



July 27, 2007

L-MT-07-058
10 CFR 50.55a

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Monticello Nuclear Generating Plant
Docket 50-263
License No. DPR-22

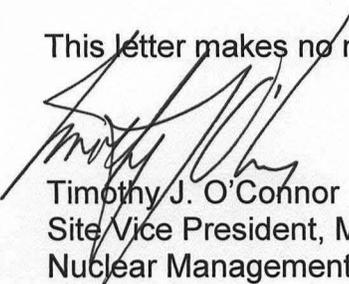
Cycle No. 23 Inservice Inspection Summary Report for Inservice Inspection and ASME Section XI Repairs and Replacements

Pursuant to 10 CFR Part 50, Section 50.55a and IWA-6240 of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1995 Edition, with the 1996 Addenda, the Nuclear Management Company, LLC (NMC) hereby submits the Cycle No. 23 Inservice Inspection Summary Report for Inservice Inspections (ISI) and ASME Section XI Repairs and Replacements completed from April 13, 2005 to April 28, 2007.

Pursuant to 10 CFR Part 50, Section 50.55a(b)(2)(ix)(D), evaluations for ASME Section XI, Subsection IWE Class MC components that did not meet the acceptance standards of IWE-3410-1 are included. The evaluations are applicable to the examinations performed during the refueling outage for the first interval of the metal containment inspection program.

This summary report also provides amended information to the previously submitted Cycle No. 22 ISI Summary Report, including a supplemental NIS-1 Form and a supplemental NIS-2 Form.

This letter makes no new commitments or changes to any existing commitments.



Timothy J. O'Connor
Site Vice President, Monticello Nuclear Generating Plant
Nuclear Management Company, LLC

Enclosure

cc: Administrator, Region III, USNRC (2 copies of enclosure)
Project Manager, Monticello, USNRC (1 copy of enclosure)
Resident Inspector, Monticello, USNRC (1 copy of enclosure)
Mr. Joel Amato, Chief Boiler Inspector, State of Minnesota (1 copy of enclosure)
Mr. Paul Fisher, Hartford Insurance (1 copy of enclosure)

ENCLOSURE 1

**NUCLEAR MANAGEMENT COMPANY, LLC
MONTICELLO NUCLEAR GENERATING PLANT
DOCKET 50-263**

**INSERVICE INSPECTION SUMMARY REPORT
CYCLE NO. 23 REFUELING OUTAGE**



Nuclear Management Company, LLC
700 1st Street
Hudson, WI 54016



Xcel Energy, LLC
800 Nicollet Mall
Minneapolis, MN 55402

INSERVICE INSPECTION

SUMMARY REPORT

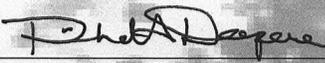
Monticello Nuclear Generating Plant
2807 West County Road 75
Monticello, MN 55362

Commercial Service Date: June 30, 1971

Cycle No. 23 Refueling Outage

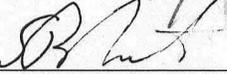
Outage Dates: March 14, 2007 to April 28, 2007

Inservice Inspection Interval 4, Periods 1 and 2

Prepared By: 
Section XI ISI Coordinator

Reviewed By: 
Site NDE Level III

Approved By: 
Supervisor, MIR

Approved By: 
Manager, Program Engineering

ANII: 
Hartford Steam Boiler - CT

Report Date: July 26, 2007

TABLE OF CONTENTS

<u>SECTIONS</u>	<u>PAGES</u>
I. Summary	4
1.0 Introduction	
2.0 Personnel	
3.0 Examination Reports, Equipment, and Materials	
4.0 Inspection Summary	
5.0 Abbreviations	
II. NIS-1 Form for Cycle 23	2
III. NIS-2 Summary for Cycle 23	5
NIS-2 Forms for Cycle 23	140
IV. New NIS-1 Form for Cycle 22 Supplemental Information	2
V. NIS-2 Form for Cycle 22 Supplemental Information	3
<u>APPENDICES</u>	<u>PAGES</u>
A. Class 1, 2, 3, Augmented, and Non-Code Examinations (Sorted by Class, Category, and Item No.)	19
B. Reactor Vessel IVVI Examinations	7
C. System Pressure Test Examinations	2
D. Snubber Functional Testing	1
E. Engineering Evaluations per Subsection IWE-3122.4 (1992 Edition with 1992 Addenda) and 10CFR50.55a(b)(2)(ix)(D)	
1. CAP CE 01082766-03 for degraded containment nuclear coatings	28
• CAP CE 01082766-03 Attachment 1	
• CAP CE 01082766-03 Attachment 2	
• CAP CE 01082766-03 Attachment 3	
2. CAP CE 01084149-01 for Arc Strikes on Torus Shell Plates	2

I. Summary

Page 1 of 4

1.0 Introduction

This report encompasses all Inservice Inspection (ISI) activities performed during Cycle 23 of operation at Monticello Nuclear Generating Plant (MNGP). Cycle 23 began April 13, 2005, immediately following completion of the 22nd refueling outage. This report also includes activities performed during the 23rd refueling outage that began March 14, 2007 and ended April 28, 2007.

All examinations and tests for Cycle 23 were scheduled and conducted in accordance with the 4th Interval ISI Plan. The majority of the exams and tests were credited to the 2nd period of the 4th ISI Interval. Nine Class 2 and Class 3 system pressure tests conducted during Cycle 23 were credited to the 1st Period.

Also included in this report is supplemental information for the Cycle 22 ISI Summary Report submitted in July 2005.

2.0 Personnel

Visual and nondestructive examinations for inservice inspection (ISI) and preservice inspection (PSI) activities were performed by Nuclear Management Company (NMC), Lambert-MacGill-Thomas (LMT), International Quality Consultants (IQC), Inc., and AREVA-NP. Hartford Steam Boiler Inspection & Insurance Co. of Connecticut (HSB-CT) provided the Authorized Inspection. Personnel certification records are maintained by NMC.

3.0 Examination Reports, Equipment and Materials

The examination reports listed in the attached appendices contain references to procedures, equipment, and materials used to complete specific examinations. Records of the reports and referenced information are maintained by NMC.

4.0 Inspection Summary

In accordance with the 4th Interval ISI Plan and ISI Relief Request #1, Class 1 and 2 piping welds were examined using methodologies applicable to Risk-Informed ISI (RI-ISI). The remainder of the components and their supports were examined per the requirements set forth in American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI.

I. Summary (continued)

Page 2 of 4

Comprehensive in-vessel visual inspections (IVVI) of reactor vessel internal components were conducted to ensure continued integrity of the Reactor Pressure Vessel (RPV) internals. Additionally, enhanced visual inspections (EVT-1) were performed per the recommendations set forth in Boiling Water Reactor Vessel Internals Project (BWRVIP) guideline BWRVIP-03.

Examinations performed during Cycle 23 are listed in attached appendices. Code Case N-460, which allows a reduction of examination volume coverage <10% to be considered essentially 100%, was applied to exams where coverage was reduced due to interference or part geometry. These limitations are recorded on the examination reports.

Components with exam limitations that did not meet the required coverage requirements of essentially 100% will be included in a 10CFR50.55a Request for Relief under separate cover.

Results of the Class 1, 2, and 3 ISI and PSI examinations, identified conditions, and corrective actions are noted and summarized in the attached appendices. Listed anomalies were either corrected, or an engineering evaluation was performed to accept "as-is" conditions. Other non-code and augmented exams are also provided in the attached appendices for site information tracking purposes only.

Evaluations were also performed during the 2007 outage for the Containment Inspection Examination Plan, Interval 1, Period 3, Cycle 23 examinations that exceeded acceptance standards of Table IWE-3410-1. (Code of Record is ASME Section XI, 1992 Edition with 1992 Addenda) These evaluations were performed per IWE-3122.4 and per 10CFR50.55a(b)(2)(ix)(D) as an alternative to IWE-2430. As required by 10CFR50.55a(b)(2)(ix)(D)(1), the evaluations are included in the attached appendices of this report.

Results of system pressure tests and snubber functional tests performed during Cycle 23 are summarized in attached appendices. The pressure testing appendix includes 1st Period pressure tests that were not completed prior to beginning Cycle 23. These test were subsequently completed during Cycle 23 prior to the end of the 1st Period .

Repair-replacement activities for Cycle 23 were performed to the requirements of the 4th ISI Interval Relief Request #7 which is based on ASME Section XI, 2001 Edition with no Addenda as approved by the Nuclear Regulatory Commission (NRC). A summary of the activities and copies of the NIS-2 forms are attached to this report in the applicable appendix.

I. Summary (continued)

Page 3 of 4

During Cycle 23, an audit of Cycle 22's maintenance activities identified the replacement of a valve sub-component that constituted a repair/replacement activity. However, no Repair/Replacement Plan had been prepared prior to, or during performance of the work. This condition was identified subsequent to submittal of the Cycle 22 ISI Summary Report, thus no NIS-2 form for this activity was submitted with the Cycle 22 ISI Summary Report. As part of the corrective action process, a Repair/Replacement Plan was prepared and approved for the activity. An NIS-2 form was also prepared and signed to document the Repair/Replacement activity. This Cycle 22 supplement NIS-2 form, along with a new Cycle 22 supplemental NIS-1 form, are included in this Summary Report.

In conclusion, based on the results of the Cycle 23 ISI activities, the integrity of the plant systems has been adequately maintained.

I. Summary (continued)

Page 4 of 4

5.0 List of Abbreviations

The Summary Report Examination Appendices contains several abbreviations identified below (System acronyms, plant component identifiers, and standard Section XI identifiers are not included in list):

AUG	=	Augmented Exam Program
B-	=	Bolt (plus number identifier)
BWRVIP	=	Boiling Water Reactor Vessel and Internals Project
CA	=	Corrective Action
CAP	=	Corrective Action Process
CE	=	Condition Evaluation
D	=	Downstream
DC / EC	=	Design / Engineering Change
DS	=	Downstream
EVT	=	Enhanced Visual Examination
GEO	=	Geometry / Geometric Reflector
H-	=	Hanger (plus number identifier)
ISO	=	ISI Isometric Drawing
IVVI	=	In-Vessel Visual Inspection
LS	=	Longitudinal Seam
MT	=	Magnetic Particle Examination
NC	=	Non-Code Commitment Exam
NRI	=	No Reportable Indications
OE	=	Operating Experience
PM	=	Preventative Maintenance
PMT	=	Post-Maintenance Testing
PSI	=	Pre-Service Inspection
PT	=	Liquid Penetrant Examination
PEI	=	Program Engineering Instruction
PP	=	PassPort
RFO	=	Refueling Outage
RI	=	Reportable Indication
RPV	=	Reactor Pressure Vessel
R/R	=	Repair / Replacement
U&D	=	Upstream & Downstream
U	=	Upstream
US	=	Upstream
UT	=	Ultrasonic Examination
V-	=	Valve (plus number identifier)
VT	=	Visual Examination
W-	=	Weld (plus number identifier)
WCR	=	Weld Control Record
WO	=	Work Order

II. Form NIS-1 Information

Page 1 of 2

Form NIS-1 Owner's Data Report for Inservice Inspections
As required by the Provisions of the ASME Code Rules

-
1. **Owner:** Xcel Energy, LLC / Nuclear Management Company, LLC
Address: 800 Nicollet Mall, Mpls, MN 55402 / 700 1st St., Hudson WI 54016
2. **Plant:** Monticello Nuclear Generating Plant
Address: Monticello, MN 55362
3. **Plant Unit:** 1 4. **Owner (Certificate of Authorization):** N/A
5. **Commercial Service Date:** 6/30/71 6. **National Board No. for Unit:** N/A

7. **Components Inspected:**

<u>Component or Appurtenance</u>	<u>Manufacturer or Installer</u>	<u>Manufacturer or Installer Serial No.</u>	<u>State or Province No.</u>	<u>National Board No.</u>
Reactor Pressure Vessel	Chicago Bridge & Iron	B-4697	---	---

(See appendices for components inspected during the cycle)

8. **Examination Dates:** 04/13/05 to 04/28/07 .
9. **Inspection Interval:** 05/01/03 to 05/31/12 .
10. **Applicable Edition of Section XI:** 1995 Addenda: 1996 .
11. **Abstract of Examinations.** Include a list of examinations and a statement concerning status of work required for current interval.
- See appendices A through D (Attached)
12. **Abstract of Conditions Noted.**
- See appendices A through E (Attached)
13. **Abstract of Corrective Measures Recommended and Taken.**
- See appendices A through E (Attached)

II. Form NIS-1 Information

Page 2 of 2

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Certification of Authorization No. (if applicable) N/A Expiration Date N/A

Date July 26 20 07 Signed XCEL ENERGY / NMC By [Signature]
Owner Section XI ISI Coordinator

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/13/05 to 7/26/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions NB 11863 A, N.E. MN 004928-00
National Board, State, Province, Endorsements

Date JULY 26 2007

III. Form NIS-2 Information

Cycle 23 NIS-2 Summary

R/R ITEM NUMBER	WORK DOCUMENT(S)	COMPONENT OR SYSTEM	WORK DESCRIPTION
2005-23-0001	WO-311347	P-109 (Spare Pump)	Rebuilt spare RHRSW pump bowl assembly
2005-23-0002	WO-507209 (PP 141066) WO-506359 (PP 140418)	#12 Recirc Pump (P-200B)	Replacement of stuffing box cover
2005-23-0004	WO-156335	TCV-4030B	Replaced existing valve
2005-23-0005	WO-507758	P-109B, #12 RHRSW Pump	Replaced existing pump bowl assembly w/ rebuilt spare
2005-23-0006	WO-136399	SR-26	Removed welded stanchion, realigned and repaired by welding
2006-23-0008	WO-141353	SS-14	Removed existing snubbers and replaced with rebuilt spares
	WO-141354	SS-184	
	WO-141355	SS-316	
	WO-141356	SS-13	
	WO-141357	SS-38A	
	WO-141358	SS-17A	
	WO-141359	SS-38B	
	WO-141360	SS-121	
	WO-141361	SS-11	
	WO-141362	SS-12	
	WO-141363	SS-37	
	WO-141365	SS-17B	
WO-141366	CRD-H227		

III. Form NIS-2 Information

Cycle 23 NIS-2 Summary

R/R ITEM NUMBER	WORK DOCUMENT(S)	COMPONENT OR SYSTEM	WORK DESCRIPTION
(Cont'd) 2006-23-0008	WO-141369	SS-303	(Cont'd) Removed existing snubbers and replaced with rebuilt spares
	WO-141371	CRD-H232	
	WO-141372	SS-8AR	
	WO-141374	SS-8BR	
	WO-141375	SS-2AR	
	WO-141377	SS-1BR	
	WO-141378	SS-3AR	
	WO-141379	SS-1AR	
	WO-141382	SS-20	
	WO-141383	SS-35	
2006-23-0010	WO-153676 WO-153175	P-200B	Replaced pump stuffing box (EC-806 and EC-10317)
2006-23-0011	WO-153676	P-200B	Replaced 16 pump case-to-stuffing box nuts (EC-806)
2006-23-0013	WO-151841	MO-2373	Replaced valve and associated piping (EC-748)
2006-23-0014	WO-285967 WO-285967	MO-2374	Replaced valve and associated piping (EC-748)
2006-23-0015	WO-265697	ESW-1-2	Removed / restored disc stem-to-nut tack weld to permit inspection
2006-23-0016	WO-289152	TW(H)-12	Modified support by welding, integral attachment weld removed (EC-775)
2006-23-0017	WO-289151	TWH-86	Modified support by installing replacement spring can (EC-775)
2006-23-0018	WO-285768 WO-286583	CRH	Replaced 6 scram discharge volume level switches (EC-901)

III. Form NIS-2 Information

Cycle 23 NIS-2 Summary

R/R ITEM NUMBER	WORK DOCUMENT(S)	COMPONENT OR SYSTEM	WORK DESCRIPTION
2006-23-0019	WO-140381	MO-2008	Replace bonnet
2006-23-0020	WO-285273 WO-267425 WO-284077	EDG-ESW A and B	Replaced 2 check valves (SW-16 and SW-18) with manual gate valves (SW-228 and SW-229) and associated piping. (EC-768)
2006-23-0021	WO-289151	SR-852	Installed new pipe restraint (EC-775)
2006-23-0022	WO-152929	P-111B	Replaced P-111B (EDG-ESW Pump 12)
2006-23-0023	WO-139591	ESW-32	Replaced valve and pup piece
2006-23-0024	WO-142128	ESW-31	Replaced valve and pup piece
2006-23-0025	WO-141349	SS-707	Changed out snubber with new spare and also replaced 2 end pins
2006-23-0026	WO-280592	Snubber, s/n 529	Replaced main cylinder
2006-23-0027	WO-280592	Snubber, s/n 044	Replaced main cylinder, piston, piston rings, piston rod, piston rod nut
2006-23-0028	WO-280592	Snubber, s/n 460	Replaced valve manifold
2006-23-0029	WO-280592	Snubber, s/n 126	Replaced valve manifold
2006-23-0030	WO-280592	Snubber, s/n 157	Replaced main cylinder, piston rings, piston
2006-23-0031	WO-135620	RV-11-39B (#12 SBLC Pump Disch RV)	Replaced nozzle and disc
2006-23-0032	WO-280592	Snubber, s/n 131	Replaced valve manifold
2006-23-0033	WO-280592	Snubber, s/n 487	Replaced valve manifold
2006-23-0034	WO-280592	Snubber, s/n 466	Replaced valve manifold
2006-23-0035	WO-280592	Snubber, s/n 132	Replaced valve manifold
2006-23-0036	WO-280592	Snubber, s/n 148	Replaced valve manifold

III. Form NIS-2 Information

Cycle 23 NIS-2 Summary

R/R ITEM NUMBER	WORK DOCUMENT(S)	COMPONENT OR SYSTEM	WORK DESCRIPTION
2006-23-0037	WO-280592	Snubber, s/n 132	Replaced main cylinder
2006-23-0038	WO-280592	Snubber, s/n 102	Replaced main cylinder
2006-23-0039	WO-280592	Snubber, s/n 154	Replaced main cylinder
2006-23-0040	WO-280592	Snubber, s/n 174	Replaced main cylinder
2006-23-0041	WO-280592	Snubber, s/n 133	Replaced valve manifold
2006-23-0042	WO-280592	Snubber, s/n 125	Replaced valve manifold
2006-23-0043	WO-152929	P-111B (Spare)	Repaired small crack in 1st intermediate bowl
2007-23-0044	WO-154343	RPV - WRFM	Removed 2 abandoned in-place wide range flux monitors and replaced with new blank flanges and bolting (EC-7462)
2007-23-0045	WO-314220	Main Steam	Installed 64 strain gauges (2 locations on each of the 4 steam lines in the drywell, 8 strain gages per location) (EC-9174)
2007-23-0046	WO-288806	CS	Removed and reinstalled a section of pipe due to interference with adjacent work
2007-23-0047	WO-267368	RX Head Vent, Flange #3	Replaced 1 stud and 2 nuts
2007-23-0048	WO-314233	RCIC Steam	Removed / Reinstalled a section of pipe to allow repair work on AO-2-86D.
2007-23-0049	WO-322082	RHRWS-B	Replaced approximately 4 ft section of SW10-18"-GF pipe
2007-23-0050	WO-141889	Snubber, S/N 400	Replaced main cylinder
2007-23-0051	WO-288859	MO-1742	Replaced 14 bonnet studs and 28 nuts
2007-23-0052	WO-289152	MO-1987	Replaced 18 bonnet studs and 36 nuts
2007-23-0053	WO-289154	MO-2009	Replaced disc
2007-23-0054	WO-289151	MO-1750	Removed / restored disc to nut tack weld to permit disassembly and reassembly

III. Form NIS-2 Information

Cycle 23 NIS-2 Summary

R/R ITEM NUMBER	WORK DOCUMENT(S)	COMPONENT OR SYSTEM	WORK DESCRIPTION
2007-23-0056	WO-322070	RV-2-71C	Replaced main body with rebuilt spare
2007-23-0057	WO-322070	RV-2-71C	Replaced topworks with rebuilt spare
2007-23-0058	WO-291884	RV-2-71B	Replaced 5 bolts and 6 nuts for main body to topworks flange
2007-23-0059	WO-291592	CV-1729	Replaced valve body.
2007-23-0060	WO-267369	Drywell Head	Replaced 2 bolts and 3 nuts
2007-23-0061	WO-141358	SS-17A	Replaced 4 flange mounting bolts
2007-23-0062	WO-291592	CV-1729	Replaced plug
2007-23-0063	WO-291883	RV-2-71E	Replace 12 studs and 24 nuts for main body to piping inlet flange
2007-23-0064	WO-291885	RV-2-71G	Replace 12 bolts and 12 nuts for main body to topworks flange
2007-23-0065	WO-322083	RV-2-71A	Replace 12 bolts and 12 nuts for main body to topworks flange
2007-23-0066	WO-142056	MO-2035	Replaced 1 stud and 16 nuts, and the pressure retaining disc
2007-23-0067	WO-141016	RHRWS	Replaced approximately 4 ft section of piping and removed / re-installed, by welding, portion of restraint SR-396
2007-23-0068	WO-324116	RHRWS	Replaced approximately 4 ft section of piping
2007-23-0069	WO-323428	V15-H3 (PH-28)	Replaced slightly bent hanger rod
2007-23-0070	WO-325774	ESW1-4"-HBD	Replaced approximately 6" section of pipe and adjacent 90 degree elbow
2007-23-0071	WO-326373	ESW5-4"-HBD	Replaced 90 degree elbow

III. Form NIS-2 Information

NIS-2 Forms

(140 pages to follow)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2005-23-0001

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Design Change 75M041 is applicable to existing pumps. Work was performed by NMC/Xcel Energy (2807 W. Cty Rd 75, Monticello, MN 55362) and Johnston Pump Co., Nuclear Service Div. (2601 East 34th Street, Chattanooga, TN 37407). Visual exam performed, as applicable, on weld repaired area. (out-of-round male register ground down, restored with welding, and machined to tolerances). Post-reassembly pump performance testing was performed by Johnston Pump Co. per procedure JC-TP-0069, Rev.8.

Rebuild pump returned to stock for future use. Pressure testing and pump testing will be performed following installation under separate Work Order and R/R Plan.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR AND REPLACEMENT
repair/replacement

Signed

T. Randolph Dapere SECTION XI ISI COORD
Owner or Owner's Designee, Title

Date

January 18, 2006

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period DEC 3, 2003 to JAN 18, 2006, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Furt A. Dapere
Inspector's Signature

Commissions NB 11863 A, N, I MN 004028-CO
National Board, Province and Endorsements

Date JANUARY 18, 2006

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2005-23-0002

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/12/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 0506359, 0507209 (PP 140418, 141066) Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Reactor Recirc (REC), Loop B, Pump #12 (P-200B), Class 1
- (a) Applicable Construction Code ASME Section III, Class C, 1965 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Stuffing Box Cover	Sulzer Pump Co.	p/n 5122530	n/a	Ht. K564	2004	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Stuffing Box Cover	Bingham Willamette	11N35-3	n/a	Ht. 83852	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replace stuffing box cover on Recirc Pump 12 (P-200B) as part of seal ass'y
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2005-23-0002

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Pressure testing satisfied per WO 0506359 PMT (PP 141066), successful completion of 4290-02-OCD. No PSI or IST required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *D. DeSore*, SECT XI ISI COORD
Owner of Owner's Designee, Title

Date July 12, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+I of CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/28/05 to 7/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Aulenti
Inspector's Signature

Commissions NB 11863 A, P, I, MN 054028-00
National Board, Province and Endorsements

Date JULY 13, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2005-23-0004

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Planned 4-year like-for-like replacement of valve TCV-4030B. System Leakage Test with VT-2 per work order PMT. No PSI or IST required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *Frank Dopera*, SECT. XI ISI COORD. Date July 12, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER #1 OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/1/05 to 7/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Salschi Commissions NB 11863 A, N, S, MN 004028-00
 Inspector's Signature National Board, Province and Endorsements

Date JULY 13, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2005-23-0005

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 1/18/2006
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 0507758
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
4. Identification of System RHR Service Water (RHRSW), Pump #12 (P-109B), Class 3
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Bowl Ass'y	Johnston Pump Co	NRW-6523	n/a	n/a	2003	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Bowl Ass'y	Johnston Pump Co	PG-2265	n/a	n/a	2005	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced pump bowl assembly with rebuilt spare
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 236 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2005-23-0005

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Design Change 75M041 is applicable to existing pumps.

System leakage test with VT-2 was performed per the work order PMT.

IST was performed per the WO PMT Attachment.

This satisfies post-repair/replacement testing for the spare which was rebuilt under WO 0311347 and PO P502929 (ref. R/R Plan 2005-23-0001).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPLACEMENT

repair/replacement

Signed

[Signature], SECTION XI ISI GORD

Date

JANUARY 18, 2006

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER & CO. OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period SEPT 21, 2005 to JAN 18, 2006, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]

Inspector's Signature

Commissions

NB 11863 AN, I MN 034028-CO

National Board, Province and Endorsements

Date

JANUARY 18, 2006

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2005-23-0006

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/12/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 136399
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Emergency Service Water (ESW), Class 3
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SR-26	n/a	n/a	n/a	n/a	1969	Repaired	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SR-26 Stanchion	n/a	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SR-26 Stanchion	Dubose National	n/a	n/a	Ht #203524	2005	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Removed welded stanchion, realigned, and repaired by welding.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
		Freq: 0 yrs	Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2005-23-0006

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Repair of SR-26 included removal of existing stanchion pipe and replacement with new stanchion pipe, minor weld repairs to baseplate, and final welding after realignment of components. No pre-service VT-3 required for the applicable pipe size.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 12, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 12/1/05 to 7/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. DeLoche Commissions NP 11863 A, M, I PW 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 13, 2007

APPROVED (Signatures available in Master File)

Monticello Nuclear Generating Plant
 Cycle 23 / 2007 Refueling Outage
 Snubber Changeout Repair/Replacement (R/R) Sequence Log
 R/R Item Number 2006-23-0008
 NIS-2 Attachment

R/R Item Number Sequence	Snubber Location	System	Snubber Size (KIPS)	ASME Class	Snubber Replacement WO	S/N of Removed Snubber	Installation Year of Removed Snubber	S/N of Replacement Snubber	Replacement Snubber Rebuild WO	Applicable Drawing(s)	ISI Iso Dwg, ISI Item No.
A	SS-14	CFW	10	1	141353	500	3/29/1998	533	280592	NX-14086-32, 34	ISI-13142-53-A H-5
B	SS-184	PCT	10	2	141354	040	3/31/1998	042	280592	NL-94712	ISI-94966-A H-4
C	SS-316	CSP-B	10	2	141355	035	3/31/1998	534	280592	NL-95946	ISI-13142-26-B H-1
D	SS-13	CFW	10	1	141356	171	3/29/1998	044	280592	NX-14086-17, 31, 32	ISI-13142-52A H-4
E	SS-38A	RCI-STM	10	2	141357	013	3/25/1998	536	280592	NX-14086-75	ISI-13142-19B H-1
F	SS-17A	RHR-B	10	1	141358	530	3/31/1998	157	280592	NL-97036-5-1	ISI-97004-A H-4
G	SS-38B	RCI-STM	10	2	141359	033	4/1/1998	535	280592	NX-14086-77	ISI-13142-19B H-1
H	SS-121	PCT	10	2	141360	037	3/27/1998	007	02-02066	NX-14084-122	ISI-94966-A H-2
I	SS-11	CFW	10	1	141361	047	3/27/1998	039	280592	NX-14086-27, 28, 29	ISI-13142-52A H-7
J	SS-12	CFW	10	1	141362	095	3/29/1998	043	280592	NX-14086-28, 30	ISI-13142-53A H-7
K	SS-37	HPC-STM	10	2	141363	028	3/25/1998	003	03-11522	NX-14086-74	ISI-13142-19A H-1
L	SS-17B	RHR-B	10	1	141365	054-1	3/31/1998	012	280592	NL-97036-6-1	ISI-97004-A H-3
M	CRD-H227	CRD-A	20	2	141366	099	1/24/2000	128	03-11522	NQ-93567-37	ISI-93268-IA H-14

Monticello Nuclear Generating Plant
 Cycle 23 / 2007 Refueling Outage
 Snubber Changeout Repair/Replacement (R/R) Sequence Log
 R/R Item Number 2006-23-0008
 NIS-2 Attachment

R/R Item Number Sequence	Snubber Location	System	Snubber Size (KIPS)	ASME Class	Snubber Replacement WO	S/N of Removed Snubber	Installation Year of Removed Snubber	S/N of Replacement Snubber	Replacement Snubber Rebuild WO	Applicable Drawing(s)	ISI Iso Dwg, ISI Item No.
N	SS-303	RHR-A	10	2	141369	001	1/23/2000	532	280592	NL-95936	ISI-13142-37E H-2
O	CRD-H232	CRD-A	20	2	141371	214	1/24/2000	019A	02-02066	NQ-93567-42	ISI-93268-IA H-12
P	SS-8AR	REC-A	20	1	141372	307	3/30/1998	132	280592	NL-97033-20-1	ISI-97005-A H-8
Q	SS-8BR	REC-B	20	1	141374	152	3/30/1998	126	280592	NL-97035-20-1	ISI-97006-A H-8
R	SS-2AR	REC-A	30	1	141375	005	4/2/1998	154	280592	NL-97033-13-1	ISI-97005-C H-8
S	SS-1BR	REC-B	30	1	141377	470	4/4/1998	146	280592	NL-97035-12-1	ISI-97006-C H-7
T	SS-3AR	REC-A	30	1	141378	145	4/2/1998	050	280592	NL-97033-14-1	ISI-97005-C H-9
U	SS-1AR	REC-A	30	1	141379	468	4/4/1998	102	280592	NL-97033-12-1	ISI-97005-C H-7
V	SS-20	RHR-A	10	1	141382	539	4/13/1998	529	280592	NL-97034-12-1	ISI-97003-B H-2
W	SS-35	HPC-WTR	10	2	141383	540	4/1/1998	202	03-11522	NX-14086-72,-1	ISI-13142-40A H-5

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0010

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Like-for-like replacement of the stuffing box in association with pump PM, including replacement of the rotating assembly. System Leakage Test with VT-2 during 0255-20-IIC-2. Pressure and temperature listed are the minimum requirements for the Reactor Coolant Boundary System Leakage Test. No PSI or IST required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 13, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER FIT OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/5/06 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N, I MA 004028-C0
Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0011

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/13/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 153676 and EC-806
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Reactor Recirculation (REC), Loop B, Pump P-200B, Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
(16) Stuffing Box Nuts	Bingham Willamette	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(16) Stuffing Box Nuts	Nova Machine	n/a	n/a	Ht#7420672 Q1405	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced A194 Gr 2H with A194 Gr 7 in support of EC 806 (installed 2007)
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0011

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

PSI VT-1 performed on replacement nuts. Reference ISI Report 2007VT045. No pressure testing or IST required.

CERTIFICATE OF COMPLIANCE

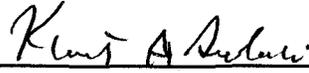
We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECT XI ISI COORD Date July 13, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INT OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/5/06 to 7/17/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 A, N, I MW 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date July 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 23 <small>RAD 7/14/07</small>

ITEM 2006-23-0013

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/16/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 3
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 EC-748 and WO 151841
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Co Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Main Steam (MST), Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-2373	BW/IP	E163A-1-1	n/a	n/a	1998	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2373	Flowserve	BC-390	n/a	n/a	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
PS15-3"-ED pipe	unknown	n/a	n/a	n/a	1998	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
PS15-3"-ED pipe	Dubose	n/a	n/a	Ht # L40039	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SEE	PAGE	2	FOR	ADD'L	ITEMS		<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced MO-2373 and associated piping to improve Appx J performance.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 1000 psi Test Temp 181.8 °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____

FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

ITEM 2006-23-0013

WO No. 151841

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
<i>PS15-3"-ED Elbow</i>	<i>Unknown</i>	<i>n/a</i>	<i>n/a</i>	<i>n/a</i>	<i>1998</i>	<i>replaced</i>		X
<i>PS15-3"-ED Elbow</i>	<i>Chicago Tube & Iron</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht.# 5850A</i>	<i>2007</i>	<i>replacement</i>		X

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 23 of 3

RSD
7/16/07

ITEM 2006-23-0013

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

The existing double-disk gate valve, including the associated piping and elbow, was replaced with a globe valve to improve Appendix J performance per EC-748. PT and RT performed as applicable. System leakage test with VT-2 during performance of 0255-20-IIC-2. IST performed during work order pre-operational testing task (stroke time closed, position indication) and Appendix J Testing per 0137-07A.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECTION XI ISI COORD Date July 16, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INS OF CONNECTICUT of HARTFORD have inspected the components described in this Owner's Report during the period 9/16/06 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N.J. NW 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0014

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/16/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 EC 748, WO's 151841 and 285967
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Co Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Main Steam (MST), Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-2374	BW/IP	E163A-2-1	n/a	n/a	1998	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2374	Flowserve	BC-391	n/a	n/a	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
PS15-3"-ED pipe	US Steel	n/a	n/a	n/a	2003	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
PS15-3"-ED pipe	Dubose	n/a	n/a	Ht # L40039	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced MO-2374 and associated piping to improve Appx J performance.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 1000 psi Test Temp 181.8 °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0014

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

The existing double-disk gate valve, including the associated piping, was replaced with a globe valve to improve Appendix J performance per EC-748. PT and RT performed as applicable. System leakage test with VT-2 during performance of 0255-20-IIC-2. IST performed during work order pre-operational testing task (stroke time closed, position indication) and Appendix J Testing per 0137-07A.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed T. D. [Signature] SECTION XI ISI COORD Date July 16, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MARYLAND and employed by HARTFORD STEAM BOILER I & P OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/16/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. [Signature] Commissions WB 11863 A, M, F MW 004028-00
Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0015

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/16/2007
Name

2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 265697
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address

4. Identification of System EDG Emergency Service Water (EDG-ESW), Valve ESW-1-2, Class 3

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
ESW-1-2	OIC Corp	Valve s/n 10568	n/a	n/a	1969	Repaired	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Removed / restored disc stem-to-nut tack weld to permit full inspection.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____

FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0015

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

DC 94A051 applies.
 No PSI or pressure testing required for valve internal components.
 IST close exercise per procedure 0187-02, open exercise per 0187-02B.
 (reference CAP's 01099755 and 01102080)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
 repair/replacement

Signed *J. DeA. Dagan*, SECT XI ISI COORD Date July 17, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/1/06 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Bulus Commissions NB 11863 A, N, I 144004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0016

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/18/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 289152, EC-775
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Residual Heat Removal (RHR), B Loop, Class 2
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
TW-12	n/a	n/a	n/a	TWH-12	1987	Repaired	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Removed integral attachment and modified for Engineering Change (EC-775)
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0016

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

This repair/replacement activity also included use of AWS D1.1, 1989 Edition.

EC-775 replaces several Rotork valve actuators with Limitorque actuators. Pipe support TW-12 required removal of the integrally welded stanchion and modification to meet piping analysis models.

NDE was performed at the formerly welded pipe attachment area. VT-3 PSI was also performed. No pressure testing was required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *[Signature]* SECT XI ISI COORD
Owner or Owner's Designee, Title

Date July 18, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ITS OF CONNECTICUT of HARTFORD have inspected the components described in this Owner's Report during the period 9/1/06 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions UB 11867 A, W, I MK 004028-CO
National Board, Province and Endorsements

Date July 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0017

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/18/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 289151, EC-775
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Core Spray (CSP), Loop B, Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
TWH-86 Spring Can	Bergen Patterson	n/a	n/a	Type IF Size 10	1987	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
TWH-86 Spring Can	Bergen Patterson	n/a	n/a	Type F Size 11	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced the spring can on support TWH-86 as required by EC-775
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0017

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*This repair/replacement activity also included use of AWS D1.1, 1989 Edition.
 EC-775 replaced several Rotork valve actuators with Limitorque actuators.
 Pipe support TWH-86 required a different spring can size to meet the revised piping analysis models. This included welding to structural components.
 VT-3 PSI performed. No pressure testing required.*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT Activity conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECT XI ISI COORD Date July 19, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM POWER INT OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/11/06 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions UB 11863 A, K, J MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2-3 <small>RAP 1/21/07</small>

ITEM 2006-23-0018

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/21/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 3
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 285768 (Div I), 286583 (Div II), EC-901
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Control Rod Drive Hydraulics (CRDH), Scram Discharge Volume, Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
LS-7428A (Div I)	Fluid Components	870-1	n/a	n/a	1981	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
LS-7428A (Div I)	Fluid Components	7802-S	n/a	n/a	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
LS-7428B (Div I)	Fluid Components	871-1	n/a	n/a	1981	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
LS-7428B (Div I)	Fluid Components	7803-S	n/a	n/a	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SEE	PAGE	2	FOR	ADD'L	ITEM		<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced obsolete CRD SDV level switches with new, equivalent switches
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 1000 psi Test Temp 181.8 °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

ITEM 2006-23-0018

WO Nos. 285768
286583
EC-901

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
LS-7429A (Div I)	Fluid Components	3337-1	n/a	n/a	1981	replaced		X
LS-7429A (Div I)	Fluid Components	7806-S	n/a	n/a	2007	replacement		X
LS-7430A (Div I)	Fluid Components	708	n/a	n/a	1981	replaced		X
LS-7430A (Div I)	Fluid Components	7808-S	n/a	n/a	2007	replacement		X
LS-7428C (Div II)	Fluid Components	872-1	n/a	n/a	1981	replaced		X
LS-7428C (Div II)	Fluid Components	7804-S	n/a	n/a	2007	replacement		X
LS-7428D (Div II)	Fluid Components	873-1	n/a	n/a	1981	replaced		X
LS-7428D (Div II)	Fluid Components	7805-S	n/a	n/a	2007	replacement		X
LS-7429B (Div II)	Fluid Components	707	n/a	n/a	1981	replaced		X
LS-7429B (Div II)	Fluid Components	7810-S	n/a	n/a	2007	replacement		X
LS-7430B (Div II)	Fluid Components	709	n/a	n/a	1981	replaced		X
LS-7430B (Div II)	Fluid Components	7809-S	n/a	n/a	2007	replacement		X

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 23 of 3

R&D
7/21/07

ITEM 2006-23-0018

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Replaced obsolete level switches (FCI Model FR-72) with new, equivalent level switches (FCI Model FLT93) per Engineering Change (EC) 901.
 New mechanical joint (bolted) pressure tested per IWA-4540(c), 1998 Edition per ISI Relief Request #7. System leakage test with VT-2 performed during 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR / REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECTION XI ISI Coord Date July 21, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I&I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/7/06 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions WB 11863 A, N, I MW 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0019

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

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- Owner Xcel Energy Company Date 7/19/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 140381
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System RHR Torus Cooling / Test Return, Loop A, Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-2008 Bonnet	Flowserve	Valve s/n AY260	n/a	Ht. # M3156	2005	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2008 Bonnet	Flowserve	Valve s/n AY260	n/a	Ht. # 227H096	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced MO-2008 bonnet like for like (replacement built 2006, installed 2007)
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 110 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

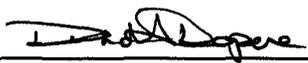
ITEM 2006-23-0019

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Design Change 04Q015 applies.
 MO-2008 bonnet replaced as a precautionary measure due to possible stresses induced by overthrusting during valve testing.
 No NDE or PSI required.
 System leakage test with VT-2 performed per work order PMT.
 IST stroke time testing (STO and STC) and position indication testing (PIT) performed per work order PMT.*

CERTIFICATE OF COMPLIANCE

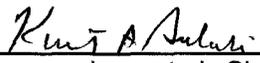
We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECTION XI ISI COORD Date July 19, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/8/06 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 A, N.E., MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 23 ²³ _{7/23/07}

ITEM 2006-23-0020

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/23/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 3
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 EC-768, WOs 267425, 284077, 285273
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Co Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Emergency Service Water (EDG-ESW), Loops A and B, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SW-16 (Loop A)	OIC Corp	n/a	n/a	n/a	1969	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SW-228 (Loop A)	Flowserve	BC-455	n/a	n/a	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SW21-4"-HF (Loop A)	Tioga	n/a	n/a	52TA4665, S/N 1	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
SEE	PAGE	2	FOR	LOOP B	ITEMS		<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced check valves with manual gate valves & associated pipe for EC-768
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure Remark psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

ITEM 2006-23-0020

WO Nos. 267425
284077
285273
EC-768

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
SW-18 (Loop B)	OIC Corp	n/a	n/a	n/a	1969	replaced		X
SW-229 (Loop B)	Flowserve	BC-456	n/a	n/a	2007	replacement		X
SW21-4"-HF (Loop B)	Tioga	n/a	n/a	52TA4665, S/N 1	2007	replacement		X

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2 of 3 7/23/07

ITEM 2006-23-0020

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Engineering Change (EC) 768 replaced Class 3 check valves with manual gate valves and a new section of upstream piping. The EC also replaced non-class items, not applicable to the Section XI boundary.

NDE performed as applicable.

System leakage test with VT-2 performed per work order PMT attachments. Loop A pressure: 47 psig, Loop B pressure: 48 psig.

No IST or PSI required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature] SECTION XI ISI COORD Date July 23, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 8/30/06 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NO 11863 A, N, I MN 004028-00
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0021

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*This repair/replacement activity also included use of AWS D1.1, 1989 Edition.
 EC-775 replaced several Rotork valve actuators with Limitorque actuators.
 Pipe support SR-852 is a new support installed to meet the revised piping analysis models.
 VT-3 PSI performed. No pressure testing required.*

CERTIFICATE OF COMPLIANCE	
We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.	<u>REPAIR / REPLACEMENT ACTIVITY</u> repair/replacement
Signed <u>[Signature]</u> , SECTION XI ISI COORD Owner or Owner's Designee, Title	Date <u>July 19</u> , <u>2007</u>

CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of <u>MINNESOTA</u> and employed by <u>HARTFORD STEAM BOILER DEP OF CONNECTICUT</u> of <u>HARTFORD, CT</u> have inspected the components described in this Owner's Report during the period <u>7/1/07</u> to <u>7/22/07</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer SHALL be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.</p>	
<u>[Signature]</u> Inspector's Signature	Commissions <u>NB 11863 A, N, I MN 004025-CU</u> National Board, Province and Endorsements
Date <u>JULY 22</u> , <u>2007</u>	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0022

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/19/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 152929
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System EDG Emerg. Service Water (EDG-ESW), Pump #12 (P-111B), Class 3
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
<i>P-111B</i>	<i>Johnston</i>	<i>QD-6702</i>	<i>n/a</i>	<i>n/a</i>	<i>2004</i>	<i>Replaced</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
<i>P-111B</i>	<i>Johnston</i>	<i>QD-6703</i>	<i>n/a</i>	<i>n/a</i>	<i>2006</i>	<i>Replacement</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced EDG-ESW Pump #12 (P-111B) with rebuilt spare.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 47 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

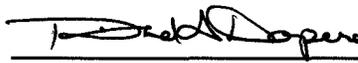
ITEM 2006-23-0022

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Pumps meet requirements of Design Change 92Q300.
 System leakage test with VT-2 of discharge flange joint per work order PMT attachment.
 IST pump performance testing per work order PMT.
 This satisfies post-repair/replacement testing for the spare which was rebuilt under WO 0403298.*

CERTIFICATE OF COMPLIANCE

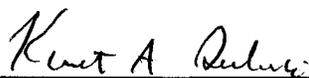
We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECTION XI ISI COORD. Date July 19, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MASSACHUSETTS and employed by HARTFORD STEAM BOILER & PIPE CONNECTION of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/1/06 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 AND MA 004028-EO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0023

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/18/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 139591
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
- Identification of System EFT Emergency Service Water (FSW), Div. I, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
ESW-32	Anchor Darling	E1241-1-2	n/a	n/a	1983	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW-32	Flowserve	AZ-899	n/a	n/a	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW1-3"-HBD	unknown	n/a	n/a	n/a	1983	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW1-3"-HBD	Dubose	n/a	n/a	Ht. Code L22569	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced due to corrosion on valve internals (built 2006 / installed 2007)
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 74.3 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0023

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*NDE performed as applicable.
System leakage test with VT-2 performed per work order PMT attachment
No PSI or IST required.*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 18, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER & PIPE CONTRACTORS of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/6/06 to 2/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N.I. MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0024

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/18/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 142128
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
- Identification of System EFT Emergency Service Water (FSW), Div. II, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
ESW-31	Anchor Darling	E1241-1-1	n/a	n/a	1983	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW-31	Flowserve	AZ-532	n/a	n/a	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW5-3"-HBD	unknown	n/a	n/a	n/a	1983	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW5-3"-HBD	Dubose	n/a	n/a	Ht. Code L22569	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced due to corrosion on valve internals (built 2006 / installed 2007)
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 75.8 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:
			Freq: 0 yrs
			Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0024

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

NDE performed as applicable.

System leakage test with VT-2 performed per work order PMT attachment

No PSI or IST required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 18, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ID OF CONVERT CUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/6/06 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions MB 11863 A, N, I MW 004028-CD
Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0025

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/7/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address
WO-141349 Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System High Pressure Coolant Injection (HPCI), Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SS-707	Lisega	614900/052	-	MNGP s/n 700	2001	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SS-707	Lisega	615060/055	-	MNGP s/n 701	2001	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(2) Snubber End Pins	Bergen-Paterson	-	-	-	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(2) Snubber End Pins	Bergen-Paterson	-	-	Ht# 306610	1991	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replace existing snubber and (2) end pins like-for-like with new.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0025

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functional Testing per procedure 0251. Pre-service VT-3 Examination as required by procedure 4903.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD. Date July 7, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/8/06 to 7/9/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Hulsh Commissions WB 11863 A, N, D, MN 004028-00
 Inspector's Signature National Board, Province and Endorsements

Date JULY 9, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0026

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/10/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Spare - 10 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	529	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	529	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced main cylinder during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0026

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT Activity
repair/replacement

Signed *[Signature]*, SECT. XI ISI COORD. Date July 10, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO. OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/21/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB11867 A, N.I. MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

ITEM 2006-23-0027

WO No. 280592

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
<i>Piston Rings</i>	<i>Bergen Paterson</i>	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>Un-known</i>	<i>replaced</i>		X
<i>Piston Rings</i>	Bergen Paterson	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>2006</i>	<i>replacement</i>		X
<i>Piston Rod</i>	<i>Bergen Paterson</i>	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>Un-known</i>	<i>replaced</i>		X
<i>Piston Rod</i>	Bergen Paterson	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>2006</i>	<i>replacement</i>		X
<i>Piston Nut</i>	<i>Bergen Paterson</i>	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>Un-known</i>	<i>replaced</i>		X
<i>Piston Nut</i>	Bergen Paterson	<i>044</i>	<i>n/a</i>	<i>n/a</i>	<i>2006</i>	<i>replacement</i>		X

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 23 of 3

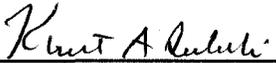
RAD
7/10/07

ITEM 2006-23-0027

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functional test per 0251, VT-3 to be completed when installed.

<p>CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.</p>		<p><u>REPAIR/REPLACEMENT ACTIVITY</u> repair/replacement</p>
<p>Signed <u></u>, <u>SECT XI ISI COORD</u> Owner or Owner's Designee, Title</p>	<p>Date <u>July 10, 2007</u></p>	

<p>CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of <u>MINNESOTA</u> and employed by <u>HARTFORD STEAM BOILER IPT OF CONNECTICUT</u> of <u>HARTFORD, CT</u> have inspected the components described in this Owner's Report during the period <u>9/21/06</u> to <u>7/12/07</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer SHALL be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.</p>	
<p><u></u> Inspector's Signature</p>	<p>Commissions <u>MB 11863 A, N, I, M, N 004028-CO</u> National Board, Province and Endorsements</p>
<p>Date <u>JULY 12, 2007</u></p>	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0028

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/10/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Spare - 20 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	460	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	460	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced valve manifold during rebuild PM.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0028

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECT. XI ISI COORD
Owner or Owner's Designee, Title

Date July 11, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I&I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/29/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions LB 11863 A, N, I MN 004028-CO
National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0029

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *Dada Lopez*, SECT. XI ISI COORD. Date July 11, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & F OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/29/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Snelson Commissions WB 11863 A, N, I, 14N004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 23 ^{ADD} 7/11/07

ITEM 2006-23-0030

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/11/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 3
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Spare - 10 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	157	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	157	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Piston	Bergen Paterson	157	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Piston	Bergen Paterson	157	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
See	Page	2	for	add'l	items		<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced Main Cylinder, Piston, Piston Rings during rebuild.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

ITEM 2006-23-0030

WO No. 280592

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
<i>Piston Rings</i>	<i>Bergen Paterson</i>	<i>157</i>	<i>n/a</i>	<i>n/a</i>	<i>Un-known</i>	<i>replaced</i>		X
<i>Piston Rings</i>	<i>Bergen Paterson</i>	<i>157</i>	<i>n/a</i>	<i>n/a</i>	<i>2006</i>	<i>replacement</i>		X

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 23 of 3

047
7/4/07

ITEM 2006-23-0030

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functional test per 0251, VT-3 to be completed when installed.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *Richard Deperera*, SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER DEP OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/21/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Pulisti Commissions NB 11863 A, N, I MN 004028-CO
Inspector's Signature National Board, Province and Endorsements
Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0031

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/8/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 135620, GE 21A5829 Rev1, DC 96Q065
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Standby Liquid Control (SBL), Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
RV-11-39B Disc	Consolidated Dresser, Inc.	Valve s/n TM 41591	n/a	AOD71	2001	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
RV-11-39B Disc	Consolidated Dresser, Inc.	Valve s/n TM 41591	n/a	s/n ADV03 Ht Cd 67269	2005	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
RV-11-39B Nozzle	Consolidated Dresser, Inc.	Valve s/n TM 41591	n/a	ADD47	2001	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
RV-11-39B Nozzle	Consolidated Dresser, Inc.	Valve s/n TM 41591	n/a	s/n ADV72 Ht Cd 72470	2005	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced nozzle and disc during normal PM
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0032

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Spare - 20 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	131	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	131	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced valve manifold during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0032

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR / REPLACEMENT ACTIVITY
repair/replacement

Signed *D. D. Dugane*, SECT XI ISI COORD Date July 12, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of CONNECTICUT and employed by HARTFORD STEAM BOILER I & J OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/4/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Aulevich Commissions NB 11863 A, N, I, I, NW 004128-00
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0033

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/11/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Spare - 20 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	487	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	487	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced valve manifold during rebuild PM.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____					
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs	
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0033

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

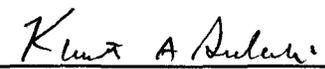
We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR /REPLACEMENT ACTIVITY
repair/replacement

Signed , SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+J OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/4/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 A, N, I, MN 004028-C0
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0034

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Spare - 20 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	466	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	466	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced valve manifold during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0034

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *Patrick Dague*, SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ISJ OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/4/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Salschi Commissions NB 11863 A, N, I, MN 004028-00
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0035

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Spare - 20 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	132	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	132	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced valve manifold during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0035

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed

[Signature] SECT. XI ISI COORD.
Owner or Owner's Designee, Title

Date

July 11, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & F OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/29/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions

NB 11863 A, N, I, MN 304028-CD
National Board, Province and Endorsements

Date

JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0036

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Spare - 20 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
<i>Valve Manifold</i>	<i>Bergen Paterson</i>	148	n/a	n/a	<i>un-known</i>	<i>Replaced</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
<i>Valve Manifold</i>	<i>Bergen Paterson</i>	148	n/a	n/a	2006	<i>Replacement</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced valve manifold during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0036

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed [Signature], SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER IIT OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/26/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N, I, MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0037

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/11/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Spare - 20 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	132	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	132	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced main cylinder during rebuild PM.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0037

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *D. A. Drape*, SECT XI ISI COORD Date July 11, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 9/29/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kevin A. Auluck Commissions NB 11863 A, I, II, MA 004028-CU
 Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0038

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR / REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *D. A. Dapone*, SECT XI ISI COORD Date July 11, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER IJS OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 10/2/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Dulanti Commissions NB 11863 A, N, I, MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0039

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/11/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Spare - 30 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	154	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	154	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced main cylinder during rebuild PM.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____								
FOR ADMINISTRATIVE USE ONLY	Resp Supv:	PRGE	Assoc Ref:	PRGE	SR:	N	Freq:	0 yrs
	ARMS:	3613	Doc Type:	REF ONLY	Admin initials:		Date:	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0039

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *David Dugan*, SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER #1 OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 10/6/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Subedi Commissions NB 11863 A,N,I, MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0040

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Spare - 130 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	174	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	174	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced main cylinder during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____					
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs	
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0040

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature] SECT XI ISI COORD Date July 11, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INT OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/4/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions MB 11863 A, N, F, MW 00 4028-00
 Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0041

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/11/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 280592
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Spare - 20 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	133	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	133	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced valve manifold during rebuild PM.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____					
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs	
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0041

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *[Signature]*, SECT XI ISI COORD
Owner or Owner's Designee, Title

Date July 11, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEEL BOILER I+J OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/20/06 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions NB 11863 A, N, I, MW 004028-00
National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-23-0042

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/11/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 280592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Spare - 20 Kip Snubber
5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Valve Manifold	Bergen Paterson	125	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Valve Manifold	Bergen Paterson	125	n/a	n/a	2006	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced valve manifold during rebuild PM.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0042

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functionally tested per 0251. VT-3 pre-service exam to be completed following installation under separate work order.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR / REPLACEMENT ACTIVITY
repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 11, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & J OF CONNECTICUT HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/5/07 to 7/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N, I, MA 004028-60
Inspector's Signature National Board, Province and Endorsements

Date JULY 12, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0043

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Pump meets requirements of Design Change 92Q300.

During rebuild, excavated small crack in 1st intermediate bowl's impeller shroud, performed NDE to verify removal, and repaired by welding.

No PSI or Pressure Testing required per this activity.

Pre-service testing will be performed under separate work order when removed from spare stock and installed in plant.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECTION XI ISI COORD Date July 19, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER [I+] OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 1/4/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 1863, A, NJ PLW 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 23 ²³ 7/21/07

ITEM 2007-23-0044

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/20/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 3
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 154343, EC-7462
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Reactor Vessel (RPV), Wide Range Flux Monitor Housing, Class 1
- (a) Applicable Construction Code ASME Section III, 1965 Edition
Year
Addenda Summer 1966 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
WRFM 12-45	GE Reuter Stokes	Marking 6549798	n/a	Ht. # 132567	1984	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Blind Flange spare 12-45	GE Reuter Stokes	s/n 11-76748-1	n/a	Ht. # 2F353	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
WRFM 20-05	GE Reuter Stokes	Marking 6549796	n/a	Ht. # 132567	1984	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Blind Flange spare 20-05	GE Reuter Stokes	s/n 12-76748-1	n/a	Ht. # 2F353	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SEE	PAGE	2	FOR	ADD'L	ITEMS		<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Removed (2) abandoned WRFM's, replaced with blind flanges including bolting
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 1000 psi Test Temp 181.8 °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

ITEM 2007-23-0044

WO No. 154343

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							Yes	No
<i>WFRM 12-45 (4) capscrews</i>	<i>GE Reuter Stokes</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht. # 85483</i>	<i>1984</i>	<i>replaced</i>		<i>X</i>
<i>Blind Flange spare 12-45 (4) capscrews</i>	<i>GE Reuter Stokes</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht. # 96903</i>	<i>2007</i>	<i>replacement</i>		<i>X</i>
<i>WFRM 20-05 (4) capscrews</i>	<i>GE Reuter Stokes</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht. # 85483</i>	<i>1984</i>	<i>replaced</i>		<i>X</i>
<i>Blind Flange spare 20-05 (4) capscrews</i>	<i>GE Reuter Stokes</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht. # 96903</i>	<i>2007</i>	<i>replacement</i>		<i>X</i>

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 23 of 3

7/21/07

ITEM 2007-23-0044

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Replaced abandoned wide range flux monitors with blind flanges to restore the original configuration per Engineering Change (EC) 7462.
 No PSI required.
 System leakage test with VT-2 performed during 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature] SECTION XI ISI COORD. Date July 21, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER F&E OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/23/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB (186) A, N, I MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0045

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/23/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 314220, EC-9174
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address

4. Identification of System Main Steam (MST)); Loops A, B, C, D; Class 1

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Strain Gage PS1-18"-ED	HITEC Product	n/a	n/a	See Remarks	2007	Repair	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Strain Gage PS2-18"-ED	HITEC Product	n/a	n/a	See Remarks	2007	Repair	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Strain Gage PS3-18"-ED	HITEC Product	n/a	n/a	See Remarks	2007	Repair	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Strain Gage PS4-18"-ED	HITEC Product	n/a	n/a	See Remarks	2007	Repair	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Spot welded 64 strain gages to Main Steam Piping for EC-9174

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:
			Freq: 0 yrs
			Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0045

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*In support of Engineering Change (EC) 9174, 64 (total) Hastaloy X strain gages and covers were spot welded by capacitive discharge methods to the main steam piping per ASME Section III 2001 Edition NB-4311.2 provisions. 8 strain gages were installed around the circumference at 2 location on each the 4 loops of main steam piping. The strain gages are part of a new data acquisition system that will provide dynamic pressure measurement for current conditions to compare to increased flow associated with future extended power uprate. With the exception of having the strain gages attached to it, the Main Steam system is not affected. The HITEC strain gage part number is HBWAK-35-250-6-10FG-SHIELD.
No pressure testing or PSI was required.*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECTION XI ISI COORD Date July 23, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER & TANK CO. OF CONNECTICUT HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/18/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 NBI MW004028-C0
Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0046

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/20/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 288806
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Core Spray (CSP), Min Flow Loop B, Class 2
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter '78 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
<i>TW9-2 1/2"-HE</i>	<i>unknown</i>	<i>n/a</i>	<i>n/a</i>	<i>n/a</i>	<i>1969</i>	<i>Repaired</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Removed / Reinstalled piping due to interference with adjacent work.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 300 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0046

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Piping removed and reinstalled due to ingress / egress interference with adjacent Torus interior work. Also disassembled and reassembled associated supports TWH-135 and TWH-136. RT performed following reinstallation of piping. Owner-elected VT-3 performed on supports. (reference ISI Report BOP-VT-07-053 and -054) System leakage test with VT-2 performed per work order PMT attachment.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR / REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]* SECTION XI ISI COORD Date July 20, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/19/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions MB 11863 A, N, I MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0047

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/20/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 267368
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Reactor Head Vent, Flange #3 (2" Cavity Flange), Class 1
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
(1) 5/8" Stud	Nova	n/a	n/a	Ht. # 7404456	2005	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(1) 5/8" Stud	Nova	n/a	n/a	Ht. # 230111	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(2) 5/8" Nuts	Nova	n/a	n/a	Ht. # B87035	2005	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
(2) 5/8" Nuts	Nova	n/a	n/a	Ht. # 7220464	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Like for like replacement of 1 stud and 2 nuts due to disassembly damage.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0047

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

VT-1 PSI performed on replacement bolting. No pressure testing required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR / REPLACEMENT ACTIVITY
repair/replacement

Signed

[Signature], SECTION XI ISI COORD.
Owner or Owner's Designee, Title

Date

July 20, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/3/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions

WB 11863 A, M, I MN 004026-00
National Board, Province and Endorsements

Date

July 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0048

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/21/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 314233
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Reactor Core Isolation Cooling (RCIC), Steam Supply, Class 2
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter '78 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
PS17-3"-ED	unknown	n/a	n/a	n/a	2001	Repaired	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Removed / Reinstalled piping due to interference with adjacent work.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 1040 psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0048

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Piping removed and reinstalled due to interference with adjacent MSIV AO-2-86D actuator maintenance activities. Also disassembled and reassembled associated supports PSH-114, PSH-115, and SR-564.

RT performed following reinstallation of piping.

No PSI required.

System leakage test with VT-2 performed per work order PMT attachment / work plan.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *Richard Lopez*, SECTION XI ISI COORD.
Owner or Owner's Designee, Title

Date *July 21*, *2007*

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER DIV OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/21/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Dulmus
Inspector's Signature

Commissions *NB 11863 A, N.E. MW 004028-00*
National Board, Province and Endorsements

Date *JULY 22*, *2007*

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0049

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/20/2007
Name

2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 322082
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address

4. Identification of System Residual Heat Removal Service Water (RHRSW), Loop B, Class 3

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SW10-18"-GF	n/a	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SW10-18"-GF	Dubose	n/a	n/a	Ht# A83222	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced approx 4 ft of A-106 Gr. B pipe (line designation SW10-18"-GF located in the intake) like-for-like by welding due to localized ID corrosion (replacement pipe built 2000, installed 2007).

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 253 psi Test Temp n/a °F

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2 <i>RSP 7/20/07</i>

ITEM 2007-23-0049

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

NDE of welds performed as applicable.

Support SR-385 was also disassembled and re-assembled to accommodate access for the piping replacement. VT-3 PSI performed following re-installation.

System leakage test with VT-2 in lieu of a hydro per IWA-4540(a)(2) per WO PMT attachment.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT Activity
repair/replacement

Signed *T. Joseph Deane*, SECTION XI ISI COORD Date July 20, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/22/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Dulake Commissions NB 11863 A.M.J. ML 004028 -CO
Inspector's Signature National Board, Province and Endorsements

Date July 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0050

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/8/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 141889
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC/Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cnty. Rd. 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Spare - 130 Kip Snubber
- (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Cylinder	Bergen Paterson	400	n/a	n/a	un-known	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Cylinder	Bergen Paterson	400	n/a	n/a	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced main cylinder during rebuild.
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp _____ °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0050

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Functional test per 0251, VT-3 to be completed when installed.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPLACEMENT
repair/replacement

Signed *[Signature]* SECT XI ISI COORD. Date July 8, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & II OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/23/07 to 7/9/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11867 A, N, I, MW 00 4028-00
Inspector's Signature National Board, Province and Endorsements

Date JULY 9, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0051

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/18/2007
Name

2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 288859, EC-775
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address

4. Identification of System Core Spray (CSP), Loop B, Class 2

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-1742 studs (14)	Wm Powell Valve	Valve s/n 58767	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1742 studs (14)	Cardinal	n/a	n/a	D42339	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1742 nuts (28)	Wm Powell Valve	Valve s/n 58767	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1742 nuts (28)	Nova	n/a	n/a	B87035	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Like for like replacement of 14 bonnet studs and 28 nuts on valve MO-1742

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____

FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0051

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Like for like replacement of studs and nuts due to damage during removal in support of activities for Engineering Change (EC) 775.
 Studs built 1984, installed 2007. Nuts built 2002, installed 2007.
 No pressure testing or PSI required.
 IST stroke time testing and position indication test performed per Work Order preoperational testing instructions.*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]* SECT XI ISI COORD Date July 19, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER DIV OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/26/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11763 A, IV, I MN 004028-00
 Inspector's Signature National Board, Province and Endorsements

Date July 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0052

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/18/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 289152, EC-775
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address

4. Identification of System Residual Heat Removal (RHR), B Loop, Valve MO-1987, Class 2

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-1987 Studs (18)	unknown	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1987 Studs (12)	Nova	n/a	n/a	Ht. #240902	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1987 Studs (6)	Nova	n/a	n/a	Ht. # 53410	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1987 Nuts (36)	unknown	n/a	n/a	n/a	2007	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-1987 Nuts (36)	Nova	n/a	n/a	Ht. #8994826	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>

7. Description of Work Replaced 18 Studs and 36 Nuts that were damaged during disassembly.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:
		Freq: 0 yrs	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2006-23-0016

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*EC-775 replaces several Rotork valve actuators with Limitorque actuators.
 No PSI or pressure testing required for bolting replacement.
 Stroke Time and Position Indication testing for IST performed per work order PMT attachment.*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *Pho Lapere*, SECT XI ISI COORD Date July 18, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ITS OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/26/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Aular Commissions NB 11863 A.I.N.J. MW 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0053

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/18/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 289154
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address

4. Identification of System Residual Heat Removal (RHR), Div II, Class 2

5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-2009 Disc	Anchor Darling	n/a	n/a	n/a	1989	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2009 Disc	Anchor Darling	n/a	n/a	Ht# A34626	1989	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replacement of disc and weld connecting the steel tube portion of the skirt to the disc in valve MO-2009 (replacement built 1989, installed 2007).

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 3 of 2 ^{RAD} _{7/18/07}

ITEM 2007-23-0053

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Disc replaced to accommodate maintenance activities in support of EC-775 to replace the stem and actuator.
No PSI or pressure testing required for internal valve components.
IST position indication (PIT) and stroke time testing (STO and STC) completed per work order Testing Instruction.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR /REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 18, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I+J OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/26/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Dulski Commissions NB 11863 A, N.J. MA 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date July 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0054

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/18/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 289151, EC-775
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Core Spray (CSP), Loop B, Class 2
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-1750 disc	Anchor Darling	n/a	n/a	n/a	1969	Repair	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Welding on MO-1750 disc nut to disc for disassembly / reassembly.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0054

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

The disc nut to disc tack weld was required to be removed and rewelded to permit disassembly and reassembly of the valve to accommodate maintenance activities for Engineering Change (EC) 775. No pressure testing or PSI required. IST position indication testing was performed per work order preoperational testing instructions.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed

[Signature], SECT XI ISI COORD
Owner or Owner's Designee, Title

Date

July 19, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/27/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions NB 11863 A, N, I 12A 004028-CO
National Board, Province and Endorsements

Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0056

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/23/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 322070
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Automatic Pressure Relief (APR); Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Main Stage Body	Target Rock	1458	n/a		2005	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Main Stage Body	Target Rock	53	n/a		2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced Main Stage Body for RV-2-71C with rebuilt spare
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0056

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Replacement was pre-emptive and valve showed no signs of leakage at normal operating conditions.

Though no pressure test is required per Relief Request #7, a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

repair/replacement activities
repair/replacement

Signed *[Signature]*, SECTION XI ISI COORD Date July 23, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER IJI OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/3/07 to 7/27/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions MB 11863 A, N, I MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date July 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0057

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Replacement was pre-emptive and valve showed no signs of leakage at normal operating conditions.

Though no pressure test is required per Relief Request #7, a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. repair/replacement activities
repair/replacement

Signed D. D. Dopen, SECTION XI ISI COORD Date July 23, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INS OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/3/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Ambler Commissions WB 11863 A.W.I. RW 004028-CD
Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0058

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

GE SPEC 21A9206 and Alteration 97A011 Rev.1 apply. Like for like, partial replacement of bolting damaged during disassembly for procedure 4280-PM. (built 2003, installed 2007)
 PSI VT-1 performed on replacement bolting. (reference ISI Report 2007VT459)
 No pressure testing or IST required for bolting replacement.
 Although no pressure testing or IST is required for bolting replacement, the valve was tested per Operational Test 0112 and a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECTION XI ISI COORD Date July 24, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ITS OF CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/31/07 to 7/24/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NO 11863 A, M, I MW 004028-C0
 Inspector's Signature National Board, Province and Endorsements
 Date July 24, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0059

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/17/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 291592
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System RHR Service Water (RSW), Loop B, Class 3
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
CV-1729	Valtek	47907.001.1	n/a	n/a	1987	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
CV-1729	Valtek	n/a	n/a	Ht. #ZFE	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced CV-1729 valve body like-for-like. (built 1998, installed 2007)
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
		Freq: 0 yrs	Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0059

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Design Change 87Z015 applies.

New mechanical joint pressure tested per IWA-4540(c), 1998 Edition per ISI Relief Request #7.

System Leakage Test with VT-2, IST position indication test (PIT) and open exercise test (STO) performed per work order PMT.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed

Frank A. Depere, SECT XI ISI WORD
Owner or Owner's Designee, Title

Date

July 17, 2007

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by MARY FORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/30/07 to 7/19/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. Anselmi
Inspector's Signature

Commissions

NB 11863 A, N, E, MN 004028-00
National Board, Province and Endorsements

Date

JULY 19, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0060

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/20/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 267369
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
4. Identification of System Primary Containment, Drywell Head, Class 2/MC
5. (a) Applicable Construction Code ASME Section III, 1965 Edition
Year
Addenda Winter '65 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
DW Head Flange Bolts	Chicago Bridge & Iron	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
DW Head Flange Bolts	Chicago Bridge & Iron	n/a	n/a	Ht. #32508	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
DW Head Flange Nuts	Chicago Bridge & Iron	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
DW Head Flange Nuts	Chicago Bridge & Iron	n/a	n/a	Ht. # 6390344	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced 2 Bolts and 3 Nuts on the Drywell Head main flange.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0060

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

(2) Bolts and (3) nuts replaced due to normal wear, nicks, gouges, thread damage, etc. resulting from multiple disassembly / reassembly occurrences since original installation.
 VT-1 PSI performed on replacement bolting items. (reference ISI Report BOP-VT-07-051)
 No pressure testing required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT Activity conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECTION XI ISI COORD. Date July 20, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & P OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/3/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Schultz Commissions MB 11863 A, N, I MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements
 Date July 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0061

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/3/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO-141358
Address Repair Organization P.O. No., Job No. etc.

3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a

4. Identification of System Residual Heat Removal (RHR), Class 1

5. (a) Applicable Construction Code USAS B31.1, 1967 Edition
Year
Addenda No Addenda Code Case n/a

(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SS-17A	Bergen Patterson	157	-	4 bolts, Ht# unknown	unknown	replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SS-17A	Bergen Patterson	157	-	4 bolts, Ht# 8868492	2007	replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced 4 flange mounting bolts that were misplaced during maintenance.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:				
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0061

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Installation performed per procedure 4903.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPLACEMENT
repair/replacement

Signed *[Signature]*, ISI COORDINATOR Date JULY 3, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER I & I OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/30/07 to 7/10/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Schubert Commissions NB 11863 A, N, I, MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 10, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0062

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/17/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address
WO 291592 Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System RHR Service Water (RSW), Loop B, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
CV-1729 Plug	Valtek	V12499	n/a	n/a	1995	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
CV-1729 Plug	Flowserve	n/a	n/a	Ht. Code 86966-1	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replaced CV-1729 plug. (built 2006, installed 2007)
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0062

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Design Changes 87Z015 and 95A011 apply.

No pressure testing required for valve internal components.

IST position indication test (PIT) and open exercise test (STO) performed per work order PMT.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI.

REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *[Signature]*, SECT XI ISI COORD Date July 17, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INC OR CONNECTICUT OF HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/30/07 to 7/19/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N, I, MN 004028-CO
Inspector's Signature National Board, Province and Endorsements

Date JULY 19, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0063

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/23/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address
WO 291883 Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Automatic Pressure Relief (APR); Safety Relief Valve RV-2-71E, Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
Inlet flange studs (12)	Nova	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Inlet flange studs (12)	Nova	n/a	n/a	Ht # 3M41345	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Inlet flange nuts (24)	Nova	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Inlet flange nuts (24)	Nova	n/a	n/a	Ht # M92614	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced inlet flange bolting due to disassembly damage & thread pitch spec

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____

FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N	Freq: 0 yrs
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:	Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0063

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

GE Spec 21A9206 applies. Replaced all inlet flange bolting (built 2004, installed 2007): some damaged during disassembly for procedure 4280-PM, and all had a thread pitch different than specified by drawings specs.

PSI VT-1 performed on replacement bolting. (reference ISI Report 2007VT486).

No pressure testing or IST required for bolting replacement.

Although no pressure testing or IST is required for bolting replacement, the valve was tested per Operational Test 0112 and a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECTION XI ISI COORD Date July 23, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEEL BOILER I&E OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/13/07 to 7/24/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions WB11863 A, N, I MN 004028-C0
Inspector's Signature National Board, Province and Endorsements

Date JULY 24, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0064

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/23/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address
WO 291885 Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Automatic Pressure Relief (APR); Safety Relief Valve RV-2-71G, Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
topworks flg bolts (12)	Nova	n/a	n/a	H250	2003	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
topworks flg bolts (12)	Nova	n/a	n/a	Ht # 84013	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
topworks flg nuts (12)	Nova	n/a	n/a	H251	2003	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
topworks flg nuts (12)	Nova	n/a	n/a	Ht # 66036	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Replacement of topworks flange bolting due to disassembly damage
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____								
FOR ADMINISTRATIVE USE ONLY	Resp Supv:	PRGE	Assoc Ref:	PRGE	SR:	N	Freq:	0 yrs
	ARMS:	3613	Doc Type:	REF ONLY	Admin initials:		Date:	

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0064

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

GE SPEC 21A9206 and Alteration 97A011 Rev.1 apply. Like for like, replacement of bolting damaged during disassembly for procedure 4280-PM. (built 1998, installed 2007)
 PSI VT-1 performed on replacement bolting. (reference ISI Report 2007VT488)
 No pressure testing or IST required for bolting replacement.
 Although no pressure testing or IST is required for bolting replacement, the valve was tested per Operational Test 0112 and a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *[Signature]*, SECTION XI ISI COORD Date July 23, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/3/07 to 7/24/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, M, I MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 24, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0065

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

GE SPEC's 21A1060, 21A9206, and Alteration 97A011 Rev.1 apply. Like for like, replacement of bolting damaged during disassembly for procedure 4280-PM. (built 1998, installed 2007)
 PSI VT-1 performed on replacement bolting. (reference ISI Report 2007VT487)
 No pressure testing or IST required for bolting replacement.
 Although no pressure testing or IST is required for bolting replacement, the valve was tested per Operational Test 0112 and a VT-2 was performed as part of the RCPB System Leakage Test 0255-20-IIC-2.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed [Signature], SECTION XI ISI COORD Date July 23, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER ITC OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/3/07 to 7/24/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, NJ 144004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 24, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0066

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/19/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address
WO 142056 Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name
2807 W. Cty Rd 75, Monticello, MN 55362 Address Authorization No. n/a
Expiration Date n/a
- Identification of System High Pressure Coolant Injection (HPC), Class 1
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
MO-2035 stud. nuts	Nova/Flowserve	n/a	n/a	n/a	various	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2035 stud. nuts	Nova/Flowserve	n/a	n/a	n/a	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2035 Disc	Flowserve	S/N 2	n/a	Ht. Code TRH-1	2000	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
MO-2035 Disc	Flowserve	S/N 1	n/a	Ht. Code TRH-2	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Like for like replacement of 1 bonnet stud and 16 nuts on valve MO-2035
- Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials: _____
			Freq: 0 yrs
			Date: _____

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0066

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Like for like replacement of stud and nuts due to damage during removal.
 Like for like replacement of disc due to slight warpage visually observed during removal.
 Studs built 2004, installed 2007. Nuts built 2004, installed 2007. Disc built 1999, installed 2007.
 No pressure testing required. VT-1 PSI performed on replacement bolting.
 IST stroke time testing and position indication test performed per Work Order PMT. Appendix J
 LLRT performed per applicable steps of 0137-07A (4/7/2007)*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *D. DeAgave*, SECTION XI ISI COORD. Date July 19, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INT OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/4/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Aulucki Commissions NB 11563 A, V, S MN 004028-00
 Inspector's Signature National Board, Province and Endorsements

Date July 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0067

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/14/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 141016
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
- Identification of System Residual Heat Removal Service Water (RHRSW), Loop A, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
SW9-18"-GF	n/a	n/a	n/a	n/a	1969	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SW9-18"-GF	Dubose	n/a	n/a	Ht# A83222	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
SR-396	n/a	n/a	n/a	n/a	2007	Repaired	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced approx 4ft of A-106 Gr. B pipe (line designation SW9-18"-GF in the Torus Room) like-for-like by welding due to localized ID corrosion (replacement pipe built 2000, installed 2007). Removed portions of SR-396 to facilitate piping replacement and reinstalled by welding.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 235-240 psi Test Temp n/a °F

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 3 ² of 2 <small>RWP 7/14/07</small>

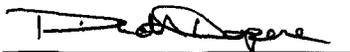
ITEM 2007-23-0067

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

PSI VT-3 performed on SR-396. Reference ISI Report 2007VT733. System leakage test in lieu of a hydro per IWA-4540(a)(2) per WO PMT attachment.

CERTIFICATE OF COMPLIANCE

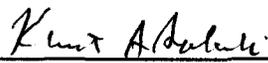
We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , SECT XI ISI COORD Date July 14, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER DIV OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/7/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions MB11863 A, N, E MW004125-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

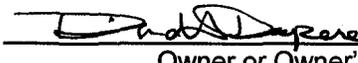
MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 3 of 2 2007 7/14/07

ITEM 2007-23-0068

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)
 System leakage test in lieu of a hydro per IWA-4540(a)(2) per WO PMT attachment.

CERTIFICATE OF COMPLIANCE

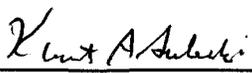
We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
 repair/replacement

Signed  SECT XI ISI COORD Date July 14, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER IJS OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/9/07 to 7/23/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11867 A, N, I MN 004528-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 23, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

ITEM 2007-23-0069

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

Initial ISI VT-3 exam reported a bend in the upper rod. (reference ISI Report 2007VT023 and CAP-01082758). Condition evaluated as acceptable, but recommendation was made to replace the rod. Preservice VT-3 performed following rod replacement. (reference ISI Report 2007VT747)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *Richard Depere*, Sect XI ISI Coord Date July 15, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEEL BUILDER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/14/07 to 7/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kurt A. Schubert Commissions WB 11863 A, N, E MW 00428-20
Inspector's Signature National Board, Province and Endorsements
Date JULY 22, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0070

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

- Owner Xcel Energy Company Date 7/14/2007
Name
- Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 Address WO 325774
Repair Organization P.O. No., Job No. etc.
- Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Address Expiration Date n/a
Address
- Identification of System Emergency Service Water (ESW), Div I, Class 3
- (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
(b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
- Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
ESW1-4"-HBD Pipe	unknown	n/a	n/a	n/a	1981	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW1-4"-HBD Pipe	Tioga Pipe	n/a	n/a	Ht# A83222	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW1-4"-HBD Elbow	unknown	n/a	n/a	n/a	1981	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
ESW1-4"-HBD Elbow	Consolidated Power	n/a	n/a	Ht# 7662	2007	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced approx 6 inches of A-106 Gr. B pipe and adjacent A-234 WPB elbow (line designation ESW1-4"-HBD) like-for-like by welding due to localized ID corrosion (replacement pipe built 2000, replacement elbow built 1991, both installed 2007).

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 77 psi Test Temp n/a °F

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 3 of 2 2 JUL 14 2007

ITEM 2007-23-0070

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)
 System leakage tests in lieu of a hydro per IWA-4540(a)(2) per WO PMT attachments.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPAIR/REPLACEMENT ACTIVITY conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed *Donald A. Opere*, SECT XI ISI COORD Date July 14, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/20/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

Kent A. DiLenti Commissions NP 11863 A, N, I, MN 004028-C0
 Inspector's Signature National Board, Province and Endorsements

Date JULY 20, 2007

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2007-23-0071

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/14/2007
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 326373
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System Emergency Service Water (ESW), Div II, Class 3
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Addenda Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
<i>ESW5-4"-HBD Elbow</i>	<i>unknown</i>	<i>n/a</i>	<i>n/a</i>	<i>n/a</i>	<i>1981</i>	<i>Replaced</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
<i>ESW5-4"-HBD Elbow</i>	<i>Consolidated Power</i>	<i>n/a</i>	<i>n/a</i>	<i>Ht# 7662</i>	<i>2007</i>	<i>Replacement</i>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced A-234 WPB elbow (line designation ESW5-4"-HBD) like-for-like by welding due to localized ID corrosion (replacement elbow built 1991, installed 2007).

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure 76 psi Test Temp n/a °F

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2 7/14/07

ITEM 2007-23-0071

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

System leakage test in lieu of a hydro per IWA-4540(a)(2) per WO PMT attachment.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this conforms to the rules on the ASME Code, Section XI. REPAIR/REPLACEMENT ACTIVITY
repair/replacement

Signed *Dino Deperis*, SECT XI ISI COORD Date July 14, 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER CO OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/24/07 to 7/20/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

King A. Deluca Commissions NB11863 A, W, I MN 004028-C0
Inspector's Signature National Board, Province and Endorsements
Date July 20, 2007

APPROVED (Signatures available in Master File)

IV. Form NIS-1 Information

Page 1 of 2

Form NIS-1 Owner's Data Report for Inservice Inspections
(Supplement to Submittal for Cycle 22, 2005 RFO**)**
As required by the Provisions of the ASME Code Rules

-
1. **Owner:** Xcel Energy, LLC / Nuclear Management Company, LLC
Address: 800 Nicollet Mall, Mpls, MN 55402 / 700 1st St., Hudson WI 54016
 2. **Plant:** Monticello Nuclear Generating Plant
Address: Monticello, MN 55362
 3. **Plant Unit:** 1
 4. **Owner (Certificate of Authorization):** N/A
 5. **Commercial Service Date:** 6/30/71
 6. **National Board No. for Unit:** N/A
 7. **Components Inspected:**

<u>Component or Appurtenance</u>	<u>Manufacturer or Installer</u>	<u>Manufacturer or Installer Serial No.</u>	<u>State or Province No.</u>	<u>National Board No.</u>
Reactor Pressure Vessel	Chicago Bridge & Iron	B-4697	---	---

(See appendices for components inspected during the cycle)

8. **Examination Dates:** 05/26/03 to 04/13/05.
9. **Inspection Interval:** 05/01/03 to 05/31/12.
10. **Applicable Edition of Section XI:** 1995 Addenda: 1996.
11. **Abstract of Examinations.** Include a list of examinations and a statement concerning status of work required for current interval.
 - See Sections IV and V for supplemental Form NIS-2 Cycle 22 information
12. **Abstract of Conditions Noted.**
 - See Sections IV and V for supplemental Form NIS-2 Cycle 22 information
14. **Abstract of Corrective Measures Recommended and Taken.**
 - See Sections IV and V for supplemental Form NIS-2 Cycle 22 information
 -

IV. Form NIS-1 Information

(**Supplement to Submittal for Cycle 22, 2005 RFO**)

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Certification of Authorization No. (if applicable) N/A Expiration Date N/A

Date July 26 20 07 Signed XCEL ENERGY / NMC By [Signature]
Owner Section XI ISI Coordinator

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by MARTFORD STEAM BOILER LTD OF CONNECTICUT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 5/26/07 to 7/26/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11867 A, N, I 144 004028-00
Inspector's Signature National Board, State, Province,
Endorsements

Date July 26 2007

V. Form NIS-2 Information

**Supplemental NIS-2 Form
for Cycle 22
2005 RFO Summary Report**

Previously unsubmitted Form NIS-2

R/R ITEM NUMBER	WORK DOCUMENT	COMPONENT OR SYSTEM	WORK DESCRIPTION
2006-22-0007	WO-0307010	CV-1728	Replaced valve plug note: replaced during 2005 RFO (reference CAP 01006064)

(2 pages to follow)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 1 of 2

ITEM 2006-22-0007

NOTE 1: This form may not address all items needed for documentation of a repair/replacement program, see 4 AWI-09.04.03 (ASME SECTION XI REPAIR/REPLACEMENT PROGRAM) for further guidance.

NOTE 2: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. Owner Xcel Energy Company Date 7/13/2006
Name
2. Plant Monticello Nuclear Generating Plant Sheet 1 of 2
Name
2807 W. Cnty. Rd. 75, Monticello, Minnesota 55362 WO 0307010
Address Repair Organization P.O. No., Job No. etc.
3. Work Performed By NMC / Xcel Energy Type Code Symbol Stamp n/a
Company Name Authorization No. n/a
2807 W. Cty Rd 75, Monticello, MN 55362 Expiration Date n/a
Address
4. Identification of System RHR Service Water (RHRSW), CV-1728, Class 3
5. (a) Applicable Construction Code ANSI B31.1, 1977 Edition
Year
Addenda Winter 1978 Code Case n/a
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacement 2001 Edition, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced or Replacement	ASME Code Stamped	
							YES	NO
CV-1728	Valtek	47907-001 (valve)	n/a	0411236-29970-01	1987	Replaced	<input type="checkbox"/>	<input checked="" type="checkbox"/>
CV-1728	Flowserve	47907-001 (valve)	n/a	0411236-29970-01	2005	Replacement	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

7. Description of Work Replaced plug
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure n/a psi Test Temp n/a °F

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:			
FOR ADMINISTRATIVE USE ONLY	Resp Supv: PRGE	Assoc Ref: PRGE	SR: N
	ARMS: 3613	Doc Type: REF ONLY	Admin initials:
			Freq: 0 yrs
			Date:

APPROVED (Signatures available in Master File)

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS - FORM NIS-2	Revision 4
		Page 2 of 2

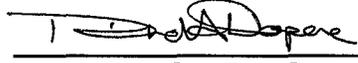
ITEM 2006-22-0007

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Design Change 87Z015 is applicable to valve and Alteration 95A011 is applicable to plug.
 No pressure testing required for internal component.
 IST was performed per the WO PMT Attachment.
 This R/R was performed during 2005 refueling outage (Cycle 22). This NIS-2 will be submitted with the Cycle 23 ISI Summary Report noting amendment. (Ref. CAP A/R 01006064).*

CERTIFICATE OF COMPLIANCE

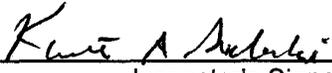
We certify that the statements made in the report are correct and this replacement conforms to the rules on the ASME Code, Section XI. repair/replacement

Signed , Sec XI ISI Coord Date July 14, 2007
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER IPI OF CONNECTICUT HARTFORD, CT have inspected the components described in this Owner's Report during the period DECEMBER 12, 2005 to JULY 14, 2006, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury of property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 A, NJ MN 004028-CO
 Inspector's Signature National Board, Province and Endorsements

Date JULY 14, 2006

APPROVED (Signatures available in Master File)

APPENDIX A
(19 pages to follow)

(in order as listed below)

Class 1 Category B-A

- ISI and PSI Exams

Class 1 Category B-D

- ISI and PSI Exams

Class 1 Category B-G-1

- ISI and PSI Exams

Class 1 Category B-G-2

- ISI and PSI Exams

Class 1 Category B-L-2

- ISI Exams

Class 1 Category B-M-2

- ISI Exams

Class 1 Category B-N-1

- ISI Exams

Class 1 Category B-N-2

- ISI Exams

Class 1 Category F-A

- ISI and PSI Exams

Class 1 Category R-A(H)

- ISI Exams

Class 1 Category R-A(L)

- ISI, PSI, and AUG Exams

Class 1 Category R-A(M)

- ISI, PSI, and AUG Exams

Class 2 Category C-C

- ISI and PSI Exams

Class 2 Category F-A

- ISI and PSI Exams

Class 2 Category R-A(M)

- ISI Exams

Class 3 Category F-A

- ISI and PSI Exams

Non-Code Exams

- AUG Exams

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
-------------	---------	------------	----------	------	-----------	----------------------	--------	---------	-----------

Class 1 Category B-A, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

102643	VLAA-2	Long Seam		B1.12 ISI	PEI-02.03.15	UT 2007UT059 NRI	Reactor Vessel	ISI Fig 4	3/22/2007
Comments:		Limited exam, Supplemental for future ID exam, no relief request required at this time.							

Class 1 Category B-D, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

102374	N- 6A IR	N- 6A Inner Radius		B3.100 ISI	FP-PE-NDE-UT-03	UT 2007UT105 NRI	Reactor Vessel	ISI Fig 1	3/25/2007
102375	N- 6A NV	Nozzle/Head Weld		B3.90 ISI	PEI-02.03.15	UT 2007UT104 NRI	Reactor Vessel	ISI Fig 1	3/25/2007
Comments:		Limited exam less than 90% coverage - Relief Request required							
102657	N- 2B IR	N- 2B Inner Radius		B3.100 ISI	FP-PE-NDE-UT-03	UT 2007UT057 NRI	Reactor Vessel	ISI Fig 5	3/21/2007
102658	N- 2B NV	N- 2B Noz / Vsl Weld		B3.90 ISI	PEI-02.03.15	UT 2007UT058 NRI	Reactor Vessel	ISI Fig 5	3/21/2007
Comments:		Limited exam less than 90% coverage - Relief Request required							
102667	N- 2G IR	N- 2G Inner Radius		B3.100 ISI	FP-PE-NDE-UT-03	UT 2007UT060 NRI	Reactor Vessel	ISI Fig 5	3/22/2007
102668	N- 2G NV	N- 2G Noz/Vsl Weld		B3.90 ISI	PEI-02.03.15	UT 2007UT061 NRI	Reactor Vessel	ISI Fig 5	3/22/2007
Comments:		Limited exam less than 90% coverage - Relief Request required							

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
102683	N- 4A IR	N- 4A Inner Radius		B3.100 ISI	FP-PE-NDE-UT-03	UT 2007UT101 NRI	Reactor Vessel	ISI Fig 5	3/24/2007
102684	N- 4A NV	N- 4A Noz/Vsl Weld		B3.90 ISI	PEI-02.03.15	UT 2007UT103 NRI	Reactor Vessel	ISI Fig 5	3/24/2007
Comments:		Limited exam less than 90% coverage - Relief Request required							
102699	N- 9 IR	N- 9 Inner Radius		B3.100 ISI	FP-PE-NDE-UT-03	UT 2007UT100 NRI	Reactor Vessel	ISI Fig 5	3/24/2007
102700	N- 9 NV	N- 9 Ves/Noz Weld		B3.90 ISI	PEI-02.03.15	UT 2007UT102 NRI	Reactor Vessel	ISI Fig 5	3/24/2007
Comments:		Limited exam less than 90% coverage - Relief Request required							

Class 1 Category B-G-1, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

102076	B- 2	RC Pump B Studs, Nuts		B6.180 ISI	PEI-02.03.06	UT 2007UT125 NRI	Recirculation B	ISI-97006-A	3/25/2007
				ISI	PEI-02.05.01	VT 2007VT044 NRI	Recirculation B	ISI-97006-A	3/25/2007
				PSI	PEI-02.05.01	VT 2007VT045 NRI	Recirculation B	ISI-97006-A	3/25/2007
107184	P200-B	Flange Surface		B6.190 ISI	PEI-02.05.01	VT 2007VT475 NRI	Recirculation B	ISI-97006-A	3/25/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
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Class 1 Category B-G-2, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

100567	B- 2	Flange Bolts @ W-14	B7.50 ISI	PEI-02.05.01	VT 2007VT032	NRI	Main Steam A	ISI-13142-33-A	3/19/2007
100568	B- 3	Flange Bolts @ W-16	B7.50 PSI	PEI-02.05.01	VT 2007VT486	NRI	Main Steam A	ISI-13142-33-A	4/3/2007
100574	B- 9	Valve Bolts	B7.70 PSI	PEI-02.05.01	VT 2007VT487	NRI	Main Steam A	ISI-13142-33-A	4/3/2007
100622	B- 2	Flange Bolts @ W-15	B7.50 ISI	PEI-02.05.01	VT 2007VT031	NRI	Main Steam B	ISI-13142-34-A	3/19/2007
100626	B- 6	Valve Bolts	B7.70 PSI	PEI-02.05.01	VT 2007VT488	NRI	Main Steam B	ISI-13142-34-A	4/3/2007
100629	B- 7	Valve Bolts	B7.70 PSI	PEI-02.05.01	VT 2007VT459	NRI	Main Steam B	ISI-13142-34-A	4/1/2007
100974	B- 2	Valve Bolts	B7.70 PSI	PEI-02.05.01	VT 2007VT494	NRI	HPCI Steam	ISI-13142-42-A	4/4/2007
			ISI	PEI-02.05.01	VT 2007VT495	NRI	HPCI Steam	ISI-13142-42-A	4/4/2007
101408	B- 1	Flange Bolts	B7.50 ISI	PEI-02.05.01	VT 2007VT732	NRI	Recirc B Drain Line	ISI-74210-1A	4/11/2007
101492	B- 2	Flange Bolts	B7.50 PSI	PEI-02.05.01	VT 2007VT477	NRI	Head Vent	ISI-782A-A	4/3/2007
101929	B- 2	Valve Bolts	B7.70 ISI	PEI-02.05.01	VT 2007VT474	NRI	RHR Return B	ISI-97004-A	3/27/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
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Class 1 Category B-L-2, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

102099	P200-B	Pump Casing Internal Surfaces	B12.20 ISI	PEI-02.05.04	VT 2007VT043	NRI	Recirculation B	ISI-97006-A	3/25/2007
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Class 1 Category B-M-2, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

100992	V- 2	Valve Int Surfaces	B12.50 ISI	PEI-02.05.04	VT 2007VT493	NRI	HPCI Steam	ISI-13142-42-A	4/4/2007
101938	V- 2	Valve Int Surfaces	B12.50 ISI	PEI-02.05.04	VT 2007VT446	NRI	RHR Return B	ISI-97004-A	3/27/2007

Class 1 Category B-N-1, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

102716	C-1A	Top Guide (areas made access., fuel cell vacated)	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007
102718	C-3A	Shroud Shelf 0-180 deg	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107690	C-3B	Shrd Shf 180-360 deg	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107737	C-11	Incore Dry Tubes (if made accessible during RFO)	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107738	C-12A	Core Plate (areas made access., fuel cell vacated)	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007
102668	N- 2G NV	Core Plate (areas made access., fuel cell vacated)	B13.10 ISI	PEI-02.05.05	VT-3 AREVA-RPT	NRI	Reactor Vessel	ISI Fig 0	3/31/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
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Class 1 Category B-N-2, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

107691	C-3C	Shroud Shelf H-9 Weld 0-180 deg	B13.30 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107692	C-3D	Shroud Shelf H-9 Weld 180-360 deg	B13.30 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107739	C-12B	Core Plate (areas made access., fuel cell vacated)	B13.40 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107744	C-17A	Fuel Spt Cast (areas access, fuel cell vacated)	B13.40 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107746	C-18	Ctrl Rod Guide Tube - Int (when accessible)	B13.40 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107747	C-3E	Shroud Shelf 0-180 deg	B13.40 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007
107748	C-3F	Shroud Shelf 180-360 deg	B13.40 ISI	PEI-02.05.05	VT-3 AREVA-RPT NRI	Reactor Vessel	ISI Fig 0	3/31/2007

Class 1 Category F-A, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

100582	H- 6	Snubber / Clamp	F1.10b PSI	PEI-02.05.02	VT 2007VT022 NRI	Main Steam A	ISI-13142-33-A	3/15/2007
			ISI	PEI-02.05.02	VT 2007VT021 NRI	Main Steam A	ISI-13142-33-A	3/15/2007
100682	H- 2	Snubber / Clamp	F1.10b PSI	PEI-02.05.02	VT 2007VT013 NRI	Main Steam C	ISI-13142-35-A	3/15/2007
100683	H- 3	Snubber / Clamp	F1.10b PSI	PEI-02.05.02	VT 2007VT012 NRI	Main Steam C	ISI-13142-35-A	3/15/2007
101198	H- 4	Snubber / Clamp	F1.10b PSI	PEI-02.05.02	VT 2007VT717 NRI	Feedwater	ISI-13142-52-A	3/30/2007
101201	H- 7	Snubber / Clamp	F1.10b PSI	PEI-02.05.02	VT 2007VT713 NRI	Feedwater	ISI-13142-52-A	3/30/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
101253	H-5	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT710 NRI	Feedwater	ISI-13142-53-A	3/30/2007
101255	H-7	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT482 NRI	Feedwater	ISI-13142-53-A	3/31/2007
101855	H-3	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT016 NRI	RHR Return A	ISI-97003-A	3/14/2007
101896	H-2	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT709 NRI	RHR Suction A	ISI-97003-B	4/5/2007
				PSI	PEI-02.05.02	VT 2007VT014 NRI	RHR Suction A	ISI-97003-B	3/14/2007
101934	H-2	Variable/Slide/Clamp		F1.10c ISI	PEI-02.05.02	VT 2007VT030 NRI	RHR Return B	ISI-97004-A	4/14/2007
101935	H-3	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT484 NRI	RHR Return B	ISI-97004-A	3/30/2007
101936	H-4	Snubber / Clamp		F1.10b PSI	PEI-02.05.02	VT 2007VT481 NRI	RHR Return B	ISI-97004-A	3/30/2007
102027	H-10	Restraint		F1.10a ISI	PEI-02.05.02	VT 2007VT041 NRI	Recirc Manifold A	ISI-97005-B	3/25/2007
105020	H-5A	Hanger		F1.10a ISI	PEI-02.05.02	VT 2007VT026 NRI	Head Vent	ISI-782A	3/18/2007
105022	H-17	Spring / Clamp		F1.10c ISI	PEI-02.05.02	VT 2007VT023 RI	Head Vent	ISI-782A	3/18/2007
				PSI	PEI-02.05.02	VT 2007VT747 NRI	Head Vent	ISI-782A	4/14/2007
105023	H-1	Hanger		F1.10a PSI	PEI-02.05.02	VT 2007VT745 NRI	Head Vent	ISI-782A-A	4/12/2007
105024	H-2	Spring / Clamp		F1.10c PSI	PEI-02.05.02	VT 2007VT748 NRI	Head Vent	ISI-782A-A	4/12/2007
105025	H-3	Hanger		F1.10a PSI	PEI-02.05.02	VT 2007VT746 NRI	Head Vent	ISI-782A-A	4/12/2007
107523	H-3	Dbl Spring / 4 Lugs		F1.10c ISI	PEI-02.05.02	VT 2007VT029 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
107528	H- 8	Snubber / Lugs		F1.10b	PEI-02.05.02	VT 2007VT712 NRI	Recirculation A	ISI-97005-A	3/31/2007
				PSI	PEI-02.05.02	VT 2007VT020 NRI	Recirculation A	ISI-97005-A	3/15/2007
				ISI	PEI-02.05.02	VT 2007VT019 NRI	Recirculation A	ISI-97005-A	3/15/2007
107534	H- 1	Rod/Clevis Grip/Lugs/Constant-Support		F1.40c ISI	PEI-02.05.02	VT 2007VT034 NRI	Recirc Pump A	ISI-97005-C	3/19/2007
107535	H- 2	Rod/Clevis Grip/Lugs/Constant-Support		F1.40c ISI	PEI-02.05.02	VT 2007VT033 NRI	Recirc Pump A	ISI-97005-C	3/19/2007
107540	H- 7	Snubber / Lugs		F1.40b PSI	PEI-02.05.02	VT 2007VT024 NRI	Recirc Pump A	ISI-97005-C	3/18/2007
107541	H- 8	Snubber / Lugs		F1.40b PSI	PEI-02.05.02	VT 2007VT714 NRI	Recirc Pump A	ISI-97005-C	4/6/2007
107542	H- 9	Snubber / Lugs		F1.40b PSI	PEI-02.05.02	VT 2007VT715 NRI	Recirc Pump A	ISI-97005-C	4/6/2007
107547	H- 8	Snubber / Lugs		F1.10b PSI	PEI-02.05.02	VT 2007VT480 NRI	Recirculation B	ISI-97006-A	3/31/2007
107555	H- 3	Rod/Clevis Grip/Lugs/Constant-Support		F1.40c PSI	PEI-02.05.02	VT 2007VT485 NRI	Recirc Pump B	ISI-97006-C	3/29/2007
107559	H- 7	Snubber / Lugs		F1.40b PSI	PEI-02.05.02	VT 2007VT025 NRI	Recirc Pump B	ISI-97006-C	3/18/2007

Class 1 Category R-A(H), EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

101874	W-15	Pipe-to-Elbow		R1.11 ISI	FP-PE-NDE-401	UT 2007UT118 NRI	RHR Return A	ISI-97003-A	3/27/2007
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NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date	
Class 1 Category R-A(L), EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams), PSI, and AUGMENTED (HELB / SUPERPIPE)										
100618	W-30	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT068	NRI	Main Steam A	ISI-13142-33-A	3/17/2007
				AUG						
100619	W-31	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT066	NRI	Main Steam A	ISI-13142-33-A	3/17/2007
				AUG						
				AUG						
100668	W-30	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT005	NRI	Main Steam B	ISI-13142-34-A	3/16/2007
				AUG						
				AUG						
100669	W-31	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT008	NRI	Main Steam B	ISI-13142-34-A	3/16/2007
				AUG						
				AUG						
100670	W-32	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT007	NRI	Main Steam B	ISI-13142-34-A	3/16/2007
				AUG						
				AUG	FP-PE-NDE-401	UT 2007UT006	NRI	Main Steam B	ISI-13142-34-A	3/16/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
100720	W-30	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT030	Main Steam C	ISI-13142-35-A	3/16/2007
				AUG		NRI			
				AUG		2007UT031			
100721	W-31	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT032	Main Steam C	ISI-13142-35-A	3/16/2007
				AUG		NRI			
				AUG		2007UT033			
100722	W-32	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT034	Main Steam C	ISI-13142-35-A	3/16/2007
				AUG		NRI			
				AUG		2007UT037			
100723	W-33	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT038	Main Steam C	ISI-13142-35-A	3/16/2007
				AUG		NRI			
				AUG		2007UT039			
100724	W-33	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT040	Main Steam C	ISI-13142-35-A	3/16/2007
				AUG		NRI			
				AUG		2007UT041			
100778	W-32	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT002	Main Steam D	ISI-13142-36-A	3/16/2007
				AUG		GEO			
				AUG		2007UT023			
100779	W-33	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT001	Main Steam D	ISI-13142-36-A	3/16/2007
				AUG		NRI			
				AUG		2007UT026			
100780	W-34	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT025	Main Steam D	ISI-13142-36-A	3/16/2007
				AUG		NRI			
				AUG		2007UT024			
100780	W-34	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT028	Main Steam D	ISI-13142-36-A	3/16/2007
				AUG		GEO			
				AUG		2007UT027			
100780	W-34	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT029	Main Steam D	ISI-13142-36-A	3/16/2007
				AUG		NRI			

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date		
101056	W-14	Pipe-to-Valve		R1.20	FP-PE-NDE-401	UT 2007UT062	RCIC Steam	ISI-13142-43-A	3/21/2007		
				AUG		NRI					
				AUG	FP-PE-NDE-401	UT 2007UT063	RCIC Steam	ISI-13142-43-A	3/21/2007		
101529	W- 7	Tee-to-Pipe		R1.20 PSI	PEI-02.01.01	PT 2007PT003	MS Condensate Lkoff	ISI-786A	4/1/2007		
101530	W- 8	Valve-to-Pipe		R1.20 PSI	PEI-02.01.01	PT 2007PT004	MS Condensate Lkoff	ISI-786A	1/25/2007		
101531	W- 9	Elbow-to-Pipe		R1.20 PSI	PEI-02.01.01	PT 2007PT005	MS Condensate Lkoff	ISI-786A	1/25/2007		
101535	W-13	Pipe-to-Pipe		R1.20 PSI	PEI-02.01.01	PT 2007PT006	MS Condensate Lkoff	ISI-786A	4/4/2007		
101536	W-14	Pipe-to-Valve		R1.20 PSI	PEI-02.01.01	PT 2007PT007	MS Condensate Lkoff	ISI-786A	2/6/2007		
107749	W-29A	Pipe-to-Pipe		R1.20	FP-PE-NDE-401	UT 2007UT082	Main Steam A	ISI-13142-33-A	3/17/2007		
				ISI		UT 2007UT081				GEO	3/17/2007
				AUG		UT 2007UT080				NRI	3/17/2007
				AUG		UT 2007UT079				GEO	3/17/2007
				AUG		UT 2007UT077				NRI	3/17/2007
				ISI	FP-PE-NDE-401	UT 2007UT078	Main Steam A	ISI-13142-33-A	3/17/2007		
107751	W- 7A	Pipe-to-Valve		R1.20 PSI	PEI-02.01.01	PT 2007PT008	MS Condensate Lkoff	ISI-786A	1/25/2007		
107752	W- 9A	Pipe-to-Pipe		R1.20 PSI	PEI-02.01.01	PT 2007PT009	MS Condensate Lkoff	ISI-786A	4/1/2007		

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date	
Class 1 Category R-A(M), EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams), PSI, and AUGMENTED (HELB / SUPERPIPE)										
100439	W- 6	Valve-to-Pipe	R1.20	AUG	FP-PE-NDE-401	UT 2007UT122	GEO	Core Spray B	ISI-13142-26-A	3/25/2007
				AUG	FP-PE-NDE-401	UT 2007UT120	NRI	Core Spray B	ISI-13142-26-A	3/25/2007
				AUG	FP-PE-NDE-401	UT 2007UT121	GEO	Core Spray B	ISI-13142-26-A	3/25/2007
100440	W- 7	Pipe-to-Pipe	R1.20	AUG	FP-PE-NDE-401	UT 2007UT128	GEO	Core Spray B	ISI-13142-26-A	3/25/2007
				AUG	FP-PE-NDE-401	UT 2007UT126	NRI	Core Spray B	ISI-13142-26-A	3/25/2007
				AUG	FP-PE-NDE-401	UT 2007UT127	GEO	Core Spray B	ISI-13142-26-A	3/25/2007
100500	W- 1	Valve-to-Pipe	R1.20	AUG	FP-PE-NDE-401	UT 2007UT072	GEO	Core Spray A	ISI-13142-31-A	3/18/2007
				AUG	FP-PE-NDE-401	UT 2007UT071	NRI	Core Spray A	ISI-13142-31-A	3/18/2007
100501	W- 2	Pipe-to-Pipe	R1.20	ISI	FP-PE-NDE-401	UT 2007UT074	GEO	Core Spray A	ISI-13142-31-A	3/18/2007
				ISI	FP-PE-NDE-401	UT 2007UT073	NRI	Core Spray A	ISI-13142-31-A	3/18/2007
100591	W- 3	Elbow-to-Pipe	R1.20	ISI	FP-PE-NDE-401	UT 2007UT137	NRI	Main Steam A	ISI-13142-33-A	3/27/2007
				ISI	FP-PE-NDE-401	UT 2007UT133	NRI	Main Steam A	ISI-13142-33-A	3/27/2007
100642	W- 4	Pipe-to-Elbow	R1.20	ISI	FP-PE-NDE-401	UT 2007UT132	NRI	Main Steam B	ISI-13142-34-A	3/27/2007
				ISI	FP-PE-NDE-401	UT 2007UT136	NRI	Main Steam B	ISI-13142-34-A	3/27/2007
101219	W-13	Elbow-to-Pipe	R1.20	ISI	FP-PE-NDE-401	UT 2007UT076	GEO	Feedwater	ISI-13142-52-A	3/22/2007
				ISI	FP-PE-NDE-401	UT 2007UT075	NRI	Feedwater	ISI-13142-52-A	3/22/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
101232	W-26	Tee-to-Reducer	R1.18	ISI	FP-PE-NDE-401	UT 2007UT021 NRI	Feedwater	ISI-13142-52-A	3/17/2007
				ISI	FP-PE-NDE-401	UT 2007UT022 NRI	Feedwater	ISI-13142-52-A	3/17/2007
101233	W-27	Reducer-to-Pipe	R1.18	ISI	FP-PE-NDE-401	UT 2007UT020 NRI	Feedwater	ISI-13142-52-A	3/17/2007
				ISI	FP-PE-NDE-401	UT 2007UT019 NRI	Feedwater	ISI-13142-52-A	3/17/2007
101371	W- 6 LS U	Pipe-to-Elbow	R1.20	ISI	FP-PE-NDE-401	UT 2007UT088 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
				ISI	FP-PE-NDE-401	UT 2007UT087 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
101379	W-14 LS U&D	Pipe-to-Pipe	R1.20	ISI	FP-PE-NDE-401	UT 2007UT085 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
				ISI	FP-PE-NDE-401	UT 2007UT086 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
101380	W-15 LS U	Pipe-to-Elbow	R1.20	ISI	FP-PE-NDE-401	UT 2007UT084 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
				ISI	FP-PE-NDE-401	UT 2007UT083 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/18/2007
101391	W-26	Pipe-to-Valve	R1.20	ISI	FP-PE-NDE-401	UT 2007UT124 GEO	Reactor Wtr Cleanup	ISI-73880-A	y
				ISI	FP-PE-NDE-401	UT 2007UT123 NRI	Reactor Wtr Cleanup	ISI-73880-A	3/26/2007
101860	W- 1	Valve-to-Pipe	R1.20	AUG	FP-PE-NDE-401	UT 2007UT129 NRI	RHR Return A	ISI-97003-A	3/28/2007
				AUG	FP-PE-NDE-401	UT 2007UT131 NRI	RHR Return A	ISI-97003-A	3/28/2007
				AUG	FP-PE-NDE-401	UT 2007UT130 GEO	RHR Return A	ISI-97003-A	3/28/2007
101866	W- 7	Elbow-to-Pipe	R1.11	ISI	FP-PE-NDE-401	UT 2007UT119 NRI	RHR Return A	ISI-97003-A	3/27/2007
101997	W- 3 LS U&D	Pipe-to-Elbow	R1.20	ISI	FP-PE-NDE-402	UT 2007UT048 NRI	Recirculation A	ISI-97005-A	3/17/2007
				ISI	FP-PE-NDE-402	UT 2007UT047 NRI	Recirculation A	ISI-97005-A	3/17/2007
101998	W- 4 LS U&D	Elbow-to-Pipe	R1.20	ISI	FP-PE-NDE-402	UT 2007UT046 NRI	Recirculation A	ISI-97005-A	3/17/2007
				ISI	FP-PE-NDE-402	UT 2007UT045 NRI	Recirculation A	ISI-97005-A	3/17/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
102003	W- 9 LS U&D	Pipe-to-Bent Pipe	R1.20 ISI		FP-PE-NDE-402	UT 2007UT011 NRI	Recirculation A	ISI-97005-A	3/18/2007
					FP-PE-NDE-402	UT 2007UT012 NRI	Recirculation A	ISI-97005-A	3/18/2007
102041	W-11	Safe End-to-Nozzle	R1.20 PSI		PEI-02.01.01	PT 2007PT002 NRI	Recirc Manifold A	ISI-97005-B	3/26/2007
			ISI		FP-PE-NDE-410	UT 2007UT108 NRI	Recirc Manifold A	ISI-97005-B	3/26/2007
			ISI		FP-PE-NDE-410	UT 2007UT106 NRI	Recirc Manifold A	ISI-97005-B	3/26/2007
			ISI		FP-PE-NDE-410	UT 2007UT107 NRI	Recirc Manifold A	ISI-97005-B	3/26/2007
Comments:	Limited exam, reference Code Case N-460								
102164	W-25	Safe End-to-Nozzle	R1.20 ISI		FP-PE-NDE-410	UT 2007UT110 NRI	Recirc Manifold B	ISI-97006-B	3/22/2007
			ISI		FP-PE-NDE-410	UT 2007UT109 NRI	Recirc Manifold B	ISI-97006-B	3/22/2007
			ISI		FP-PE-NDE-410	UT 2007UT111 NRI	Recirc Manifold B	ISI-97006-B	3/22/2007
Comments:	Limited exam less than 90% coverage - Relief Request required								
102211	W-14	Pipe-to-Elbow	R1.20 ISI		FP-PE-NDE-401	UT 2007UT090 GEO	RHR Equalizer	ISI-97027-A	3/18/2007
			ISI		FP-PE-NDE-401	UT 2007UT089 GEO	RHR Equalizer	ISI-97027-A	3/18/2007
			ISI		FP-PE-NDE-401	UT 2007UT091 GEO	RHR Equalizer	ISI-97027-A	3/18/2007
102212	W-15	Pipe-to-Elbow	R1.20 ISI		FP-PE-NDE-401	UT 2007UT094 NRI	RHR Equalizer	ISI-97027-A	3/18/2007
			ISI		FP-PE-NDE-401	UT 2007UT093 GEO	RHR Equalizer	ISI-97027-A	3/18/2007
			ISI		FP-PE-NDE-401	UT 2007UT092 GEO	RHR Equalizer	ISI-97027-A	3/18/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
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Class 2 Category C-C, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

100815	H- 3	Strut / 8 Lugs		C3.20 ISI	PEI-02.02.01	MT 2007MT003 NRI	Containment Spray	ISI-13142-37-B	3/30/2007
100978	H- 3	Dbl Spring / 4 Lugs		C3.20 ISI	PEI-02.02.01	MT 2007MT002 NRI	HPCI Steam	ISI-13142-42-A	3/20/2007
102753	Support, Pump A	Pump Support		C3.30 ISI	PEI-02.02.01	MT 2007MT004 NRI	Core Spray Pumps	ISI-49	4/4/2007
107616	H- 5	Variable / Support		C3.20 ISI	PEI-02.02.01	MT 2007MT001 NRI	Core Spray B	ISI-13142-20-B	3/18/2007
107650	H- 4	Slide		C3.20 PSI	PEI-02.01.01	PT 2007PT001 NRI	RHR Suction B	ISI-13142-17-C	4/7/2007

Class 2 Category F-A, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

100098	H- 1	Rod / Clevis		F1.20a ISI	PEI-02.05.02	VT 2007VT040 NRI	HPCI Water Side Sctn	ISI-13142-17-B	3/22/2007
100131	H- 4	Slide		F1.20b PSI	PEI-02.05.02	VT 2007VT720 NRI	RHR Suction B	ISI-13142-17-C	4/7/2007
100132	H- 5	Strut / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT721 NRI	RHR Suction B	ISI-13142-17-C	4/7/2007
100168	H- 1	Spring / Clamp		F1.20c ISI	PEI-02.05.02	VT 2007VT750 NRI	RHR Discharge B	ISI-13142-18-A	3/22/2007
100263	H- 1	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT483 NRI	HPCI Steam Disch	ISI-13142-19-A	4/1/2007
100365	H- 5	Variable / Support		F1.20a ISI	PEI-02.05.02	VT 2007VT027 NRI	Core Spray B	ISI-13142-20-B	3/18/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
100824	H- 9	Spring / Clamp		F1.20c ISI	PEI-02.05.02	VT 2007VT456 NRI	Containment Spray	ISI-13142-37-B	3/29/2007
100879	H- 5	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT749 NRI	HPCI Water Side Dsch	ISI-13142-40-A	4/6/2007
100987	H-11	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT015 NRI	HPCI Steam	ISI-13142-42-A	3/16/2007
				PSI	PEI-02.05.02	VT 2007VT716 NRI	HPCI Steam	ISI-13142-42-A	4/7/2007
101682	DVMX Supp A	DVMX Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT008 NRI	HPCI Pumps	ISI-8292-42A	3/14/2007
101683	DVMX Supp B	DVMX Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT005 NRI	HPCI Pumps	ISI-8292-42A	3/14/2007
101684	DVMX Supp C	DVMX Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT006 NRI	HPCI Pumps	ISI-8292-42A	3/14/2007
101688	DVS Supp C	DVS Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT007 NRI	HPCI Pumps	ISI-8292-42A	3/14/2007
101689	DVS Supp D	DVS Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT009 NRI	HPCI Pumps	ISI-8292-42A	3/14/2007
101710	H- 9	Strut / Clamp		F1.20b ISI	PEI-02.05.02	VT 2007VT018 NRI	CRD Scram Header A	ISI-93268-1A	3/15/2007
101713	H-12	Snubber/ Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT017 NRI	CRD Scram Header A	ISI-93268-1A	3/15/2007
101715	H-14	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT028 NRI	CRD Scram Header A	ISI-93268-1A	3/19/2007
102741	Pump Supp A	Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT002 NRI	RCIC Turbine & Pump	ISI-47	3/14/2007
102742	Pump Supp B	Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT001 NRI	RCIC Turbine & Pump	ISI-47	3/14/2007
102743	Pump Supp C	Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT004 NRI	RCIC Turbine & Pump	ISI-47	3/14/2007
102744	Pump Supp D	Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT003 NRI	RCIC Turbine & Pump	ISI-47	3/14/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
106042	H- 2	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT708 NRI	Vacuum Relief & CGCS Outlet Div.1	ISI-94966-A	4/6/2007
106044	H- 4	Snubber / Clamp		F1.20b PSI	PEI-02.05.02	VT 2007VT707 NRI	Vacuum Relief & CGCS Outlet Div.1	ISI-94966-A	4/6/2007
106810	H- 1	Double Snubber		F1.20b PSI	PEI-02.05.02	VT 2007VT719 NRI	RCIC Steam Discharge	ISI-13142-19-B	4/5/2007
				PSI	PEI-02.05.02	VT 2007VT718 NRI	RCIC Steam Discharge	ISI-13142-19-B	4/5/2007
106820	H- 1	Snubber		F1.20b PSI	PEI-02.05.02	VT 2007VT042 NRI	CORE SPRAY B	ISI-13142-26-B	3/25/2007
106825	H- 6	RESTRAINT HANGER TWH-121		F1.20a ISI	PEI-02.05.02	VT 2007VT476 NRI	CORE SPRAY B	ISI-13142-26-B	3/22/2007
106838	H- 9	Restraint Hanger		F1.20a PSI	PEI-02.05.02	VT 2007VT445 NRI	Core Spray B Discharge	ISI-13142-26-C	3/27/2007
106881	H- 2	Restraint Hanger		F1.20a PSI	PEI-02.05.02	VT 2006VT001 NRI	Core Spray A Discharge	ISI-13142-31-D	12/19/2006
106996	H- 7	Seismic Restraint		F1.20a ISI	PEI-02.05.02	VT 2007VT478 NRI	Fuel Pool Emergency Cooling	ISI-13142-67	4/2/2007
107049	H- 2	Snubber		F1.20b PSI	PEI-02.05.02	VT 2007VT711 NRI	RHR A	ISI-13142-37-E	4/6/2006
107573	H- 2	Strut / 8 Lugs		F1.20b ISI	PEI-02.05.02	VT 2007VT458 NRI	Containment Spray	ISI-13142-37-B	3/30/2007
107574	H- 3	Strut / 8 Lugs		F1.20b ISI	PEI-02.05.02	VT 2007VT457 NRI	Containment Spray	ISI-13142-37-B	3/30/2007
107579	H- 3	Dbl Spring / 4 Lugs		F1.20c ISI	PEI-02.05.02	VT 2007VT039 NRI	HPCI Steam	ISI-13142-42-A	3/20/2007
107589	H-14	Tank Support		F1.20a ISI	PEI-02.05.02	VT 2007VT010 NRI	CRD Scram Header B	ISI-93268-3A	3/15/2007
107594	Support, Pump A	Pump Support		F1.40a ISI	PEI-02.05.02	VT 2007VT699 NRI	Core Spray Pumps	ISI-49	4/4/2007
107750	H-1A	SEISMIC RESTRAINT HANGER		F1.20a PSI	PEI-02.05.02	VT 2007VT700 NRI	CORE SPRAY B	ISI-13142-26-B	3/29/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
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Class 2 Category R-A(M), EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams)

100945	W-20	Pipe-to-Elbow	R1.20	FP-PE-NDE-401	UT	2007UT015	NRI	HPCI Water Side Dsch	ISI-13142-40-B	3/20/2007
			ISI	FP-PE-NDE-401	UT	2007UT016	NRI	HPCI Water Side Dsch	ISI-13142-40-B	3/20/2007
100948	W-23	Elbow-to-Pipe	R1.20	FP-PE-NDE-401	UT	2007UT044	GEO	HPCI Water Side Dsch	ISI-13142-40-B	3/20/2007
			ISI	FP-PE-NDE-401	UT	2007UT043	GEO	HPCI Water Side Dsch	ISI-13142-40-B	3/20/2007
			ISI	FP-PE-NDE-401	UT	2007UT042	NRI	HPCI Water Side Dsch	ISI-13142-40-B	3/20/2007
100962	W-13	Pipe-to-Tee	R1.20	FP-PE-NDE-401	UT	2007UT070	NRI	RCIC Water Suction	ISI-13142-41-A	3/18/2007
			ISI	FP-PE-NDE-401	UT	2007UT069	NRI	RCIC Water Suction	ISI-13142-41-A	3/18/2007

Class 3 Category F-A, EXAMS PERFORMED FOR ISI (4th Interval, Period 2 Credited Exams) and PSI

102243	SWH-304	Double Spring	F1.30c ISI	PEI-02.05.02	VT	2007VT455	NRI	RHR Service Water	nd-isi-102	3/29/2007
102244	SWH-305	Base Plate	F1.30a ISI	PEI-02.05.02	VT	2007VT011	NRI	RHR Service Water	nd-isi-102	3/15/2007
102251	SR-396	Seismic Restraint	F1.30a PSI	PEI-02.05.02	VT	2007VT733	NRI	RHR Service Water	nd-isi-103	4/11/2007
102303	SR-385	Strut	F1.30b PSI	PEI-02.05.02	VT	2007VT447	NRI	RHR Service Water	nd-isi-108	3/28/2007
102326	SR-88	Stanchion	F1.30b ISI	PEI-02.05.02	VT	2007VT038	NRI	RHR Service Water	nd-isi-111	3/18/2007

NIS-1 Attachment (Appendix A)
Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
Class n/a Category NON CODE, AUGMENTED EXAMS PERFORMED FOR THERMAL FATIGUE									
105009	W- 1	Pipe-to-Pipe	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT017 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT018 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
105010	W- 2	Pipe-to-Tee	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT014 GEO	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT013 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
105011	W- 3	Tee-to-Pipe	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT049 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT050 GEO	RCIC Feedwater	NC-ISI-37	3/17/2007
105012	W- 4	Tee-to-Elbow	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT053 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT054 GEO	RCIC Feedwater	NC-ISI-37	3/17/2007
105013	W-12	Pipe-to-Pipe	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT055 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT056 GEO	RCIC Feedwater	NC-ISI-37	3/17/2007
105014	W-12A	Pipe-to-Tee	NCR95-068	AUG	FP-PE-NDE-401	UT 2007UT052 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
				AUG	FP-PE-NDE-401	UT 2007UT051 NRI	RCIC Feedwater	NC-ISI-37	3/17/2007
105016	W-11	Pipe-to-Tee	NCR95-068	AUG	PEI-02.03.11	UT 2007UT112 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	PEI-02.03.11	UT 2007UT116 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	PEI-02.03.11	UT 2007UT117 GEO	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	PEI-02.03.11	UT 2007UT115 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	PEI-02.03.11	UT 2007UT114 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	PEI-02.03.11	UT 2007UT113 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007

NIS-1 Attachment (Appendix A)
 Cycle 22 Inservice Inspection Report (2007 RFO)

Owner: Xcel Energy, LLC, 800 Nicollet Mall, Minneapolis, MN 55402
 Operator: Nuclear Management Company, LLC, 700 First Street, Hudson, WI 54016
 Plant: Monticello Nuclear Generating Plant, Unit 1, 2807 West County Rd #75, Monticello, MN 55362

Owner Certificate of Authorization (If Req.): N/A
 Commercial Service Date: 6/30/1971
 National Board Number for Unit: N/A

Summary No.	Comp ID	Comp Desc.	Category	Item	Procedure	Method/Sheet/Results	System	ISO Num	Exam Date
105017	W-12	Pipe-to-Tee		NCR95-068	FP-PE-NDE-402	UT 2007UT098 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	FP-PE-NDE-402	UT 2007UT099 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
105018	W-13	Tee-to-Pipe		NCR95-068	FP-PE-NDE-402	UT 2007UT096 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007
				AUG	FP-PE-NDE-402	UT 2007UT097 NRI	CRD TO RWCU	NC-ISI-51	3/24/2007

APPENDIX B

Reactor Vessel IVVI Examinations

(7 pages to follow)

**Monticello Nuclear Generating Plant
2007 RFO-23 Refuel Outage
In-Vessel Visual Examination Summary Report**

1.0 In-Vessel Visual Examination Project Overview:

During the 2007 RFO-23 Monticello Nuclear Generating Plant refueling outage, comprehensive visual examinations of reactor vessel internal components were conducted to ensure the continued integrity of the RPV internals. These examinations were conducted in accordance with Monticello procedure PEI-02.05.05, Revision 1. Additionally, all enhanced visual inspections (EVT-1) were performed per the recommendations set forth in BWRVIP-03. Details of the examinations performed can be found in the individual component data sheets contained in Section 3 of this report. A synopsis of the components examined, the methods utilized, and the examination results are provided below.

1.1 Scope

The following table shows the final scope of In-Vessel Visual Examinations, along with the examination techniques employed:

Component	Scope of Work	Inspection Technique
Jet Pump Assemblies 1-10	Adapter top to Adapter bottom (AD-1)	EVT-1
Jet Pump Assemblies 11,12,13,15,18,19,20	Adapter top to Adapter bottom (AD-1)	EVT-1
	Wedge Bearing Surface (WD-1)	VT-1
Jet Pump Assemblies 14,16,17	Wedge Bearing Surface (WD-1)	EVT-1
Shroud Ledge	H8, H9, and shroud support ledge between JP pairs	VT-3
Access Hole Cover 0° and 180°	Cover to shroud support plate	VT-1
Core Spray Headers and Downcomers on Loop A	Clamp Assemblies on Tee Box Junctions at 90°	VT-3
	Evaluate previous known indication AP3a for apparent growth	EVT-1
	Welds AP1, AP2, AP3a, AP3b, AP4a, AP4b, AP4c, AP4d, AP5, AP6, AP7, AP8a, AP8b, BP5, BP6, BP7, BP8a, BP8b	EVT-1
Core Spray Headers and Downcomers on Loop B	Clamp Assemblies on Tee Box Junctions at 270°	VT-3
	Welds BP1, BP2, BP3a, BP3b, CP5, CP6, CP7, CP8a, CP8b, DP5, DP6, DP7, DP8a, DP8b	EVT-1

Component	Scope of Work	Inspection Technique
Core Spray Sparger - A	Welds S1, S2-1, S2-2, S4-1, S4-2	EVT-1
	S3a&b, S3c-1&2	VT-1
Core Spray Sparger - D	Welds S1, S2-1, S2-2, S4-1, S4-2	EVT-1
	S3a&b	VT-1
Core Spray Sparger Brackets	CS-SB @ 13 ^o , 54 ^o , 87 ^o , 272 ^o , 306 ^o & 346 ^o	VT-1
Fuel Cells at: 10-27, 14-23, 14-31, 18-27, 22-07, 22-15, 26-19, 26-35, 26-43, 30-11, 30-15, 34-27, 38-23, 38-31, 42-23, 42-27	FSC, Top Guide Grid, Core Spt Plate, Anti-rotation pin, CRGT Lug welds, CRGT ID, CRD spud	VT-3
Core Shroud	Vertical welds V-3 and V-4 from ID	EVT-1
Top Guide	Aligner Pins and Sockets at 252 ^o & 341 ^o	VT-1
Incore Dry Tubes	IRM 36-09	VT-1
	SRM 12-25, SRM 28-17	VT-1
Moisture Separator	Shroud Head Bolts 1 - 36	VT-3
Steam Dryer	Dryer OD Previously Identified Ind. V3@90, V10@90, V10@270, Jacking Bolt @ 325, Sup Brkt.@215	VT-1
	Dryer ID Horiz. Welds: HA-H2,H3,H4,H6,H8,H10 HE- H2,H3,H4,H6,H8,H10	VT-1
	Dryer ID Vert. Welds: HA-V2,V3,V5,V6, HE-V2,V3,V5,V6	VT-1
	Dryer ID Strut Welds: HA-A,B; HB-A,B; HC-A,B; HD-A,B; HE-A,B	VT-1
	Dryer ID Drain Channel and Drain Pipe Welds:DC-V1,A-H1, A-H2, A-H3,V1A, B-H1, B-H2, B-H3,V2,V3,V4, C-H1, C-H2, C-H3,V4A, D-H1, D-H2, D-H3,V4B,E-H1,E-H2, E-H3,V4C, F-H1, F-H2, F-H3,V5,V6, G-H1, G-H2,	VT-1

Component	Scope of Work	Inspection Technique
Steam Dryer	G-H3,V6A, H-H1, H-H2, H-H3,V7,V8,V9, J-H1, J-H2, J-H3,V9A, K-H1, K-H2, K-H3,V10,V11, M-H1, M-H2, M-H3,V11A, N-H1, N-H2, N-H3,V12,V13,V14, P-H1, P-H2, P-H3,V14A, Q-H1, Q-H2, Q-H3,V14B, R-H1, R-H2, R-H3,V14C, S-H1, S-H2, S-H3,V15,V16, T-H1, T-H2, T-H3,V16A, U-H1, U-H2, U-H3,V17,V18,V19, V-H1, V-H2, V-H3,V19A, W-H1, W-H2, W-H3,V20	VT-1

1.2 Examination Results

The examinations confirmed previously reported indications. Thorough evaluations concluded there were no discernable changes in the indications from the original discovery. The previously reported indications included:

- Crack indication in the 90° core spray t-box to header weld in the 80° side header.
- Crack indication in Lifting Lug 325° Jacking Bolt tack weld
- Crack indication in Dryer Weld V3 90
- Crack indication in Dryer Weld V10 90
- Crack indication in Dryer Weld V10 270
- Crack indication in 215° Dryer support Bracket Guide Channel

Current RFO-23 IVVI examinations identified the following new indications and/or conditions:

- Crack indication in Dryer Weld V10 90.
- Fifteen Shroud Head Bolts are incorrectly installed (Indicator window facing the Separator).
- Crack indication on Drain Channel F access plate adjacent to Drain Pipe weld H1.

1.3 Equipment

The equipment used during the Monticello In-Vessel Visual Examinations is as follows:

- **Camera Systems**

- RJ Electronics RCS-8110 Camera System
 - RJ Electronics Camera with Pan and Tilt
 - RJ Electronics CCU
 - RJ Electronics Twin 35 Watt Light Unit
- RJ Electronics RCS-600 Color Camera System
 - RCS 600 Camera
 - RJ Cable
 - RJ CCU
- IST-Rees ETV-1250 Black and White Camera System
 - IST-Reese 1250 Camera
 - IST-Reese Cable
 - IST-Reese CCU
 - Twin 50W Lighting System
- REES B&W Camera System
 - R93 Camera
 - R93 Cable
 - R93 CCU
- Deep Ocean Engineering ROV
 - Firefly Submersible
 - Sony IX-11, CCD Camera
 - Quad 20W Onboard Lights
 - 175' Cable
 - CCU

- **Monitors and Recording Equipment**

- Sharp 20" LCD Flat-Panel Color Monitors
- Sony RDR-GX300 DVD Recorders
- Microphones

- **Cleaning Equipment**

- 10,000 PSI Hydrolazer Supplied by Monticello with AREVA NP Supplied Foot Pedal, Wands, and Weights

- ***Underwater Lights and Power Supply***
 - UL-212 35w Light
 - UL-210 35w Light
 - UL-210 Cable
 - PS-160 Power Supply
 - Birns 500w Light w/cable
 - Variac

- ***Sensitivity, Resolution and Contrast Standards***
 - S/N 7001343 (DB# 14081)
 - S/N 7001342 (DB# 14097)

- ***Support Equipment***
 - AREVA 1" dia. Handling Poles (various lengths)
 - Videonics Titlemaker 3000 Character Generator
 - Various Cables

- ***Level III Computer Work Station***
 - 1-Dell, Pentium™ 2.4G Computer
 - Sony DVD ± R/RW
 - CD-R/W
 - Pinnacle Graphics Card
 - Canon MP830 Color Printer
 - 30 G Hard Drive
 - 110 G Hard Drive
 - 2- 20" LCD Flat Panel Color Monitors
 - 2-Sony DVD Recorder RDR-GX300
 - 1-Alera Technologies DVD Copy Cruiser, DVD/CD Duplicator

Monticello Nuclear Generating Plant
2007 RFO-23 Refuel Outage
In-Vessel Visual Examination Indication History

The following table provides a history of the indications that have been identified prior to RFO-23 in the reactor vessel internal components at the Monticello Nuclear Generating Plant.

Component	Indication	Associated Engineering Evaluations
Steam Dryer	Steam Impingement Wear in Dryer Head in Areas of all Four Main Steam Nozzles	CR20000209
	Crack Indication in 325 ^o Jacking Bolt Tack Weld - Unrepaired	CR98000794
	Crack Indication at the top of weld V3 90	CAP 037603
	Crack Indication at the top of weld V10 90	CAP 037603
	Crack Indication on Dryer Support Bracket Guide weld at 215	CAP 037603
	Crack Indication at the top of weld V10 270	CAP 037603
Jet Pumps	Crack Indication in Jet Pump #10 vessel Side Tack Weld - Unrepaired	CR20000209
	Crack Indication in Jet Pump #14 Vessel Side Tack Weld - Repaired	SRI 94-019
	Crack indication in Jet Pump #15 Vessel Side Tack Weld - Repaired	SRI 94-019
	Crack indication in Jet Pump #18 Vessel Side Tack Weld - Repaired	SRI 94-019
	Crack indication in Jet Pump #14 Beam Bolt retainer Tack Weld - Unrepaired	CR98000794
	Crack indication in Jet Pump #19 Beam Bolt retainer Tack Weld - Unrepaired	CR98000794
	Crack Indication in Jet Pump #8 Secondary Riser Brace Lower Leaf to Block Weld in Leaf Material - Unrepaired	SRI-89-028
	Crack Indication in Jet Pump #19 Secondary Riser Brace Lower Leaf to Block Weld in Block Material - Unrepaired	CR20000209
	Small 0.25 Inch Crack Indication Extending from Arc Strike in Jet Pump #20 Diffuser Weld DF-2 - Unrepaired	Evaluated by NSP as Oxide Layer Break (Non-Relevant)
Shroud Support Legs	Crack indication in 210 ^o Shroud Support Leg to Shroud Support Cylinder Weld - Unrepaired	CR20000209

Component	Indication	Associated Engineering Evaluations
Core Spray	Crack Indication in 90° Tee Box Junction to Header - Repaired	SRI-93-006
N11A and N11B Nozzles	Removal of Typical Oxides from Nozzle Areas Due to Steam Flashing or Other Phenomena	CR20000209
Surveillance Sample Holder	Bent Alignment Bracket on the 30 degree Surveillance Sample Holder	CAP 038142

APPENDIX C

System Pressure Test Examinations credited to Period 1

Test Number	Procedure Title	Date(s)	Results
0255-03-IIC-1	Core Spray System Leakage Test – Loop A	12/20/05	Satisfactory
0255-03-IIC-2	Core Spray System Leakage Test – Loop B	11/16/05	Satisfactory
0255-04-IIC-3	RHR Shutdown Cooling Suction Piping System Leakage Test	5/22/06	Satisfactory
0255-05-IIC-1	RHR Service Water System Leakage Test - Loop A	9/9/05	Partial, Satisfactory (see note 1)
0255-05-IIC-2	RHR Service Water System Leakage Test - Loop B	11/3/05	Partial, Satisfactory
		4/21/06	Partial, Satisfactory (see note 2)
0255-06-IIC-1	HPCI System Leakage Test	2/26/06	Partial, Satisfactory
		3/14/06	Partial Satisfactory (see note 2)
0255-11-IIC-1	Emergency Service Water System Leakage Test - Loop A	10/5/05	Partial, Satisfactory (see note 3)
0255-11-IIC-3-1	EDG-ESW System Leakage Test Loop A Accessible Pressure Boundary Components	10/10/05	Satisfactory
0255-11-IIC-4-1	EDG-ESW System Leakage Test Loop B Accessible Pressure Boundary Components	10/24/05	Satisfactory

Note 1: Partial test, when combined with the partial test performed during Cycle 22 (3/20/2005), satisfies completion of 4th Interval, Period 1 requirements.

Note 2: Partial tests, when combined, satisfy completion of 4th Interval, Period 1 requirements.

Note 3: Partial test, when combined with the partial tests performed during Cycle 22 (3/19/05 and 3/20/05), satisfies completion of 4th Interval, Period 1 requirements.

System Pressure Test Examinations credited to Period 2

Test Number	Title	Date(s)	Results
0255-20-IIC-2	Reactor Coolant Boundary System Leakage Test	4/14/07	Satisfactory

APPENDIX C (continued)

Class 2 System Pressure Tests Per Code Case N-522 Credited to Period 2

Performed in accordance with 10CFR50, Appendix J Methodology

Procedure No.	Procedure Title	Test Date	Results
0137-04 (0255-10-IIB-1)	Primary Containment Purge and Vacuum Breaker, Pressure and Isolation Valve LLRT Test	4/11/07	SAT Sec XI Boundary
0137-05 (0255-10-IIB-2)	Primary Containment Nitrogen Control and Sampling System Valves Pressure and Local Leak Rate Test	3/31/07	SAT Sec XI Boundary
0137-18 (0255-17-IIB-1)	Instrument Air System Isolation Valves Pressure And Local Leak Rate Test	3/27/07	SAT Sec XI Boundary
0137-19 (0255-17-IIB-2)	Service Air System Isolation Valve Pressure and Local Leak Rate Test	3/22/07	SAT Sec XI Boundary
0137-22 (0255-26-IIB-1)	Demineralized Water Supply to Drywell Isolation Valve Pressure and Leak Rate Test	4/10/07	SAT Sec XI Boundary
0137-24 (0255-10-IIB-3)	Primary Containment Vent and Hard Pipe Vent Local Leak Rate Test	4/13/07	SAT Sec XI Boundary
0137-27 (0255-18-IIB-1)	TIP System Isolation Valves and Local Leak Rate Test	3/16/07	SAT Sec XI Boundary
0137-30 (0255-17-IIB-3)	Alternate N2 Supply Pressure and Local Leak Rate Test	3/24/07	SAT Sec XI Boundary
0137-31 (0255-13-IIB-1)	RBCCW System Isolation Valve Pressure and Local Leak Rate Test	3/26/07	SAT Sec XI Boundary
0138 (0255-10-IIB-4)	Drywell Personnel Airlock Pressure and Leak Test	4/15/07	SAT Sec XI Boundary

APPENDIX D

Snubber Functional Testing

The following snubbers were tested during Cycle 23 and the 2007 RFO to meet Technical Specifications, ASME Section XI requirements and verification testing as a result of replacement. There are 147 safety related snubbers at Monticello. The snubbers are grouped into two groups: One main group of 146 (Bergen-Paterson) and second group of 1 (Lisege – snubber SS-707). Fifteen snubbers from the main group and the single snubber from the second group resulted in 16 snubbers being tested per plant procedure 0251 during the 2007 RFO.

All snubbers were tested under one work order, 141385. Below is listed each individual work order that controlled removal, replacement if required and re-installation.

No.	Snubber Location	Size	Work Order No.	Snubber Serial No.	Test Results
1	SS-707	3	141349	700	Passed
2	SS-357	130	141350	400	Passed
3	CRD-H210	3	141351	538	Passed
4	SS-14	10	141353	500	Passed
5	SS-184	10	141354	040	Passed
6	SS-316	10	141355	035	Passed
7	SS-13	10	141356	171	Passed
8	SS-38B	10	141359	033	Passed
9	CRD-H227	20	141366	099	Passed
10	CRD-H232	20	141371	214	Passed
11	RV-25-NS2	30	141373	020	Passed
12	SS-2AR	30	141375	005	Passed
13	SS-3AR	30	141378	145	Passed
14	SS-20	10	141382	539	Passed
15	RV-25A-H2A	20	153475	478	Passed
16	RV-24-H5	20	153475	016	Passed

APPENDIX E

**Engineering Evaluations per Subsection IWE-3122.4
(1992 Edition with 1992 Addenda) and 10CFR50.55a(b)(2)(ix)(D)**

1. CAP CE 01082766-03 for degraded containment nuclear coatings
 - CAP CE 01082766-03 Attachment 1
 - CAP CE 01082766-03 Attachment 2
 - CAP CE 01082766-03 Attachment 3

(28 pages to follow)

CAP Number: 01082766

Condition Evaluation Number: 01082766-03

Description of problem:

Subject: Numerous small areas of PCT coatings failed acceptance Criteria

Assignment Description

Prepare a condition evaluation for the containment coatings that did not meet acceptance criteria.

Evaluation Details:

Description:

Numerous small areas of the Containment interior coatings did not meet the inspection procedure acceptance criteria. Comparison with previous performance of procedures 0135 and 0140 and IWE ISI exams shows expected deterioration from the degradation already discussed in previous CAPs (Reference 8).

This condition has occurred because of the following:

1. Torus environment of ~80 degrees F and 100% RH is harsh on the coating system.
2. Installation and removal of scaffolding over numerous outages have left the coating chipped in high traffic areas.
3. Lack of coating maintenance over the last several outages has lead to continue coating degradation.

The evaluation will address the following:

1. ASME code requirements.
2. Containment pressure boundary evaluation.
3. Containment debris loading and ECCS pump performance.
4. Future coating repair plans

Evaluation Details:

This Condition Evaluation (CE) was initiated to document the required engineering evaluation per ASME, Section XI IWE-3122.4 (1992 edition with 1992 Addenda), 10CFR50.55a(b)(2)(ix)(D), and to satisfy the requirements of Fleet procedure FP-PA-ARP-01, CAP Action Request Process. Procedure FP-PA-ARP-01, section 3.4 states:

“All personnel are responsible for identifying conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective material and equipment, and non conformances.”

Areas in containment were identified that did not meet the acceptance criteria of the inspection procedures. The inspection acceptance criteria is that the surface has no bare metal, little or no rusting on shell, and no flaking, peeling, blistering, cracking, or checking in excess of values established in CA-01-156.

Primary Containment (Drywell and Suppression chamber) internal coated surfaces were inspected by plant engineering staff, and ASME IWE inspectors during the 2007 refueling outage. These examinations included the coated surfaces of the drywell, the vent lines and vent header, downcomers and the entire suppression pool surfaces. The procedures used were, 0140 DRYWELL INTERIOR SURFACE INSPECTION, 0135 PRESSURE SUPPRESSION CHAMBER PAINTED SURFACE INTERNAL INSPECTION, 1367 PRESSURE-SUPPRESSION CHAMBER BELOW WATER LINE PAINTED SURFACE INTERNAL INSPECTION and 0515 PRIMARY CONTAINMENT VISUAL EXAMINATION FOR STRUCTURAL PROBLEMS. The observations made by these procedures include IWE inspection observations from work orders 135253 and 288456. These examinations satisfied ASME Table IWE-2500-1 examination categories E1.11, E1.12, E1.20 and E4.11.

Technical Specification compliance is not an issue associated with condition. MNGP Technical Specifications no longer contains the requirements of MNGP Custom Technical Specification 4.7.A.2.d and 4.7.A.1.c.

ASME Code Requirements

The ASME, Section XI Table IWE-2500-1 requires general visual and VT-3 visual examinations of coated and uncoated surfaces of the containment vessel. IWE-3510.2 VT-3 Visual Examinations of Coated areas states in part the following:

“The inspected area, when painted or coated, shall be examined for evidence of flaking, blistering, peeling, discoloration, and other signs of distress. Areas that are suspect shall be accepted by engineering evaluation or corrected by repair or replacement...”

The ASME, Section XI IWE-3122.4, Acceptance by Evaluation states in part the following:

“Components whose examination results reveal flaws or areas of degradation that do not meet the acceptance standards listed in Table IWE-3410-1 shall be acceptable for service without the removal or repair of the flaw or area of degradation or replacement if an engineering evaluation indicates that the flaw or area of degradation is nonstructural in nature or has no effect on the structural integrity of the containment...”

CONTAINMENT PRESSURE BOUNDARY EVALUATION

The following items provide facts concerning the containment pressure boundary:

- Ultrasonic testing performed by MDC in 1986 (MDC Document No. NSPT-12-4621) showed that the plate material used in the construction of the containment shell was thicker than that assumed in the design. Generally the plate thickness was at least 0.026" greater than that used in the CBI Certified Stress Report.
- Pacific Nuclear Calculation MN.9201.0104 determined that a 0.001" overall torus shell thickness reduction relative to design was acceptable for the pressure rating and Mark I hydrodynamic loads.
- Structural Integrity Associates justified localized shell reduction up to 0.312". When metallurgical boat samples of the torus shell were taken in 1991, cavities up to maximum depth of 0.312" were initially produced. These cavities were weld repaired but the structural integrity of the torus shell was evaluated without the cavities repaired. Structural Integrity Associates determined that the cavities could have been left in the torus in an unrepaired condition without violating required ASME Code margins.
- Calculation CA-97-004 indicates that for carbon steel in the containment atmosphere the corrosion rate would be 0.0006" per year.
- Automated Engineering Services Corp. determined the allowable pitting depth in the Torus shell and Submerged portions of the vent system downcomers, (Ref 9). The analysis determined the following as acceptable:

Description of Region	Thinning area diameter (inches)	Thinning depth (inches)
Torus upper clean shell region	12	0.21
Torus lower clean shell region	13	0.23
Torus upper shell region near discontinuities	15	0.07
Torus lower shell region near discontinuities	16	0.09
Downcomer clear region	1.25	0.17
Downcomer region near discontinuities	2.5	0.08

The containment pressure boundary including the suppression chamber, drywell, vent line and vent header and downcomers had General Visual examinations, VT-1 examinations, and VT-3 visual examinations during the 2007 refueling outage. These examinations are required by Table IWE-2500-1 Examinations Categories. The general visual examinations were performed to satisfy Item E1.11 (General Visual Examination shall be performed prior to each Type A test). The VT-3 visual examinations were performed to satisfy Items E1.12 and E1.20 (VT-3 examinations of coated and uncoated accessible surfaces of the containment vessel and vent system). Additionally, VT-1 examinations were performed as applicable to satisfy Item E4.11 for areas previously identified by IWE that were dispositioned use-as-is.

The observed areas of degradation (coatings and surface rust) are nonstructural in nature. No conditions that could affect the structural integrity were observed.

DRYWELL EVALUATION

The observations made by 0140 DRYWELL INTERIOR SURFACE INSPECTION are attached to this evaluation, Attachment 1. Coating degradation, consisting of rusting, flaking, discoloration and peeling were noted. In most areas of flaking, discoloration or peeling coatings the base coats were still intact.

Work order 0291852 was initiated to remove flaking and peeling coatings from the SRV discharge lines and to repair the degraded drywell, vent line and vent header coating. This work order removed the loose coatings from the SRV discharge lines. Repairs to the degraded drywell, vent line and vent header coating was not accomplished during the current 2007 refueling outage.

The internal surfaces of the drywell, vent lines, vent header, and downcomers had General Visual, VT-3, and VT-1 visual examinations performed on them as applicable. The observed areas of degradation (coatings and surface rust) are nonstructural in nature. No conditions that could affect the structural integrity were observed.

SUPPRESSION CHAMBER EVALUATION

The observations made by 0135 PRESSURE SUPPRESSION CHAMBER PAINTED SURFACE INTERNAL INSPECTION and 1367 PRESSURE-SUPPRESSION CHAMBER BELOW WATER LINE PAINTED SURFACE INTERNAL INSPECTION are attached to this evaluation, Attachments 2 & 3. Coating degradation, consisting of rusting, flaking, discoloration and peeling were noted. In most areas of flaking, discoloration or peeling coatings the base coats were still intact.

The suppression chamber, vent lines, and vent header assembly had VT-3 visual examinations performed on them that identified areas of degraded coatings. No conditions that could affect the structural integrity were observed.

Work order 0134819 to repair coatings in the torus was performed. All of the submerged portions of the torus shell and approximately 1/3 of the vapor region near the torus catwalk was restored. All of degraded coating in those areas was removed, prepared for coating, VT-3 visually examined prior to coating, then coated and cured per the manufacture's requirements. Large sections of coating discoloration were removed by scrubbing the area. This action removed the red-oxide stain from the intact coating system.

The remaining 2/3 of the suppression chambers vapor region, vent header and downcomers were left in their as-found state. The areas of degraded coatings that still remain were: light rust bleed through at the top of the torus shell interior; rust inside the downcomers from where the coating system was burnt off during downcomer support installation welding, and also slightly above the waterline where the coating is degraded; and several areas of flaking paint located near the top of the torus shell interior. The estimated total area of flaking paint remaining is estimated to be less than 50 square feet.

Ultrasonic shell thickness measurements (UT) were taken in accessible areas where rust has bled through the topcoat, and coating discoloration has occurred.

These ultrasonic measurements taken during the 2007 refueling outage are contained in Table 1.

Although essentially no evidence of pitting was identified in the areas where degraded coatings were removed and subsequently recoated, any UT measurements less than design values are well within the acceptance criteria for pits as shown above.

Table 1

Bay	Component	Design Thickness	UT Measurement
1	Upper Shell Plate	0.533"	0.579"
1	Lower Shell Plate	0.584"	0.618"/0.578"/ 0.618"
1	Downcomers	0.250"	0.250"/0.268"/ 0.254"/0.284"
2	Upper Shell Plate	0.533"	0.617"
2	Lower Shell Plate	0.584"	0.655"/0.593"/ 0.584"
2	Downcomers	0.250"	0.256"/0.251"/ 0.246"/0.277"
3	Upper Shell Plate	0.533"	0.574"
3	Lower Shell Plate	0.584"	0.633"/ 0.629"
3	Downcomers	0.250"	0.296"/0.256"/ 0.279"/0.278"
4	Upper Shell Plate	0.533"	0.610"
4	Lower Shell Plate	0.584"	0.631"/0.669"/ 0.628"
4	Downcomers	0.250"	0.340"/0.256"/ 0.248"/0.267"
5	Upper Shell Plate	0.533"	0.602"
5	Lower Shell Plate	0.584"	0.606"/0.576"
5	Downcomers	0.250"	0.285"/0.293"/ 0.312"/0.302"
6	Upper Shell Plate	0.533"	0.572"
6	Lower Shell Plate	0.584"	0.582"/0.632"/ 0.623"/0.592"
6	Downcomers	0.250"	0.290"/0.324"/ 0.301"/0.270"
7	Upper Shell Plate	0.533"	0.562"
7	Lower Shell Plate	0.584"	0.618"/0.596"/ 0.625"/0.640"
7	Downcomers	0.250"	0.298"/0.297"/ 0.346"/0.298"
8	Upper Shell Plate	0.533"	0.578"
8	Lower Shell Plate	0.584"	0.604"/0.606"/ 0.593"/0.619"
8	Downcomers	0.250"	0.287"/0.260"/ 0.265"/0.274"
9	Upper Shell Plate	0.533"	0.626"
9	Lower Shell Plate	0.584"	0.590"/ 0.607"
9	Downcomers	0.250"	0.258"/0.295"/ 0.292"/0.265"
10	Upper Shell Plate	0.533"	0.586"
10	Lower Shell Plate	0.584"	0.604"/ 0.675"
10	Downcomers	0.250"	0.261"/0.314"/ 0.257"/0.307"
11	Upper Shell Plate	0.533"	0.579"
11	Lower Shell Plate	0.584"	0.652"/ 0.681"

11	Downcomers	0.250"	0.298"/0.335"/ 0.271"/0.357"
12	Upper Shell Plate	0.533"	0.554"
12	Lower Shell Plate	0.584"	0.581"/0.611"/ 0.709"/0.646"
12	Downcomers	0.250"	0.298"/0.330"/ 0.308"/0.256"
13	Upper Shell Plate	0.533"	0.606"
13	Lower Shell Plate	0.584"	0.638"/0.600"/ 0.622"/0.605"
13	Downcomers	0.250"	0.279"/0.290"/ 0.248"/0.271"
14	Upper Shell Plate	0.533"	0.562"
14	Lower Shell Plate	0.584"	0.718"/0.620"/ 0.645"
14	Downcomers	0.250"	0.285"/0.294"/ 0.269"/0.273"
15	Upper Shell Plate	0.533"	0.574"
15	Lower Shell Plate	0.584"	0.630"/0.750"
15	Downcomers	0.250"	0.273"/0.295"/ 0.257"/0.271"
16	Upper Shell Plate	0.533"	0.589"
16	Lower Shell Plate	0.584"	0.631"/0.622"/ 0.601"/0.649"
16	Downcomers	0.250"	0.269"/0.288"/ 0.279"/0.245"

The internal surfaces of the Suppression chamber, external surfaces of the vent lines, vent header and downcomers had General Visual and VT-3 visual examinations performed on them. In addition, the visual examinations, ultrasonic (UT) measurements were taken. The UT measurement confirmed that no degradation of the pressure boundary was occurring. As identified above, localized areas on the torus shell could have significant thickness reduction and still meet code allowables. The small areas of bare metal on the torus shell were estimated to be less than 1/2- inch in diameter with most being very small nicks. Inspection of these bare metal areas showed only light rust film without evidence of corrosion pitting. The VT-3 inspections of the surfaced prepared areas for coating found several minor arc strikes and no areas of corrosion pitting. The arc strikes were further inspected by a mag-particle examination and were found to be acceptable with no indications. The observed areas of degradation (coatings) are nonstructural in nature. No conditions that could affect the structural integrity were observed.

Containment Debris Loading and ECCS Pump Performance Evaluation

Calculation CA-01-135 Evaluation of Effect of Paint Chips on Debris Loading of ECCS Suction Strainers assumes 1000 square feet of paint chip debris can collect on the ECCS suction strainers following an accident. Calculation CA-01-177 evaluated the impact of debris loading on the suction strainers with respect to post accident NPSH for the ECCS pumps and determined that the assumed debris loading was acceptable.

The results from the coated surfaces examinations of the drywell, the vent lines and vent header, downcomers and suppression pool surfaces are shown in Table 2.

Table 2

Inspection	As-Found Degraded coatings (ft ²)	As-Left Degraded coatings (ft ²)	Coating repaired (ft ²)
0135, PRESSURE SUPPRESSION CHAMBER PAINTED SURFACE INTERNAL INSPECTION (Note 2)	365.8	88.4	277.4
1367, PRESSURE-SUPPRESSION CHAMBER BELOW WATER LINE PAINTED SURFACE INTERNAL INSPECTION (Note 2)	453.76	213.49	240.27
0140, DRYWELL INTERIOR SURFACE INSPECTION (Note 2)	96	61	35
0515, PRIMARY CONTAINMENT VISUAL EXAMINATION FOR STRUCTURAL PROBLEMS	Note 1	Note 1	
Totals	915.56	362.89	552.67

Note 1: Degraded coatings were documented in procedures 0135, 0140 and 1367.

Note 2: The observations made by these procedures include IWE inspection observations from work orders 135253 and 288456.

The results show that the as-found degraded coating did not exceed the 1000 ft² of coating chips assumed in Calculation CA-01-135. Therefore, the as-found condition was within the calculation limits for debris loading on the ECCS suction strainers.

The results show that the as-left degraded coating did not exceed the 500 ft² administrative limit of degraded coating that could become paint chips. Therefore, the as-left condition is acceptable and will not challenge the debris loading on the ECCS suction strainers.

During the inspections, the coatings were classified as being "loose" if the coatings were flaking or peeling. Areas identified as light rust bleed through the top coat or discoloration was checked for adhesion and was found as tightly bonded as the non-degraded coatings. The other type of degraded coatings was the small areas where the coating had been chipped off and bare metal was exposed. Of these types of degraded coating only the "loose" coating are expected to add to the debris loading on the ECCS suction strainers. The amount of identified "loose" coating left inside primary containment is approximately 50 ft².

Therefore, since the as-left degraded coatings are a fraction of the amount of degraded coatings assumed to contribute to the ECCS suction strainer debris loading, the as-left condition is acceptable.

For the conditions identified that have not been repaired, they are acceptable for further service since the observed areas of degradation (coatings and surface rust) are nonstructural in nature and no conditions that could affect the structural integrity were observed.

Future Coating Repair Plans

Accessible areas of the drywell and suppression chamber will continue to be inspected each refueling outage. Degraded coatings will be identified, evaluated, and totaled to assure the amount of degraded coating remains within the established administrative limit and that there is no impact to the structural integrity of the containment.

Repairs to the drywell coatings should be scheduled and completed as needed to minimize the amount of degraded coatings inside the containment.

Containment coating inspections will be performed per procedures 0135, 140 and reexamination requirements of IWE-3122.4 during the 2009 RFO. It is predicted that approximately 50 to 60 ft² of additional degraded coats will be identified during those inspections.

For repairs to the torus, vent header and downcomers this would be at the next scheduled drain down of the torus. The next scheduled torus drain down for coating repairs should be performed when evaluations determine that the administrative limit of 500 ft² may be exceeded, or if degradation is observed that could impact structural integrity.

Conclusion

Based on results of the extensive scope of examinations performed on the various components of the Primary Containment and the engineer evaluation presented above, there is sufficient evidence that degraded areas of coatings is not leading to degradation of the base metal, is nonstructural in nature, and has no effect on the structural integrity of the containment. (IWE-3122.4) Therefore, it is acceptable for continued service.

The reported degraded coatings are non-structural in nature, and are not affecting the structural integrity of the underlying containment base metal. Therefore, there is no evidence to suggest possible degradation of the base metal in inaccessible areas. Additionally, with the exception of plant shutdown conditions where containment entry is necessary, the Drywell and Suppression Chamber interiors are maintained in an inerted atmosphere (nitrogen) which provides environmental conditions that are not conducive to oxidation of exposed base metal at areas of degraded coatings.

10CFR50.59 Screening SCR-01-0553 Rev. 3 was prepared for this Condition Report with a "Use-As-Is" disposition.

The examination conducted inspected 100% of the suppression chambers and nearly 100% of the drywell internal surfaces. No supplemental examinations are specified as a result of this engineering evaluation or 10CFR50.55a(b)(2)(ix)(D).

Actions:

Repairs of the suppression chamber and drywell coatings during the 2007 refueling outage were conducted under work orders 0134819 and 291852 respectively. No additional actions are required at this time.

REFERENCES:

1. Procedure 0135, Pressure-Suppression Chamber Painted Surface Internal Inspection, completed March 2007.
2. Procedure 0140, Drywell Internal Surface Inspection, completed March 2007.
3. Procedure 1367, Pressure-Suppression Chamber Below water line Painted Surface Internal Inspection, completed March 2007.
4. Procedure 0515, Primary Containment Visual Examination for Structural Problems, completed April 2007.
5. Work Order 00134819, REPAIR TORUS INTERNAL COATINGS.
6. Work Order 00291852, DRYWELL COATING REPAIR
7. Calculation –CA-01-156, ADD 0; IWE Recording Criteria. Approved 11/13/01.
8. CAPs 691537, 625896, 634151, 636491, and 819170
9. Calculation – AES calculation number MN02-P017471-100; Rev. 0 Determination of Torus Pitting Acceptance Criteria. Dated 04/25/03.
10. Calculation – AES calculation number MN02-P017471-200; Rev. 0, Pitting Acceptance Criteria for the Submerged Portions of the downcomers. Dated 04/25/03.
11. Work Orders 135253 and 288456 for Visual Examination of IWE Surfaces.
12. ASME, 1992 Edition, 1992 addenda, Section XI, IWE-3122.4
13. Torus Painting Procedure 8280, Revision 9.
14. Data Sheet for Phenoline 368WG paint.
15. MDC Document No. NSPT-12-4621, Monticello Plant Life Extension
16. Program - Ultrasonic wall Thickness Survey or the Drywell shell Test Results Package, July 1986.
17. CBI Certified Stress Report Monticello Project -II, Rev. 1, 3/7/69.
18. Pacific Nuclear Calculation MN.9201.0104, Calculate the Minimum Wall Thickness for the Monticello Torus for an Internal Pressure of 56 PSI and for Mark I Long Term Program Hydrodynamic Loadings, 7/6/92.
19. Letter from P. Riccardella (Structural Integrity Associates, Inc.) to B. Day (NSP), Torus Crack Indication - Final Evaluation and Recommendations, 5/16/91 (Appendix 2 to 6/17/91 letter to NRC).
20. NSP-74-103, MNGP Plant Unique Analysis Report (PUAR), Volume 3, Vent System, Revision 1, November 1982.

- 21. Calculation CA-97-004, Corrosion Allowances for Primary Containment Components.
- 22. Calculation CA-01-135, Evaluation of Effect of Paint Chips on Debris Loading of ECCS Suction Strainers.
- 23. Calculation CA-01-177, Determination of Containment Overpressure Required for Adequate NPSH of Low Pressure ECCS Pumps Updated for Suction Strainer Debris Loading.
- 24. FP-PA-ARP-01, CAP Action Request Process

Preparer's Name: Alan Wojchowski

Date: April 12, 2007

No human performance, process, organizational errors were identified. No training needs were identified.

Attachment 1



0140 Drywell Interior
Surface Inspection Re

Attachment 2



0135-COMMENTS-20
07 -AS LEFT-4-8-07.c

Attachment 3



1367-COMMENTS-wit
h downcomers -2007-

CE-01082766-03
Embedded Attachment 1

0140 Drywell Interior Surface Inspection Results

Area 1 - 920'-6"

SRV Tailpipe inside X-5A, flaking, 10 sq ft, recommend remove

X5A, floor support to left as you face X-5A, 6 sq in degraded, recommend remove

Shell above X-5B, Behind SRV Tailpipe, 2 sq ft degraded, recommend no rework

X-5B, vent tube, 2 sq in plus 2 sq in, marked, recommend no rework

X-5B, SRV tailpipe, 5 sq ft degraded, recommend remove

X-5B, floor support to right as you face X-5B, 4 sq in degraded, recommend remove

Two small bare spots between X5C and X5D, marked, recommend no rework

X-5 D brackets in vent tube, 6 sq in degraded, recommend remove

X-5D vent tube, 4 sq in degraded on bottom below SRV tailpipe penetration, marked, recommend remove.

X-5E vent tube, 2 sq in degraded on bottom below SRV tailpipe penetration, marked, recommend remove.

X-5E, SRV tailpipe inside vent tube, 20 sq ft degraded, recommend remove

X-5E, SRV tailpipe outside vent tube, 5 sq ft degraded, recommend remove

Floor support to left of X-5E as you face X-5E, 2 sq ft degraded, recommend remove.

X-5E, 9 o'clock, 4 sq in degraded, recommend remove

X-5E, X-5F, sheel behind duct, 6 sq in degraded, recommend no rework

HVAC Support near X-5F, 2 sq ft degraded, recommend remove

Floor support to right of X-5F as you face X-5F , 6 sq in degraded, recommend remove

Floor support to left of X-5F as you face X-5F , 4 sq in degraded, recommend remove

X-5F, SRV tailpipe, 10 sq ft degraded, recommend remove

X-5F, floor support to left as you face X-5F, 1 sq ft degraded, recommend remove

CE-01082766-03
Embedded Attachment 1

X-5F, bracket above SRV tailpipe, 2 sq in degraded, recommend no rework

SRV tailpipe, inside X-5G, 3 sq ft degraded, recommend remove.

SRV Tailpipe above X-5G, 10 sq ft degraded, recommend remove

X-5G, vent tube, several degraded areas marked, 6 sq in, recommend remove

Tailpipe inside X5H, 5 sq ft degraded, recommend remove

Shell above X-5H, Behind SRV Tailpipe, 2 sq ft degraded, recommend no rework

Shell near X-5H, between X-5H and X-5A, near concrete, 1 sq ft degraded, recommend repair

Support between X-5H and X-5A degraded 5 sq ft, recommend remove

X-5H, SRV tailpipe inside vent tube, 10 sq ft degraded, recommend remove

Area 5 - 984"

AZ 60, Refuel Cavity floor support, 6 sq in flaky, not marked

SRP cover plates recessed area, 3 sq in each flaky, 24 sq in total, not marked

AZ 180, about 989, small spots top coat missing, no corrosion, marked

AZ 193, under refuel cavity floor, small spot missing top coat, not marked

AZ 240, Refuel Cavity floor support, 1 sq ft flaky, not marked

AZ 347, neck area, small spot missing topcoat, no corrosion, not marked

AZ 167, about 992 elev, large spot and small spot missing top coat, no corrosion, marked

AZ 149, elev 990 and 989, small spots missing top coat, no corrosion, marked

SRP above N4B, small spot missing top coat, no corrosion, not marked

AZ 211, elev 985-990, shield, small area missing topcoat, no corrosion, not marked

AZ 119, shield, elev 988, small spot missing topcoat, no corrosion, marked

Area near X-25, 6 sq in flaky, not marked

AZ 85, elev 992, small area missing top coat, no corrosion, not marked.

COMMENTS

Area **Loose**
FT² **Y/N**

BAY 1:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	2	N
5	Ring girder at top of torus patches of rust bleed through	0.5	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
	Total degraded coatings:	2.6	
	Total loose coatings:	0	

Area **Loose**
FT² **Y/N**

BAY 2:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	1.5	N
5	Ring girder at top of torus patches of rust bleed through	0.5	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header has small patches of rust bleed through	2	N
9	Air temperature penetration heavily rusted	0.1	N
	Total degraded coatings:	4.2	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 3:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	10	N
5	Back wall center, patches of rust bleed through	3	N
6	Air temperature penetration heavily rusted.	0.1	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
	Total degraded coatings:	13.2	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 4:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	5	N
5	Top half of torus- peeling coating	1	Y
6	Torus manway cover and access many chipped spots	0.8	N
7	Monorail and it's supports has small patches of rust.	0.1	N
	Total degraded coatings:	6.9	
	Total loose coatings:	1	

Area Loose
FT² Y/N

BAY 5:

1			
2			
3			
4	Top of torus, and girders patches of rust bleed through.	5	N
5			
6	Vent header has small patches of rust bleed through	0.5	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
	Total degraded coatings:	5.6	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 6:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	1.5	N
5	Ring girders at top of torus patches of rust bleed through	0.5	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
9	Air temperature penetration, HPCI vent line & penetration heavily rusted	2	N
10.			
11			
	Total degraded coatings:	4.1	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 7:

1			
2			
3			
4	Top of torus, patches of rust bleed through around penetration.	6	N
5	Ring girders at top of torus patches of rust bleed through	1	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header, has small patches of rust bleed through	4	N
9	Air temperature penetration, HPCI vent line & penetration heavily rusted	2	N
	Total degraded coatings:	13.1	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 8:

1			
2			
3	Air temperature penetration heavily rusted.	0.2	N
4	Top of torus, patches of rust bleed through.	3	N
5	Ring girders at top of torus patches of rust bleed through	0.5	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header, at support has small patches of loose coating	1	Y
9	Air temperature penetration heavily rusted	0.2	N
10			
	Total degraded coatings:	5	
	Total loose coatings:	1	

Area Loose
FT² Y/N

BAY 9:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	1	N
5	Ring girders at top of torus patches of rust bleed through	0.5	N
6			
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header, at support has small patches of rust bleed through	0.2	N
	Total degraded coatings:	1.8	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 10:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	1	N
5	Ring girders at top of torus patches of rust bleed through	0.5	N
6	Vent header has small patches of rust bleed through	0.1	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header, at support has small patches of rust bleed through	0.2	N
9	Air temperature penetration heavily rusted.	0.2	N
	Total degraded coatings:	2.1	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 11:

1			
2			
3			
4	Top of torus, patches of rust bleed through.	1	N
5	Ring girders has patches of rust.	0.5	N
6	Vent header supports has small patches of rust bleed through	0.1	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8	Vent header, has small patches of rust bleed through	1	N
9	Air temperature penetration heavily rusted.	0.2	N
	Total degraded coatings:	2.9	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 12:

1			
2			
3			
4	Top of torus, patches of loose coatings.	2	Y
5	Top of torus, light rusting	1	N
6	Vent header has small patches of rust bleed through	0.1	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
9	Torus manway cover and access many chipped spots	0.8	N
10			
	Total degraded coatings:	4	
	Total loose coatings:	2	

Area Loose
FT² Y/N

BAY 13:

1			
2			
3			
4	Top of torus, patches of rust bleed through	3	N
5	Ring girders has patches of rust.	0.5	N
6	Vent header has small patches of rust bleed through	0.5	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
	Total degraded coatings:	4.1	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 14:

1			
2			
3	Top half of torus ring girdter has patches of rust bleed through	0.5	N
4	Top of torus, patches of rust bleed through	8	N
5	Torus spray line has patches of rust.	0.5	N
6	Vent header, has small patches of rust bleed through	0.2	N
7	Monorail and it's supports has small patches of rust.	0.1	N
	Total degraded coatings:	9.3	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 15:

1			
2			
3			
4	Top of torus, patches of rust bleed through	5	N
5	Ring girders has patches of rust.	1	N
6	Vent header, has small patches of rust bleed through	0.2	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
	Total degraded coatings:	6.3	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 16:

1			
2			
3			
4	Top of torus, patches of rust bleed through	2	N
5	Vent header gusset at top has patches of rust.	1	N
6	Vent header, has small patches of rust bleed through	0.2	N
7	Monorail and it's supports has small patches of rust.	0.1	N
8			
9			
10	Air temperature penetration heavily rusted.	0.2	N
	Total degraded coatings:	3.5	
	Total loose coatings:	0	

	TOTALS OF ALL BAYS	As-Left	
	Bay 1 through Bay 16 Total degraded coatings: (365.8 as-found)	88.4	
	Bay 1 through Bay 16 Total loose coatings: (14.0 as-found)	4.0	

Note: "loose" means peeling/flaking, not tightly adhering. "Not loose" means no peeling, flaking, blistering, cracking or checking, but does have bare metal or rust spots on shell.

Area Loose
FT² Y/N

BAY 1:

1			
2			
3			
4			
5			
6			
7	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² + 5-ft ²	15	N
8	Downcomer-OD- 6 spots of degraded coating- Totaling-1-ft ²	1	N
9	Downcomer-OD- 2 spots of degraded coating- Totaling-0.4-ft ²	0.4	Y
10			
	Total degraded coatings:	16.4	
	Total loose coatings:	0.4	

Area Loose
FT² Y/N

BAY 2:

1			
2			
3			
4			
5			
6			
7			
8			
9	Ring Girder - 16 small rust spots -1/2-inch square	0.02	Y
10	Downcomer-ID- 20 spots weld damaged coating- 6- inch ² + 4-ft ²	9	N
11	Downcomer-OD- 3 spots of degraded coating- Totaling-0.23-ft ²	0.23	N
12			
	Total degraded coatings:	9.25	
	Total loose coatings:	0.02	

Area Loose
FT² Y/N

BAY 3:

1			
2			
3			
4			
5			
6	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² + 5.5-ft ²	15.5	N
7	Downcomer-OD- 5 spots of degraded coating- Totaling-1-ft ²	0.23	N
8	Downcomer-OD- 1 spots of degraded coating- Totaling-0.06-ft ²	0.06	Y
9			
10			
	Total degraded coatings:	15.79	
	Total loose coatings:	0.06	

Area Loose
FT² Y/N

BAY 4:

1			
2			
3			
4	Downcomer-ID- 20 spots weld damaged coating- 6- inch ² + 2-ft ²	7	N
5	Downcomer-OD- 3 spots of degraded coating- Totaling- 0.13-ft ²	0.13	N
6			
7			
8			
9			
10			
	Total degraded coatings:	7.13	
	Total loose coatings:	0	

Area Loose
FT² Y/N

BAY 5:

1			
2			
3			
4			
5	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² + 12-ft ²	22	N
6	Downcomer-OD- 10 spots of degraded coating- Totaling-4.45-ft ²	4.45	N
7	Downcomer-OD- 3 spots of degraded coating- Totaling- 3.25-ft ²	3.25	Y
8			
9			
10			
	Total degraded coatings:	29.7	
	Total loose coatings:	3.25	

Area Loose
FT² Y/N

BAY 6:

1			
2			
3			
4			
5			
6			
7			
8			
9	Downcomer-ID- 20 spots weld damaged coating- 6- inch ²	5	N
10	Downcomer-OD- 3 spots of degraded coating- Totaling-2.17-ft ²	2.17	Y
	Total degraded coatings:	7.17	
	Total loose coatings:	2.17	

Area Loose
FT² Y/N

BAY 7:

1			
2			
3			
4			
5			
6			
7	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² +4-ft ²	10	N
8	Downcomer-OD- 7 spots of degraded coating- Totaling-2.34-ft ²	2.34	N
9	Downcomer-OD- 2 spots of degraded coating- Totaling- 7.02-ft ²	7.02	Y
10	Downcomer-ID- 5 spots of degraded coating-Totaling 1-ft ²	1	Y
	Total degraded coatings:	20.36	
	Total loose coatings:	8.02	

Area Loose
FT² Y/N

BAY 8:

1			
2			
3			
4			
5			
6			
7	Downcomer-ID- 20 spots weld damaged coating- 6- inch ² +2-ft ²	7	N
8	Downcomer-OD- 3 spots of degraded coating- Totaling-0.067-ft ²	0.07	N
9	Downcomer-OD- 4 spots of degraded coating- Totaling- 9.1-ft ²	9.1	Y
10	Downcomer-ID- 1 spots of degraded coating-Totaling 2-ft ²	2	Y
	Total degraded coatings:	18.17	
	Total loose coatings:	11.1	

Area Loose
FT² Y/N

BAY 9:

1			
2			
3			
4			
5			
6			
7			
8			
9			
10	Downcomer-ID- 40 spots weld damaged coating- 6- inch ²	10	N
11	Downcomer-OD- 3 spots of degraded coating- Totaling-0.095-ft ²	0.1	N
12	Downcomer-OD- 4 spots of degraded coating- Totaling- 0.187-ft ²	0.19	Y
	Total degraded coatings:	10.3	
	Total loose coatings:	0.19	

Area Loose
FT² Y/N

BAY 10:

1			
2			
3			
4			
5			
6			
7			
8			
9	Downcomer-ID- 20 spots weld damaged coating- 6- inch ² +0.5-ft ²	5.5	N
10	Downcomer-OD- 3 spots of degraded coating- Totaling-0.17-ft ²	0.17	N
11	Downcomer-OD- 3 spots of degraded coating- Totaling- 0.077-ft ²	0.08	Y
	Total degraded coatings:	5.75	
	Total loose coatings:	0.08	

Area Loose
FT² Y/N

BAY 11:

1			
2			
3			
4			
5			
6			
7			
8			
9			
10	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² +0.2-ft ²	10.2	N
11	Downcomer-OD- 6 spots of degraded coating- Totaling-0.83-ft ²	0.83	N
12	Downcomer-OD- 4 spots of degraded coating- Totaling- 3.06-ft ²	3.06	Y
	Total degraded coatings:	11.85	
	Total loose coatings:	3.06	

Area Loose
FT² Y/N

BAY 12:

1			
2			
3			
4			
5			
6			
7			
8			
9			
10	Downcomer-ID- 20 spots weld damaged coating- 6-inch ² +0.36-ft ²	5.36	N
11	Downcomer-OD- 3 spots of degraded coating- Totaling-0.292-ft ²	0.3	N
12	Downcomer-OD- 4 spots of degraded coating- Totaling- 10-ft ²	10	Y
	Total degraded coatings:	15.66	
	Total loose coatings:	10	

Area Loose
FT² Y/N

BAY 13:

1			
2			
3			
4			
5			
6			
7			
8			
9	Downcomer-ID- 40 spots weld damaged coating- 6- inch ²	10	N
10	Downcomer-OD- 8 spots of degraded coating- Totaling- 6.4-ft ²	0.1	N
11	Downcomer-OD- 3 spots of degraded coating- Totaling- 0.47-ft ²	0.47	Y
	Total degraded coatings:	10.57	
	Total loose coatings:	0.47	

Area Loose
FT² Y/N

BAY 14:

1			
2			
3			
4			
5			
6			
7			
8			
9			
10	Downcomer-ID- 20 spots weld damaged coating- 6- inch ²	5	N
11	Downcomer-OD- 4 spots of degraded coating- Totaling-0.165-ft ²	0.17	N
12	Downcomer-OD- 4 spots of degraded coating- Totaling- 0.567-ft ²	0.57	Y
13	Downcomer-ID- 1 spots of degraded coating-Totaling 3-ft ²	3	Y
	Total degraded coatings:	8.74	
	Total loose coatings:	3.57	

Area Loose
FT² Y/N

BAY 15:

1			
2			
3			
4			
5			
6			
7			
8	Downcomer-ID- 40 spots weld damaged coating- 6- inch ² +2-ft ²	12	N
9	Downcomer-OD- 7 spots of degraded coating- Totaling-4.6-ft ²	4.6	N
10	Downcomer-OD- 4 spots of degraded coating- Totaling- 3.15-ft ²	3.15	Y
	Total degraded coatings:	19.75	
	Total loose coatings:	3.15	

Area Loose
FT² Y/N

BAY 16:

1			
2			
3			
4			
5			
6			
7			
8			
9			
10	Downcomer-ID- 20 spots weld damaged coating- 6- inch ²	5	N
11	Downcomer-OD- 4 spots of degraded coating- Totaling-2.88-ft ²	2.9	N
	Total degraded coatings:	7.9	
	Total loose coatings:	0	

	TOTALS OF ALL BAYS for 1367		
	Bay 1 through Bay 16 Total degraded coatings:	213.49	
	Bay 1 through Bay 16 Total loose coatings:	45.54	

APPENDIX E (continued)

**Engineering Evaluations per Subsection IWE-3122.4
(1992 Edition with 1992 Addenda) and 10CFR50.55a(b)(2)(ix)**

2. CAP CE 01084149-01 for Arc Strikes on Torus Shell Plates

(2 pages to follow)

CAP Number: 01084149

Condition Evaluation Number: 01

Description of problem:

Arc strikes were noted on the torus shell during IWE examinations.

Evaluation Details:

ASME Section XI, Subsection IWE requires examination of containment surfaces. Arc strikes were noted on the interior of the torus shell during base metal examination associated with the torus coating project. These arc strikes occurred during construction of the torus. The arc strikes only became visible after all coating was removed. The arc strikes are superficial as evidenced by the visual examination results (little or no removal of metal).

Inspection of the containment is governed by IWE. 10CFR50.55a states that 50.55a(b)(2)(ix)(D) may be used in as an alternative to the requirements of IWE 2430. 50.55a(b)(2)(ix)(D) follows.

(1) If the examinations reveal flaws or areas of degradation exceeding the acceptance standards of Table IWE-3410-1, an evaluation must be performed to determine whether additional component examinations are required. For each flaw or area of degradation identified which exceeds acceptance standards, the licensee shall provide the following in the ISI Summary Report required by IWA-6000:

(i) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation;

(ii) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components, and;

(iii) A description of necessary corrective actions.

For the arc strikes noted during this examination, additional examinations are not required. The torus shell is a coated structure. Serious arc strikes would result in metal removal. Such metal removal would not meet the base metal profile requirements for coating and would have been rejected and reworked. Superficial arc strikes such as those noted during this examination would not interfere with base metal profile for coating and may not have been rejected and reworked. The code of fabrication (ASME III 1965, Winter 1965 and Section VIII 1965 Winter 1965) does not specifically address arc strikes. Arc strikes are required to be addressed by ASME Section XI, Subsection IWE. As noted above, 50.55a(b)(2)(ix)(D) allows evaluation of IWE indications by the alternate means detailed in 50.55a.

An additional concern with arc strikes is the effect of rapidly heating and cooling the metal. Such rapid heating and cooling may lead to cracks. Cracks could lead to failure of the pressure boundary.

Actions:

Visual and pitting examinations of the torus (IWE exams and coating prep exams) have confirmed the superficial nature of the arc strikes. The thickness of the torus shell is acceptable. The base metal profile for coating is acceptable. No further actions are required to address metal removal/profile for these superficial arc strikes.

All of the arc strikes were subjected to Section XI compliant magnetic particle examinations. The intent of these examinations was to confirm the visual examination results that no linear indications (i.e. cracks) were present in the area of the arc strikes. The magnetic particle examinations confirmed that there were no linear indications.

Additional examinations to detect arc strikes are not required.

Severe arc strikes (if any) would have been rejected during initial construction either for base metal profile concerns (e.g. undercut, thinning) or for coating profile concerns. Any superficial arc strikes remaining under the existing coating are acceptable based on the absence of linear indications from the magnetic particle examinations performed for this CAP. The torus has been subjected to (essentially) design level stress a number of times (during Type A tests and performance tests). If the superficial arc strikes had created cracks or removed metal or had created hard spots, these tests would have encouraged the growth of linear indications. The absence of any linears in the areas subjected to magnetic particle examinations and the acceptability of the base metal thickness and profile provides strong evidence that only superficial arc strikes occurred. Additional examinations would only reveal additional superficial arc strikes (if there are additional arc strikes). A large amount of work and dose would be required to reveal these additional (if any) superficial arc strikes.

No corrective actions are required for these superficial arc strikes. Visual and magnetic particle examinations have confirmed that there is no thinning/profile concern and that there are no linear indications (cracks). Since the initial construction codes did not require removal of arc strikes (that did not otherwise violate acceptance criteria), there is no code reconciliation or other document rework required to achieve code compliance.

A GAR has been created to verify that the evaluation results are reported per IWA-6000 (Reference AR 01084776). Inclusion of IWE results in the ISI summary report is unusual. The GAR will verify that this action occurs.

Preparer's Name: Jim Bridgeman

Date: 3/29/2007

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