

August 1, 2007

Mr. Michael A. Balduzzi
Sr. Vice President & COO
Regional Operations, NE
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT - REQUEST FOR
ADDITIONAL INFORMATION REGARDING REQUEST FOR RELIEF NO. RR-6
(TAC NO. MD4758)

Dear Mr. Balduzzi:

On February 27, 2007, Entergy Nuclear Operations, Inc. (Entergy), submitted an application for a proposed relief request for the James A. FitzPatrick Nuclear Power Plant which would use various Boiling Water Reactor Vessel and Internals Project guidelines as an alternative to certain requirements of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) for inservice inspection of reactor pressure vessel internal components.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). During a telephone call on July 24, 2007, the Entergy staff indicated that a response to the RAI would be provided within 30 days of the date of this letter.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure:
RAI

cc w/encl: See next page

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ADAMS ACCESSION NUMBER: ML072110014

*See memo dated July 5, 2007

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| DATE | 8/01/07 | 8/01/07 | 7/5/07 | 8/01/07 |

OFFICIAL RECORD COPY

FitzPatrick Nuclear Power Plant

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FitzPatrick Nuclear Power Plant

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REQUEST FOR ADDITIONAL INFORMATION

REGARDING RELIEF REQUEST RR-6

ENTERGY NUCLEAR OPERATIONS, INC.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

By letter dated February 27, 2007, Entergy Nuclear Operations Inc. (the licensee) submitted Relief Request RR-6 for the James A. FitzPatrick Nuclear Power Plant (JAFNPP). The licensee proposes to use Boiling Water Reactor Vessel and Internals Project (BWRVIP) guidelines for the inspection of reactor vessel internals (RVI) components as an alternative to certain requirements of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) for the fourth inservice inspection interval. The NRC staff has reviewed the information the licensee provided that supports the proposed relief request and would like to discuss the following issues to clarify the submittal:

1. Results of inspections that were conducted through refueling outage (RO)16 (year 2004) are recorded in pages 12 through 24 of the submittal. However, RO17 (year 2006) inspection results of the RVI components were not included in this submittal. The staff requests that the licensee provide this information so that it can perform an effective review of the relief request.
2. Editorial corrections:
 - (A) The staff requests that the licensee add attachment numbers 2, 3, 4, and 5 to pages 10, 12, 25, and 28, respectively.
 - (B) Section 5.0 of the relief request indicates that RO16 inspection results are recorded in Attachment 3 (page 12). However, the title on page 12 reads that RO17 inspection results are included in Attachment 3. The staff requests that the applicant make necessary corrections on page 12 of the submittal.
3. Section 4.1 item 5 of the BWRVIP-100-A report, "Updated Assessment of the Fracture Toughness of Irradiated Stainless Steel for BWR Core Shrouds," dated August 2006, states that fracture toughness values of stainless steel materials that are exposed to neutron fluence values greater than 1×10^{21} n/cm² (E > 1 MeV) are lower than those used in Appendix C of the BWRVIP-76 Report, "BWR Core Shroud Inspection and Flaw Evaluation Guidelines." Confirm that your planned inspections of the JAFNPP core shroud and/or core shroud repair hardware for the unit's fourth ISI interval are consistent with both BWRVIP-76 and BWRVIP-100-A.

Enclosure