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SDP PHASE 1 SCREENING WORKSHEET FOR <u>INITIATING EVENTS</u>, <u>MITIGATION</u> <u>SYSTEMS</u>, AND <u>BARRIERS</u> CORNERSTONES

Reference/Title (LER #, Inspection Report #, etc):

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Performance Deficiency (concise statement clearly stating deficient licensee performance):

Incorrect fuses placed in PORV operating circuits.

Factual Description of Condition (statement of facts known about the condition that resulted from the performance deficiency, without hypothetical failures included): PORVs would not have opened in SGTR or bleed and feed situation.

System(s)/Train(s) Degraded by Condition: Both PORVs inoperable.

Licensing Basis Function of System(s)/Train(s): SGTR, loss of AFW

Other Safety Function of System(s)/Train(s):

Maintenance Rule Category (check one):

_ risk-significant _____non risk-significant

Time condition existed or is assumed to have existed: 2 days

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0609, App A

Issue Date: 03/23/07

CORNERSTONES AND FUNCTIONS DEGRADED AS A RESULT OF DEFICIENCY () Check the appropriate boxes								
INITIATING EVENTS CORNERSTONE	MITIGATION SYSTEMS CORNERSTONE	BARRIERS CORNERSTONE						
Primary System LOCA initiator contributor - (e.g., RCS leakage from pressurizer heater sleeves, RPV piping penetrations, CRDM nozzles, PORVs, SRVs, ISLOCA issues, etc.)	 Core Decay Heat Removal Degraded Short Term Heat Removal Degraded Primary (e.g., Safety Inj, 	 RCS Boundary as a mitigator following plant upset (e.g., pressurized thermal shock). Note: all other RCS boundary issues, such as leaks, will be considered under the Initiating Events Cornerstone. 						
 Transient initiator contributor (e.g., reactor/turbine trip, loss of offsite power, loss of service water, main steam/feedwater piping degradations, etc.) Fire initiator contributor (e.g., transient loadings and combustibles, hotwork) 	 Imain (e.g., Curvey, My, [main feedwater, HPCI, and RCIC - BWR only]) <u>—</u> High Pressure <u>—</u> Low Pressure Secondary - PWR only (e.g. AFW, main feedwater, ADVs) Long Term Heat Removal Degraded (e.g., ECCS sump recirculation, suppression 	 Containment Barrier Degraded Reactor Containment Degraded Actual Breach or Bypass Heat Removal, Hydrogen or Pressure Control Degraded Control Room, Aux 						
Internal/external flooding initiator contributor	 pool) Reactivity Control Degraded Seismic/Fire/Flood/Severe Weather Protection Degraded 	Bldg/Reactor Bldg, or Spent Fuel Bldg Barrier Degraded						

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SDP PHASE 1 SCREENING WORKSHEET FOR IE, MS, and B CORNERSTONES Check the appropriate boxes ✓

IF the finding is assumed to degrade:

1. fire protection defense-in-depth strategies involving: detection, suppression (equipment for both manual and automatic), barriers, fire prevention and administrative controls, and post fire safe shutdown systems, **THEN STOP. Go to** IMC 0609, Appendix F. Issues related to performance of the fire brigade are not included in Appendix F and require NRC management review.

2. steam generator tube integrity, THEN STOP. Go to IMC 0609, Appendix J.

- 3. the safety of an operating reactor, **THEN IDENTIFY** the degraded cornerstone(s):
 - Initiating Event
 - □ Mitigation Systems
 - □ RCS Barrier (e.g., PTS issues)
 - □ Fuel Barrier
 - Containment Barriers

IF TWO OR MORE of the above cornerstones are degraded -> **THEN STOP. Go to Phase 2.**

IF ONLY ONE of the above cornerstones is degraded, **THEN CONTINUE** in the appropriate column on page 4 of 5 of this worksheet.

NOTE: When assessing the significance of a finding affecting multiple cornerstones, the finding should be assigned to the cornerstone that best reflects the dominant risk of the finding.

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Init	ating Events Cornerstone	Miti	gation Systems Cornerstone	RCS Barrier or Fuel		ontainment Barriers
<u>LO</u> 1.	<u>CA Initiators</u> Assuming worst case degradation, would the finding result in exceeding the Tech Spec limit for identified RCS leakage or could the finding have likely affected other mitigation systems resulting in a total loss of their safety function.	1. 2.	Is the finding a design or qualification deficiency confirmed <u>not</u> to result in loss of operability per "Part 9900, Technical Guidance, Operability Determination Process for Operability and Functional Assessment." If YES, screen as Green.	Barrier 1. <u>RCS Barrier</u> (e.g.,pressurized thermal shock issues) Stop. Go to Phase 3. 2. <u>Fuel Barrier</u> Screen as Green.		Does the finding only represent a degradation of the radiological barrier function provided for the control room, or auxiliary building, or spent fuel pool, or SBGT system (BWR)?
	lf YES → Stop. Go to Phase 2.	□ 2.	If NO, continue. Does the finding represent a			If YES → screen as Green.
□ Tra	If NO, screen as Green. nsient Initiators	0	loss of system safety function? If YES →Stop. Go to Phase		0	If NO, continue.
1.	Does the finding contribute to both the likelihood of a reactor trip AND the likelihood that mitigation equipment or functions will not be available?	□ 3.	If NO, continue.Does the finding represent actual loss of safety function of		2.	Does the finding represent a degradation of the barrier function of the control room against smoke or a
	If YES→Stop. Go to Phase 2.		a single Train, for > its Tech Spec Allowed Outage Time?			toxic atmosphere? If YES → Stop.
	ernal Event Initiators Does the finding increase the likelihood of a fire or		If YES → Stop. Go to Phase 2.			Go to Phase 3. If NO, continue.
	If YES → Use the IPEEE or other existing plant-specific analyses to identify core damage scenarios of concern and factors that increase the frequency. Provide this input for Phase 3 analysis.	4.	If NO, continue. Does the finding represent an actual loss of safety function of one or more non-Tech Spec Trains of equipment designated as risk-significant per 10CFR50.65, for >24 hrs? If YES → Stop. Go to Phase 2.		3.	Does the finding represent an actual open pathway in the physical integrity of reactor containment, or involve an actual reduction in defense-in-depth for the atmospheric
	If NO, screen as Green.		If NO, continue.			pressure control or hydrogen control functions of the reactor
		5.	Does the finding screen as potentially risk significant due to a seismic, flooding, or severe weather initiating event, using the criteria on page 5 of this Worksheet?			containment? If YES → Stop. Go to Appendix H of IMC 0609.
			If YES → Use the IPEEE or other existing plant-specific analyses to identify core damage scenarios of concern and provide this input for Phase 3 analysis.			If NO, screen as Green.
			If NO, screen as Green.			

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