplementary ventilation system starts automatically whenever the generator is started. The roof fans stop automatically on low temperature.

### **Smoke Removal Operating Mode**

Supply AHUs and Exhaust fans stop automatically if smoke is detected. Once the fire has been extinguished, smoke removal operation is initiated manually by starting the smoke fans from the MCR.

### **Radiological Event Operation (TSCVS only)**

Upon detection of outside air high radiation signal a filtration unit fan starts automatically and opens its normally closed isolation damper. The signal also closes the filtration unit damper and the exhaust air dampers.

#### **9.4.7.3 Safety Evaluation**

The EBVS does not perform or ensure any safety-related function.

The EBVS components are designed as Seismic Category NS.

Where a system is provided with a redundant device, failure of an operating device automatically starts the standby device to maintain continuity of ventilation.

The intake air is monitored for radioactivity and filtered (if required) prior to being introduced in the Technical Support Center. Filter efficiencies meet or exceed the guidance of RG 1.140.

#### **9.4.7.4 Testing and Inspection Requirements**

Major components are tested and inspected as separate components prior to installation and as integrated systems after installation to ensure design performance. Ductwork system airflows are measured and adjusted to meet design requirements and instruments are calibrated to the design setpoints. The systems are preoperational tested in accordance with the requirements of Chapter 14.

Periodic inspections and measurements including air flows, water flows, air and water temperatures, filter pressure drops, controls positions, are taken to verify the systems operating conditions and to ensure the integrity of the systems for normal plant operation.

The EBVS filtration components are periodically tested in accordance with ASME AG-1, Code On Nuclear Air And Gas Treatment to meet the requirements of RG 1.140.

#### **9.4.7.5 Instrumentation Requirements**

All control actuations, indicators, and alarms for normal plant operation are located in the MCR. Controls and instrumentation for the EBVS include:

- Heating and cooling temperature indicators, controls and alarms for the entering mixed air and recirculated air;
- Low and high temperature sensors and alarms for heated and cooled air supply;

- Differential pressure transmitters to monitor filters pressure drop, and alarm high filter pressure drop;
- Airflow indicators, controls, alarms and trips for each supply fan;
- Airflow failure sensor and alarm for each exhaust fan;
- Differential pressure transmitters to monitor and control positive pressure of the TSC with respect to the surrounding areas; and
- Radiation and smoke detection for the TSCVS outside air intake.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### ***9.4.7.6 COL Information***

COL applicants are responsible for the amount and location of possible sources of toxic chemicals in or near the plant and for TSC toxic gas monitoring, as required. COL applicants are responsible for verifying that TSC toxic gas protection equipment, procedures and training for TSC habitability are consistent with NUREG-0696, RG 1.78, and the intent of Generic Issue 83 as applicable. Regulatory Guide 1.78 addresses control room protection for toxic chemicals and evaluation of offsite toxic releases (including the potential for toxic releases beyond 72 hours) in order to meet the requirements of TMI Action Plan Item III.D.3.4 and GDC 19.

#### ***9.4.7.7 References***

The applicable HVAC codes and standards are shown in Table 9.4-17.

#### **9.4.8 Drywell Cooling System**

Regarding the ESBWR nonsafety-related Drywell Cooling System (DCS), this subsection addresses the applicable requirements of the General Design Criteria (GDC) 2, 5 and 60, discussed in Standard Review Plan (SRP) 9.4.3. The ESBWR:

- Meets GDC 2 via compliance with the guidance of Regulatory Guide 1.29, Position C.2 for nonsafety-related portions. The DWVS is classified as nonsafety-related and Seismic Category II.
- Meets GDC 5 for shared systems and components important to safety. The ESBWR does not share any safety-related structure, system or component with any other unit.
- Meets GDC 60 by suitably controlling the release of gaseous radioactive effluents to the environment. During normal operation, the DWVS re-circulates air with no connection to any HVAC system outside primary containment. Only during DW purge operations, is the primary containment air connected with the CONAVS subsystem of RBV. During DW purge operations, the containment purge fan can be used to discharge primary containment air to the CONAVS subsystem. The CONAVS system has RB HVAC Purge Exhaust Filter Units that are designed, tested and maintained in accordance with Regulatory Guide 1.140.

### **9.4.8.1 Design Basis**

#### **Safety (10 CFR 50.2) Design Bases**

The Drywell Cooling System has no safety-related function, and thus, has no safety design basis.

#### **Power Generation Design Bases**

The Drywell Cooling System (DCS) is classified as a nonsafety-related and Seismic Category II system. The DCS performs the following functions during stable and transient operating conditions through the entire operating range, from startup to full load condition to refueling:

- Maintain temperature in the upper and the lower drywell spaces within specified limits during normal operation
- Accelerate drywell cooldown during Loss of Preferred Power (LOPP) the period from hot shutdown to cold shutdown
- Aid in complete purging of nitrogen from the drywell during shutdown
- Maintain a habitable environment for plant personnel during plant shutdowns for refueling and maintenance
- Limit drywell temperature during (LOPP)

Tables 9.4-12, 9.4-13 and 9.4-14 contain the DCS design parameters.

### **9.4.8.2 System Description**

#### **Summary Description**

The DCS maintains the thermal environment within the drywell to specified conditions during normal reactor operation, hot standby and refueling using Fan Cooling Units (FCUs). The cooling medium of the FCUs is Chilled Water System (CWS) water. There are separate FCUs for the upper and the lower drywell regions.

Figure 9.4-13 shows a simplified DCS diagram, subsection 9.2.7 describes the CWS and subsection 9.4.10 describes component information.

#### **Detailed System Description**

The DCS is a closed loop recirculating air/nitrogen cooling system with no outside air/nitrogen introduced into the system except during refueling. The system uses direct-drive type FCUs to deliver cooled air/nitrogen to various areas of the upper and the lower drywell. Ducts distribute the cooled, recirculated air/nitrogen through diffusers and nozzles.

The drywell heat loads are transferred to the Nuclear Island subsystem of the (CWS) circulating through the cooling coils of the FCUs. The DCS consists of four FCUs, two located in the upper drywell and two in the lower drywell.

Each upper drywell FCU has a cooling capacity of 50% of the upper drywell design heat load during normal plant operating conditions. Both FCUs are normally operating. Each FCU comprises a cooling coil and two fans downstream of the coil. Nuclear Island subsystem of CWS train A supplies one FCU, and Nuclear Island subsystem of CWS train B supplies the other. One of the fans operates while the other is on standby status. The fan on standby

automatically starts upon loss of the lead fan. Cooled air/nitrogen leaving the FCUs enters a common plenum and is distributed to the various zones in the upper drywell through distribution ducts. Return ducts are not provided. The FCUs draw air/nitrogen directly from the upper drywell. Each FCU is equipped with a condensate collection pan.

Each lower drywell FCU has a cooling capacity of 50% of the lower drywell design heat load. Each FCU comprises a cooling coil and two fans downstream of the coil. One of the fans operates while the other is on standby status. The fan on standby automatically starts upon loss of the lead fan. Nuclear Island subsystem of CWS train A supplies one FCU, while Nuclear Island of subsystem CWS train B supplies the other. Cooled air/nitrogen is supplied below the RPV and in the RPV support area through supply ducts. Return ducts are not provided. The FCUs draw air/nitrogen directly from the lower drywell.

Each FCU has a condensate collection pan. The condensate collected from the FCUs in the upper and the lower drywell is piped to a Leak Detection and Isolation System (LD&IS) flowmeter to measure the condensation rate contribution to unidentified leakage.

The piping for train A and train B of the Nuclear Island subsystem of CWS independently penetrate the containment. The cooling coils of one FCU in the upper drywell and one FCU in the lower drywell are piped in parallel to Nuclear Island subsystem of CWS train A and the remaining two are piped in parallel to Nuclear Island subsystem of CWS train B. The system is designed so both FCUs in the upper drywell and both FCUs in the lower drywell are always operating during normal plant operation assuming the loss of a single electrical group or failure of any single FCU motor or fan. Upon failure of one FCU, the two fans of the remaining FCU are in service. One FCU with two fans in operation maintains the drywell temperature below the maximum allowed.

Table 9.4-13 provides a description of the upper DCS and lower DCS FCUs. Table 9.4-14 provides a summary of the drywell heat loads.

The FCU fans and fan motors are designed to be operable during containment integrated leak testing (ILRT).

### **System Operation**

During normal plant operating condition, two FCUs in the upper drywell and two FCUs in the lower drywell are continuously operating to maintain the ambient conditions described in Table 9.4-12.

During plant refueling conditions, one FCU in the upper drywell and one FCU in the lower drywell continuously operate with two fans in service to maintain a habitable environment in the drywell for maintenance activities.

nonsafety-related on-site diesel generators power the FCUs during a LOPP as long as there is no loss-of-coolant-accident (LOCA) signal.

#### **9.4.8.3 Safety Evaluation**

The DCS does not perform any plant safety-related function. Failure of the system does not compromise any safety-related system or component nor does it prevent safe shutdown of the plant.

#### ***9.4.8.4 Testing and Inspection Requirements***

The FCU coils are tested for pressure integrity in conjunction with the Nuclear Island subsystem of CWS after the installation is completed. Hydrostatic testing of piping systems is performed at least 1.5 times the design pressure, for a minimum of 30 minutes with no indicated leakage. Pneumatic testing may be substituted for hydrostatic testing in accordance with applicable codes.

The supply ducts have test connections for verifying calibration of operating controls.

#### ***9.4.8.5 Instrumentation Requirements***

Each FCU motor can be controlled manually from the MCR. Indicating lights in the MCR display the status of each unit. Failure of an FCU and a subsequent temperature rise in the discharge stream activates an alarm in the MCR.

A flow switch is provided in each fan discharge duct at the downstream side of its gravity backdraft damper. The standby fan automatically starts upon failure of a fan. Failure of a fan activates a low flow alarm in the MCR.

Preset flow reducing devices are provided in the supply ducts so that, regardless of which FCUs are operating, the air/nitrogen distribution requirements are met. Gravity-type backdraft dampers prevent reverse flow through a unit that is not operating.

There are temperature instruments for measuring the temperatures of the supply and return air/nitrogen to each FCU.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### ***9.4.8.6 COL Unit-Specific Information***

None

#### ***9.4.8.7 References***

The applicable HVAC codes and standards are shown in Table 9.4-17.

### **9.4.9 Containment Inerting System**

#### ***9.4.9.1 Design Bases***

##### **Safety (10 CFR 50.2) Design Bases**

The Containment Inerting System (CIS) does not perform any safety-related function. Therefore, the CIS has no safety design bases other than provision for safety-related containment penetrations and isolation valves, as described in Subsection 6.2.4.

##### **Power Generation Design Bases**

- The CIS is designed to establish an inert atmosphere (i.e., less than 4% oxygen by volume) throughout the containment in less than 4 hours and less than 2% oxygen by volume in the next 8 hours following an outage.

- The CIS is designed to maintain the containment oxygen concentration below the maximum permissible limit (3%) during normal, abnormal, and accident conditions to assure an inert atmosphere.
- The CIS is designed to maintain a positive pressure in the primary containment during normal, abnormal, and accident conditions to prevent air (oxygen) in-leakage into the inerted spaces from the Reactor Building. The CIS nitrogen gas makeup supply line is designed for the normal daily operating capacity to maintain approximately 4.8 kPaG (0.7 psig) positive pressure within the containment. The system has the capability to replenish containment atmosphere leakage at a design value of 0.5% per day based on containment operating pressure.
- The inerting auxiliary steam vaporizer is sized to provide at least 2.5 times the containment (wetwell and drywell) free volume of nitrogen within the allotted 4 hours. The temperature of the injected nitrogen is within the range of 10°C (50°F) to 65°C (150°F).
- The CIS is designed to permit de-inerting the containment for safe operator access without breathing apparatus in less than 12 hours.
- The CIS is designed to perform continuous containment leakage rate monitoring and detect gross leakage of containment atmosphere during normal reactor operation.
- The CIS is also designed to release containment pressure before uncontrolled containment failure could occur. (See Subsection 6.2.5)

#### ***9.4.9.2 System Description***

##### **Summary Description**

The CIS establishes and maintains an inert atmosphere within the primary containment during all plant operating modes except during plant shutdown for refueling or maintenance and during limited periods of time to permit access for inspection during reactor low power operation. The purpose of the system is to provide an inert containment atmosphere ( $\leq 3\%$  oxygen) during normal operation to minimize hydrogen burn inside the containment.

A simplified CIS system diagram is shown in Figure 9.4-14 and subsection 9.4.10 describes component information.

##### **Detailed System Description**

The CIS consists of a pressurized liquid nitrogen storage tank, a steam-heated main vaporizer for large nitrogen flow, electric heater for vaporizing makeup flow, two injection lines, an exhaust line, a bleed line and associated valves, controls and instrumentation. All CIS components are located inside the Reactor Building except the liquid nitrogen storage tank and the steam-heated main vaporizer that are located in the yard.

The first of the injection lines is used only for makeup. It includes an electric heater to vaporize the nitrogen and to regulate the nitrogen temperature to acceptable injection temperatures. Remotely operated valves, together with a control valve, enable the operator to accomplish low rates of nitrogen injection into the drywell and suppression pool airspace.

The second injection line is used for the inerting function where larger flow rates of nitrogen are required. This line provides the flow path for vaporized nitrogen at an appropriate temperature from the steam-heated main vaporizer to be injected into the containment through remotely operated valves and a control valve to injection points common with the makeup supply. The inerting and makeup lines converge to common injection points in the upper drywell and suppression pool airspace.

The CIS includes an exhaust line from the lower drywell on the opposite side of containment from the injection points. The discharge line connects to the Reactor Building HVAC system exhaust before being diverted to the plant stack. The Reactor Building HVAC system is discussed in Subsection 9.4.6.

A small bleed line bypassing the main exhaust line is also provided for manual pressure control of the containment during normal reactor operation.

Redundant containment isolation valves provided in the inerting, makeup, exhaust and bleed lines close automatically upon receipt of an isolation signal from the Leak Detection and Isolation System (LD&IS). Discussion of these signals is provided in Subsection 7.3.3.

Upstream of the pressure-reducing valve in the makeup line, a small branch line is provided and connected to the High Pressure Nitrogen Supply System (HPNSS) (see Subsection 9.3.8).

### **System Operation**

During plant startup, large flow rates of nitrogen from the liquid nitrogen storage tank are vaporized by the steam-heated vaporizer and injected into the drywell and the wetwell. The exhaust line is kept open to displace containment resident atmosphere with nitrogen. Once the desired concentration of oxygen is reached, the exhaust line is closed. When the required inerted containment operating pressure is attained, the nitrogen supply shutoff valve and the inerting isolation valves are closed to terminate the inerting process. The system is capable of inerting the containment to  $\leq 4\%$  oxygen by volume within four hours. The CIS is capable of establishing a more completely inert atmosphere, equal to or less than 2% in containment with the next eight hours after reaching 4% conditions.

Containment pressure is maintained automatically after manually aligning the nitrogen makeup subsystem. Low flows of liquid nitrogen are vaporized and heated to the desired temperature and injected into the drywell and the wetwell to makeup for the nitrogen out-leakage. The containment atmosphere is kept constant at a positive pressure relative to the Reactor Building to preclude air (oxygen) in-leakage. In response to a change in containment pressure, the pressure control valve modulates (opens or closes) to provide nitrogen makeup and thereby maintaining the containment pressure. The flow integrator monitors the nitrogen makeup to compensate for leakage during normal containment pressure control and to the HPNSS. Large makeup flow indicates gross or excessive leakage and is annunciated in the MCR. Manual venting through the exhaust bleed line controls increases in containment pressure greater than the normal operating range.

During plant shutdown, the containment atmosphere is de-inerted to allow safe personnel access inside the containment. Breathable air from the Reactor Building HVAC system is injected into the drywell and wetwell air space through the inerting injection line. The incoming air displaces containment gases (mostly nitrogen) into the exhaust line. The Reactor Building HVAC system

exhaust fans, filters, and radiation detectors remove the vented gases and then they are diverted to the plant stack. The CIS is capable of reaching a volumetric oxygen concentration of  $\geq 19\%$  within 12 hours after de-inerting begins.

#### **9.4.9.3 Safety Evaluation**

The CIS has no safety-related function except the containment isolation function, which is discussed in Subsection 6.2.4. Failure of the CIS does not compromise any safety-related system or component, nor does it prevent a safe shutdown of the plant.

#### **9.4.9.4 Testing and Inspection Requirements**

CIS containment penetrations, including isolation valves, undergo routine in-service inspection and testing as required by ASME Code, Section XI.

Permanently installed instrumentation inside containment is maintained, tested, and calibrated during every refueling outage.

#### **9.4.9.5 Instrumentation Requirements**

CIS instrumentation requirements are discussed in Subsection 7.7.7.

This instrumentation conforms to GDC 13. Refer to Subsection 3.1.2 for a general discussion of the GDC.

#### **9.4.9.6 COL Information**

None.

#### **9.4.9.7 References**

The applicable HVAC codes and standards are shown in Table 9.4-17.

### **9.4.10 HVAC Component Information**

#### **9.4.10.1 Filtration**

##### **Low, Medium and High Efficiency Filters**

The various building filtration level is specified within each specific subsection. Filters meet the applicable efficiency rating as stated below.

ASHRAE (American Society of Heating, Refrigerating and Air Conditioning Engineers) Standard 52.2 establishes a filter's MERV (Minimum Efficiency Reporting Value) and Standard 52.1 establishes a filter's Average Atmospheric Dust Spot Efficiency. The following ASHRAE filter classifications are MERV specified (with Dust Spot Efficiency in parenthesis) below:

- Low Efficiency:       MERV 1-4 (Less than 20%)
- Medium Efficiency:   MERV 5-12 (As least 20% but less than 80%)
- High Efficiency:     MERV 13-16 (Greater than 80%)

**HEPA Filters**

HEPA (High Efficiency Particulate Filter) are specified for various building filtration systems. Filters meet the applicable efficiency rating as stated below.

Filters with efficiency greater than MERV 16 by ASHRAE Standard 52.2 (MERV 17-20) are usually rated by the DOP (dioctylphthalate) test method. This test is based on the ability of a filter to remove an aerosol consisting of 0.3 micrometer (micron) particles of a test challenge. HEPA filters are extended-medium dry-type filters in a rigid frame, having minimum particle-collection efficiency of 99.97% on 0.3 micron particles which meets ASME AG-1, Code on Nuclear Air and Gas Treatment.

HEPA filters are constructed, qualified and testing per UL-586, High Efficiency, Particulate, Air Filter Units.

**Carbon Adsorbers**

Constructed and tested per ASME AG-1 Section FE and RG 1.140. Single assembly, welded construction conforming to IE Bulletin 80-03. Carbon for adsorption filters is tested per ASTM D3803 to meet the requirements of RG 1.140.

**General**

Guidance for the design, installation, operation, and maintenance of air filtration systems is per the National Air Filtration Association Manual (see Table 9.4-17).

**Air Handling Units**

Each air handling unit consists of an inlet area, filters (as specified by the system), heating and/or cooling coils (as required) and the respective fans (supply or exhaust). The Air-Cleaning Units and Components are designed in accordance with ASME N509-2002, Nuclear Plants Air-Cleaning Units and Components and ASME/ANSI AG-1-2003.

**9.4.10.2 Supply and Exhaust Fans**

The various building ventilation systems are provided with supply and exhaust fans. These fans are either centrifugal or axial fans depending on the suitability to the specific system. The fans are designed, manufactured and supplied in accordance with the standards of AMCA (Air Movement and Control Association International). Fans in various areas are equipped with Variable Frequency / Speed Drive mechanisms to control airflows for the specific system application. See Table 9.4-17 for applicable codes and standards.

**9.4.10.3 Heating Coils**

Various Air Handling Units are equipped with either electrical or hot water heating coils. Electric coils are designed and supplied to the requirements of UL-1995, Heating and Cooling Equipment. Hot water heating coils are designed and supplied per ARI 410, Force-circulation Air-cooling and Air-heating Coils.

**9.4.10.4 Cooling Coils**

The Tubular fin type cooling coils are designed, constructed and installed in accordance with ASHRAE 33, Methods of Testing Forced and ANSI/ARI 410 and UL-1995.

#### **9.4.10.5 Dampers**

##### **Isolation, Control (Vortex), Backdraft and Balancing Dampers**

Dampers are designed and specified in accordance with AMCA 502-01, Damper Application Manual. These dampers include isolation, balance, backdraft, and modulating control types.

##### **Fire / Smoke Dampers**

The combination Fire and Smoke Dampers in the various systems meet the design, leakage testing, and installation requirements of UL-555(S) Fire Dampers (Smoke Dampers), and AMCA 503, Fire, Ceiling (Radiation), Smoke and Fire / Smoke Dampers Application Manual.

#### **9.4.10.6 Ductwork and Accessories**

Ductwork, duct supports and accessories are constructed of galvanized steel. The ductwork including supports is constructed to meet the design and construction requirements of SMACNA (Sheet Metal & Air Conditioning Contractors' National Association). The support guidance of AISI-2001 and 2004 Supplement, American Iron and Steel Institute Specification for the Design of Cold Formed Steel Structural Members is also employed in the design. The structural requirements for HVAC Ducts and HVAC Duct Supports are also specified in Subsection 3.8.4.1.7. See Table 9.4-17 for list of applicable HVAC codes and standards.

**Table 9.4-1**  
**Design Parameters for the CBVS**

<b>CRHAVS and CBGAVS</b>	
Operating periods:	Normal plant operation, plant startup, and plant shutdown
Outside Air Design Conditions:	
For CRHAVS and EFUs (Limiting values)	Summer:      46.1°C (115°F) Dry Bulb 26.7°C (80°F) Wet Bulb 29.4°C (85°F) wb (non-coincident) Winter:        -40.0°C (-40°F) Dry bulb
For CBGAVS: (1% Exceedance values)	Summer:      37.8°C db (100°F) 26.1°C wb (79°F) (coincident), 27.8°C (82°F) wb (non-coincident) Winter:        -23.3°C (-10°F) Dry bulb
Inside Design temperatures and humidity:	
CRHA (normal operation)	22.8°C (73°F) to 25.6°C (78°F) and 25% to 60% relative humidity (RH)
CRHA (SBO)	Maximum 8.3°C (15°F) rise above normal operating temperature, RH not controlled
DCIS rooms/miscellaneous areas	18.3°C (65°F) to 25.6°C (78°F), RH not controlled
1E DCIS rooms (SBO)	50°C (122°F) maximum
HVAC equipment room:	10°C (50°F) to 40°C (104°F), RH not controlled
Pressurization:	CBGAVS > atmospheric pressure CRHAVS > 31 Pa (1/8" wg) positive differential
CBGAVS	18.3°C (65°F) to 25.6°C (78°F), RH not controlled
CRHAVS Breathing air supply capacity:	9.5 l/s (20 cfm) per person for up to 21 persons (200 l/s or 424 cfm total) for 72 hours (Ref. 9.4.1-1)

**Table 9.4-2  
Major Equipment for CBVS**

<b>CRHAVS</b>		
Air Handling Units	Quantity:	2 - 100% capacity each
	Capacity:	Flow – 5,250 l/s per unit
		Cooling ≤ 100,000 watts (28.4 tons)
		Heating - Smoke Removal Mode: approximately 330,000 watts at Limiting Values approximately 206,000 watts at 1% Exceedance Heating - Normal Mode: approximately 18,000 watts at Limiting Values approximately 14,000 watts at 1% Exceedance
Type	Precision Floor Mounted Downflow containing medium efficiency filters, chilled water cooling coil, electric heating coil, humidifier and centrifugal fans	
<b>CRHAVS EFU</b>		
Emergency Filter Unit	Quantity:	2 - 100% capacity each
	Capacity:	Flow – 200 l/s per unit
	Type:	Metal housing containing medium efficiency filter, HEPA filter, carbon filter, and post filter
Efficiency :	99% credited removal for HEPA and 99% credited removal for carbon filter per RG 1.52	
EFU fans	Quantity:	1 - 100% capacity each per EFU
	Capacity:	Flow – 200 l/s per unit
Type	Centrifugal, ≤ 1.1 kW (1.5 hp)	

**Table 9.4-2  
Major Equipment for CBVS (continued)**

CRHAVS (continued)		
Restroom Exhaust fan	Quantity:	1 - 100% capacity each
	Capacity:	Flow - 50 l/s per unit
	Type	In-line centrifugal
CRHA Isolation Dampers	Quantity	8 Total, redundant supply, smoke intake, smoke exhaust, and restroom exhaust isolation dampers
	Capacity	Varies
	Type	Safety-related Bubble tight, Air operated, fail-closed
<b>CBGAVS - SET A</b>		
Supply Air Handling Units	Quantity:	2 - 100% capacity each (per set)
		Cooling: approximately 239,000 watts (67.5 tons)
		Heating: approximately 405,000 watts (1% exceedance-full flow)
	Type	Sheet Metal housing containing high and low efficiency filters, chilled water cooling coil, heating coil and centrifugal fan

**Table 9.4-2**  
**Major Equipment for CBVS (continued)**

AHU Supply fans	Quantity:	1 - 100% capacity each per AHU
	Capacity:	Flow – 10,300 l/s per unit
	Type	Axial or Centrifugal, with inlet vanes or variable speed drives, approximately 30 kW (40 hp)
Return/Exhaust fan	Quantity:	1 - 100% capacity each per AHU
	Capacity:	Flow – 9,700 l/s per unit
	Type	Axial or Centrifugal with inlet vanes or variable speed drives, approximately 11.2 kW (15 hp)
<b>CBGAVS - SET B</b>		
Supply Air Handling Units	Quantity:	2 - 100% capacity each (per set)
		Cooling: approximately 223,000 watts (63.6 tons)
		Heating: approximately 371,000 watts (1% exceedance)
	Type	Sheet Metal housing containing high and low efficiency filters, chilled water cooling coil, heating coil and centrifugal fan
AHU Supply fans	Quantity:	1 - 100% capacity each per AHU
	Capacity:	Flow – 9,450 l/s per unit
	Type	Axial or Centrifugal, with inlet vanes or variable speed drives, approximately 30kW (40 hp)
Return/Exhaust fan	Quantity:	1 - 100% capacity each per AHU
	Capacity:	Flow – 8,950 l/s per unit
	Type	Axial or Centrifugal with inlet vanes or variable Speed drives, approximately 11.2 kW (15 hp)

**Table 9.4-3  
Design Parameters for FBVS**

<b>FBGAVS</b>	
Operating periods:	Normal plant operation, plant startup, and plant shutdown
Temperatures:	
Outdoor Design:	Summer:     37.8°C (100°F) Dry Bulb 26.1°C (79°F) Wet Bulb (coincident) 27.8°C (82°F) wb (non- coincident) Winter:       -23.3°C (-10°F) Dry bulb
Indoor Design:	
Occupied Areas	23°C to 26°C (73°F to 78°F)
Occupied areas with moderate work and areas with electronic equipment:	18°C to 29°C (65°F to 85°F)
Areas with frequent inspection/maint. (without sensitive electronic equipment):	10°C to 40°C (50°F to 104°F)
HVAC equipment room:	10°C to 40°C (50°F to 104°F)
Pressurization:	-62 Pa (-1/4 in. wg) minimum relative to surrounding areas
<b>FBFPVS</b>	
Operating periods:	Same as FBGAVS
Temperatures:	Same as FBGAVS
Areas with infrequent inspections or maintenance activities (without sensitive equipment):	10°C to 40°C (50°F to 104°F)
Spent Fuel Pool area wet bulb global temperature:	Below 27°C (80°F)
Pressurization:	-62 Pa (-1/4 in. wg) minimum relative to surrounding areas

**Table 9.4-4  
Major Equipment for FBGAVS**

Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 12,300 l/s (26,060 cfm) per unit
		Filtration – Low and medium efficiency
		Cooling – approximately 756,500 watts (2,582,400 Btu/hr) Heating – approximately 579,700 watts (1,978,656 Btu/hr)
AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 12,300 l/s (26,060 cfm) per unit
	Type:	Axial or Centrifugal with variable speed drive, approximately 37 kW (50 hp)
Exhaust fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 13,550 l/s (28,710 cfm) per unit
	Type:	Axial or Centrifugal with variable speed drive, approximately 11.2 kW (15 hp)
Building Supply Isolation Damper	Quantity:	2 (Safety-Related, Seismic Category I)
	Capacity:	12,300 l/s (26,060 cfm)
	Type:	Parallel Blade
	ASME AG-1 Seat Leakage Class	I - Low Leakage
	Actuator Type	Pneumatic, fail close
Building Exhaust Isolation Damper	Quantity:	2 (Safety-Related, Seismic Category I)
	Type:	Parallel Blade
	Capacity:	13,550 l/s (28,710 cfm)
	ASME AG-1 Seat Leakage Class	I – Low Leakage
	Actuator Type	Pneumatic, fail close
FMCRD Recirculation AHU	Quantity:	1 - 100% capacity
	Capacity:	Flow – 600 l/s (1,272 cfm) per unit
		Filtration – low efficiency
AHU Supply fan	Quantity:	1 - 100% capacity
	Capacity:	Flow – 600 l/s (1,272 cfm) per unit
	Type:	Centrifugal, direct drive

**Table 9.4-5  
Major Equipment for FBFVPS**

Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 14,350 l/s (30,405 cfm) per unit
		Filtration – Low and medium efficiency
		Cooling – approximately 882,300 watts (3,012,000 Btu/hr)
		Heating – approximately 676,000 watts (2,307,737 Btu/hr)
AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 14,350 l/s (30,405 cfm) per unit
	Type:	Centrifugal or Axialwith variable speed drive, approximately 45 kW (60 hp)
Exhaust fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 15,790 l/s (33,457 cfm) per unit
	Type:	Centrifugal or Axialwith variable speed drive, approximately 11.2 kW (15 hp)
Building Supply Isolation Damper	Quantity:	2 (Safety-Related, Seismic Category I)
	Capacity:	14,350 l/s (30,405 cfm)
	ASME AG-1 Seat Leakage Class	I - Low Leakage
	Actuator Type	Pneumatic, fail close
Building Exhaust Isolation Damper	Quantity:	2 (Safety-Related, Seismic Category I)
	Capacity:	15,800 l/s (32,843cfm)
	ASME AG-1 Seat Leakage Class	I - Low Leakage
	Actuator Type	Pneumatic, fail close

**Table 9.4-6**  
**RWVS Design Conditions**

Control Room and areas with electronic equipment:	23°C to 26°C (73°F to 78°F) and relative humidity between 35-50%
Corridors and general access areas:	10°C to 40°C (50°F to 104°F)
Equipment cells:	10°C to 50°C (50°F to 122°F)
Tank rooms:	10°C to 50°C (50°F to 122°F)
HVAC equipment room:	10°C to 40°C (50°F to 104°F)

Design room differential pressure

Control Room areas and electronic equipment room areas: +31Pa

General areas: -31Pa

**Table 9.4-7**

**Major Equipment for the RWVS**

RWGAVS Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Normal flow – Established during detailed design
	Filtration	Medium Efficiency
		Cooling – Established during detailed design
		Heating – Established during detailed design
RWGAVS AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Normal flow - Established during detailed design
		Pressure - Established during detailed design
Type:	Centrifugal or Axial with variable speed drive	
RWGAVS Return/exhaust fans	Quantity:	3 - 50% capacity (two running and one standby)
	Capacity:	Flow – Established during detailed design
	Filtration	Medium Efficiency & HEPA (99.97% DOP)
Type:	Centrifugal or Axial with variable speed drive	
RWCRVS Air Handler	Quantity:	2 - 100% capacity
	Capacity:	Established during detailed design
	Filtration	Low Efficiency
	Type:	Centrifugal or Axial, approximately 18.6 kW (25 hp)
RWCRVS Smoke Exhaust Fan	Quantity:	1 - 100% capacity
	Capacity:	Flow – Established during detailed design
		Pressure - Established during detailed design
	Type:	Centrifugal or Axial

**Table 9.4-8**  
**Design Parameters for RBVS**

<b>Description</b>	<b>Requirements</b>
<b>CONAVS and REPAVS</b>	
Operation periods:	Normal plant operation, plant startup, and plant shutdown.
Temperatures: Outdoor Design: Summer:	37.8°C db (100°F), 26.1°C wb (79°F) (coincident) 27.8°C (82°F) wet bulb (non-coincident)
Winter:	-23.3°C db (-10°F)
Indoor Design: Occupied areas:	23°C to 26°C (73°F to 78°F)
Occupied areas with moderate work and areas with electronic equipment:	18°C to 29°C (65°F to 85°F)
Areas with frequent inspections/maintenance (without sensitive electronic equipment):	10°C to 40°C (50°F to 104°F)
HVAC equipment room:	10°C to 40°C (50°F to 104°F)
Pressurization:	-62 Pa (-0.25 in. wg) minimum relative to surrounding areas
<b>CLAVS</b>	
Operation periods:	Normal plant operation, plant startup, and plant shutdown.
Temperatures: Outdoor Design: Summer:	37.8°C db (100°F), 26.1°C wb (79°F) (coincident) 27.8°C (82°F) wet bulb (non-coincident)
Winter:	-23.3°C db (-10°F)
Indoor Design: Occupied areas:	23°C to 26°C (73°F to 78°F)
Occupied areas with moderate work and areas with electronic equipment:	18°C to 29°C (65°F to 85°F)
Areas with frequent inspections/maintenance activities (without sensitive electronic equipment):	10°C to 40°C (50°F to 104°F)
Pressurization (battery rooms):	Slightly negative relative to surrounding areas
Pressurization (other CLAVS rooms):	Slightly positive relative to surrounding areas

**Table 9.4-9  
Major Equipment for CLAVS**

Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Normal flow – 27,250 l/s (57,739 cfm) per unit
		Filtration - medium efficiency
		Cooling – approximately 686,400 watts (2,344,203 Btu/hr)
		Heating – approximately 100,500 watts (343,406 Btu/hr)
AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Normal flow - 27,250 l/s (57,739 cfm) per fan
Type:	Centrifugal or Axial with variable inlet vanes or Variable Speed Drive, approximately 75 kW (100 hp)	
Return/exhaust fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 24,800 l/s (52,548 cfm) per fan
Type:	Centrifugal or Axial with variable inlet vanes or Variable Speed Drive, approximately 30 kW (40 hp)	
Smoke exhaust fans	Quantity:	2 - 100% capacity (both standby)
	Capacity:	Flow – 17,600 l/s (37,292 cfm) per fan
Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 18.6 kW (25 hp)	
Battery Room exhaust fan	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 2,050 l/s (4,345 cfm) per fan
Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 2.2 kW (3 hp)	

**Table 9.4-10**  
**Major Equipment for REPAVS**

Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow – 29,150 l/s (61,765 cfm) per unit
		Filtration –Medium efficiency
		Cooling – approximately 1,605,800 watts (5,480,749 Btu/hr)
		Heating – approximately 1,193,900 watts (4,074,968 Btu/hr)
AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Flow - 29,150 l/s (61,765 cfm) per unit
	Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 110 kW (150 hp)
Exhaust fans	Quantity:	2 - 100% capacity
	Capacity:	Flow – 32,000 l/s (67,910 cfm) per fan
	Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 37 kW (50 hp)
Safety-related Building Isolation Damper	Quantity:	4 (2 redundant dampers for each supply and exhaust duct)
	Type:	Parallel Blade
	ASME AG-1 Seat Leakage Class	I – Low Leakage
	Actuator Type	Pneumatic, fail close

**Table 9.4-11  
Major Equipment for CONAVS**

Supply air handling units	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Max flow – 24,250 l/s (51,385 cfm) per unit
		Filtration - medium efficiency
		Cooling - approximately 1,338,892 watts (4,569,638 Btu/hr)
	Heating - approximately 993,256 watts (3,389,983 Btu/hr)	
AHU Supply fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Max flow - 24,250 l/s (51,385 cfm) per fan
	Type:	Centrifugal, or Axial with Variable Speed Drive or variable inlet vanes approximately 93 kW (125 hp)
Exhaust fans	Quantity:	2 - 100% capacity (one running and one standby)
	Capacity:	Max flow – 26,050 l/s (55,200 cfm) per fan
	Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 30 kW (40 hp)
Primary containment purge exhaust fan	Quantity:	1 - 100% capacity (one running and one standby)
	Capacity:	Flow -6,100 l/s (12,925 cfm) per fan
	Type:	Centrifugal or Axial with Variable Speed Drive or variable inlet vanes, approximately 7.5 kW (10 hp)
Safety-related Building Isolation Damper	Quantity:	4 (2 redundant dampers for each supply and exhaust duct)
	Type:	Parallel Blade
	ASME AG-1 Seat Leakage Class	I – Low Leakage
	Actuator Type	Pneumatic, fail close

**Table 9.4-11  
Major Equipment for CONAVS (continued)**

Main Steam Tunnel Recirculation AHU	Quantity:	2- 100% capacity (used when cooling is necessary)
	Capacity:	Flow - 9,000 l/s (19,070 cfm) per unit
		Filtration – low efficiency
		Cooling - approximately 312,000 watts (1,064,856 Btu/hr)
AHU Supply Fan	Quantity:	1- 100% capacity
	Capacity:	Flow - 9,000 l/s (19,070 cfm) per unit
	Type:	Centrifugal or Axial with Variable Speed Drive, approximately 11.2 kW (15 hp)
Refueling Machine Control Room Recirculation AHU	Quantity:	1- 100% capacity (used when cooling is necessary)
	Capacity:	Flow - 850 l/s (1800 cfm) per unit
		Filtration – low efficiency
		Cooling - approximately 10,550 watts (36,000 Btu/hr; nominal 3 tons)
	Type:	Centrifugal or Axial, approximately 0.75 kW (1hp)
Purge exhaust filter unit	Quantity:	2 - 100% capacity (used when purging and when temporarily filtering exhaust from other areas)
	Capacity:	Flow – 4,800 l/s (10,170 cfm)
	Type:	Medium efficiency filter, HEPA filter (99% credited), Carbon filter (99%), and post-filter (95% DOP minimum)
Purge exhaust filter unit exhaust fans	Quantity:	2 - 100% capacity (used when purging and when temporarily filtering exhaust from other areas)
	Capacity:	Flow - 4,800 l/s (10,170 cfm) per fan
	Type:	Centrifugal or Axial with Variable Speed Drive or with inlet vanes, approximately 22.4 kW (30 hp)

Table 9.4-12

**Drywell Cooling System Design Parameters**

Specifically, the DCS is designed to maintain the following conditions in the upper and lower drywell during normal and plant refueling modes of operation:	
Normal Plant Operation:	
Average dry bulb temperature	≤57°C (≤135°F)
Maximum temperature of ambient atmosphere in each drywell zone	66°C (150.8°F)
Plant Refueling:	
Average dry bulb temperature	25°C (77°F)

**Table 9.4-13**  
**Drywell Cooling System Fan Cooling Units**

Upper Drywell FCUs *	
Number of fans per FCU	2
Motor and Fan	Directly coupled
Number of coils per FCU	1 or more
Cooling coil type	Plate Fin (4 fins/cm max) (11 fins/inch max.)
Air inlet temperature	57°C (135°F)
Air outlet relative humidity	90 to 100%
Water inlet temperature	7°C (45°F)
Air flow rate (total)	114,410 m <sup>3</sup> /hr (67,400 cfm)
Water flow rate	45 m <sup>3</sup> /hr (198.1 gpm)
Cooling capacity (each FCU)	1,408 kW (4.8 x 10 <sup>6</sup> Btu/hr)
Fan type	Vaneaxial
Lower Drywell FCUs	
Number of fans per FCU	2
Motor and Fan	Directly coupled
Number of coils per FCU	1
Cooling coil type	Plate Fin (4 fins/cm max.) (11 fins/inch max.)
Air inlet temperature	57°C (135°F)
Air outlet relative humidity	90 to 100%
Water inlet temperature	7°C (45°F)
Air flow rate (total)	31,195 m <sup>3</sup> /hr (18,360 cfm)
Water flow rate	16.2 m <sup>3</sup> /hr (71.3 gpm)
Cooling capacity (each FCU)	370 kW (1.26 x 10 <sup>6</sup> Btu/hr)
Fan type	Vaneaxial

\* Design case only one FCU in operation with both FCU fans operating.

**Table 9.4-14**  
**Drywell Cooling System Heat Loads**

Upper Drywell		
	Normal kW (Btu/hr)	Refueling kW (Btu/hr)
Sensible Load	1,010.4 (3.45 x 10 <sup>6</sup> )	293 (1 x 10 <sup>6</sup> )
Latent Load	214.1 (0.73 x 10 <sup>6</sup> )	12 (0.04 x 10 <sup>6</sup> )
Total Load	1,224.5 (4.18 x 10 <sup>6</sup> )	305 (1.04 x 10 <sup>6</sup> )
Lower Drywell		
	Normal kW (Btu/hr)	Refueling kW (Btu/hr)
Sensible Load	214.8 (0.73 x 10 <sup>6</sup> )	88 (0.3 x 10 <sup>6</sup> )
Latent Load	107.1 (0.37 x 10 <sup>6</sup> )	Negligible
Total Load	321.9 (1.1 x 10 <sup>6</sup> )	88 (0.3 x 10 <sup>6</sup> )

**Table 9.4-15**  
**Design Parameters for TBVS**

<b>TBVS</b>	
Operating periods:	Normal plant operation, plant startup, and plant shutdown
Temperatures:	
Outdoor Design:	Summer: 37.8°C db (100°F), 26.1°C wb (79°F) (coincident) 27.8°C (82°F) wb (non-coincident)  Winter: -23.3°C (-10°F) Dry bulb
Indoor Design Parameters:	
<u>General Areas</u>	
Indoor Minimum temperature	15°C (59°F)
Indoor Maximum temperature	40°C to 49°C (104°F to 120°F)
Relative Humidity	Uncontrolled
<u>Areas with electrical equipment</u>	
Indoor Minimum temperature	15°C (59°F)
Indoor Maximum temperature	40°C (104°F)
Relative Humidity	Uncontrolled
<b>TBAS Subsystem</b>	
<u>Air Handling Units</u>	
Number	1 x 100%
Capacity	48,000 l/s
<u>Fans</u>	
Number	3 x 50%
Type	Centrifugal or Axial with Variable Speed Drive
Capacity	24,000 l/s
<u>Filters</u>	
Type	Medium Efficiency Filters

**Table 9.4-15**  
**Design Parameters for TBVS (continued)**

<b>TBE Subsystem</b>	
<u>Air Filtration Units (AFU)</u>	
Number	5
Capacity	14,158 l/s (30,000 cfm max. each)
<u>Fans</u>	
Number	3 x 50%
Type	Centrifugal or Axial with Variable Speed Drive
Capacity	26,400 l/s each
<u>Prefilters</u>	
Type	Medium Efficiency Filters
<u>Filters</u>	
Type	High Efficiency Filters
<b>TBCE Subsystem</b>	
<u>Fans</u>	
Number	2 x 100%
Type	Centrifugal or Axial
Capacity	Established during detailed design
<u>Filters</u>	
Type	High Efficiency Filters
<b>TBLOE Subsystem</b>	
<u>Fans</u>	
Number	2 x 100%
Type	Centrifugal or Axial
Capacity	Established during detailed design

**Table 9.4-15  
Design Parameters for TBVS (continued)**

<b>TBDRE Subsystem</b>	
<u>Fans</u>	
Number	1 x 100%
Type	Centrifugal or Axial
Capacity	Established during detailed design
<u>Prefilters</u>	
Type	Medium Efficiency Filters
<u>Filters</u>	
Type	HEPA filters (99.97% DOP)

**Table 9.4-16**  
**Design Parameters for EBVS**

<b>EBVS</b>	
Operating periods:	Normal plant operation, plant startup, and plant shutdown
Temperatures:	
Outdoor Design:	Summer: 37.8°C db (100°F), 26.1°C wb (79°F) (coincident) 27.8°C (82°F) wb (non-coincident)  Winter: -23.3°C (-10°F) Dry bulb
Indoor Design:	
Switchgear and Cable Spreading Rooms	10°C to 40°C (50°F to 104°F) RH uncontrolled
Diesel Generator General Areas and Day Tank Room:	10°C to 50°C (50°F to 122°F) RH uncontrolled
DG Electronic Equipment Areas	10°C to 40°C (50°F to 104°F) RH uncontrolled
Battery Rooms	Nominal 25°C (77°F) Range 15.5°C to 32°C (60°F to 90°F) RH uncontrolled Hydrogen Concentration 2% maximum
Electronic Rooms	18.3°C to 29.4°C (65°F to 85°F) RH uncontrolled
Technical Support Center	23°C to 26°C (73°F to 78°F) RH 25 – 60% +31Pa (0.125" wg) pressure with respect to surrounding areas Pressurization flow of approximately 667 l/s (1413 cfm)

Table 9.4-16

**Design Parameters for EBVS (continued)**

Filter Unit	Quantity:	2 - 100% capacity each
	Capacity:	Flow – approximately 667 l/s per unit
	Type:	Metal housing containing medium efficiency filter, HEPA filter, carbon filter, and post filter
	Efficiency :	99% credited removal for HEPA and 99% credited removal for carbon filter per RG 1.140

**Table 9.4-17**  
**Industrial Codes and Standards<sup>1</sup> Applicable to ESBWR HVAC**

<b>Code or Standard Number</b>	<b>Year</b>	<b>Title</b>
<b>American Iron and Steel Institute (AISI)</b>		
CF02-1		Cold-Formed Steel Framing Design Guide (Latest edition based on the 1996 edition and 2000 supplement of the AISI Specification for the Design of Cold-Formed Steel Structural Members)
SG02-1 and SG02-2	2001	North American Specification for the Design of Cold-Formed Steel Structural Members, and Commentary
SG05-1e	2004	Supplement 2004 to the North American Specification for the Design of Cold-Formed Steel Structural Members, 2001 Edition
<b>Air-Conditioning and Refrigeration Institute (ARI)</b>		
410-01	2001	Force-circulation Air-cooling and Air-heating Coils
430-99	1999	Central Station Air Handling Units
450-99	1999	Water-Cooled Refrigerant Condensers, Remote Type
550/590-03	2003	Water Chilling Packages Using the Vapor Compression Cycle
575-94	1994	Method of Measuring Machinery Sound Within an Equipment Space
<b>Air Movement and Control Association (AMCA)</b>		
99-03	2003	Standards Handbook
200-95	1995	Air Systems
201-02	2002	Fans and Systems
202-98	1998	Troubleshooting
203-90	1990	Field Performance Measurements of Fan Systems
210-99	1999	Laboratory Methods of Testing Fans for Rating – Addenda A, August 21, 2001
301	1990	Methods for Calculating for Sound Ratings from Laboratory Test Data
302	1973	Sone Rating Applications Publication
303-79	1979	Sound Power Level Ratings Applications Publication
410-96	1996	Safety Practices for Users and Installers of Industrial & Commercial Fans
500-D	1998	Laboratory Methods of Testing Dampers for Rating
500-L	1999	Laboratory Methods of Testing Louvers for Rating
502	2001	Damper Application Manual

<sup>1</sup> The listing of a code or standard does not necessarily mean that it is applicable in its entirety. See Table 1.9-22 for applicable revision.

**Table 9.4-17**  
**Industrial Codes and Standards<sup>1</sup> Applicable to ESBWR HVAC**

<b>Code or Standard Number</b>	<b>Year</b>	<b>Title</b>
503	2003	Fire, Ceiling (Radiation), Smoke and Fire/Smoke Dampers Application Manual
801-01	2001	Industrial Process/ Power Generation Fans: Specification Guidelines
<b>American Nuclear Society (ANS)</b>		
56.7-1978	1978 (R 1987)	Boiling Water Reactor Containment Ventilation Systems
59.2-1985	1985	Safety Criteria for HVAC Systems Located Outside Primary Containment
<b>American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE)</b>		
15-2001	2001	Safety Standard for Refrigeration Systems
30-1995	1995	Methods of Testing Liquid-Chilling Packages
33-2000	2000	Methods of Testing Forced Circulation Air Cooling and Air Heating Coils
51-1999	1999	Laboratory Methods of Testing Fans for Aerodynamic Performance Rating
52-1976	1976	Testing Air-Cleaning Devices Used in General Ventilation for Removing Particulate Matter
52.1-1992	1992	Gravimetric and Dust-Spot Procedures for Testing Air-Cleaning Devices Used in General Ventilation for Removing Particulate Matter
52.2-1999	1999	Method of Testing General Ventilation Air-Cleaning Devices for Removal Efficiency by Particle Size
62-2001	2001	Ventilation for Acceptable Indoor Air Quality
<b>American Society of Mechanical Engineers (ASME)</b>		
AG-1-2003	2003	Code on Nuclear Air and Gas Treatment
B31.1-2004	2004	Power Piping
B31.5-2001	2001	Refrigeration Piping and Heat Transfer Components
N509-2002	2002	Nuclear Power Plant Air-Cleaning Units and Components
N510-1989	1989 (R 1995)	Testing of Nuclear Air-Treatment Systems – Errata: January 1991
<b>American Society for Testing and Materials (ASTM)</b>		
D3803	1989 (R 1995)	Standard Test Methods for Nuclear-Grade Activated Carbon
E741-00	2000	Quality Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution
<b>American Water Works Association (AWWA)</b>		
C200-97	1997	Steel Water Pipe – 6 in. (150mm) and Larger, 2nd Edition

**Table 9.4-17**  
**Industrial Codes and Standards<sup>1</sup> Applicable to ESBWR HVAC**

<b>Code or Standard Number</b>	<b>Year</b>	<b>Title</b>
C203-02	2002	Coal-Tar Protective Coatings and Linings for Steel Water Pipelines – Enamel and Tape – Hot Applied
D100-96	1996	Welded Steel Tanks for Water Storage
<b>Institute of Electrical and Electronics Engineers (IEEE)</b>		
338-1987	1987 (R 2000)	Standard Criteria for the Periodic Surveillance Testing of Nuclear Power Generating Station Safety Systems
484-2002	2002	Recommended Practice for Installation Design and Installation of Vented Lead-Acid Batteries for Stationary Applications
<b>Manufacturers Standardization Society of the Valve and Fittings Industry, Inc (MSS)</b>		
SP 67A-02	2002	Butterfly Valves
<b>National Fire Protection Association (NFPA)</b>		
NFPA 80	1999	Standard for Fire Doors and Windows
NFPA 90A	2002	Standard for the Installation of Air-Conditioning and Ventilating Systems
NFPA 90B	2002	Standard for the Installation of Warm Air Heating and Air-Conditioning Systems
NFPA 91	2004	Standard for Exhaust Systems for Air Conveying of Vapors, Gases, Mists and Noncombustible Particulate Solids
NFPA 92A	2006	Standard for Smoke Control Systems Utilizing Barriers and Pressure Differences
NFPA 101	2003	Life Safety Code
NFPA 110	2002	Standard for Emergency and Standby Power Systems
NFPA 204	2002	Standard for Smoke and Heat Venting
NFPA 214	2000	Standard on Water-Cooling Towers
NFPA 251	1999	Standard Methods of Tests of Fire Endurance of Building Construction and Materials
NFPA 252	2003	Standard Methods of Fire Tests of Door Assemblies
NFPA 804	2006	Standard for Fire Protection for Advanced Light Water Reactor Electric Generating Plants
<b>National Air Filtration Association (NAFA)</b>		
	2006	Installation, Operation and Maintenance of Air Filtration Systems Manual
<b>Sheet Metal and Air Conditioning Contractors' National Association (SMACNA)</b>		
1208	1990	HVAC Systems – Duct Design, 3rd Edition
1481	2005	HVAC Duct Construction Standards – Metal and Flexible, 3rd Edition
1299	1980	Rectangular Industrial Duct Construction Standards

**Table 9.4-17**  
**Industrial Codes and Standards<sup>1</sup> Applicable to ESBWR HVAC**

<b>Code or Standard Number</b>	<b>Year</b>	<b>Title</b>
1520	1999	Round Industrial Duct Construction Standards
1143	1985	HVAC Air Duct Leakage Test Manual
1780	2002	HVAC Systems – Testing, Adjusting and Balancing
<b>Underwriters Laboratories, Inc. (UL)</b>		
555	1999	UL Standard for Safety Fire Dampers, 6th Edition (Reprint with Revisions through and Including January 2, 2002)
555S	1999	Smoke Dampers
586	1996	High-Efficiency, Particulate, Air Filter Units
900	2004	UL Standard for Safety Air Filter Units, 7th Edition
1096	1988	UL Standard for Safety Electric Central Air Heating Equipment, 4th Edition
1950	1995	UL Standard for Safety Information Technology Equipment, Including Electrical Business Equipment; Third Edition
1995	2005	Heating and Cooling Equipment

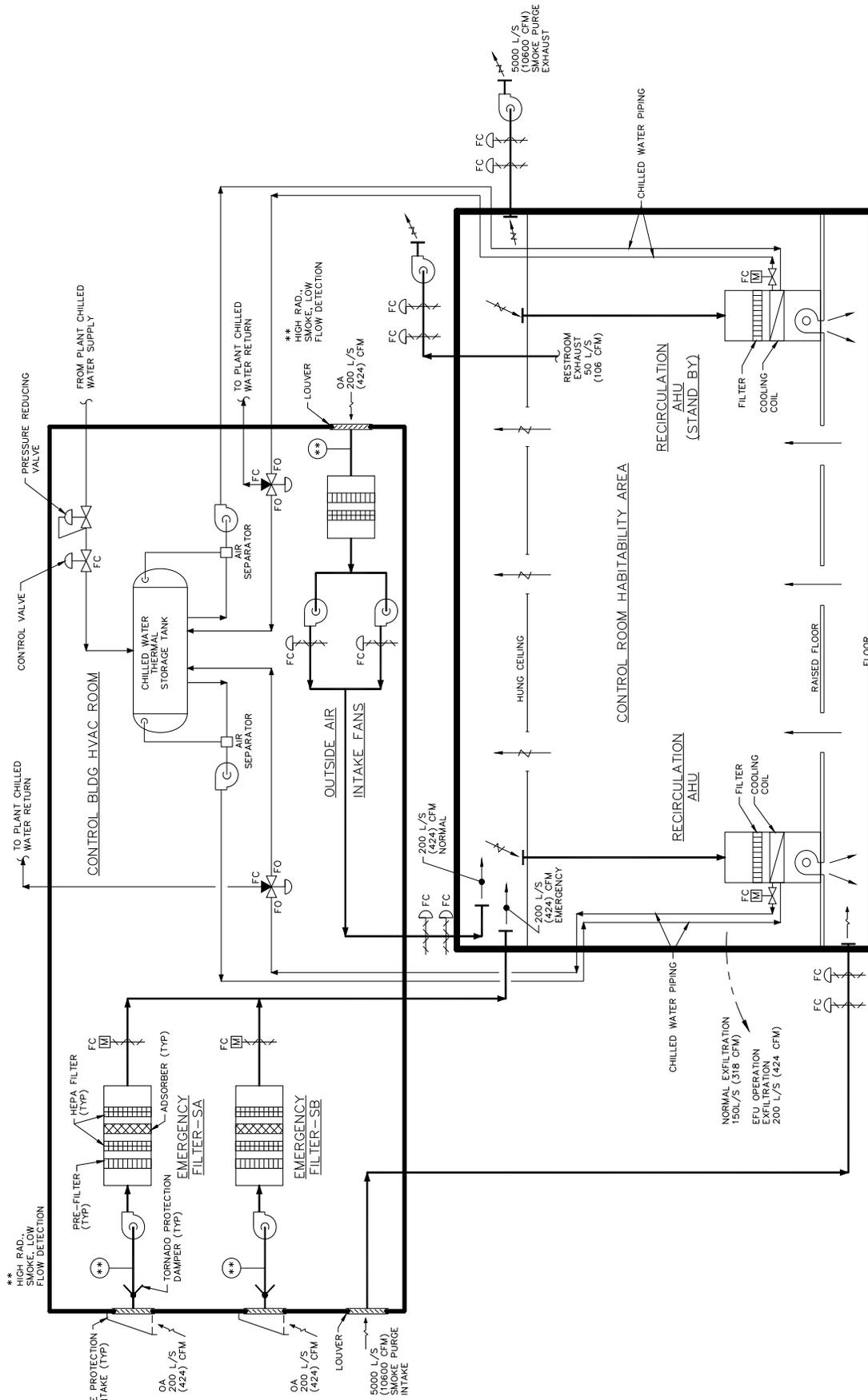


Figure 9.4-1. CRHAVS Simplified System Diagram

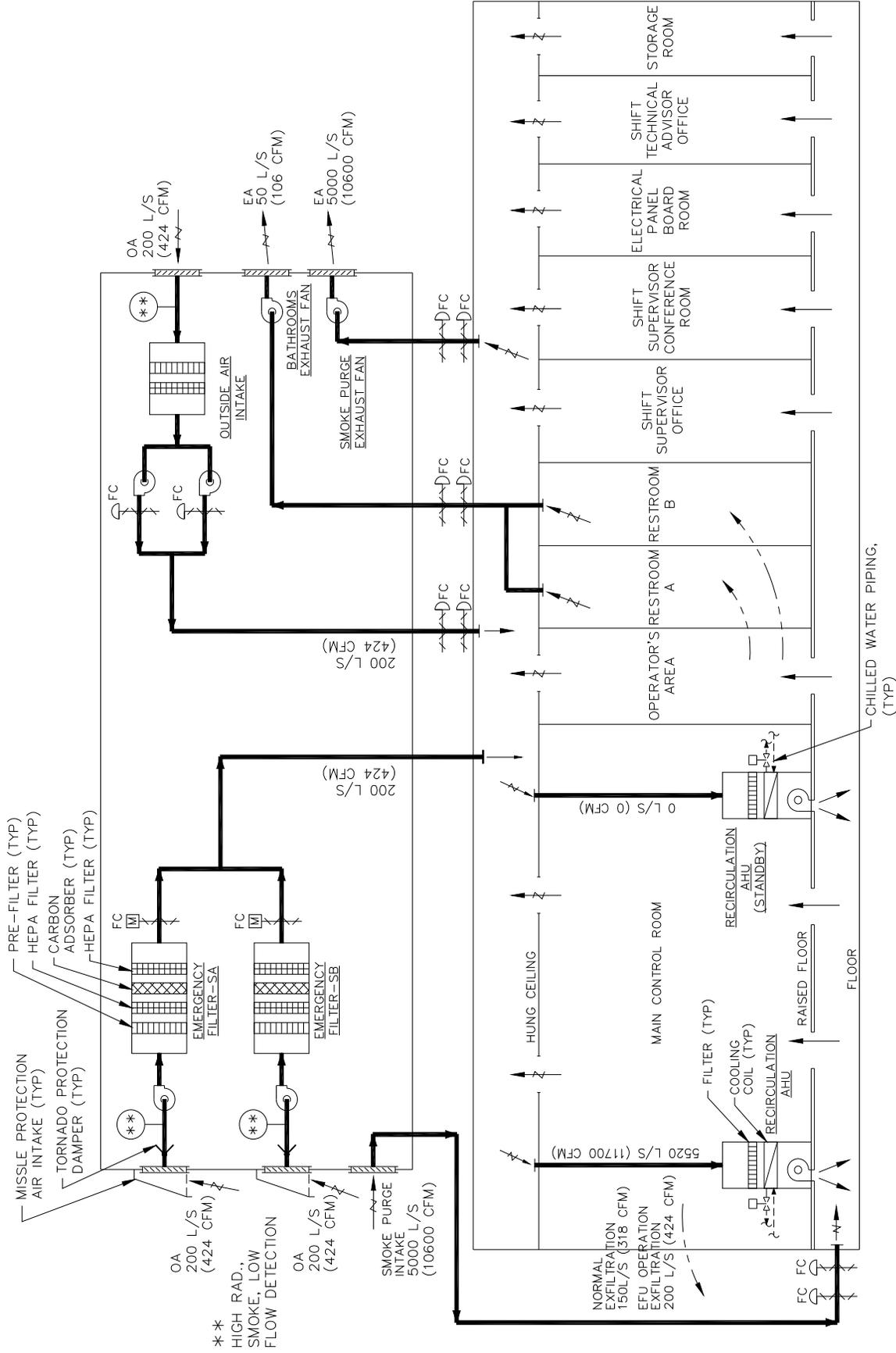


Figure 9.4-2. CRHAVS Air Flow Diagram

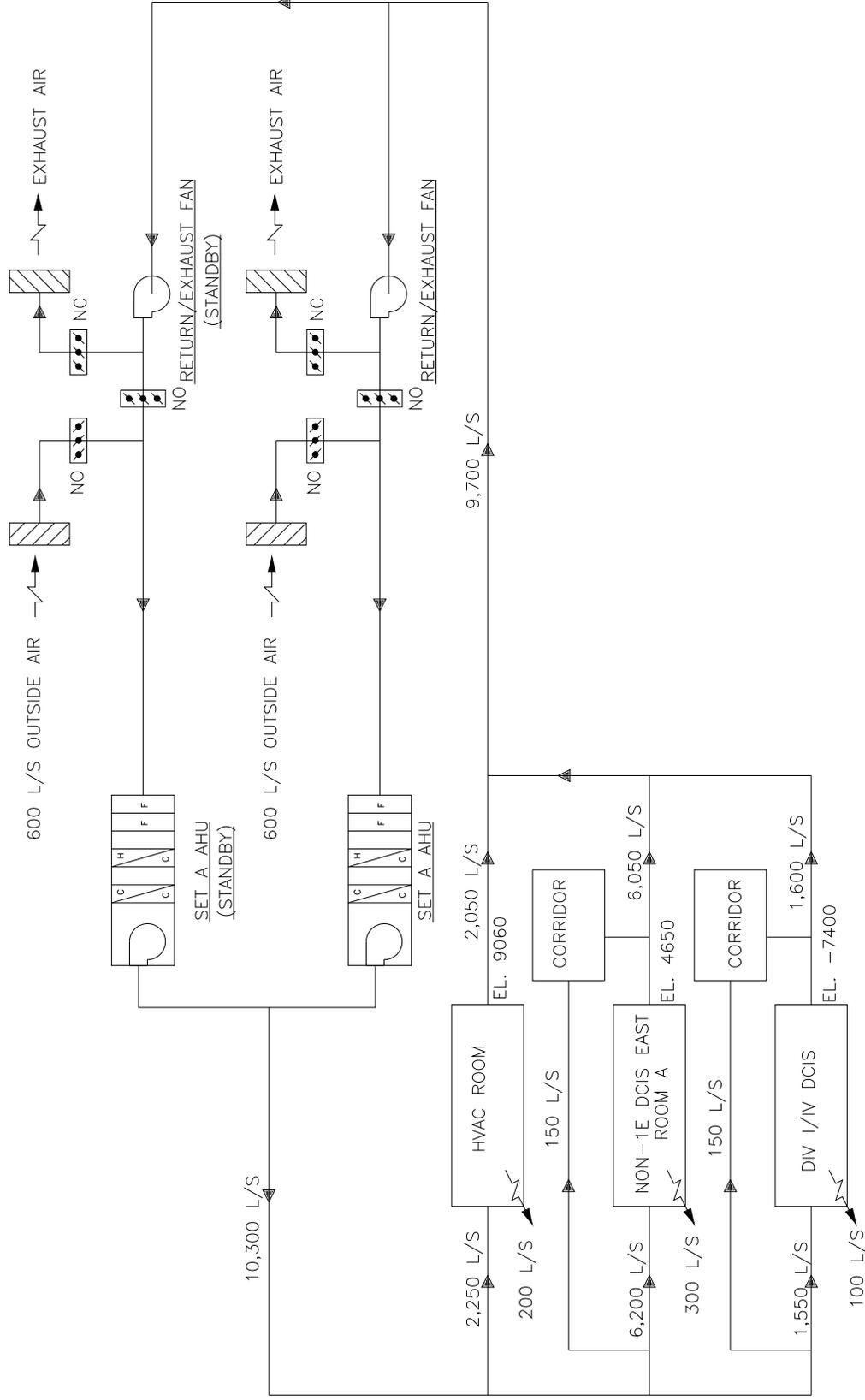


Figure 9.4-3. CBGAVS SET A Simplified System Diagram

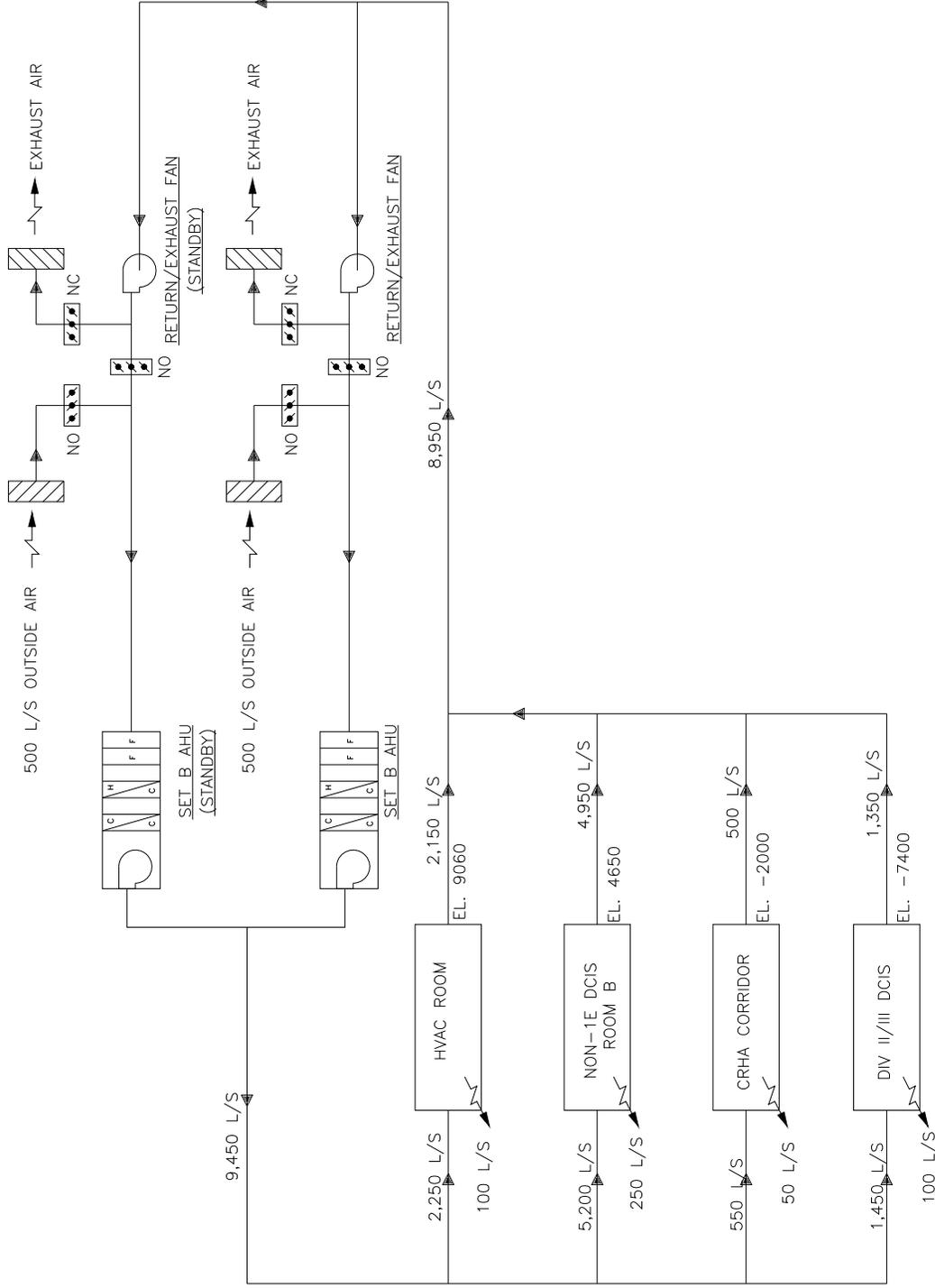


Figure 9.4-4. CBGAVS SET B Simplified System Flow Diagram

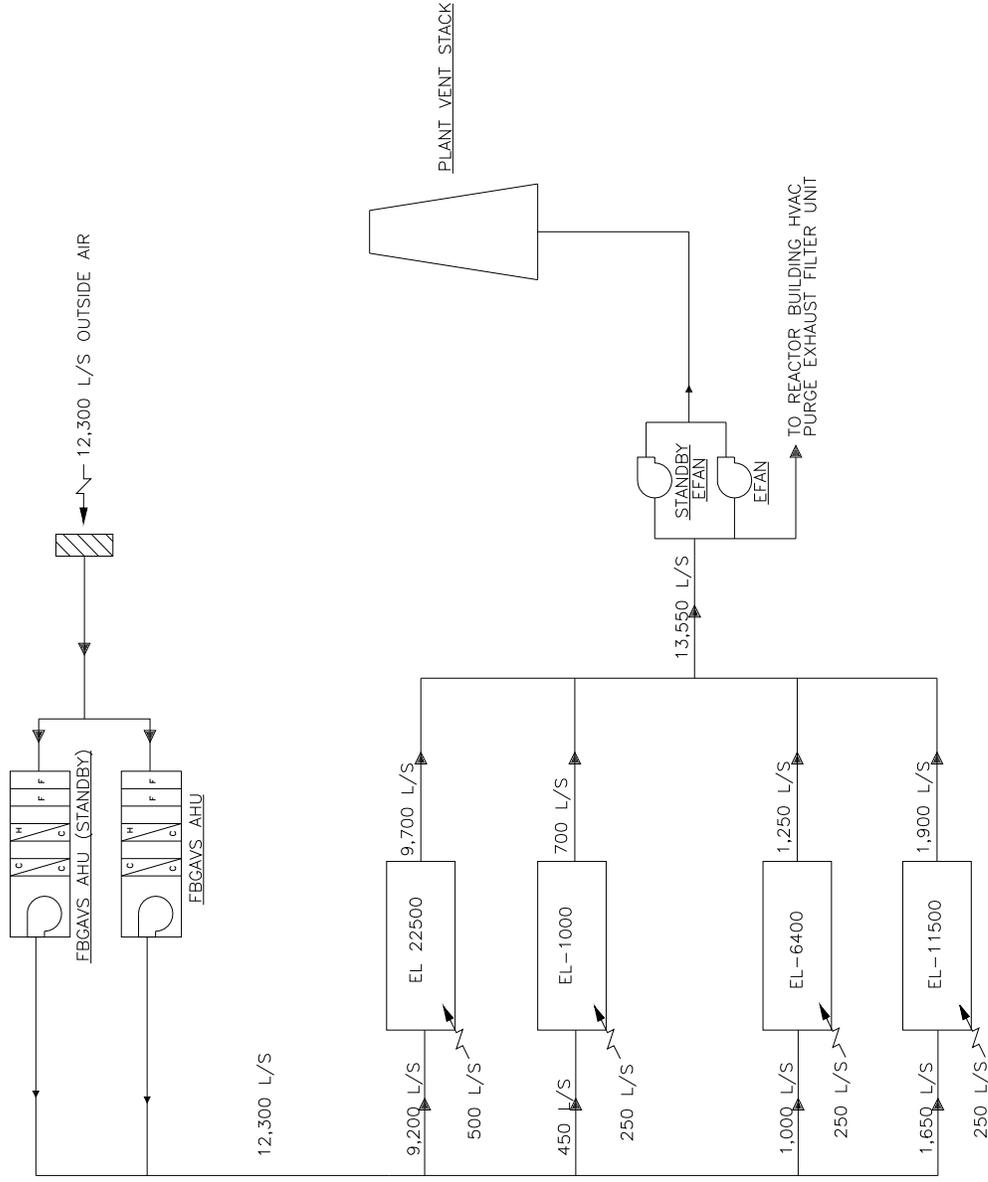


Figure 9.4-5. FBGAVS Simplified System Diagram

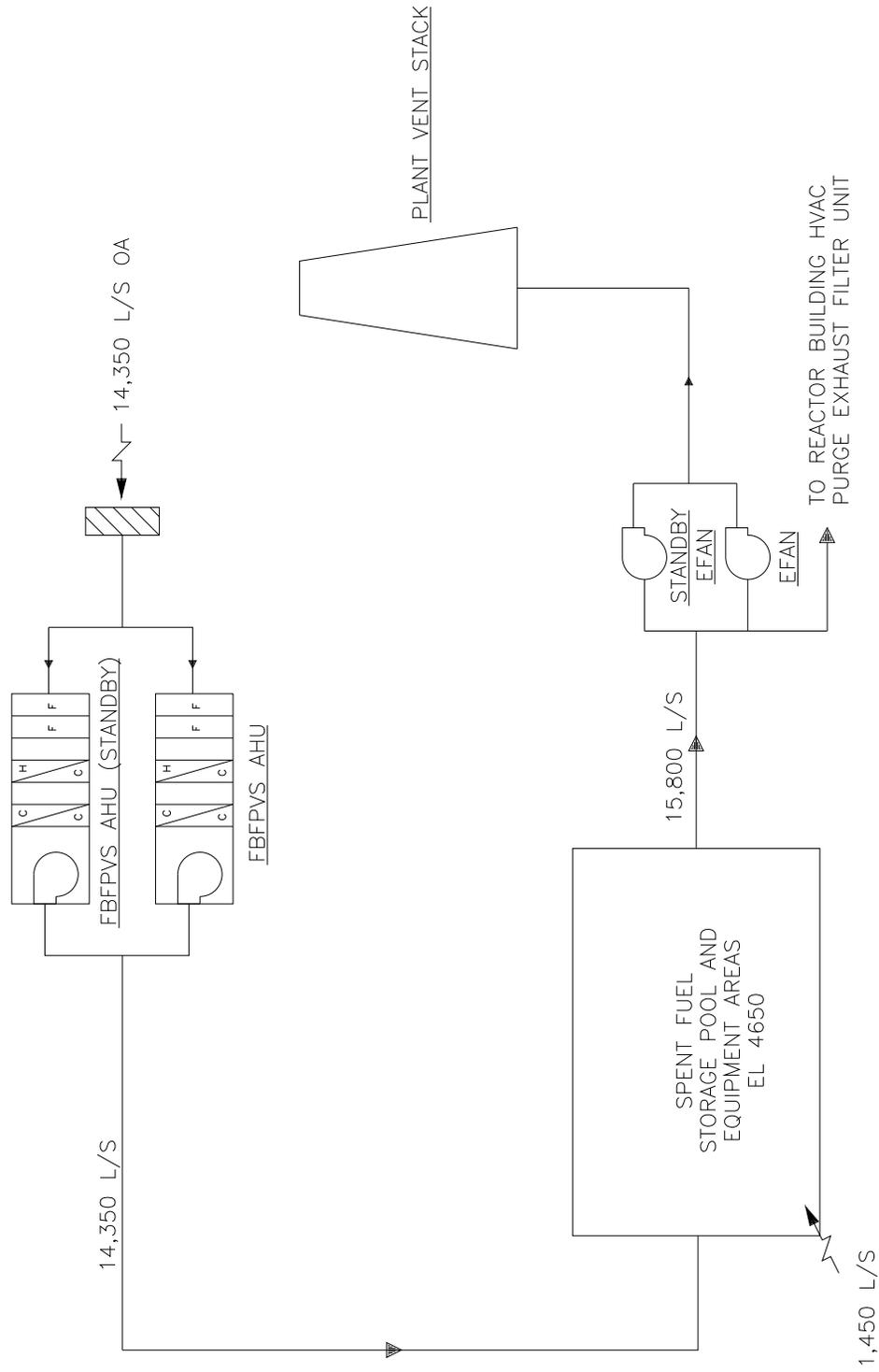
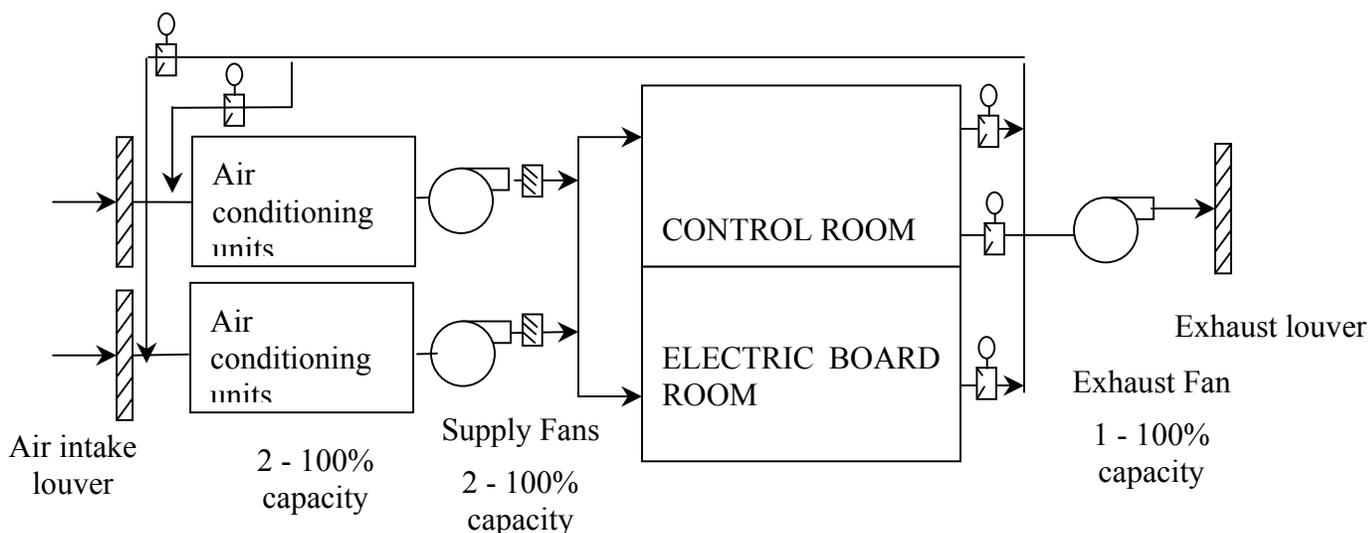
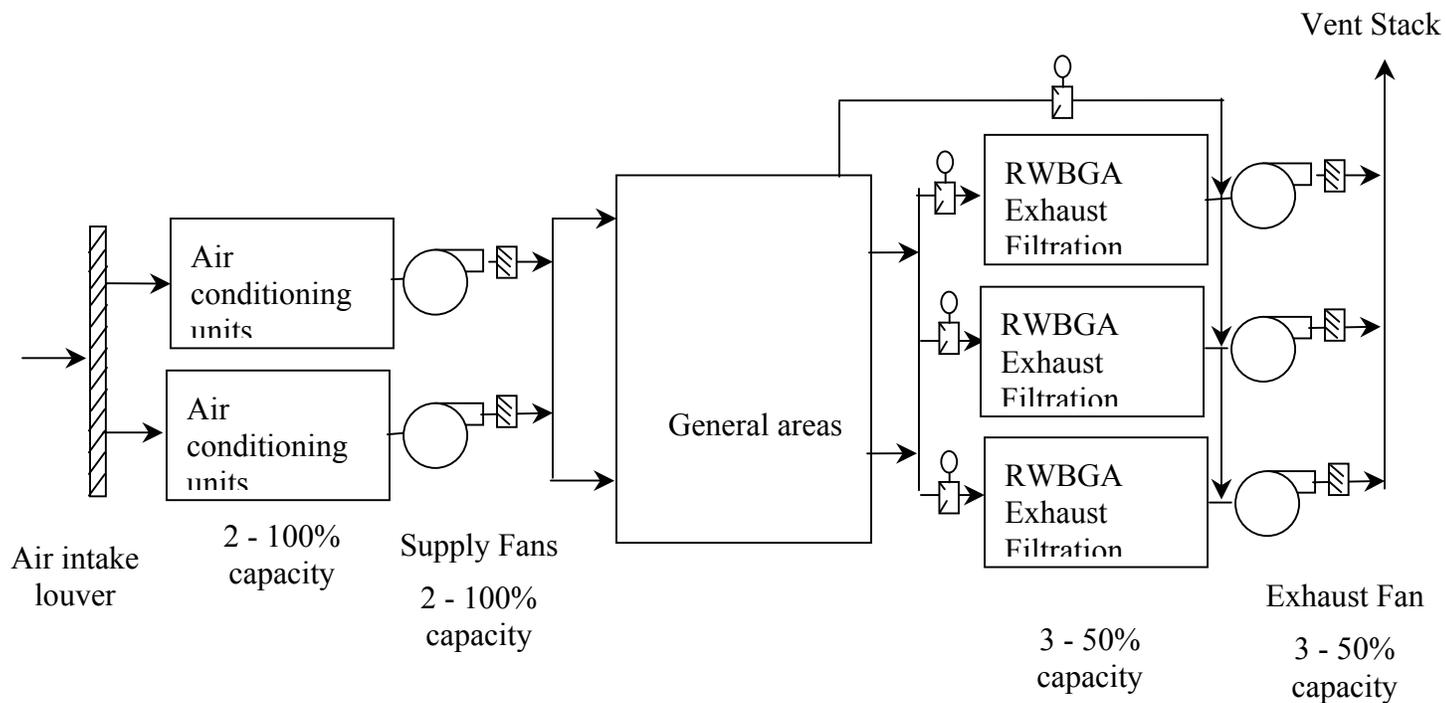


Figure 9.4-6. FBFPVS Simplified System Diagram



RWCRVS Simplified System Diagram



RWGAVS Simplified System Diagram

**Figure 9.4-7. RWVS Simplified System Diagrams**

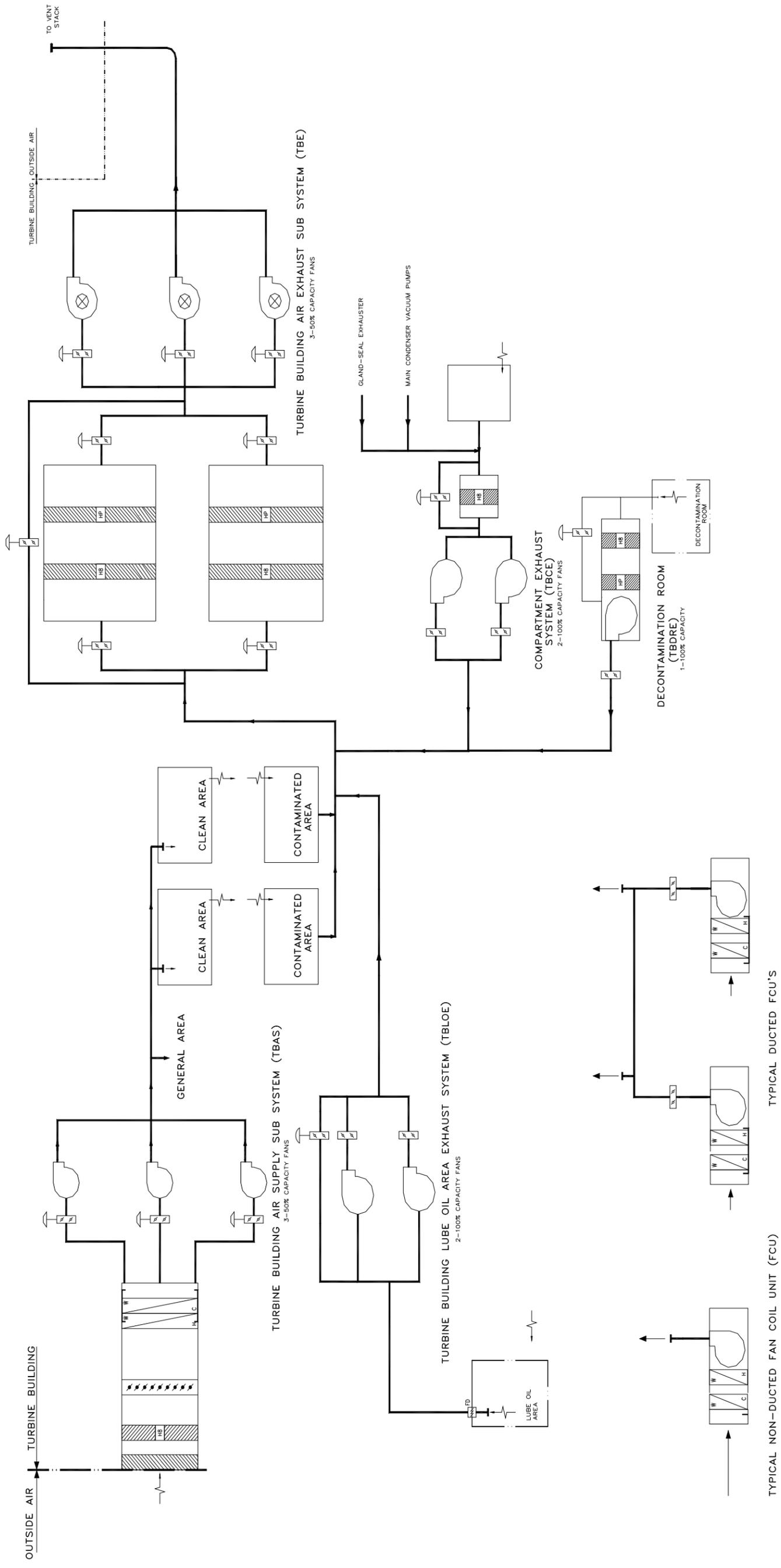


Figure 9.4-8. TBVS Simplified System Diagram



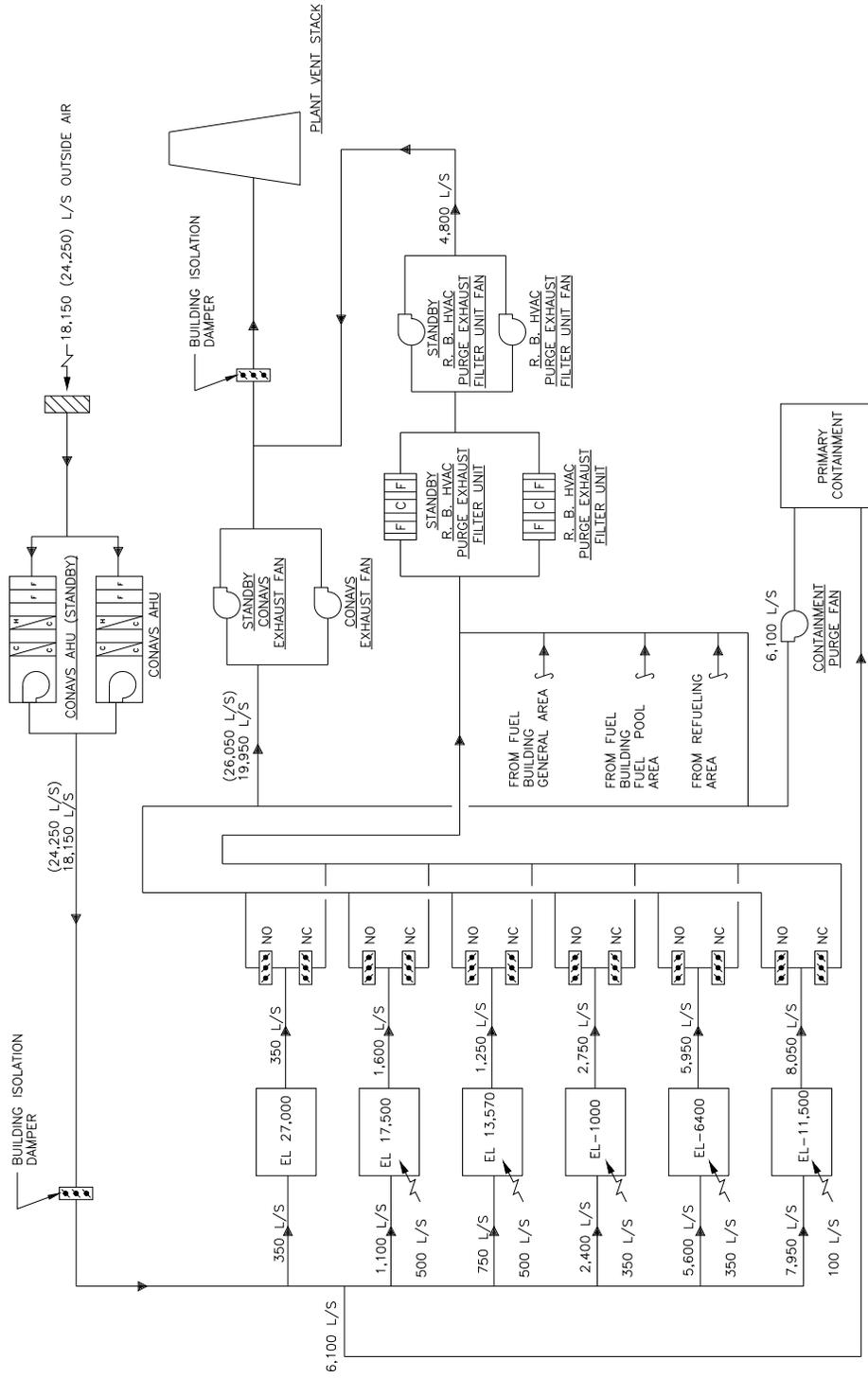


Figure 9.4-10. CONAVS Simplified System Diagram

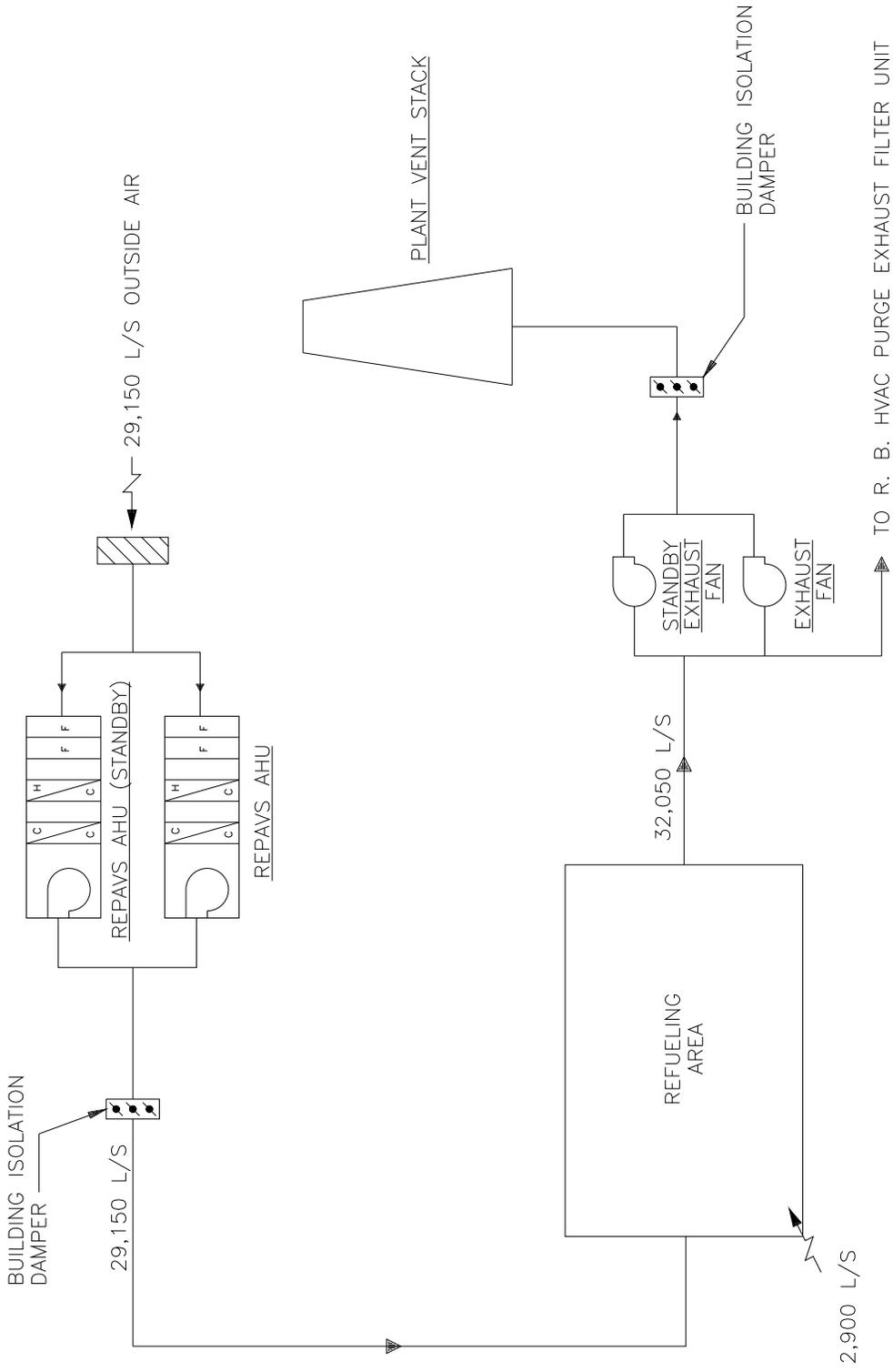


Figure 9.4-11. REPAVS Simplified System Diagram

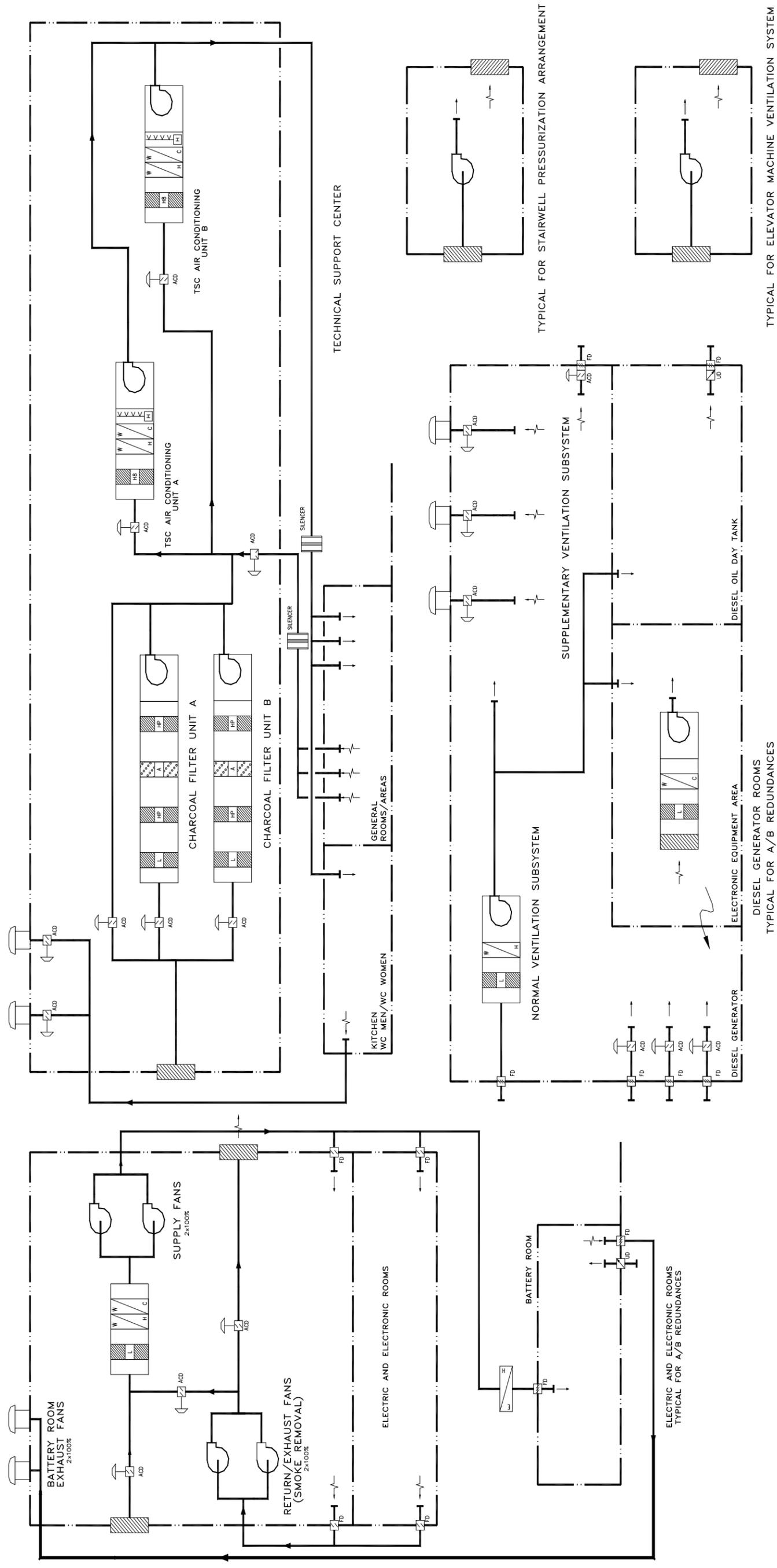


Figure 9.4-12. Electrical Building HVAC System

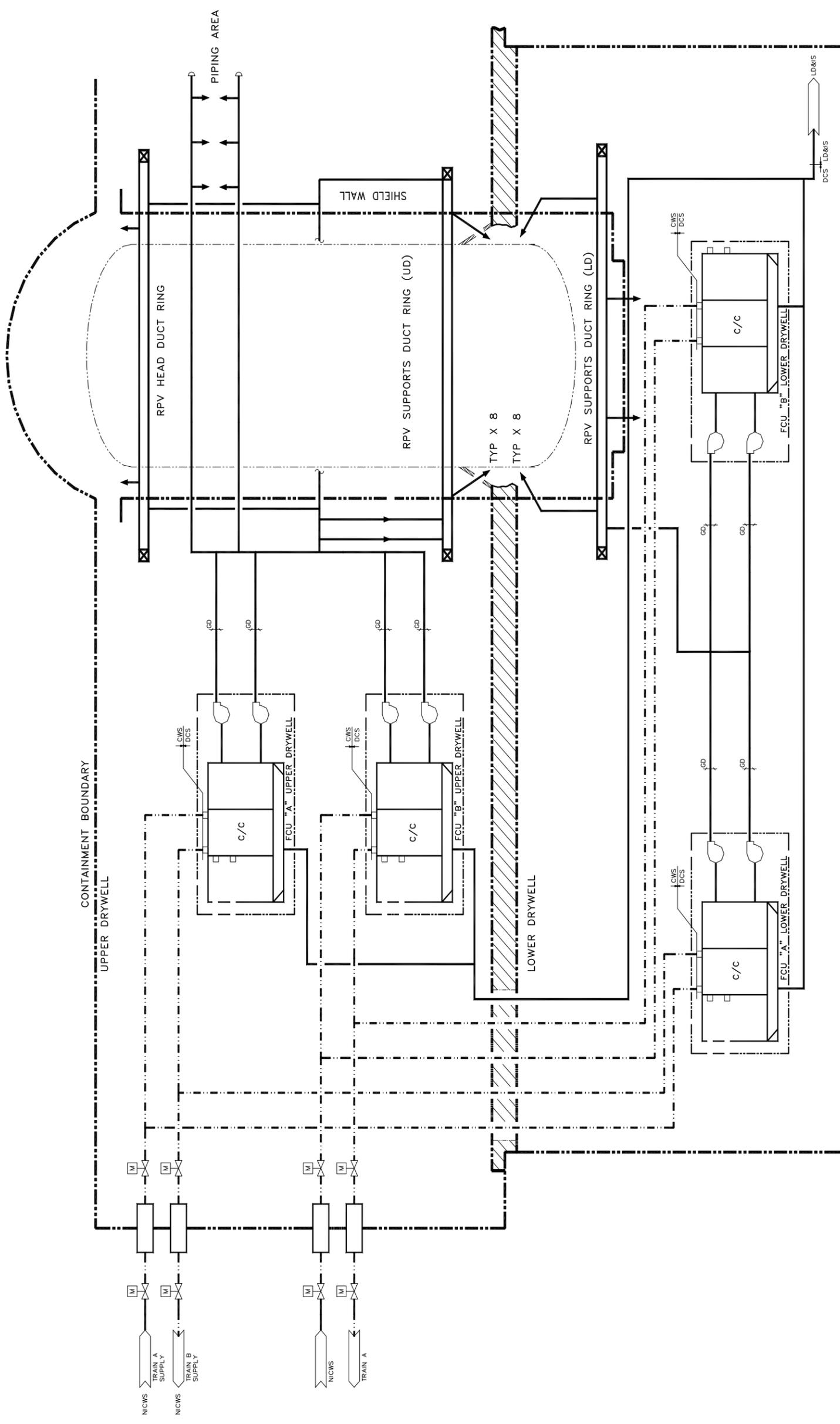


Figure 9.4-13. DCS Simplified System Diagram

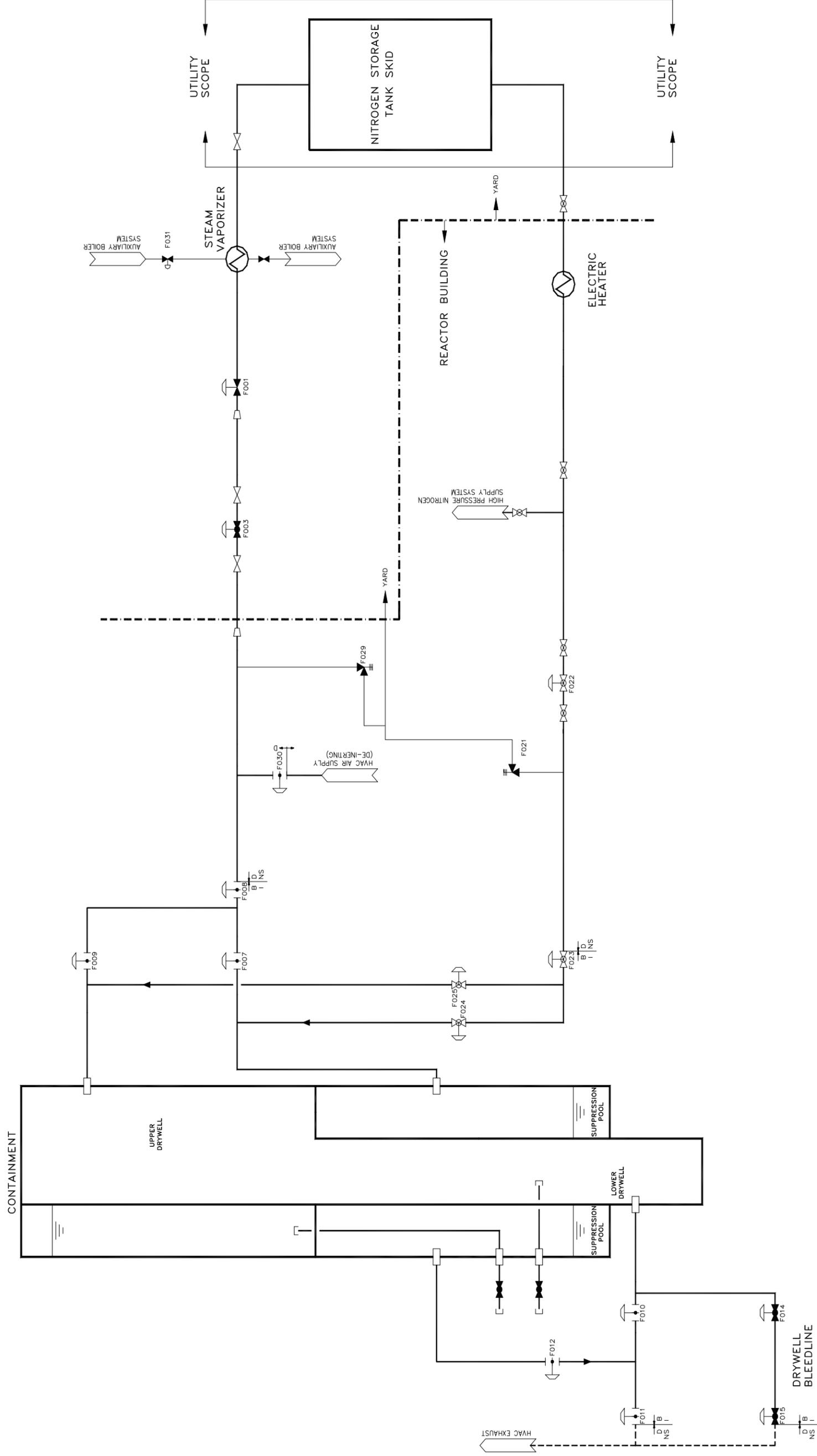


Figure 9.4-14. CIS Simplified System Diagram

## 9.5 OTHER AUXILIARY SYSTEMS

### 9.5.1 Fire Protection System

The term “fire protection system” refers to the integrated complex of components and equipment provided for detection, notification, annunciation, and suppression of fires. In addition to this system, the “fire protection program” includes the concepts of design and layout implemented to prevent or mitigate fires, administrative controls and procedures, and the training of personnel to combat fires.

#### 9.5.1.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The Fire Protection System (FPS) does not perform any safety-related function. The FPS connects to the safety-related portion of the Fuel and Auxiliary Pools Cooling System (FAPCS).

##### Power Generation Design Bases

In accordance with NUREG-0800 SRP 9.5.1 and Regulatory Guide 1.189, fire protection for nuclear power plants uses the concept of defense in depth to achieve the required degree of reactor safety by using administrative controls, fire protection systems and features, and safe shutdown capability. These defense-in-depth principles achieve the following objectives:

- To prevent fires from starting;
- To rapidly detect, control, and extinguish promptly those fires that do occur; and
- To provide protection for structures, systems, and components important to safety so that a fire that is not promptly extinguished by the fire suppression activities does not prevent the safe shutdown of the plant and will not significantly increase the risk of radioactive release to the environment.

The ESBWR fire protection program design bases incorporate the following elements:

- To maintain the ability to safely shut down the reactor and keep it shut down by providing adequate separation of safety-related equipment. This capability is to be achievable during all modes of plant operation.
- To minimize the probability of the spread of fire by the use of fire barriers between areas of significant combustible loading; and
- To maintain the ability to minimize the potential for radioactive releases to the environment in the event of a fire.

The ESBWR FPS design bases incorporate the following elements:

- To maintain the ability to safely shut down the reactor and keep it shut down by providing the capability to control the spread of and extinguish the postulated fires in all plant areas by the use of fixed and/or portable fire fighting equipment. This capability is achievable during all modes of plant operation.
- To provide automatic fire detection and annunciation for selected areas of the plant as required by the Fire Hazards Analysis for personnel safety and fire brigade notification;

## ESBWR

- To supply the maximum firewater demand at any point throughout the system, with one fire pump out of service;
- To prevent inadvertent operation of the FPS from jeopardizing the capability to achieve safe shutdown of the plant;
- To preclude damage to plant safety-related structures, systems, or components caused by seismic loading of the FPS;
- To keep equipment required for safe shutdown free from fire damage during a safe shutdown earthquake (SSE). To this end, one source of firewater supply, including a water source; one fire pump and its associated suction and discharge lines; and firewater lines, including standpipes and hose connections, are designed and analyzed to remain functional after an SSE. This includes analysis to the first isolation valve on all branches connected to the seismically analyzed firewater lines.
- To ensure a continuous firewater supply for the fire pumps in the event of failure of one firewater source. Two separate firewater sources are connected to FPS such that there is no interruption in supply and that failure of one water source or its piping does not drain the other source.
- To provide manual suppression capability to all plant areas, including those that have automatic fire suppression systems;
- To ensure that a single active failure in a moderate-energy line cannot impair both the primary and backup fire suppression systems;
- To permit isolation from the fire main or outside hydrants for maintenance or repair without interrupting FPS water supply;
- To ensure at least one effective hose stream can reach any location containing safe shutdown equipment, for preventing a fire exposure hazard to the equipment;
- To provide an emergency backup source of makeup water for auxiliary refueling pools and reactor water inventory control through a piping connection to the Fuel and Auxiliary Pools Cooling System (FAPCS); and
- To have a useful life of 60 years with normal maintenance and replacement of parts/components subject to normal wear and deterioration.

### Codes, Standards, and Regulatory Guidance

Table 9.5-1 lists the codes, standards, and guidelines used in the fire protection program and FPS design.

#### 9.5.1.2 System Description

Figure 9.5-1 shows the FPS simplified system diagram. Table 9.5-2 lists the Component Design Characteristics.

The FPS is the integrated complex of equipment and components that provides early fire detection and suppression to limit the spread of fires. The FPS is part of the overall fire protection program including the plant design and layout to prevent or mitigate fires and includes administrative controls and procedures.

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The type of fire suppression is based on the combustible loading and the extent of safety-related equipment within a fire area. Fixed automatic fire suppression systems are installed in areas identified as having a high fire hazard rating by the Fire Hazards Analysis (FHA) (Appendix 9A). Building standpipes and hose stations are provided in major buildings. Portable fire extinguishers are strategically located throughout the plant in accordance with NFPA 10, except in highly radioactive areas.

An automatic fire detection, alarm, supervisory control, and indication system is also provided in selected areas of the plant, as required by the fire hazards analysis for personnel safety and fire brigade notification.

A main fire alarm panel (MFAP), is located in the Main Control Room (MCR), monitors and receives system actuation, supervisory, and trouble alarm signals from the individual local panels.

***9.5.1.3 Facility Features for Fire Protection***

Consistent with applicable safety-related requirements, structures, systems, and components are designed and located to minimize the probability and effect of fires. To the maximum extent practical, noncombustible and fire-resistant materials minimize the combustible loading and thereby reduce the expected duration, severity, and intensity of fires.

Within the safety-related structures, interior walls, partitions, structural components, materials for insulation, and radiation shielding are either noncombustible or have low ratings for contribution. The flame spread and smoke development rating of these materials is 25 or less. Materials having a flame spread and smoke development rating of 50 or more are considered to be combustible when analyzing fire hazards.

Exposed structural steel protecting safety-related areas is fireproofed with material with a fire rating of up to 3 hours as determined from the FHA.

Access stairwells are enclosed in minimum 2-hour rated firewalls and equipped with self-closing fire-rated doors. Openings in fire barriers or firewalls are equipped with fire doors, frames, and hardware rated the same as the barriers they penetrate.

Safety-related raceway and circuit routing comply with Branch Technical Position (BTP) SPLB 9.5-1 except that separation by fire barriers rather than distance is used outside the MCR or primary containment. Exceptions to this requirement are analyzed and justified as acceptable on an individual basis. The acceptance criterion is that a single fire cannot degrade the performance of more than one division of safe shutdown equipment controlled from the main control room. However, if alternate means of control or indication are provided or available that remain unaffected by the same fire, then exception to the BTP SPLB 9.5-1 requirements for circuit routing and separation may be taken. The alternate means of control or indication are not required to be safety-related. All electrical cables (safety-related and nonsafety-related) conform to IEEE-1202 flame test criteria.

The intent is to avoid the use of electrical raceway fire barrier systems (ERFBS) for ESBWR, relying instead on divisional separation by fire area and structural fire barriers.

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#### ***9.5.1.4 Fire Protection Water Supply System***

##### **Water Source**

Water for the Fire Protection System is supplied from a minimum of two reliable sources. The primary source shall be two dedicated, Seismic Category I, firewater storage tanks. Each source has sufficient capacity to meet the maximum firewater demand of the system for a period of 120 minutes. The secondary source may be a second firewater storage tank, a cooling tower water basin, or a large body of water, with the minimum capacity to meet the total water demand for a period of 120 minutes. Water sources that are used for multiple purposes ensure that the required quantity of firewater is dedicated for fire protection use only. The COL applicant will provide final quantity and capacity of secondary firewater storage.

Fire department connections on all major buildings allow a fire department pumper truck to pump water into the FPS as an additional fire protection water supply source.

The primary Seismic Category I, firewater storage tanks provide the required emergency makeup water volume for the IC/PCCS pools and Spent Fuel Pool to the FAPCS for 72 hours after this time on-site or offsite makeup sources can be used. This function of FPS is considered to be RTNSS rather than Safety-related due to being required post 72 hrs. A deviation to this acceptance criterion II.1.a of SRP 9.1.3 (which requires Quality Group C for Fuel pool make up components) will be added for this nonsafety-related make up water RTNSS function from quality group C to D in Table 1.9-9.

If both primary fire pumps and primary firewater storage tanks are available, make-up water is capable of being provided from both primary fire pumps and primary firewater storage tanks at the same time.

##### **Fire Pumps**

Two primary fire pumps each provide 100% of the firewater demand to the worst-case fire within the nuclear island (Reactor Building, Fuel Building, and Control Building) and 50% of the firewater demand to the worst-case fire within the Turbine Building (TB). Two secondary fire pumps each provide 50% of the firewater demand for the worst-case fire in the TB and 100% of the firewater demand for the worst-case fire in the remainder of the balance of plant (BOP). All fire pumps are capable of delivering the flow and pressure required to the location that is farthest from the firewater supply source. The two primary fire pumps are located near the nuclear island power block in a fire pump enclosure (FPE). The two secondary fire pumps are located remote from the other two pumps to avoid any common-location failures. The COL applicant shall provide the final secondary fire pump flow rate and pump head. For the two primary nuclear island fire pumps, the lead fire pump is motor-driven and the backup is a Seismic Category I diesel driven fire pump. The backup diesel-driven fire pump provides firewater in the event of failure of the motor-driven fire pump or loss of preferred power (LOPP).

For the two secondary fire pumps, the lead fire pump shall be motor-driven and the backup fire pump shall be Non-Seismic (NS) diesel-driven. The secondary diesel-driven fire pump provides firewater in the event of failure of the motor-driven fire pump.

Two motor-driven jockey pumps shall be provided to prevent initiation of the primary and secondary motor-driven fire pumps due to minor pressure losses. The motor-driven jockey

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pumps shall maintain the system pressure at a minimum of 34.4 kPa (5psi) above the highest start pressure setpoint of the fire pumps.

Booster pumps shall be provided to maintain minimum standpipe pressure in accordance with NFPA 14. Redundant booster pumps shall be separated by a 3-hour fire-rated wall.

The fuel oil tanks for the diesel-driven fire pumps have a capacity sufficient to allow operation of the diesel engines for approximately 96 hours before refilling based upon the fuel consumption and margin criteria provided in NFPA 24.

***9.5.1.5 Firewater Supply Piping, Yard Piping, and Yard Hydrants***

The firewater supply piping consists of a buried non-seismic, yard main loop and a suspended ASME B31.1, Seismic Category I nuclear island piping loop. The Seismic Category I loop is designed to remain functional following a SSE. The primary fire pumps supply firewater to the Seismic Category I, loop, supplying firewater within the structures of the nuclear island (Reactor Building, Control Building, and Fuel Building). The secondary fire pumps supply firewater directly to the yard main loop. Isolation valves are provided between the buried, non-seismic, yard piping loop and the suspended ASME B31.1 Seismic Category I piping loop.

The yard main loop consists of code compliant material, FM (Factory Mutual) approved for fire main service, with concrete thrust blocks or cement-lined ductile iron piping, in accordance with NFPA 24. Locked open sectionalizing post-indicator valves installed in the yard main loop permit isolation of any part of the main without completely removing the system from service. Valves between connections separate individual fire pump connections from the yard main. The COL applicant will determine the design characteristics of the yard main loop.

Check valves are provided between each building connection from the main yard piping loop. Additionally, a check valve is provided in the nuclear island to turbine building piping connection to prevent flow from the turbine building to the nuclear island.

Fire hydrants located at approximately 76.2 m (250 ft) intervals along the fire main loop provide fire-fighting capability especially in the vicinity of areas or buildings containing combustible materials. The fire hydrants are located no closer than 12.2 m (40 ft) from the protected buildings.

Fire hydrants are protected against freezing and damage from vehicles.

The COL applicant shall provide FPS piping and instrumentation diagrams showing complete site-specific systems.

***9.5.1.6 Manual Suppression Means***

Manual suppression means are provided for all plant areas. The sprinkler systems and the hose station standpipes have separate connections to the firewater main; therefore, no single failure can impair both systems.

**Standpipe and Hose Systems (Wet)**

Standpipes and hose stations are provided in all major buildings. Standpipes and hose stations in areas adjacent to stairways and other locations provide sufficient hose coverage. Minimum

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standpipe size is 100 mm (4 inch). Standpipe size for building heights exceeding 30.5 meters (100 feet) is 150 mm (6 inch).

The wet standpipes and hose stations are designed to NFPA 14 Class III Service.

Each Class III hose station is provided with a 65 mm (2.5 inch) hose valve with cap and a 40 mm (1.5 inch) hose valve. A 6.35-cm (2.5 inch) to 3.8-cm (1.5-inch) reducer is utilized on the 6.35-cm (2.5-inch) hose valve.

Each Class III hose rack has 30.5m (100 ft) of 40 mm (1.5-inch) jacket-lined fire hose.

The water supply pressure maintains a gauge pressure of 448.2 kPaG (65 psig) at the most hydraulically remote 40 mm (1.5 inch) hose station and 689 kPaG (100 psig) at the most hydraulically remote 65 mm (2.5 inch) hose station. If the gauge pressure at a 40 mm (1.5 inch) hose station exceeds 689 kPaG (100 psig), orifice discs installed in the hose couplings reduce the reaction force at the hose end.

Areas containing equipment required for safe shutdown, standpipes and hose connections for manual firefighting remain functional following an SSE. Provisions are made to supply water to at least two standpipes and hose connections for manual firefighting in areas containing equipment required for safe plant shutdown in the event of an SSE. The piping system serving such hose stations is analyzed for SSE loading and is provided with supports to ensure system pressure integrity. The piping and valves for the portion of the hose standpipe system affected by this functional requirement, as a minimum, satisfy ASME B31.1 requirements..

All rooms within the plant buildings are within the reach of at least one effective hose stream from a Class III hose station. A hose station covers each room not covered by a fixed fire suppression system. Effective hose streams from two separate hose stations cover each room that contains equipment required for safe shutdown that is not protected by a fixed fire suppression system. The need for two hose station coverage is also based upon the fire hazard present. Rooms not covered by a fixed fire suppression system and with coverage by only one hose station are furnished with portable fire extinguishers for secondary coverage.

Hose stations also provide secondary coverage for fixed suppression systems.

Hose stations are located outside of highly radioactive areas where possible; however, hose stations are located such that any location that contains or could present a hazard to safety-related equipment can be reached by at least one effective hose stream with a maximum of 30.5 meters (100 feet) of hose.

Fixed fog hose nozzles protecting high-voltage electrical equipment rooms preclude electrical shock hazards with ON/OFF isolation valves. Adjustable fog and straight stream nozzles are provided for all hose stations located away from high-voltage electrical equipment.

**Fire Extinguishers**

Portable multipurpose Class ABC dry chemical-type fire extinguishers are provided for general hazard areas throughout the buildings in the plant. Portable Carbon Dioxide Class BC fire extinguishers are provided for electrical areas. Special use portable extinguishers are provided based upon the hazard present. The fire extinguishers are located in accordance with NFPA 10.

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## **Water Spray for Charcoal Filters**

Charcoal filters in the off-gas and ventilation systems of the plant are provided with water spray systems for fire protection. Water is supplied to the charcoal by means of fixed piping terminating at the exterior of the equipment assembly with manual shutoff valves. In the event of charcoal ignition, the piping can be connected to the firewater supply system through a standard hose or jumper fitting.

### ***9.5.1.7 Fixed Automatic Water Extinguishing Systems***

The selection of specific types of fire suppression systems and the areas requiring protection are based on equipment arrangements and combustible loading in each fire area.

Sprinkler piping for safety-related areas meets the requirements of NFPA 13 and Seismic Category I criteria (assurance that any failure of FPS piping caused by an earthquake does not damage a safety-related item).

### **Wet Pipe Sprinkler System**

Automatic sprinklers provide protection for the areas identified in the fire hazard analysis, except where conditions dictate the use of other types of systems or fire suppression agents.

Each system consists of an outside screw and yoke valve with a position switch, an alarm check valve assembly, piping network containing water under pressure, and thermally actuated closed-head sprinklers. Water discharges immediately from sprinklers opened by heat from a fire. The wet pipe sprinkler system meets the requirements of NFPA 13.

### **Preaction Sprinkler System (Manual or Automatic)**

The preaction sprinkler system employs thermally actuated closed-head sprinklers attached to piping containing air under pressure. Fire detectors are installed in the same area as the sprinklers. A preaction system is used in areas where there is danger of serious water damage as a result of a leaking automatic sprinkler head, spurious actuation, or a pipe break. The preaction sprinkler system meets the requirements of NFPA 13.

### **Deluge System (Manual and/or Automatic)**

Deluge systems are actuated automatically or manually depending upon the area and the nature of the equipment being protecting. Deluge systems employ open head sprinklers or nozzles attached to dry piping and water supply. Fire detectors are installed to cover the protected equipment or are in the same area as the nozzles. Deluge systems are normally used for hazards requiring an immediate application of water over an entire hazard area. The deluge system meets the requirements of NFPA 15.

### ***9.5.1.8 Foam System***

A foam system is provided for concentrated fuel oil or lube oil hazards.

A combination of infrared/ultraviolet detectors and thermal rate compensated detectors are used for fire detection that actuates preaction foam-water sprinkler or deluge foam-water spray systems to protect against inadvertent actuation.

The foam systems meet the requirements of NFPA 11 and NFPA 16.

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**9.5.1.9 Smoke Detection and Fire Alarm System**

Smoke detectors installed in rooms containing safety-related equipment, except primary containment, and in areas containing significant amounts as determined by the FHA (Appendix 9A) of combustible materials provide early detection and warning of fires. Primary containment is an inerted space during normal operation. Portable fire detection equipment is used inside primary containment during maintenance outages when the space is not inerted.

Fire detector types are selected on the basis of the nature and burning characteristics of the materials within the room, and on the basis on the FHA (Appendix 9A). The selection of the detector type also includes specific consideration of the combustion products, type of hazard, and fire load expected as well as the effects of humidity, air velocity, temperature, ambient air conditions, radiation, air pressure, area configuration, and response time required. The detector locations are based upon environmental conditions, maintenance access, and As Low as Reasonably Achievable (ALARA) concerns.

A minimum of two detectors is installed in any single room containing safety-related equipment.

Smoke detection systems for early warning and annunciation of fire conditions are separate from fire detection and releasing devices for suppression system actuation.

Local fire alarm panels continuously monitor the zone and area detection systems and circuits. Upon receipt of an indication of fire from any of the area smoke detectors, the local fire alarm panel activates a visual and audible fire alarm at the panel.

All fire and smoke detection circuits are electrically supervised to detect circuit breaks, ground faults, and power failure. The detector circuits are designed so that the failure, removal, or replacement of a detector does not affect the performance of the fire detection loop. Fire or trouble alarms register on an audible-visual annunciator on a MFAP in the MCR.

Preaction fire detection systems have 90-hour (minimum) backup battery packs located at the local fire alarm panel (releasing panel). Other fire detection and alarm systems have 24-hour (minimum) backup battery packs located at each local fire alarm panel (supervisory panel) and at the MFAP.

Manual fire alarm stations (pull box stations) are provided at the normal exit paths or every 61 meters (200 feet), whichever is less, throughout normally occupied buildings. Manual fire alarm stations (pull box stations) are provided at normal exit paths only for normally unoccupied buildings. Visible fire notification is provided in manned office areas such as in the Control Building and the Service Building.

**9.5.1.10 Fire Barriers**

Fire barriers of 3-hour fire resistance rating are provided separating:

- Safety-related systems from any potential fires in nonsafety-related areas that could affect the ability of safety-related systems to perform their safety function;
- Redundant divisions or trains of safety-related systems from each other to prevent damage that could adversely affect a safe shutdown function from a single fire;
- Components within a single safety-related electrical division that present a fire hazard to components in another safety-related division; and

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- Electrical circuits (safety-related and nonsafety-related) whose fire-induced failure could cause a spurious actuation that could adversely affect a safe shutdown function.

Penetrations through fire barriers are sealed or closed to provide fire resistance ratings at least equal to that of the barrier. Only noncombustible materials qualified per ASTM E-119 are used for construction of fire barriers. Fire dampers protect ventilation duct openings in fire barriers as required by NFPA 90A.

Fire barrier separation of electrical circuits (safety-related and nonsafety-related) whose fire-induced failure could cause a spurious actuation that could prevent safe shutdown per NRC Regulatory Guides 1.75 and 1.189, GDC 17 and 18, and IEEE Standard 384 is described in detail in DCD Tier 2 Subsection 8.3.1.4.1.

The COL applicant shall provide specific design and certification testing details for fire barriers and electrical raceway fire barrier systems in accordance with applicable sections of NFPA 251, ASTM E-119 and guidance in Regulatory Guide 1.189.

**9.5.1.11 Building Ventilation**

Fire protection/smoke control provisions for ventilation for the various building areas are designed as follows:

Smoke control in accordance with NFPA 92A guidelines is provided for unsprinklered areas where the FHA identifies a potential for heavy smoke or heat conditions. Additionally, safe egress and safe smoke refuge areas during a fire incident are provided in accordance with NFPA 92A guidelines for building occupants and the fire brigade. NFPA 101 guidelines are utilized for the design and labeling of safe egress routes.

Smoke removal meets NFPA 804 with exception to NFPA 804 Sections 8.4.3 (3) and 8.4.3.2. Automatic sprinkler protection is provided for the high density cable tunnels, fuel oil tank rooms, diesel-generator rooms and a significant portion of the turbine building to limit heat and smoke generation. The COL Applicant will establish provisions for manual smoke control by manual actions of the fire brigade for all plant areas in accordance with NFPA 804 guidelines.

**Control Building (CB) Smoke Removal**

The CB HVAC System (CBVS) provides smoke removal through two CBVS subsystems: Control Room Habitability Subsystem (CRHAVS) and Control Building General Area HVAC Subsystem (CBGAVS). Fire-rated penetration seals and smoke dampers are provided to prevent smoke and hot gases from migrating into other fire areas.

**Control Room Habitability HVAC Area Subsystem (CRHAVS)**

The MCR is separated from the rest of the Control Building by a 1-hour fire barrier and separated from other major plant areas by 3-hour fire barriers.

Manual fire fighting capability in the MCR consists of portable dry Class ABC chemical fire extinguishers. Additionally, hose stations with UL-approved fixed fog nozzles are installed outside both entrances to the MCR. No hose stations are located within the MCR.

The MCR is provided with smoke detectors that actuate audible and visible alarms on the MFAP in the MCR.

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Smoke detection capability in the CRHAVS automatically detects and annunciates the presence of smoke. Upon receipt of the outside air intake smoke alarm, the CRHAVS automatically isolates and is placed in full recirculation mode to isolate the MCR from the outside air. To purge smoke from the MCR when there is no smoke in the outside air intake, the CRHAVS is placed in the smoke removal mode, in which the MCR air recirculation path is closed, 100% outside air flows through the MCR, the return/exhaust fan is started, and the exhaust dampers are checked for an open path. The normal ventilation return/exhaust fans are used for smoke removal. The MCR operators manually initiate the smoke removal operating mode of the CRHAVS.

Safe shutdown of the plant is not dependent on the operation of the CRHAVS. Two remote shutdown stations are provided inside the Reactor Building, which is serviced by a separate ventilation system. Plant shutdown and maintenance of shutdown conditions may be accomplished from these stations.

**Control Building General Area HVAC Subsystem (CBGAVS)**

The smoke removal mode of the CBGAVS is manually initiated from the MCR by closing the recirculation damper, de-energizing the normal exhaust fans, and energizing the smoke exhaust fan after its isolation dampers are opened to allow outside air purging of the affected room(s).

**Reactor Building (RB) Smoke Removal**

The RB HVAC System (RBVS) provides smoke removal through three RBVS subsystems: Reactor Building Clean Area HVAC Subsystem (CLAVS), Reactor Building Contaminated Area HVAC Subsystem (CONAVS), and the Refueling and Pool Area HVAC Subsystem (REPAVS).

**Reactor Building Clean Area HVAC Subsystem (CLAVS)**

The CLAVS provides smoke removal in the clean areas of the Reactor Building. The CLAVS is a recirculation-type ventilation system that uses minimal outside air for fresh air makeup for exhaust fans. Fire, in any of the areas served by CLAVS, is isolated by the closure of the fire dampers in the supply and return air ducts serving the fire area. Area fire detectors annunciate the fire condition in the MCR. After the fire is extinguished, smoke is removed by placing the CLAVS in the smoke removal mode, which closes the recirculation damper, stops the exhaust fan, opens the exhaust fan bypass damper, and starts the smoke removal fan. This changes the HVAC operation from recirculating to once through and vents smoke and heat from the clean areas to the outside atmosphere.

**Reactor Building Contaminated Area HVAC Subsystem (CONAVS)**

The CONAVS provides smoke removal in the potentially contaminated areas of the Reactor Building. The CONAVS is a once-through ventilation system. In areas served by CONAVS, a fire is isolated by the closure of the fire dampers in the supply and return air ducts serving the fire area. Area fire detectors annunciate the fire in the MCR. After the fire is extinguished, CONAVS provides smoke removal by exhausting air through the exhaust fan to the outside atmosphere through the plant vent stack and by drawing in fresh air from outside. During a radiological event, the exhaust airflow is diverted to the purge exhaust filter unit to remove airborne contamination prior to discharge through the plant stack.

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Fire on the refueling floor or pool areas is annunciated in the MCR. Fire dampers in the supply and return ducts close during a fire. After the fire is extinguished, smoke is removed by placing the REPAVS in the smoke removal mode. REPAVS provides smoke removal by exhausting air through the exhaust fan to the outside atmosphere through the plant vent stack and by drawing in fresh air from outside. During a radiological event, the exhaust airflow is diverted to the purge exhaust filter unit to remove airborne contamination prior to discharge through the plant stack.

**Fuel Building (FB) Smoke Removal**

The FB HVAC System provides smoke removal for the Fuel Building. Following a recovery from a fire, the exhaust fans remove smoke from the area by exhausting air to the outside atmosphere.

**Electrical Building (EB) Smoke Removal**

The EB HVAC System provides smoke removal for the Electrical Building. The smoke removal mode of the EB HVAC System provides smoke removal from the diesel generator engine rooms and diesel generator day tank rooms.

**Turbine Building (TB) Smoke Removal**

The TB HVAC System provides smoke removal for the Turbine Building. A fire in the Turbine Building is annunciated in the MCR. Fire dampers in the supply and exhaust ducts close to preclude the spread of a Turbine Building fire to other areas. Following fire suppression in the Turbine Building, fire dampers are opened as appropriate for smoke removal. The smoke removal mode of the TB HVAC System provides smoke removal from the Turbine Building areas. Turbine Building Exhaust (TBE) fans can be operated to expedite smoke removal as described in Subsection 9.4.4.

**9.5.1.12 Safety Evaluation**

The FHA, contained in Appendix 9A, demonstrates the adequacy of the ESBWR fire protection design to provide the required protection in the event of a postulated fire.

The methodology for performing the FHA consistent with the level of ESBWR design completion is described in the FHA.

The FHA includes the following information:

- Fire Protection System description;
- Comparison of the ESBWR Fire Protection Program with NRC Branch Technical Position SPLB 9.5-1, demonstrating conformance of the ESBWR program and design with the guidance in the Branch Technical Position;
- Methodology for evaluation of potential fire hazards; and
- Safe shutdown analyses on a fire area by fire area basis.

Included in the above are complete descriptions of the fire areas, fire loadings in the areas, and fire detection and suppression capabilities provided in each area.

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The COL licensee referencing the ESBWR Standard Plant conducts a compliance review of the as-built design against the assumptions and requirements stated in the FHA. Based on this review, the FHA is updated as necessary.

The ESBWR design satisfies the following guidance from the NUREG-0800 SRP 9.5.1 and BTP SPLB 9.5-1:

**Guidance** — “Therefore, the designers of standard plants have been informed that they must demonstrate that safe shutdown of their designs can be achieved, assuming that all equipment in any one fire area has been rendered inoperable by fire and that reentry to the fire area for repairs and for operator actions is not possible. The control room should be excluded from this approach, subject to the need for an independent alternate shutdown capability that is physically and electrically independent of the control room.”

**Conformance** — The design of the fire barrier system and safe shutdown systems for the ESBWR are such that complete burnout of any single fire area without recovery does not prevent safe shutdown of the plant.

Safe shutdown is achieved primarily by using the Isolation Condenser System (ICS), described in Subsection 5.4.6. This is a system employed for both hot standby and long-term core cooling modes, which can operate at full reactor coolant system pressure, and is thereby able to place the reactor in the long-term cooling mode immediately after reactor shutdown. Operation of the plant in the long-term cooling mode is automatic. The system does not require any ac power or other support systems such as cooling water. Operation does not require any pumps or valve operation once initial alignment is established.

The system initiation is based on a two-out-of-four logic. Actuation does still occur with one division failed due to a fire.

A Remote Shutdown System is provided to ensure safe shutdown capability. The Remote Shutdown System is physically and electrically independent of the MCR.

**Guidance** — “Fire protection for redundant shutdown systems in the reactor containment building should ensure, to as great an extent as possible, that one shutdown division is free of damage.”

**Conformance** — The ESBWR design conforms to this guidance in that the relevant redundant systems are separated by fire barriers, or, in the case of the primary containment, as much separation as possible and by inerting the containment atmosphere during operation to preclude the initiation or propagation of a fire.

**Guidance** — “Consideration should be given for safety-grade provisions for the fire protection systems to ensure that the remaining shutdown capabilities are protected.”

**Conformance** — Fire protection piping and associated components that could become a hazard during an earthquake are seismically supported. Manual suppression capability for safe shutdown equipment areas remains functional following an earthquake. The fire barriers in safety-related areas of buildings are Seismic Category I.

**Guidance** — “In addition, it should be demonstrated that smoke, hot gases, or the fire suppressant does not migrate into other fire areas to the extent that safe shutdown capabilities, including operator actions, could be adversely affected.”

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**Conformance** — The ESBWR fire protection design satisfies this guidance with a combination of fire dampers and other barriers, smoke evacuation capabilities, and minimal required operator actions. Details are provided in the fire hazards analysis in Appendix 9A.

#### 9.5.1.12.1 Design Exceptions

The ESBWR fire protection design follows the recommendations of BTP SPLB 9.5-1 or Regulatory Guide 1.189 with the following exceptions:

##### 9.5.1.12.1.1 No Fire Detection within Electrical Cabinets in Main Control Room Complex

Section C.7.1.4 of BTP SPLB 9.5-1 recommends that electrical cabinets should be protected as described in Regulatory Guide 1.189. Section 6.1.2.2 of Regulatory Guide 1.189 states in part:

"Smoke detectors should be provided in the control room, cabinets, and consoles."

ESBWR consoles and electrical cabinets do not have fire detectors installed inside them, unless identified as a significant fire hazard in the Fire Hazards Analysis.

Justification: The electrical cabinets and consoles contain limited combustibles and are air-cooled so that smoke from an interior fire will exhaust to the room. Early warning fire detection, primarily consisting of ionization smoke detectors, is provided in all rooms containing consoles or electrical cabinets. A fire in any single cabinet or console will not disable the capability to safely shut down the plant. Except in the Main Control Room Complex, all safety-related electrical cabinets and consoles are located in divisional rooms, and all divisional rooms are separated from each other by 3-hour fire-rated barriers such that a single fire will not affect electrical cabinets or consoles from multiple divisions. The Main Control Room Complex is continuously manned so that any fire will be quickly detected and manual fire suppression activities would be initiated quickly upon discovery of a fire. In the unlikely event that a fire in the Main Control Room were to require evacuation, use of either the Division I or Division II Remote Shutdown Panels (located remotely from Main Control Room, in the Reactor Building) enable the operators to bring the reactor to a safe shutdown.

##### 9.5.1.12.1.2 No Automatic Fire Suppression in Office Areas of Main Control Room Complex

Section C.8.1.2.c of BTP SPLB 9.5-1 recommends that automatic suppression capability should be provided in the Control Room Complex as described in Regulatory Guide 1.189. Section 6.1.2 of Regulatory Guide 1.189 states in part:

"Peripheral rooms in the control room complex should have automatic water suppression..."

The office spaces contained in the ESBWR Main Control Room Complex do not have automatic fire suppression systems installed, unless identified as a significant fire hazard in the Fire Hazards Analysis.

Justification: The Main Control Room Complex is considered to be a low risk fire area, due to the lack of high- or medium-voltage equipment or cabling. Interior finishing materials within the Main Control Room Complex are noncombustible or have a flame spread and smoke developed rating of 25 or less. The amount of transient combustibles within this fire area is limited. Papers within the Main Control Room Complex are stored in file cabinets, bookcases, or other storage locations except when in use. Ionization or photoelectric smoke detectors are installed throughout the Main Control Room Complex to provide early warning of fire during the

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incipient stage. The Main Control Room Complex is continuously manned so that any fire will be quickly detected and manual fire suppression activities would be initiated quickly upon discovery of a fire. Should manual fire fighting in the Main Control Room Complex be necessary using either portable fire extinguishers or hand held fire hoses, accumulation or drainage of firewater will not affect the ability to safely shutdown the reactor. If the firewater is assumed to transport immediately to the basement of the Control Building, the resulting accumulation of water will not affect safety-related equipment located in the basement. In either case, the fire fighting activities will not prevent the reactor from being safely shutdown.

Finally, in the unlikely event that a fire in the Main Control Room were to require evacuation, use of either the Division I or Division II Remote Shutdown Panels (located remotely from Main Control Room, in the Reactor Building) enable the operators to bring the reactor to a safe shutdown.

#### **9.5.1.12.1.3 No Automatic Fire Suppression Below Raised Floor in Main Control Room Complex**

Section C.8.1.2.c of BTP SPLB 9.5-1 recommends cable raceways under raised floors should be reviewed to determine if adequate fire detection and suppression are provided for potential fires in these areas. Section 6.1.2.1 of Regulatory Guide 1.189 states in part:

"...Fully enclosed electrical raceways located in under-floor and ceiling spaces, if over 0.09 m<sup>2</sup> (1 sq ft) in cross-sectional area, should have automatic fire suppression inside."

The Main Control Room Complex has a raised floor over a subfloor volume, which is used for routing of cables between the electrical cabinets, control panels, computer equipment, and the divisional electrical rooms. Divisional separation of the subfloor cabling is maintained per the requirements of IEEE 384. The subfloor volume includes full fire detection but does not include any automatic fire suppression system, unless identified as a significant fire hazard in the Fire Hazards Analysis.

Justification: The Main Control Room Complex and subfloor volume is considered to be a low risk fire area, due to the lack of high- or medium-voltage equipment or cabling. The characteristics of the subfloor cables are such that the probability of a fire ignition is very low and any fire that were to occur would be self-extinguishing or very slow to spread. There are no transient combustibles stored in the subfloor volume during normal activities. Ionization smoke detectors are installed throughout the subfloor volume to provide early warning of fire during the incipient stage. The raised floor consists of noncombustible sectional panels that can be individually removed to provide fire-fighting access to a subfloor fire. Because the Control Room is continuously manned, manual fire suppression activities would be initiated quickly upon discovery of a fire in the subfloor volume. Since fire resistant cables are used, the amount of water needed to extinguish a fire within the subfloor volume is relatively small. Any water that is introduced into the subfloor volume can be removed by floor drains in the subfloor volume or through the use of temporary portable sump pumps. Accumulation of water in the subfloor volume is limited in depth to less than the raised floor height and will not adversely effect water sensitive safety-related equipment, which is installed above the raised floor. Effectiveness of a permanently installed fire suppression system within the subfloor volume may be somewhat limited due to the relatively small height between raised floor and top of cabling, as well as physical barriers within the subfloor volume to meet IEEE 384 separation criteria. Not including automatic fire suppression within the subfloor volume has the indirect benefit of

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avoiding the potential for missiles (from gaseous suppression cylinders) or flooding/wetting (from water piping) during maintenance or testing activities to impact safety-related equipment within the Main Control Room Complex.

Finally, in the unlikely event that a fire in the Main Control Room were to require evacuation, use of either the Division I or Division II Remote Shutdown Panels (located remotely from Main Control Room, in the Reactor Building) enable the operators to bring the reactor to a safe shutdown.

**9.5.1.12.1.4 Diesel Day Tank Capacity within Building**

Section C.8.1.8.b of BTP SPLB 9.5-1 recommends that diesel day tanks comply with Regulatory Guide 1.189. Section 6.1.8 of Regulatory Guide 1.189 states in part:

"Day tanks with total capacity up to 4164 L (1100 gallons) may be located in the diesel generator area under the following conditions:

- The day tank is located in a separate enclosure with fire resistance rating of at least 3 hours.

Based on the size of the nonsafety-related diesel generators, the capacity of each of the diesel day tanks will likely exceed 4164 L (1100 gallons) to allow enough fuel for at least 8 hours of diesel operation at the maximum load demand.

Technical Justification: The ESBWR design includes two independent and physically separated nonsafety-related diesel generators, either of which is capable of providing the full electrical load for the redundant nonsafety-related electrical buses. Neither diesel generator is necessary to achieve and maintain safe shutdown conditions for the 72-hour period following an accident or fire event. Each day tank is located in the Electrical Building in a dedicated 3-hour fire rated compartment of masonry or concrete construction. There is no safety-related equipment located in the same building as the day tank rooms. Additionally, the day tank rooms are located in individual fire areas adjacent to the Diesel Generator (DG) rooms and are positioned such that the 3 hour fire rated walls, ceiling, and floor of the day tank room are not common to the other redundant DG.

Each day tank room is protected by a foam water deluge system that can deliver foam to the room for a minimum of 30 minutes without operator intervention. The day tank is seismically designed and supported. Potential ignition sources, with enough energy to ignite diesel fuel, are limited inside the day tank rooms. Furthermore, the supply of fresh air to support combustion is limited. In the event of a fire, the automatic foam water deluge system is designed to extinguish a fire in this room in 10 minutes. In the unlikely event the day tank were to fail, the entire contents of the day tank plus the foam water volume can be contained in the sunken volume of the day tank room. Additional foam capacity beyond 10 minutes provides added assurance that a postulated fire will be extinguished. In the unlikely event the fire cannot be extinguished, the day tank room can be isolated by closing doors and dampers to allow the fire to burn out on its own without spreading to other fire areas.

In the event that the fuel oil transfer line from the day tank to the DG were to fail outside of the day tank room, the curbed area within the DG room can accommodate the contents of the day tank plus the foam water volume applied by the preaction foam water automatic sprinkler system. This automatic sprinkler system is designed to extinguish a fire within the DG room

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within 10 minutes. In the unlikely event the fire is still not extinguished, the DG room can be isolated by closing doors and dampers to allow the fire to burn out on its own without spreading to other fire areas. Alternatively, if the fire brigade is required to fight the fire manually, the curbed area within the DG room can accommodate additional water/foam application from two hand-held foam hose lines before reaching the lowest door opening. The lowest door opening to these rooms are the exterior equipment doors which could be opened if fire fighting activities necessitate so that any overflow would spill outside the building and not spread to other parts of the electrical building. Therefore, any overflow from the sump area of the room will not affect any other equipment, nor will it affect safe shutdown equipment or equipment needed for support of safe shutdown equipment.

**9.5.1.12.1.5 Allowing Continued Diesel-Generator Operation During a Fire**

Section 8.1.8.c of BTP SPLB 9.5-1 recommends that impacts of suppression systems on operating generators should be addressed in the fire hazard analysis. Section 6.1.8 of Regulatory Guide 1.189 states in part:

“Automatic fire suppression should be installed to suppress or control any diesel generator or lubricating oil fires. Such systems should be designed for operation when the diesel is running without affecting the diesel.”

The automatic sprinkler systems in the diesel generator rooms are installed to extinguish any fire in those rooms and do not place restrictions on the positioning and direction of the application of the fire suppressant.

Justification: The automatic sprinkler systems used in the diesel generator rooms are designed to prevent inadvertent actuation by utilizing preaction automatic sprinkler type. The sprinkler piping and closed head sprinklers are pneumatically supervised for leakage, and any inadvertent actuation of the deluge valve during testing or maintenance will not result in unintentional water release due to the normally closed sprinkler heads.

Two actuation signals are required to automatically actuate the deluge valve, the first of which will annunciate an alarm to alert the operators to any potential problems. Automatic actuation of the sprinkler system to release water requires three independent events: 1) detection of a specific range of infrared wavelengths, consistent with burning oil, by at least one infrared detector; 2) detection of a significant heat release by at least one heat detector; and, 3) opening of at least one fusible link sprinkler head. Furthermore, each redundant diesel generator has its own dedicated fire detectors and preaction deluge valve for the control of the fire sprinklers in that room, and loss of power to the deluge valve does not cause actuation.

The ESBWR design includes two independent and physically separated nonsafety-related diesel generators, either of which is capable of providing the full electrical load for the redundant nonsafety-related electrical buses. Neither diesel generator is necessary to achieve and maintain safe shutdown conditions for the 72-hour period following an accident or fire event. The ESBWR design also includes four independent and physically separated safety-related divisions, any two of which are capable of bringing the plant to a safe shutdown in the event of a fire. For design purposes, it is assumed that a fire anywhere in a fire area results in the immediate loss of function of all equipment associated with that division. Even with this conservative assumption,

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the remaining independent safety-related divisions are available for full utilization by the operators.

**9.5.1.12.1.6 No Automatic Fire Suppression in Safety-Related Computer Rooms**

Section 8.1.4 of SPLB BTP 9.5-1 recommends protecting computer rooms with fire protection systems as described in Regulatory Guide 1.189. Section 6.1.4 of Regulatory Guide 1.189 states in part:

“Computer rooms for computers performing functions important to safety that are not part of the control room complex should be separated from other areas of the plant by barriers having a minimum fire resistance rating of 3 hours and should be protected by automatic detection and fixed automatic suppression.”

ESBWR computer rooms containing safety-related equipment do not have fire suppression installed inside them.

Justification: Computer rooms are considered to be low risk fire areas, due to the lack of high- or medium-voltage equipment or cabling. Interior finishing materials within computer rooms are noncombustible. The amount of transient combustibles within computer rooms is limited. Papers within computer rooms, if any, are stored in file cabinets, bookcases, or other storage locations except when in use.

Ionization smoke detectors are installed throughout computer rooms to provide early warning of fire during the incipient stage. The Main Control Room Complex is continuously manned so that any fire will be quickly detected and manual fire suppression activities would be initiated quickly upon discovery of a fire in a computer room. Should manual fire fighting in a computer room be necessary using either portable fire extinguishers or hand held fire hoses, accumulation or drainage of firewater will not affect the ability to safely shutdown the reactor. If the firewater is assumed to transport immediately to the basement of the building, the resulting accumulation of water will not affect safety-related equipment located in the basement. In either case, the fire fighting activities will not prevent the reactor from being safely shutdown.

Except in the Main Control Room Complex, all safety-related computers are located in divisional rooms, and all divisional rooms are separated from each other by 3-hour fire-rated barriers such that a single fire will not affect computer equipment from multiple divisions. In the unlikely event that a fire in the Main Control Room were to require evacuation, use of either the Division I or Division II Remote Shutdown Panels (located remotely from Main Control Room, in the Reactor Building) enable the operators to bring the reactor to a safe shutdown.

**9.5.1.13 Inspection and Testing Requirements**

Preoperational inspection and testing requirements for each fire protection system are described in Section 14. Periodic inspection and testing to assure system operability is conducted in accordance with applicable codes and approved procedures.

**9.5.1.14 Instrumentation Requirements**

Controls and instrumentation are provided for a fully functioning system. There are three main types of FPS instrumentation: instrumentation supporting fire detection, instrumentation supporting automatic suppression systems, and instrumentation supporting firewater delivery.

**ESBWR****Instrumentation for the Fire Detection System**

Instrumentation for the fire detection system provides signals for early detection and warning of fires. Local fire alarm panels per NFPA 72 supervise fire and smoke detectors. The local fire alarm panels are in turn connected to the alarm MFAP via a dedicated data link. Signals transmitted include detector status (normal, alarm, supervisory, trouble) as well as local fire alarm panel status.

Upon receipt of a signal from any of the area fire detectors, audible and visual annunciation is activated at the MFAP in the MCR and at the local fire alarm panel.

Instrumentation for fire detection is either FM approved or UL listed, where available.

**Instrumentation Supporting Fire Suppression Systems**

Each fire suppression system automatically actuated by a fire detection system has the control logic and capability for manual actuation available at the local fire alarm panel for the protected area. Remote manual actuation of these suppression systems is also available from the MCR. Automatic sprinkler systems that do not require separate detection systems for actuation are not equipped with manual actuation means.

Instrumentation for fixed fire suppression systems provides local and remote monitoring capability for the suppression system status. All instruments for automatic suppression systems are wired to the local fire alarm panels for control. Dedicated data links transmit command and status information to and from the local fire alarm panels and the MFAP in the MCR.

All instrumentation for automatically actuated fire suppression systems is either FM approved or UL listed, where available.

**Instrumentation Supporting Firewater Delivery**

Instrumentation supporting firewater delivery provides status indication of firewater storage level, firewater main pressure, primary and secondary jockey pump status, and primary and secondary fire pump status conditions.

The primary motor-driven fire pump is designed to start first, if the primary jockey pump cannot maintain pressure in the system. The secondary motor-driven fire pump is set to initiate second if the secondary jockey pump cannot maintain pressure in the system. The primary Seismic Category I diesel-driven fire pump is set to initiate third and the secondary NS diesel-driven fire pump is set to initiate last if the other fire pumps fail to start or cannot maintain the required pressure. The starting of any fire pump shall depend on the location of the firewater demand. All fire pumps are stopped manually. Any fire pump can be started manually from the MFAP in the MCR or locally.

Pressure instrumentation is used to automatically start and stop the primary and secondary motor-driven jockey pumps.

**9.5.1.15 Fire Protection Program**

The ESBWR Fire Protection Program is established to ensure that a fire will not prevent safe shutdown of the plant uses not endanger the health and safety of the public. Fire protection at the plant uses a defense-in-depth concept that includes fire prevention, detection, control and extinguishing systems and equipment, administrative controls, procedures, and trained personnel.

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The COL applicant shall provide a complete description of the applicant's fire protection program.

**9.5.1.15.1 Fire Protection Program Criteria**

The ESBWR Plant Fire Protection Program is based on the criteria of several industry and regulatory documents that are referenced in Table 9.5-1.

**9.5.1.15.2 Organization and Responsibilities**

The organizational staffing structure of the Fire Protection Program is discussed in Chapter 13.1.

The site engineer in charge of the Fire Protection Program is responsible for the following:

- Ensuring that programs and periodic inspections are implemented to;
  - Minimize the amount of combustibles in safety-related areas; and
  - Determine the effectiveness of housekeeping practices.
- Assure the availability and acceptability of the following;
  - Fire Protection System and components;
  - Manual fire fighting equipment;
  - Emergency breathing apparatus;
  - Emergency lighting;
  - Communication equipment;
  - Fire barriers including fire rated walls, floors and ceilings, fire rated doors, dampers, etc., fire stops and wraps, and fire retardant coatings. Procedures specifically address the administrative controls to be put in place, including fire watches, when a fire barrier is breached for maintenance; and
  - Assure prompt and effective corrective actions are taken to correct conditions adverse to fire protection and preclude their recurrence.
- Ensuring that periodic maintenance and testing of fire protection systems, components, and manual fire fighting equipment is conducted, test results are evaluated, and the acceptability of systems under test is determined in accordance with established plant procedures;
- Designing and selecting equipment related to Fire Protection;
- Reviewing and evaluating proposed work activities to identify potential transient fire loads;
- Managing the Plant Fire Brigade, including;
  - Developing, implementing and administering the Fire Brigade Training Program;
  - Scheduling and conducting fire brigade drills;
  - Critiquing fire drills to determine how well training objectives are met;

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- Performing a periodic review of the fire brigade roster and initiating changes as needed;
  - Maintaining the fire training program records for members of the fire brigade and other personnel; and
  - Ensuring that sufficient fire brigade personnel are identified at the beginning of each shift.
- Developing and conducting the Fire Extinguisher Training Program;
  - Implementing a program for indoctrination of personnel gaining unescorted access to the protected area in appropriate procedures which implement the fire protection program, such as fire prevention and fire reporting procedures, plant emergency alarms, including evacuation;
  - Implementing a program for instruction of personnel on the proper handling of accidental events such as leaks or spills of flammable materials;
  - Preparing procedures to meet possible fire situations in the plant and for assuring assistance is available for fighting fires in radiological areas;
  - Implementing a program that utilizes a permit system that controls and documents inoperability of fire protection systems and equipment. This program also initiates proper notifications and compensatory actions, such as fire watches, when inoperability of any fire protection system or component is identified;
  - Developing and implementing preventive maintenance, corrective maintenance, and surveillance test fire protection procedures;
  - Ensuring plant modifications, new procedures and revisions to procedures associated with fire protection equipment and systems that have significant impact on the Fire Protection Program are reviewed by an individual who possesses the qualifications of a fire protection engineer. The COL applicant shall provide the proposed fire protection license condition for making changes to the fire protection system without prior review and approval of the NRC.
  - Ensuring a continuing evaluation of fire hazards during construction or modification of other units on the site. Additional fire barriers, fire protection capability and administrative controls are provided as necessary to protect the operating unit(s) from construction or modification activities.

**9.5.1.15.3 Fire Protection Program Staffing Requirements**

Fire Protection Staffing requirements are described in Section 13.1.

**9.5.1.15.4 Fire Brigade**

The COL applicant shall provide provisions for manual fire-fighting capability for all plant areas.

**9.5.1.15.4.1 General**

The Fire Brigade organization is discussed in Subsection 13.1.2.1.5.

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**9.5.1.15.4.2 Fire Brigade Training**

Fire protection training consists of training in three specific areas:

- Employees designated to be members of the station fire brigade;
- Employees assigned to the fire protection staff; and
- Offsite fire departments.

Specific training requirements for each of the above categories of personnel are described in the following sections.

Personnel assigned as fire brigade members receive formal training prior to assuming brigade duties. The course subject matter is selected to satisfy the requirements of Regulatory Guide 1.189. In addition, course material selection also includes guidance from NFPA Codes 600 and 1500 as appropriate. Additional training may also include material selection from NFPA 1404 and 1451.

Course material includes the following classroom instruction:

- Chemistry of fire;
- Classification of fires and principles of extinguishment;
- Fire prevention and inspection techniques;
- Fire protection systems;
- Radiological safety aspects of fires at nuclear facilities;
- Indoctrination of plant firefighting plans with specific identification of individual responsibilities;
- Identification of the type and location of fire hazards and associated types of fires that could occur in the plant;
- The toxic and corrosive characteristics of expected products of combustion;
- Identification of the location of firefighting equipment for each fire area and familiarization with the layout of the plant, including access and egress routes;
- The proper use of available firefighting equipment and the correct method of fighting each type of fire including: fires in energized electrical equipment, fires in cables and cable trays, hydrogen fires, fires involving flammable and combustible liquids or hazardous process chemicals, fires resulting from construction or modifications (welding), and record file fires;
- The proper use of communication, lighting, ventilation, and emergency breathing equipment;
- The proper method for fighting fires inside buildings and confined spaces;
- The direction and coordination of firefighting activities (fire brigade leaders only); and
- Detailed review of firefighting strategies and procedures.

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Field exercises are conducted to reinforce the classroom training and provide an opportunity to practice the skills learned. These exercises include:

- Fighting small fires with portable fire extinguishers;
- Fighting interior fires using breathing apparatus;
- Controlling incidents involving flammable gases or pressurized liquid fuels;
- Fighting large flammable liquid fires using hose lines and/or foam; and
- Fighting flammable liquid fires inside building.

The classroom instruction and field exercises are provided by qualified individuals who are knowledgeable, experienced, and suitably trained in fighting the types of fires that could occur and in using the types of equipment available at the power station.

**9.5.1.15.4.3 Fire Protection Staff Training**

The Fire Protection Engineer responsible for the formulation and implementation of the Fire Protection Program meets the qualification requirements listed in Table 13.1-1.

The station fire protection staff receives training in:

- Design and maintenance of fire detection, suppression, and extinguishing systems;
- Fire prevention techniques and procedures;
- Firefighting techniques and procedures for plant personnel and the fire brigades; and
- Hazardous material identification and handling. Specific courses to achieve the above training objectives are provided for the System Engineers assigned to the fire protection staff if they are not fully trained when hired. Other training organizations may be used to provide this training on a case-by-case basis.

**9.5.1.15.4.4 Offsite Fire Department Training**

Training for offsite fire departments that have agreed to assist during a major fire onsite is provided to make members aware of the need for radiological protection of personnel, the special hazards and operational precautions associated with fire fighting at a nuclear power plant. The course is provided annually and includes instruction in the following:

- Basic radiation protection, including the use of personal dosimetry devices;
- Plant familiarization, including hazards, available fire fighting equipment, and fire protection systems;
- Firefighting procedures; and
- Security procedures, including entry to and exit from the plant.

**9.5.1.15.4.5 Fire Brigade Retraining****Classroom**

Regular planned meetings are held at least once each calendar quarter for brigade members to review changes in the fire protection program and other subjects as necessary. Periodic refresher

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training sessions are held to repeat the classroom instruction program for brigade members over any 2-year period. These sessions may be concurrent with the regular planned meetings.

**Practice**

Practice sessions are held for each shift fire brigade on the proper method of fighting the various types of fires that could occur in a nuclear power plant. These sessions provide brigade members with experience in actual fire extinguishment and the use of emergency breathing apparatus under strenuous conditions encountered in firefighting. These practice sessions are provided at least once per year for each fire brigade member.

**Drills**

Fire brigade drills are conducted in various plant areas, especially in those areas identified by the fire hazards analysis to be critical to plant operation and to contain significant fire hazards. Fire brigade drills are performed in the plant so that the fire brigade can practice as a team. Unannounced drills are in full dress. Regularly scheduled drills are also in full dress. Full dress includes helmet, coat, boots, gloves, and emergency breathing apparatus. Donning of face mask and use of emergency air is not mandatory during drills.

Drills are performed at least once each calendar quarter for each shift fire brigade. The offsite local fire department is invited to participate in at least one drill per year. Each fire brigade participates in at least two drills per year. Critiques are conducted upon completion of each drill to evaluate the effectiveness of brigade performance and incorporate lessons learned into future drill evolutions. Drills include reviews of the latest plant modifications and corresponding changes in firefighting plans.

**9.5.1.15.4.6 Fire Brigade Records**

Individual records of training provided to each fire brigade member, including drill critiques, are maintained as part of the permanent plant files for at least 3 years to ensure that each member receives training in all parts of the training program. Retraining or broadened training for firefighting within buildings is scheduled for all those brigade members whose performance records show deficiencies. A system to document drills including critiques and corrective actions has been developed. Fire brigade training review and individual performance programs also have been developed.

The minimum equipment provided for the ESBWR Plant Fire Brigade consists of personal protective equipment such as turnout coats, boots, gloves, hard hats, emergency communications equipment, portable lights, portable ventilation equipment and portable extinguishers. Self-contained breathing apparatus (SCBA) approved by NIOSH, using full face positive pressure masks, are provided for selected fire brigade, emergency repair and control room personnel. At least ten masks are provided for fire brigade personnel. At least two extra air bottles are located on-site for each SCBA. An additional on-site 6-hour supply of reserve air is provided to permit quick and complete replenishment of exhausted supply air bottles. Additional SCBAs are provided near the personnel containment entrance for the exclusive use of the Fire Brigade. The Fire Brigade Leader has ready access to keys for any locked fire doors.

The on-duty Shift Supervisor has responsibility for taking certain actions based on an assessment of the magnitude of the fire emergency. These actions include safely shutting down the plant, making recommendations for implementing the Emergency Plan, notification of emergency

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personnel and requesting assistance from off-duty personnel, if necessary. Emergency Plan consideration of fire emergencies includes the guidance of Regulatory Guide 1.101.

To qualify as a member of the Fire Brigade, an individual must meet the following criteria:

- Is available to answer fire alarms;
- Has attended the required training sessions for the position he occupies on the Fire Brigade; and
- Has passed an annual physical exam.

**9.5.1.15.5 Administrative Controls**

Administrative controls for the Fire Protection Program are implemented through plant administrative procedures. These procedures are available for review and inspection prior to implementation of the program.

These controls establish procedures to:

- Control actions to be taken by an individual discovering a fire, such as notification to the Control Room, attempting to extinguish the fire, and actuation of local fire suppression systems;
- Control actions to be taken by the Control Room operator, such as sounding fire alarms, and notifying the Shift Supervisor of the type, size and location of the fire;
- Control actions to be taken by the Fire Brigade after notification of a fire, including location to assemble, directions given by the fire brigade leader, the responsibilities of brigade members such as selection of fire fighting and protective equipment and use of preplanned strategies for fighting fires in specific areas;
- Control actions to be taken by the Security force upon notification of a fire; and
- Define the strategies established for fighting fires in safety-related areas and areas presenting a hazard to safety-related equipment, including the designation of the;
  - Fire hazards in each plant zone covered by a fire fighting procedure;
  - Fire extinguishers best suited for controlling fires with the combustible loadings of the zone and the nearest location of these extinguishers;
  - Most favorable direction from which to attack a fire in each area in view of the ventilation direction, access hallways, stairs, and doors that are most likely to be free of fire, and the best station of elevation for fighting the fire. All access and egress routes that involve locked doors are specifically identified in the procedure with the appropriate precautions and methods for access specified;
  - Plant systems that should be managed to reduce the damage potential during a local fire and the location of local and remote controls for such management (e.g., any hydraulic or electrical system in the zone covered by the specific fire fighting procedure that could increase the hazards in the area because of overpressurization or electrical hazards);

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- Vital heat-sensitive system components that need to be kept cool while fighting a local fire. Particularly hazardous combustibles that need cooling are designated;
- Potential radiological and toxic hazards in fire zones;
- Ventilation system operation that ensures desired plant air distribution when the ventilation flow is modified for fire containment or smoke clearing operations;
- Operations requiring Control Room and Operating Supervisor coordination or authorization
- Instructions for plant operators and general plant personnel during a fire;
- Organize the Fire Brigade and assign special duties according to job title so that all fire fighting functions are covered by any complete shift personnel complement. These duties include command and control of the brigade, transporting fire suppression and support equipment to the fire scenes, applying the extinguishing agent to the fire, communication with the Control Room, and coordination with outside fire departments.

#### 9.5.1.15.6 Control of Combustible Materials, Hazardous Materials and Ignition Sources

The control of combustible materials is defined by administrative procedures. Those procedures impose the following controls:

- Prohibit the storage of combustible materials (including unused ion exchange resins) in areas that contain or expose safety-related equipment or establish designated storage areas with appropriate fire protection;
- Govern the handling of and limit transient fire loads such as flammable liquids, wood and plastic materials in the power block;
- Assign responsibility to the appropriate supervisor for reviewing work activities to identify transient fire loads;
- Govern the use of ignition sources by use of a flame permit system to control welding, flame cutting, grinding, brazing and soldering operations, and temporary electrical power cables. A separate permit is issued for each area where such work is done. If work continues over more than one shift, the permit is valid for not more than 24 hours when the plant is operating or for the duration of a particular job during plant shutdown per NFPA 51B and 241.
- Minimize waste, debris, scrap, and oil spills or other combustibles resulting from a work activity in the power block while work is in progress and remove the same upon completion of the activity or at the end of each work shift;
- Govern periodic inspections for accumulation of combustibles and to ensure continued compliance with these administrative controls;
- Prohibit the storage of acetylene-oxygen and other compressed gasses in areas that contain or expose safety related equipment or the fire protection system that serves those areas. A permit system is required to use this equipment in the powerblock.
- Govern the use and storage of hazardous chemicals in the powerblock;

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- Control the use of specific combustibles in the powerblock. All wood used in the power block during maintenance, modification, or refueling operation (such as lay-down blocks or scaffolding) is treated with a flame retardant. Equipment or supplies (such as new fuel) shipped in untreated combustible packing containers may be unpacked in the power block if required for valid operating reasons. However, all combustible materials are removed from the area immediately following unpacking. Such transient combustible material, unless stored in approved containers, is not left unattended in the powerblock during lunch breaks, shift changes, or other similar periods. Loose combustible packing material such as wood or paper excelsior, or polyethylene sheeting is placed in metal containers with tight-fitting self-closing metal covers. Only noncombustible panels or flame-retardant tarpaulins or approved materials of equivalent fire-retardant characteristics are used in the powerblock. Any other fabrics or plastic films used in the powerblock are certified to conform to the large-scale fire test described in NFPA 701.
- Govern the control of electrical appliances in areas that contain or expose safety-related equipment.

#### 9.5.1.15.7 Control of Radioactive Materials

As stated in the Fire Hazards Analysis, (FHA), the primary objectives of a fire protection program are to minimize both the probability of occurrence and the consequences of a fire. To meet these objectives, the fire protection program provides reasonable assurance, through defense in depth, which a fire will not prevent the performance of necessary safe shutdown functions and that radioactive releases to the environment in the event of a fire will be minimized. Table 9A.5-1 specifically addresses the radiological release potential for each defined fire area.

#### 9.5.1.15.8 Testing and Inspection

The Fire Protection System is initially tested in accordance with Chapter 14.

Testing and inspection requirements is implemented through administrative procedures. Post maintenance or modifications testing to the Fire Protection System is subject to review to ensure conformance to design requirements. Installation of portions of the system where performance cannot be verified through post modification tests, such as penetration seals, fire retardant coatings, cable routing, and fire barriers is inspected. Inspections are performed by individuals knowledgeable of fire protection design and installation requirements. Open flame or combustion generated smoke are not be used for leak testing or similar procedures such as air flow determination. Inspection and testing procedures address the identification of items to be tested or inspected, responsible organizations for the activity, acceptance criteria, documentation requirements and signoff requirements.

Fire Protection materials subject to degradation (such as fire stops, seals and fire retardant coatings are visually inspected periodically to assure they are not degraded or damaged. Fire hoses are hydrostatically tested in accordance with NFPA-1962. Hoses stored in outside hose stations are tested annually and interior standpipe hoses are tested every 3 years.

The Fire Protection System is periodically tested in accordance with plant procedures. Testing includes periodic operational tests and visual verification of damper and valve positions. Fire doors and their closing and latching mechanisms are also included in these procedures. Fire

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doors separating safety-related areas are self-closing or provided with closing mechanisms and are inspected semiannually to verify that the automatic hold open, release and closing mechanisms and latches are operable. Watertight and missile resistant doors are not provided with closing mechanisms. Fire doors with automatic hold open and release mechanisms are inspected daily to verify that the doorways are free of obstructions.

Fire doors separating fire areas are normally closed and latched. Fire doors that are locked closed are inspected weekly to verify position. Fire doors that are closed and latched are inspected daily to assure that they are in the closed position. However, fire doors that are closed and electrically supervised at a continuously manned location are not inspected.

**9.5.1.15.9 Quality Assurance**

Quality assurance controls are applied to the activities involved in the design, procurement, installation, testing, and maintenance of fire protection systems for safety-related areas, in accordance with the programs outlined in Chapter 17. The COL applicant shall provide details of the QA program for the fire protection program.

**9.5.1.15.10 Emergency Planning**

Emergency planning is described in DCD Section 13.3.

**9.5.1.16 COL Information**

**The following site specific items shall be determined by the COL applicant:**

- Details of the QA program for the fire protection program;
- Details of manual fire-fighting capability, including smoke control during a fire;
- Complete description of the applicant's fire protection program;
- Proposed fire protection license condition for making changes to the fire protection system without prior review and approval of the NRC;
- Fire protection system piping and instrumentation diagram showing complete site-specific system;
- Compliance review of the as-built design against the assumptions and requirements stated in the FHA;
- Provisions for manual smoke control by manual actions of the fire brigade for all plant areas;
- Final quantity and capacity of secondary firewater storage;
- Final secondary fire pump flow rate and head; and
- Specific design and certification testing details for fire barriers and electrical raceway fire barrier systems in accordance with applicable section of NFPA 251, ASTM E-119 and guidance in Regulatory Guide 1.189.

**9.5.1.17 References**

All applicable references are listed in Table 9.5-1.

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### 9.5.2 Communications System

The communication system provides the means to conveniently and effectively communicate between various plant locations and with off-site locations during normal, maintenance, transient, fire, and accident conditions under maximum potential noise levels. The communication system consists of the following subsystems:

- Plant page/party-line (PA/PL) subsystem;
- Private automatic branch exchange (PABX) subsystem;
- Plant sound-powered telephone subsystem;
- Plant radio subsystem;
- Evacuation alarm and remote warning subsystem;
- Emergency offsite communication subsystem; and
- Completely independent radio subsystem for security purposes as described in Section 13.6.

#### 9.5.2.1 Design Bases

##### Safety (10 CFR 50.2) Design Bases

The communication system serves no safety-related function and thus has no safety design basis.

##### Power Generation Design Bases

The communication system power generation design bases are as follows:

- Communication subsystems are independent of one another, therefore, a failure in one subsystem does not degrade the performance of the other subsystems;
- The communication system is in accordance with applicable codes and standards and the equipment is shielded as necessary, from the adverse effects of electromagnetic interference (EMI) and radio frequency interference (RFI); and
- The communication subsystems are operable during a loss of off-site power.

#### 9.5.2.2 System Description

##### Summary

The PA/PL, PABX, and plant radio systems are physically independent systems powered from diverse nonsafety-related power supplies backed from the standby on site AC power supply system. They serve as backup to one another in the event of system failures. These three independent voice communication systems are designed and installed to provide assurance that any single event does not cause a complete loss of intraplant communication. This is accomplished by the use of diverse technology, separate routing of cables, and separate standby diesel-generator-backed power supplies.

Particular attention is given to the supports and anchoring of emergency communication system components and components of the other communication systems located in normally occupied areas or in areas containing safety-related equipment so as to enhance the earthquake

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survivability of these components and ensure that they do not present a personnel or equipment hazard when subjected to seismic loading.

The plant simulator, the Emergency Operations Facility (EOF) and the Technical Support Center (TSC) connect to the plant nonsafety-related Distributed Control and Instrumentation System (N-DCIS) firewall via high speed dedicated data lines as discussed in Section 7.1 for data communicatons.

Descriptions of these systems are given in the following sections.

**In-Plant Page/Party Line (PA/PL)**

This system provides communication means such as ringing, mutual telephonic communication, and simultaneous broadcasting in various select buildings and areas including outdoor locations of the plant. The system permits merging with and separation from other units of the plant. The system is primarily used for intraplant communications. It is a hard-wired communication system used during plant operations, testing, calibration, startup, and limited emergencies.

The PA/PL system is a nonsafety-related system and mounting of system components is in accordance with standard design engineering practices. The PA/PL system components are mounted to Seismic Category II requirements in safety-related areas if located above safety-related equipment.

The PA/PL system consist of handsets, speakers, branch boxes, main distribution boards, a control board, amplifiers, amplifier boards, battery, battery chargers, DC distribution board, cable wiring materials, junction boxes and jacks. The system is a multiple-channel, multiple-system-split-type design with a separate set of amplifiers and a distribution board for each branch. This permits simultaneous use of a page line and four party lines for in-plant use. The block diagrams of the systems are shown in Figures 9.5-2, 9.5-3 and 9.5-4.

Handsets and speakers are installed in places which are important for plant operation and necessary for personnel safety, and where communication is frequent, including the areas below:

- Main Control Room (MCR);
- Electrical equipment rooms;
- Refueling area;
- Turbine operating area;
- Periphery of control rod hydraulic units areas;
- Feedwater pump rooms;
- Elevators;
- Exterior of plant buildings;
- Technical Support Center (TSC);
- Electrical building; and
- Cooling tower area.

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Each handset station can be used to communicate with any other handset station or the central station of another unit.

One circuit of the handset station is connected to a telephone line, thereby permitting simultaneous broadcasting from a security telephone unit. In addition to the basic function, the equipment can be used for an automatic surveillance of main amplifier output alarm indication in the event of failure of main equipment and manual switching to spare amplifier as necessary.

The system is operated from a battery source with a normal and a spare battery charger. The chargers are powered from a nonsafety-related power supply backed from the standby on site AC power supply system. The PA/PL equipment is backed by an exclusive DC power supply with a dedicated battery. The battery has capacity for 10 hours of operation following the loss of AC power. The charger is sized to recharge the battery from a fully discharged condition within 10 hours while supplying the normal DC loads.

The handsets are located at the same relative position on each floor, at a conspicuous location in personnel routes, at uniform intervals in corridors and large rooms, close to panels where possible, and low level radiation areas.

Paging equipment for outdoor facilities is designed to automatically limit the sound volume at night to a level manually set from the MCR.

Speakers and handsets are installed at the farthest practical distance from noise sources. However, in rooms where the noise level increases during equipment operation (such as the feedwater pump room, diesel generator room, etc.), handsets are enclosed within a soundproof booth.

Box-type speakers are installed in small rooms where reverberations make hearing difficult. The speakers are of two different types as described below:

- Horn shaped (trumpet shaped); and
- Cone shaped (box type).

The circuits from the main paging equipment to each junction box are ring-wired to preclude loss of system function in the event of a single cable failure.

**Private Automatic Branch Telephone Exchange (PABX)**

The private automatic branch telephone exchange (PABX) equipment and cabling is supplied and installed by the telephone company. The PABX is a multinode system with telephones located throughout the plant. The nodes of the PABX are located in separate communication rooms.

The PABX is connected to the commercial telephone system and the utility private network that allows off-site communications for normal and emergency conditions.

The PABX is powered from the plant nonsafety-related buses and consists of independent chargers and batteries for each node with the capability of operating the entire plant telephone system for approximately 8 hours after a loss of the normal AC supply.

**ESBWR****Sound-Powered Telephone System**

A separate telephone communication system using portable sound-powered telephone units, independent of the PA/PL and PABX systems, is provided for normal and abnormal/accident conditions. This sound-powered system allows uninterrupted private communication between the MCR and the control rod drive equipment area; refueling platform area; turbine-generator operating deck; areas containing switchgear, load centers, motor control centers; and other high-maintenance activity areas.

The communication facilities for use during plant maintenance consist of local terminal jacks and boxes and a system main communication board with storage for patch cords. Terminal jacks are attached to the central control boards and to local panels and racks where communication links are frequently required.

The system provides communication capability between boards in the MCR, between the MCR and field stations, or between field stations during testing and periodic inspection of the plant. The communication between field stations is by means of portable telephone units and patch cords at the system main communication board. The sound-powered telephone system does not require external power supply for operation.

**Plant Radio System**

Normal and emergency communications within the plant can be maintained independent of the PA/PL and PABX systems through the plant radio system. This system consists of antennas distributed throughout the plant with a centralized rebroadcast transmitter. Lower power portable radios are used with this system to ensure that there is no EMI with control and instrument circuits. The system is designed to permit radio to radio and radio to console communications from any location within the plant and satellite buildings.

Communication consoles are located at selected plant locations including the MCR and remote shutdown rooms. Communications between consoles is through hardwire, therefore providing a means of communication between selected areas of the plant even with the failure of the radio base station, PA/PL system, and PABX. The power for the base station and consoles is from the security system power supply that is backed by batteries and a standby generator.

Portable, hand-held radios provide two-way voice communications between the various units for personnel who need mobility. The radios are equipped with multiple channels as follows:

- Emergency (alternate security);
- Fire brigade (alternate security);
- Operations;
- Maintenance (alternate operations);
- Management;
- Health physics; and
- Crisis management (or Unassigned).

The radios are equipped with tone-coded squelch so that a message cannot be received unless the message contains the proper address code. Therefore, individual, all-channel (zone), and all-

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system calls can be made. The emergency channel is not coded. Calls are made between the telephone system and the in-plant radio system by dialing through the PABX to a radiotelephone interconnect panel.

Any portable radio systems operate at frequencies within the EMI/RFI test envelope for DCIS components.

**Evacuation Alarm and Remote Warning System**

The evacuation alarm and remote warning system is provided to warn personnel of emergency conditions. This system supplements the Area Radiation Monitoring System described in Subsection 12.3.4.

The evacuation alarm system consists of a siren tone generator, public address system speakers, and an outdoor siren. A selector switch in the MCR is used to manually initiate the evacuation alarm. This selector switch also selects the evacuation alarm coverage in the drywell or the entire plant including the initiation of the outdoor siren and the remote broadcast speakers.

The remote warning system consists of a message storage device, microphone, remote broadcast speakers, and an output/feedback monitoring system. The message storage device transmits recorded messages and the microphone transmits warning instructions through the remote broadcast speakers. An initiation signal from the MCR starts the message storage device or opens the microphone available for transmission.

The output/feedback monitoring system monitors the output of the remote broadcast speakers and retransmits the output back to the monitoring speaker when the message storage device is initiated or to the sound level (VU) meter when the microphone is activated. The monitoring speaker and sound level (VU) meter are located in the MCR.

Power for this system is supplied from a nonsafety-related bus backed from standby on site AC power supply system and backed by the station batteries.

**Emergency Communication Systems**

Normal and emergency off-site communications are provided by public telephone lines, the private utility network connected to the PABX and radio systems.

Emergency telephones are color-coded to distinguish them from normal telephones and include, but are not limited to, the following:

- Emergency Notification System (ENS) - Provides a communications link with the Nuclear Regulatory Commission (NRC) or country-specific regulatory agency and in accordance with IE Bulletin 80-15. See Subsection 9.5.2.5.1;
- Health Physics Network (HPN) - Provides a communications link with the NRC's (or country-specific regulatory agency) health physics personnel;
- Ringdown Phone System - Provides a communications link with local and state agencies;
- Crisis management radio system in accordance with the intent of NUREG-0654;
- Fire brigade radio system consists of a base unit, mobile units, portable units and in accordance with BTP SPLB 9.5-1, position C.5.g(4); and
- Communication with the transmission system operator. See Subsection 9.5.2.5.2;

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**9.5.2.3 Safety Evaluation**

The communications systems are not safety-related and are classified as non-Class 1E. The failure of any communications system does not adversely affect safe shutdown capability. It is not necessary for plant personnel in safety-related areas of the plant to communicate with the MCR in order to achieve safe shutdown of the plant.

Diverse non-Class 1E power supplies connected to the plant standby generators power the PA/PL telephone, PABX and plant radio systems. Failure of any or all of its components does not affect any safety-related equipment.

**9.5.2.4 Inspection and Testing Requirements**

The communications system is preoperational tested. The systems described above are conventional and have a history of successful operation at similar plants. These systems are used and maintained routinely to ensure their availability.

The power sources for the PA/PL telephone system and the PABX are tested separately during the preoperational and startup test program. Measurements or tests required to identify long-term deterioration are performed on a periodic basis.

**9.5.2.5 COL Unit-Specific Information****9.5.2.5.1 Offsite Interfaces**

The COL applicant shall describe the Emergency Notification System.

**9.5.2.5.2 Grid Transmission Operator**

The COL applicant shall describe the voice communication link availability with the grid transmission operator.

**9.5.2.6 References**

- 9.5.2-1 IEEE Standard 281, "IEEE Standard Service Conditions for Power System Communication Equipment," (see Table 1.9-22).
- 9.5.2-2 EPRI Report NP 6559, "Voice Communication System Compatible with Respiratory Protection."
- 9.5.2-3 10 CFR 73 Section 55(e) and (f), "Physical Protection of Plants and Material."
- 9.5.2-4 10 CFR 50, Appendix E, IV.E.9, ERF Communication System"
- 9.5.2-5 NRC Information Notice 86-097, "Emergency Communication Systems"
- 9.5.2-6 NRC Information Notice 87-058, "Continuous Communication Following Emergency Notification"
- 9.5.2-7 NRC IE Circular No.80-09, "Problems with Plant Internal Communication Systems,"
- 9.5.2-8 NUREG-0654, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plan."

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- 9.5.2-9 IE Bulletin 80-15, "Possible Loss of Emergency Notification System (ENS) with Loss of Offsite Power." June 18, 1980.
- 9.5.2-10 NRC Generic Letter 89-15, Emergency Response Data System
- 9.5.2-11 NUREG 0696, Functional Criteria for Emergency Response Facilities

### 9.5.3 Lighting System

This section covers all on-site systems that provide artificial illumination for rooms, spaces, and outdoor areas of the plant. These systems include normal, standby, emergency and security lighting subsystems. Security lighting system is described in separate security documents. Refer to Section 13.6.

#### 9.5.3.1 Safety (10 CFR 50.2) Design Bases

The safety design bases for the emergency lighting system in the MCR are as follows:

- The system is Class 1E;
- The system is integrated with the standby lighting system and designed so that alternate emergency lighting fixtures are supplied from separate safety divisions; and
- The system maintains the lighting levels for at least 72 hours following a design basis event including the loss of all AC power sources.

#### 9.5.3.2 Power Generation Design Bases

- The lighting system design provides the illumination required for the performance of activities in the various areas and is equal to or greater than those recommended by the Illuminating Engineering Society of North America (IESNA);
- The lighting circuits of the normal, standby, and emergency lighting subsystems are routed in separate conduits;
- The design of the lighting system for areas containing rotating equipment includes special provisions to eliminate the risk of stroboscopic effects caused by flicker;
- The circuits to the individual lighting fixtures (other than the DC self-contained battery-operated lighting units) are staggered to the extent possible and the staggered circuits are supplied from separate power supplies to ensure that some lighting is retained in each room even in the event of a circuit failure;
- Mercury vapor lamps and mercury switches are not used in fuel handling areas; and
- Incandescent lamps are the only type of lamp used within the primary containment, the main steam tunnel, and the refueling level of the Reactor Building;

#### 9.5.3.3 System Description

The plant lighting systems furnish the illumination required for safe performance of plant operation, security, shutdown, and maintenance activities. Emergency lighting is provided in Safety Related areas for the safety of personnel during a power failure. Table 9.5-3 summarizes the lighting system illumination ranges for normal illumination.

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The block diagrams of the plant lighting system are shown in Figures 9.5-5, 9.5-6 and 9.5-7 and are composed of the following systems:

- Normal lighting system;
- Standby lighting system; and
- Emergency lighting system.

The lighting systems are designed in accordance with applicable industry standards for lighting fixtures, cables, grounding, penetrations, conduits, and controls.

Lighting fixtures located in the vicinity of safety-related equipment are supported so that they do not adversely impact the equipment when subjected to seismic loading of a safe shutdown earthquake.

**9.5.3.3.1 Normal Lighting**

The normal lighting system, as supplemented by the standby lighting system, is used to provide normal illumination under normal plant operating, maintenance, and testing conditions.

Power for the normal lighting system is supplied from the nonsafety-related buses that are described in Subsection 8.3.1. This system provides lighting for all indoor and outdoor areas.

The normal illumination level in any area is comprised of light from all lighting fixtures except the emergency DC self-contained battery-operated lighting units.

Table 9.5-3 illustrates the typical ranges of illumination that are achieved, based on current recommendations.

The HID and fluorescent lighting fixtures in this subsystem are powered from 480/277 VAC; 3-phase, 4-wire, grounded neutral system distribution panels supplied from the normal 480 VAC motor control centers. The incandescent lighting fixtures on refueling platforms are powered from the 480/277 VAC, 3-phase, 4-wire, grounded neutral system distribution panels. Other incandescent lighting fixtures are powered from dry-type transformers rated at 480-208/120 VAC, 3-phase, 4-wire, grounded, or 480-240/120 VAC, single-phase, 3-wire, grounded.

**9.5.3.3.2 Standby Lighting**

The standby lighting system, in addition to supplementing the normal lighting system, supplements the emergency lighting system in selected areas of the plant where emergency operations are performed, including the access and egress routes to and from those areas.

The standby lighting system is designed to provide a minimum level of illumination to selected areas of the plant to aid in emergencies, safe shutdown, or in restoring the plant to normal operation. This system consists of fluorescent lighting fixtures powered from 480/277 VAC, 3-phase, 4-wire, grounded neutral system lighting distribution panels normally supplied by the PIP nonsafety-related buses. The primary areas served by this system are as follows:

- Main Control Room;
- Remote shutdown rooms;

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- Operational support center;
- Technical support center;
- Auxiliary switchgear rooms;
- Class 1E DC equipment rooms;
- Stairwells and aisle way;
- DCIS equipment rooms;
- Diesel generator room; and
- Diesel generator control room.

The standby lighting distribution panels also serve as the preferred power supply to the 8-hour emergency lighting units and the stair lighting units.

The standby lighting system is connected to the PIP nonsafety-related buses of the plant auxiliary AC power distribution system as described in Subsection 8.3.1.1.2. The standby lighting system is energized as long as power from an off-site source or a standby on-site AC power supply system is available.

The standby lighting system is limited to those areas of the plant where power can be supplied from the standby on-site AC power supply system, insofar as practical, and could be involved in emergencies, shutdown or recovery operations.

The standby lighting system and the normal lighting system provide normal illumination. Standby lighting is maintained as long as power is available from the PIP Nonsafety-Related buses.

**9.5.3.3.3 Emergency Lighting**

The emergency lighting system is used to provide acceptable levels of illumination throughout the station and, particularly, in areas where emergency operations are performed, such as control rooms, battery rooms, containment, etc., upon loss of the normal lighting system.

The emergency lighting system is comprised of the following:

- Control room emergency lighting; and
- DC self-contained battery-operated lighting units for exit lights, emergency lighting units, and stair lighting units;

The circuits to emergency lighting fixtures are continuously energized and are not switched.

The emergency lighting system components and installation inside and outside the MCR remain functional during design basis events and in particular withstand the seismic loads of a design basis earthquake.

**9.5.3.3.3.1 Control Room Emergency Lighting**

The control room emergency lighting is integrated with the standby lighting that consists of identical lighting fixtures and is supplied from the four divisions of 72-hour Class 1E Uninterruptible AC power supply system (UPS). The 72-hour Class 1E batteries and the standby

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on-site AC power supply system provide backup to the 72-hour Class 1E UPS. The MCR emergency lighting complies with the human factor requirements by utilizing semi-indirect, low-glare lighting fixtures. The MCR emergency lighting is Class 1E Seismic Category I. Both the standby and emergency lighting fixtures, switches and associated cables used in the main control room are non-Class 1E.

**9.5.3.3.2 DC Self-Contained Battery-Operated Lighting Units**

In areas outside the MCR and the remote shutdown area, emergency light is provided by 8-hour, self-contained, battery pack, sealed beam lighting units. These units are powered from the non-Class 1E and provide illumination for safe ingress and egress of personnel following a loss of normal lighting in following areas:

- Areas required for power restoration / recovery to comply with the requirement of BTP SPLB 9.5-1. Two-hour rated units as a minimum are used in other areas of the plant;
- Areas where normal actions are required for operation of equipment needed during fire; and
- Stairwells serving as escape or access routes for fire fighting.

The DC self-contained battery-operated emergency and stair lighting units are powered from the same circuit that powers the normal or standby lighting fixture whose loss of power then causes the operation of the particular emergency or stair lighting unit.

Emergency exit lighting consists of battery-powered self-contained “exit” light units. Each unit consists of a 90-minute battery, a battery charger and exit sign and is normally energized by 277 VAC or 120 VAC from the normal lighting system power supply.

Emergency lighting units provide emergency lighting instantaneously and automatically upon the failure or interruption of the normal or standby lighting power supply, as applicable. Each emergency lighting unit consists of a battery, a charger, and control and monitoring circuits, enclosed in a self-contained unit. Each emergency lighting unit is capable of supplying sealed beam lamps locally mounted on the battery pack unit, remotely mounted near the battery pack unit, or a combination thereof for 8 hours without the charger.

The emergency lighting units are provided with a time delay following restoration of AC power, the emergency lighting only turns off after adequate time for the normal or standby lighting to restart. The units are normally energized from the same circuit whose loss of light requires the operation of the unit.

**9.5.3.4 Safety Evaluation**

The MCR emergency lighting system is safety-related and classified as Class 1E. The remaining lighting systems are not safety-related and are classified as non-Class 1E. Components of the lighting systems associated with safety-related systems are supported to Seismic Category I requirements.

During normal plant operation, lighting systems are energized from on-site buses and off-site feeders. MCR emergency lighting permits the operators to safely operate and maintain the plant and to safely shutdown the plant and maintain it in a safe shutdown condition. The lighting system provides lighting at all times in areas used during reactor shutdown or emergency.

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In the event of a loss of AC power from both load groups of the unit auxiliary buses, the normal lighting system is inoperable. The standby lighting system is temporarily inoperable until the standby generators reenergize the permanent Nonsafety-Related buses. The emergency lighting system remains operable and is energized from the permanent Nonsafety-Related buses and the batteries of the self-contained lighting units. The security lighting system remains energized from the security system uninterruptible power supply (UPS) and standby generators.

In the event of a loss of AC power from both load groups of the unit auxiliary buses, the standby generators are signaled to start and energize their respective PIP Nonsafety-Related buses within one minute. During the 1-minute delay (standby generator startup time), the emergency lighting system in the MCR remains energized from the station Class 1E 250 VDC battery, and the self-contained battery-operated lighting units immediately turn on. After the standby generators have started and reenergized their respective PIP Nonsafety-Related buses, the standby lighting system is available. After an additional time delay, the DC self-contained battery-operated lighting units connected to the standby lighting power supply automatically turn off.

**9.5.3.5 Tests and Inspections**

The lighting systems are preoperational tested. System operability is demonstrated by normal use during plant operation.

The AC lighting systems are normally energized and maintained continuously and require only routine testing. The emergency lighting system is tested to ensure the operability of the DC self-contained battery-operated lighting units and other major components of the system.

**9.5.3.6 COL Unit-Specific Information**

None

**9.5.3.7 References**

9.5.3-1 Illuminating Engineering Society of North America (IESNA), IESNA Lighting Handbook (see Table 1.9-22).

**9.5.4 Diesel Generator Fuel Oil Storage and Transfer System****9.5.4.1 Design Bases****Safety (10 CFR 50.2) Design Bases**

The diesel generator fuel oil storage and transfer system is not safety-related and has no safety-related design basis.

**Power Generation Design Bases**

Each DG is supplied by a separate fuel oil system. The diesel generator systems are standby power supply systems. The fuel oil and transfer systems for the diesel generators design bases are as follows:

- Provide day tank of sufficient capacity to supply fuel oil to the DG for a minimum of 8 hours of operation at full load;

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- Provide a long-term fuel oil storage capacity (fuel oil storage tank) sufficient to support continuous Diesel operation for a minimum of 7 days without refueling. The use of the Diesel Generators for peaking service will not challenge the 7 day fuel oil supply reserves for diesel generator operation relative to plant investment protection or RTNSS functions;
- Ensure adequate separation between the two Diesel Generators Systems including their auxiliary and fuel oil supplies so that failure in one DG does not incapacitate the other diesel-generator;
- Provide protection against contamination of the ground or ground water through failure of tanks or buried piping;
- The diesel engine is designed to be compatible with the use of low and ultra-low sulfur diesel fuel;
- Diesel Fuel tanks will be designed in accordance with State and Federal regulations for required berm holding requirements; and
- The Diesel Fuel tanks will provide fuel to the Auxiliary Boiler system.

**9.5.4.2 System Description****Summary Description**

A simplified diagram of a typical DG fuel oil system is provided as Figure 9.5-9. The system is designed and supplied per a Diesel Generator equipment specification. The DG fuel oil system from the fuel oil day tank to the engine equipment is manufacturer provided. (See Subsection 9.5.4.6 for COL information)

The DG fuel oil system for each of the two engines consists of separate fuel oil storage tanks, fuel oil day tanks, fuel oil transfer pumps, strainers/filters, oil purifier, instrumentation and controls, and the necessary interconnecting piping and valves.

The DG fuel oil system has piping connections to supply fuel to the Auxiliary Boiler system.

**Detailed System Description**

There are two diesel generators, DG-A and DG-B, each housed in a separate enclosure in the Electrical Building adjacent to the Turbine Building. The units are identical and are held in reserve to furnish standby AC power in the event of Loss of Offsite Power (LOOP). Each DG has its own day tank, which holds sufficient fuel oil to operate its corresponding DG set for a minimum of 8 hours at full load. Each DG has its own storage tank, which holds sufficient fuel oil to operate its corresponding DG a minimum of 7 days without refueling. Transfer pumps supply fuel oil to each day tank from the fuel oil storage tank. A bleed line returns excess fuel oil from the day tank for recirculation to the fuel oil storage tank.

An engine-driven fuel oil pump supplies fuel from the day tank to the diesel engine fuel manifold then to the engine fuel pumps and injectors. Day tank elevation is such that the engine fuel oil pump operates with flooded suction. There are no intermediate powered components to fail. A suction strainer limits foreign matter from entering the pump and causing malfunction.

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Corrosion protection for underground piping for the fuel oil system will be determined based on the piping material. If piping subject to corrosion, such as carbon steel piping, is utilized, underground portions will be coated in accordance with DCD Tier 2, Chapter 8A,

A single DG can meet full site standby power demands. Fuel oil transfer system piping and components up to the engine skid connection are designed and constructed in accordance with the ASME Boiler and Pressure Vessel Code, Section VIII, Division 1, and ASME B31.1 for above ground piping runs. The underground piping portions is designed and constructed in accordance with the latest industry standards for buried pipe including provisions for corrosion protection. The diesel fuel oil storage and transfer systems are capable of supporting the start requirements of the diesel-generators (refer to Subsection 8.3.1). The stored fuel oil meets the requirements of the ASTM D975 “Standard Specification for Diesel Fuel Oils” and the requirements of the diesel engine manufacturer (see Subsection 9.5.4.6). The quality of the fuel oil used for the diesel engine is assured by routine testing. Similarly, the piping supplying the Auxiliary Boiler system meets the above requirements.

The DG fuel oil system has piping connections to supply fuel to the Auxiliary Boiler system. The piping connections for the auxiliary boiler tie into the diesel oil storage tank at an elevated nozzle connection. This location ensures that fuel storage quantity, stored below this level will not be affected by Auxiliary Boiler system usage. This ensures the 7 days Diesel Fuel Oil storage requirements cannot be used for any other purposes.

**System Operation**

Transfer pumps supplying fuel oil to the day tanks from the yard tanks can be operated manually; however, level sensors on the day tanks normally operate them automatically. A “low” level signal starts the first transfer pump, a “low-low” level signal starts the standby transfer pump and a “high” level signal stops both pumps. The engine-driven fuel oil pump supplies fuel to the diesel engine fuel manifold from the day tank. Administrative controls ensure a minimum of fuel oil capacity is maintained on site at all times.

**9.5.4.3 Safety Evaluation**

The DG and its auxiliary systems are not safety-related, and are not credited in any safety analysis. The storage tanks are located at a sufficient distance away from other plant buildings or, if required, buildings are protected with 3-hr rating barriers. The fuel oil day tank is located in a separate room with 3-hr fire rated concrete walls. Corrosion protection is provided for underground fuel oil piping. To prevent any fuel oil contamination during storage, biocides and other fuel additives are added, as required, to the stored fuel oil to prevent deterioration, accumulation of sludge in the tank, and the growth of algae and fungi. The design incorporates a fuel oil purification system to prevent tank contamination and thus ensure the diesel oil storage tank maintains the fuel at the desired quality.

**9.5.4.4 Tests and Inspections**

The DG fuel oil storage and transfer system permits periodic testing and inspection.

DG fuel oil storage and transfer system operability is demonstrated during the regularly scheduled operational tests of the DG. Periodic testing of instruments, controls, sensors and alarms assures reliable operation.

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The ASTM standard fuel sample tests are conducted at regular intervals to ensure compliance with fuel composition limits recommended by the diesel engine manufacturer. The design incorporates the use of a fuel oil purification system to ensure the diesel oil storage tank maintains the fuel at the desired quality.

Each fuel oil storage tank is emptied and accumulated sediments are removed every 10 years to conform to Federal and State examination requirements.

New fuel oil is tested for specific gravity, cloud point and viscosity and visually inspected for appearance prior to addition to ensure that the limits of ASTM D975 are not exceeded. Analysis of other properties of the fuel oil is completed within thirty days of the receipt of the new fuel.

**9.5.4.5 Instrumentation Requirements**

Fuel supply levels in the storage and day tanks are indicated locally and in the Main Control Room (MCR). Also, alarms on the local DG panel annunciate low level and high level in the day tanks. The setting of the low level alarm provides fuel for at least 60 minutes of DG operation at 100% load with 10% margin between the alarm and when the suction line inlet level sensors in the day tank signal automatic start of the first fuel oil transfer pump.

**9.5.4.6 COL Unit-Specific Information**

The COL applicant shall describe the material and corrosion protection for the underground piping portion of the fuel oil transfer system.. The COL applicant has responsibility to establish the procedural controls to ensure a minimum fuel oil capacity is maintained on site at all times.

**9.5.4.7 References**

- 9.5.4-1 ASME Boiler and Pressure Vessel Code (B&PVC), "Section VIII – Rules for Construction of Pressure Vessels" (see Table 1.9-22)
- 9.5.4-2 ASME B31.1, "Power Piping" (see Table 1.9-22)
- 9.5.4-3 ASTM D975, "Standard Specification for Diesel Fuel Oils" (see Table 1.9-22)

**9.5.5 Diesel Generator Jacket Cooling Water System****9.5.5.1 Design Bases****Safety (10 CFR 50.2) Design Bases**

The DG jacket cooling water system is not safety-related and has no safety design basis.

**Power Generation Design Bases**

A separate jacket cooling water system supplies each DG.

The DG jacket cooling water system is self-contained and sized to meet the full load cooling demands of the diesel-generator.

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### **9.5.5.2 System Description**

#### **Summary Description**

The diesel generator manufacturer shall specify the design of the DG jacket water system. A simplified diagram of a typical DG jacket cooling water system is shown in Figure 9.5-10.

The DG systems are standby power supply systems. The DG jacket cooling water system is a self-contained, closed-loop system that circulates cooling water through the diesel engine, to maintain system operating temperature. The jacket cooling water, which picks up heat from the operating engine is cooled as it circulates through the jacket water heat exchanger. The system incorporates a keep-warm system to maintain the diesel in a pre-warmed state while in the standby mode to support required engine start times.

#### **Detailed System Description**

Each DG unit is supplied with a complete closed-loop cooling system mounted integrally with the engine-generator package. Included in each cooling package are a jacket water heater and keep warm pump, temperature regulating valve, lube oil cooler, jacket water pumps, a manifold, a head tank and a jacket water heat exchanger. The JW heat exchanger rejects engine heat to the Reactor Component Cooling Water System (RCCWS). RCCWS water supply is from the same train as that of the DG served. In addition to the engine (jacket and head), the jacket water cools the turbocharger, the governor, the engine air coolers, the exhaust manifold and the lube oil cooler. However, depending upon the engine manufacturer, the above components may vary slightly.

The installed electric heater is designed to keep the engine jacket water at a manufacturer recommended standby temperature.

The heat exchanger for the DG jacket cooling water system is designed for a total heat removal rate based on the maximum permissible output of the DG. Additional margin is included to assure reliable system operation. The Jacket Cooling water system piping is manufacturer designed and constructed.

The system is filled with high quality, treated water, to prevent long term corrosion, from the Makeup Water System (MWS) to prevent long-term deterioration of the system internal surfaces. This makeup water is either manually or automatically controlled to the JW head tank. The level of the head tank allows the detection and control of system leakage.

#### **System Operation**

During the standby mode, the jacket water temperature is maintained within the range specified by the DG manufacturer based on normal ambient temperature, by the heater. The heated water is circulated through the engine by the keep-warm pump to maintain a uniform engine temperature. This portion of the system is not required for the DG operation; it only reduces stress during starts.

Depending upon the engine manufacturer either one or two engine driven jacket water circulating pumps are provided to circulate the cooling water through the engine during DG operation. The jacket cooling water passes through temperature control valve(s) that modulate the flow of water through or around the jacket water heat exchanger to maintain required water

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temperature. The functional capability of the system during high water levels (i.e. flooding) is assured since the system is engine driven and is a closed loop.

**9.5.5.3 Safety Evaluation**

The DG jacket cooling water system serves no safety-related function. However it is designed robustly by the manufacturer to ensure operation in high impact Marine type environments and Stationary power plants.

**9.5.5.4 Tests and Inspection**

To ensure the availability of the DG jacket cooling water system, scheduled inspection and testing of the equipment are performed, as part of the overall engine performance checks. Instrumentation is provided to monitor cooling water temperatures, pressure, and head tank level. Instruments receive periodic calibration and inspection to verify their accuracy. During standby periods, the keep-warm feature of the engine water jacket cooling closed-loop system is checked at scheduled intervals to ensure that the water jackets are warm. The cooling water in the engine water jacket cooling closed-loop system is sampled and analyzed at regular intervals and is treated, as necessary, to maintain the desired quality.

**9.5.5.5 Instrumentation Requirements**

Pressure, temperature, and level instrumentation are provided for monitoring of system operating parameters. Alarms, locally and in the MCR, provide warning in case of system low or high water temperature, low pressure, or low water inventory. The diesel-generators trip on high-high cooling water temperature.

**9.5.5.6 COL Unit-Specific Information**

None.

**9.5.5.7 References**

9.5.5-1 ASME B31.1, "Power Piping" (see Table 1.9-22)

**9.5.6 Diesel Generator Starting Air System****9.5.6.1 Design Bases****Safety (10 CFR 50.2) Design Bases**

The DG starting air auxiliary system is not safety-related and has no safety design basis.

**Power Generation Design Bases**

Each DG is supplied by a separate starting air system. The DG systems are standby power supply systems. The DG starting air system meets the following design bases:

- Provides a supply of compressed air for starting the generator diesel engines without external power; and

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- Starting Air Receivers have sufficient air storage capacity for three consecutive starts of the engine and perform the starting function so that the DG meets the readiness criteria defined in Subsection 8.3.1- Onsite AC Power Systems.

### **9.5.6.2 System Description**

#### **Summary Description**

Each system includes two air compressors, air receiver(s), and redundant air admission valves. When standby DG electric power is required, the air admission valves are opened to initiate engine cranking.

The diesel generator manufacturer provides the DG air starting system design. A simplified diagram of a typical DG air starting system is provided in Figure 9.5-11

#### **Detailed System Description**

Each of the two standby DGs is provided with its own starting air system, consisting of two redundant 100% capacity air compressors, air receiver, a 100% capacity air dryer, associated piping, and valves. Two redundant starting air valves, one in each engine starting air manifold, are provided for each engine. Failure of one valve does not affect the ability of the other valve to start the engine.

The air compressors are motor driven. Each compressor is equipped with appropriate moisture removal equipment. The air receiver(s) are sized so each air starting system has sufficient capacity for cranking its engine for three automatic starts without recharging. Each air receiver is provided with an automatic drain trap at the receiver bottom to remove any water accumulated in the tank.

Each DG air start system is provided with an air dryer to ensure clean dry air for engine starting. The dryer is capable of controlling the dew point as recommended by the diesel engine manufacturer. It is equipped with filters to remove oil, dust and pipe scale from the air stream.

The air receiver, valves, and piping up to the first connection on the engine skid are designed to the ASME Boiler and Pressure Vessel Code, Section VIII, Division 1 and ASME B31.1. A low-pressure alarm setpoint is established to notify operators when the receiver pressure drops and multiple engine starts attempts are not available.

#### **System Operation**

Normally, the air compressors operate automatically and are controlled by pressure sensors located on their respective air receivers. The pressure sensors signal the start and stop of the compressors, as necessary to maintain the required system pressure. Manual override of the automatic sequence is provided for emergency operation.

The valves receive a signal to open and initiate the engine starting sequence in the event of a total loss of off site power (LOOP). If a start attempt is unsuccessful after a manufacturer specified period of time, the control system automatically detects the failure, activates an alarm to alert the operator, and allows for a restart attempt.

### **9.5.6.3 Safety Evaluation**

The DG starting air system serves no safety-related function.

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**9.5.6.4 Tests and Inspection**

Periodic tests and inspections are performed on the following:

- Air receiver pressure control switches;
- Low pressure alarm signal for low receiver pressure;
- Engine air start valves and the admission line vent valve;
- Pressure gages on the receivers;
- Air receivers to clear accumulated moisture using the blowdown connection as required; and
- Air Quality – oil, particulates, and dew point.

**9.5.6.5 Instrumentation Requirements**

The starting air receivers are designed to store sufficient amount of compressed air to ensure three engine start attempts. An air receiver low-pressure alarm is provided to alert the MCR operator of low starting air pressure with a setpoint established, to ensure that multiple engine starts are still available. A failed start attempt alarms if the engine fails to start within a manufacturer specified amount of time.

**9.5.6.6 COL Unit-Specific Information**

None

**9.5.6.7 References**

- 9.5.6-1 ASME Boiler and Pressure Vessel Code (B&PVC), "Section VIII – Rules for Construction of Pressure Vessels" (see Table 1.9-22)
- 9.5.6-2 ASME B31.1, "Power Piping" (see Table 1.9-22)

**9.5.7 Diesel Generator Lubrication System****9.5.7.1 Design Bases****Safety (10 CFR 50.2) Design Bases**

The DG lubrication system is not safety-related and has no safety design basis.

**Power Generation Design Bases**

Each DG is supplied by a separate lubrication system. The DG lubrication system supplies clean filtered oil to the engine bearings at controlled pressure and temperature.

**9.5.7.2 System Description****Summary Description**

The DG includes a self-contained lubrication system including lube oil sump tank, circulating pump, filtering elements, and a cooler. The system filters the lubricating oil and delivers it at controlled pressure and temperature to the engine and generator bearing surfaces. Built-in lube

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oil storage capacity ensures adequate lubrication of wearing surfaces and cooling as necessary. An electric heater and a keep-warm circulating pump continuously circulate warm oil to maintain the engine in standby readiness. This keep-warm system is not required for the engine to perform its function.

The diesel generator manufacturer supplies the lubrication system as part of the engine design. A simplified diagram of a typical DG lubrication system is provided in Figure 9.5-12.

**Detailed System Description**

Each of the two DG lubrication systems consists of an oil sump in the engine frame, an engine-driven positive displacement pump, a cooler, a main header, strainer, and filters. The main engine-driven lube oil pump takes oil from the lube oil sump tank, passes it through the lube oil cooler and lube oil filter, through a strainer, through the main header, through the lube oil loads and back to the lube oil sump tank. The lubrication system (LO) system also supplies oil to the combustion air turbochargers. Pressure-regulating valves maintain constant oil pressure to the main header by bypassing excess oil back to the lube oil sump tank. The system is manufacturer designed to provide engine lubrication of moving surfaces and to remove engine heat.

Each lube oil cooler is built to TEMA Class C or Industry Standards as per the engine manufacturer. Cooling water for the coolers is from the DG jacket cooling water system.

The DG sets have lube oil heating systems to keep the oil warm during standby. An electric lube oil heater heats the oil, which is then circulated through the engine oil circuit by a motor-driven keep-warm circulating pump. The keep-warm system circulates oil through a filter to ensure cleanliness. The lube oil portion of the system to keep the engine warm and prelubed is not required for the engine to start and perform its function. The engine lube oil sump is provided with level indication and can be manually replenished. The lube oil may be added to the engine oil sump during engine operation. The tanks, pumps, piping, and valves up to the engine connections are built to ASME B31.1 or other Industry standards as per the engine manufacturer. The engine is equipped with explosion proof doors to protect the engine in the event of a crankcase explosion.

**System Operation**

The lube oil keep-warm systems are in operation when the DGs are in their normal standby mode. When an engine starts, the lube oil keep-warm subsystem shuts down. When each DG is started, the engine-driven lube oil pump begins circulating oil through the filters and manifold to the engine bearing surfaces and back to the sump. The oil pumps are engine driven and therefore are not affected by flooding or power losses so lube oil circulation continues until the engine is shutdown.

**9.5.7.3 Safety Evaluation**

The DG lubrication system serve no safety-related function.

**9.5.7.4 Tests and Inspection**

The operability of the DG lubrication system is tested and inspected during scheduled operational testing of the overall engine. Instrumentation is provided to monitor lube oil temperature, pressure and sump level, ensuring proper operation of the system. During standby

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periods, the keep-warm system is checked at scheduled intervals to ensure that the oil is warm. The lube oil is periodically sampled and analyzed to ensure quality.

Instruments are periodically calibrated and tested to verify their accuracy.

The lubrication systems are located in the DG rooms, which are provided with personnel control access, precluding unauthorized personnel from interfering with system operation. Also, any contamination of the lubricating oil is thereby prevented.

**9.5.7.5 Instrumentation Requirements**

Pressure, temperature, and level instrumentation is provided for monitoring of important system operating parameters. Alarms in the MCR provide warning in case of system low or high temperature, low pressure, or low oil inventory. The diesel-generators trip on low-low lubricating oil pressure.

**9.5.7.6 COL Unit-Specific Information**

None

**9.5.7.7 References**

- 9.5.7-1 TEMA - Standards of Tubular Exchanger Manufacturers Association (see Table 1.9-22)
- 9.5.7-2 ASME Boiler and Pressure Vessel Code (B&PVC), "Section VIII – Rules for Construction of Pressure Vessels" (see Table 1.9-22)
- 9.5.7-3 ASME B31.1, "Power Piping" (see Table 1.9-22)

**9.5.8 Diesel Generator Combustion Air Intake and Exhaust System****9.5.8.1 Design Bases****Safety (10 CFR 50.2) Design Bases**

The DG combustion air intake and exhaust system is not safety-related and has no safety design basis.

**Power Generation Design Bases**

Each DG is supplied by a separate air intake and exhaust system.

The DG systems supply standby electric power. The DG combustion air intake and exhaust system is designed to meet the following design bases:

- Provide a supply of combustion air for operating the diesel engines;
- Prevent ingress to the diesel engines of contaminating substances that could degrade the diesel engine performance;
- Provide a diesel engine exhaust system capable of exhausting the products of combustion to the atmosphere; and
- Federal, State and Local Air Quality Emission Standards.

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### **9.5.8.2 System Description**

#### **Summary Description**

Each DG is provided with a separate intake and exhaust system. The combustion air intake and exhaust system supplies filtered air for engine fuel combustion. The system also exhausts engine combustion products out of the DG enclosure to the atmosphere. It includes intake and exhaust silencers to ensure quiet engine operation.

The diesel generator manufacturer supplies the DG air intake and exhaust system as part of their engine design. A simplified diagram of a typical DG air intake and exhaust system is provided in Figure 9.5-9 .

#### **Detailed System Description**

Each engine takes combustion air from its own cubical in the DG room. Air enters the cubical through the outside wall and is filtered before entering the air intake plenum (which may be integral to the turbocharger). Combustion air is drawn from the intake plenum through an air intake silencer (which may be integral to the turbocharger) and then is drawn into the engine turbocharger(s) which discharges through an intercooler into the diesel engine combustion air manifold.

A differential pressure gage is installed across the filters to monitor the filter condition.

The engine exhaust is collected in an exhaust manifold and directed to the turbine section of the turbocharger where it imparts energy to spin the compressor. The hot exhaust is discharged from the turbo to the exhaust silencers. Each engine has its own exhaust system. The exhaust outlet is well removed from the air inlet to prevent any recirculation of exhaust gases. The engine design provides for crankcase venting.

Nitrogen, hydrogen and CO<sub>2</sub> storage tanks are located at a sufficient distance away from the DG air inlet to preclude engine malfunction or explosions and to ensure that the engine intake air, oxygen level is not diluted.

In order to ensure exhaust emissions meet Federal, State and Local Air Quality Emissions Standards, the Diesel Engine design employs either an engine control system or a direct water injection system.

#### **System Operation**

There are no active components in the combustion air intake and exhaust system. Combustion air is drawn into the turbocharger by vacuum when the diesel engine is started and exhausted after combustion. The turbocharger pressurizes the intake manifold gases prior to entry to the cylinders to improve combustion and overcome the potential effects of such reduced atmospheric pressure conditions as may result in a tornado.

### **9.5.8.3 Safety Evaluation**

The DG combustion air intake and exhaust system serves no safety-related function.

**ESBWR*****9.5.8.4 Inspection and Testing Requirements***

Visual inspection of the DG combustion air intake and exhaust system is performed concurrently with regularly scheduled DG testing and inspection. Inspection of the integrity of the ducting and joints, filter condition, intake and exhaust silencer condition is included in DG maintenance procedures.

***9.5.8.5 Instrumentation Requirements***

The air intake filters are equipped with differential pressure gages to monitor the filter condition. An alarm is provided in the MCR to signal a high differential pressure condition. Engine high combustion air exhaust temperature or high crankcase pressure provide alarms in the MCR.

***9.5.8.6 COL Unit-Specific Information***

None

***9.5.8.7 References***

None

**Table 9.5-1**  
**Lists of Applicable Codes for Fire Protection**

American Society of Mechanical Engineers (ASME)
A17.1, Safety Code for Elevators and Escalators B31.1, Power Piping NQA-1, Quality Assurance Programs Requirements for Nuclear Facilities
American Society for Testing and Materials (ASTM)
ASTM E-84, Method of Test of Surface Burning Characteristics of Building Material ASTM E-119, Fire Test of Building Construction Materials ASTM E-814, Standard Test Method for Fire Tests for Through-Penetration Fire Stops
Applicable Building Codes
International Building Code International Fire Code American Disability Act (ADA) Accessibility Guidelines – 28 CFR 36
Factory Mutual
Factory Mutual (FM) Approval Guide
Institute of Electrical and Electronics Engineers (IEEE)
C2, National Electric Safety Code IEEE 383, Standard for Type Test of Class 1E Electric Cables IEEE 384, Criteria for Independence of Class 1E Equipment and Circuits
IEEE 1202, Standard for Flame-Propagation Testing of Wire and Cable
National Fire Protection Association (NFPA)
NFPA 10, Standard for Portable Fire Extinguishers NFPA 11, Standard for Low-, Medium, and High-Expansion Foam NFPA 12, Standard for Carbon Dioxide Extinguishing Systems NFPA 13, Standard for Installation of Sprinkler Systems NFPA 14, Standard for Standpipe and Hose Systems NFPA 15, Standard for Water Spray Fixed Systems NFPA 16, Standard for the Installation of Foam-Water Sprinkler Systems and Foam-Water Spray Systems NFPA 20, Standard for the Installation of Stationary Pumps NFPA 22, Standard for Water Tanks for Private Fire Protection NFPA 24, Standard for the Installation of Private Fire Service Mains and their Appurtenances NFPA 30, Flammable and Combustible Liquids Code NFPA 37, Standard for the Installation and Use of Stationary Combustion Engines and Gas Turbines NFPA 70, National Electric Code NFPA 72, National Fire Alarm Code NFPA 80, Standard for Fire Doors and Fire Windows NFPA 80A, Recommended Practice for Protection of Buildings from Exterior Exposure Fire

**Table 9.5-1**  
**Lists of Applicable Codes for Fire Protection (continued)**

<p>NFPA 90A, Standard for the Installation of Air-Conditioning and Ventilating Systems</p> <p>NFPA 92A, Standard for Smoke Control Systems Utilizing Barriers and Pressure Differences</p> <p>NFPA 101, Life Safety Code</p> <p>NFPA 214, Standard on Water Cooling Towers</p> <p>NFPA 241, Standard for Safeguarding Construction, Alteration, and Demolition Operations</p> <p>NFPA 204, Standard for Smoke and Heat Venting</p> <p>NFPA 251, Standard Method of Tests of Fire Endurance of Building Construction and Materials</p> <p>NFPA 252, Standard Method of Fire Tests of Door Assemblies</p> <p>NFPA 255, Standard Method of Test of Surface Burning Characteristics of Building Materials</p> <p>NFPA 497, Recommended Practices for the Classification of Flammable Liquids, Gases, or Vapors and of Hazardous (Classified) Locations for Electrical Installations in Chemical Process Areas</p> <p>NFPA 600, Standard on Industrial Fire Brigades</p> <p>NFPA 701, Standard Methods of Fire Tests for Flame Propagation of Textiles and Films</p> <p>NFPA 780, Standard for the Installation of Lightning Protection Systems</p> <p>NFPA 801, Standard for Fire Protection Practices for Facilities Handling Radioactive Materials</p> <p>NFPA 804, Standard for Fire Protection for Advanced Light Water Reactor Electric Generating Plants</p> <p>NFPA 1404, Standard for Fire Service Respiratory Protection Training</p> <p>NFPA 1451, Standard for a Fire Service Vehicle Operations Training Program</p> <p>NFPA 1500, Standard on Fire Department Occupational Safety and Health Program</p> <p>NFPA 1961, Standard for Fire Hose</p> <p>NFPA 1962, Standard for the Inspection, Care, and Use of Fire Hose, Couplings, and Nozzles and the Service Testing of Fire Hose</p> <p>NFPA 1963, Standard for Fire Hose Connections</p> <p>NFPA 1964, Standard for Spray Nozzles</p>
<b>Occupational Safety and Health Act (OSHA)</b>
<p>29 CFR 1910, Occupational Safety and Health Standards</p> <p>29 CFR 1926, Safety and Health Regulations for Construction</p>
<b>Underwriters Laboratories (UL)</b>
Underwriters Laboratories (UL) Fire Protection Equipment Directory
<b>NRC Regulations and Guidance</b>
<p>10 CFR 50.48, Fire Protection</p> <p>10 CFR 50, Appendix A, GDC 3, Fire Protection</p> <p>10 CFR 50, Appendix A, GDC 5, Sharing of Structures, Systems, and Components</p> <p>10 CFR 50, Appendix A, GDC 19, Control Room</p> <p>10 CFR 50, Appendix A, GDC 23, Protection System Failure Modes</p> <p>NUREG-0800, Standard Review Plan (SRP) 9.1.3 Spent Fuel Pool Cleaning and Cleanup System</p> <p>NUREG-0800, Standard Review Plan (SRP) 9.5.1 Fire Protection Program</p> <p>NUREG-0800, Branch Technical Position SPLB 9.5-1, Guidelines for Fire Protection for Nuclear Power Plants</p>

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NUREG-0800, Branch Technical Position SPLB 9.5-1, Appendix B, Supplemental Fire Protection Review Criteria for Advance Reactors

NUREG-1552, Fire Barrier Penetration Seals in Nuclear Power Plants

Regulatory Guide 1.13, Spent Fuel Storage Facility Design Basis

Regulatory Guide 1.39, Housekeeping Requirements for Water-Cooled Nuclear Power Plants

Regulatory Guide 1.189, Fire Protection for Operating Nuclear Power Plants

**Table 9.5-2**  
**FPS Component Design Characteristics**

<b>Firewater Pumps</b>	
Primary motor-driven fire pump	454.2 m <sup>3</sup> /hr (2,000 gpm) ***
Primary diesel-driven fire pump	454.2 m <sup>3</sup> /hr (2,000 gpm) ***
Secondary motor-driven fire pump*	454.2 m <sup>3</sup> /hr (2,000 gpm) ***
Secondary diesel-driven fire pump*	454.2 m <sup>3</sup> /hr (2,000 gpm) ***
Primary motor-driven jockey pump	4.54 m <sup>3</sup> /hr (20 gpm) minimum as required to maintain the NI firewater loop pressure 34.4 kPa (5 psi) above the start pressure of the fire pumps
Secondary motor-driven jockey pump	4.54 m <sup>3</sup> /hr (20 gpm) minimum as required to maintain the yard main loop pressure 34.4 kPa (5 psi) above the start pressure of the fire pumps
Required minimum total makeup flow rate to IC/PCC and spent fuel pools at 72 hours into an event	46 m <sup>3</sup> /hr (200 gpm)
<b>Firewater Storage</b>	
Primary storage tanks combined minimum usable firewater storage	3900 m <sup>3</sup> (1,030,000 gallons)
Secondary storage minimum firewater storage**	2081.8 m <sup>3</sup> (550,000 gallons)

\* Secondary fire pumps may be new or existing depending upon available site-specific provisions.

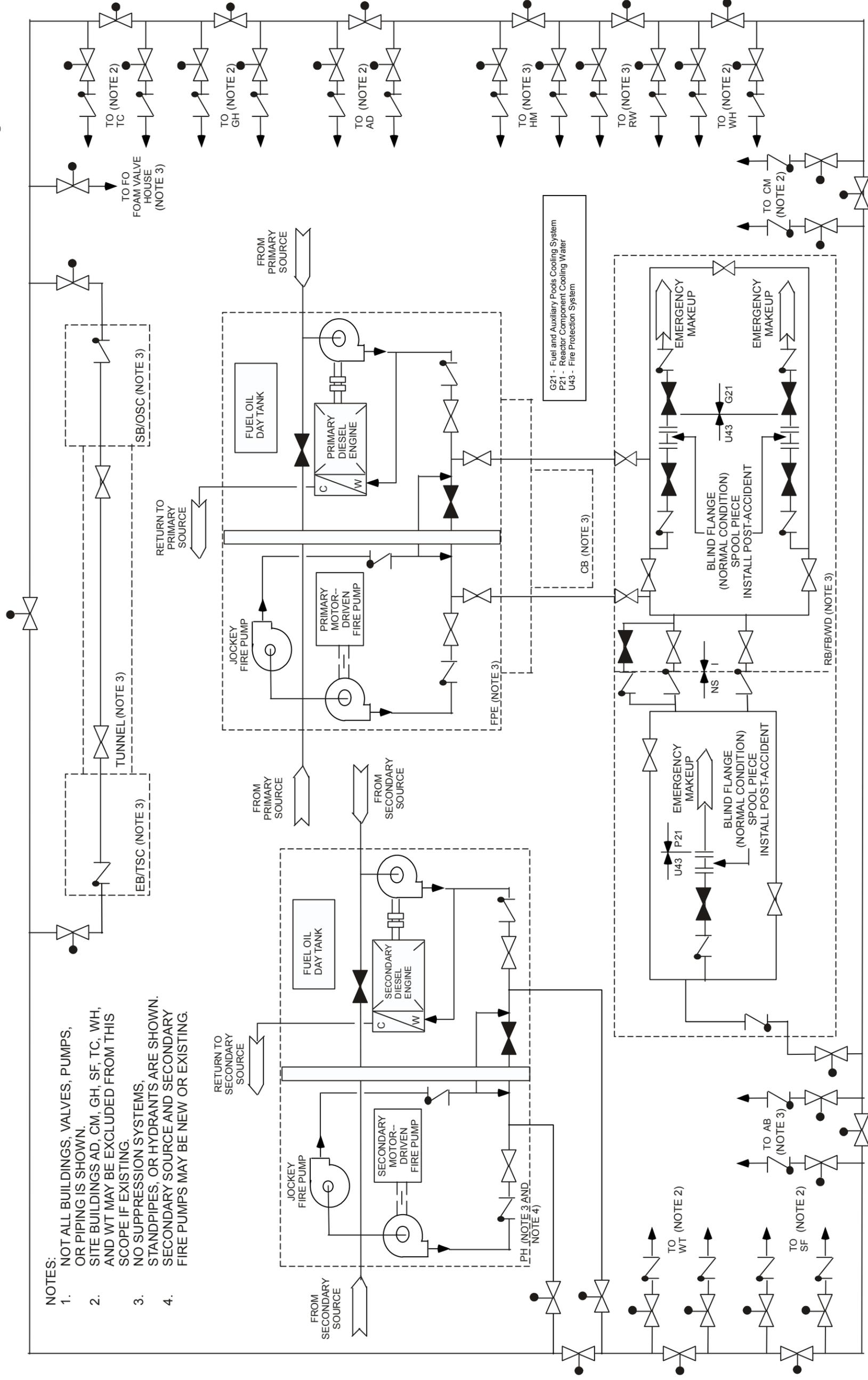
\*\* Secondary firewater storage may be a tank, cooling tower basin, or a large body of water depending upon available site-specific provisions. Storage volume listed is the minimum storage volume to be dedicated for fire protection use.

\*\*\* Based on the largest firewater demand of 967 m<sup>3</sup>/hr (4256 gpm) for Turbine Building, including hose stream.

**Table 9.5-3**  
**Typical Luminance Ranges for Normal Lighting**

Location	Luminance Range	
	(Lux)	(Foot-candles)
Auxiliary Building and Uncontrolled Access Areas	150-200	15-20
Controlled Access Areas		
– Count Room	750-1000	75-100
– Laboratories	750-1000	75-100
– Health Physics Office	1500-2000	150-200
– Medical Aid Room	1500-2000	150-200
– Hot Laundry	300-500	30-50
– Storage Room	150-200	15-20
– Engineered Safety Features Equipment	300-500	30-50
– Battery Rooms	300-500	30-50
Electrical Building (w/ DG)	300-500	30-50
Fuel Handling Area		
– Operating Floor	300-500	30-50
– Below Operating Floor	150-200	15-20
Off Gas Building	150-200	15-20
Radwaste Building	300-500	30-50
Reactor Building		
– Operating Floor	300-500	30-50
– Below Operating Floor	150-200	15-20
Control Room		
– Main Control Boards	300-500	30-50
– Auxiliary Control Panels	300-500	30-50
– Operator's Station	750-1000	75-100
Turbine Building		
– Operating Floor	300-500	30-50
– Below Operating Floor	150-200	15-20
Switchgear and Motor Control Centers	300-500	30-50
HVAC Equipment Areas	50-75	5-7.5
Exterior Areas	10-20	1-2

- NOTES:
1. NOT ALL BUILDINGS, VALVES, PUMPS, OR PIPING IS SHOWN.
  2. SITE BUILDINGS AD, CM, GH, SF, TC, WH, AND WT MAY BE EXCLUDED FROM THIS SCOPE IF EXISTING.
  3. NO SUPPRESSION SYSTEMS, STANDPIPES, OR HYDRANTS ARE SHOWN.
  4. SECONDARY SOURCE AND SECONDARY FIRE PUMPS MAY BE NEW OR EXISTING.



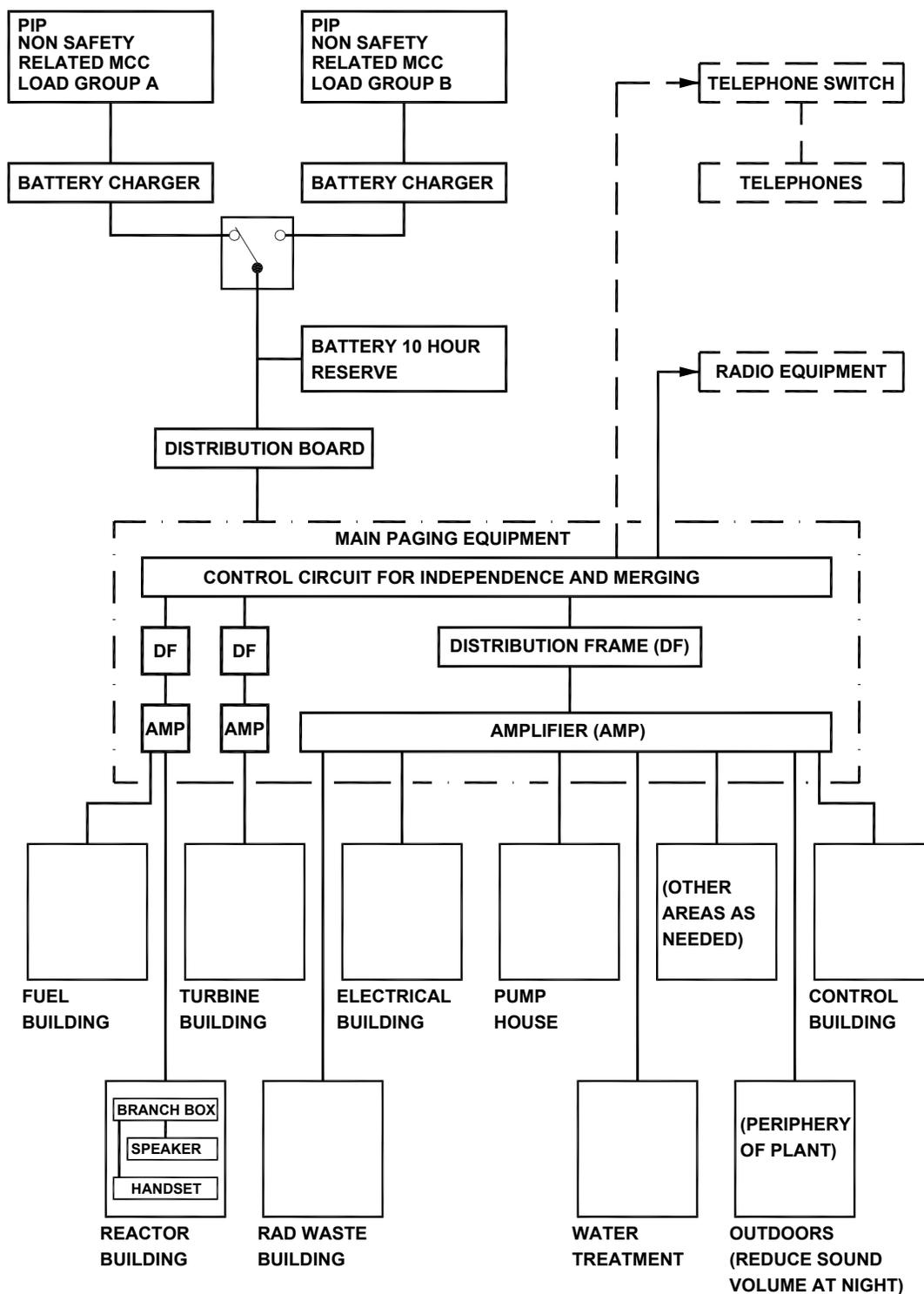
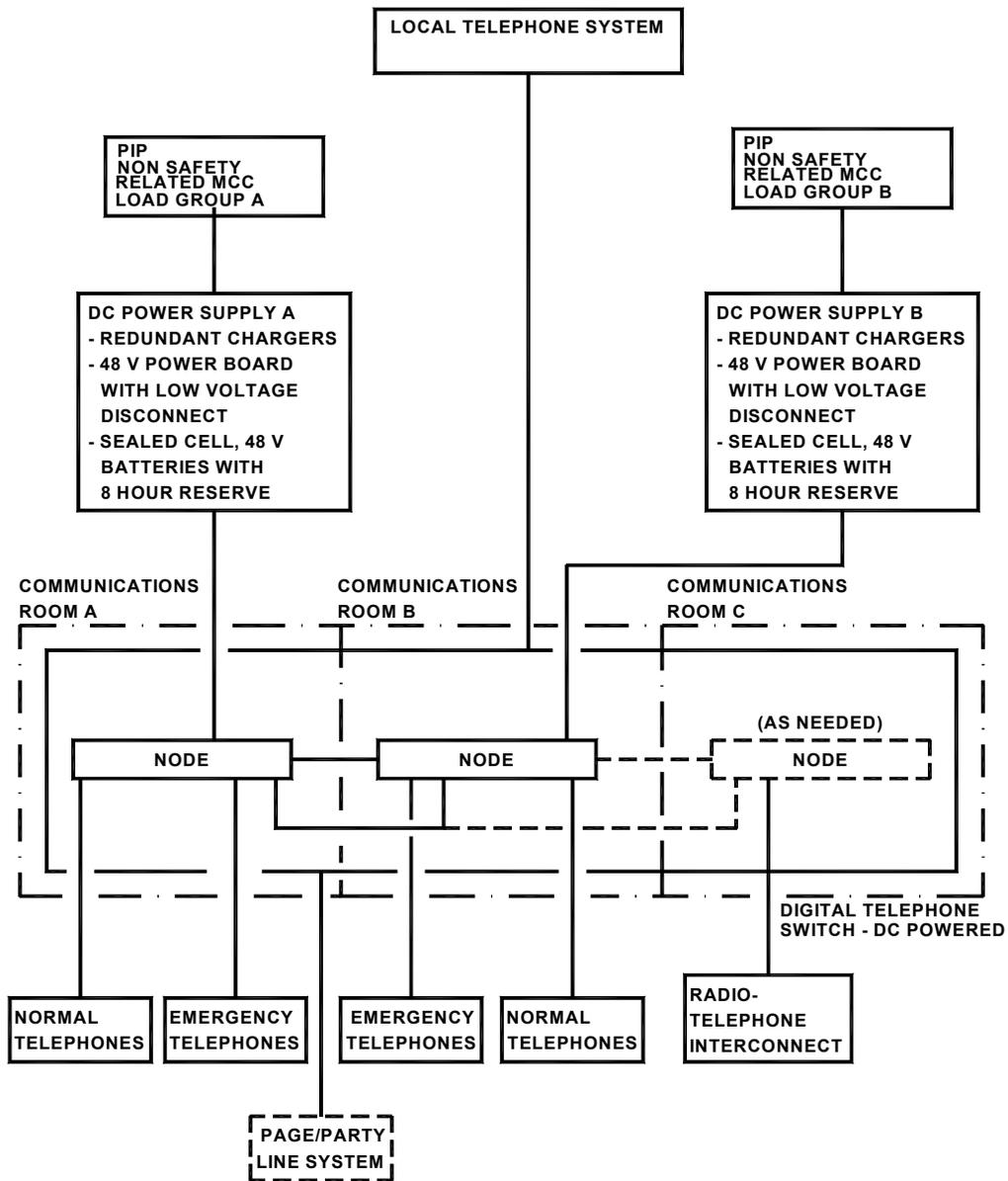


Figure 9.5-2. Page/Party-Line System Simplified Block Diagram



**Figure 9.5-3. Private Automatic Branch Exchange (PABX) Telephone System Simplified Block Diagram**

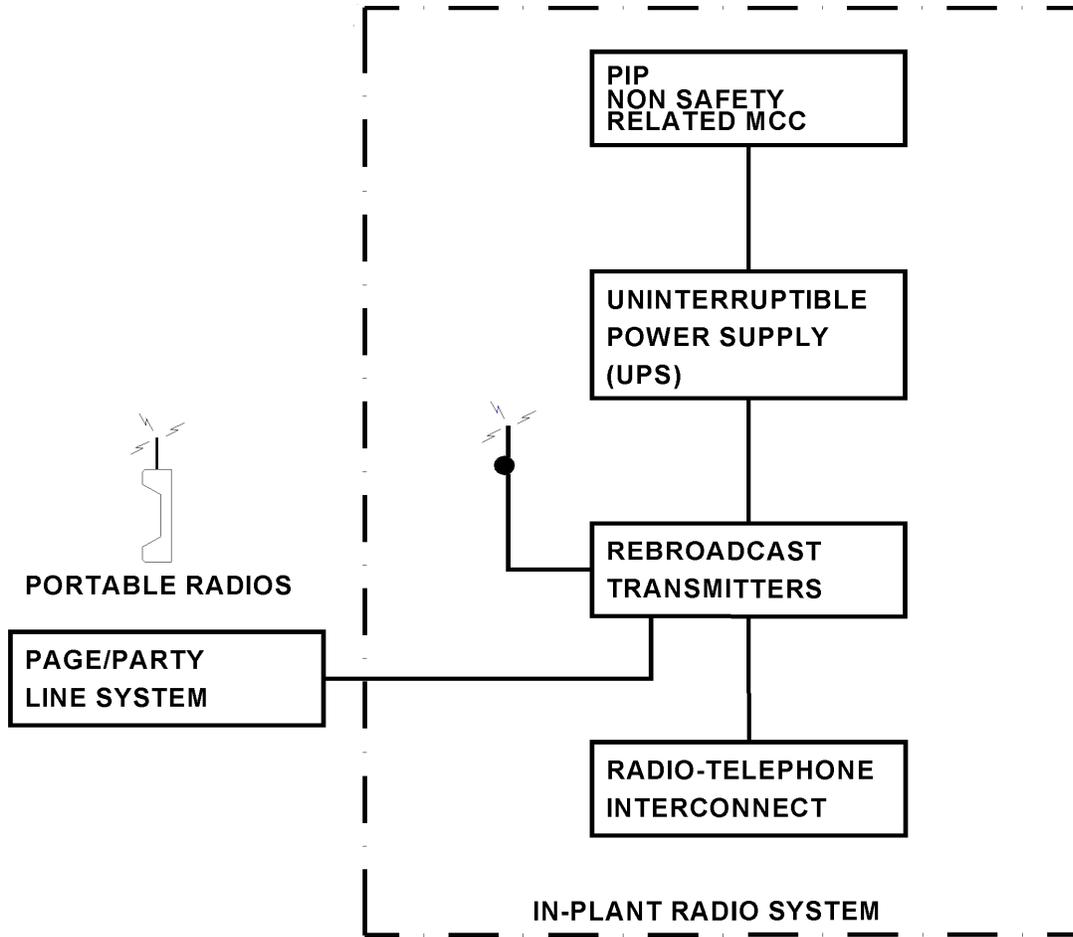


Figure 9.5-4. In-Plant Radio System Simplified Block Diagram

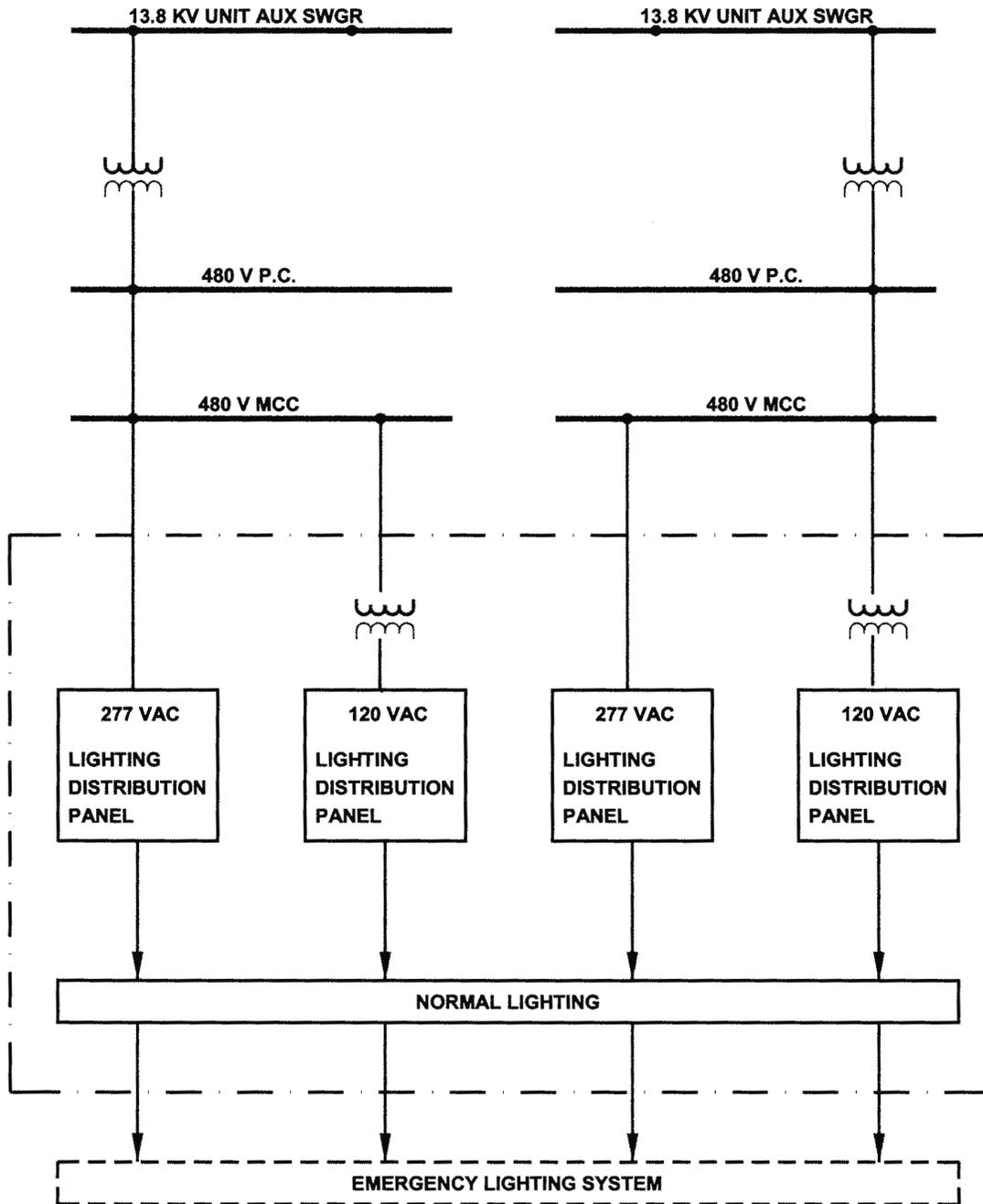


Figure 9.5-5. Normal Lighting System Simplified Block Diagram

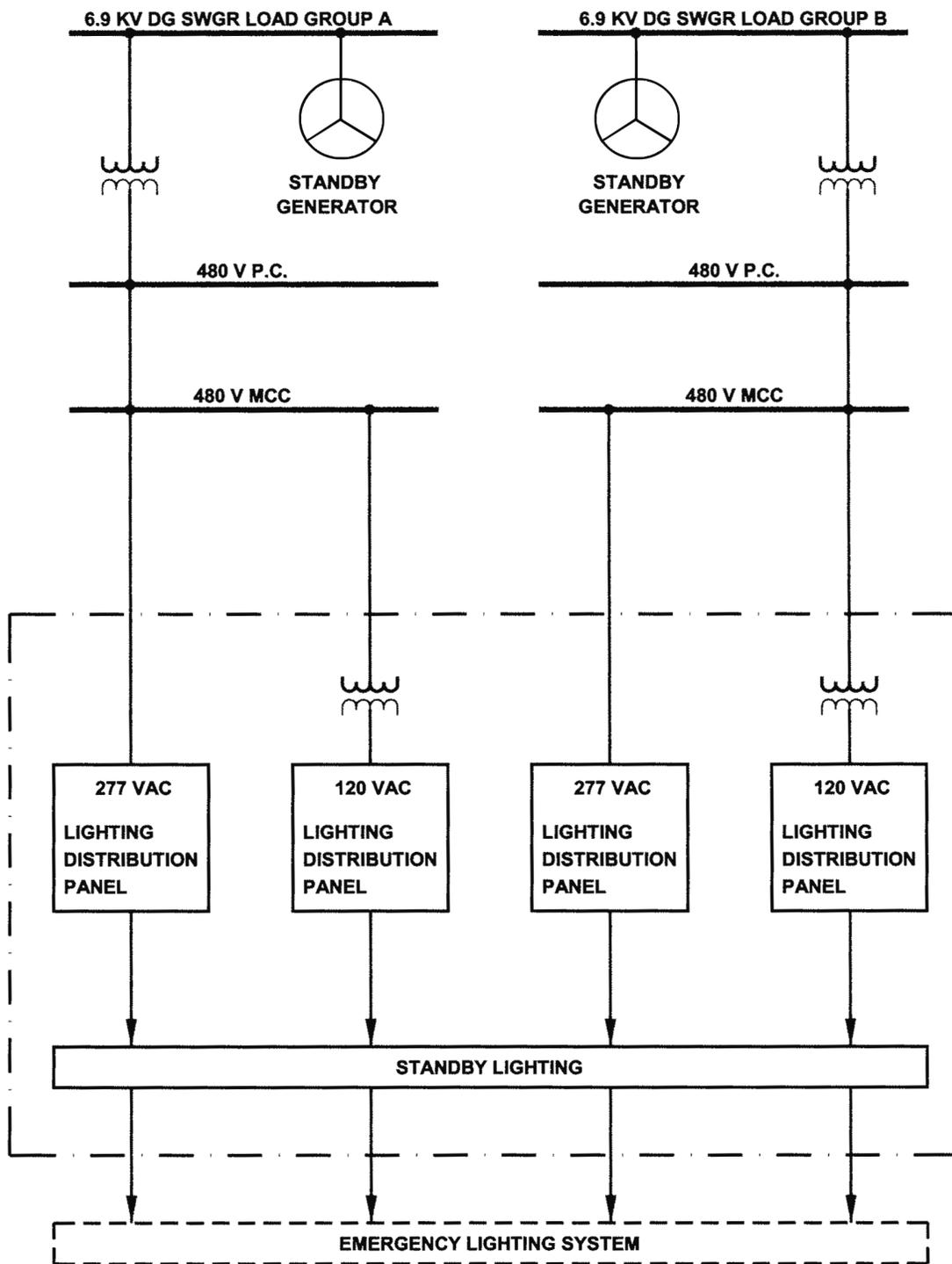


Figure 9.5-6. Standby Lighting System Simplified Block Diagram

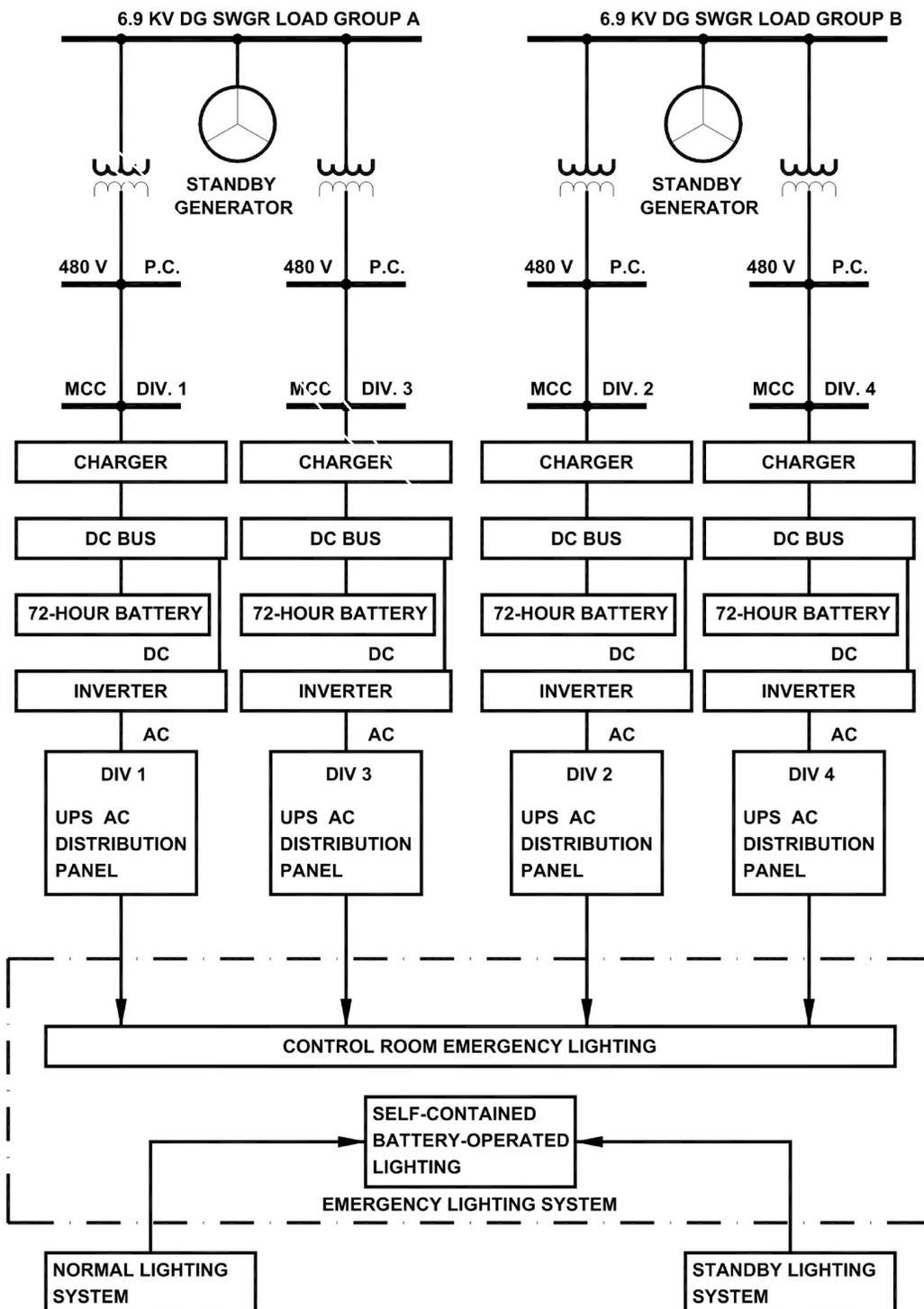


Figure 9.5-7. Emergency Lighting System Simplified Block Diagram

**Figure 9.5-8. DELETED**

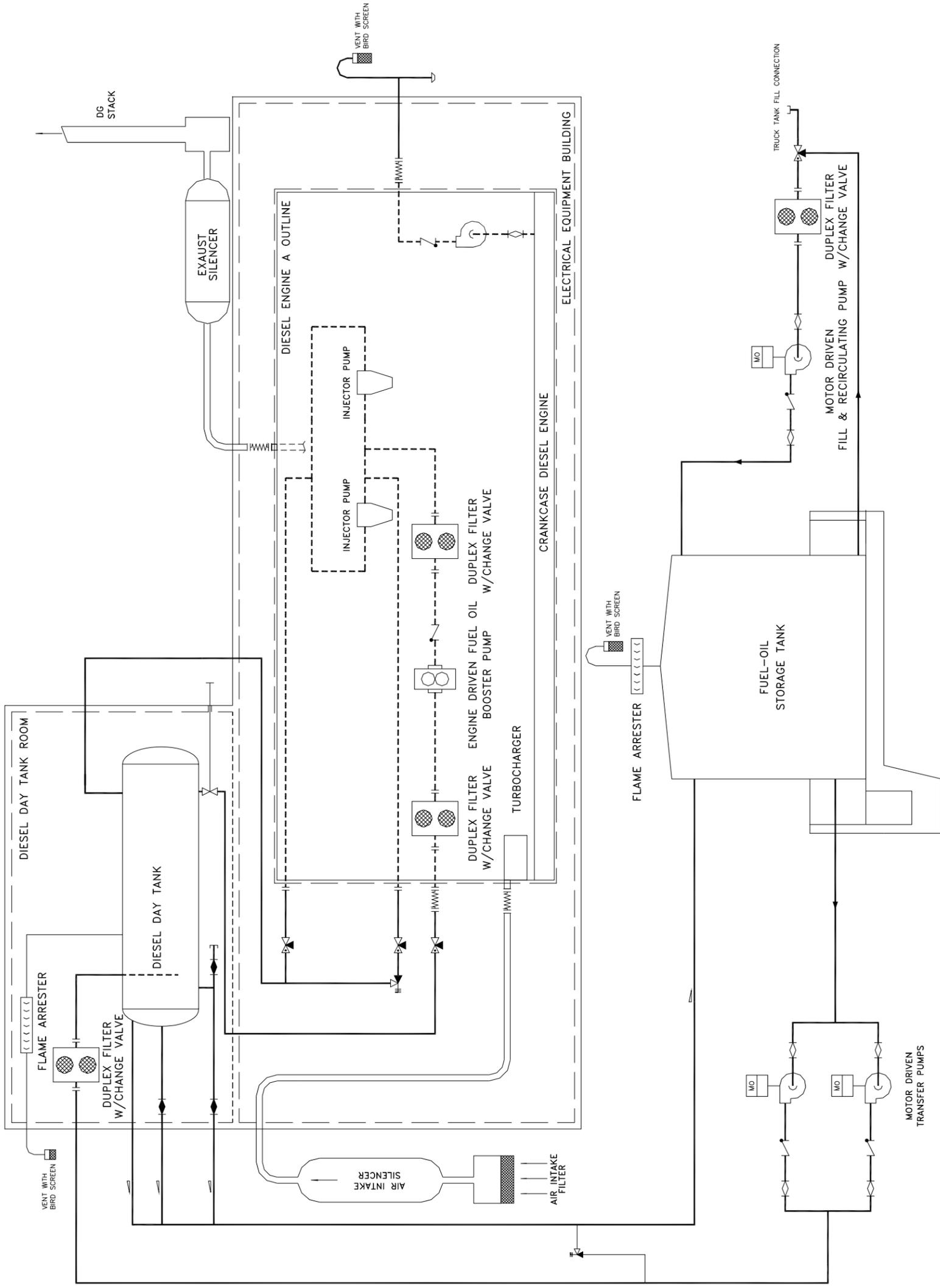


Figure 9.5-9. Diesel Generator Fuel Oil Storage and Transfer System & Air Intake and Exhaust System Diagram

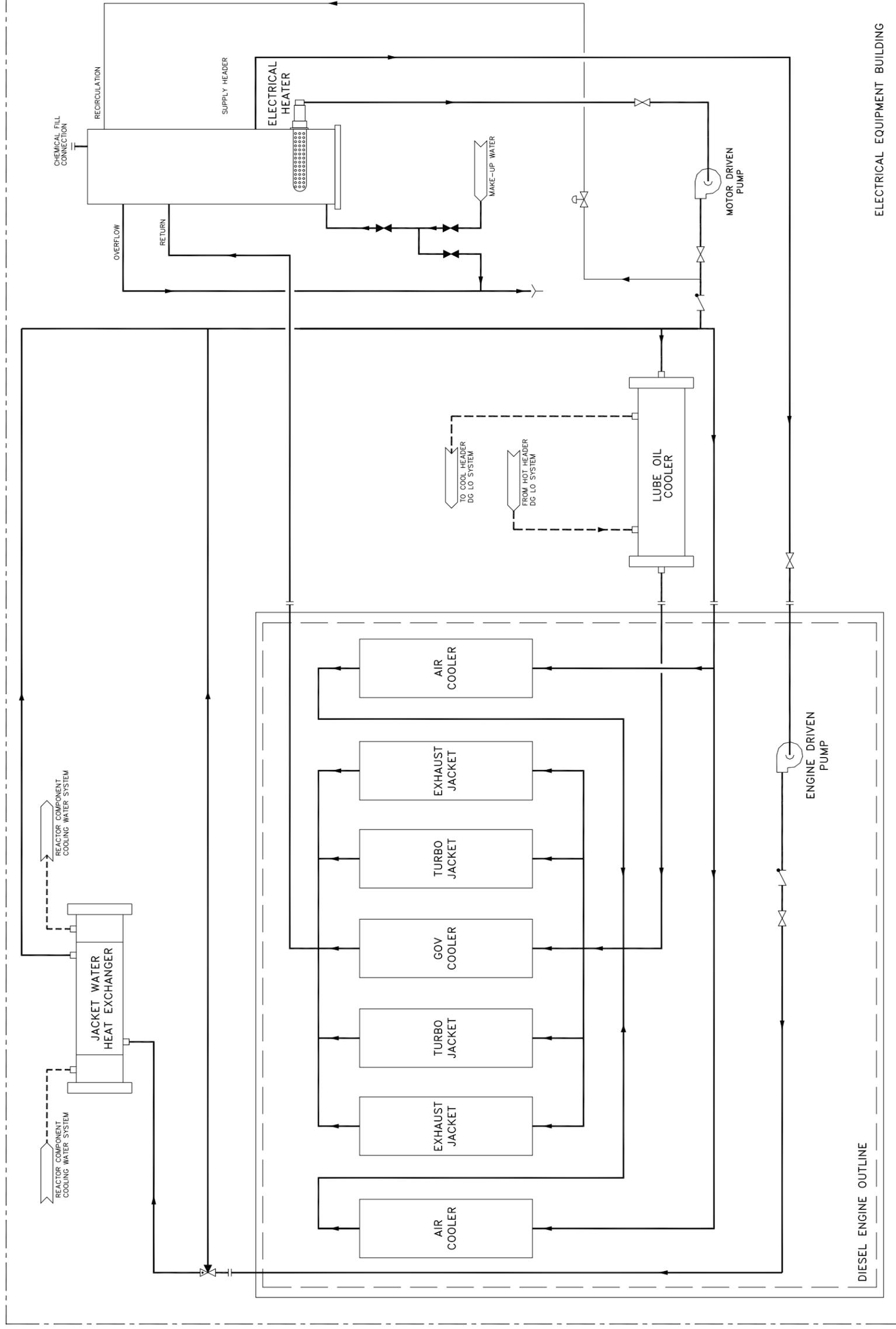


Figure 9.5-10. Diesel Generator Jacket Cooling Water System Diagram

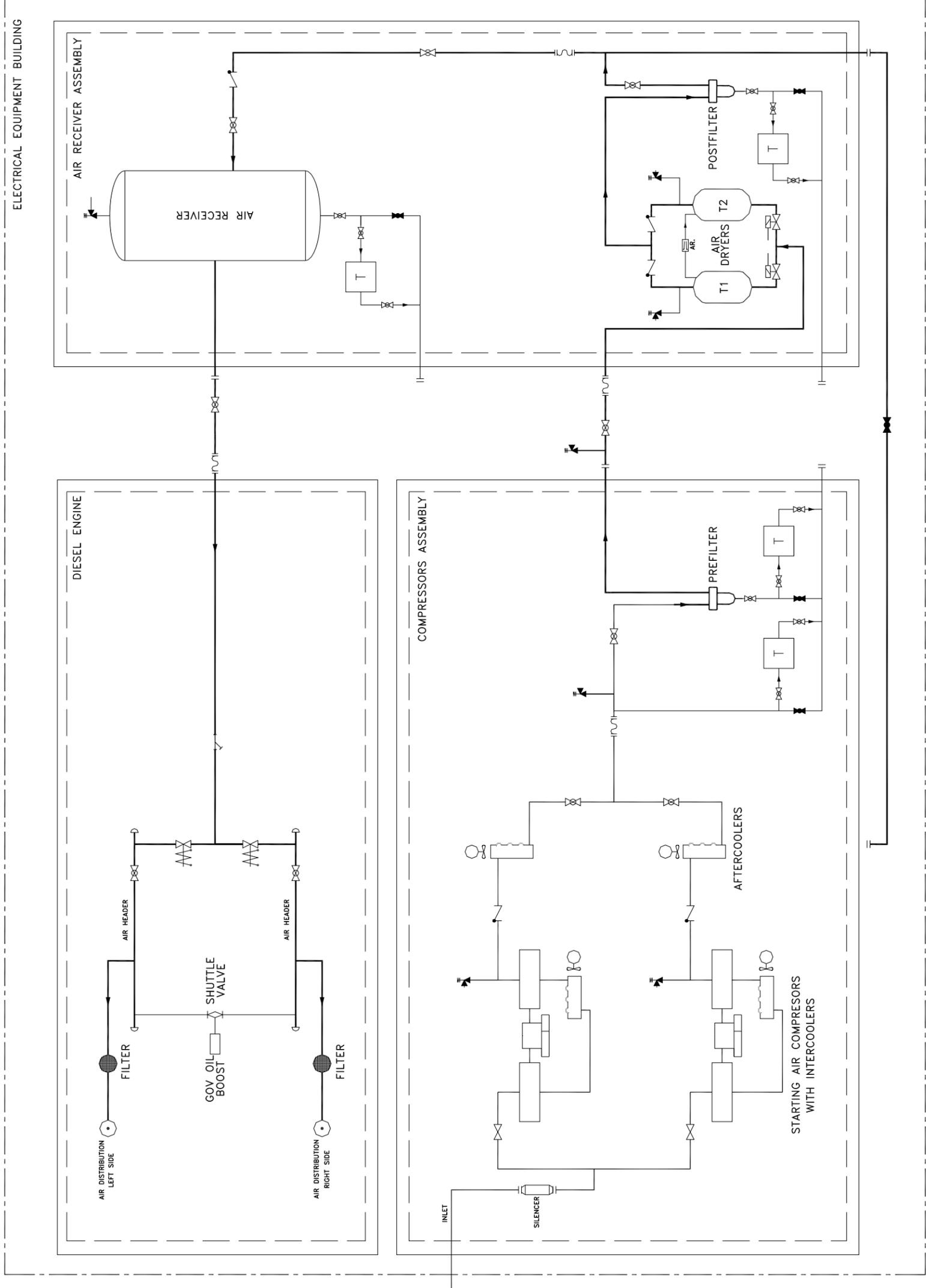


Figure 9.5-11. Diesel Generator Starting Air System Diagram

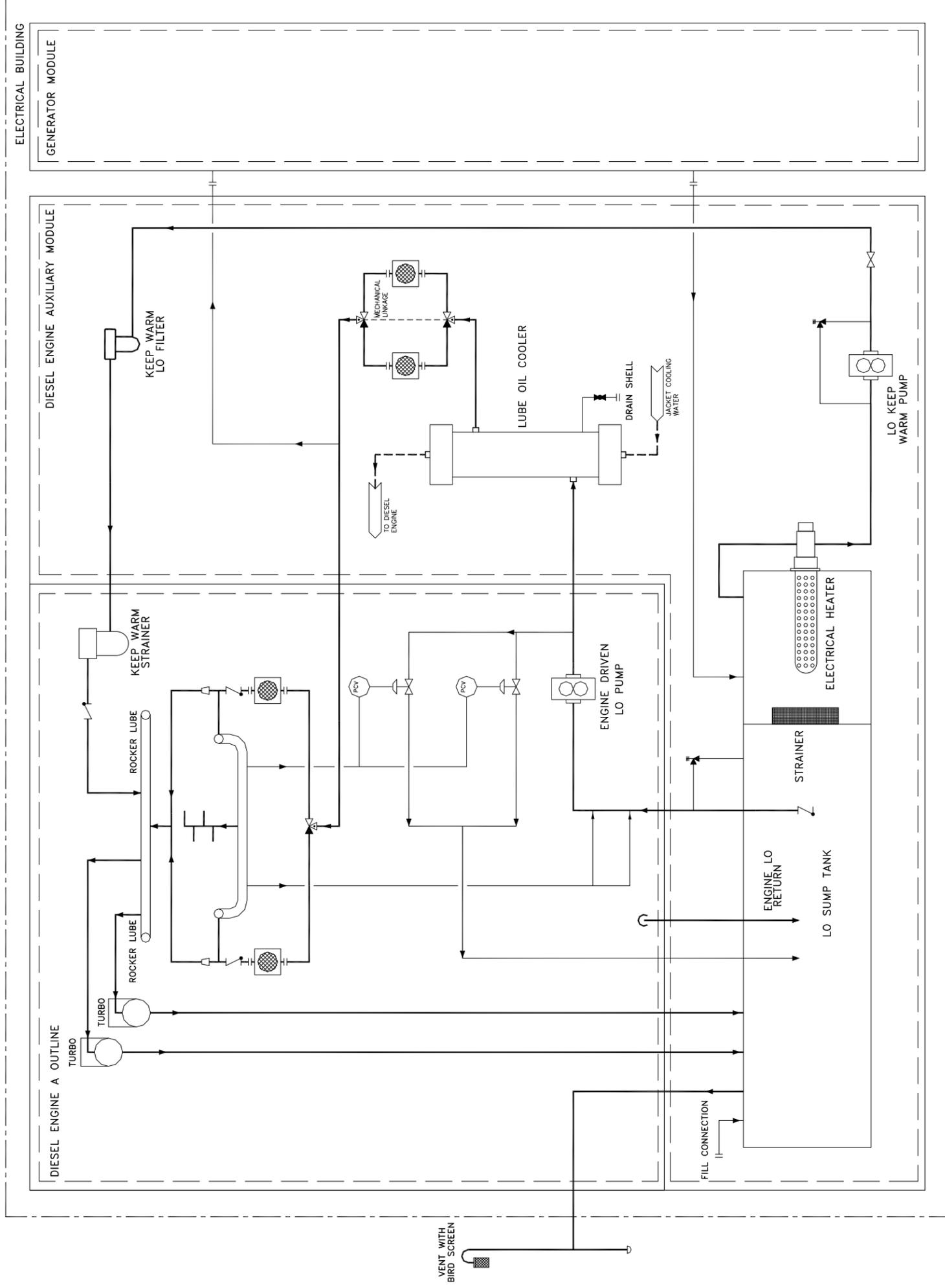


Figure 9.5-12. Diesel Generator Lubrication System Diagram