

August 8, 2007

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Vermont Yankee Nuclear Power Station

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JULY 18, 2007, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE VERMONT YANKEE NUCLEAR POWER STATION LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., held a telephone conference call on July 18, 2007, to discuss and clarify the staff's draft request for additional information (D-RAI) and the supplemental response to an RAI concerning the Vermont Yankee Nuclear Power Station license renewal application. The telephone conference call was useful in clarifying the intent of the staff's RAIs.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

***/RA/***

Jonathan G. Rowley, Project Manager  
License Renewal Branch B  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosures:

1. List of Participants
2. List of Requests for Additional Information

cc w/encls: See next page

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TELEPHONE CONFERENCE CALL  
VERMONT YANKEE NUCLEAR POWER STATION  
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS  
JULY 18, 2007

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**DRAFT REQUEST FOR ADDITIONAL INFORMATION (D-RAI)  
VERMONT YANKEE NUCLEAR POWER STATION  
LICENSE RENEWAL APPLICATION**

**JULY 18, 2007**

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., held a telephone conference call on July 18, 2007, to discuss and clarify the following draft request for additional information (D-RAI) and the supplemental response to an RAI concerning the Vermont Yankee Nuclear Power Station (VYNPS) license renewal application (LRA).

**D-RAI 4.3.3-1:**

The LRA for the VYNPS listed several reactor coolant pressure boundary components that did not receive a plant-specific metal fatigue analysis and cumulative usage factor (CUF). The applicant generated LRA Commitment No. 27 to address this issue. In the commitment, the applicant committed to take one of the following options or a combination of options to ensure that the time-limited aging analysis (TLAA) on environmentally-assisted fatigue (EAF) of these components, and any other Class 1 components requiring reanalysis for EAF, will be acceptable under 10 CFR 54.21(c)(1) for the period of extended operation:

Option (1): either refine the existing fatigue analyses or perform new fatigue analyses using at least one of four sub-options under this option.

Option (2): manage aging by an aging management program for the period of extended operation.

Option (3): repair or replace the impacted component prior to exceeding a CUF of 1.0.

In order to make a determination under the requirements of 10 CFR 54.21(c)(1), the staff requests that the applicant provide additional information on the option (or options) that will be used to comply with the actions committed under LRA Commitment No. 27. Specifically, the methodology that will be used for the chosen option or options needs to be described in sufficient detail such that the staff can perform a review and make a determination on the acceptability of the option when implemented. The staff requests that:

- A. If Option (1) will be the option used to comply with the pertinent commitment, describe the methodology and the process that will be used to ensure that assumptions, transients, cycles, external loadings,  $F_{en}$  values, and analysis methods are valid for the refined or new fatigue analyses.
- B. If Option (2) will be the option used to comply with the pertinent commitment, describe the AMP in sufficient detail with regard to inspection scope, inspection methods, inspection frequency, and inspection qualification techniques.
- C. If Option (3) will be the option used to comply with the pertinent commitments, describe how the repair or replacement activity will be implemented in accordance with applicable repair or replacement requirements of the ASME Code Section XI.

**Discussion:** The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

**Follow-up questions to RAI 2.1-2**

1. For the Service Water, Circulating Water, and Reactor Water Cleanup Systems, has the applicant identified the non-safety-related portions of the system, attached to safety-related systems, structures, and components (SSCs), which are required to be included within the scope of license renewal in accordance with 10 CFR 54.4(a)(2)? Are all scoping activities complete and have all component types been identified and included in the applicable tables in the LRA?
2. Are there any non-safety-related systems for which the applicant has not identified the non-safety-related portion of the system, attached to safety-related SSCs, which are required to be included within the scope of license renewal in accordance with 10 CFR 54.4(a)(2)?
3. For all non-safety-related systems, are all scoping activities complete and have all component types been identified and included in the applicable tables in the LRA?
4. Is the applicant taking credit for, or committing to perform, any future scoping activities?

**Discussion:** The applicant provided a supplemental response to these follow-up questions in a July 3, 2007, letter to the NRC. Based on the discussion with the applicant, the staff indicated that the supplemental response to this RAI requires clarification. Therefore, the applicant will clarify the information.

Memo to Entergy Nuclear Operations, Inc. from J. Rowley dated August 8, 2007

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