



Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000
July 16, 2007

10 CFR 50.55a

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Stop: OWFN P1-35
Washington, D.C. 20555-0001

Gentlemen:

In the Matter of) Docket No. 50-260
Tennessee Valley Authority)

BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2 - AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) SECTION XI, INSERVICE INSPECTION, SYSTEM PRESSURE TEST, CONTAINMENT INSPECTION (IWE), AND REPAIR AND REPLACEMENT PROGRAMS - SUMMARY REPORTS (NIS-1 AND NIS-2) FOR CYCLE 14 OPERATION

In accordance with paragraphs IWA-6220, IWA-6230, and IWA-6240 of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1995 Edition through the 1996 addenda, TVA is submitting BFN Unit 2 outage summary reports for NRC review. The summary reports are for Inservice Inspection, System Pressure Test, and Containment Inspection (NIS-1 Report), and Repair and Replacement activities (NIS-2 Report) for Unit 2 Cycle 14 operation.

TVA has determined that certain BFN Unit 2 components had nondestructive examination (NDE) coverage limitations (less than 90 percent coverage completed) which exceed that specified in NRC Information Notice 98-42, "Implementation of 10 CFR 50.55a(g) Inservice Inspection Requirements."

A047

MRR

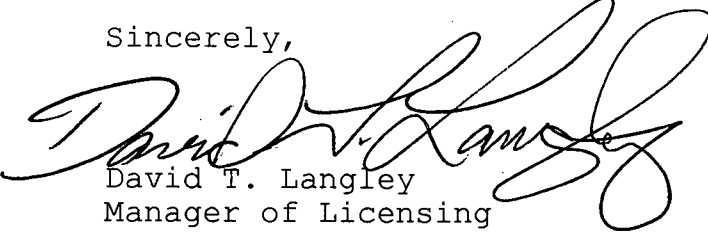
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Specifically, four Reactor Recirculation System piping welds, and nineteen reactor pressure vessel (RPV) nozzles received ultrasonic examination coverage of less than essentially 100 percent (i.e., less than 90 percent). Also, four Reactor Water Cleanup System piping welds received coverage less than essentially 100 percent (i.e., less than 90 percent). These examination limitations will be addressed by TVA in requests for relief and submitted to NRC for staff review and approval at a later date.

Enclosure 1 of this letter contains the BFN Unit 2 Inservice Inspection, System Pressure Test, and Containment Inspection Summary Report (NIS-1) for Code Class 1 and 2 pressure retaining components and their supports. Enclosure 2 contains the Repair and Replacement Summary Report (NIS-2) for Code Class 1 and 2 components and supports.

There are no new regulatory commitments in this letter. If you have any questions regarding these reports, please contact me at (256) 729-2636.

Sincerely,



David T. Langley
Manager of Licensing
and Industry Affairs

Enclosures
cc (see page 3)

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cc (Enclosures):

Mr. James H. Moorman, III, Branch Chief
U.S. Nuclear Regulatory Commission
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NRC Senior Resident Inspector
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Rockville, Maryland 20852-2739

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DTL:JWD:BAB

Enclosures.

cc (w/o Enclosures):

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EDMS WT CA-K (w/Enclosures)

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ENCLOSURE 1

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNIT 2

AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME),
SECTION XI, THIRD TEN-YEAR INSPECTION INTERVAL

INSERVICE INSPECTION (ISI), SYSTEM PRESSURE TEST (SPT),
CONTAINMENT INSPECTION, AND AUGMENTED EXAMINATIONS PROGRAM

SUMMARY REPORT (NIS-1) FOR CYCLE 14 OPERATION

(SEE ATTACHED)

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

UNIT 2 CYCLE 14

“OWNER’S REPORT FOR INSERVICE INSPECTION”

TABLE OF CONTENTS

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
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1101 MARKET STREET DECATUR, ALABAMA 35609-2000
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NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX I

NIS-1 OWNER'S REPORT

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35609-2000
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Inservice Inspection Introduction Summary

In accordance with paragraph IWA-6220 of 1995 Edition, 1996 Addenda of Section XI of the ASME Boiler and Pressure Vessel Code the following information is provided.

1. Date of document completion: June 05, 2007

2. Name of owner and address of principal offices:

Tennessee Valley Authority
Office Of Nuclear Power
1101 Market Street
Chattanooga, Tennessee 37402-2801

3. Name and address of the nuclear generating plant:

Browns Ferry Nuclear Plant
P.O. Box 2000
Decatur, Alabama 35602

4. Name or number assigned to the nuclear power unit by TVA:

Browns Ferry Nuclear Plant, Unit 2.

5. Commercial operation date of unit:

March 1, 1975

6. Numbers assigned to the components by the state:

No numbers assigned

7. National Board Number assigned to the components by the manufacturer:

No numbers assigned

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8. Names of the components and descriptions including size, capacity, material, location, and drawings to aid identification.

The Class 1 and 2 components examined as part of this Inservice inspection are listed in Appendix V.

9. Name and address of principal manufacturer and the principal contract which will identify the subcontractors/manufacturer's component identification numbers.

The majority of components examined were supplied by:

General Electric Corporation
San Jose, Ca.
Contract Number: 66C31-90744

10. Date of completion of the examinations:
March 25, 2007

11. Name of ANII who witnessed or otherwise verified the examinations and his employer and business address:

Samuel Flood and Bruce Eamigh
Hartford Steam Boiler of Connecticut
200 Ashford Center North, Suite 300
Atlanta, Georgia 30338

12. Abstract of examinations, conditions observed, and corrective measures recommended or taken:

See Appendix III, IV, V and VI

13. Signature of ANII:

See NIS-1 Form.

FORM NIS-1 OWNERS' REPORT FOR INSERVICE INSPECTIONS

As required by the Provisions of the ASME Code Rules

1. Owner Tennessee Valley Authority, 1101 Market St. Chattanooga, TN. 37402
(Name and Address of Owner)
2. Plant Browns Ferry Nuclear Plant, P.O. Box 2000 Decatur, AL. 35609-2000
(Name and Address of Plant)
3. Plant Unit 2
4. Owner Certificate of Authorization Not Required
5. Commercial Service Date 03/01/1975
6. National Board Number for Unit Not Required
7. Components Inspected:

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor Pressure Vessel	General Electric	Contract No. 67C21-90744	N/A	N/A
Various systems and components. (See Appendix V)	TVA	N/A	N/A	N/A
<u>The NIS-1 Owners Report for Inservice Inspections includes Appendices I, II, III, IV, V, and VI.</u>				

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8.5 in. X 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-1 (back)

- 8. Examination Dates 08/31/2005 to 04/11/2007
- 9. Inspection Period Identification: Second Period, 05/24/2004 to 05/25/2008
- 10. Inspection Interval Identification: 05/25/2001 to 05/24/2011
- 11. Applicable Edition of Section XI 1995 Edition through 1996 Addenda
- 12. Date/Revision of Inspection Plan: 2-SI-4.6.G Revision, 032
- 13. Abstract of Examinations and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan.

See Appendix II, III, IV, V, VI.
- 14. Abstract of Results of Examinations and Tests.

See Appendix II, III, IV, V, VI.
- 15. Abstract of Corrective Measures. See Appendix VI

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. Not Applicable Expiration Date Not Applicable

Date 7-3, 2007 Signed Tennessee Valley Authority By [Signature]
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, The undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB-CT of Hartford, CT. have inspected the components described in this Owners' Report during the period 8/31/05 to 4/11/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owners' Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations tests, and corrective measures described in this Owners' Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province and No.

Date 7/3/2007

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
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COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX II

SCOPE AND INTRODUCTION

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35609-2000
UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.	
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NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.	

Scope:

The scope of this appendix is to provide an overview of the Inservice inspections performed during the Unit 2/Cycle 14 Outage on Class 1 and 2 components for ASME Section XI Code credit and other augmented examinations.

Introduction:

The examinations were performed in accordance with implementing plant surveillance instruction 2-SI-4.6.G "Inservice Inspection and Risk Informed Inservice Inspection Program Unit 2". 2-SI-4.6.G is organized to comply with the ISI NDE requirements of the 1995 Edition, 1996 Edition of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Division 1, Articles IWX-1000, IWX-2000, IWX-3000, and IWX-6000 in accordance with Title 10 Code of Federal Regulations (CFR) Part 50; 50.55a (g); to implement the Browns Ferry Nuclear Plant (BFN) Technical Requirements TR-3.4.3; and to fulfill the requirements of SPP-9.1, ASME Section XI Inservice Inspection Program.

Beginning in the third Period of the second Interval, Surveillance Instruction 2-SI-4.6.G implemented the NRC approved BFN Risk-Informed Inservice Inspection Program to address all piping locations that are subject to service induced degradation. In accordance with Regulatory Guides 1.174 and 1.178 and Code Case N-577, this program provides an acceptable alternative approach to the existing ASME Section XI requirements for scope and frequency of piping weld examinations, and satisfies the criteria of 10CFR50.55a(a)(3)(i) providing an acceptable level of quality and safety.

2-SI-4.6.G reflects the built-in limitations of the original plant design, geometry, construction, component materials and the current technology or state-of-the-art nondestructive examination techniques. The SI specifies the methods to be used and provides schedule tables from which specific items were scheduled for examination during the outage. Examinations were witnessed or verified by an Authorized Nuclear Inservice Inspector (ANII) and performed in accordance with the Section XI of the ASME Boiler and Pressure Vessel Code.

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The majority of examinations were performed by the TVA Inspection Services Organization (ISO). Augmentation of personnel was provided by Washington Group, Inc., and AREVA (Framatome) ANP.

An overview of ISI activities consists of the following:

- . ASME Section XI Class 1 and 2 Piping Examinations
- . ASME Section XI Class 1 Reactor Pressure Vessel Weld Examinations
- . ASME Section XI Class 1 and 2 Support Examinations
- . Reactor Pressure Vessel In-Vessel Visual Inspection Examinations (RPVII)
- . Augmented Examinations

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APPENDIX III

ISI SUMMARY

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Examination Summary:

The Unit 2, Cycle 14 outage Inservice Inspection (ISI) examinations were performed during the second scheduled refueling outage during the second inspection period of the third ASME Section XI 10-year inspection interval. Approximately 184 (one-hundred and eighty-four) visuals, 10 (ten) of which were RPV Nozzle Inner Radius Sections Code Category B-D, Item No. B3.100, performed from vessel ID, (reference RFR # 2-ISI-16 and 2-ISI-17), 249 (two-hundred and forty-nine) ultrasonic, 10 (ten) of which were expanded scope for RPV nozzle to vessel welds, 2 (two) for piping welds and 9 (nine) magnetic particle examinations were performed in support of code credit components. Also, preservice examinations were performed; 53 (fifty-three) visual, 7 (seven) ultrasonic, 2 (two) magnetic particle, and 5 (five) liquid penetrant examinations.

Three (3) Notification of Indications (NOI's) were issued for Inservice Inspection (ISI), to document indications identified during the performance of the examinations. These NOI's were evaluated by engineering and dispositioned (see Appendix VI, Summary of Indications).

Other examinations were performed in accordance with BFN's augmented inspection program and are included in Attachment 1 for information. Approximately 454 (four-hundred and fifty-four) visual, 29 (twenty-nine) ultrasonic, of which 20 (twenty) were credited for BWRVIP-75A/IGSCC, 9 (nine) for RPV Core Shroud welds, and 45 (forty-five) radiography examinations were performed (excluding valve RT) in accordance with the augmented programs. These totals are inclusive of the Reactor Pressure Vessel Internals Inspection (RPVII) Augmented examination program on Unit 2 RPV internals. Reference RPVII Procedure 0-TI-365.

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ASME Code Cases

The following code cases have been approved for use as applicable during the Unit 2/Cycle 14 outage:

- N-323-1 Alternate Examination For Welded Attachments To Pressure Vessels, Section XI, Division 1.
- N-498-4 Alternate Rules for 10-Year System Hydrostatic Testing for Class 1, 2 and 3 Systems Section XI Division 1
- N-526 Alternate Requirements for Successive Inspections of Class 1 and 2 Vessels, Section XI, Division 1.
- N-552 Alternate Methods - Qualification For Nozzle Inside Radius Section From the Outside Surface section XI, Division 1.

To achieve consistency with the 10 CFR 50.55a rule change published September 22, 1999 (64 FR 51370), incorporating Appendix VIII, "Performance Demonstration for Ultrasonic Examination Systems," to Section XI, add the following to the specimen requirements:

"At least 50 percent of the flaws in the demonstration test set must be cracks and the maximum misorientation angles must be demonstrated with cracks. Flaws in nozzles with bore diameters equal to or less than 4 inches may be notches."

The number of false calls must not exceed three.

- N-577 Risk-Informed Requirements for Class 1, 2, and 3 Piping, Method A, , Section XI, Division 1, (RIMS # R08 000601 846), with the more detailed provisions provided in WCAP-14572, Revision 1-NPA, "Westinghouse Owners Group Application Of Risk - Informed Methods To Piping Inservice Inspection Topical Report." Reference Safety Evaluation Report (SER) from NRC Date January 19, 2001.

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ASME Code Cases (cont'd)

N-586 Alternate Additional Examination Requirements for Class 1, 2, and 3 Piping, Components, and Supports, Section XI, Division 1.

The engineering evaluations addressed under item (a) and the additional examinations addressed under Item (b) shall be performed during this outage.

If the additional examinations performed under (b) reveal indications exceeding the applicable acceptance criteria of Section XI, the engineering evaluations and the examinations shall be further extended to include additional evaluations and examinations at this outage.

N-598 Alternate requirements to Required Percentages of Examinations Section XI, Division 1.

N-623 Deferral of Inspections of Shell-to-Flange and Head-to-Flange Welds of a Reactor Vessel Section XI, Division 1.

N-624 Successive Inspections, Section XI, Division 1.

N-652 Alternate Requirements to Categories B-G-1, B-G-2, and C-D Bolting Examination and Selection Criteria Section XI, Division 1.

N-658 Qualification Requirements for Ultrasonic Examination of Wrought Austenitic Piping Welds Section XI, Division 1.

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UNIT 2 INTERVAL STATUS

The BFN Unit 2 Cycle 14 outage ISI examinations were performed during the second scheduled refueling outage of the second period of the third interval. The component quantities examined were determined from 2-SI-4.6.G, Section 8.1 (Parts 1, 2, 4, 6, and 7), Unit 2 Class 1 and 2 components, and from applicable BFN Unit 2 relief requests. This Owner's Report for Inservice Inspection, covers the Cycle 14 Outage for Browns Ferry Unit 2. The following table summarizes the percentage of Code required examinations completed to date.

Table 1 summarizes code credited examinations by category, percentages completed, and complies with ASME Section XI percentage requirements.

**TABLE 1
ASME SECTION XI EXAMINATION SUMMARY FOR THE SECOND
PERIOD OF THE THIRD TEN-YEAR INSPECTION INTERVAL**

<u>CATEGORY/CLASS</u>	<u>% COMPLETE</u>	<u>COMMENTS</u>
B-A/1	17%	Reference RFR # 2-ISI-9 (Category B-A, Item No. B1.11)
B-B	N/A	
B-D/1	74%	Reference RFR # 2-ISI-16 and 2-ISI-17, approved by NRC
B-F/1	N/A	Risk-Informed ISI implemented in the second interval
B-G-1/1	70%	Reference Code Case N-652
B-G-2/1	80%	Item No. B7.50 Pipe Bolting, Reference Code Case N-652
B-G-2/1	44%	Item No. B7.70 Valve Bolting, inspect when the valve is disassembled
B-J/1	N/A	Risk-Informed ISI implemented in the second interval
B-K/1	62%	
B-L-1	N/A	
B-L-2/1	100%	Unit 2 RECIRC Pumps A & B changed out in Cycle 14, 2nd period
B-M-1	N/A	
B-M-2/1	33%	When disassembled
B-N-1/1	66%	Each period
B-N-2/1	70%	Deferral permissible
B-O	N/A	Make-up exclusion reference 2-SI-4.6.G, Paragraph 7.1.3.G
B-P	N/A	Refer to pressure test program
C-A/2	50%	
C-B/2	60%	
C-C/2	56%	
C-D	N/A	
C-F-1/2	N/A	Risk-Informed ISI implemented in the second interval
C-F-2/2	N/A	Risk-Informed ISI implemented in the second interval
C-G	N/A	
F-A/ 1 and 2	66%	ASME Code Class 1 and 2 Component Supports only
R-A / 1 and 2	59%	Risk-Informed ISI implemented in the second period second interval.

This percentage does not include Flow Accelerated Corrosion (FAC) Item # R1.18. Reference BFPER 99-008578-000, 02-005018-000, 02-004920-000.

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APPENDIX IV
EXAMINATION LIMITATIONS

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
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METHOD OF CALCULATION OF LIMITATIONS

During the performance of Inservice Inspections, the ASME Section XI Code 1995 Edition, 1996 Addenda, requires the determination of the ultrasonic examination volume to establish the required beam path angles needed to maximize coverage and verify technique parameters. This information is necessary in those instances where there may be a reduction in the examination volume.

Surface examinations are typically conducted on 100% of the weld length plus a defined amount of base material on each side of the weld. Surface areas are calculated in those instances where there may be a reduction in the examination area.

The Code required ultrasonic examination volume or surface examination area for each type of piping weld or nozzle-to-vessel weld is depicted in the figures of IWB-2500 or IWC-2500. As depicted for piping welds, volume width generally constitutes the weld plus 1/4" on each side while volume thickness generally constitutes the lower 1/3 of the piping thickness for the length of the weld. As depicted, for nozzle-to-vessel welds, the volume width generally constitutes the weld plus 1/2t (ts/2) on each side of the weld while volume thickness generally constitutes the entire component thickness (i.e. full volume). The volume changes with variations in weld configuration (e.g. transition between different pipe thickness or nozzle-to-vessel configuration). Therefore, it is necessary to determine the required volume for each group of similar welds to allow setting of scanner limits for automated ultrasonic examinations and scan paths for manual ultrasonic examinations. Surface examination area is generally the weld plus 1/2-inch of base material on each side of the weld.

Reactor Pressure Vessel Nozzle to shell or head weld examination volume has been reduced to 1/2" beyond the widest part of the boundary of the deposited weld material in lieu of the requirements of ASME Section XI Figures IWB-2500-7 (a) and IWB-2500-7 (b) per Request For Relief No. PDI-2.

Paragraph IWA-2232 of the Code requires that the ultrasonic examination of piping systems be conducted in accordance with Appendix I of ASME Section XI. Appendix I requires that the ultrasonic examination of piping systems be conducted in accordance with Appendix VIII of ASME Section XI, and the nozzle-to-vessel welds be conducted in accordance with Article 4 of ASME Section V, 1995 Edition, 1996 Addenda as supplemented by Appendix I of ASME Section XI. Appendix VIII and Article 4 define the applicable examination methods (e.g., examination angles, scan directions) to be used during examination.

TVA developed procedures N-GP-28 and N-GP-31 to provide a standardized methodology for calculation of Code coverage in those instances where configuration or other components cause an examination limitation. Components/welds with limitations were evaluated in terms of the feasibility of other NDE techniques or methods to increase coverage.

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EXAMINATION LIMITATIONS: (CONTINUED)

The following items/components had examination limitations outside those specified in 1995 Edition, 1996 Addenda of ASME Section XI Code and NRC Information Notice, 98-42 "Implementation Of 10 CFR 50.55a(g) Inservice Inspection Requirements." The Inservice Inspection Program 2-SI-4.6.G will be revised to incorporate these limitations in the form of Requests for Relief (RFR). Program revisions, including Requests for Relief, will be submitted to the NRC. Reference RFR# 2-ISI-19, Revision 01.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>COVERAGE CALCULATED</u>	<u>REPORT NO.</u>	<u>RFR No.</u>
RPV	N1B-NV	31%	R-137	2-ISI-19
RPV	N2A-NV	44%	R-138	2-ISI-19
RPV	N2B-NV	44%	R-142	2-ISI-19
RPV	N2C-NV	44%	R-139	2-ISI-19
RPV	N2D-NV	44%	R-173	2-ISI-19
RPV	N2G-NV	44%	R-136	2-ISI-19
RPV	N2H-NV	50%	R-143	2-ISI-19
RPV	N2K-NV	44%	R-174	2-ISI-19
RPV	N3A-NV	41%	R-151	2-ISI-19
RPV	N3B-NV	41%	R-175	2-ISI-19
RPV	N3C-NV	41%	R-152	2-ISI-19
RPV	N4B-NV	44%	R-176	2-ISI-19
RPV	N4C-NV	44%	R-177	2-ISI-19
RPV	N4E-NV	44%	R-178	2-ISI-19
RPV	N4F-NV	44%	R-179	2-ISI-19
RPV	N5A-NV	27%	R-156	2-ISI-19
RPV	N5B-NV	27%	R-157	2-ISI-19
RPV	N7-NV	69%	R-080	2-ISI-19
RPV	N9-NV	40%	R-117	2-ISI-19

The following BFN Unit 2 Reactor Pressure Vessel Inner Radius Sections, Code Category, B-D; Item No. B3.100, received an Enhanced Remote Visual (EVT-1) examination, capable of a 1-mil wire resolution in accordance with ASME Section XI, VT-1 requirements. This was in lieu of a volumetric examination required in accordance with the 1995 Edition, 1996 Addenda of ASME Section XI Code.

This was in accordance with Request For Relief # 2-ISI-17 for the Reactor Pressure Vessel Nozzles, N2A, N2B, N2C, N2D, N2E, N2F, N2G, N2H, N2J, N2K, N5A, N5B, N8A, and N8B, approved by the NRC on October 07, 2001, (TAC NO. MB4880). TVA provided the NRC specific limitations and estimated coverage's for each nozzle in the Request For Relief # 2-ISI-17. TVA is reporting the actual coverage's obtained during the Enhanced Remote Visual (EVT-1) examination, capable of a 1-mil wire resolution in this report below. Reference BFN RFR# 2-ISI-16 and 2-ISI-17.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>ESIMATED COVERAGE</u>	<u>ACTUAL COVERAGE</u>	<u>REPORT NO.</u>
RPV	N1B-IR	100%	80%	R-181
RPV	N2C-IR	50%	40%	R-181
RPV	N2G-IR	50%	40%	R-181
RPV	N2H-IR	50%	40%	R-181
RPV	N5A-IR	40%	40%	R-181
RPV	N5B-NV	40%	40%	R-181

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX V

EXAMINATION PLAN

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

The following printout is an outage ISI report designed to meet the reporting requirements of IWA-6000 of the ASME Section XI Code. This report contains Unit 2/Cycle 14 Inservice Inspection data for Code Class 1 and Code Class 2 components selected for ASME Section XI credit. Essential unit and system files are contained herein as a reference to describe abbreviations and features in the printout. The aforementioned precedes the outage ISI report.

Code Class 3 Inservice data and reports are contained in the Browns Ferry Inservice Inspection (ISI) Final Plant Report.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

J. Calibration Standard- If required

K. Exam Date- Date of Inspection

L. Exam Report No.- Examination Report Number

M. Exam Results - P - Pass

R- Rejectable

E- Evaluated acceptable for continued operation by Engineering

O. COMMENTS

NOTE (1): EXREQ Identifiers:

96E-03 - ASME Section XI Code 1995 Edition, 1996 Addenda/ Third Interval

P95-96 - Preservice Examination Cycle 12/ Third Interval

A14-03 - 1st additional sample per IWB-2430 Code Category R-A

A15-03 - 1st additional sample per IWB-2430 Code Category B-D

B01-02 - Feedwater Sparger Visual (VT-1) Examinations to O-TI-365

B02-02 - Examinations performed to BWRVIP-75 for IGSCC detection

B04-02 - Weld inspection for Pipe Whip Protection

B07-02 - Examinations performed to the recommendations of BWRVIP-27 and BWRVIP-49

B12-02 - Augmented examination expanded scope per BWRVIP-75A

S01-03 - Successive Examinations

V01-02 - Voluntary Examinations

OTI365 - Augmented examinations of RPV Internals

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

UNIT 2/CYCLE 14
ISI REPORT OF CLASS 1 AND CLASS 2
COMPONENTS

"ORIGINAL COPY"

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	RECIRC	PMP-B-STUD-Z-01 THROUGH PMP-B-STUD-Z-16	Y	ADDED CALIBRATION STANDARD BF-119.	CALIBRATION STANDARD NOT LISTED IN DATA BASE	W. W. W. W. 2/9/07	W. W. W. W. 3/5/07	F.W. Frazzelle 2/8/07	W. W. W. W. 3/5/07
2/14	001	RECIRC	RBC-2-1	Y	ADDED EXAM EXSCND-VT1 NDEPROC - N-VT-1 CAT-BG2 EXREQ-96E-03	W/O# 06-726736-000	W. W. W. W. 2/9/07	W. W. W. W. 3/5/07	F.W. Frazzelle 2/8/07	W. W. W. W. 3/5/07
2/14	001	HPCIS	2-47B455S0010-IA	Y	DELETE EXAM - MT ON IA	IA INACCESSABLE FOR MT EXAM.	W. W. W. W. 2/9/07	W. W. W. W. 3/5/07	F.W. Frazzelle 2/9/07	W. W. W. W. 3/5/07
2/14	001	RHRS	2-47B452R0064-IA	Y	DELETE MTEXAM ON IA CAT-C-C, C3.20	EXAM IN LOCKED HIGH RAD AREA (SUBSTITUTED FOR 3/1/07 2-47B452H0069-IA 2-47B452R0064-IA	W. W. W. W. 3/1/07	W. W. W. W. 3/1/07	F.W. Frazzelle 3/1/07	W. W. W. W. 3/1/07
2/14	001	RHRS	2-47B452H0069-IA	Y	ADD MT EXAM ON IA Cat C-C, C3.20	REPLACED MT EXAM ON 2-47B452R0064-IA 2-47B452H0069-IA 3/1/07	W. W. W. W. 3/1/07	W. W. W. W. 3/1/07	F.W. Frazzelle 3/1/07	W. W. W. W. 3/1/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	MSS	Valve 2-FCV-01-015 B-M-2 B12.50	Y	Add Value 2-FCV-01-015 VT-3 B-M-2 B12.50	Valve Disassembled	Steve Wilkens 3/1/07	Walter Wilkens 3/1/07	FW Fraschetti 3/1/07	Walter Wilkens 3/1/07
2/14	001	MSS	2-FCV-01-015-BC B-G-2 B7.70	Y	Add Value Bolted Connection 2-FCV-01-015-BC VT-1 B-G-2 B7.70	Valve Disassembled	Steve Wilkens 3/1/07	Walter Wilkens 3/1/07	FW Fraschetti 3/1/07	Walter Wilkens 3/1/07
2/14	001	MSS	2-47B40050038 2-47B40050039 2-47B40050040 2-47B40050041	Y	ADD THE FOLLOWING SUPPORTS 2-47B40050038 CAT. F-A, ITEM 1.10A* 2-47B40050039 CAT. F-A, ITEM 1.10A* 2-47B40050040 CAT. F-A, ITEM 1.10A* 2-47B40050041 CAT. F-B PAG-96 F-A, ITEM 1.10B*	DISASSEMBLED FOR MSIV EPU WORK. ADDED NEW BOLTING. PSI EXAM REF: W/O# 06-711366-018 *Items F.10A & F.10B	Steve Wilkens 3/8/07	Walter Wilkens 3/6/07	FW Fraschetti 3/6/07	Walter Wilkens 3/6/07
2/14	001	CSS	Valve 2-FCV-075-26	Y	Add Value 2-FCV-075-26 VT-3 B-M-2 B12.50	Valve Disassembled	Steve Wilkens 3/8/07	Walter Wilkens 3/8/07	FW Fraschetti 3/8/07	Walter Wilkens 3/8/07
2/14	001	CSS	Valve 2-FCV-075-26- BC	Y	Add Value Bolting 2-FCV-075-26-BC B-G-2, B7.70 VT-1	Valve Disassembled	Steve Wilkens 3/8/07	Walter Wilkens 3/8/07	FW Fraschetti 3/8/07	Walter Wilkens 3/8/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	RHRS	DSRHR-2-04	Y	2/11/07 Remove DSRHR-2-04 UT EXREQ TS3432, N B-J, B04-02.	TSR3432 exam has been eliminated by Engineering	Blue Willet 3/12/07	Wans Willet 3/11/07	J.W. Troncello 3/11/07	Wans Willet 3/11/07
2/14	074	RHRS	DRHR-2-11	N	Add WELD DRHR-2-11 UT exam Cat. B12-02 IT.# NU0313 Cat. D	EXPANDED Scope EXREQ: B12-02	Blue Willet 3/12/07	Wans Willet 3/11/07	J.W. Troncello 3/11/07	Wans Willet 3/11/07
2/14	085	RPV/ CRDS	RCRD-2-33	N	Add Weld RCRD-2-33 UT exam. Cat. B12-02 IT.# NU0313, Cat. D	EXPANDED Scope EXREQ: B12-02	Blue Willet 3/12/07	Wans Willet 3/11/07	J.W. Troncello 3/11/07	Wans Willet 3/11/07
2/14	074	RHRS	DRHR-2-03	Y	Add Weld DRHR-2-03 UT exam Cat. A14-03 IT# R-A, R1.16D	EXPANDED Scope EXREQ: A14-03	Blue Willet 3/12/07	Wans Willet 3/11/07	J.W. Troncello 3/11/07	Wans Willet 3/11/07
			DRHR-2-03	N	NU0313 - Cat. D	B12-02	SCD 3/14/07			
2/14	085	CRDS	RCRD-2-50	Y	Add Weld RCRD-2-50 UT exam Cat. A14-03 IT# R-A, R1.16D	EXPANDED Scope EXREQ: A14-03	Blue Willet 3/12/07	Wans Willet 3/11/07	J.W. Troncello 3/11/07	Wans Willet 3/11/07
			RCRD-2-50	N	NU0313 - Cat. D	B12-02	SCD 3/14/07			

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2/14	001	RWCW	2-ISV-69-500	Y	Add valve VT-3 EXAM 2-ISV-69-500 Cat. B-M-2 IT# B12.50	Valve Disassembled	<i>Steph Willard</i> 3/12/07	<i>Max Willard</i> 3/12/07	<i>F.W. Trucello</i> 3/12/07	<i>Max Willard</i> 3/12/07
2/14	001 069 7/8 3/15/07	RWCUS	DRWC-2-01 DRWC-2-01A	Y	Delete welds: DRWC-2-01, DRWC-2-01A Cat. R-A, IT# R1.16C & B02-02 - Cat. C NU0313	Welds to be cut out to remove valve 2-ISV-69-500. ALARA consideration	<i>Steph Willard</i> 3/15/07	<i>Max Willard</i> 3/15/07	<i>F.W. Trucello</i> 3/15/07	<i>Max Willard</i> 3/15/07
2/14	001	RPV 3/18/07 RPV	RPV N-V welds: N2B, N2D, N2K N3A, N3C, N4F N4B, N4C, N4D, N4E N8B	Y	ADD THE LISTED RPV N-V WELDS: N2B, N2D, N2K, N3A, N3C, N4B, N4C, N4E, N4F and N8B CATEGORY B-D ITEM B3.90 EXREQ A15-03	IWB-2430 EXPANDED SCOPE ADDITIONAL EXAMINATIONS	<i>Steph Willard</i> 3/21/07	<i>Max Willard</i> 3/18/07	<i>F.W. Trucello</i> 3/18/07	<i>Max Willard</i> 3/18/07
2/14	001	MS	2-FCV-01-037	Y	Add valve VT-3 PSI EXAM 2-FCV-01-037 EXREQ: P95-96 Cat. B-M-2, IT# B12.50	Value Disassembled Repair of Valve Guide Weld MS-2-003-003 CORO	<i>Steph Willard</i> 3/21/07	<i>Max Willard</i> 3/21/07	<i>F.W. Trucello</i> 3/21/07	<i>Max Willard</i> 3/21/07
2/14	001	FW	ZRFWZA-10E ZRFWZA-22E ZRFWZA-33E R-A, R1.18	Y	DELETE COMPONENTS FROM SCAN PLAN	DELETED BY BFN FAC ENGINEERING	<i>Steph Willard</i> 4/25/07	<i>Max Willard</i> 4/12/07	<i>F.W. Trucello</i> 4/12/07	<i>Max Willard</i> 4/12/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	MSS	ZMSZ-MS1D-9FN ZMSZ-MS2D-9FN R-A, R1.18	Y	Delete components from scan plan.	Deleted by FAC BFN engineering	Stephen W. Willard 4/25/07	Mark Wickel 4/12/07	F.W. Froncello 4/12/07	Mark Wickel 4/12/07
2/14	001	MSS	PCV1-2-030 Valve	Y	Delete component from scan plan.	Exam item B12.50, B-N-2 category satisfied in previous RFO.	Stephen W. Willard 4/27/07	Mark Wickel 4/12/07	F.W. Froncello 4/12/07	Mark Wickel 4/12/07
2/14	001	RECIRC	PMP-A-NUT-2-01 THRU -16 PMP-B-NUT-2-01 THRU -16 item B6.200 category B-G-1, P95-96	Y	Add to RFO U2C14 ISI scan plan.	Preservice examinations New material W.O. 06-718735-006 06-718765-006	Stephen W. Willard 4/27/07	Mark Wickel 4/12/07	F.W. Froncello 4/12/07	Mark Wickel 4/12/07
2/14	001	RECIRC	PMP-A-WASH-2-01 to -16 PMP-B-WASH-2-01 to -16 item B6.200 cat B-G-1 spec P95-96	Y	Add to RFO U2C14 ISI scan plan.	Preservice examinations New material 06-718765-006 W.O. 06-718735-006	Stephen W. Willard 4/27/07	Mark Wickel 4/12/07	F.W. Froncello 4/12/07	Mark Wickel 4/12/07
2/14	001	RECIRC	Z-47B 40850069-IE cat F-A item F1.40D spec P95-96	Y	Add to RFO U2C14 ISI scan plan.	Preservice examination Ref w/o # 04-718365-000	Stephen W. Willard 4/25/07	Mark Wickel 4/12/07	F.W. Froncello 4/12/07	Mark Wickel 4/12/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	RWCU	RWCU-2-003-069 RWCU-2-003-070 Cat B-J, B9.11, P95-96 165CL, Cat D, B02-02 Exam req. UT & PT	Y	Add to RFO VZC14 scan plan	Pre-service examinations New welds. Ref w/o # 06-723636-001 and 00-003350-000.	<i>Stephen Wilk</i> 4/25/07	<i>Mark Wilk</i> 4/12/07	<i>F.W. Forscello</i> 4/12/07	<i>Mark Wilk</i> 4/12/07
2/14	001	RWCU	RWCU-2-003-071 Cat B-J, B9.11, P95-96 145CL, Cat A, B02-02 Exam req. UT & PT	Y	Add to RFO VZC14 scan plan	Pre-service examinations New welds. Ref w/o # 06-723034-001	<i>Stephen Wilk</i> 4/25/07	<i>Mark Wilk</i> 4/12/07	<i>F.W. Forscello</i> 4/12/07	<i>Mark Wilk</i> 4/12/07
2/14	001	NSS	2-471407 24 Cat F-A, F1.100 2-47B40050110 2-47B40050110-IA Cat B-K, B10.20 Exam P95-96	Y	Add to RFO VZC14 scan plan, VT-3 and PT exams.	Pre-service examinations Ref w/o # 02-011925-001	<i>Stephen Wilk</i> 4/25/07	<i>Mark Wilk</i> 4/12/07	<i>F.W. Forscello</i> 4/12/07	<i>Mark Wilk</i> 4/12/07
2/14	001	RHRS500	2-47B450-R003B Cat F-A, F1.30B Exam P95-96	Y	Add to RFO VZC14 scan plan. VT-3 exam	Pre-service examinations Ref: w/o # 06-718765-017	<i>Stephen Wilk</i> 4/25/07	<i>Mark Wilk</i> 4/12/07	<i>F.W. Forscello</i> 4/12/07	<i>Mark Wilk</i> 4/12/07
2/14	001	HPCI	HPCI-2-001-003 HPCI-2-001-004 HPCI-2-001-005 B-J, B9.11, P95-96	Y	Add to RFO VZC14 scan plan Surface and VT exams.	Pre-service exams New welds Ref: w/o # 00-003350-000 00-003350-000	<i>Stephen Wilk</i> 4/25/07	<i>Mark Wilk</i> 4/12/07	<i>F.W. Forscello</i> 4/12/07	<i>Mark Wilk</i> 4/12/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE.	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	RWELVS	CRD-2-005-003 R-A, R1.16D, P95-96 145CC, D, B02-02	Y	Add to RFO UCL14 Scan plan. UT exam	New weld Pressure exam. Ref: w/p # 07-713160-000	Stephen W. King 4/25/07	Mark W. King 4/24/07	F.W. Forsythe 4/12/07	Mark W. King 4/19/07
2/14	001	HPRE	2-47B455-2123 2-47B455-2124 F-A, F1.10D, P95-96	Y	Add to RFO UCL14 Scan plan. UT-3 exam	New supports Pressure exams Ref: w/p # 00-003350-003 and 00-003350-002	Stephen W. King 4/25/07	Mark W. King 4/24/07	F.W. Forsythe 4/12/07	Mark W. King 4/19/07

BFN SCAN PLAN REVISION LOG

UNIT/ CYCLE	SCAN PLAN REV.	SYSTEM	COMPONENT IDENTIFIER	ASME XI	REVISION	REASON FOR REVISION	APPROVED BY ISI/NDE SIGN AND DATE	APPROVED BY NDE LEVEL III SIGN AND DATE	ISI DATA BASE REVISED BY ISO SIGN AND DATE	ISI DATA BASE REVISION VERIFIED BY ISI/NDE SIGN AND DATE
2/14	001	RPV	RPV-INTERIOR	Y	ADD THE FOLLOWING: ITEM NO/ EXSCHED/CAT B13.20/VT-1/B-N-2 B13.30/VT-3/B-N-2 B13.40/VT-3/B-N-2	UPDATE/CLARIFICATION OF CODE EXAMS.	<i>Stephen Willard 5/24/07</i>	<i>Mark Wheeler 4/20/07</i>	<i>J.W. Procella 4/20/07</i>	<i>Mark Wheeler 4/20/07</i>
2/14	001	RPV	Z-SI-3.3.1.A	Y	ADD THE FOLLOWING: ITEM NO./CAT/EXSCHED B15.10/B-P/VT-2 B15.50/B-P/VT-2 B15.60/B-P/VT-2 B15.70/B-P/VT-2	SYSTEM LEAKAGE TEST PER ASME SEC. XI, 95/96. EXAM CAT B-P.	<i>Stephen Willard 4/25/07</i>	<i>Mark Wheeler 4/23/07</i>	<i>J.W. Procella 4/23/07</i>	<i>Mark Wheeler 4/23/07</i>
2/14	001	RPV	Z-SI-3.3.1.A	Y	ADD THE FOLLOWING: ITEM/CAT/EXSCHED C7.30/C-H/VT-2 C7.70/C-H/VT-2	SYSTEM LEAKAGE TEST PER ASME SEC. XI, 95- 96, EXAM CAT C-H	<i>Stephen Willard 4/25/07</i>	<i>Mark Wheeler 4/23/07</i>	<i>J.W. Procella 4/23/07</i>	<i>Mark Wheeler 4/23/07</i>

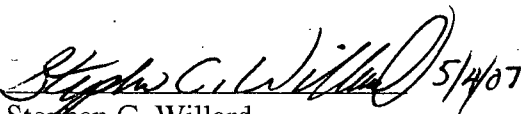
May 03, 2007

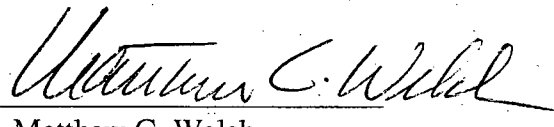
Sam Flood, ANI/ANII, PEC-1C, BFN

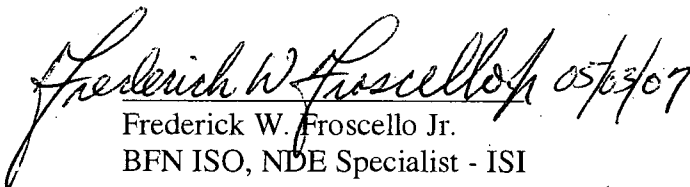
**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE INSERVICE INSPECTION (ISI) SCAN PLAN REVISION 001**

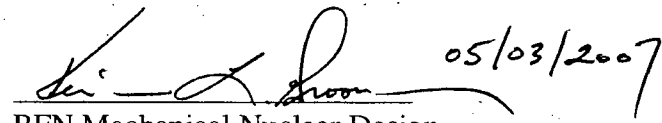
Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage ISI Scan Plan, Revision 001, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, 1995 Edition, 1996 Addenda.


This document was prepared by Fred Froscello of Inspection Service Organization (ISO) and coordinated with Stephen C. Willard of BFN Components Engineering and Matthew Welch of TVAN Inspection Services Organization (ISO).


Stephen C. Willard
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BFN Components Engineering


Matthew C. Welch
BFN ISO, NDE Level III


Frederick W. Froscello Jr.
BFN ISO, NDE Specialist - ISI


Ken L. Brown
BFN Mechanical Nuclear Design
Engineering (TSR3.4.3.2, BWRVIP-27,
BWRVIP-49, BWRVIP-75, SPP-9.7,
APPENDIX. "B")


Sam Flood ANI/ANII
Concurrence

cc: T. L. Shults, SAB-1B, BFN
M. L. Turnbow, STC-1I, SQN

Revision 001

05/02/2007

Total Examinations: 632

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
CRDS	2-SI-3.3.1.D		N/A	14	C7.10	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CRDS	2-SI-3.3.1.D		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CRDS	2-SI-3.3.1.D		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CSS	2-47B458S0029		2-ISI-0280-C-01	14	F1.10B	F-A	96E-03	VT-3	N-VT-1					RGD HGR
CSS	2-SI-3.3.6		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CSS	2-SI-3.3.6		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CSS	2-SI-3.3.6		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4					SYSLEAK
CSS	FCV-75-26		2-ISI-0271-C-01	14	B12.50	B-M-2	96E-03	VT-3	N-VT-1		12.75			INT
CSS	FCV-75-26-BC		2-ISI-0271-C-01	14	B7.70	B-G-2	96E-03	VT-1	N-VT-1					BLTG
EECWS	0-37B205S0053		ISI-0368-C-03	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	0-37B205S0055		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0056		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0064		ISI-0368-C-01	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		24.00			RGD HGR
EECWS	0-37B205S0067		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	0-37B205S0071		ISI-0368-C-03	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0072		ISI-0368-C-03	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	1-47B451H0017		ISI-0368-C-13	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0013		ISI-0368-C-13	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0039		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD STRT
EECWS	1-47B451R0040		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0045		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD STRT
EECWS	1-47B451S0055		ISI-0368-C-15	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		06.00			RGD HGR
EECWS	1-47B451S0290		ISI-0368-C-10	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		06.00			RGD HGR
EECWS	2-47B451R0009-1A		ISI-0368-C-06	14	D1.20	D-A	96E-03	VT-1	N-VT-1			1.500		WLD ATT
EECWS	2-SI-3.3.14A		N/A	14	D2.10	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14A		N/A	14	D2.30	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14A		N/A	14	D2.50	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14A		N/A	14	D2.70	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14B		N/A	14	D2.10	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14B		N/A	14	D2.30	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14B		N/A	14	D2.50	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
EECWS	2-SI-3.3.14B		N/A	14	D2.70	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
FPCS	2-SI-3.3.3		N/A	14	D2.10	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
FPCS	2-SI-3.3.3		N/A	14	D2.30	D-B	96E-03	VT-2	N-VT-4					SYSLEAK

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDA	NOMTHCK	COMPDESA	COMPDESB
FPCS	2-SI-3.3.3		N/A	14	D2.70	D-B	96E-03	VT-2	N-VT-4					SYSLEAK
FWS	2RFW2A-14R	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8729				RED
FWS	2RFW2A-17R	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	01-7476	20	1.281		RED
FWS	2RFW2A-20E	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	01-7474				EL
FWS	2RFW2A-21P	2-003-037	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	01-7474				P
FWS	2RFW2A-24E	2-003-037	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8730				EL
FWS	2RFW2A-39E	2-003-039	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8729	12	1.012		EL
FWS	2RFW2A-41E	2-003-039	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8729	12	0.844		EL
FWS	2RFW2B-22E	2-003-041	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8730	12.75	0.844		EL
FWS	2RFW2B-36E	2-003-042	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8730	12.75	0.844		EL
FWS	2RFW2B-41E	2-003-043	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	048730	12.75	0.844		EL
FWS	2RFW2B-44E	2-003-043	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8734/01-7474	12.75	0.844		EL
HPCIS	2-47B455-2123		ISI-0275-C-01	14	F1.10D	F-A	P95-96	VT-3	N-VT-1		10.00			SNBR
HPCIS	2-47B455-2124		ISI-0275-C-01	14	F1.10D	F-A	P95-96	VT-3	N-VT-1		10.00			SNBR
HPCIS	2-47B455H0061-IA		2-ISI-0130-C-01	14	C3.20	C-C	96E-03	MT	N-MT-6			0.500		WLD ATT
HPCIS	2-SI-3.3.9		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4					HYDRO
HPCIS	2-SI-3.3.9		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4					HYDRO
HPCIS	2-SI-3.3.9		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4					HYDRO
HPCIS	HPCI-2-001-003		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	MT	N-MT-6		10.00	0.593		P EL
HPCIS	HPCI-2-001-003		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	UT	N-UT-76	SQ-115	10.00	0.593		P EL
HPCIS	HPCI-2-001-004		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	PT	N-PT-9		10.00	0.593		VLV P
HPCIS	HPCI-2-001-004		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	UT	N-UT-76	ALTCS/WB 78	10.00	0.593		VLV P
HPCIS	HPCI-2-001-005		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	MT	N-MT-6		10.00	0.593		P VLV
HPCIS	HPCI-2-001-005		2-ISI-0273-C-01	14	B9.11	B-J	P95-96	UT	N-UT-76	SQ-115	10.00	0.593		P VLV
MSS	2-47B400S0006		2-ISI-0279-C-02	14	F1.10B	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0007		2-ISI-0279-C-02	14	F1.10B	F-A	96E-03	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0007		2-ISI-0279-C-02	14	F1.10B	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0010		2-ISI-0279-C-01	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0010-IA		2-ISI-0279-C-01	14	B10.20	B-K	96E-03	MT	N-MT-6			0.578		WLD ATT
MSS	2-47B400S0018		2-ISI-0279-C-02	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0021-IA		2-ISI-0279-C-01	14	B10.20	B-K	96E-03	MT	N-MT-6			1.625		WLD ATT
MSS	2-47B400S0022		2-ISI-0279-C-01	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0038		2-ISI-0279-C-01	14	F1.10A	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0039		2-ISI-0279-C-02	14	F1.10A	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0040		2-ISI-0279-C-02	14	F1.10A	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0041		2-ISI-0279-C-01	14	F1.10B	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0096		2-ISI-0279-C-01	14	F1.10D	F-A	96E-03	VT-3	N-VT-1		26.00			SNBR
MSS	2-47B400S0105-IA		2-ISI-0279-C-02	14	B10.20	B-K	96E-03	MT	N-MT-6			1.250		WLD ATT

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
MSS	2-47B400S0107		2-ISI-0279-C-02	14	F1.10D	F-A	96E-03	VT-3	N-VT-1		26.00		SNBR	
MSS	2-47B400S0110		2-ISI-0279-C-01	14	F1.10D	F-A	P95-96	VT-3	N-VT-1		26.00		VAR SUP	
MSS	2-47B400S0110-1A		2-ISI-0279-C-01	14	B10.20	B-K	P95-96	PT	N-PT-9			0.500	WLD ATT	
MSS	2MSZ-MS1A-6E	2-001-036	2-ISI-0222-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	01-7476	26	1.012	EL	
MSS	2MSZ-MS1D-4E	2-001-039	2-ISI-0222-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26	04-8730	26	1.012	EL	
MSS	2-SI-3.3.1.C		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
MSS	2-SI-3.3.1.C		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
MSS	FCV-01-015		2-ISI-0222-C-01	14	B12.50	B-M-2	96E-03	VT-3	N-VT-1		26.00		INT	
MSS	FCV-01-015-BC		2-ISI-0222-C-01	14	B7.70	B-G-2	96E-03	VT-1	N-VT-1				BLTG	
MSS	FCV-01-037		2-ISI-0222-C-02	14	B12.50	B-M-2	P95-96	VT-3	N-VT-1		26.00		INT	
MSS	FCV-01-037-BC		2-ISI-0222-C-02	14	B7.70	B-G-2	P95-96	VT-1	N-VT-1				BLTG	
MSS	FCV-01-037-BC		2-ISI-0222-C-02	14	B7.70	B-G-2	V01-02	VT-1	N-VT-1				BLTG	
RCICS	2-47B456H0027		2-ISI-0131-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		06.00		VAR SUP	
RCICS	2-SI-3.3.10		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RCICS	2-SI-3.3.10		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RCICS	2-SI-3.3.10		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RCICS	RCICH-2-1		2-ISI-0131-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				PUMP	
RCICS	RCICH-2-2		2-ISI-0131-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				PUMP	
RECIR	2-47B408S0063		2-ISI-0278-C-01	14	F1.40D	F-A	96E-03	VT-3	N-VT-1				SNBR	
RECIR	2-47B408S0067-IE		2-ISI-0278-C-01	14	F1.40D	F-A	96E-03	VT-3	N-VT-1				SNBR	
RECIR	2-47B408S0069-IE		2-ISI-0278-C-01	14	F1.40D	F-A	P95-96	VT-3	N-VT-1				SNBR	
RECIR	GR-2-15(OL)	2-068-006	2-ISI-0270-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-66	BF-133	12.00	1.139	SDL	P
RECIR	GR-2-15(OL)	2-068-006	2-ISI-0270-C-01	14	NU0313	E	B02-02	UT	N-UT-66	BF-133	12.00	1.139	SDL	P
RECIR	GR-2-38	2-068-010	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS/WB 85	12.00	0.569	SDL	P
RECIR	GR-2-38	2-068-010	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS/WB 85	12.00	0.569	SDL	P
RECIR	GR-2-41	2-068-011	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS/SQ1 23	12.00	0.569		P
RECIR	GR-2-41	2-068-011	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS/SQ1 23	12.00	0.569		P
RECIR	GR-2-48	2-068-013	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	12.00	0.569	SDL	P
RECIR	GR-2-48	2-068-013	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS/SQ1 23	12.00	0.569	SDL	P
RECIR	GR-2-53	2-068-016	2-ISI-0270-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-64	ALTSS/WB 85	28.00	1.138	SE	P
RECIR	GR-2-53	2-068-016	2-ISI-0270-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS/BF1 33	28.00	1.138	SE	P
RECIR	PMP-2-1B		2-ISI-0407-C-01	14	B12.20	B-L-2	96E-03	VT-3	N-VT-1				INT	
RECIR	PMP-A-NUT-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PNUTS	
RECIR	PMP-A-NUT-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PNUTS	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPESA	COMPESB
RECIR	PMP-A-NUT-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-NUT-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-A-WASH-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-A-WASH-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-FLG		2-ISI-0407-C-01	14	B6.190	B-G-1	96E-03	VT-1	N-VT-1					FLG SUR
RECIR	PMP-B-NUT-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDES B
RECIR	PMP-B-NUT-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-STUD-2-01		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-02		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-03		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-04		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-05		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-06		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-07		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-08		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-09		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-10		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-11		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-12		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-13		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-14		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-15		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-STUD-2-16		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67	BF-119	03.00			BLTG
RECIR	PMP-B-WASH-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
RECIR	PMP-B-WASH-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RECIR	PMP-B-WASH-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	P95-96	VT-1	N-VT-1		03.00		PWASH	
RECIR	RBC-2-1		2-ISI-0270-C-01	14	B7.50	B-G-2	96E-03	VT-1	N-VT-1				BLTG	
RHR	2-SI-3.3.8.C		N/A	14	C7.10	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHR	2-SI-3.3.8.C		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHR	2-SI-3.3.8.C		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHR	2-SI-3.3.8.C		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-47B452H0015		2-ISI-0324-C-08	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		24.00		RGD STRT	
RHRS	2-47B452H0022		2-ISI-0324-C-09	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		24.00		RGD STRT	
RHRS	2-47B452H0035		2-ISI-0324-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0037		2-ISI-0324-C-01	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		20.00		RGD STRT	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPdia	NOMTHCK	COMPDESA	COMPDESB
RHRS	2-47B452H0064		2-ISI-0324-C-05	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0064-IA		2-ISI-0324-C-05	14	C3.20	C-C	96E-03	MT	N-MT-6			1.000	WLD ATT	
RHRS	2-47B452H0069		2-ISI-0324-C-07	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0069-IA		2-ISI-0324-C-07	14	C3.20	C-C	96E-03	MT	N-MT-6			0.500	WLD ATT	
RHRS	2-47B452H0088		2-ISI-0324-C-02	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0089		2-ISI-0324-C-02	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0106		2-ISI-0324-C-03	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		14.00		RGD HGR	
RHRS	2-47B452H0126		2-ISI-0324-C-07	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0158		2-ISI-0324-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0158-IA		2-ISI-0324-C-01	14	C3.20	C-C	96E-03	MT	N-MT-6			0.237	WLD ATT	
RHRS	2-47B452R0054		ISI-0324-C-06	14	F1.20D	F-A	96E-03	VT-3	N-VT-1		20.00		SNBR	
RHRS	2-47B452R0059		2-ISI-0324-C-05	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		20.00		RGD HGR	
RHRS	2-47B452R0064		2-ISI-0324-C-07	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		20.00		RGD HGR	
RHRS	2-47B452S0308		2-ISI-0324-C-10	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		06.00		RGD HGR	
RHRS	2-SI-3.3.8.A		N/A	14	C7.10	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-SI-3.3.8.A		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-SI-3.3.8.A		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-SI-3.3.8.A		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-SI-3.3.8.B		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	2-SI-3.3.8.B		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRS	DRHR-2-03	2-074-002	2-ISI-0221-C-01	14	R1.16D	R-A	A14-03	UT	N-UT-64	SQ-123	24.00	1.219	VLV	FH
RHRS	DRHR-2-03	2-074-002	2-ISI-0221-C-01	14	D	NU0313	B12-02	UT	N-UT-64	SQ-123	24.00	1.219	VLV	FH
RHRS	DRHR-2-03B	2-074-005	2-ISI-0221-C-01	14	R1.16G	R-A	96E-03	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-03B	2-074-005	2-ISI-0221-C-01	14	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-09		2-ISI-0221-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS/BFI	24.00	1.219	P	P
RHRS	DRHR-2-11		2-MSG-0018-C-09	14	NU0313	D	B12-02	UT	N-UT-82	BF-102	24.00	1.531	P	VLV
RHRS	DRHR-2-12		2-ISI-0221-C-01	14	NU0313	D	B02-02	UT	N-UT-64	ALTSS/WB	24.00	1.219	VLV	P
RHRS	DRHR-2-13B	2-074-013	2-ISI-0221-C-01	14	R1.16G	R-A	96E-03	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-13B	2-074-013	2-ISI-0221-C-01	14	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-22		2-ISI-0221-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS/BFI	20.00	1.031	VLV	P
RHRS	DSRHR-2-04		2-ISI-0221-C-01	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS/SQ1	24.00	1.219	P	EL
RHRS	RHR-2-H-343-IE		2-ISI-0324-C-08	14	F1.40C	F-A	96E-03	VT-3	N-VT-1		18.00		RGD HGR	
RHRS	RHR-2-R-223		2-ISI-0324-C-02	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		24.00		RGD HGR	
RHRS	RHRG-2-05-A		2-ISI-0406-C-01	14	C2.33	C-B	96E-03	VT-2	N-VT-4		10.50	0.875	NOZ	
RHRS	RHRG-2-05A-A		2-ISI-0406-C-01	14	C2.31	C-B	96E-03	MT	N-MT-6		24	0.875	SHL	NOZ
RHRS	RHRG-2-05B-A		2-ISI-0406-C-01	14	C2.31	C-B	96E-03	MT	N-MT-6		24	0.875	NOZ	SHL
RHRS	RHRG-2-06-A		2-ISI-0406-C-01	14	C2.33	C-B	96E-03	VT-2	N-VT-4		10.50	0.875	NOZ	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPdia	NOMTHCK	COMPESA	COMPESB
RHRS	RHRG-2-09-A		2-ISI-0406-C-01	14	C1.10	C-A	96E-03	UT	N-UT-18	BF-40	54.00	0.875	HD	FLG
RHRS	RHRG-2-12-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-13-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-14-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRPH-2-B		ISI-0310-B-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HPMP	
RHRS	TRHR-2-281	2-074-031	MSG-0018-C-02	14	R1.11	R-A	96E-03	UT	N-UT-76	ALTCS/WB 78	24.00	0.500	VLV	EL
RHRS	TRHR-2-295	2-074-030	MSG-0018-C-02	14	R1.11	R-A	96E-03	UT	N-UT-76	ALTCS/WB 78	24.00	0.500	VLV	EL
RHRSW	2-17B300S0070		2-ISI-0145-C-01	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		16.00		RGD HGR	
RHRSW	2-47B450R0027		2-ISI-0145-C-02	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		16.00		RGD HGR	
RHRSW	2-47B450R0028-IA		2-ISI-0145-C-02	14	D1.20	D-A	96E-03	VT-1	N-VT-1			1.00	WLD ATT	
RHRSW	2-47B450R0038		2-ISI-0145-C-03	14	F1.30B	F-A	P95-96	VT-3	N-VT-1		12.00		RGD HGR	
RHRSW	2-SI-3.3.13		N/A	14	D2.10	D-B	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRSW	2-SI-3.3.13		N/A	14	D2.30	D-B	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRSW	2-SI-3.3.13		N/A	14	D2.50	D-B	96E-03	VT-2	N-VT-4				SYSLEAK	
RHRSW	2-SI-3.3.13		N/A	14	D2.70	D-B	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	B15.10	B-P	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	B15.50	B-P	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	B15.60	B-P	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	B15.70	B-P	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	2-SI-3.3.1.A		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
RPV	N10-SE		2-ISI-0380-C-01	14	N/A	BWRVIP-27	B07-02	VT-2	N-VT-4		1.5	0.218	SE	NOZ
RPV	N11A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N11B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N12A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N12B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	P	PC,PIPE
RPV	N-16A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.218	NOZ	SE
RPV	N-16B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	NOZ	SE
RPV	N1B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		28.00	6.600	NOZ IR	
RPV	N1B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	28.00	6.600	SHL	NOZ
RPV	N2A-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12	6.600	NOZ IR	
RPV	N2A-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2B-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2C-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2C-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2D-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2G-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2G-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	CALSTD	COMPDIS	QOMTHCK	COMPDESA	COMPDESB
RPV	N2H-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2H-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2K-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N3A-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	26.00	6.600	NOZ	SHL
RPV	N3B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		26.00	6.600	NOZ IR	
RPV	N3B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	26.00	6.600	NOZ	SHL
RPV	N3C-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	26.00	6.600	NOZ	SHL
RPV	N4A-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4B-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4B-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N4C-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4C-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N4D-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4E-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4E-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N4F-FW-SPARG		2-CHM-2046-C-02	14	N/A	EPU74383	B01-02	VT-1E	VENDOR VT					
RPV	N4F-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N5A-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		10.00	6.600	NOZ IR	
RPV	N5A-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	10.00	6.6	NOZ	SHL
RPV	N5B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		10.00	6.600	NOZ IR	
RPV	N5B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	10.00	6.600	NOZ	SHL
RPV	N6A-2-1-BC		ISI-0408-C-01	14	B7.50	B-G-2	96E-03	VT-1	N-VT-1				BLTG	
RPV	N7-IR		ISI-0408-C-01	14	B3.100	B-D	96E-03	VT-1E	N-VT-1		04.00	4.340	NOZ IR	
RPV	N7-NV		ISI-0408-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-19	04.00	4.310	CL HD	NOZ
RPV	N8B-NV		2-CHM-2046-C-01	14	B3.90	B-D	A15-03	UT	N-UT-78	BF-18	04.06	6.600	NOZ	SHL
RPV	N9-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		04.00	6.600	NOZ IR	
RPV	N9-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	04.00	6.600	NOZ	SHL
RPV	RCH-2-4V		ISI-0408-C-01	14	B1.22	B-A	96E-03	UT	N-UT-78	BF-19	-96L	4.000	VERT LS	
RPV	RCH-2-6V		ISI-0408-C-01	14	B1.22	B-A	96E-03	UT	N-UT-78	BF-19	-96L	4.00	VERT LS	
RPV	RCRD-2-33		2-ISI-0272-C-01	14	NU0313	D	B12-02	UT	N-UT-82	BF-131	4.00	0.674	NOZ	CAP
RPV	RPV CORE PLATE		2-CHM-2046-C-02	14	N/A	BWRVIP-25	0TI365	VT-3	VENDOR VT				INT	
RPV	RPV CORE SUPPORT		2-CHM-2046-C-02	14	B13.40	B-N-2	96E-03	VT-3	VENDOR VT				INT	
RPV	RPV CR GUIDE TUBES		2-CHM-2046-C-02	14	N/A	BWRVIP-47	0TI365	VT-3	VENDOR VT				INT	
RPV	RPV CR GUIDE TUBES		2-CHM-2046-C-02	14	N/A	BWRVIP-47	0TI365	VT-1E	VENDOR VT				INT	
RPV	RPV CS PIPING		2-CHM-2046-C-02	14	N/A	BWRVIP-18	0TI365	VT-3	VENDOR VT		6.000		INT	
RPV	RPV CS PIPING		2-CHM-2046-C-02	14	N/A	BWRVIP-18	0TI365	VT-1E	VENDOR VT		6.000		INT	
RPV	RPV CS PIPING		2-CHM-2046-C-02	14	N/A	BWRVIP-18	0TI365	VT-1	VENDOR VT		6.000		INT	
RPV	RPV INT ATT BLR		2-CHM-2046-C-02	14	B13.20	B-N-2	96E-03	VT-1	VENDOR VT				INT	
RPV	RPV INT ATT NBLR		2-CHM-2046-C-02	14	B13.30	B-N-2	96E-03	VT-3	VENDOR VT				INT	
RPV	RPV JET PUMPS		2-CHM-2046-C-02	14	N/A	BWRVIP-41	0TI365	VT-1	VENDOR VT				INT	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDLA	NOMTHCK	COMPDESA	COMPDES
RPV	RPV JET PUMPS		2-CHM-2046-C-02	14	N/A	BWRVIP-41	0TI365	VT-1E	VENDOR VT				INT	
RPV	RPV JET PUMPS		2-CHM-2046-C-02	14	N/A	BWRVIP-41	0TI365	VT-3	VENDOR VT				INT	
RPV	RPV STEAM DRYER		2-CHM-2046-C-02	14	N/A	BWRVIP13	0TI365	VT-1	VENDOR VT				INT	
RPV	RPV STEAM SEPARATO		2-CHM-2046-C-02	14	N/A	EPU74383	0TI365	VT-1	VENDOR VT				INT	
RPV	RPV TOP GUIDE		2-CHM-2046-C-02	14	N/A	BWRVIP-26	0TI365	VT-1	VENDOR VT				INT	
RPV	RPV TOP GUIDE		2-CHM-2046-C-02	14	N/A	BWRVIP-26	0TI365	VT-1E	VENDOR VT				INT	
RPV	RPV-BUSH-2-22		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-23		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-24		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-25		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-INTERIOR		2-CHM-2046-C-02	14	B13.10	B-N-1	96E-03	VT-3	VENDOR VT				INT	
RPV	RPV-NUTS-2-31		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-32		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-33		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-34		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-35		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-36		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-37		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-38		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-39		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-40		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-41		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-42		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-43		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-44		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-45		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-46		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-47		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-48		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-49		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-50		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-51		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-52		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-53		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-54		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-55		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-56		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-57		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-58		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-59		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-WASH-2-45		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-46		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-47		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-48		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-49		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-50		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-51		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	BUSH	
RPV	RPV-WASH-2-52		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-53		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-54		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-55		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-56		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-57		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-58		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-59		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-60		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-61		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	SHROUD WELD H-1		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-2		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-3		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-4		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-5		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-6		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD H-7		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD V7		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RPV	SHROUD WELD V8		2-CHM-2046-C-02	14	N/A	BWRVIP-76	0TI365	UT	VENDOR UT				INT	
RWCUS	69-500		2-ISI-0272-C-01	14	B12.50	B-M-2	96E-03	VT-3	N-VT-1		06.00		INT	
RWCUS	CRD-2-005-003	2-085-031	2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-82	SQ-116	4.00	0.531	VLV	P
RWCUS	CRD-2-005-003	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	P95-96	UT	N-UT-82	SQ-116	4.00	0.531	VLV	P
RWCUS	DSRWC-2-03(OL)	2-069-001	2-ISI-0272-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-66	BF-133	06.00	0.700	P	EL
RWCUS	DSRWC-2-03(OL)	2-069-001	2-ISI-0272-C-01	14	NU0313	E	B02-02	UT	N-UT-66	BF-133	06.00	0.700	P	EL
RWCUS	RCRD-2-49	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	EL
RWCUS	RCRD-2-49	2-085-031	2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	EL
RWCUS	RCRD-2-50	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	A14-03	UT	N-UT-82	BF-132	4.00	0.531	EL	VLV
RWCUS	RCRD-2-50	2-085-031	2-ISI-0272-C-01	14	D	NU0313	B12-02	UT	N-UT-82	BF-131/BF-132	4.00	0.531	EL	VLV
RWCUS	RCRD-2-52	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	P

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
RWCUS	RCRD-2-52	2-085-031	2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	P
RWCUS	RWCU-2-003-069		2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-003-069		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	PT	N-PT-9		06.00	0.432	P	P
RWCUS	RWCU-2-003-069		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-003-070		2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-003-070		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	PT	N-PT-9		06.00	0.432	P	P
RWCUS	RWCU-2-003-070		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-003-071		2-ISI-0272-C-01	14	NU0313	A	B02-02	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-003-071		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	PT	N-PT-9		06.00	0.432	P	P
RWCUS	RWCU-2-003-071		2-ISI-0272-C-01	14	B9.11	B-J	P95-96	UT	N-UT-64	SQ-116	06.00	0.432	P	P
RWCUS	RWCU-2-004-083	2-069-003	2-ISI-0272-C-01	14	R1.16A	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.337	VLV	P
SLCS	2-SI-3.3.4		N/A	14	C7.10	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
SLCS	2-SI-3.3.4		N/A	14	C7.30	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
SLCS	2-SI-3.3.4		N/A	14	C7.50	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	
SLCS	2-SI-3.3.4		N/A	14	C7.70	C-H	96E-03	VT-2	N-VT-4				SYSLEAK	


December 06, 2006

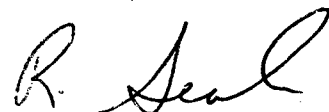
Sam Flood, ANI/ANII, PEC-1C, BFN

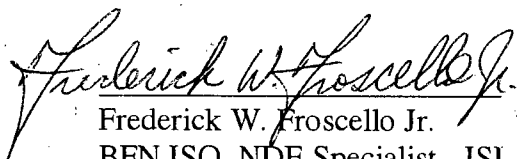
**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE INSERVICE INSPECTION (ISI) SCAN PLAN REVISION 000**


Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage ISI Scan Plan, Revision 000, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, 1995 Edition, 1996 Addenda.


This document was prepared by Harold E. Hodges of BFN Components Engineering and coordinated with Richard Seals and Fred Froscello of TVAN Inspection Services Organization (ISO).


1/8/2007
Harold E. Hodges
ISI Engineer
BFN Components Engineering


Richard A. Seals
BFN ISO, NDE Level III


12/06/06
1/8/07
Frederick W. Froscello Jr.
BFN ISO, NDE Specialist - ISI


VDS
02-05-2007
BFN Mechanical Nuclear Design
Engineering (TSR3.4.3.2, BWRVIP-27,
BWRVIP-49, BWRVIP-75, SPP-9.7,
APPENDIX. "B")


2/5/07
Sam Flood ANI/ANII
Concurrence

cc: T. L. Shults, SAB-1B, BFN
M. L. Turnbow, STC-1I, SQN

Revision 000

01/19/2007

Total Examinations: 450

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
CSS	2-47B458S0029		2-ISI-0280-C-01	14	F1.10B	F-A	96E-03	VT-3	N-VT-1					RGD HGR
EECWS	0-37B205S0053		ISI-0368-C-03	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	0-37B205S0055		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0056		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0064		ISI-0368-C-01	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		24.00			RGD HGR
EECWS	0-37B205S0067		ISI-0368-C-01	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	0-37B205S0071		ISI-0368-C-03	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	0-37B205S0072		ISI-0368-C-03	14	F1.30A	F-A	96E-03	VT-3	N-VT-1		14.00			RGD HGR
EECWS	1-47B451H0017		ISI-0368-C-13	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0013		ISI-0368-C-13	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0039		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD STRT
EECWS	1-47B451R0040		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD HGR
EECWS	1-47B451R0045		ISI-0368-C-12	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		18.00			RGD STRT
EECWS	1-47B451S0055		ISI-0368-C-15	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		06.00			RGD HGR
EECWS	1-47B451S0290		ISI-0368-C-10	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		06.00			RGD HGR
EECWS	2-47B451R0009-1A		ISI-0368-C-06	14	D1.20	D-A	96E-03	VT-1	N-VT-1			1.500		WLD ATT
FWS	2RFW2A-10E	2-003-006	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		24	1.531		EL
FWS	2RFW2A-14R	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					RED
FWS	2RFW2A-17R	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		20	1.281		RED
FWS	2RFW2A-20E	2-003-036	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					EL
FWS	2RFW2A-21P	2-003-037	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					P
FWS	2RFW2A-22E	2-003-037	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					EL
FWS	2RFW2A-24E	2-003-037	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					EL
FWS	2RFW2A-33E	2-003-038	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26					EL
FWS	2RFW2A-39E	2-003-039	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12	1.012		EL
FWS	2RFW2A-41E	2-003-039	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12	0.844		EL
FWS	2RFW2B-22E	2-003-041	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12.75	0.844		EL
FWS	2RFW2B-36E	2-003-042	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12.75	0.844		EL
FWS	2RFW2B-41E	2-003-043	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12.75	0.844		EL
FWS	2RFW2B-44E	2-003-043	2-ISI-0269-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		12.75	0.844		EL
HPCIS	2-47B455H0061-1A		2-ISI-0130-C-01	14	C3.20	C-C	96E-03	MT	N-MT-6			0.500		WLD ATT
HPCIS	2-47B455S0010-1A		2-ISI-0130-C-02	14	C3.20	C-C	96E-03	MT	N-MT-6			0.500		WLD ATT
MSS	2-47B400S0006		2-ISI-0279-C-02	14	F1.10B	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0007		2-ISI-0279-C-02	14	F1.10B	F-A	96E-03	VT-3	N-VT-1		26.00			RGD HGR

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
MSS	2-47B400S0007		2-ISI-0279-C-02	14	F1.10B	F-A	P95-96	VT-3	N-VT-1		26.00			RGD HGR
MSS	2-47B400S0010		2-ISI-0279-C-01	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0010-IA		2-ISI-0279-C-01	14	B10.20	B-K	96E-03	MT	N-MT-6			0.578		WLD ATT
MSS	2-47B400S0018		2-ISI-0279-C-02	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0021-IA		2-ISI-0279-C-01	14	B10.20	B-K	96E-03	MT	N-MT-6			1.625		WLD ATT
MSS	2-47B400S0022		2-ISI-0279-C-01	14	F1.10C	F-A	96E-03	VT-3	N-VT-1		26.00			VAR SUP
MSS	2-47B400S0096		2-ISI-0279-C-01	14	F1.10D	F-A	96E-03	VT-3	N-VT-1		26.00			SNBR
MSS	2-47B400S0105-IA		2-ISI-0279-C-02	14	B10.20	B-K	96E-03	MT	N-MT-6			1.250		WLD ATT
MSS	2-47B400S0107		2-ISI-0279-C-02	14	F1.10D	F-A	96E-03	VT-3	N-VT-1		26.00			SNBR
MSS	2MSZ-MS1A-6E	2-001-036	2-ISI-0222-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		26	1.012		EL
MSS	2MSZ-MS1D-4E	2-001-039	2-ISI-0222-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		26	1.012		EL
MSS	2MSZ-MS1D-9FN	2-001-039	2-MSG-0021-C-01	14	R1.18	R-A	96E-03	UT	N-UT-26		26	1.012		NOZ BR
MSS	2MSZ-MS2B-9FN	2-001-037	2-ISI-0222-C-02	14	R1.18	R-A	96E-03	UT	N-UT-26		26	1.012		NOZ BR
MSS	FCV-01-037-BC		2-ISI-0222-C-02	14	B7.70	B-G-2	P95-96	VT-1	N-VT-1					BLTG
MSS	FCV-01-037-BC		2-ISI-0222-C-02	14	B7.70	B-G-2	V01-02	VT-1	N-VT-1					BLTG
MSS	PCV1-2-030		2-ISI-0312-B-01	14	B12.50	B-M-2	96E-03	VT-3	N-VT-1		26.00			INT
RCICS	2-47B456H0027		2-ISI-0131-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		06.00			VAR SUP
RCICS	RCICH-2-1		2-ISI-0131-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1					PUMP
RCICS	RCICH-2-2		2-ISI-0131-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1					PUMP
RECIR	2-47B408S0063		2-ISI-0278-C-01	14	F1.40D	F-A	96E-03	VT-3	N-VT-1					SNBR
RECIR	2-47B408S0067-IE		2-ISI-0278-C-01	14	F1.40D	F-A	96E-03	VT-3	N-VT-1					SNBR
RECIR	GR-2-15(OL)	2-068-006	2-ISI-0270-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-66	BF-133	12.00	1.139		SDL P
RECIR	GR-2-15(OL)	2-068-006	2-ISI-0270-C-01	14	NU0313	E	B02-02	UT	N-UT-66	BF-133	12.00	1.139		SDL P
RECIR	GR-2-38	2-068-010	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-38	2-068-010	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-41	2-068-011	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-41	2-068-011	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-48	2-068-013	2-ISI-0270-C-02	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-48	2-068-013	2-ISI-0270-C-02	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	12.00	0.569		SDL P
RECIR	GR-2-53	2-068-016	2-ISI-0270-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-64	ALTSS	28.00	1.138		SE P
RECIR	GR-2-53	2-068-016	2-ISI-0270-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS	28.00	1.138		SE P
RECIR	PMP-2-1B		2-ISI-0407-C-01	14	B12.20	B-L-2	96E-03	VT-3	N-VT-1					INT
RECIR	PMP-B-FLG		2-ISI-0407-C-01	14	B6.190	B-G-1	96E-03	VT-1	N-VT-1					FLG SUR
RECIR	PMP-B-NUT-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RECIR	PMP-B-NUT-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-NUT-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PNUTS
RECIR	PMP-B-STUD-2-01		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-02		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-03		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-04		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-05		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-06		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-07		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-08		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-09		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-10		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-11		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-12		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-13		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-14		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-15		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-STUD-2-16		2-ISI-0407-C-01	14	B6.180	B-G-1	96E-03	UT	N-UT-67		03.00			BLTG
RECIR	PMP-B-WASH-2-01		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-02		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-03		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-04		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-05		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-06		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-07		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-08		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-09		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-10		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-11		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-12		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-13		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-14		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH
RECIR	PMP-B-WASH-2-15		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00			PWASH

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
RECIR	PMP-B-WASH-2-16		2-ISI-0407-C-01	14	B6.200	B-G-1	96E-03	VT-1	N-VT-1		03.00		PWASH	
RHRS	2-47B452H0015		2-ISI-0324-C-08	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		24.00		RGD STRT	
RHRS	2-47B452H0022		2-ISI-0324-C-09	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		24.00		RGD STRT	
RHRS	2-47B452H0035		2-ISI-0324-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0037		2-ISI-0324-C-01	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		20.00		RGD STRT	
RHRS	2-47B452H0064		2-ISI-0324-C-05	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0064-IA		2-ISI-0324-C-05	14	C3.20	C-C	96E-03	MT	N-MT-6			1.000	WLD ATT	
RHRS	2-47B452H0069		2-ISI-0324-C-07	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0088		2-ISI-0324-C-02	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0089		2-ISI-0324-C-02	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0106		2-ISI-0324-C-03	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		14.00		RGD HGR	
RHRS	2-47B452H0126		2-ISI-0324-C-07	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0158		2-ISI-0324-C-01	14	F1.20C	F-A	96E-03	VT-3	N-VT-1		20.00		VAR SUP	
RHRS	2-47B452H0158-IA		2-ISI-0324-C-01	14	C3.20	C-C	96E-03	MT	N-MT-6			0.237	WLD ATT	
RHRS	2-47B452R0054		ISI-0324-C-06	14	F1.20D	F-A	96E-03	VT-3	N-VT-1		20.00		SNBR	
RHRS	2-47B452R0059		2-ISI-0324-C-05	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		20.00		RGD HGR	
RHRS	2-47B452R0064		2-ISI-0324-C-07	14	F1.20A	F-A	96E-03	VT-3	N-VT-1		20.00		RGD HGR	
RHRS	2-47B452R0064-IA		2-ISI-0324-C-07	14	C3.20	C-C	96E-03	MT	N-MT-6			0.500	WLD ATT	
RHRS	2-47B452S0308		2-ISI-0324-C-10	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		06.00		RGD HGR	
RHRS	DRHR-2-03B	2-074-005	2-ISI-0221-C-01	14	R1.16G	R-A	96E-03	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-03B	2-074-005	2-ISI-0221-C-01	14	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-09		2-ISI-0221-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS	24.00	1.219	P	P
RHRS	DRHR-2-12		2-ISI-0221-C-01	14	NU0313	D	B02-02	UT	N-UT-64	ALTSS	24.00	1.219	VLV	P
RHRS	DRHR-2-13B	2-074-013	2-ISI-0221-C-01	14	R1.16G	R-A	96E-03	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-13B	2-074-013	2-ISI-0221-C-01	14	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	P	P
RHRS	DRHR-2-22		2-ISI-0221-C-01	14	NU0313	E	B02-02	UT	N-UT-65	ALTSS	20.00	1.031	VLV	P
RHRS	DSRHR-2-04		2-ISI-0221-C-01	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	24.00	1.219	P	EL
RHRS	DSRHR-2-04		2-ISI-0221-C-01	14	TS3432	B-J	B04-02	UT	N-UT-64	ALTSS	24.00	1.219	P	EL
RHRS	RHR-2-H-343-IE		2-ISI-0324-C-08	14	F1.40C	F-A	96E-03	VT-3	N-VT-1		18.00		RGD HGR	
RHRS	RHR-2-R-223		2-ISI-0324-C-02	14	F1.20B	F-A	96E-03	VT-3	N-VT-1		24.00		RGD HGR	
RHRS	RHRG-2-05-A		2-ISI-0406-C-01	14	C2.33	C-B	96E-03	VT-2	N-VT-4		10.50	0.875	NOZ	
RHRS	RHRG-2-05A-A		2-ISI-0406-C-01	14	C2.31	C-B	96E-03	MT	N-MT-6		24	0.875	SHL	NOZ
RHRS	RHRG-2-05B-A		2-ISI-0406-C-01	14	C2.31	C-B	96E-03	MT	N-MT-6		24	0.875	NOZ	SHL
RHRS	RHRG-2-06-A		2-ISI-0406-C-01	14	C2.33	C-B	96E-03	VT-2	N-VT-4		10.50	0.875	NOZ	
RHRS	RHRG-2-09-A		2-ISI-0406-C-01	14	C1.10	C-A	96E-03	UT	N-UT-18	BF-40	54.00	0.875	HD	FLG
RHRS	RHRG-2-12-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-13-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-14-A		2-ISI-0406-C-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HVES	
RHRS	RHRPH-2-B		ISI-0310-B-01	14	F1.40B	F-A	96E-03	VT-3	N-VT-1				HPMP	
RHRS	TRHR-2-281	2-074-031	MSG-0018-C-02	14	R1.11	R-A	96E-03	UT	N-UT-76	ALTCS	24.00	0.375	VLV	EL

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RHRS	TRHR-2-295	2-074-030	MSG-0018-C-02	14	R1.11	R-A	96E-03	UT	N-UT-76	ALTCS	24.00	0.375	VLV	EL
RHRWSW	2-17B300S0070		2-ISI-0145-C-01	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		16.00		RGD HGR	
RHRWSW	2-47B450R0027		2-ISI-0145-C-02	14	F1.30B	F-A	96E-03	VT-3	N-VT-1		16.00		RGD HGR	
RHRWSW	2-47B450R0028-1A		2-ISI-0145-C-02	14	D1.20	D-A	96E-03	VT-1	N-VT-1			1.00	WLD ATT	
RPV	N10-SE		2-ISI-0380-C-01	14	N/A	BWRVIP-27	B07-02	VT-2	N-VT-4		1.5	0.218	SE	NOZ
RPV	N11A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N11B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N12A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	SE	P
RPV	N12B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	P	PC,PIPE
RPV	N-16A-SE		2-ISI-0383-C-01	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.218	NOZ	SE
RPV	N-16B-SE		2-ISI-0383-C-02	14	N/A	BWRVIP-49	B07-02	VT-2	N-VT-4		2.00	0.250	NOZ	SE
RPV	N1B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		28.00	6.600	NOZ IR	
RPV	N1B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	28.00	6.600	SHL	NOZ
RPV	N2A-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12	6.600	NOZ IR	
RPV	N2A-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2C-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2C-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2G-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2G-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N2H-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		12.00	6.600	NOZ IR	
RPV	N2H-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	12.00	6.600	NOZ	SHL
RPV	N3B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		26.00	6.600	NOZ IR	
RPV	N3B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	26.00	6.600	NOZ	SHL
RPV	N5A-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		10.00	6.600	NOZ IR	
RPV	N5A-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	10.00	6.6	NOZ	SHL
RPV	N5B-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		10.00	6.600	NOZ IR	
RPV	N5B-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	10.00	6.600	NOZ	SHL
RPV	N6A-2-1-BC		ISI-0408-C-01	14	B7.50	B-G-2	96E-03	VT-1	N-VT-1				BLTG	
RPV	N7-IR		ISI-0408-C-01	14	B3.100	B-D	96E-03	VT-1E	N-VT-1		04.00	4.340	NOZ IR	
RPV	N7-NV		ISI-0408-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-19	04.00	4.310	CL HD	NOZ
RPV	N9-IR		2-CHM-2046-C-01	14	B3.100	B-D	96E-03	VT-1E	VENDOR VT		04.00	6.600	NOZ IR	
RPV	N9-NV		2-CHM-2046-C-01	14	B3.90	B-D	96E-03	UT	N-UT-78	BF-18	04.00	6.600	NOZ	SHL
RPV	RCH-2-4V		ISI-0408-C-01	14	B1.22	B-A	96E-03	UT	N-UT-78	BF-19	-96L	4.000	VERT LS	
RPV	RCH-2-6V		ISI-0408-C-01	14	B1.22	B-A	96E-03	UT	N-UT-78	BF-19	-96L	4.00	VERT LS	
RPV	RPV-BUSH-2-22		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-23		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-24		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-BUSH-2-25		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1				BUSH	
RPV	RPV-INTERIOR		2-CHM-2046-C-02	14	B13.10	B-N-1	96E-03	VT-3	VENDOR VT				INT	
RPV	RPV-NUTS-2-31		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-NUTS-2-32		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-33		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-34		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-35		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-36		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-37		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-38		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-39		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-40		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-41		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-42		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-43		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-44		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-45		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-46		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-47		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-48		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-49		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-50		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-51		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-52		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-53		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-54		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-55		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-56		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-57		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-58		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-59		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-60		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-NUTS-2-61		ISI-0266-C-01	14	B6.10	B-G-1	96E-03	VT-1	N-VT-1		08.50		CL HD BLT	
RPV	RPV-STUDS-2-01		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-02		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-03		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-04		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-05		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-06		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-07		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-08		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-09		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	
RPV	RPV-STUDS-2-10		ISI-0266-C-01	14	B6.20	B-G-1	96E-03	UT	N-UT-67	BF-126	06.00	65.56	CL HD BLT	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIS	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-THR IN FLG-2-79		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-80		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-81		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-82		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-83		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-84		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-85		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-86		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-87		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-88		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-89		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-90		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-91		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-THR IN FLG-2-92		ISI-0266-C-01	14	B6.40	B-G-1	96E-03	UT	N-UT-67	BF-126	0.000	0.000	LIG	
RPV	RPV-WASH-2-31		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-32		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-33		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-34		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-35		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-36		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-37		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-38		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-39		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-40		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-41		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-42		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-43		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-44		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-45		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-46		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-47		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-48		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-49		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-50		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-51		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	BUSH	
RPV	RPV-WASH-2-52		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-53		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-54		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-55		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-56		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	

SYSTEM	WELDNO	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-WASH-2-57		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-58		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-59		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-60		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RPV	RPV-WASH-2-61		ISI-0266-C-01	14	B6.50	B-G-1	96E-03	VT-1	N-VT-1		06.00	08.62	WASH	
RWCUS	DRWC-2-01	2-069-001	2-ISI-0272-C-01	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	06.00	0.432	VLV	P
RWCUS	DRWC-2-01	2-069-001	2-ISI-0272-C-01	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	06.00	0.432	VLV	P
RWCUS	DRWC-2-01A	2-069-001	2-ISI-0272-C-01	14	R1.16C	R-A	96E-03	UT	N-UT-64	ALTSS	06.00	0.432	P	VLV
RWCUS	DRWC-2-01A	2-069-001	2-ISI-0272-C-01	14	NU0313	C	B02-02	UT	N-UT-64	ALTSS	06.00	0.432	P	VLV
RWCUS	DSRWC-2-03(OL)	2-069-001	2-ISI-0272-C-01	14	R1.16E	R-A	96E-03	UT	N-UT-66	BF-133	06.00	0.700	P	EL
RWCUS	DSRWC-2-03(OL)	2-069-001	2-ISI-0272-C-01	14	NU0313	E	B02-02	UT	N-UT-66	BF-133	06.00	0.700	P	EL
RWCUS	RCRD-2-49	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	EL
RWCUS	RCRD-2-49	2-085-031	2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	EL
RWCUS	RCRD-2-52	2-085-031	2-ISI-0272-C-01	14	R1.16D	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	P
RWCUS	RCRD-2-52	2-085-031	2-ISI-0272-C-01	14	NU0313	D	B02-02	UT	N-UT-82	BF-131/BF-132	4.00	0.531	VLV	P
RWCUS	RWCU-2-004-083	2-069-003	2-ISI-0272-C-01	14	R1.16A	R-A	96E-03	UT	N-UT-82	BF-131/BF-132	4.00	0.337	VLV	P

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CSS	2-47B458S0029	2-ISI-0280-C-01	96E-03	F-A	F1.10B	VT-3		20070225	R-058	P	
CSS	FCV-75-26	2-ISI-0271-C-01	96E-03	B-M-2	B12.50	VT-3		20070309	R-096	P	
CSS	FCV-75-26-BC	2-ISI-0271-C-01	96E-03	B-G-2	B7.70	VT-1		20070309	R-097	P	
EECWS	0-37B205S0053	ISI-0368-C-03	96E-03	F-A	F1.30A	VT-3		20070220	R-031	P	
EECWS	0-37B205S0055	ISI-0368-C-01	96E-03	F-A	F1.30A	VT-3		20070220	R-032	P	
EECWS	0-37B205S0056	ISI-0368-C-01	96E-03	F-A	F1.30A	VT-3		20070220	R-033	P	
EECWS	0-37B205S0064	ISI-0368-C-01	96E-03	F-A	F1.30B	VT-3		20070220	R-034	P	
EECWS	0-37B205S0067	ISI-0368-C-01	96E-03	F-A	F1.30A	VT-3		20070220	R-035	P	
EECWS	0-37B205S0071	ISI-0368-C-03	96E-03	F-A	F1.30B	VT-3		20070220	R-036	P	
EECWS	0-37B205S0072	ISI-0368-C-03	96E-03	F-A	F1.30A	VT-3		20070220	R-037	P	
EECWS	1-47B451H0017	ISI-0368-C-13	96E-03	F-A	F1.30B	VT-3		20070201	R-015	P	
EECWS	1-47B451R0013	ISI-0368-C-13	96E-03	F-A	F1.30B	VT-3		20070201	R-017	P	
EECWS	1-47B451R0039	ISI-0368-C-12	96E-03	F-A	F1.30B	VT-3		20070222	R-051	P	
EECWS	1-47B451R0040	ISI-0368-C-12	96E-03	F-A	F1.30B	VT-3		20070222	R-052	P	
EECWS	1-47B451R0045	ISI-0368-C-12	96E-03	F-A	F1.30B	VT-3		20070222	R-053	P	
EECWS	1-47B451S0055	ISI-0368-C-15	96E-03	F-A	F1.30B	VT-3		20070224	R-054	P	
EECWS	1-47B451S0290	ISI-0368-C-10	96E-03	F-A	F1.30B	VT-3		20070223	R-050	P	
EECWS	2-47B451R0009-IA	ISI-0368-C-06	96E-03	D-A	D1.20	VT-1		20070201	R-016	P	
FWS	2RFW2A-14R	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8729	20070319	R-134	P	RI-ISI
FWS	2RFW2A-17R	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	01-7476	20070316	R-133	P	RI-ISI
FWS	2RFW2A-20E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	01-7474	20070316	R-128	P	RI-ISI
FWS	2RFW2A-21P	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	01-7474	20070316	R-129	P	RI-ISI

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
FWS	2RFW2A-24E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8730	20070316	R-131	P	RI-ISI
FWS	2RFW2A-39E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8729	20070319	R-135	P	RI-ISI
FWS	2RFW2A-41E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8729	20070320	R-127	P	RI-ISI
FWS	2RFW2B-22E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8730	20070323	R-145	P	RI-ISI
FWS	2RFW2B-36E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8730	20070324	R-148	P	RI-ISI
FWS	2RFW2B-41E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	048730	20070323	R-146	P	RI-ISI
FWS	2RFW2B-44E	2-ISI-0269-C-01	96E-03	R-A	R1.18	UT	04-8734/01-7474	20070324	R-147	P	RI-ISI
HPCIS	2-47B455-2123	ISI-0275-C-01	P95-96	F-A	F1.10D	VT-3		20070409	R-170	P	W.O. 00-003350-003. RFR# 2-ISI-13
HPCIS	2-47B455-2124	ISI-0275-C-01	P95-96	F-A	F1.10D	VT-3		20070411	R-172	P	W.O. 00-003350-002. RFR# 2-ISI-13
HPCIS	2-47B455H0061-1A	2-ISI-0130-C-01	96E-03	C-C	C3.20	MT		20070308	R-088	P	
HPCIS	HPCI-2-001-003	2-ISI-0273-C-01	P95-96	B-J	B9.11	MT		20070324	R-167	P	DCN# 50287 W.O. 00-003350-000
HPCIS	HPCI-2-001-003	2-ISI-0273-C-01	P95-96	B-J	B9.11	UT	SQ-115	20070324	R-149	P	W.O. 00-003350-000
HPCIS	HPCI-2-001-004	2-ISI-0273-C-01	P95-96	B-J	B9.11	PT		20070123	R-013	P	W.O. 00-003350-000. DCN# 50287A. VALVE 2-FCV-073-2.
HPCIS	HPCI-2-001-004	2-ISI-0273-C-01	P95-96	B-J	B9.11	UT	ALTCS/WB78	20070202	R-040	P	W.O. 00-003350-000 DCN#50287
HPCIS	HPCI-2-001-005	2-ISI-0273-C-01	P95-96	B-J	B9.11	MT		20070324	R-168	P	DCN# 50287 W.O. 00-003350-000
HPCIS	HPCI-2-001-005	2-ISI-0273-C-01	P95-96	B-J	B9.11	UT	SQ-115	20070324	R-150	P	W.O. 00-003350-000
MSS	2-47B400S0006	2-ISI-0279-C-02	P95-96	F-A	F1.10B	VT-3		20070307	R-083	P	IWF-2220 (b)
MSS	2-47B400S0007	2-ISI-0279-C-02	P95-96	F-A	F1.10B	VT-3		20070307	R-084	P	IWF-2220 (b)
MSS	2-47B400S0007	2-ISI-0279-C-02	96E-03	F-A	F1.10B	VT-3		20070307	R-084	P	Reference IWF-2220(b)
MSS	2-47B400S0010	2-ISI-0279-C-01	96E-03	F-A	F1.10C	VT-3		20070311	R-098	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	2-47B400S0010-IA	2-ISI-0279-C-01	96E-03	B-K	B10.20	MT		20070312	R-104	P	
MSS	2-47B400S0018	2-ISI-0279-C-02	96E-03	F-A	F1.10C	VT-3		20070309	R-099	P	
MSS	2-47B400S0021-IA	2-ISI-0279-C-01	96E-03	B-K	B10.20	MT		20070312	R-105	P	
MSS	2-47B400S0022	2-ISI-0279-C-01	96E-03	F-A	F1.10C	VT-3		20070309	R-100	P	
MSS	2-47B400S0038	2-ISI-0279-C-01	P95-96	F-A	F1.10A	VT-3		20070306	R-089	P	WO 06-711366-018
MSS	2-47B400S0039	2-ISI-0279-C-02	P95-96	F-A	F1.10A	VT-3		20070306	R-090	P	W.O. 06-711366-018
MSS	2-47B400S0040	2-ISI-0279-C-02	P95-96	F-A	F1.10A	VT-3		20070306	R-091	P	W.O. 06-711366-018
MSS	2-47B400S0041	2-ISI-0279-C-01	P95-96	F-A	F1.10B	VT-3		20070306	R-092	P	W.O. 06-711366-018
MSS	2-47B400S0096	2-ISI-0279-C-01	96E-03	F-A	F1.10D	VT-3		20070309	R-094	P	RRFR# 2-ISI-13
MSS	2-47B400S0105-IA	2-ISI-0279-C-02	96E-03	B-K	B10.20	MT		20070311	R-093	P	
MSS	2-47B400S0107	2-ISI-0279-C-02	96E-03	F-A	F1.10D	VT-3		20070309	R-095	P	RRFR# 2-ISI-13
MSS	2-47B400S0110	2-ISI-0279-C-01	P95-96	F-A	F1.10D	VT-3		20070402	R-158	P	DCN# 51303 W.O. 02-011925-001. RRFR# 2-ISI-13
MSS	2-47B400S0110-IA	2-ISI-0279-C-01	P95-96	B-K	B10.20	PT		20070312	R-162	P	Examinations performed 03/12/07 to 03/26/07. DCN# 51303 W.O. 02-011925-001
MSS	2MSZ-MS1A-6E	2-ISI-0222-C-01	96E-03	R-A	R1.18	UT	01-7476	20070316	R-130	P	RI-ISI
MSS	2MSZ-MS1D-4E	2-ISI-0222-C-01	96E-03	R-A	R1.18	UT	04-8730	20070316	R-132	P	RI-ISI
MSS	2-SI-3.3.1.C	N/A	96E-03	C-H	C7.30	VT-2		20070220	R-169	P	
MSS	FCV-01-015	2-ISI-0222-C-01	96E-03	B-M-2	B12.50	VT-3		20070301	R-071	P	VALVE BODY INTERNALS AND BONNET SURFACE
MSS	FCV-01-015-BC	2-ISI-0222-C-01	96E-03	B-G-2	B7.70	VT-1		20070301	R-072	P	
MSS	FCV-01-037	2-ISI-0222-C-02	P95-96	B-M-2	B12.50	VT-3		20070320	R-141	P	WELD MS-2-003-003 CORO, valve Guide and valve internals.
MSS	FCV-01-037-BC	2-ISI-0222-C-02	P95-96	B-G-2	B7.70	VT-1		20061016	R-009	P	EXAMINER JOSE ALEJANDRO 10/17/06. W.O. 06-723923-001. 24 NUTS AND 8 STUDS IN POSITIONS# 24, 18, 12, 6, 7, 13, 17, & 11.

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RCICS	2-47B456H0027	2-ISI-0131-C-01	96E-03	F-A	F1.20C	VT-3		20070205	R-019	P	
RCICS	RCICH-2-1	2-ISI-0131-C-01	96E-03	F-A	F1.40B	VT-3		20070205	R-018	P	
RCICS	RCICH-2-2	2-ISI-0131-C-01	96E-03	F-A	F1.40B	VT-3		20070205	R-022	P	
RECIR	2-47B408S0063	2-ISI-0278-C-01	96E-03	F-A	F1.40D	VT-3		20070307	R-087	P	RFR# 2-ISI-13
RECIR	2-47B408S0067-IE	2-ISI-0278-C-01	96E-03	F-A	F1.40D	VT-3		20070307	R-085	P	RFR# 2-ISI-13
RECIR	2-47B408S0069-IE	2-ISI-0278-C-01	P95-96	F-A	F1.40D	VT-3		20070323	R-144	P	W.O. 04-718365-000. RFR# 2-ISI-13
RECIR	GR-2-15(OL)	2-ISI-0270-C-01	96E-03	R-A	R1.16E	UT	BF-133	20070311	R-115	P	
RECIR	GR-2-38	2-ISI-0270-C-02	96E-03	R-A	R1.16C	UT	ALTSS/ WB85	20070313	R-109	P	
RECIR	GR-2-41	2-ISI-0270-C-02	96E-03	R-A	R1.16C	UT	ALTSS/S Q123	20070313	R-112	P	
RECIR	GR-2-48	2-ISI-0270-C-02	96E-03	R-A	R1.16C	UT	ALTSS	20070310	R-114	P	
RECIR	GR-2-53	2-ISI-0270-C-01	96E-03	R-A	R1.16E	UT	ALTSS/ WB85	20070310	R-106	P	
RECIR	PMP-2-1B	2-ISI-0407-C-01	96E-03	B-L-2	B12.20	VT-3		20070313	R-110	P	
RECIR	PMP-A-NUT-2-01	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-02	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-03	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-04	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-05	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-06	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070200	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-07	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-08	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-A-NUT-2-09	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-10	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-11	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-12	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-13	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-14	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-15	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-NUT-2-16	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-01	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-02	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-03	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-04	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-05	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-06	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-07	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-08	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-09	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-10	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-11	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-12	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-13	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-14	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-A-WASH-2-15	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-A-WASH-2-16	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-021	P	W.O. 06-718765-006
RECIR	PMP-B-FLG	2-ISI-0407-C-01	96E-03	B-G-1	B6.190	VT-1		20070313	R-111	P	PUMP FLANGE AND 1" ANNULAR FROM BOLT HOLES.
RECIR	PMP-B-NUT-2-01	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-01	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-02	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-02	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-03	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-03	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-04	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-04	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-05	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-05	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-06	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-06	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-07	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-07	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-08	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-08	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-09	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-09	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-B-NUT-2-10	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-10	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-11	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-11	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-12	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-12	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-13	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-13	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-14	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-14	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-15	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-NUT-2-15	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-16	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-NUT-2-16	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-102	P	
RECIR	PMP-B-STUD-2-01	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-02	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-03	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-04	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-05	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-06	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-07	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-08	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-B-STUD-2-09	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-10	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-11	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-12	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-13	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-14	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-15	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-STUD-2-16	2-ISI-0407-C-01	96E-03	B-G-1	B6.180	UT	BF-119	20070312	R-103	P	
RECIR	PMP-B-WASH-2-01	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-01	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-02	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-02	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-03	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-03	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-04	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-04	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-05	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-05	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-06	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-06	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-07	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-07	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-B-WASH-2-08	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-08	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-09	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-09	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-10	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-10	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-11	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-11	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-12	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-12	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-13	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-13	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-14	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-14	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-15	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-15	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	
RECIR	PMP-B-WASH-2-16	2-ISI-0407-C-01	P95-96	B-G-1	B6.200	VT-1		20070206	R-020	P	W.O. 06-718735-006
RECIR	PMP-B-WASH-2-16	2-ISI-0407-C-01	96E-03	B-G-1	B6.200	VT-1		20070313	R-101	P	W.O. 06-718735-006
RECIR	RBC-2-1	2-ISI-0270-C-01	96E-03	B-G-2	B7.50	VT-1		20070225	R-061	P	Code Case N-652
RHRS	2-47B452H0015	2-ISI-0324-C-08	96E-03	F-A	F1.20A	VT-3		20070221	R-046	P	
RHRS	2-47B452H0022	2-ISI-0324-C-09	96E-03	F-A	F1.20A	VT-3		20070225	R-056	P	
RHRS	2-47B452H0035	2-ISI-0324-C-01	96E-03	F-A	F1.20C	VT-3		20070207	R-024	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	2-47B452H0037	2-ISI-0324-C-01	96E-03	F-A	F1.20B	VT-3		20070221	R-042	P	
RHRS	2-47B452H0064	2-ISI-0324-C-05	96E-03	F-A	F1.20C	VT-3		20070221	R-047	P	
RHRS	2-47B452H0064-IA	2-ISI-0324-C-05	96E-03	C-C	C3.20	MT		20070223	R-055	P	
RHRS	2-47B452H0069	2-ISI-0324-C-07	96E-03	F-A	F1.20C	VT-3		20070302	R-070	P	
RHRS	2-47B452H0069-IA	2-ISI-0324-C-07	96E-03	C-C	C3.20	MT		20070302	R-073	P	
RHRS	2-47B452H0088	2-ISI-0324-C-02	96E-03	F-A	F1.20C	VT-3		20070225	R-059	P	
RHRS	2-47B452H0089	2-ISI-0324-C-02	96E-03	F-A	F1.20C	VT-3		20070225	R-060	P	
RHRS	2-47B452H0106	2-ISI-0324-C-03	96E-03	F-A	F1.20A	VT-3		20070221	R-048	P	
RHRS	2-47B452H0126	2-ISI-0324-C-07	96E-03	F-A	F1.20C	VT-3		20070302	R-069	P	
RHRS	2-47B452H0158	2-ISI-0324-C-01	96E-03	F-A	F1.20C	VT-3		20070207	R-023	P	
RHRS	2-47B452H0158-IA	2-ISI-0324-C-01	96E-03	C-C	C3.20	MT		20070207	R-025	P	
RHRS	2-47B452R0054	ISI-0324-C-06	96E-03	F-A	F1.20D	VT-3		20070225	R-057	P	RFR# 2-ISI-13
RHRS	2-47B452R0059	2-ISI-0324-C-05	96E-03	F-A	F1.20A	VT-3		20070221	R-044	P	
RHRS	2-47B452R0064	2-ISI-0324-C-07	96E-03	F-A	F1.20A	VT-3		20070302	R-065	P	
RHRS	2-47B452S0308	2-ISI-0324-C-10	96E-03	F-A	F1.20B	VT-3		20070221	R-045	P	
RHRS	DRHR-2-03	2-ISI-0221-C-01	A14-03	R-A	R1.16D	UT	SQ-123	20070310	R-121	P	EXPANDED SCOPE
RHRS	DRHR-2-03B	2-ISI-0221-C-01	96E-03	R-A	R1.16G	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-13B	2-ISI-0221-C-01	96E-03	R-A	R1.16G	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	RHR-2-H-343-IE	2-ISI-0324-C-08	96E-03	F-A	F1.40C	VT-3		20070221	R-043	P	
RHRS	RHR-2-R-223	2-ISI-0324-C-02	96E-03	F-A	F1.20B	VT-3		20070226	R-063	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	RHRG-2-05-A	2-ISI-0406-C-01	96E-03	C-B	C2.33	VT-2		20060901	R-008	P	SEE REPORT# R-008 FOR EXAM RESULTS AND M&TE.
RHRS	RHRG-2-05A-A	2-ISI-0406-C-01	96E-03	C-B	C2.31	MT		20070307	R-081	P	
RHRS	RHRG-2-05B-A	2-ISI-0406-C-01	96E-03	C-B	C2.31	MT		20070307	R-082	P	
RHRS	RHRG-2-06-A	2-ISI-0406-C-01	96E-03	C-B	C2.33	VT-2		20060901	R-008	P	SEE REPORT# R-008 FOR EXAM RESULTS AND M&TE.
RHRS	RHRG-2-09-A	2-ISI-0406-C-01	96E-03	C-A	C1.10	UT	BF-40	20070305	R-077	P	
RHRS	RHRG-2-12-A	2-ISI-0406-C-01	96E-03	F-A	F1.40B	VT-3		20070305	R-076	P	
RHRS	RHRG-2-13-A	2-ISI-0406-C-01	96E-03	F-A	F1.40B	VT-3		20070305	R-075	P	
RHRS	RHRG-2-14-A	2-ISI-0406-C-01	96E-03	F-A	F1.40B	VT-3		20070305	R-028	P	
RHRS	RHRPH-2-B	ISI-0310-B-01	96E-03	F-A	F1.40B	VT-3		20070226	R-062	P	
RHRS	TRHR-2-281	MSG-0018-C-02	96E-03	R-A	R1.11	UT	ALTCS/ WB78	20070220	R-038	P	
RHRS	TRHR-2-295	MSG-0018-C-02	96E-03	R-A	R1.11	UT	ALTCS/ WB78	20070101	R-039	P	
RHRWSW	2-17B300S0070	2-ISI-0145-C-01	96E-03	F-A	F1.30B	VT-3		20070201	R-014	P	
RHRWSW	2-47B450R0027	2-ISI-0145-C-02	96E-03	F-A	F1.30B	VT-3		20070213	R-026/026 A	P	R-026A, SUPPORT EXAMINED ON 03/05/07 BY James E. Rogers
RHRWSW	2-47B450R0028-1A	2-ISI-0145-C-02	96E-03	D-A	D1.20	VT-1		20070213	R-027/027 A	P	R-027A, SUPPORT EXAMINED ON 03/05/07 BY James E. Rogers
RHRWSW	2-47B450R0038	2-ISI-0145-C-03	P95-96	F-A	F1.30B	VT-3		20070401	R-159	P	W.O. 06-718765-017
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.10	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.50	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.60	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.70	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	C-H	C7.30	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	C-H	C7.70	VT-2		20070408	R-163	P	SEE REPORT# R-163 FOR EXAM RESULTS AND M&TE.
RPV	N1B-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N1B-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070318	R-137	P	See ISI Report R-137 for all contractor M&TE.
RPV	N2A-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N2A-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070319	R-138	P	See ISI Report R-138 for all contractor M&TE.
RPV	N2B-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070319	R-142	P	EXPANDED SAMPLE. See report R-142 for M&TE.
RPV	N2C-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N2C-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070319	R-139	P	See ISI Report R-139 for all contractor M&TE.
RPV	N2D-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070322	R-173	P	EXPANDED SCOPE. See report R-173 for contractor M&TE.
RPV	N2G-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N2G-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070313	R-136	P	See ISI Report R-136 for all contractor M&TE.
RPV	N2H-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N2H-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070320	R-143	P	See report R-143 for contractor M&TE.

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	N2K-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070321	R-174	P	EXPANDED SCOPE. See Report R-174 for contractor M&TE.
RPV	N3A-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070321	R-151	P	EXPANDED SCOPE. See report R-151 for contractor M&TE.
RPV	N3B-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N3B-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070315	R-175	P	See report R-175 for contractor M&TE.
RPV	N3C-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070322	R-152	P	EXPANDED SCOPE. See Report R-152 for contractor M&TE.
RPV	N4B-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070417	R-176	P	EXPANDED SCOPE. See report R-176 for contractor M&TE.
RPV	N4C-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070417	R-177	P	EXPANDED SCOPE. See report R-177 for contractor M&TE.
RPV	N4E-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070417	R-178	P	EXPANDED SCOPE. See report R-178 for contractor M&TE.
RPV	N4F-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070417	R-179	P	EXPANDED SCOPE. See report R-179 for contractor M&TE.
RPV	N5A-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N5A-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070314	R-156	P	See Report R-156 for contractor M&TE.
RPV	N5B-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N5B-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070323	R-157	P	See Report R-157 for contractor M&TE.
RPV	N6A-2-1-BC	ISI-0408-C-01	96E-03	B-G-2	B7.50	VT-1		20070301	R-068	P	
RPV	N7-IR	ISI-0408-C-01	96E-03	B-D	B3.100	VT-1E		20070302	R-064	P	
RPV	N7-NV	ISI-0408-C-01	96E-03	B-D	B3.90	UT	BF-19	20070302	R-080	P	See report R-080 for contractor M&TE.

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	N8B-NV	2-CHM-2046-C-01	A15-03	B-D	B3.90	UT	BF-18	20070322	R-180	P	EXPANDED SCOPE. See report R-180 for contractor M&TE.
RPV	N9-IR	2-CHM-2046-C-01	96E-03	B-D	B3.100	VT-1E		20070316	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	N9-NV	2-CHM-2046-C-01	96E-03	B-D	B3.90	UT	BF-18	20070315	R-117	F	See ISI Report R-117 for all contractor M&TE. Indication in RPV Nozzle N9-NV Weld " ACCEPTED AS IS" by Engineering. NOI# U2C14-049 is closed.
RPV	RCH-2-4V	ISI-0408-C-01	96E-03	B-A	B1.22	UT	BF-19	20070307	R-078	P	See report R-078 for contractor M&TE.
RPV	RCH-2-6V	ISI-0408-C-01	96E-03	B-A	B1.22	UT	BF-19	20070301	R-079	P	See report R-079 for contractor M&TE.
RPV	RPV CORE SUPPORT	2-CHM-2046-C-02	96E-03	B-N-2	B13.40	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV INT ATT BLR	2-CHM-2046-C-02	96E-03	B-N-2	B13.20	VT-1		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV INT ATT NBLR	2-CHM-2046-C-02	96E-03	B-N-2	B13.30	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV-BUSH-2-22	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070405	R-166	P	
RPV	RPV-BUSH-2-23	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070405	R-166	P	
RPV	RPV-BUSH-2-24	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070405	R-166	P	
RPV	RPV-BUSH-2-25	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070405	R-166	P	
RPV	RPV-INTERIOR	2-CHM-2046-C-02	96E-03	B-N-1	B13.10	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV-NUTS-2-31	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-32	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-33	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-NUTS-2-34	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-35	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-36	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-37	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-38	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-39	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-40	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-41	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-42	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-43	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-44	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-45	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-46	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-47	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-48	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-49	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-50	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-51	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-52	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-53	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-54	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-55	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE:14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-NUTS-2-56	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-57	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-58	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-59	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-60	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-NUTS-2-61	ISI-0266-C-01	96E-03	B-G-1	B6.10	VT-1		20070301	R-066	P	
RPV	RPV-STUDS-2-01	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-02	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-03	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-04	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-05	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-06	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-07	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-08	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-09	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-10	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-11	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-12	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-13	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-14	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-15	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-16	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-17	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-18	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-19	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-20	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-21	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-22	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070226	R-164	P	
RPV	RPV-STUDS-2-23	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070226	R-164	P	
RPV	RPV-STUDS-2-24	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070226	R-164	P	
RPV	RPV-STUDS-2-25	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070226	R-164	P	
RPV	RPV-STUDS-2-26	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-27	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-28	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-29	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-30	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-31	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-32	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-33	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-34	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-35	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-36	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-37	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-38	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-39	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-40	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-41	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-42	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-43	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-44	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-45	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-46	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-47	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-48	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-49	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-50	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-51	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-52	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-53	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-54	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-55	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-56	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-57	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-58	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-59	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-60	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-61	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-62	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-63	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-64	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-65	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-66	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-67	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-68	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-69	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-70	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-71	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-72	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-73	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-74	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-75	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-76	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-77	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-78	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-79	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-80	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-81	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-82	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-83	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-84	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-85	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-86	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-87	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-88	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-89	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-90	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-91	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-STUDS-2-92	ISI-0266-C-01	96E-03	B-G-1	B6.20	UT	BF-126	20070405	R-164	P	
RPV	RPV-THR IN FLG-2-01	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-02	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-03	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-04	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-05	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-06	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-07	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-08	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-09	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-10	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-11	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-12	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

**OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000**

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-THR IN FLG-2-13	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-14	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-15	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-16	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-17	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-18	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-19	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-20	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-21	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-22	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-23	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-24	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-25	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-26	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-27	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-28	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-29	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-30	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-31	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-32	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-33	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-34	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-THR IN FLG-2-35	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-36	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-37	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-38	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-39	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-40	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-41	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-42	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-43	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-44	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-45	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-46	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-47	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-48	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-49	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-50	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-51	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-52	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-53	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-54	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-55	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-56	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	

Exam Requirements:

96E-03

P95-96

A14-03

A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-THR IN FLG-2-57	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-58	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-59	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-60	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-61	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-62	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-63	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-64	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-65	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-66	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-67	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-68	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-69	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-70	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-71	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-72	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-73	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-74	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-75	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-76	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-77	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-78	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-THR IN FLG-2-79	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-80	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-81	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-82	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-83	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-84	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-85	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-86	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-87	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-88	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-89	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-90	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-91	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-THR IN FLG-2-92	ISI-0266-C-01	96E-03	B-G-1	B6.40	UT	BF-126	20070405	R-165	P	
RPV	RPV-WASH-2-31	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-32	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-33	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-34	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-35	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-36	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-37	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-38	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-WASH-2-39	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-40	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-41	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-42	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-43	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-44	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-45	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-46	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-47	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-48	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-49	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-50	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-51	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-52	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-53	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-54	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-55	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-56	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-57	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-58	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-59	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RPV	RPV-WASH-2-60	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	

Exam Requirements:

96E-03
P95-96
A14-03
A15-03

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-WASH-2-61	ISI-0266-C-01	96E-03	B-G-1	B6.50	VT-1		20070301	R-067	P	
RWCUS	69-500	2-ISI-0272-C-01	96E-03	B-M-2	B12.50	VT-3		20070320	R-140	P	W.O.06-723036-000. NOI 3 U2C14-050 CLEARED. Valve 69-500 removed from service and pipe installed.
RWCUS	CRD-2-005-003	2-ISI-0272-C-01	P95-96	R-A	R1.16D	UT	SQ-116	20070401	R-171	P	W.O. 07-713160-000
RWCUS	DSRWC-2-03(OL)	2-ISI-0272-C-01	96E-03	R-A	R1.16E	UT	BF-133	20070311	R-113	P	
RWCUS	RCRD-2-49	2-ISI-0272-C-01	96E-03	R-A	R1.16D	UT	BF-131/BF-132	20070228	R-126	P	
RWCUS	RCRD-2-50	2-ISI-0272-C-01	A14-03	R-A	R1.16D	UT	BF-132	20070311	R-118	P	EXPANDED SCOPE
RWCUS	RCRD-2-52	2-ISI-0272-C-01	96E-03	R-A	R1.16D	UT	BF-131/BF-132	20070303	R-074	F	WELD CUT-OUT AND REPLACED WITH NEW VALVE AND PIPE, WELD CRD-2-005-003, W.O. 07-713160-000. Preservice UT exam performed on new weld on 04/10/07 report R-171.
RWCUS	RWCU-2-003-069	2-ISI-0272-C-01	P95-96	B-J	B9.11	PT		20070330	R-160	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-069	2-ISI-0272-C-01	P95-96	B-J	B9.11	UT	SQ-116	20070330	R-153	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-070	2-ISI-0272-C-01	P95-96	B-J	B9.11	PT		20070329	R-160	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-070	2-ISI-0272-C-01	P95-96	B-J	B9.11	UT	SQ-116	20070330	R-154	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-071	2-ISI-0272-C-01	P95-96	B-J	B9.11	PT		20070330	R-160	P	DCN#6888A W.O. 06-723035-001
RWCUS	RWCU-2-003-071	2-ISI-0272-C-01	P95-96	B-J	B9.11	UT	SQ-116	20070330	R-155	P	DCN#6888A W.O. 06-723035-001
RWCUS	RWCU-2-004-083	2-ISI-0272-C-01	96E-03	R-A	R1.16A	UT	BF-131/BF-132	20070228	R-123	P	

REFERENCE DRAWINGS
 47W455-H-6.7.2C
 ISI-0128-C (SH.1) LD MAP

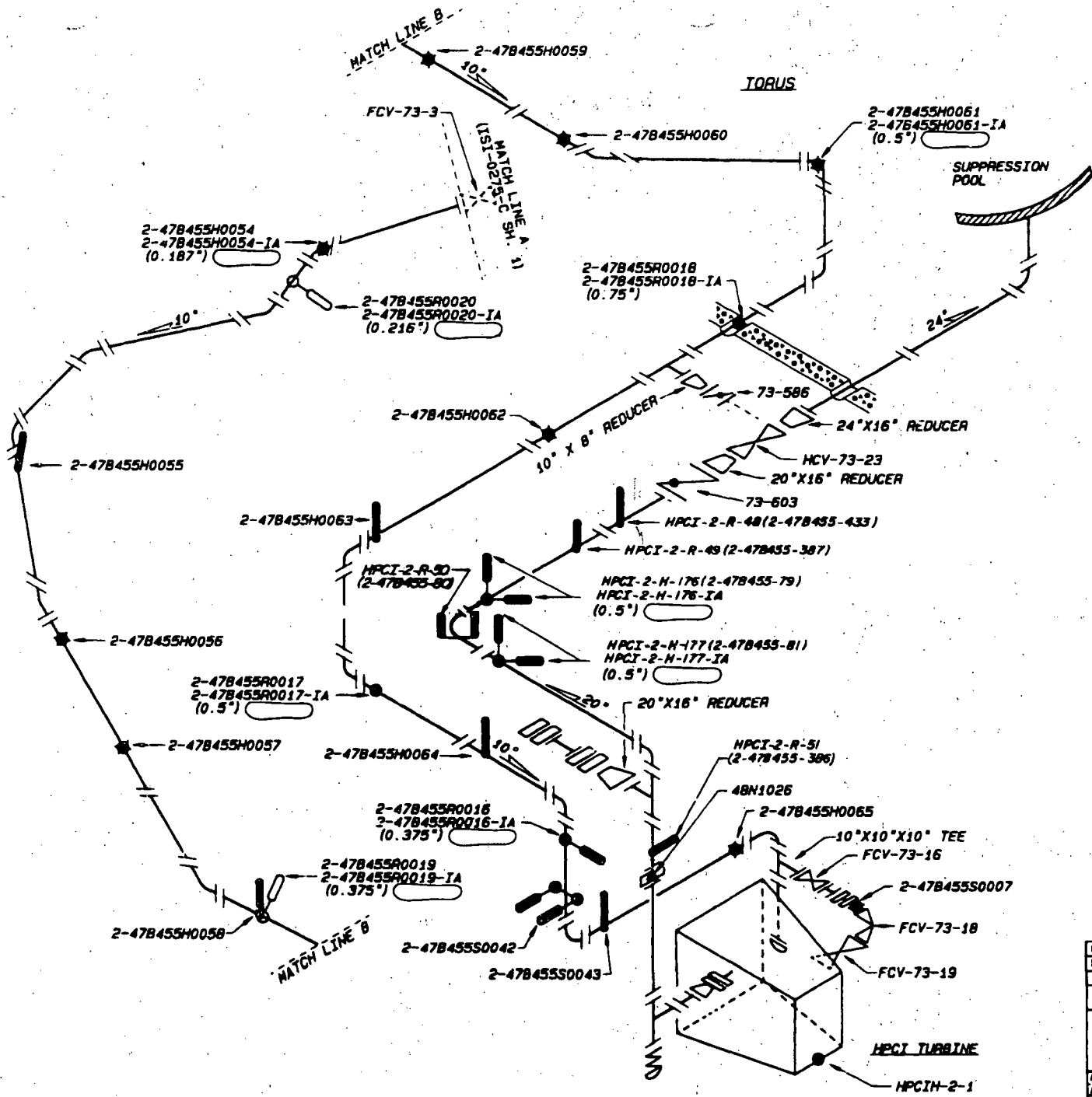
LEGEND

 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER
 ANCHOR
 RIGID STRUT

CALCULATION BRANCH
 PROJECT IDENTIFIER:
 CD-Q2073-88990

ASME CC-2 (EQUIVALENT)

NOTE
 1. SUPPORT DRAWING NUMBERS SHOWN IN "I" DRAWING MAY HAVE MULTIPLE SHEET NUMBERS.



001	ADMIN	RGP	ALB	HEH	5-29-2001
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT # 2					
HIGH PRESSURE COOLANT INJECTION SYSTEM					
SUPPORT LOCATIONS					
DATE	REV	SUBMITTED	APPROVED	SCALE	PTS
DATE	DATE	DATE	DATE	DATE	DATE
DESIGNED BY	JJK	DATE	GLB	DRAWING NO.	REV
DATE				2-47B455-0128-C	001

ALL A/D HISTORY RESEARCHED TO ROOT

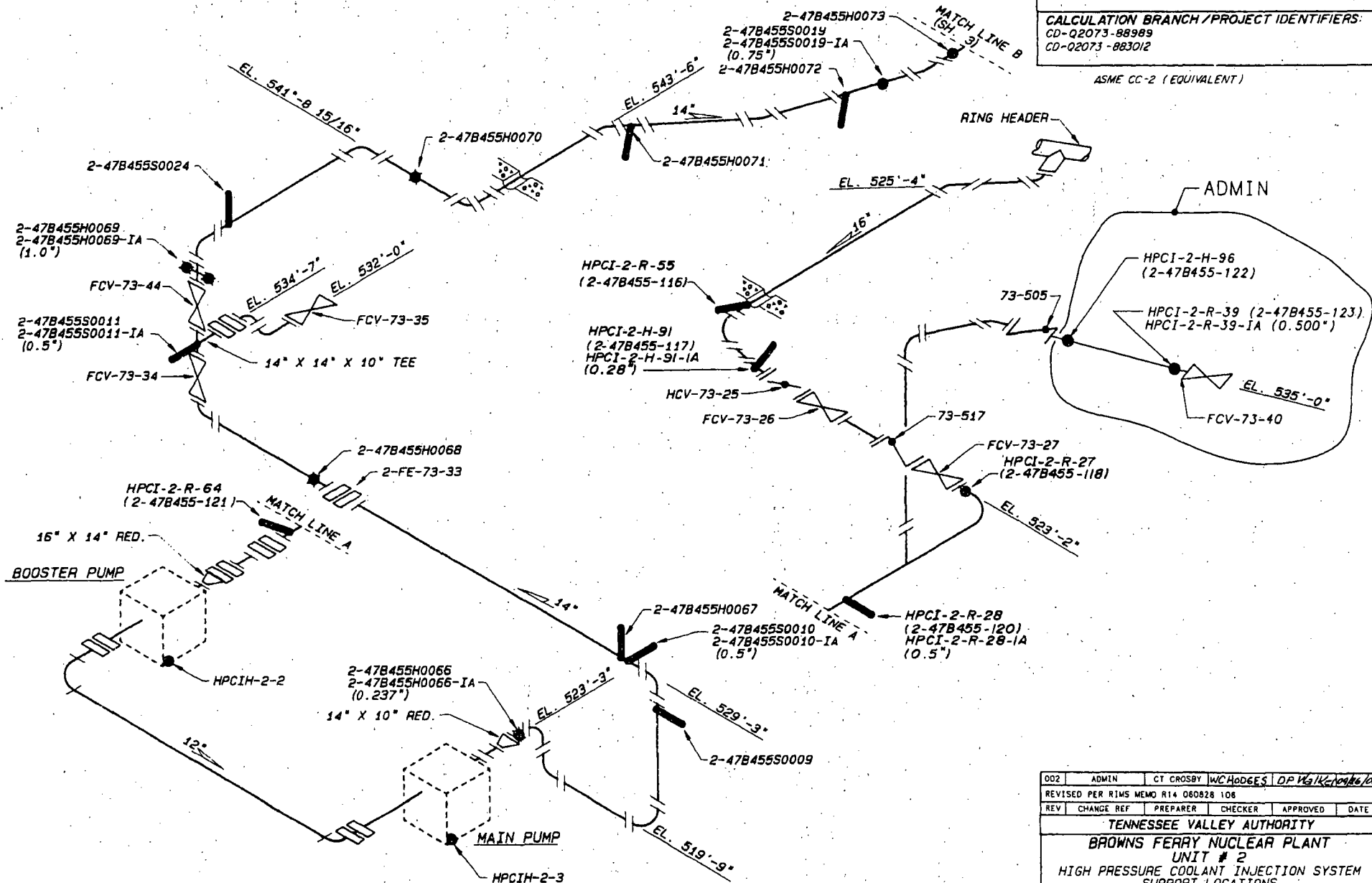
CAD MAINTAINED DRAWING

REFERENCE DRAWINGS
 47W455-H SERIES
 ISI-0128-C (SH. 2) WELD MA.

LEGEND
 ● RIGID HANGER
 ○ VARIABLE SUPPORT
 — RIGID STRUT

CALCULATION BRANCH / PROJECT IDENTIFIERS:
 CD-Q2073-88989
 CD-Q2073-883012

ASME CC-2 (EQUIVALENT)

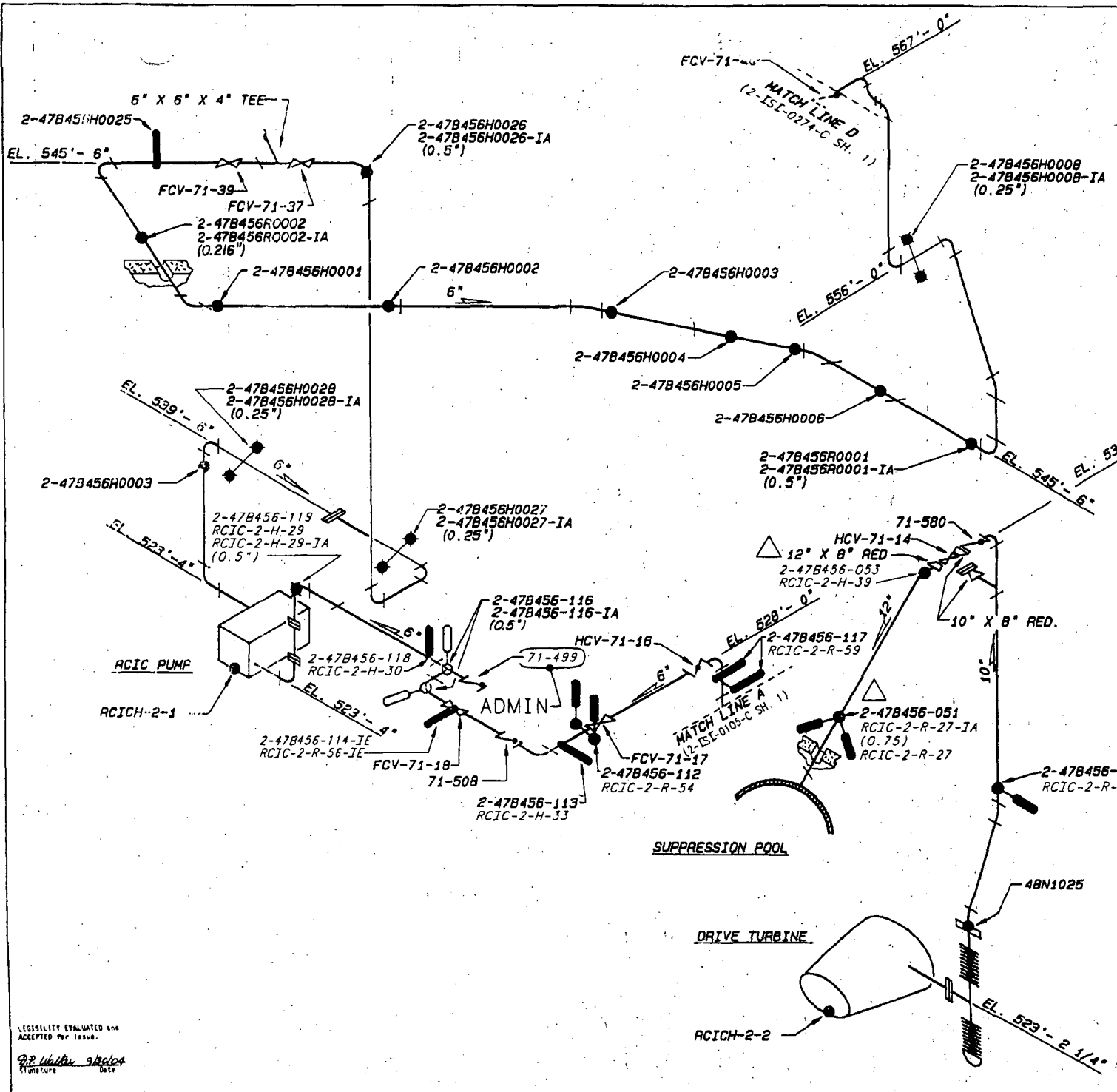


ALL A/D HISTORY RESEARCHED AT ROOD

002	ADMIN	CT CROSSBY	WCHODGES	DP Walker/10/16/06
REVISED PER RIMS MEMO R14 060828 106				
REV	CHANGE REF	PREPARER	CHECKER	APPROVED DATE
TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT				
UNIT # 2				
HIGH PRESSURE COOLANT INJECTION SYSTEM				
SUPPORT LOCATIONS				
DRAWN	REV	SUBMITTED	APPROVED	SCALE: NTS
DATE: 4-20-84	DATE: ---	DATE: ---	DATE: ---	SHEET 02 OF 03 SHEET (S)
CHECKED: JTL	DATE: ---	GLB.	DATE: ---	DRAWING NO. 2-ISI-0130-C
DATE: ---	DATE: ---	DATE: ---	DATE: ---	REV 002

CAD MAINTAINED DRAWING

CCD



REFERENCE DRAWINGS
 47B456-H SERIES
 47B456 SERIES
 2-151-0129-C (SH. 1) WELD ...

LEGEND:
 ● RIGID HANGER
 ○ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 ● MECHANICAL SNUBBER
 — RIGID STRUT

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-Q2071-88988

ASME CC-2 (EQUIVALENT)

△ EXEMPT PER ASME SECTION XI 1995 EDITION 1996 ADDENDA.

LEGIBILITY EVALUATED and ACCEPTED for issue.
 P.B. Walker 9/26/04
 DATE

008	ADMIN	J MCFARLAND	W.C. HODGES	DPH/MGL	9-30-2004
REVISED PER RIMS MEMO R14 040909 102					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
REACTOR CORE ISOLATION COOLING SYSTEM					
SUPPORT LOCATIONS					
DRAWN REV	SUBMITTED	APPROVED	SCALE	M/B	
DATE: 7-28-84	DATE: ---	DATE: ---	---	SHEET 1 OF 1 SHEET(S)	
CHECKED: JJJ	DATE: ---	GLB	DRAWING NO.	REV	
DATE: ---	---	---	2-151-0131-C	008	

ALL A/D HISTORY RESEARCHED AT ROOQ

CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS

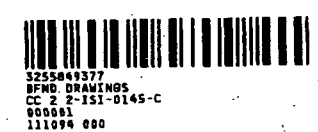
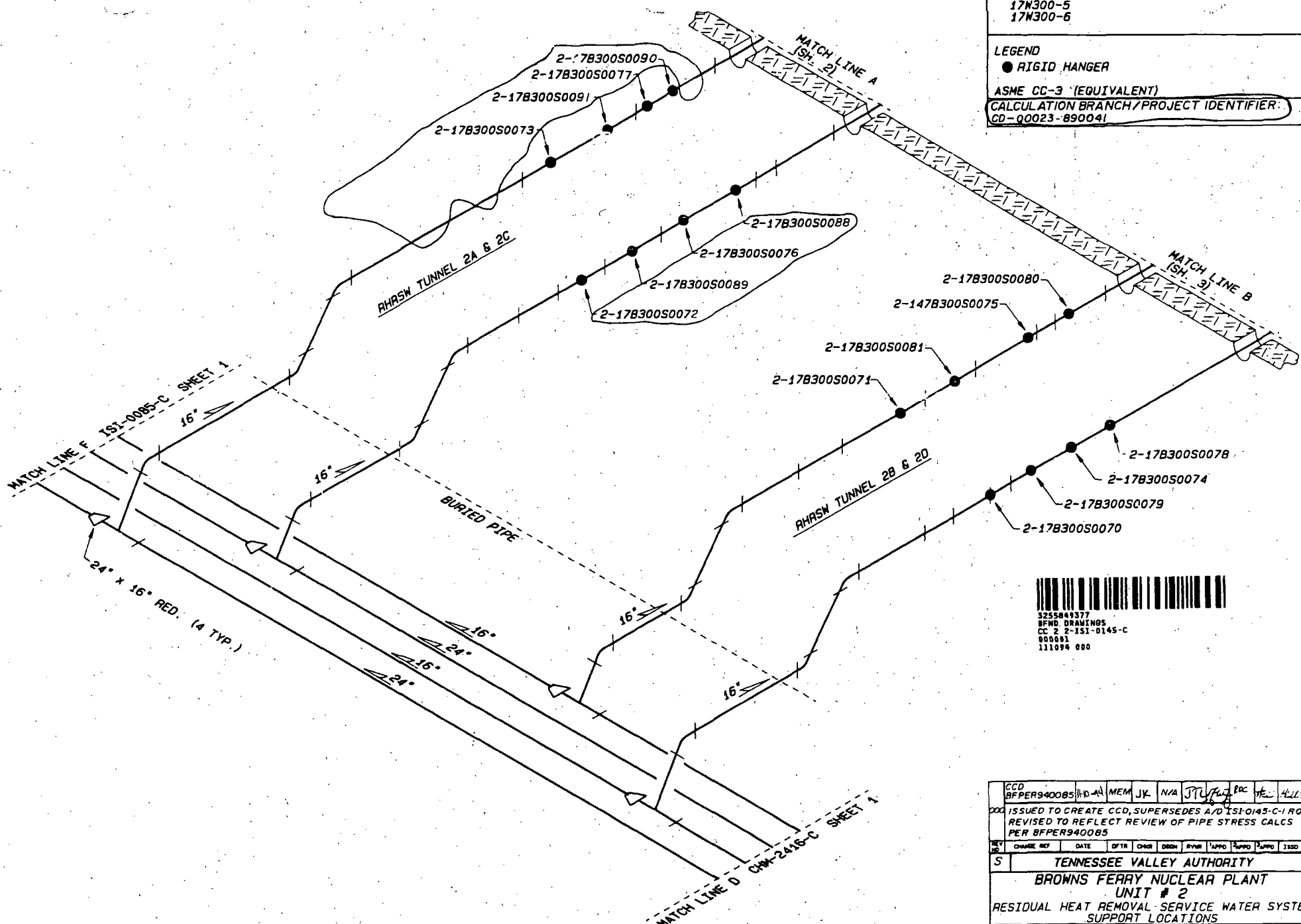
17N300-5
17N300-6

LEGEND

● RIGID HANGER

ASME CC-3 (EQUIVALENT)

CALCULATION BRANCH/PROJECT IDENTIFIER:
CD-00023-890041



CCD
BFFER9400B5 (10-A) MEM JY N/A JTL/PLC BPC MEW ALL
ISSUED TO CREATE CCD, SUPERSEDES A/D ISI-0145-C-1 R06;
REVISED TO REFLECT REVIEW OF PIPE STRESS CALCS
PER BFFER9400B5

REV	NO	CHANGE REF	DATE	BY	CHKD	DESIGN	APPD	SUPD	ISSD
S									

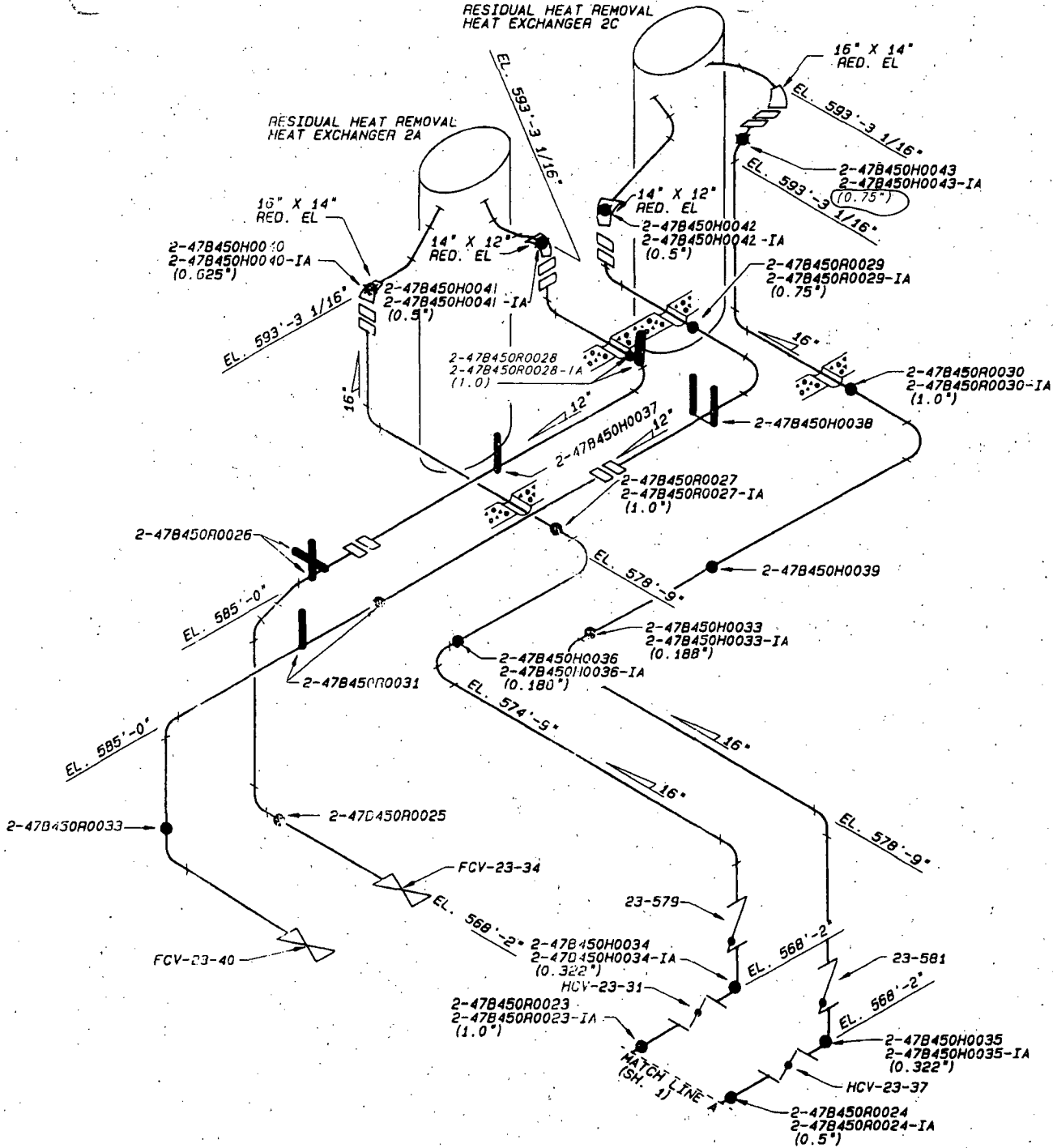
TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT
UNIT # 2
RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM
SUPPORT LOCATIONS

DRAWN: KEV	SUBMITTED	APPROVED	SCALE: NYS
DATE: 3-10-84	DATE:	DATE:	SHEET 01 OF 03 SHEET(S)
CHECKED: JTL		GLB	DRAWING NO. (REV)
DATE:			2-151-0145-C 000

CCD

RESIDUAL HEAT REMOVAL
HEAT EXCHANGER 2C

RESIDUAL HEAT REMOVAL
HEAT EXCHANGER 2A



REFERENCE DRAWINGS

2-47E450-2
0-47W450-4

LEGEND

- RIGID HANGER
- VARIABLE SUPPORT
- ▴ RIGID STRUT

CALCULATION BRANCH / PROJECT IDENTIFIER

CD - 02023 - 891361
CD - 02023 - 891362
CD - 02023 - 891363

ASME CC-3 (EQUIVALENT)

DOZ	ADMIN	RDL	WCH	RLD	12-9-99
-----	-------	-----	-----	-----	---------

REVISED PER RIMS MEMO R21 991201 003

REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
-----	------------	----------	---------	----------	------

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT
UNIT # 2
RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM
SUPPORT LOCATIONS

DRAWN	KEY	SUBMITTED	APPROVED	SCALE: NTS
DATE: 3-8-84	DATE: -	DATE: -	DATE: -	SHEET 2 OF 3 SHEET(S)
CHECKED: JIL	DATE: -	GLB	DATE: -	DRAWING NO. 2-1ST-0143-L
DATE: -	DATE: -	DATE: -	DATE: -	REV 002

MF
R0202

REFERENCE DRAWINGS:
 47K1767
 47N335-1
 KELLOGG 729E229
 CHM-2087-C (SH. 1) SUPPORT MAP
 NOTE: THIS DRAWING SUPERCEDES
 CHM-2069-C (SH. 1)

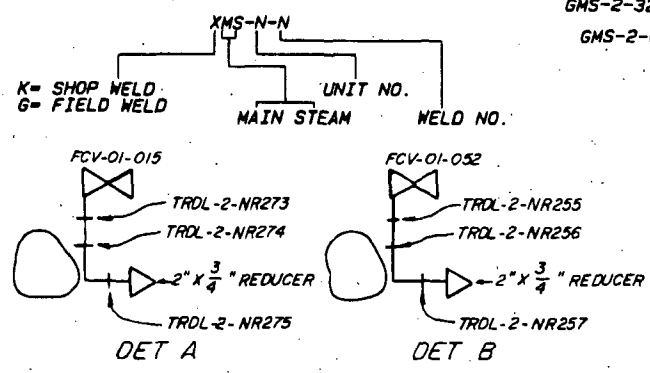
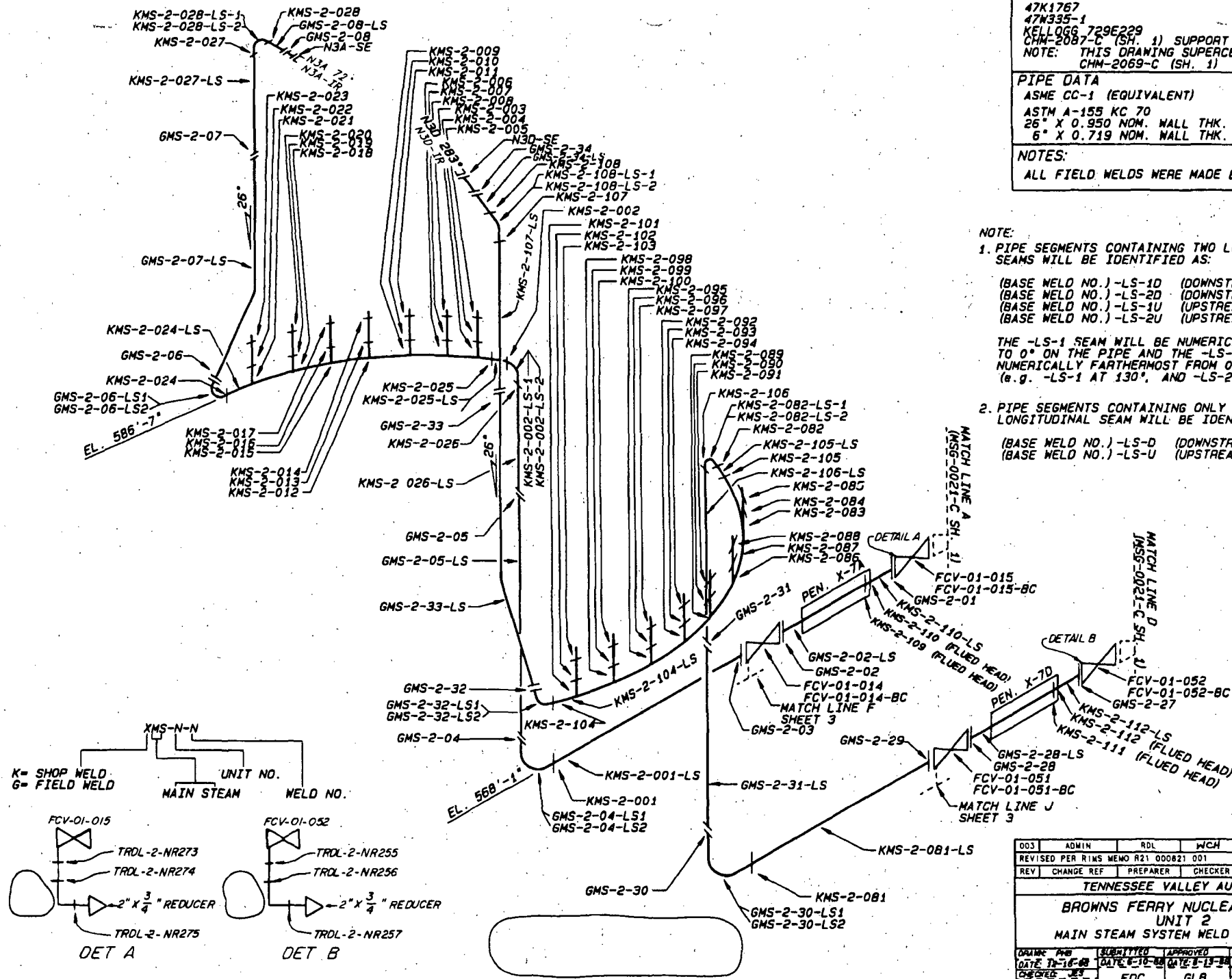
PIPE DATA
 ASME CC-1 (EQUIVALENT)
 ASTM A-155 KC 70
 26" X 0.950 NOM. WALL THK. (CS)
 6" X 0.719 NOM. WALL THK. (SCH. 160 CS)

NOTES:
 ALL FIELD WELDS WERE MADE BY TVA

NOTE:
 1. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.) -LS-1D (DOWNSTREAM)
 (BASE WELD NO.) -LS-2D (DOWNSTREAM)
 (BASE WELD NO.) -LS-1U (UPSTREAM)
 (BASE WELD NO.) -LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:
 (BASE WELD NO.) -LS-D (DOWNSTREAM)
 (BASE WELD NO.) -LS-U (UPSTREAM)



003	ADMIN	RDJ	WCH	HEH	5-24-01
REVISED PER RIMS MEMO R21 000821 001					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM SYSTEM WELD LOCATIONS					
DRAWN BY	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 7-18-00	DATE: 8-13-00	DATE: 8-13-00	SHEET 01 OF 04 SHEET(S)		
DESIGNED BY	DATE: 8-18-00	EDC	GLB	DRAWING NO. 2-TSI-0222-C 003	
CAD MAINTAINED DRAWING					

ALL A/D HISTORY RESEARCHED AT R000

REFERENCE DRAWINGS
 B&W 122863
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2001-C (UNIT 2)
 ASME CC-1 (EQUIVALENT)

NOTE:
 1. LIGAMENTS-RPV-LIGS-2-(01-92) WAS CHANGED
 IN ASME XI 95E96A TO THREADS IN
 FLANGE AND EXAM VOLUME WAS CHANGED.

270° - EAST Z

300°

0° - SOUTH
 W

60°

92 6.75" TAPPED HOLES ON
 267.25" DIA. B.C. REF.

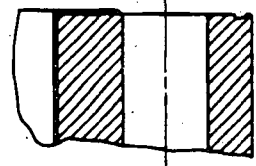
X 90° - WEST

240°

120°

NUMBERS (01 THRU 92) ARE PREFIXED BY:

- LIGAMENTS.....RPV-THRINFLG-2-
- STUDS.....RPV-STUDS-2-
- NUTS.....RPV-NUTS-2-
- WASHERS.....RPV-WASH-2-
- BUSHINGS.....RPV-BUSH-2-

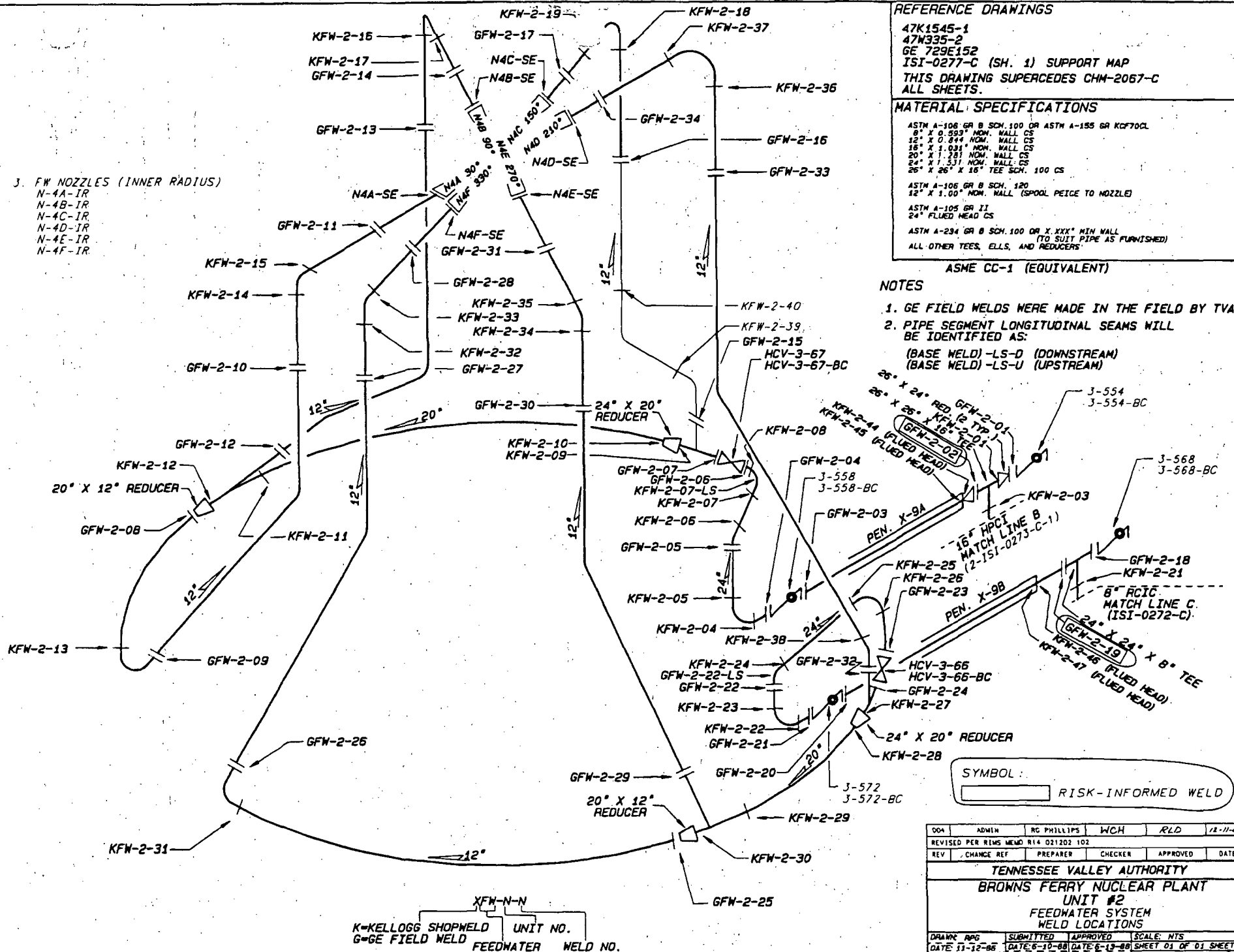


SECTION A-A

180° - NORTH
 Y

000	CCD/ADMIN	J MCFARLAND	WCHODGES	DP W/NER	4/26/85
ISSUED TO CREATE CCD SUPERSEDES ISI-0266-C-1 R2; REVISED PER RIMS MEMO #14 050328 103 (ADMINISTRATIVE REVISION)					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
HARDWARE: IBM 5085		SOFTWARE: CADAM		USER: ISICMP	
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2					
REACTOR VESSEL STUD LOCATIONS					
DRAWN: PWB	SUBMITTED	APPROVED	SCALE NTS		
DATE: 8-20-88	DATE: 8-3-89	DATE: 8-17-88	SHEET 91 OF 91 SHEET(18)		
CHECKED: JRE	EDC	DLB	DRAWING NO		
DATE: 8-3-87			2-ISI-0266-C		
CAD MAINTAINED DRAWING					CCD
					TVA: 000

3. FW NOZZLES (INNER RADIUS)
 N-4A-IR
 N-4B-IR
 N-4C-IR
 N-4D-IR
 N-4E-IR
 N-4F-IR



REFERENCE DRAWINGS
 47K1545-1
 47W335-2
 GE 729E152
 ISI-0277-C (SH. 1) SUPPORT MAP
 THIS DRAWING SUPERCEDES CHM-2067-C
 ALL SHEETS.

MATERIAL SPECIFICATIONS

ASTM A-106 GR B SCH. 100 OR ASTM A-155 GR K070CL
 8" X 0.593" NOM. WALL CS
 12" X 0.844" NOM. WALL CS
 16" X 1.031" NOM. WALL CS
 20" X 1.281" NOM. WALL CS
 24" X 1.531" NOM. WALL CS
 26" X 26" X 18" TEE SCH. 100 CS

ASTM A-106 GR B SCH. 120
 12" X 1.00" NOM. WALL (SPOOL PEICE TO NOZZLE)

ASTM A-105 GR II
 24" FLUED HEAD CS

ASTM A-234 GR B SCH. 100 OR X.XXX" MIN WALL
 (TO SUIT PIPE AS FURNISHED)
 ALL OTHER TEES, ELLS, AND REDUCERS

ASME CC-1 (EQUIVALENT)

- NOTES**
1. GE FIELD WELDS WERE MADE IN THE FIELD BY TVA.
 2. PIPE SEGMENT LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD) -LS-D (DOWNSTREAM)
 (BASE WELD) -LS-U (UPSTREAM)

SYMBOL: RISK-INFORMED WELD

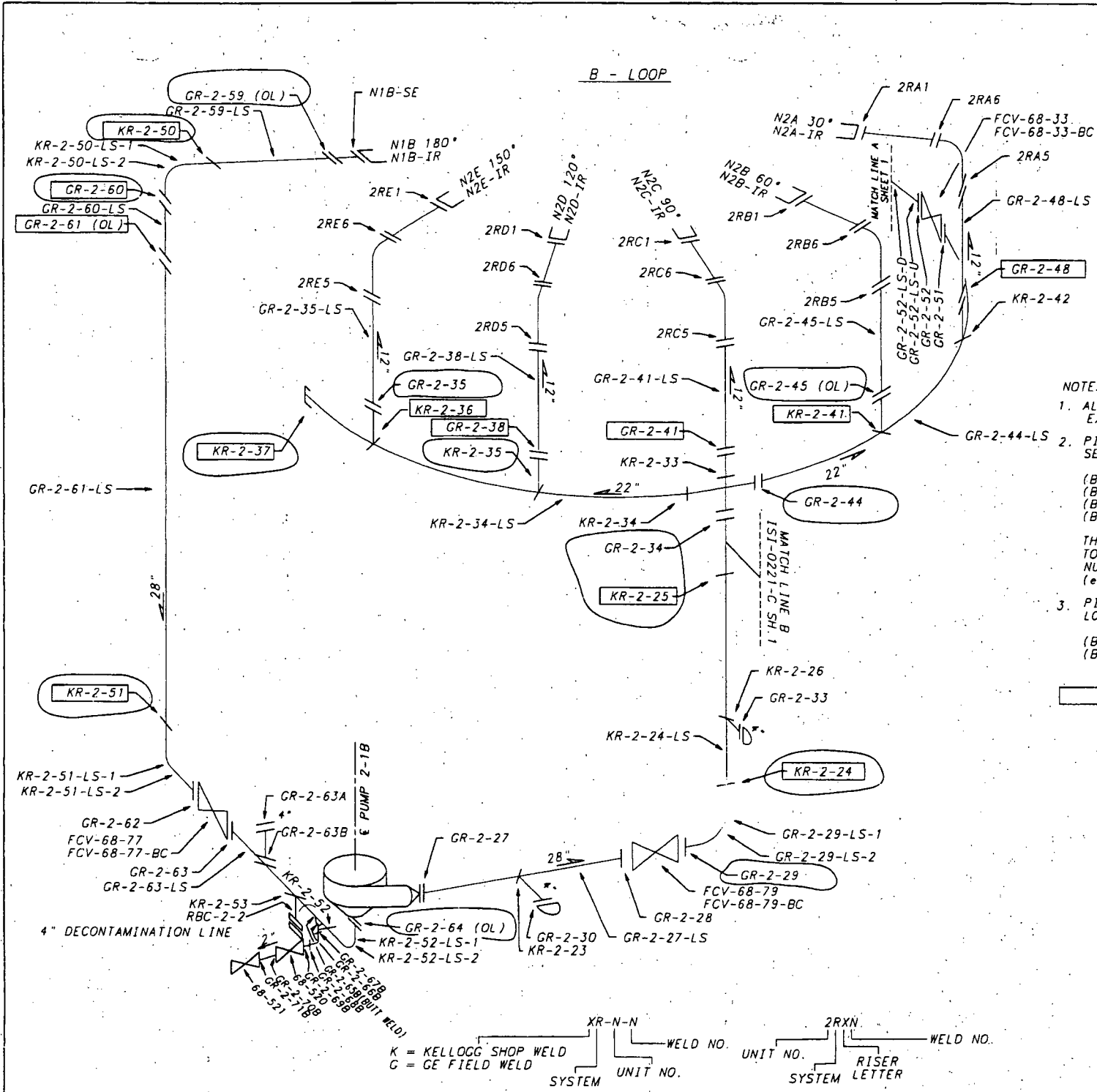
004	ADMIN	RC PHILLIPS	WCH	RLO	12-11-02
REVISED PER RIMS MEMO R14 021202 102					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT #2					
FEEDWATER SYSTEM					
WELD LOCATIONS					
DRAWN: RFG	SUBMITTED: DATE: 6-10-08	APPROVED: DATE: 6-13-08	SCALE: NTS	SHEET 01 OF 01 SHEET (S)	
CHECKED: JES	DATE: 6-10-08	EDC	GLB	DRAWING NO. 2-TSI-0269-C	REV 004

ALL A/D HISTORY RESEARCHED AT 000

CAD MAINTAINED DRAWING

CCD

K=KELLOGG SHOPWELD UNIT NO.
 G=GE FIELD WELD FEEDWATER WELD NO.



REFERENCE DRAWINGS:
 2-47W2408-8,9 (S.E. REPL ENT)
 GE 769E963 (S.E. REPLACE)
 TVA 47K1544-2
 GE 2-153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2068-C ALL SHEETS

MATERIAL SPECIFICATIONS:
 A358, TP 304
 4" X 0.337" NOM WALL THK. (SS)
 12" X 0.569" NOM. WALL THK. (SS)
 22" X 1.030" NOM. WALL THK. (SS)
 28" X 1.138" NOM. WALL THK. (SS) SUCTION
 28" X 1.322" NOM. WALL THK. (SS) DISCHARGE

2" SCH. 80, A376, TP304
 2" FITTINGS A182.F304

SAFE END REPLACEMENT
 12" X 0.688 NOM. WALL THK. (SS)
 SA 403 WP 316 N.C.

ASME CC-1 (EQUIVALENT)

- NOTES:**
- ALL 2" WELDS ARE SOCKET WELDED EXCEPT WHERE NOTED.
 - PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)
 THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)
 - PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

 RISK INFORMED WELDS

K = KELLOGG SHOP WELD
 G = GE FIELD WELD

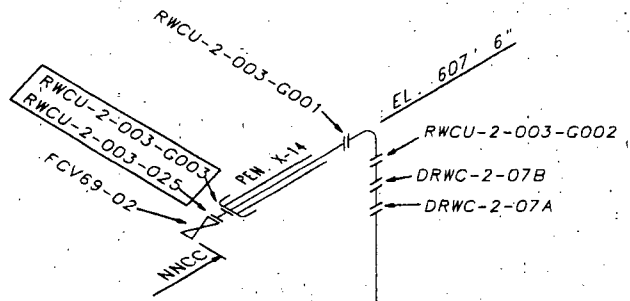
WELD NO. _____
 SYSTEM UNIT NO. _____

2RXN _____
 WELD NO. _____
 SYSTEM RISER LETTER _____

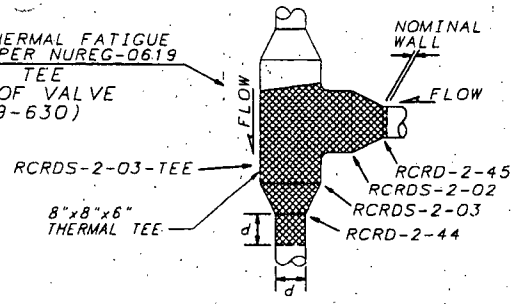
004	ADMIN	RD LOOSIER	WCH	PLD	12-11-02
REVISED PER RIMS MEMO R14 021202 102					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM WELD LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NIS
DATE	3-28-89	DATE	3-8-89	DATE	5-17-89
CHECKED	JES	EDC	GLB	DRAWING NO.	2-TSI-0270-CDD4
DATE	5-3-89			REV.	

ALL A/D HISTORY RESEARCHED • ROOD

CAD MAINTAINED DRAWING CCD



REGION OF THERMAL FATIGUE EXAMINATION PER NUREG-0619
 THERMAL TEE (UPSTREAM OF VALVE 2-FCV-69-630)



REFERENCE DRAWING
 CRD-2-005
 RCIC-2-004
 RWC-2-001
 474335-14, -17

NOTE:
 THIS DRAWING SUPERSEDES CHM-2075-C AND CHM-2072-C (ALL SHEETS)

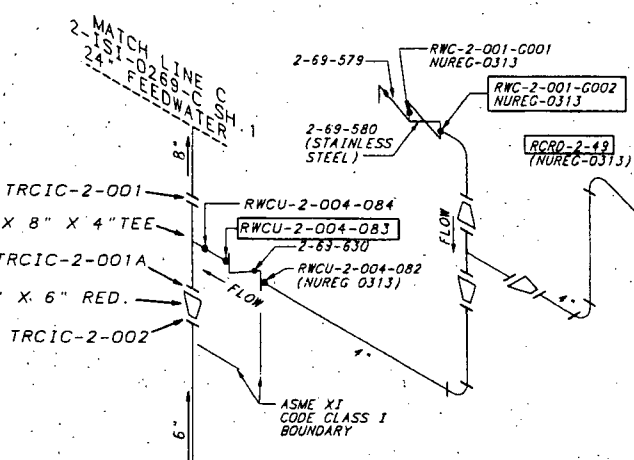
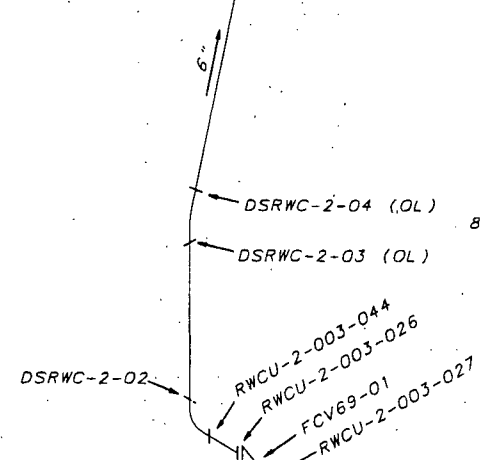
MATERIAL SPECIFICATIONS
 STAINLESS STEEL
 FITTINGS
 6" SA403 WP316NG SCH. 80 SS
 PIPING
 6" SA376 TP316NG SCH. 80 SS
 6" A376GR TP304 SCH. 80 SS
 6" A312GR TP304 SCH. 80 SS

CARBON STEEL
 4" SCH. 80 A-333, GR1 (SEAMLESS) CS
 6" X 0.562" NOM WALL SCH. 120 CS
 8" X 0.593" NOM WALL SCH. 100 CS
 VALVE
 2-69-630 SA351 CF8M SS
 CRD CAP
 4" X 0.674" NOM WALL SS

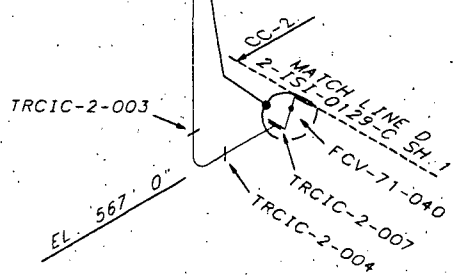
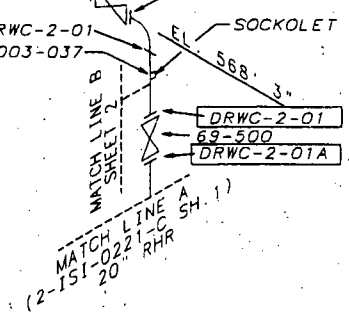
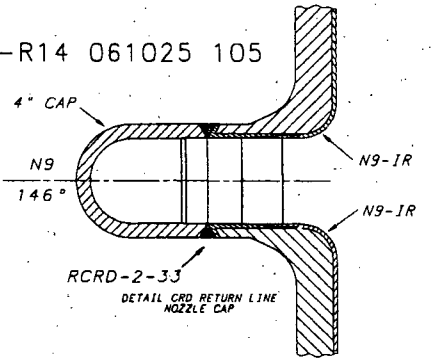
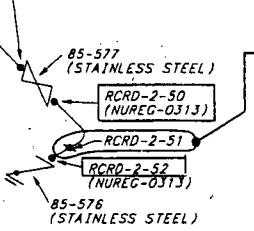
ASME CC-1 (EQUIVALENT)
 RISK INFORMED WELD

DSRWC-2-06
 CARBON STEEL
 STAINLESS STEEL
 DSRWC-2-05 (OL)

R14 061025 105



R14 060913 101

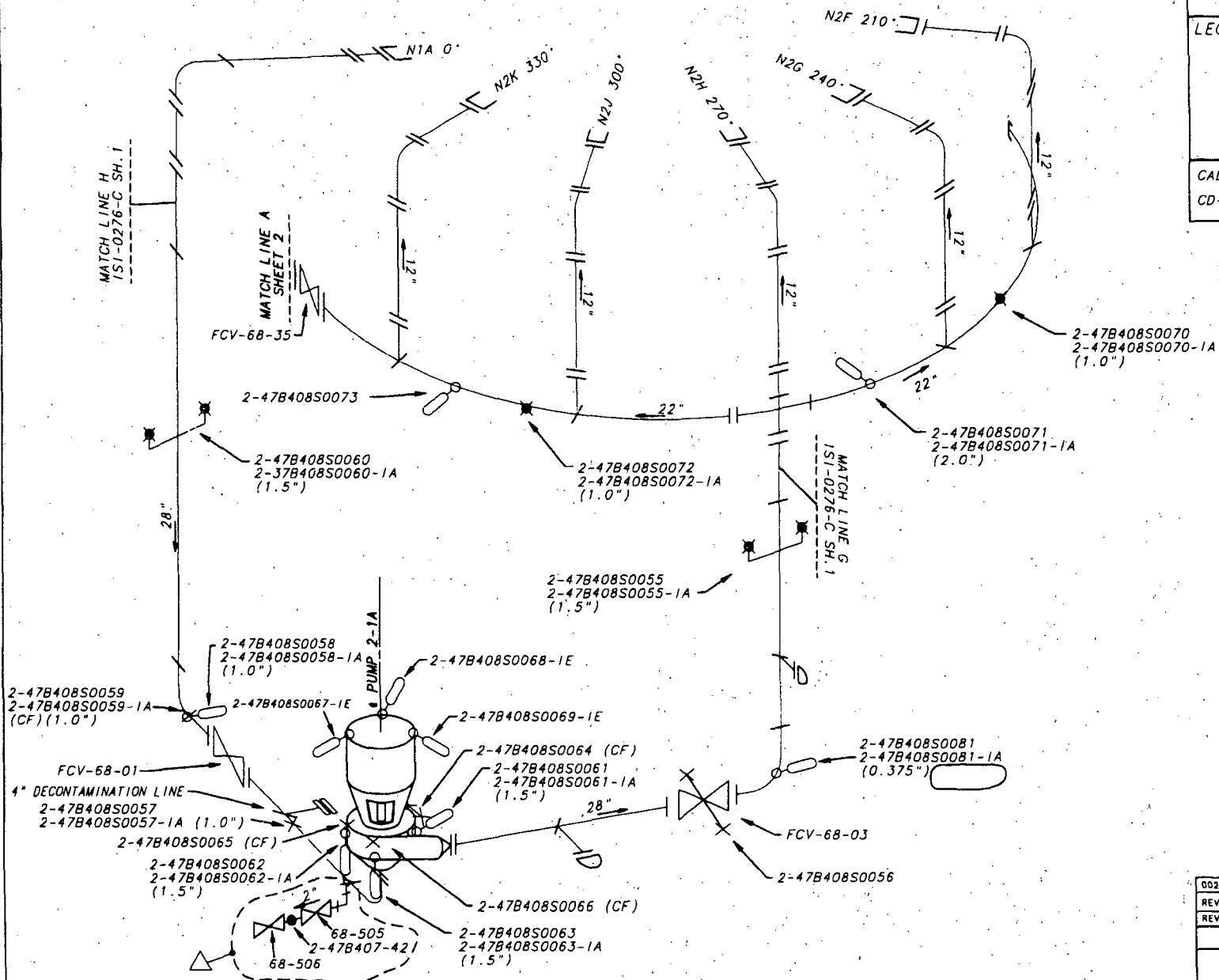


009	ADMIN	J MCFARLAND	W C HODGES	OP	WALKER	WALKER
REVISED PER RIMS MEMO R14 060913 101 AND R14 061025 105						
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE	
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT						
UNIT 2						
REACTOR WATER CLEAN UP, RCIC, AND CRD WELD IDENTIFICATION						
DRAWN:	PHB	DATE:	6-9-88	SCALE:	NTS	CADAM/TSICMP
CHECKED:	JES	APPROVED:		SHEET	01 OF 03	REV
SUBMITTED:	EDC		GLB	2-IST-0272-C	009	

ALL A/D HISTORY RESEARCHED AT ROO0

CAD MAINTAINED DRAWING
 CCD

A - LOOP



REFERENCE DRAWINGS:
 2-47W2408-B,9 (S.E. REPL. ENT)
 GE 769E963 (S.E. REPLACEMENT)
 TVA 47K1544-2
 2-153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2068-C ALL SHEETS

LEGEND:
 ● RIGID HANGER
 ✖ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 ✕ CONSTANT FORCE SUPPORT (CF)

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-Q2068-871118

ASME CC-1 (EQUIVALENT)

△ EXEMPT PER DNE CALCULATION
 (MD-Q0999-950033)
 RIMS MEMO R14 950829 109

002	ADMIN	RDL	WCH	HEH	5/30/87
REVISED PER RIMS MEMO R21.000021 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RECIRCULATION SYSTEM					
SUPPORT LOCATIONS					
DRAWN: PHB	SUBMITTED	APPROVED	SCALE NTS		
DATE: 8-22-87	DATE: 8-10-88	DATE: 11-88	SHEET 1 OF 2 SHEETS(S)		
CHECKED: JES	EDC	GLB	DRAWING NO.		
DATE: 8-10-88			2-151-0278-C002		

CAD MAINTAINED DRAWING

CCD

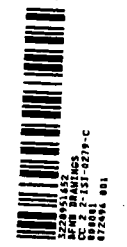
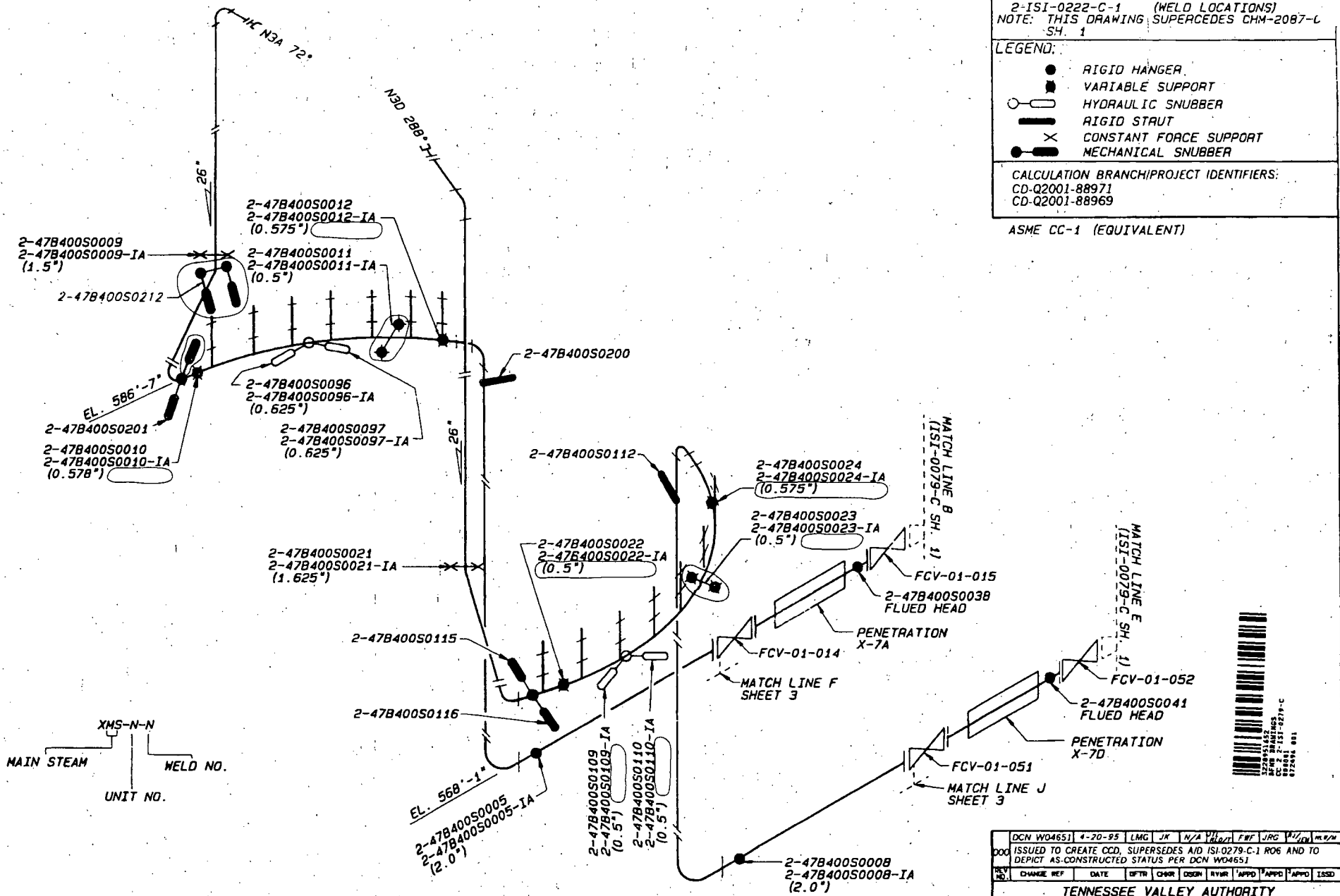
ALL A/D HISTORY RESEARCHED AT ROOD

REFERENCE DRAWINGS:
 GE 729E401-1
 0-729E401-2
 2-ISI-0222-C-1 (WELD LOCATIONS)
 NOTE: THIS DRAWING SUPERCEDES CHM-2087-L SH. 1

LEGEND:
 ● RIGID HANGER
 ○ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 — RIGID STRUT
 × CONSTANT FORCE SUPPORT
 ● MECHANICAL SNUBBER

CALCULATION BRANCH/PROJECT IDENTIFIERS:
 CD-Q2001-88971
 CD-Q2001-88969

ASME CC-1 (EQUIVALENT)



DCN	WO465J	4-20-95	LMG	JK	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
ISSUED TO	ISSUED TO CREATE CCD, SUPERSEDES AID ISI-0279-C-1 R06 AND TO DEPICT AS-CONSTRUCTED STATUS PER DCN WO465J																		
REV. NO.	CHANGE REF.	DATE	DFTR	CHGR	DSGN	RYMR	APPR	APPR	APPR	ISSD									

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT
UNIT 2
MAIN STEAM SYSTEM SUPPORT LOCATIONS

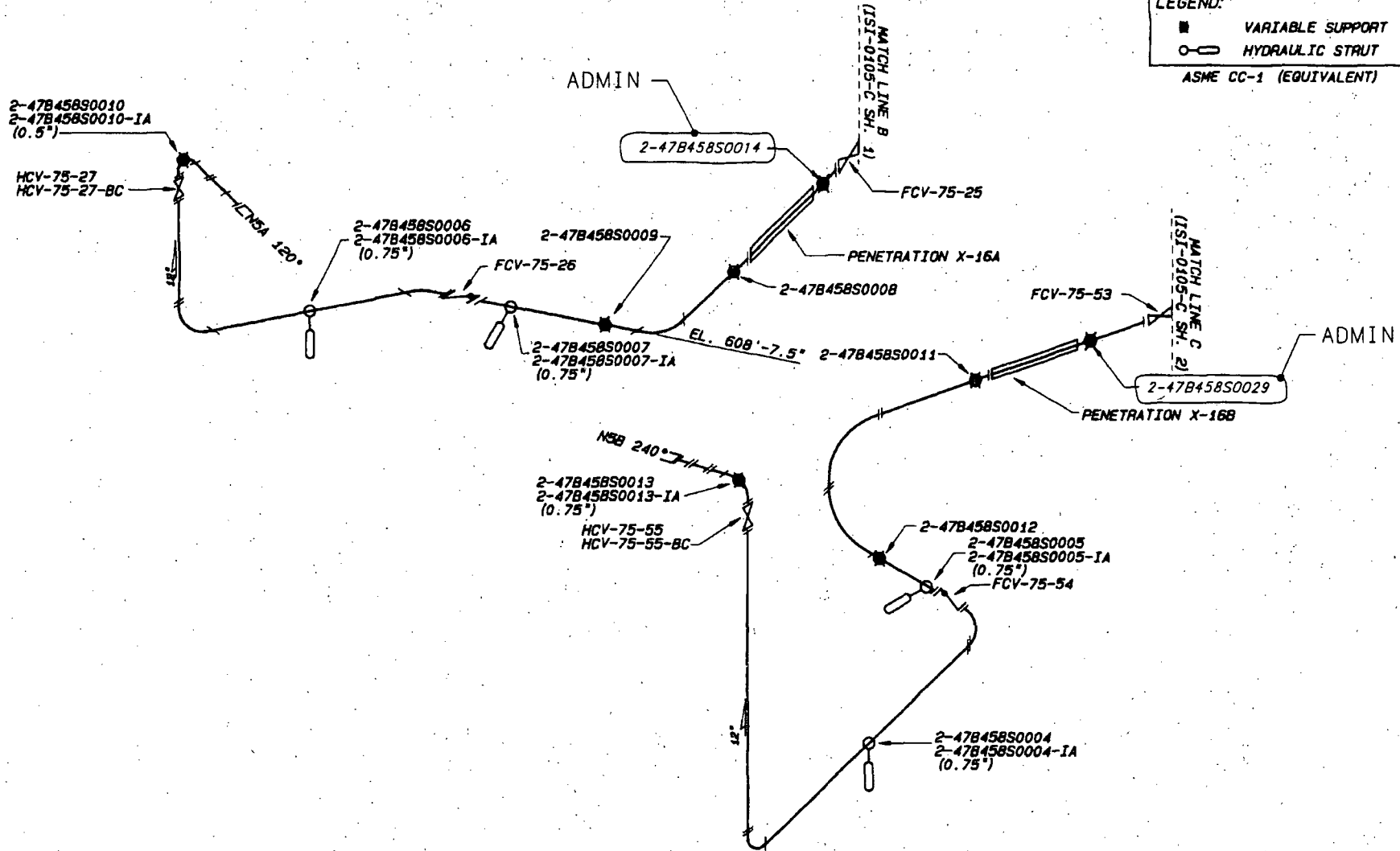
DRWNG	PHB	SUBMITTED	APPROVED	SCALE	NTS
DATE	12-16-95	DATE		SHEET	01 OF 04 (SHEET 1)
CHECKED	JES	DATE		DRWING NO.	2-ISI-0279-C
DATE		DATE		CCD	R001

ADMIN	7-24-96	RML	WAL	N/A	FLD	N/A	N/A	OLD	N/A	
REV. NO.	001	CHANGE REF.	DATE	DFTR	CHGR	DSGN	RYMR	APPR	APPR	ISSD

ALL AID HISTORY RESEARCHED @ R000

REFERENCE DRAWINGS:
 47W458-1R8
 ISI-0271-C (SH. 1) WELD MA.
 NOTE: THIS DRAWING SUPERCEDES CHM-2089-C
 ALL SHEETS.

LEGEND:
 ■ VARIABLE SUPPORT
 ○ HYDRAULIC STRUT
 ASME CC-1 (EQUIVALENT)



CS - N - N
 CORE SPRAY SUPPORT NO.
 UNIT NO.

002	ADMIN	CT CROSSBY	WCHODGES	DP H&K/r	09/16/06
REVISED PER RIMS MEMO R14 060828 104					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
CORE SPRAY SYSTEM					
SUPPORT LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NTS
DATE	8-26-07	DATE	DATE	SHEET 1 OF 1	SHEET (S)
CHECKED	JES	EDC	GLB	DRAWING NO.	REV
DATE				2-ISI-0280-C	002
CAD MAINTAINED DRAWING					CCD

ALL A/D HISTORY RESEARCHED AT ROOD

CALCULATION BRANCH/PROJECT IDENTIFIER: CD-Q2073-88301

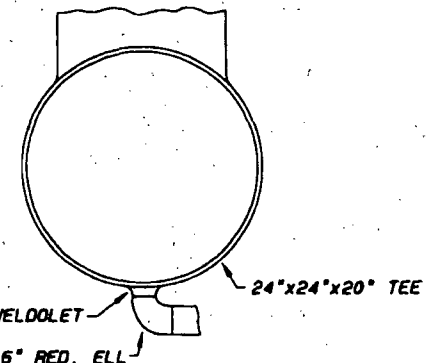
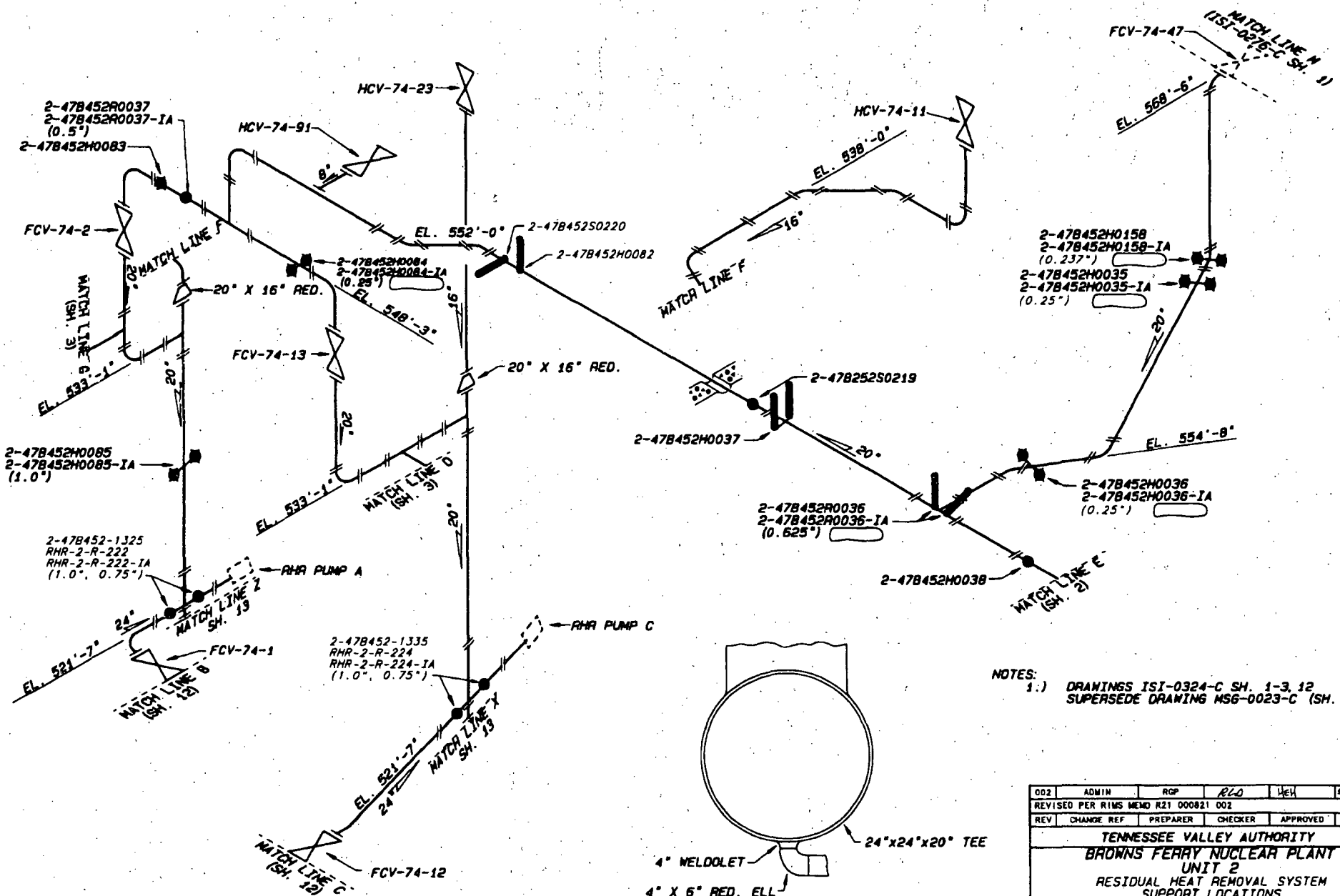
LEGEND:

- RIGID HANGER
- VARIABLE SUPPORT
- ▬ RIGID STRUT

ASME CC-2 (EQUIVALENT)

REFERENCE DRAWINGS:

47N452 SERIES
 47N335-4
 MSG-0018-C (SH. 1) WELL P
 THIS DRAWING SUPERSEDES MSG-0023-C SH. (SEE NOTE 1)



DETAIL OF 4" X 6" REDUCING ELL CONNECTION

NOTES:
 1.) DRAWINGS ISI-0324-C SH. 1-3, 12
 SUPERSEDE DRAWING MSG-0023-C (SH. 1)

002	ADMIN	RCP	RLO	WEL	5-30-2001
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DRAWN PHB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 8-8-88	DATE: 8-8-88	DATE: 8-8-88	SHEET 1 OF 13 SHEETS		
CHECKED JES	EDC	GLB	DRAWING NO.	REV	
DATE: 8-8-88			2-151-0324-C	002	

ALL A/D HISTORY RESEARCHED TO R000

CAD MAINTAINED DRAWING

CCD

NOTES:

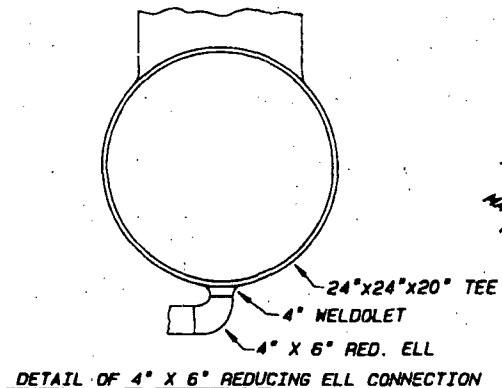
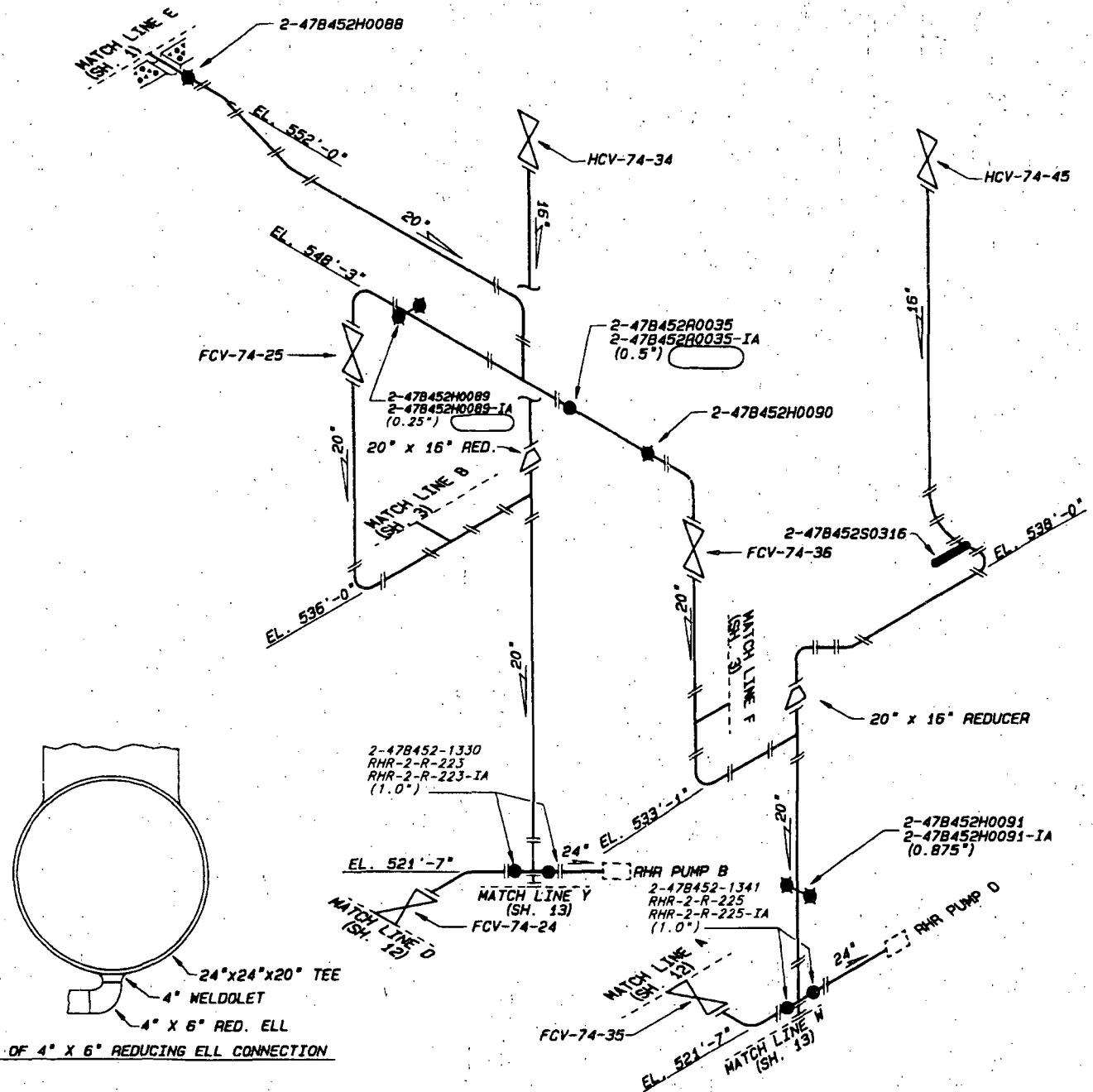
1. DRAWINGS ISI-0324-C SH. 1-3, 12 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 1)

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-4
 MSG-0018-C (SH. 2)
 THIS DRAWING SUPERCEDES MSG-0023-C SH. 1
 (SEE NOTE 1)

LEGEND
 ● RIGID SUPPORT
 ■ VARIABLE SUPPORT
 — RIGID STRUT

CALCULATION BRANCH/PROJECT IDENTIFIER: CD-Q2073-883012

ASME CC-2 (EQUIVALENT)



ALL A/D HISTORY RESEARCHED AT ROOD

001	ADMIN	ROP	RLO	HEH	6-8-88
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DRAWN: PHB	SUBMITTED: 6-8-88	APPROVED: 6-8-88	SCALE: NYS	SHEET 2 OF 13 SHEET(S)	
CHECKED: JES	DATE: 6-8-88	EDC	GLB	DRAWING NO. 2-151-0324-C	REV 001
DATE: 6-8-88					

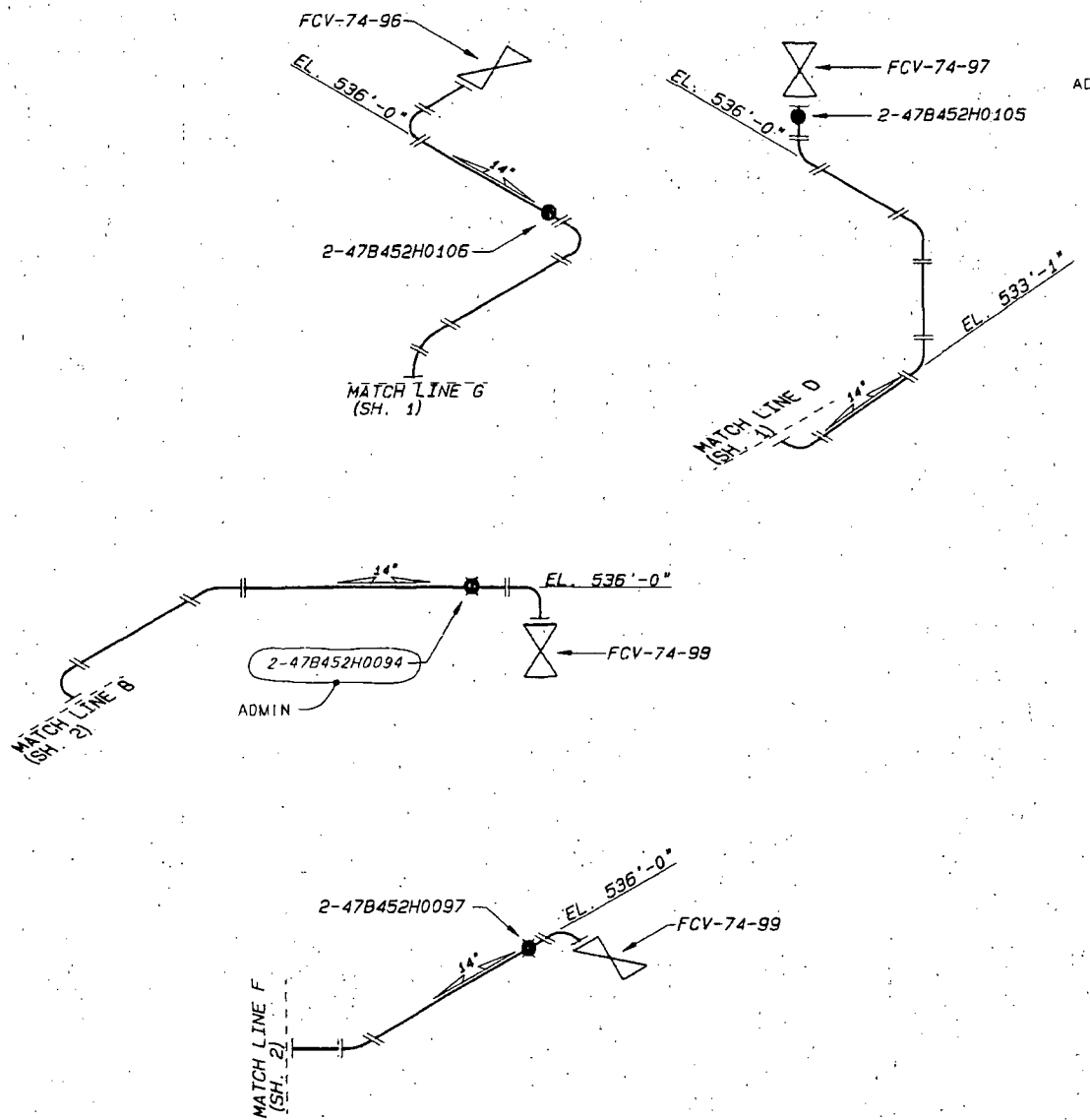
CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-4
 MSG-0018-C (SH. 3) WELL
 THIS DRAWING SUPERCEDES MSG-0023-C SH. 1
 (SEE NOTE 1)

LEGEND:
 ● RIGID HANGER
 ○ VARIABLE SUPPORT

CALCULATION BRANCH/PROJECT
 IDENTIFIER: CD-Q2074-880992
 ASME CC-2 (EQUIVALENT)



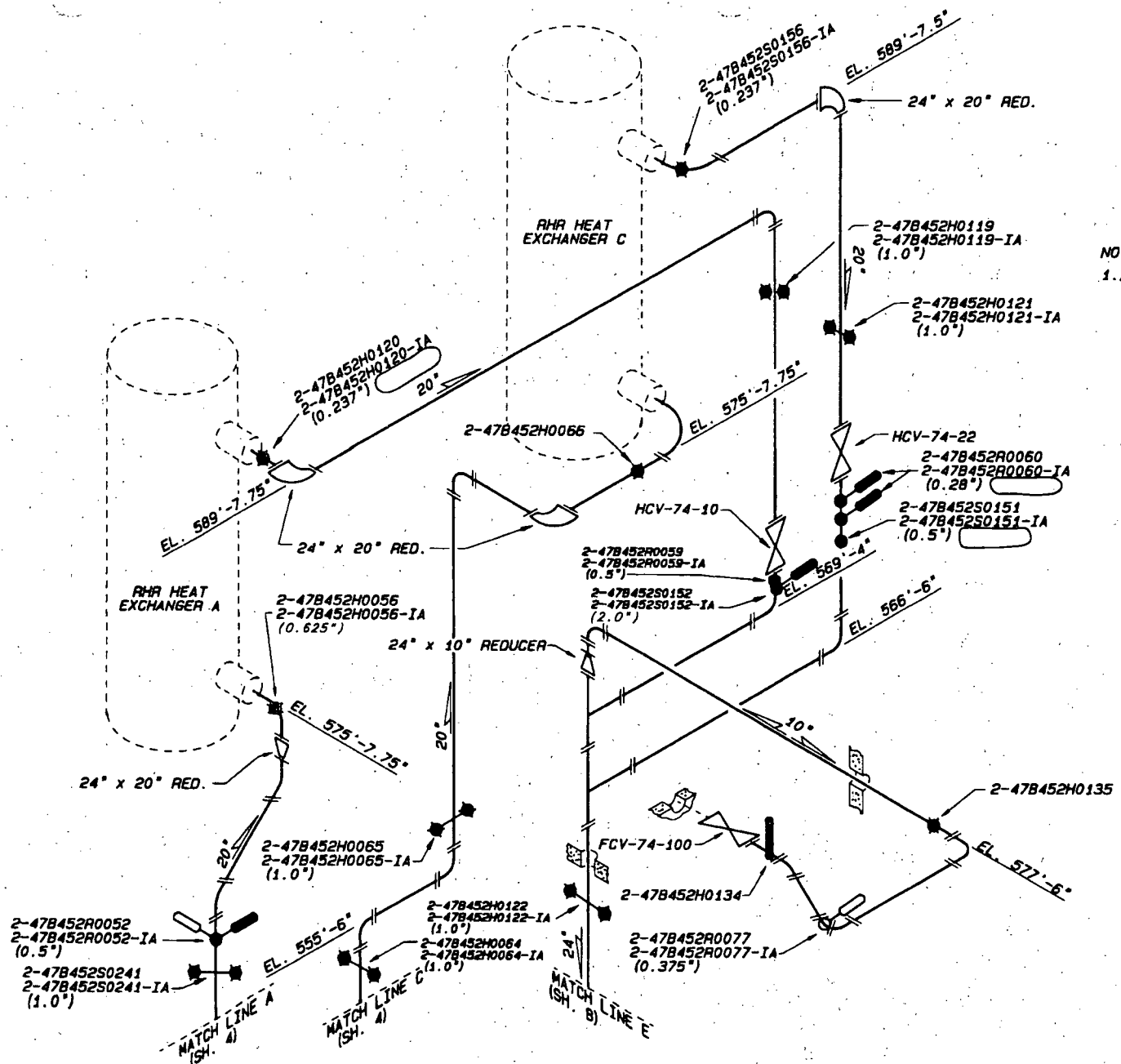
NOTE:
 1. DRAWINGS 2-151-0324-C-3, 151-0324-C
 SH. 1, 2, 12 MAKE UP SUPERCEDED DRAWING
 MSG-0023-C (SH. 1).

DESIGN	REV	DATE	BY	CHKD	APPD	ISSD														
000																				
ISSUED TO CREATE CCD. SUPERSEDES A/D 151-0324-C-3 R3 AND TO DEPICT AS-CONSTRUCTED STATUS PER A/D R0, R1, R2, R104 MEMO R32 940109 881 (ADMINISTRATIVE REVISION)																				
KEY	NO	CHANGE	REF	DATE	BY	CHKR	DSGN	RVWR	APPD	APPD	APPD	ISSD								
HARDWARE	TEK	4129	SOFTWARE	TEK/CAD	1	B	FLOPPY	BF-12												
TENNESSEE VALLEY AUTHORITY																				
S BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS																				
DRAWN: PHB			SUBMITTED			APPROVED			SCALE: NYS											
DATE: 5-8-89			DATE: 6-8-89			DATE: 6-8-89			SHEET 3 OF 13 SHEET(S)											
CHECKED: JES			EDC			GLB			DRAWING NO.			REV								
DATE: 6-8-89									2-151-0324-C			000								
												CCD								

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-7
 MS6-001B-C (SH. 5) WELD
 THIS DRAWING SUPERSEDES MS6-0023-C SH. 2
 (SEE NOTE 1)

LEGEND:
 ● RIGID HANGER
 ■ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 ○ MECHANICAL SNUBBER
 — RIGID STRUT
 ASME CC-2 (EQUIVALENT)

NOTES:
 1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP
 SUPERCEDED DRAWING MS6-0023-C (SH. 2)



001	ADMIN	RGP	ELD	WEL	5-30-70
REVISED PER RIMS MEMO R21 00021 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DRAWING NO.	SUBMITTED DATE	APPROVED DATE	SCALE: NTS		
CHECKED: JES	DATE: EDC	DATE: GLB	SHEET 5 OF 13 SHEET(S)		
DATE:			DRAWING NO.	REV	
			2-151-0324-C	001	

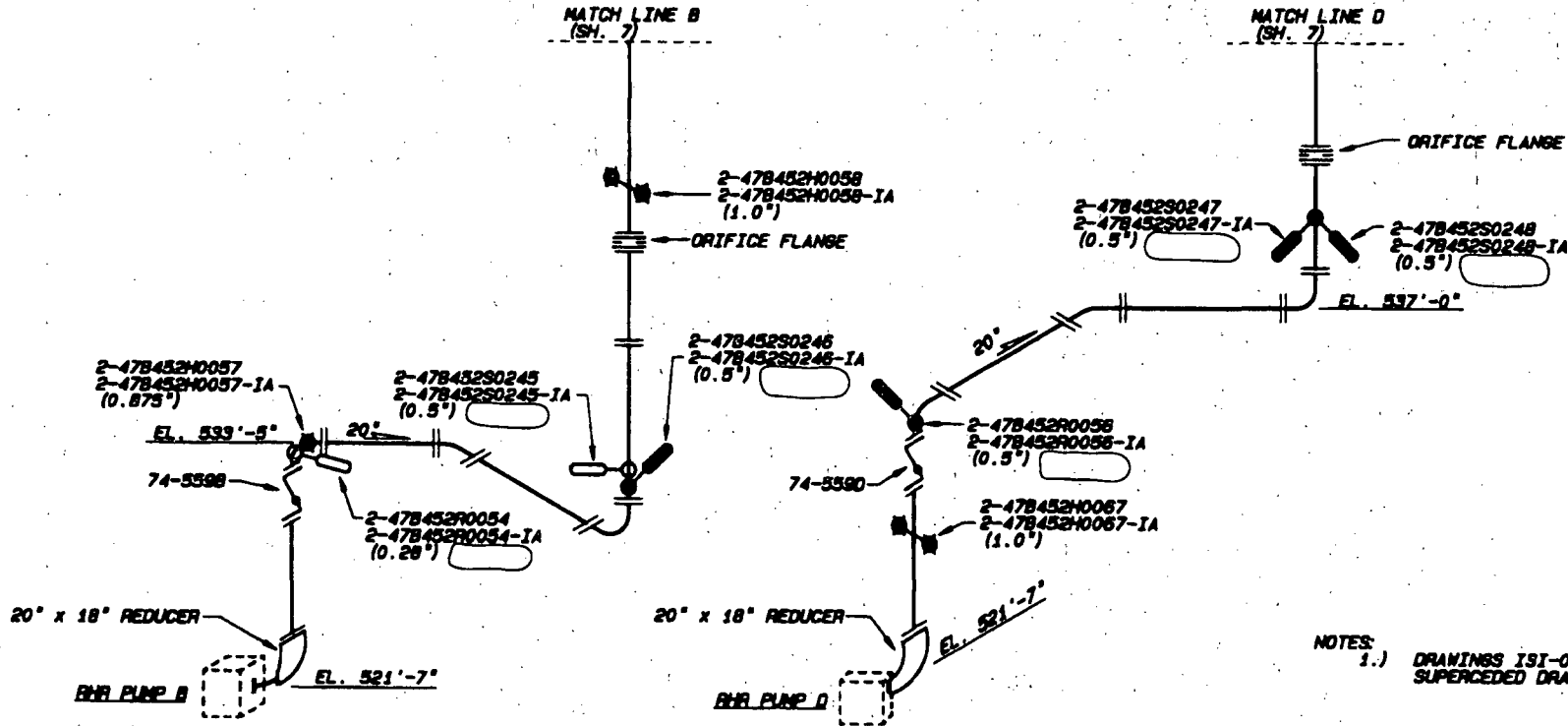
ALL A/D HISTORY RESEARCHED TO ROOT

CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS
 47W453 SERIES
 47W453-7
 NSG-0018-C (SH. 8) WELD MAP
 THIS DRAWING SUPERCEDES NSG-0023-C SH. 2
 (SEE NOTE 1)

LEGEND:
 ● RIGID HANGER
 □ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 ○ MECHANICAL SNUBBER
 ASME CC-2 (EQUIVALENT)



NOTES:
 1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP SUPERCEDED DRAWING NSG-0023-C (SH. 2)

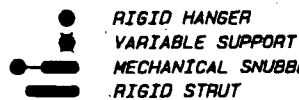
000	ADMIN	ROP	ELC	HEA	5-30-2001
ISSUED TO CREATE CDD. SUPERCEDES A/D ISI-0324-C-8 R0004 AND TO DEPICT AS-CONSTRUCTED STATUS PER RIMS MEMO R21 000821 002 (ADMINISTRATIVE REVISION)					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DATE	DATE	DATE	DATE	DATE	DATE
2-2-88	1-2-88	1-1-88	1-1-88	1-1-88	1-1-88
EDC	GLB				
2-151-0324-C					000

ALL A/D HISTORY RESEARCHED TO R000

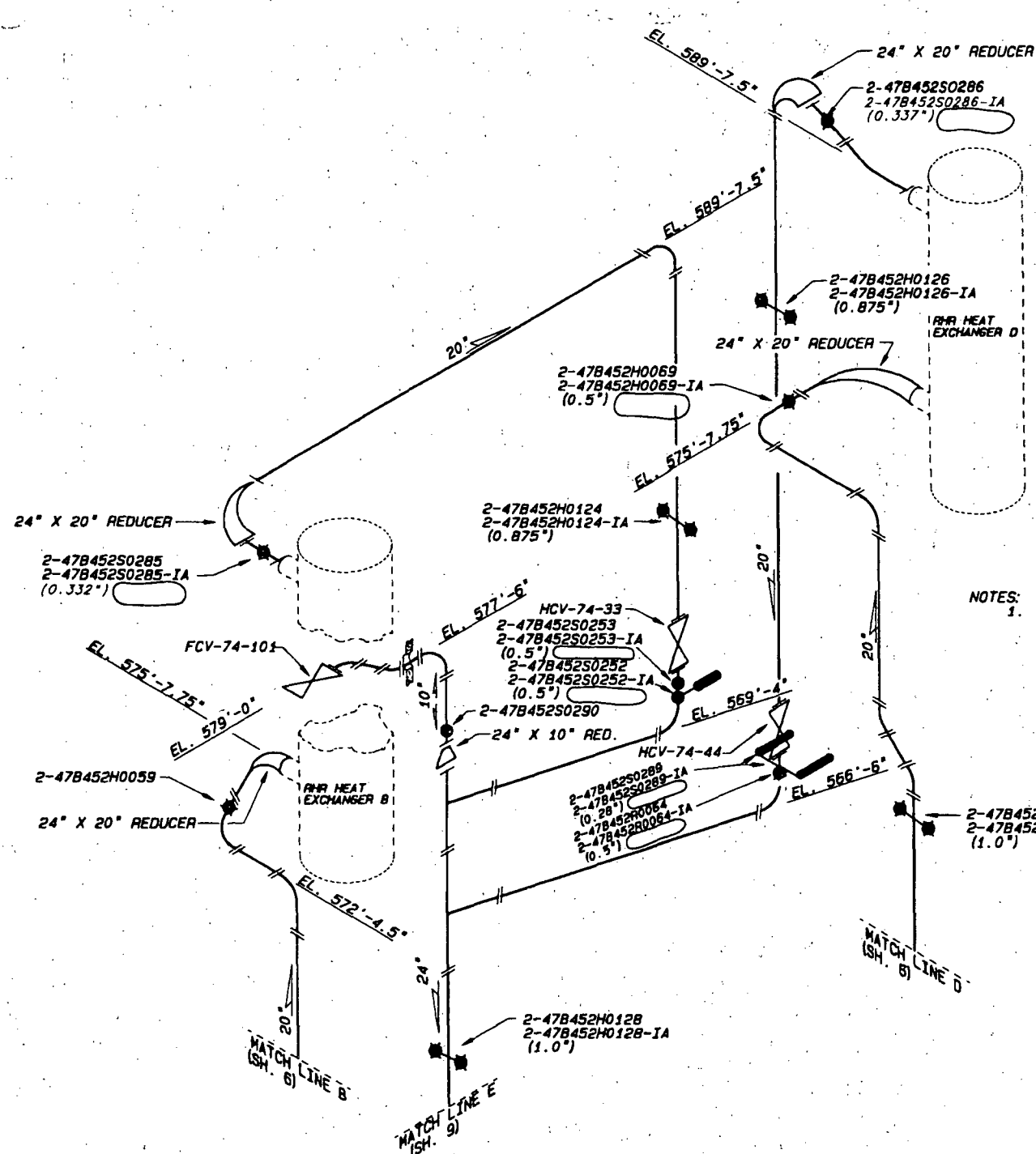
CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS
 47N452 SERIES
 47N335-7
 MSG-0018-C (SH. 7) WELD MA
 THIS DRAWING SUPERCEDES MSG-0023-C SH. 2
 (SEE NOTE 1)

LEGEND:

 RIGID HANGER
 VARIABLE SUPPORT
 MECHANICAL SNUBBER
 RIGID STRUT

ASME CC-2 (EQUIVALENT)



NOTES:
 1. DRAWINGS ISI-0324-C SH. 4-7 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 2)

TRF
 BOH
 205

001	ADMIN	RGP	ELD	MEH	5-30-88
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DRAWN P/B	SUBMITTED	APPROVED	SCALE: NYS		
DATE: 6-8-88	DATE: 6-8-88	DATE: 6-9-88	SHEET 7 OF 13 SHEET(S)		
CHECKED: JES	EOC	GLB	DRAWING NO.	REV	
DATE: 6-8-88			2-151-0324-C	001	

ALL A/O HISTORY RESEARCHED AT R000

CAD MAINTAINED DRAWING

CCD

NOTES:

1. DRAWINGS ISI-0324-C CH. 8 AND 9 MAKE UP SUPERSEDED DRAWING MSG-0023-C (SH. 3)
2. POSSIBLE EXCESSIVE GRIND SPOT BELOW RHR-2-H-7 (RS) @ 7.00
3. DRAWING NUMBERS PROVIDED IN PARENTHESIS. DRAWING MAY HAVE MULTIPLE SHEETS.

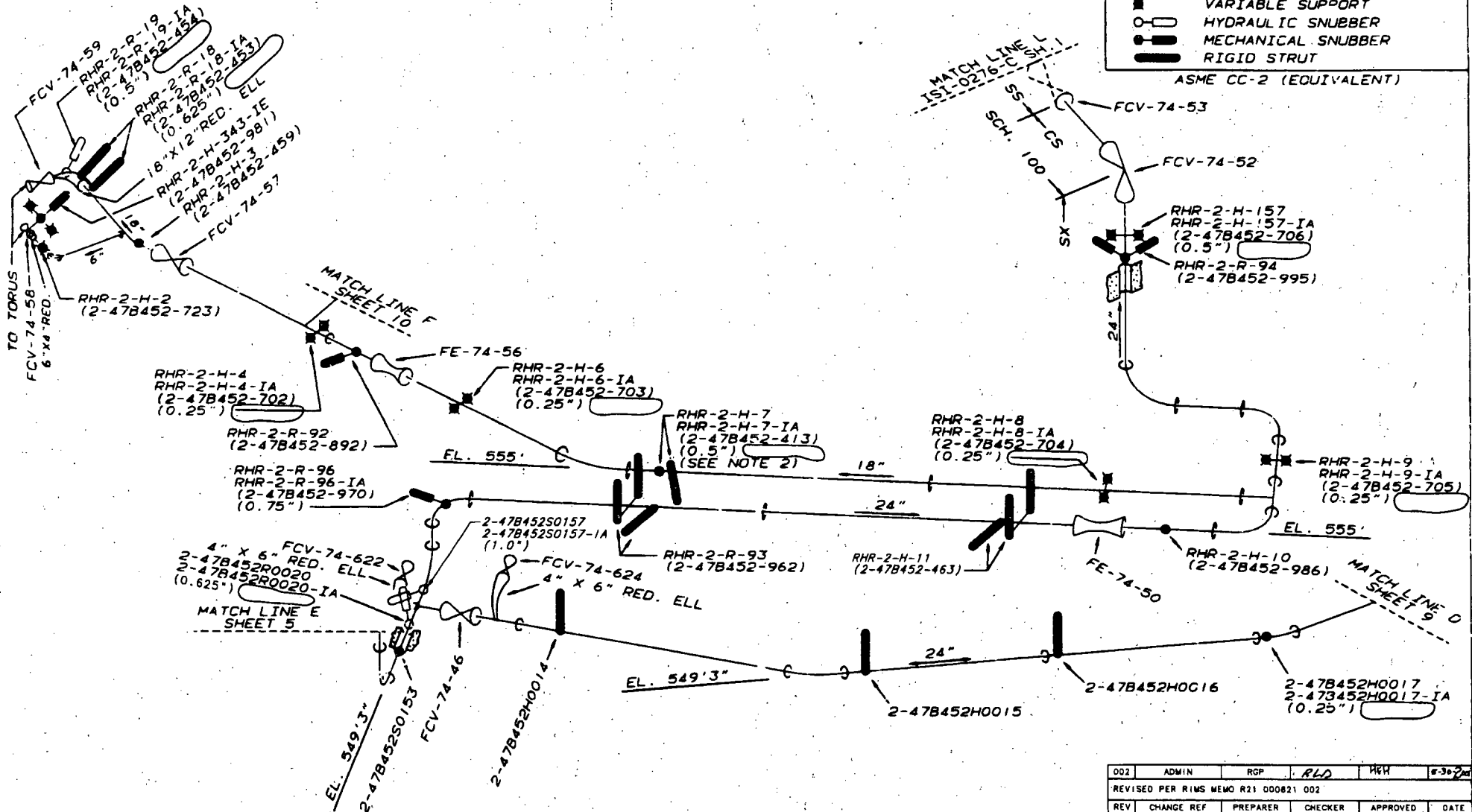
CALCULATION BRANCH/PROJECT IDENTIFIER: CD-02074-880991
CD-02074-870644

REFERENCE DRAWINGS
47W452-221
47W335-6
WP2563-88
MSG-0018-C (SH.8) WELD MAP
THIS DRAWING SUPERSEDES MSG-0023-C SH.3 (SEE NOTE 1)

LEGEND

- RIGID HANGER
- ⊠ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ▬ MECHANICAL SNUBBER
- ▬ RIGID STRUT

ASME CC-2 (EQUIVALENT)



002	ADMIN	ROP	RLD	MEH	10-30-72
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DRAWN: PMS	SUBMITTED	APPROVED	SCALE: NTS		
DATE:	DATE:	DATE:	SHEET 08 OF 13 PART 157		
CHECKED: JET	EDC	GLJ	DRAWING NO.	ISI-0324-C-0002	
DATE:					


CAD MAINTAINED DRAWING

CCD

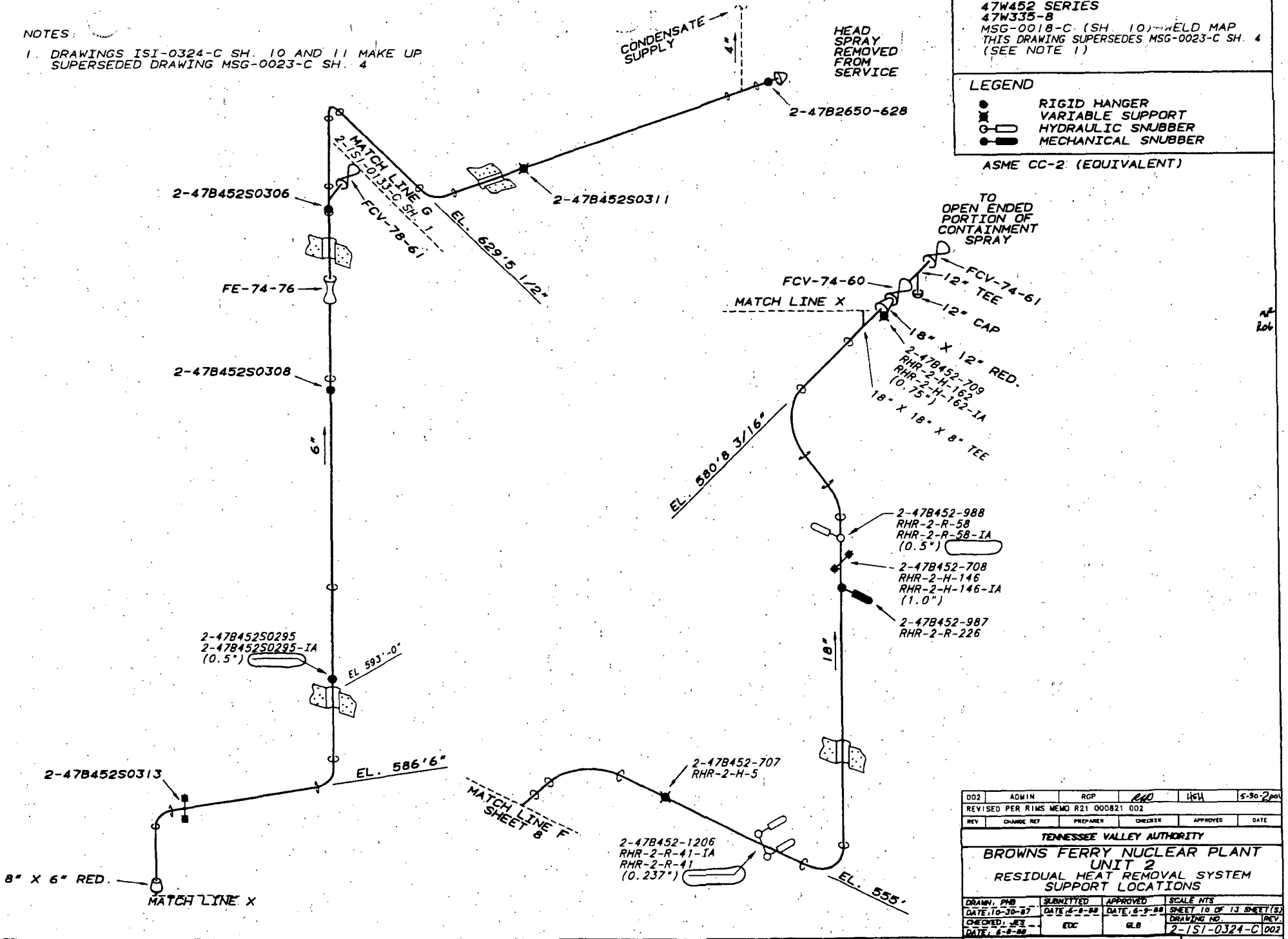
NOTES:

1. DRAWINGS ISI-0324-C SH. 10 AND 11 MAKE UP SUPERSEDED DRAWING MSG-0023-C SH. 4

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-8
 MSG-0018-C (SH. 10) WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH. 4
 (SEE NOTE 1)

LEGEND

 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER

ASME CC-2 (EQUIVALENT)



002	ADMIN	RGP	RWD	HEH	5-30-2004
REVISED PER RIMS MEMO R21 000821 002					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DRAWN: PWB	SUBMITTED	APPROVED	SCALE NTS		
DATE: 10-30-87	DATE: 5-8-88	DATE: 6-9-88	SHEET 10 OF 13 SHEET(S)		
CHECKED: JES	EDC	GLB	DRAWING NO.	REV.	
DATE: 6-8-88			2-151-0324-C	002	

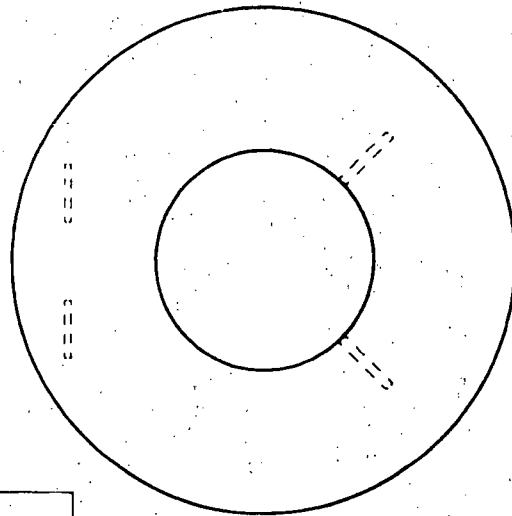
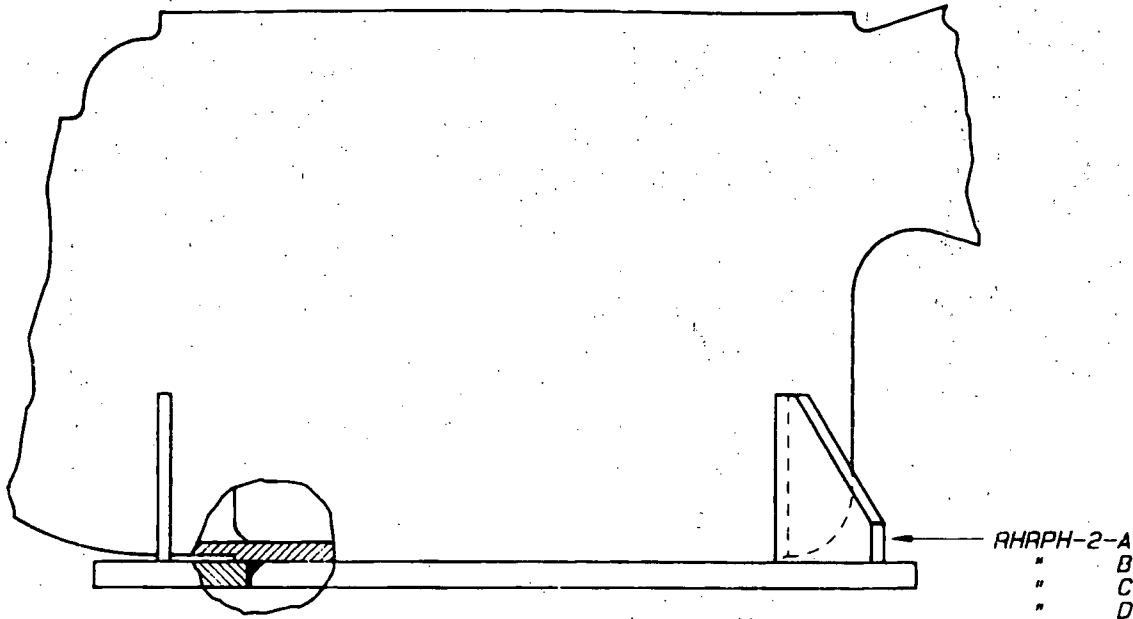
ALL A/D HISTORY RESEARCHED AT ROOD

CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS:
THIS DRAWING SUPERCEDES ISI-0022-B
FOR UNIT 2 ONLY.

ASME CC-2 (EQUIVALENT)



REV.	BY	DATE	DESCRIPTION	CK	DSUB	APP

TENNESSEE VALLEY AUTHORITY
DIVISION OF NUCLEAR ENGINEERING

BROWNS FERRY NUCLEAR PLANT
UNIT 2

RESIDUAL HEAT REMOVAL PUMP SUPPORT

REV			
	00		
HARDWARE	TEKTRONIX 4129		
SOFTWARE	TEKNICAD 8.2		
FLOPPY OR TAPE #	FLOPPY #BF4		

DRAWN:	SUBMITTED	APPROVED	SCALE:
PHB			NTS
DATE: 6-10-88	DATE: 6/10/88	DATE: 6/13/88	SHEET 1 OF 1 SHEET(S)
CHECKED: JES			DRAWING NO
DATE: 6-70-88			ISI-0310-B
			REV
			00

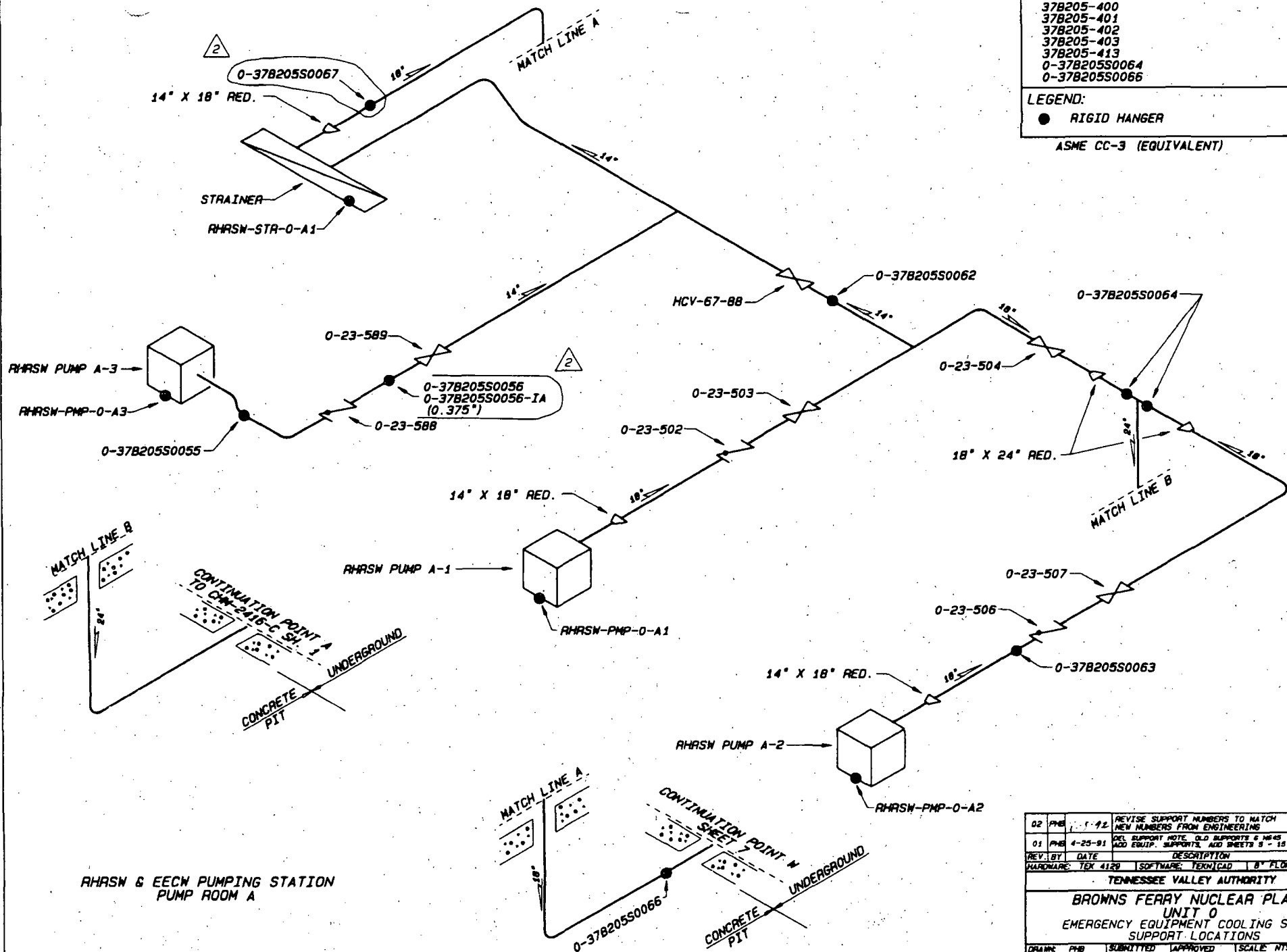
REFERENCE DRAWINGS

- 37B205-400
- 37B205-401
- 37B205-402
- 37B205-403
- 37B205-413
- 0-37B205S0064
- 0-37B205S0066

LEGEND:

- RIGID HANGER

ASME CC-3 (EQUIVALENT)



RHRSW & EECW PUMPING STATION
PUMP ROOM A

02	PHB	1-92	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	PHB	JES	GLB
01	PHB	4-25-91	DEL SUPPORT NOTE, OLD SUPPORTS & HANGERS. ADD EQUIP. SUPPORTS. ADD SHEETS 8'-15'	JAA	JES	GLB
REV.	BY	DATE	DESCRIPTION	CHK'D	DATE	APP
HARDWARE: TEX 4120			SOFTWARE: TEK/ICAD	8" FLOPPY: BP-21		

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT
UNIT 0
EMERGENCY EQUIPMENT COOLING SYSTEM
SUPPORT LOCATIONS

DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NTS
DATE	3-21-90	DATE	4-17-90	DATE	4-17-90
CHECKED	JES	EDC	GLB	DRAWING NO.	REV
DATE	4-17-90			TSI-0368-C	02

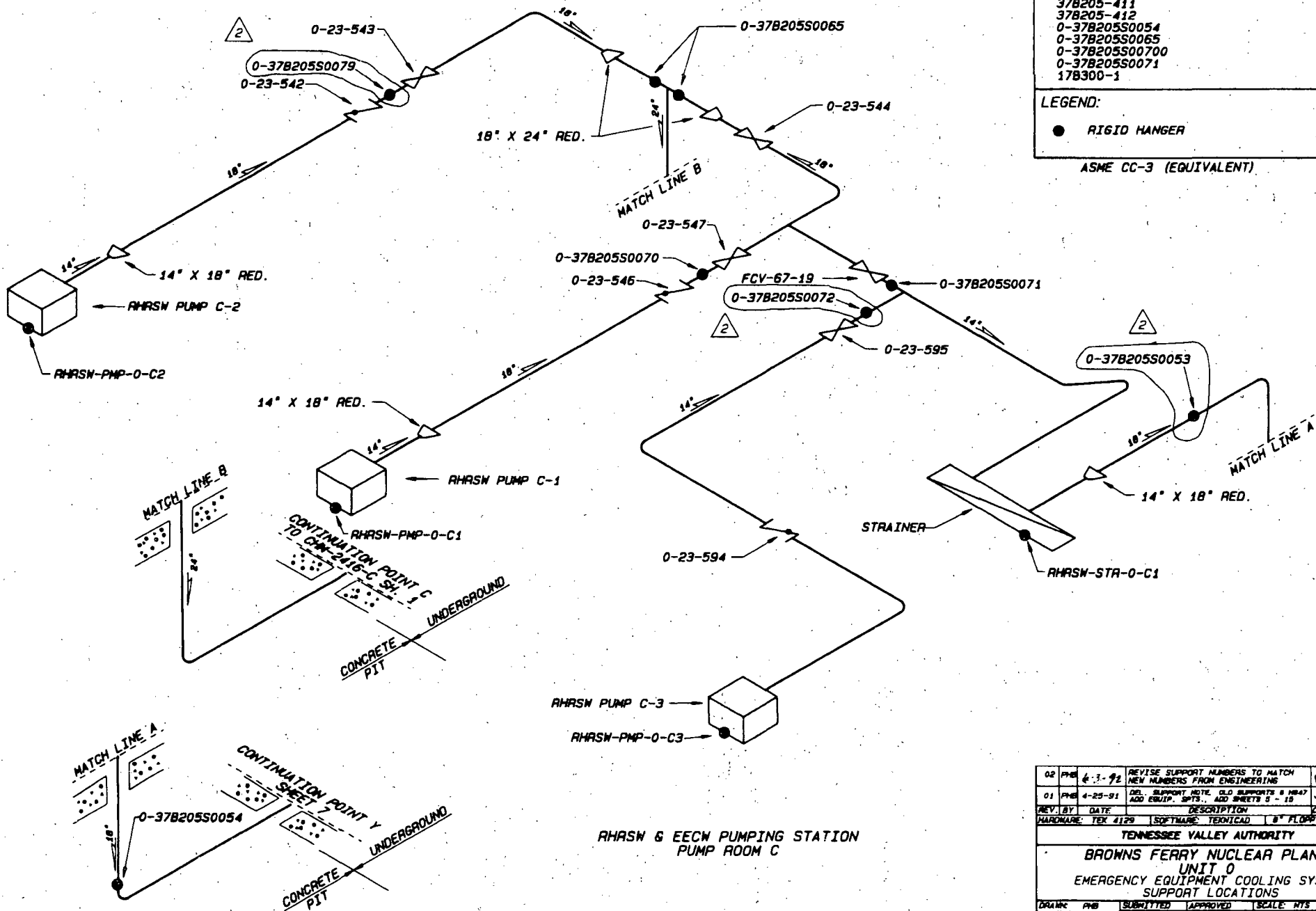
M.F.
R.L.

REFERENCE DRAWINGS

- 37B205-406
- 37B205-407/37B205-408
- 47B205-408
- 37B205-409
- 37B205-410
- 37B205-411
- 37B205-412
- 0-37B205S0054
- 0-37B205S0065
- 0-37B205S0070
- 0-37B205S0071
- 17B300-1

LEGEND:

- RIGID HANGER
- ASME CC-3 (EQUIVALENT)

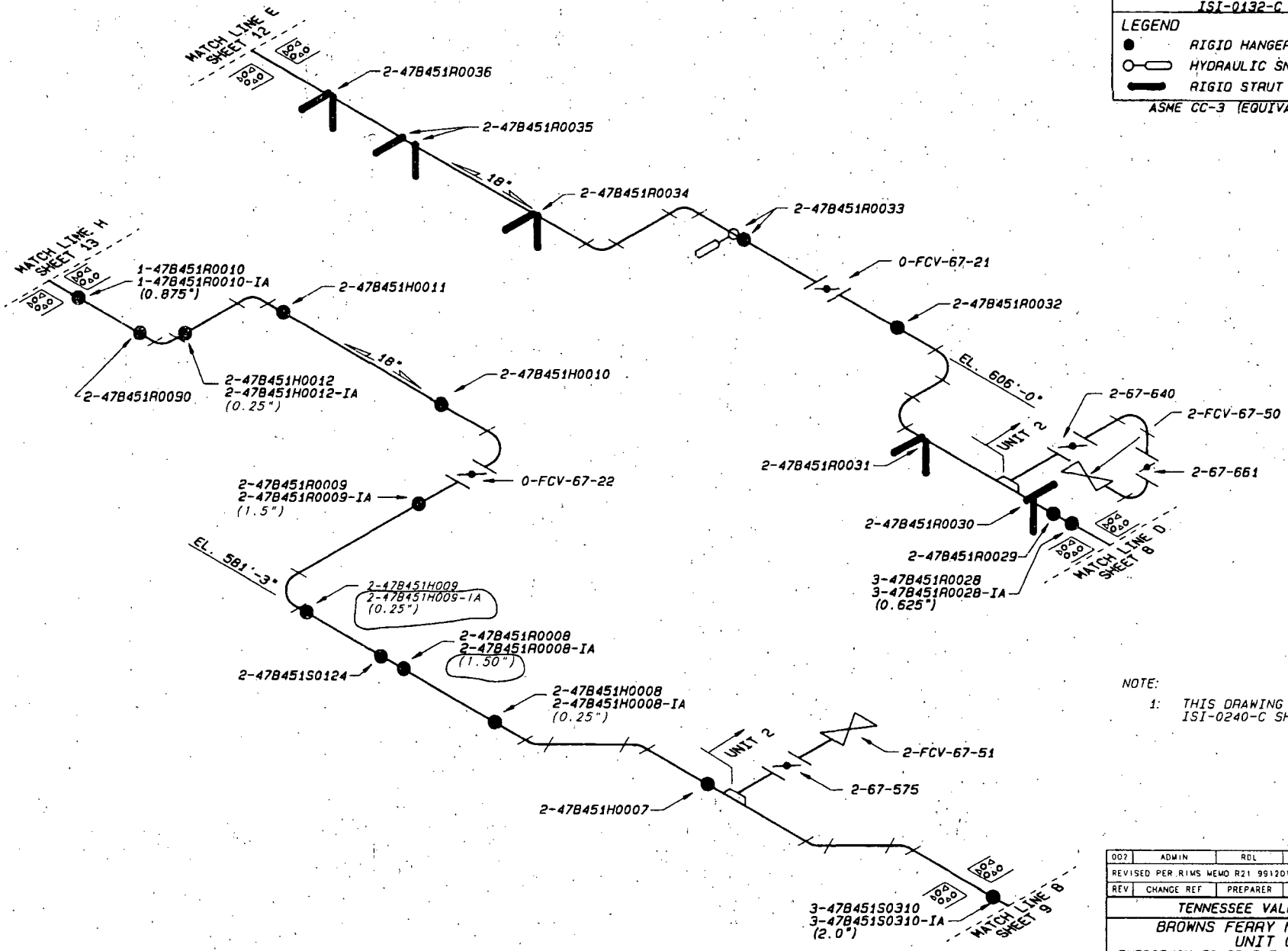


RHRSW & EECW PUMPING STATION
PUMP ROOM C

REV.	BY	DATE	DESCRIPTION	CR.	DSUB.	APP.
02	PMB	4-3-92	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	AB	MC	AB
01	PMB	4-25-91	DR. SUPPORT NOTE. OLD SUPPORTS 8 H847 ADD EQUIP. SPTS. ADD SHEETS 5 - 15	JAA	JES	GLB
HARDWARE: TEX 4129 SOFTWARE: TECHNICAL 8" FLOOR: BF-21						
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING SYSTEM SUPPORT LOCATIONS						
DRWING	PMB	SUBMITTED	APPROVED	SCALE	RTS	
DATE	3-21-90	DATE	4-17-90	DATE	4-17-90	SHEET 3 OF 15 SHEET(S)
CHECKED	JES	EOC	GLB	DRYING NO.	REV	
DATE	4-17-90			TS1-0368-C	02	

REFERENCE DRAWINGS
 47W451-H5
 47W451-H7
 NOTE: THIS DRAWING SUPERCEDES
 ISI-0132-C ALL SHEETS.

LEGEND
 ● RIGID HANGER
 ○ HYDRAULIC SNUBBER
 — RIGID STRUT
 ASME CC-3 (EQUIVALENT)



NOTE:
 1: THIS DRAWING SUPERCEDES
 ISI-0240-C SH. 1 REV. 4

00?	ADMIN	RDL	WCH	RLD	12-9-99
REVISED PER RIMS MEMO R21 991201 003					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0 & 2					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRAWN: PHB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 4-15-97	DATE: 4-18-97	DATE: 4-26-97	SHEET 6 OF 15 SHEET(S)		
CHECKED: JAA	JES	GLB	DRAWING NO.		
DATE: 4-18-97			0-157-0368-C 002		
					CCD

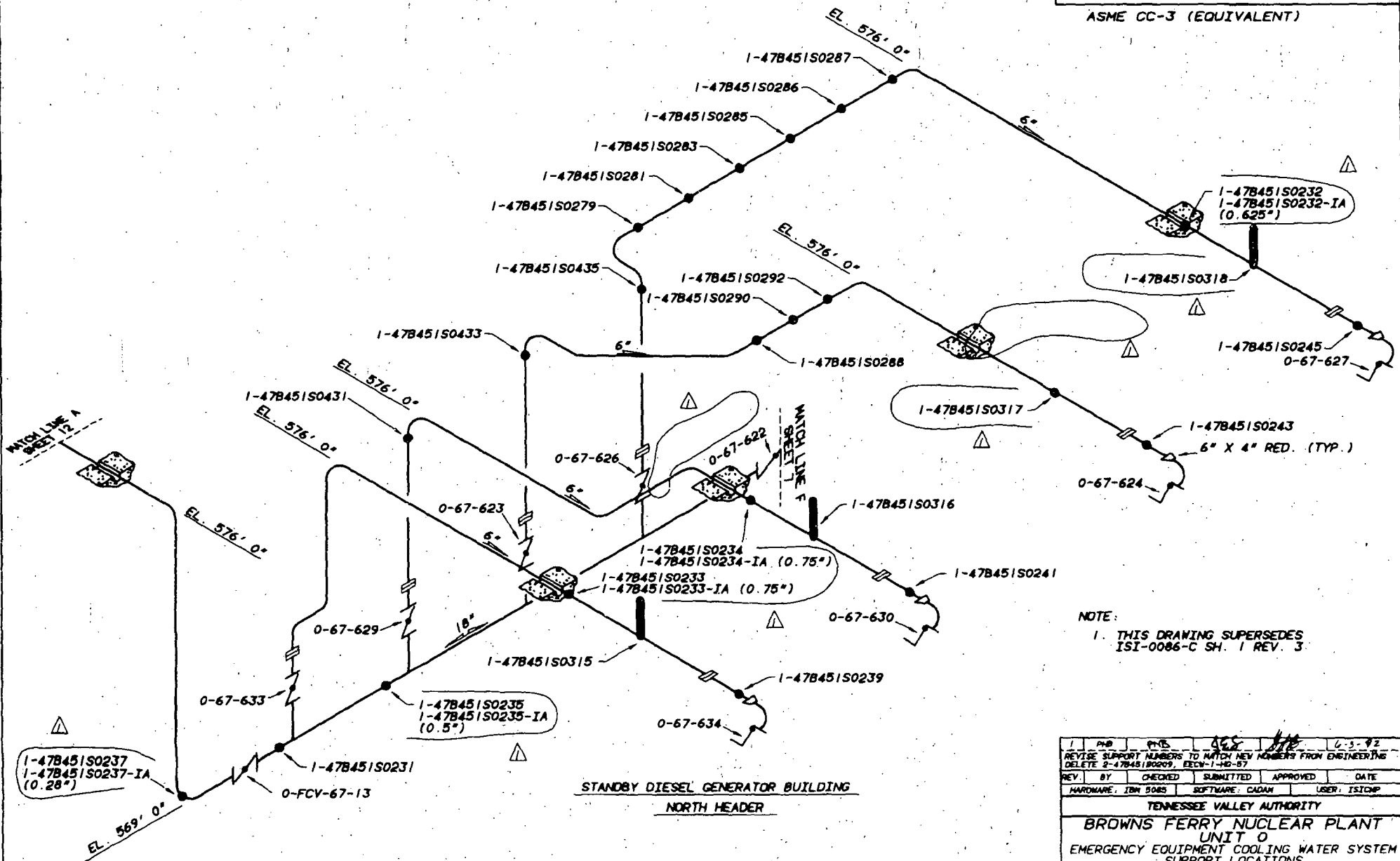
RF
 RI

REFERENCE DRAWING:
47W586 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

ASME CC-3 (EQUIVALENT)



STANDBY DIESEL GENERATOR BUILDING
NORTH HEADER

NOTE:

1. THIS DRAWING SUPERSEDES ISI-0086-C SH. 1 REV. 3

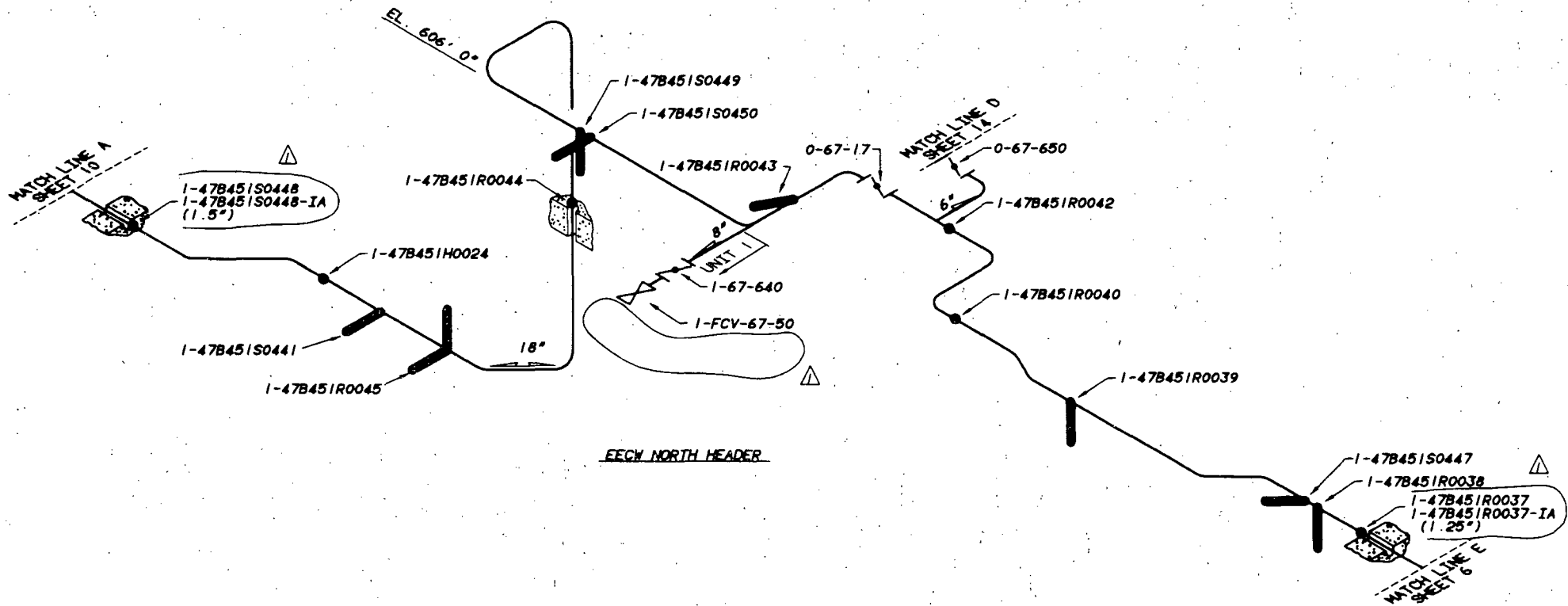
1	PHB	PHB	PHB	PHB	PHB	6-5-92
REVISE SUPPORT NUMBERS TO MATCH REV NUMBERS FROM ENGINEERING DELETE 2-47B45/S0209, EECV-1-ND-87						
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE	
HARDWARE	IBM 5085	SOFTWARE	CADAM	USER	ISTOMP	
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS						
DRWN.	PHB	SUBMITTED	APPROVED	SCALE	NTS	
DATE:	4-18-91	DATE:	4-18-91	DATE:	4-26-91	SHEET 10 OF 18 SHEET(S)
CHECKED:	JAA	DATE:	4-18-91	REV.		
			JES	GLB	ISI-0368-C101	

REFERENCE DRAWING:
47W586 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

ASME CC-3 (EQUIVALENT)



NOTE:

1. THIS DRAWING SUPERSEDES ISI-0086-C SH. 3 REV. 2

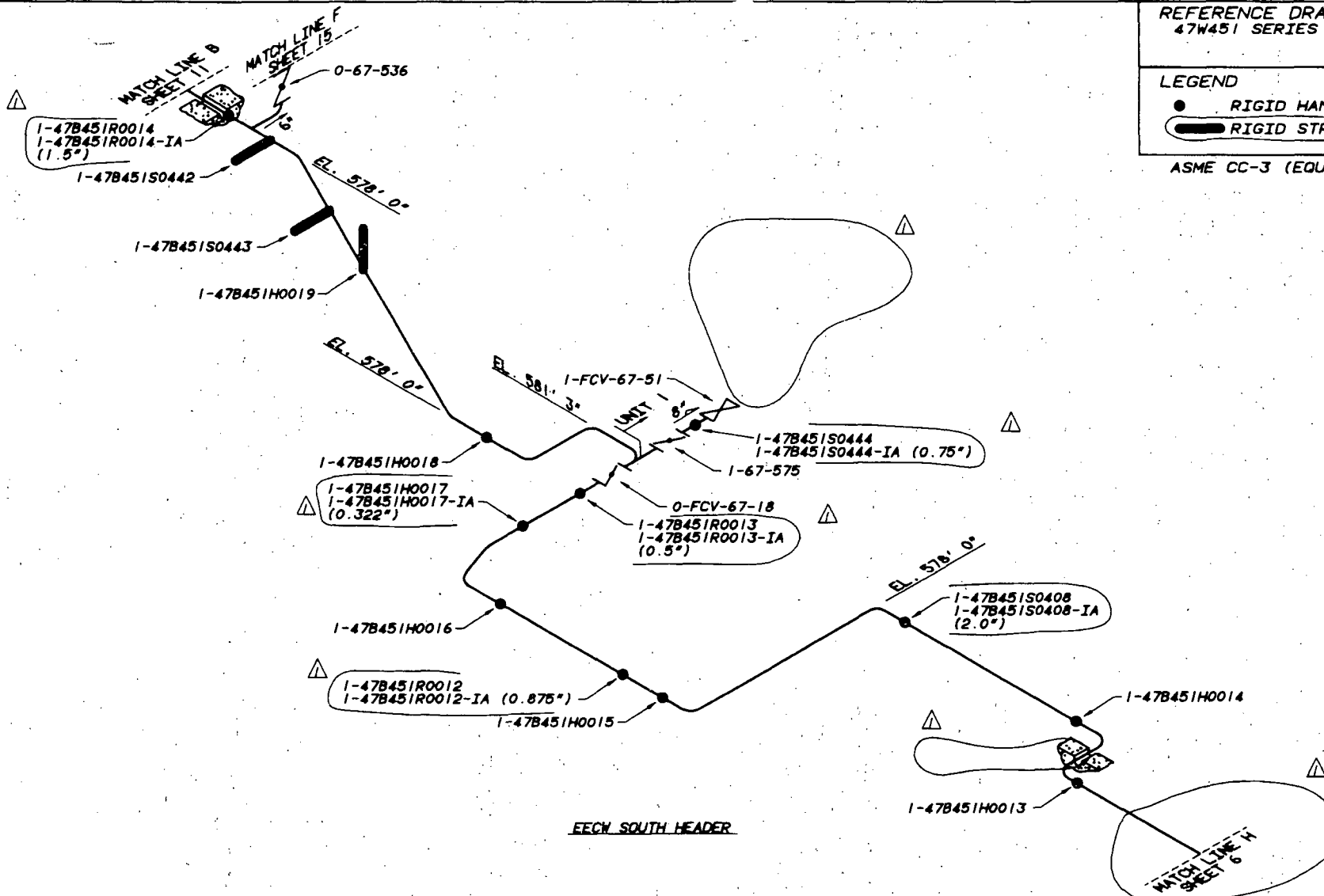
1	PHB	PHB	PHB	PHB	PHB	6-2-92
REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING DELETE SUPPORT OUTSIDE BOUNDARY						
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE	
HARDWARE: IBM 5085		SOFTWARE: CADAM		USER: ISTCP		
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT UNIT 0 & 1 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS						
DRAWN: PHB	SUBMITTED	APPROVED	SCALE: NTS			
DATE: 4-15-91	DATE: 4-16-91	DATE: 4-22-91	SHEET 12 OF 15 SHEETS(S)			
CHECKED: JAA	JES	GLB	DRAWING NO.	REV.		
DATE: 4-19-91			ISI-0368-C01			

REFERENCE DRAWING
47W451 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

ASME CC-3 (EQUIVALENT)



1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 4 REV. 2

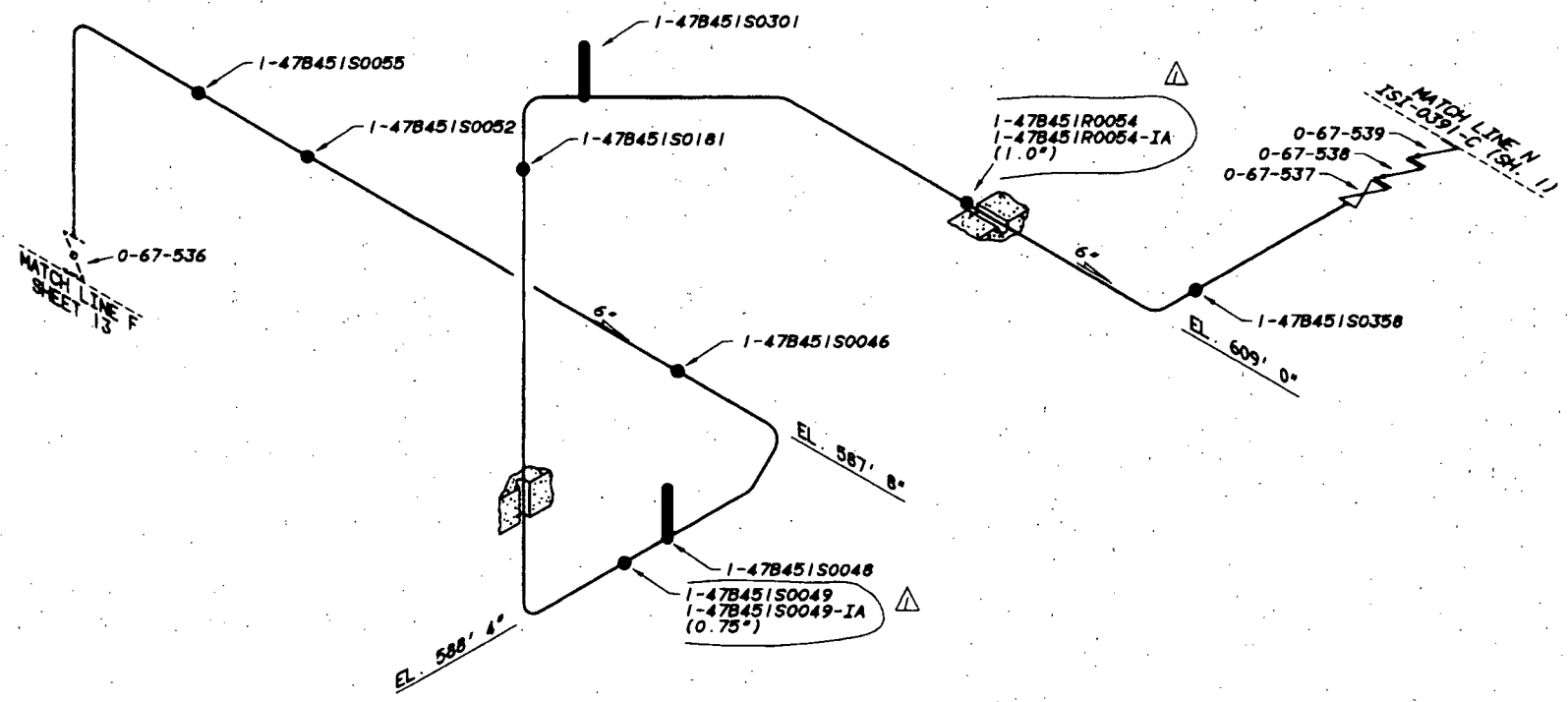
1	PHB	PNB	988	1/10	1-2-92
REVISE SUPPORT NUMBER TO MATCH NEW NUMBER FROM ENGINEERING					
CORRECT MATCH LINE H TO SHEET 6. ADD RIGID STRUT TO LEGEND					
DELETE SUPPORT OUTSIDE BOUNDARY					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE:	IBN 5088	SOFTWARE:	CADAM	USER:	ISICHP
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0 & 1					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRAWN:	PHB	SUBMITTED:	APPROVED:	SCALE:	NTS
DATE:	4-15-91	DATE:	4-16-91	DATE:	4-26-91
CHECKED:	JJA	DATE:	4-16-91	SHEET:	13 OF 15 SHEET(S)
DATE:	4-16-91	JES	CLB	DRAWING NO.:	ISI-0368-C01

REFERENCE DRAWINGS
47W451 SERIES
47W935 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

ASME CC-3 (EQUIVALENT)



NOTE:
1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 7 REV. 1

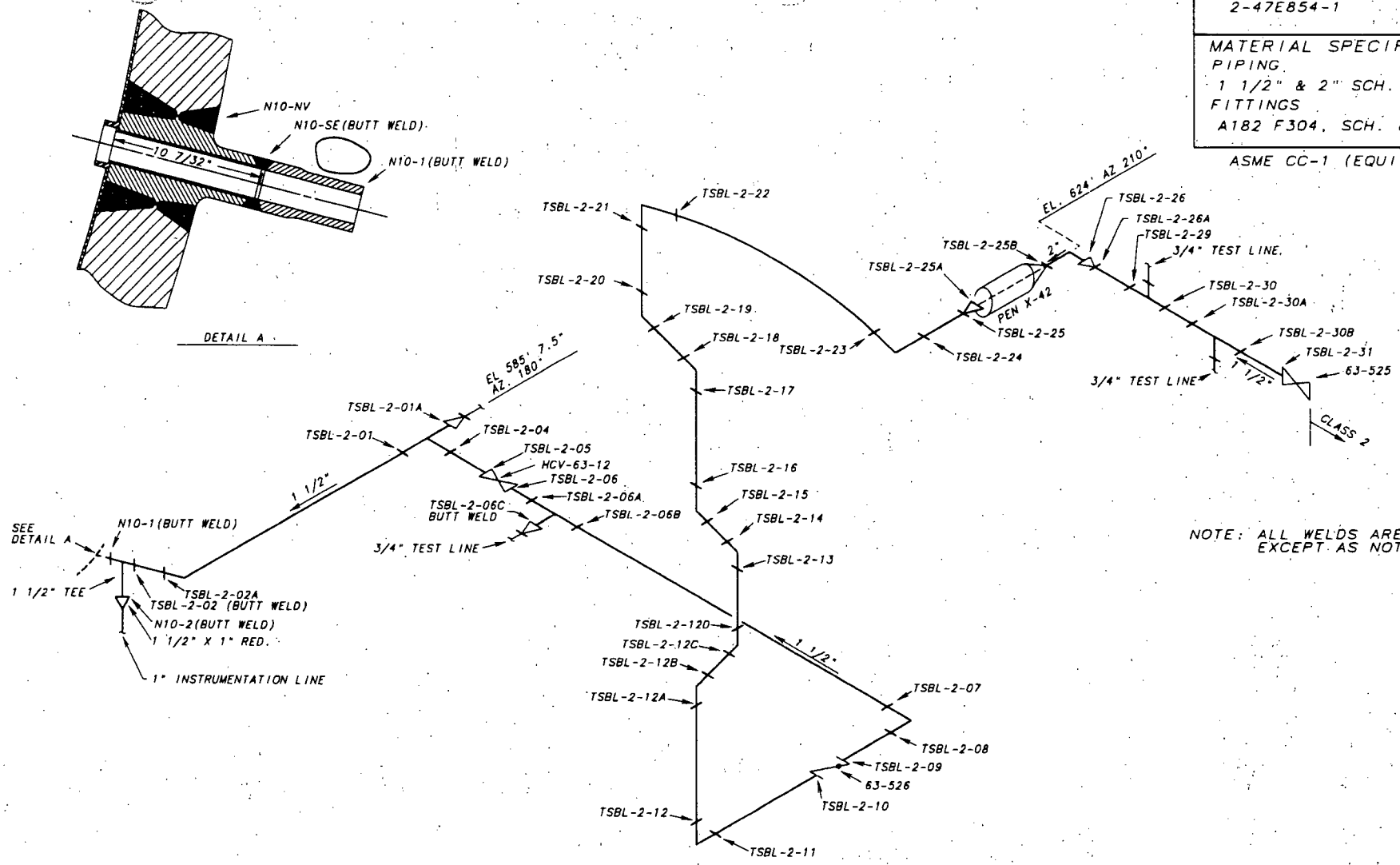
1	PHB	PHB	958	6-3-92
REVISE SUPPORT NUMBERS TO MATCH REV NUMBERS FROM ENGINEERING				
REV. BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE: IBM 5085	SOFTWARE: CADAM	USER: ISIOMP		
TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT				
UNIT: 0				
EMERGENCY EQUIPMENT COOLING WATER SYSTEM				
SUPPORT LOCATIONS				
DRAWN: PHB	SUBMITTED	APPROVED	SCALE NTS	
DATE: 4-17-91	DATE: 4-18-91	DATE: 4-26-91	SHEET 15 OF 15 SHEET(S)	
CHECKED: JAA	JES	GLB	DRAWING NO. REV.	
DATE: 4-18-91			ISI-0368-C01	

REFERENCE DRAWINGS:
 47W462 SERIES
 47B462 SERIES
 2-47E854-1

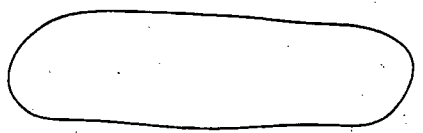
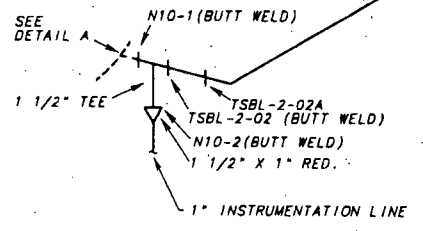
MATERIAL SPECIFICATIONS:
 PIPING:
 1 1/2" & 2" SCH. 80 A-312 TP 304
 FITTINGS:
 A182 F304, SCH. 80

ASME CC-1 (EQUIVALENT)

NOTE: ALL WELDS ARE SOCKET WELDED EXCEPT AS NOTED.



DETAIL A



001	ADMIN	RDL	WCH	HLH	5-29-91
REVISED PER RIMS MEMO R21 000821 001					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 STANDBY LIQUID CONTROL 1" TO 3" WELD LOCATIONS					
DRAWN:	PHB	DATE:	9-29-91	SCALE:	NTS CADAM/ISICMP
CHECKED:	JAA	APPROVED:	GLB	SHEET	01 OF 01 REV
SUBMITTED:	JES			2-151-0380-C 001	

ALL A/D HISTORY RESEARCHED AT ROOD

CAD MAINTAINED DRAWING

CCD

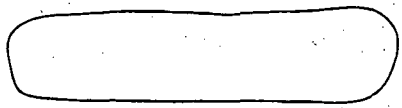
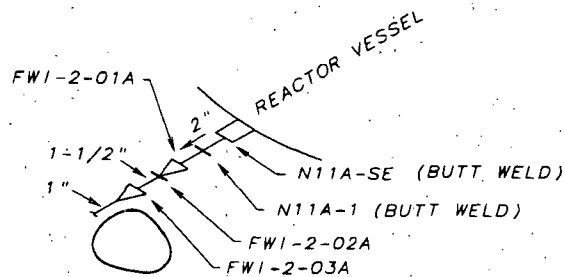
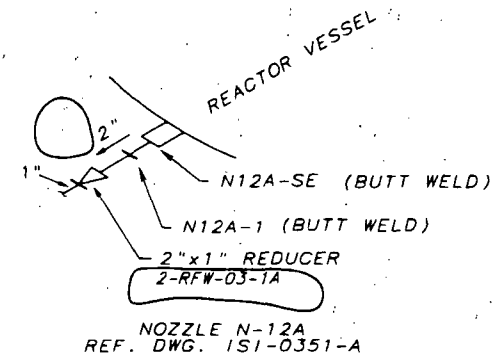
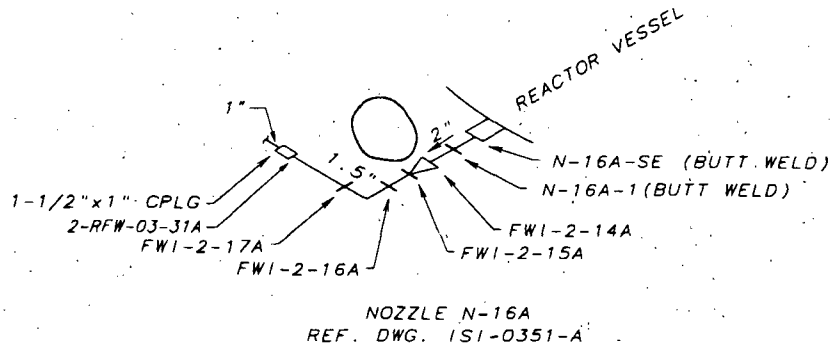
REFERENCE DRAWINGS:

2-47E803-5
 2-47W2650-100
 2-47E600-602
 47BM600-SERIES
 0-47W600-20

MATERIAL SPECIFICATIONS:

PIPING
 A312 OR A376 GR. TP-304 OR TP-316
 SCH. 80
 FITTINGS
 A182 GR. F-316

ASME CC-1 (EQUIVALENT)



001	ADMIN	ROL	ZLD	HEW	6-29-91
REVISED PER RIMS MEMO R21 000821 001					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
FEEDWATER INSTRUMENTATION					
WELD LOCATIONS					
DRAWN: PHB	SUBMITTED:	APPROVED:	SCALE: NYS		
DATE: 9-11-91	DATE: 9-20-91	DATE: 9-29-91	SHEET 01 OF 02 SHEET(S)		
CHECKED: JH	JES	CLB	DRAWING NO.	REV:	
DATE: 9-20-91			2-151-0383-C	001	

ALL A/D HISTORY RESEARCHED AT ROOD

CAD MAINTAINED DRAWING

CCD

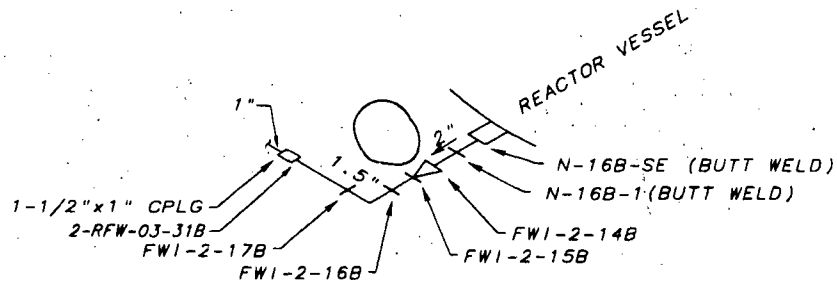
REFERENCE DRAWINGS:

2-47E803-5
 2-47W2650-100
 2-47E600-601
 47BM600-SERIES
 0-47W600-20

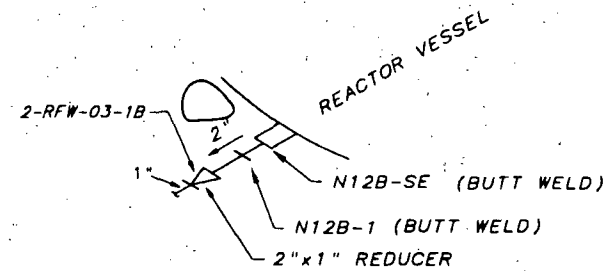
MATERIAL SPECIFICATIONS:

PIPING:
 A312 OR A376 GR. TP-304 OR TP-316
 SCH. 80
 FITTINGS:
 A182 GR. F-316

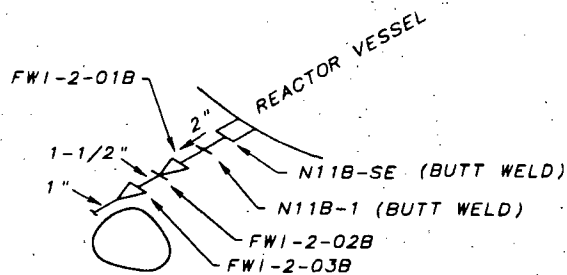
ASME CC-1 (EQUIVALENT)



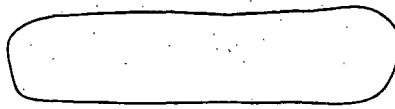
NOZZLE N-16B
 REF. DWG. ISI-0351-A



NOZZLE N-12B
 REF. DWG. ISI-0351-A



NOZZLE N-11B
 REF. DWG. ISI-0351-A



001	ADMIN	RDL	RLD	HEH	15-29-91
REVISED PER RIMS MEMO R21 000821 001					
REV	CHANGE	REF	PREPARER	CHECKER	APPROVED DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 FEEDWATER INSTRUMENTATION WELD LOCATIONS					
DRAWN: PHB	SUBMITTED	APPROVED	SCALE NYS		
DATE: 8-17-91	DATE: 8-20-91	DATE: 8-22-91	SHEET 02 OF 02 SHEET(S)		
CHECKED: JH	JES	GLB	DRAWING NO.	REV	
DATE: 8-20-91			2-151-0383-C 001		

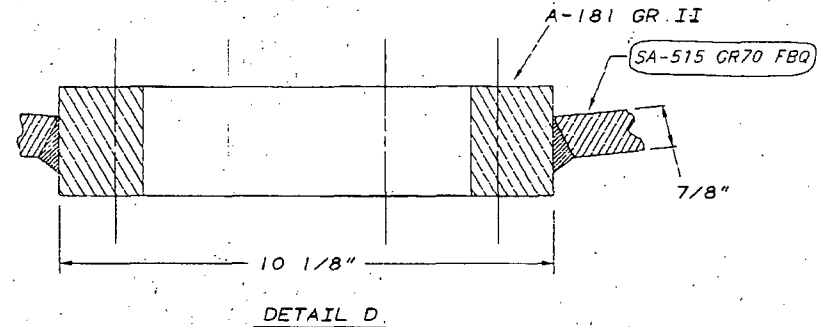
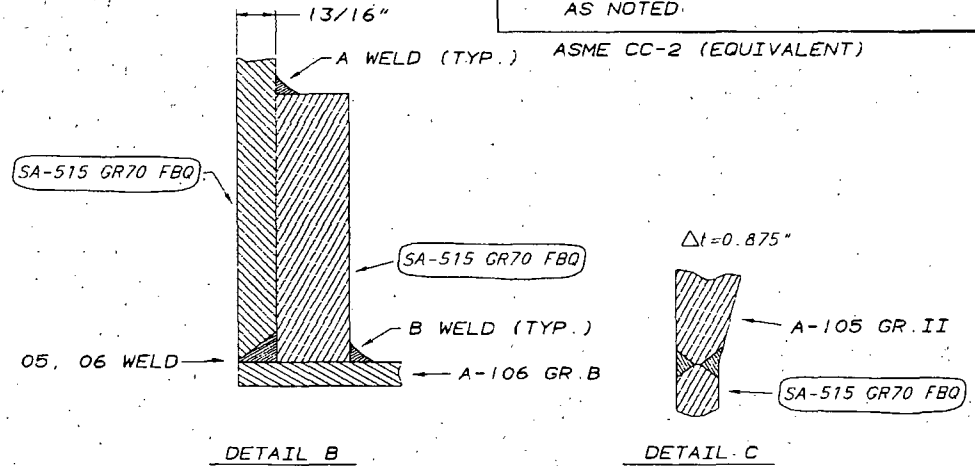
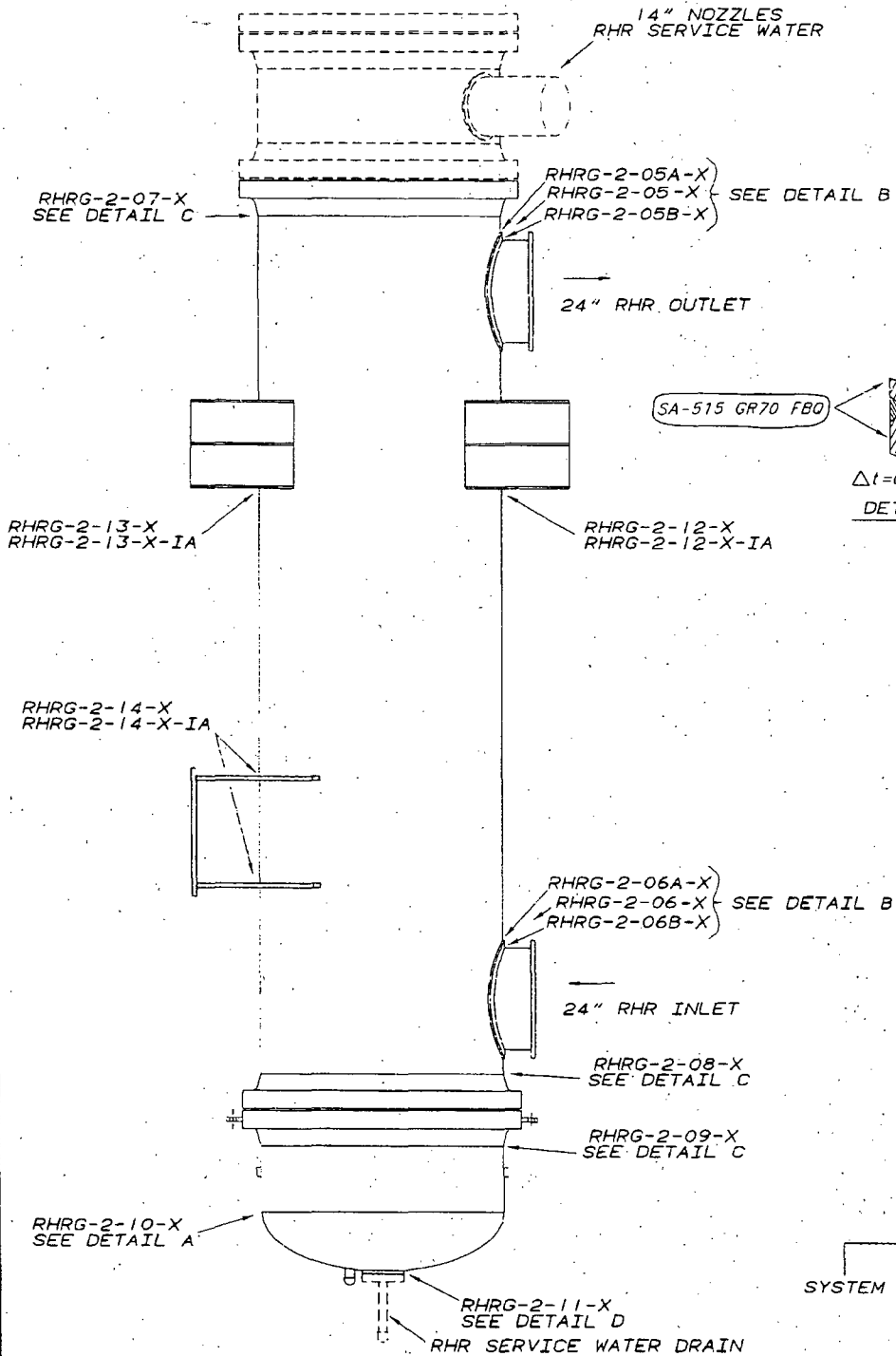
ALL A/D HISTORY RESEARCHED AT ROOD

CAD MAINTAINED DRAWING

CCD

REFERENCE DRAWINGS
 69-BF-165 PERFEX C
 69-D-160-05 PERFEX RP
 NOTE: THIS DRAWING SUPERSEDES
 ISI-0314-B

MATERIAL SPECIFICATIONS
 AS NOTED

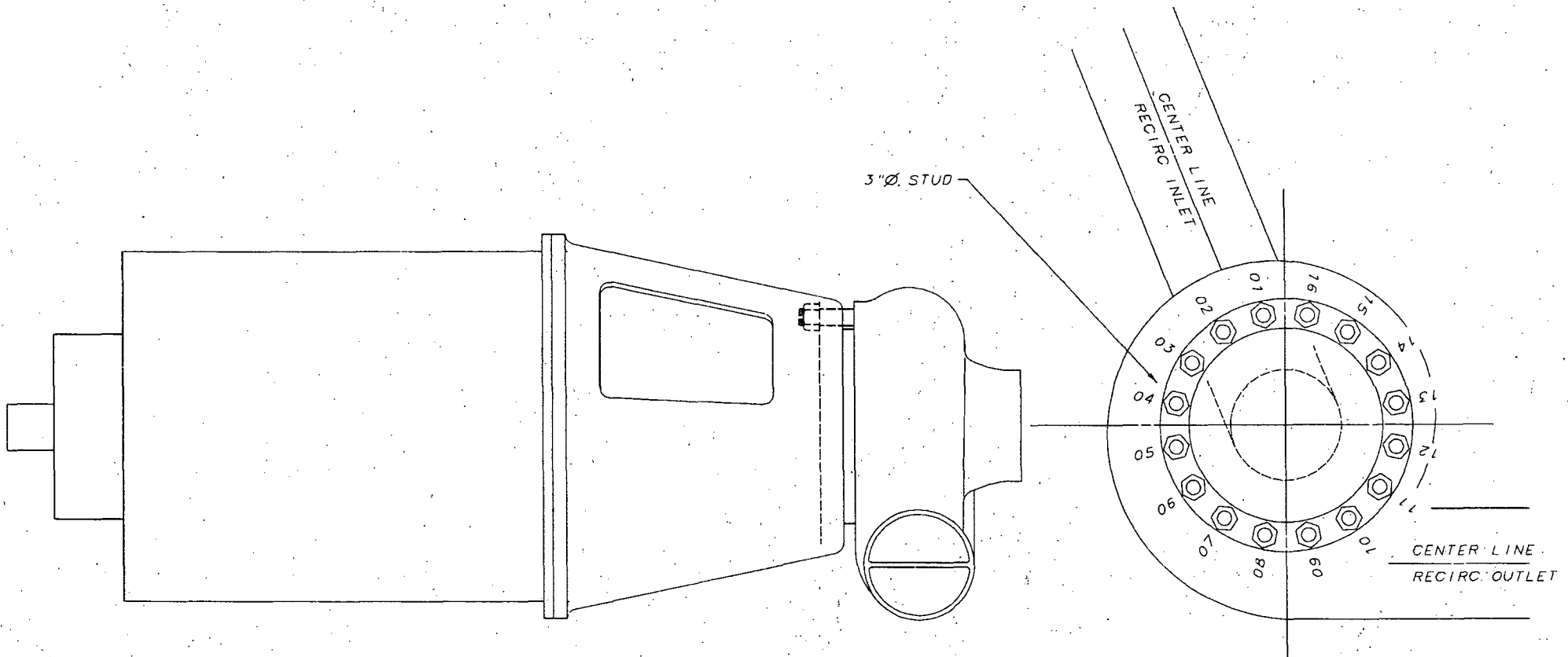


RHRG-2- -X
 SYSTEM UNIT WELD NO. HEAT EXCHANGER LETTER (A,B,C,OR D)

001	ADMIN	RJP	JT	RLD/H	
REVISED FOR RIMS MEMO R21 990604 007, REMOVED DRAWING CLASSIFICATION (ADMINISTRATIVE REVISION)					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL HEAT EXCHANGER WELD LOCATIONS					
DRAWN:	PHB	DATE:	3-13-92	SCALE:	NYS CADAM/215CMP
CHECKED:	RPG	APPROVED:		SHEET	01 OF 01 REV
SUBMITTED:	JES		DLB	2-151-0406-C	001
					CCD

REFERENCE DRAWINGS
 2F-1177 BYRON JACKSON DIV. ORG-WARNER
 1E-3429 BYRON JACKSON DIV. ORG-WARNER
 153F754 GENERAL ELECTRIC

ASME CC-1 (EQUIVALENT)



NOTE:
 1. MATERIAL ASTM A540 GR B23.

PUMP A

NUMBERS (01-16) ARE PREFIXED BY:

- PUMP APMP-A-STUD-2-
- PUMP APMP-A-NUT-2-
- PUMP APMP-A-WASH-2-
- PUMP APMP-A-FLG-2-

PUMP B

NUMBERS (01-16) ARE PREFIXED BY:

- PUMP BPMP-B-STUD-2-
- PUMP BPMP-B-NUT-2-
- PUMP BPMP-B-WASH-2-
- PUMP BPMP-B-FLG-2-

PUMP A

PUMP INTERIOR IDENTIFIERS:

- PUMP APMP-2A-INTERIOR

PUMP B

PUMP INTERIOR IDENTIFIERS:

- PUMP BPMP-2B-INTERIOR

CCO	ADMIN	ROL	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
000	ISSUED TO CREATE CCD SUPERSEDES AS-DESIGNED ISI-0407-C-1 R1 AND TO DEFICIENCY AS CONSTRUCTED STATUS PER RINS MEMO #14 870505 307; A/D RG									
REV	NO	CHANGE REF	DATE	DTTR	CHKR	DSGN	RVSR	APPC	APPD	ISSD
S	TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT										
UNIT 2										
RECIRCULATION PUMP										
BOLTING LOCATIONS & PUMP INTERIORS										
DRAWN:	PHB	SUBMITTED	DATE:	APPROVED	DATE:	SCALE	NTS			
DATE:	3-13-92	DATE:				SHEET 01 OF 01 SHEET(S)				
CHECKED:	BPG	DATE:		OPB	DATE:	DRAWING NO. 2-ISI-0407-C-000				
DATE:		DATE:			DATE:	REV.				
CCD										

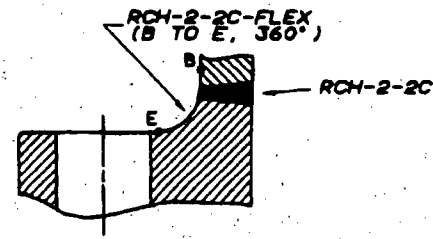
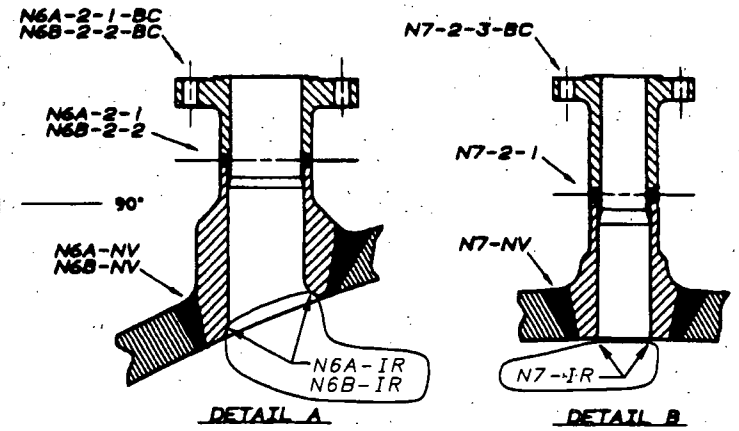
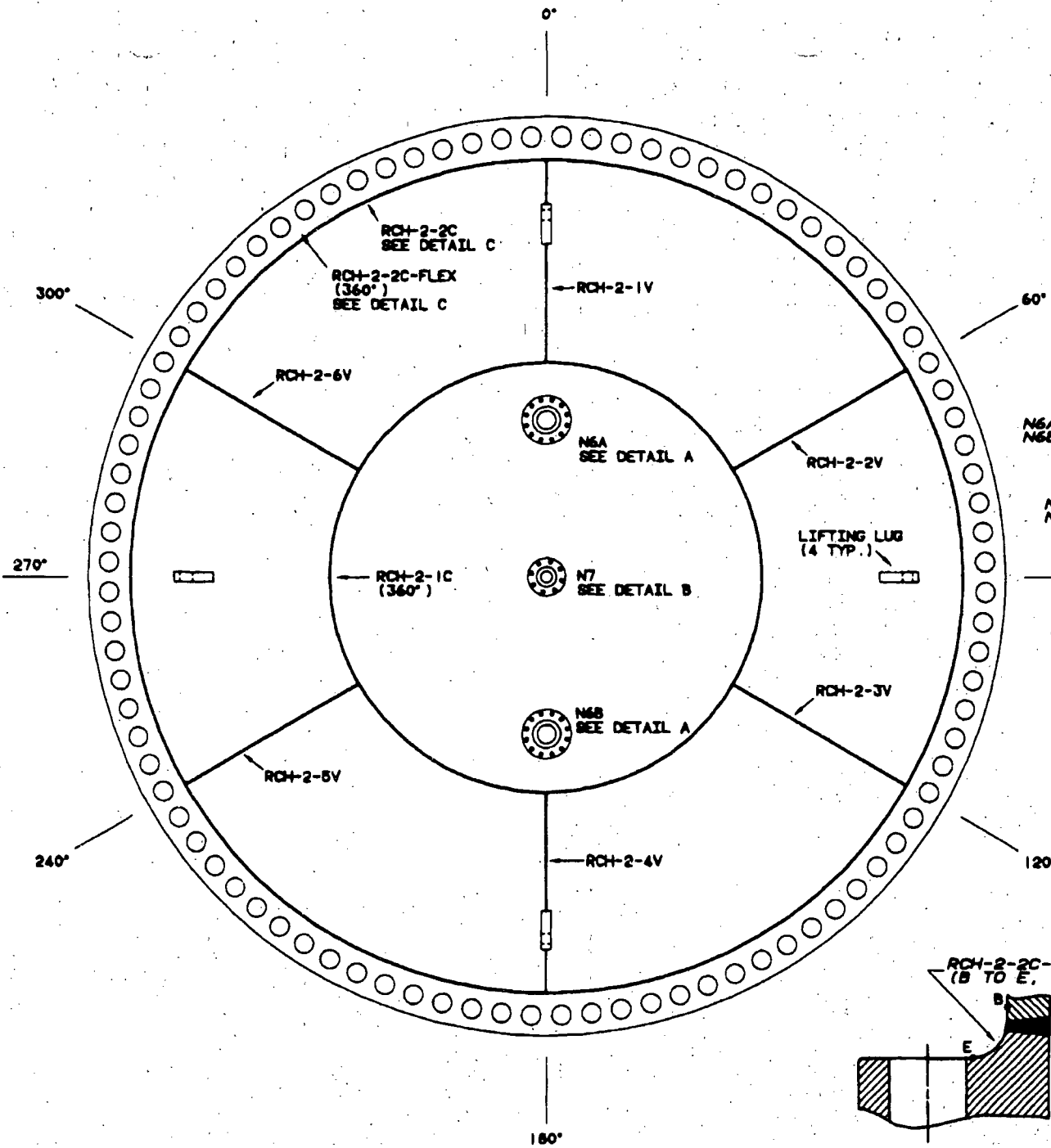
REFERENCE DRAWINGS
 B&W 122876
 B&W 122877
 NOTE: THIS DRAWING SUPERSEDES
 ISI-0294-A SH 1

MATERIAL SPECIFICATIONS

CLOSURE HEAD DOME/SEGMENTS
 CS MN/MO

N-7
 NOZZLE - A508 CL.2 (MN-MO)
 LONG WELD NECK - SA-106 GR. II CS

N6A, N6B
 NOZZLE - A508 CL.2 (MN-MO)
 LONG WELD NECK - SA-106 GR. II CS
 ASME CC-1 (EQUIVALENT)



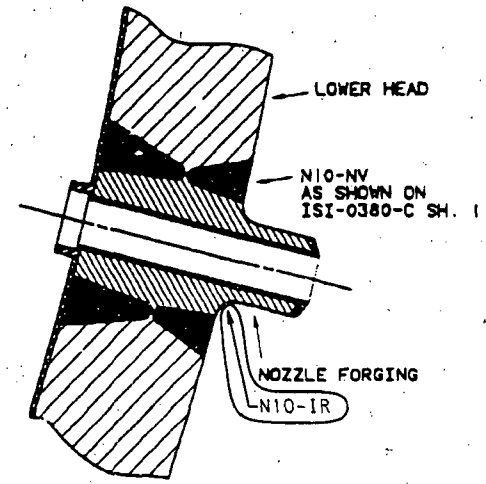
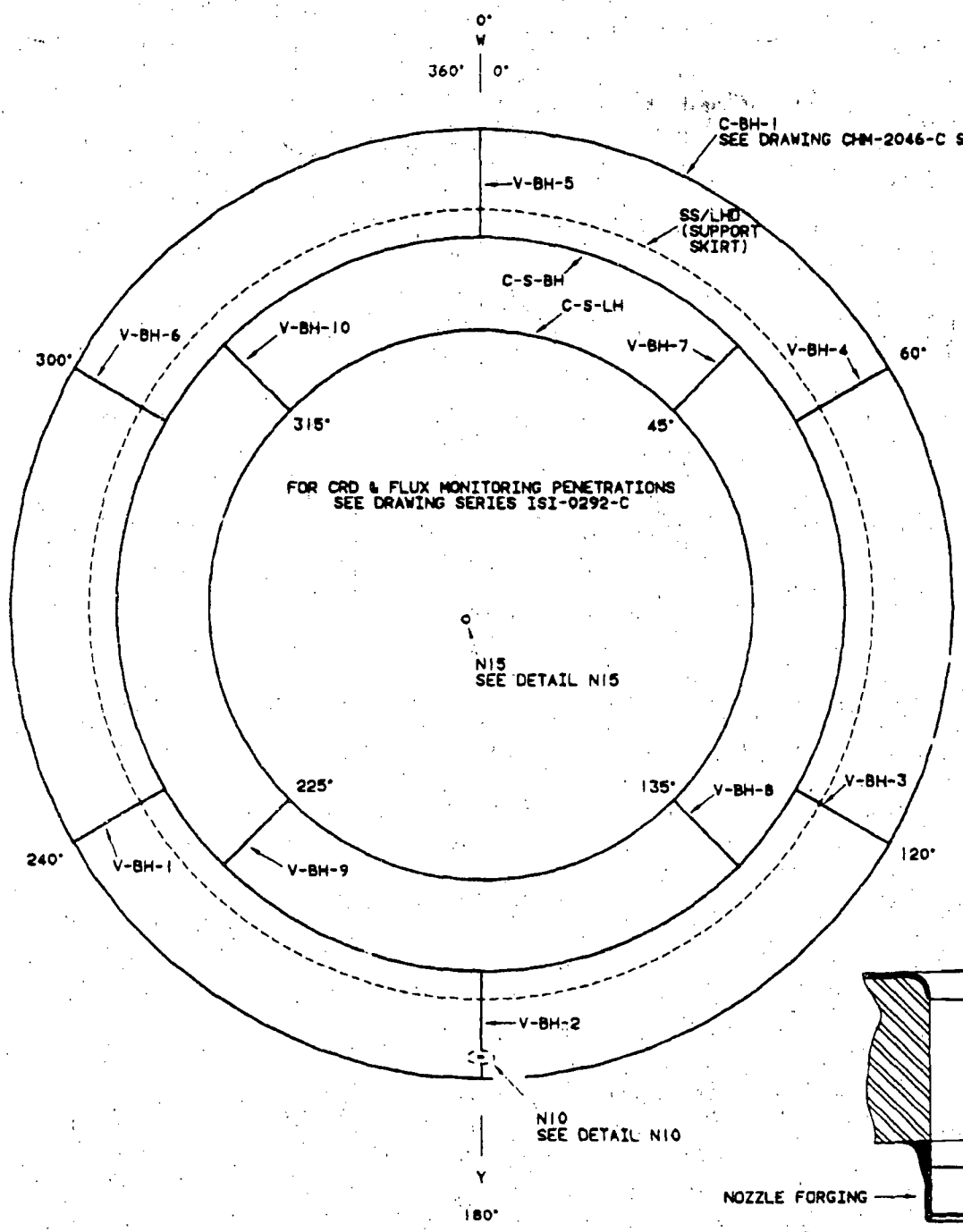
000	CCD/ADMIN	J MCFARLAND	WCHODGES	DP WILKER	PA 200
ISSUED TO CREATE CCD SUPERSEDES A/D ISI-0408-C-1 R2; REVISED PER RIMS MEMO R14 04D129 101					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
		HARDWARE: JIM BOOS	SOFTWARE: CADAM	USER: JATOMP	
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
CLOSURE HEAD ASSEMBLY WELD LOCATIONS					
DRAWN: P/B	DESIGNED: JPC	APPROVED: JPC	SCALE: NYS		
DATE: 3-75-82	DATE: 3-75-82	DATE: 3-75-82	SHEET: 01 OF 01	REV: 01	
DESIGNED: JPC	DATE: 3-75-82	DATE: 3-75-82	DRAWING NO: 2-151-0408-C-1000		

CAD MAINTAINED DRAWING CCD

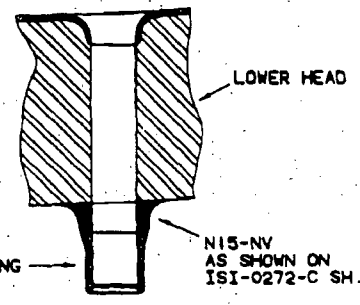
REFERENCE DRAWINGS
 GE SKETCH SK-B2008
 B&W 122856E-11
 B&W 122859E-10
 GE SKETCH SK-B2022
 GE SKETCH SK-B2023

MATERIAL SPECIFICATIONS
 BOTTOM HEAD
 CS MN/MO
 N10
 NOZZLE - A508 CL.2 (MN-MO)
 N15
 NOZZLE - SA-105

ASME CC-1 (EQUIVALENT)



DETAIL N10



DETAIL N15

000	CCD/ADMIN	J MCFARLAND	WCHOOGES	DP WALKER	2-4-04
ISSUED TO CREATE CCD SUPERSEDES A/D ISI-0444-C-1 R1; REVISED PER RIMS MEMO R14 040129 101					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 BOTTOM HEAD ASSEMBLY WELD LOCATIONS					
DRAWN:	PHB	DATE: 01/20/04	SCALE: NTS	CADIAN/ST/DP	
CHECKED:	PHB	APPROVED:		SHEET 01 OF 01	REV
SUBMITTED:	PHB	BY: PHB		2-ISI-0444-C	000

CAD MAINTAINED DRAWING

CCD

NOZZLE	DISTANCE TO MATING SURFACE
N12X	86.5"
N11X	146"
N4X	228"
N5X	246.5"
N9	259.5"
N16X	296.5"
N2X	379"
N1X	564"
N8X	583.5"
N8X	610"

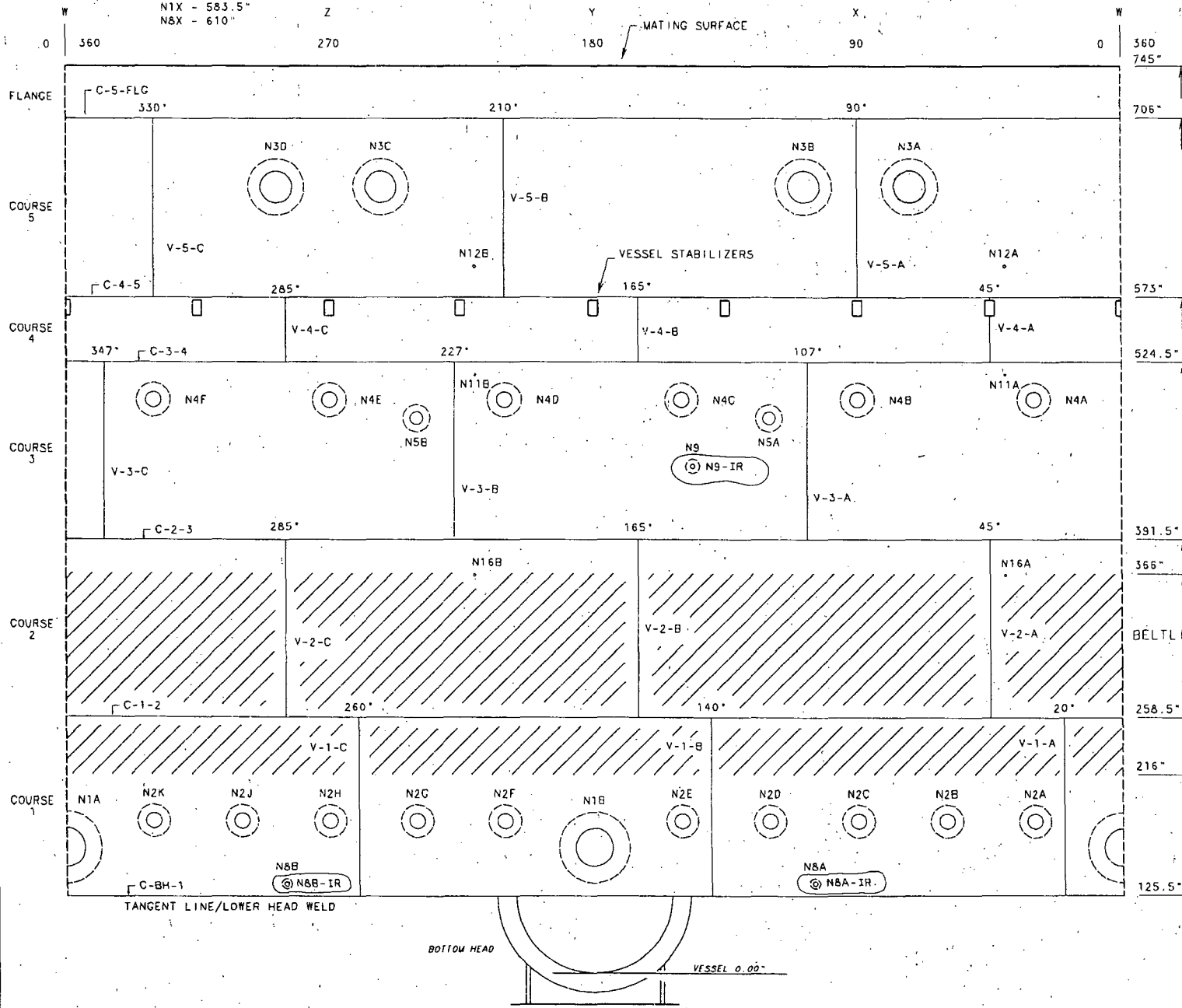
REFERENCE DRAWINGS (GE)

SKETCHES - RPV EXAMINATION V (GE)
 SK-B2001 SK-B2005 10
 SK-B2003 SK-B2007 10
 SK-B2004 SK-B2006

LEGEND

○ VESSEL NOZZLE
 ⊙ FULL PENETRATION NOZZLE WELD

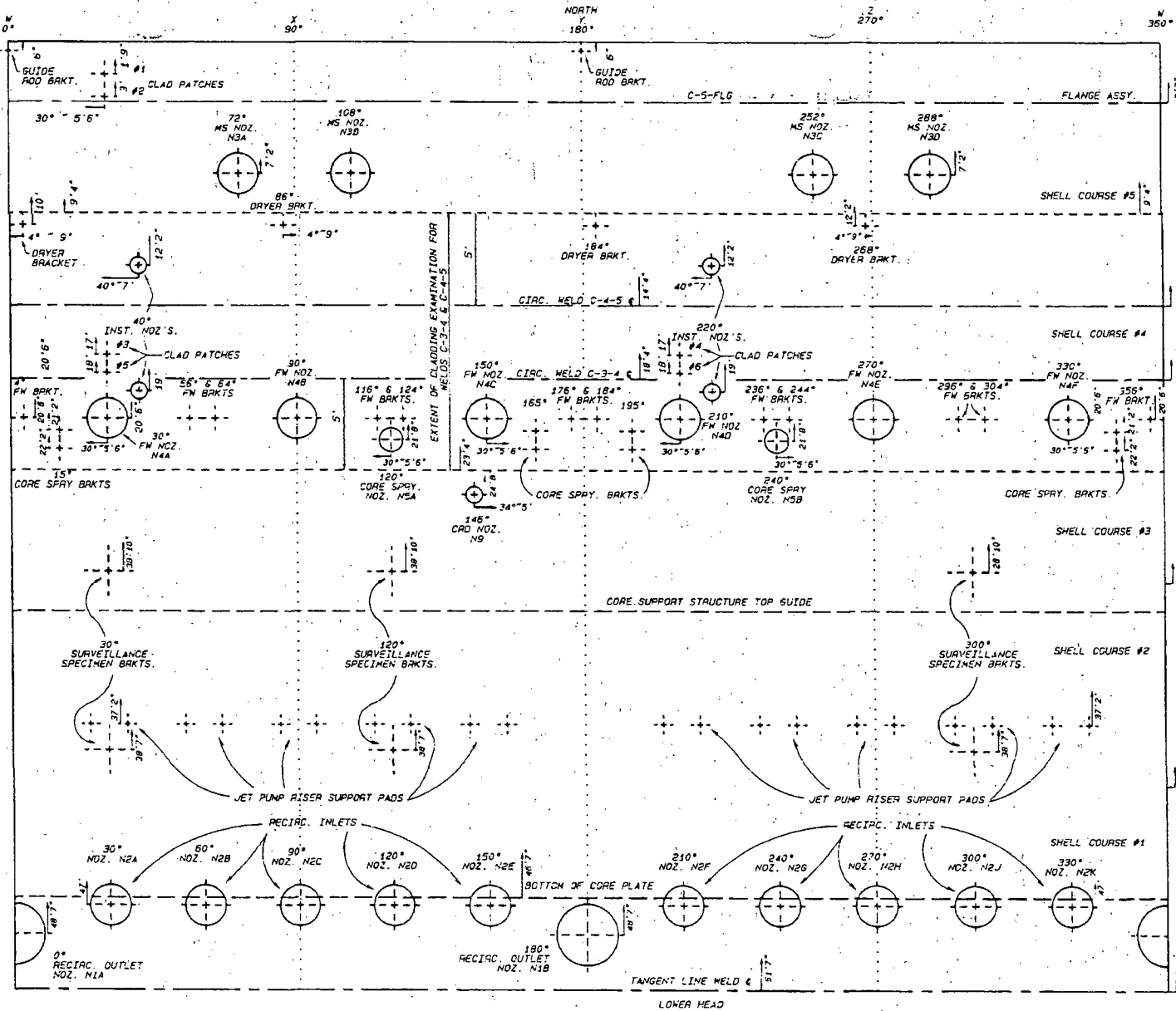
ASME CC-1 (EQUIVALENT)



NOTES:
 1. REFER TO RPV MANUAL FOR MATERIAL SPECIFICATION AND MATERIAL THICKNESS.
 2. NOZZLES N-11A, N-11B, N-12A, N-12B, N-16A, AND N-16B ARE CATEGORY B-E.

ADMIN	RDL	N/A	N/A	N/A	N/A	N/A	N/A
REVISED PER RIMS MEMO R14 970505 3D2							
CHANGE REF	DATE	DFTR	CHKR	DSGN	RVWR	APPR	ISSD
TENNESSEE VALLEY AUTHORITY							
BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR PRESSURE VESSEL (RPV) SHELL COURSE WELD/NOZZLE LOCATIONS (OUTSIDE VIEW)							
DRAWN: N/A	DATE: N/A	SCALE: NTS	CADAM/ISICMP				
CHECKED: N/A	APPROVED:	SHEET 01 OF 02		REV			
SUBMITTED: N/A	GLB	2-CHM-2046-C		002			

REFERENCE DRAWINGS
 500733-001C (MIRR (ISULATION)
 24187-F (B&W)
 122856E



- NOTES:
- FOUR (4) STEAM DRYER HOLDDOWN BRACKETS (NOT SHOWN) ARE LOCATED IN THE VESSEL CLOSURE HEAD 10" UP FROM THE FLANGE. THESE ARE AT AZIMUTH LOCATIONS 41°, 139°, 221°, AND 319°.

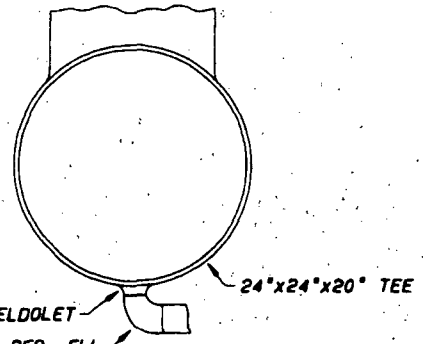
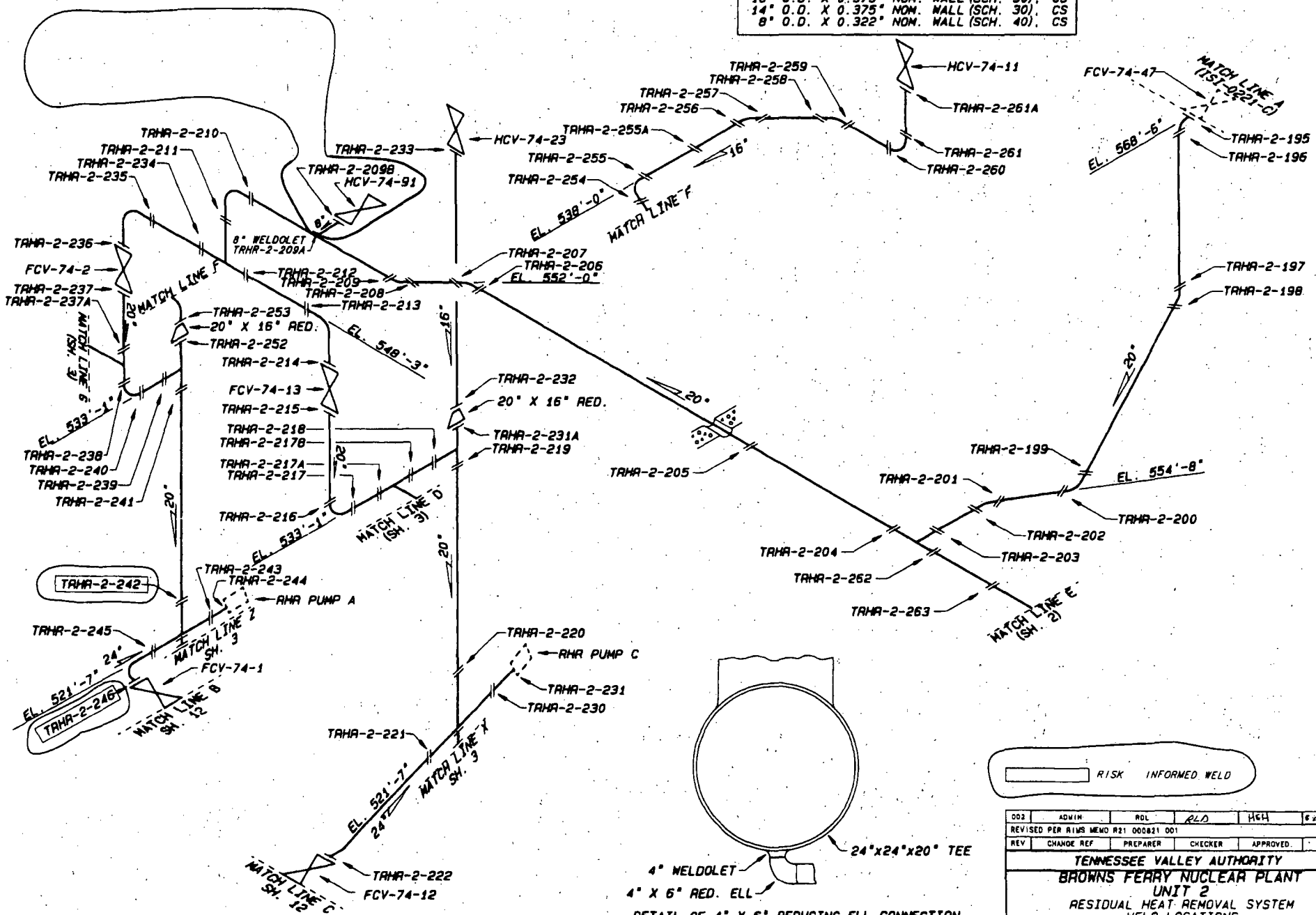
- CODE CATEGORY B-N-1 RPV INTERIOR
 CODE CATEGORY B-N-2 RPV-INT ATT BLR ①
 RPV-INT ATT NBLR ②
 RPV-CORE SUPPORT
 ① INTEGRAL ATTACHMENT BELTLINE REGION
 ② INTEGRAL ATTACHMENT NON BELTLINE REGION

M7
 Rel

REV NO	CHANGE REF	DATE	BY	CHKR	DESCR	QWR	APPR	APPR	ISSD
000									
TENSSESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR VESSEL WELD AND NOZZLE LOCATIONS (INTERIOR WALL)									
DRAWN: PHB		SUBMITTED: 5-23-89		APPROVED: 5-23-89		SCALE: NTS			
DATE: 8-26-88		DATE: 5-23-89		DATE: 5-23-89		SHEET 02 OF 02 SHEET(S)			
CHECKED: JES		EDC		GLB		DRAWING NO. 2-CHM-2046-C		REV 000	
DATE: 5-23-89									

MATERIAL SPECIFICATIONS		
ASME CC-2 (EQUIVALENT)		
24" O.D. X 0.375" NOM. WALL (SCH. 20),	CS	
20" O.D. X 0.375" NOM. WALL (SCH. 20),	CS	
16" O.D. X 0.375" NOM. WALL (SCH. 30),	CS	
14" O.D. X 0.375" NOM. WALL (SCH. 30),	CS	
8" O.D. X 0.322" NOM. WALL (SCH. 40),	CS	

REFERENCE DRAWINGS	
47N452 SERIES	
47N335-4	



4" WELDOLET
 4" X 6" RED. ELL
 DETAIL OF 4" X 6" REDUCING ELL CONNECTION

RISK INFORMED WELD

003	ADMIN	ROL	RLD	HCH	6/29/2011
-----	-------	-----	-----	-----	-----------

REVISED PER AIMS MEMO R21 000821 001

REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM WELD LOCATIONS					

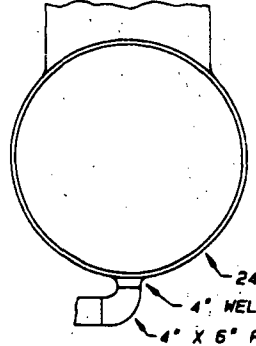
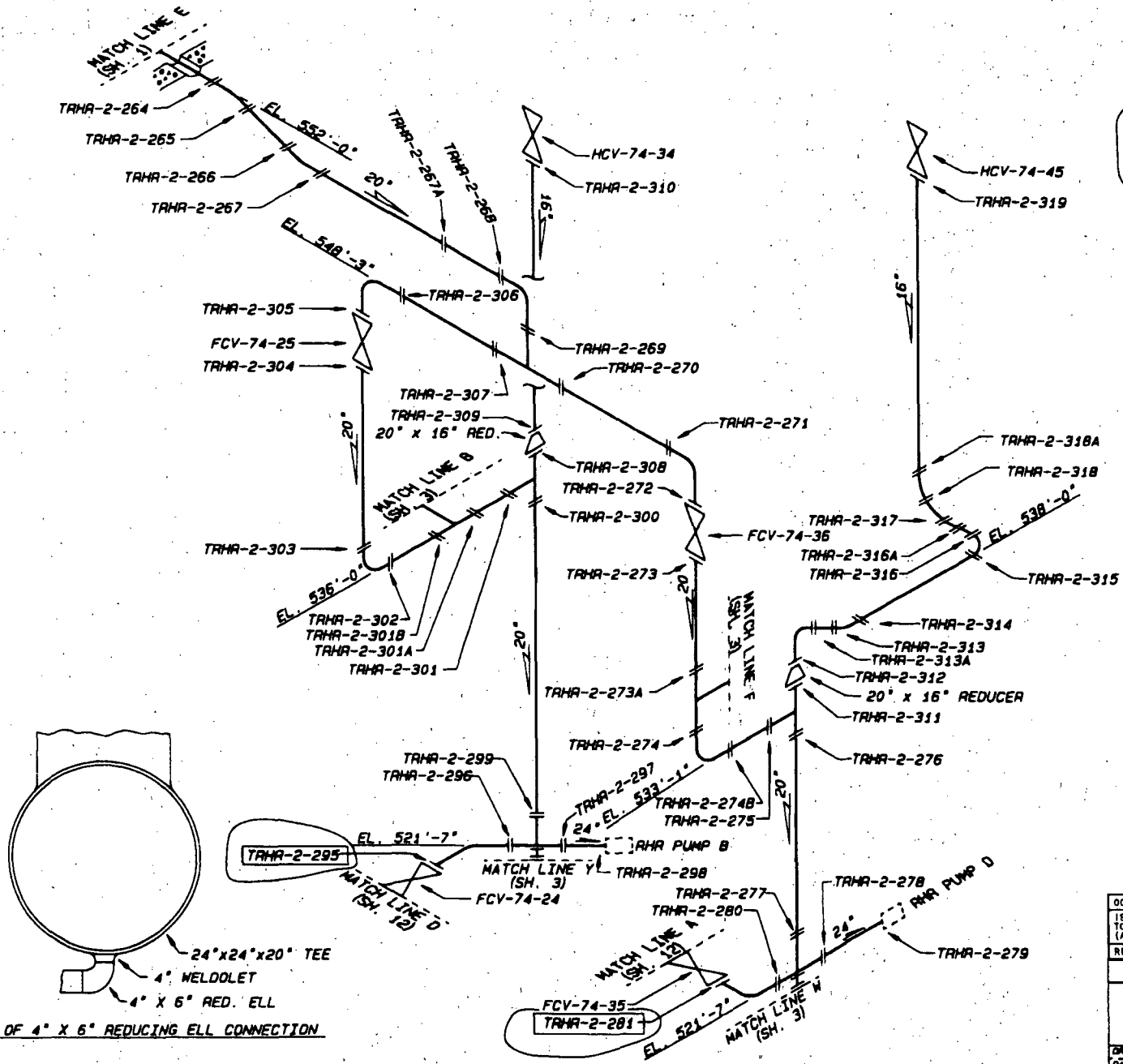
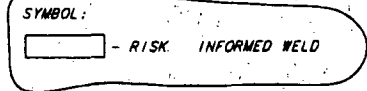
DRAWN	REV	SUBMITTED	APPROVED	SCALE	NTS
DATE	3-31-01	DATE	DATE	SHEET 1 OF 12 SHEET(S)	
CHECKED	JTL	DATE	GLB	DRAWING NO.	REV
				2-MSG-0018-C	002

CCD

ALL A/D HISTORY RESEARCHED AT ROOD

CAD MAINTAINED DRAWING

REFERENCE DRAWINGS	
47N452 SERIES	
47N335-4	
MATERIAL SPECIFICATIONS	
ASME CC-2 (EQUIVALENT)	
24" X .375" NOM. WALL (SCH. 20)	CS
20" X .375" NOM. WALL (SCH. 20)	CS
16" X .375" NOM. WALL (SCH. 30)	CS
14" X .375" NOM. WALL (SCH. 30)	CS



DETAIL OF 4" X 6" REDUCING ELL CONNECTION

000	CCD/ADMIN	RGP	RLD	HEH	9-28-2011
ISSUED TO CREATE CCD, SUPERSEDES A/D MSG-0018-C-2 R004 AND TO DEPICT AS-CONSTRUCTED STATUS PER RIMS MEMO R21 000821 001 (ADMINISTRATIVE REVISION)					
REV	CHANGE REF	PREPARER	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
WELD LOCATIONS					
DRAWN BY	DESIGNED	APPROVED	SCALE	SHEET 1 OF 18 SHEET ID	
DATE 7-1-01	DATE	DATE		REVISION NO.	REV
PREPARED BY	GLB			2-MSG-0018-C	000
DATE					

ALL A/D HISTORY RESEARCHED TO R000

CAD MAINTAINED DRAWING CCD

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX VI

SUMMARY OF INDICATIONS

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
--	--

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Summary of Indications

Indications detected during the performance of examinations for Browns Ferry Nuclear Plant Unit 2/Cycle 14 were evaluated in accordance with approved written procedures. Generally, examination results yielded either No Recordable Indications (NRI) or Recordable Indications.

Recordable Indications were evaluated to determine their origin. Indications determined to be of a geometric, metallurgical, or similar origin were typically dispositioned as non-relevant. Indications determined to be of a non-geometric, non-metallurgical, or similar origin were typically dispositioned as relevant. Such indications required additional measures such as further evaluation in accordance with ASME Section XI acceptance standards, engineering analysis, repair, or replacement.

The following is a summary indications detected and corrective measures taken during the Unit 2 Cycle 14 Refueling Outage.

NOI No.	Code Cat.	Component Identifier	Indication Description	Resolution	Additional samples
U2C14-017	R-A	RCRD-2-52	UT EXAMINATION DETECTED SUBSURFACE LINEAR INDICATION	WELD CUT-OUT REPLACED WITH WELD CRD-2-005-003, NEW VALVE AND PIPE	YES See below
U2C14-049	B-D	RPV CRD NOZZLE N9 NOZZLE TO VESSEL WLED	UT EXAMINATION DETECTED SUBSURFACE LINEAR INDICATION	USE AS IS	YES See below
U2C14-050	B-M-2	RWCU VALVE 2-ISV-69-500	DAMAGE TO VALVE SEAL RING AND GOUGES IN VALVE BODY INTENALS	VALVE CUT-OUT AND REPLACED BY PIPING.	NONE

There were unacceptable indications detected.

ADDITIONAL SAMPLES:

REFERENCE NOI# U2C14-017; EXPANDED SCOPE; DRHR-2-03 and RCRD-2-50

REFERENCE NOI# U2C14-049; EXPANDED SCOPE; N2B-NV, N2D-NV, N2K-NV, N3A-NV, N3C-NV, N4B-NV, N4C-NV, N4E-NV, N4F-NV, and N8B-NV

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 1

UNIT 2 CYCLE 14 **AUGMENTED EXAMINATION** **SUMMARY**

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

SECTION 1

AUGMENTED SUMMARY

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

This section includes augmented examinations performed to comply with NRC or TVA self-imposed requirements. Typical sources include generic letters, IE Bulletins, technical specifications, vendor recommendations, and industry experience. The following summarizes the augmented examinations performed during the Unit 2 Cycle 14 outage and references the corresponding paragraph in 2-SI-4.6.G and/or 0-TI-365.

Paragraph 7.11.5 Augmented Examination of Austenitic Stainless Steel and Dissimilar Metal Welds Susceptible to IGSCC (BWRVIP-75A)

Austenitic stainless steel and dissimilar metal circumferential welds in piping four inches or larger in nominal pipe diameter which contain reactor coolant at temperatures above 200 degrees F during power operation shall be examined. There was no new IGSCC identified in Cycle 14.

Reference: *BWR Vessel and Internals Project, Technical Basis for Revisions to Generic Letter 88-01 Inspection Schedules, BWRVIP-75A.*

NUREG-0313 CATEGORY	TOTAL NUMBER OF WELDS	WELDS EXAMINED DURING U2/C14 Outage
A	48	*0
B	N/A	N/A
C	115	4
D	9	7 (4 Expanded scope)
E	16	5
F	N/A	N/A
G	2	2 (VT-2)

*Category A Welds are sampled in accordance with the Unit 2 Risk - Informed ISI Program.

Examination Results: Indications of cracking observed in weld RCRD-2-52. Scope expanded to four (4) Category "D" welds. No indication of IGSCC noted in expanded scope.

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
--	--

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.11.6 Weld inspection For Pipe Whip Protection

Additional examinations shall be performed each inspection interval on selected circumferential pipe welds to provide additional protection against pipe whip in accordance with TSR 3.4.3.2.

There were no welds required to be scheduled this outage to be ultrasonically examined for evidence of pipe whip:

Examination results: N/A

Paragraph 7.11.7 Reactor Pressure Vessel Interior

Augmented examinations of the RPV interior components are performed in accordance with O-TI-365, Revision 023, Reactor Pressure Vessel Internals Inspection (RPVII) Units 1, 2 and 3. Reference ISI Report # R-181,

RPV Core Shroud Welds (Circumferential and Vertical) Ultrasonic (UT) Examination: Total: 9

Core Shroud Automated Phased Array UT Examinations

WELD ID#	Examination Results	Percentage of Weld Examined		Percentage of Examined Length Containing Indications
		Upper	Lower	
H1	5 Recordable Indications	N/A	77.08%	1.73%
H2	2 Recordable Indications	62.40%	82.72%	0.80%
H3	6 Recordable Indications	80.78%	82.07%	15.03%
H4	0 Recordable Indications	98.60%	98.60%	0%
H5	6 Recordable Indications	89.60%	98.60%	1.61%
H6	23 Recordable Indications	98.60%	91.70%	9.30%
H7	19 Recordable Indications	63.30%	53.80%	11.50%
V7	0 Recordable Indications	100%	100%	0%
V8	0 Recordable Indications	100%	100%	0%

Reference: O-TI-365, Paragraph 7.7, 8.0, and Appendix 9.2 and BWRVIP-76, BWR - Core Shroud Inspection and Flaw Evaluation Guidelines; EPRI TR - 114232.

Examination Results: Recordable indications observed.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Core Spray Piping, T-Box's and Sparger Inspection's Visual (VT-1 and EVT-1) Examinations

Total: 62

Welds/Component: CORE SPRAY T-BOX @ 120, P1-120, P2-120, AP3, BP3, PB-15, PB-110, PB-130, PB-165, CORE SPRAY T-BOX @ 240, P1-240, P2-240, CP3, DP3, PB-195, PB-230, PB-250, PB-345, CSSAS4-L, CSSAS2-L, CSSAS1, CSSAS2-R, CSSAS4-R, CSSAS3a, CSSAS3b, CSSBS4-L, CSSBS2-L, CSSBS1, CSSBS2-R, CSSBS4-R, CSSBS3a, CSSBS3b, CSSBS3c-L, CSSBS3c-R, CSSCS4-L, CSSCS2-L, CSSCS1, CSSCS2-R, CSSCS4-R, CSSCS3a, CSSCS3b, CSSDS4-L, CSSDS2-L, CSSDS1, CSSDS2-R, CSSDS4-R, CSSDS3a, CSSDS3b, CSSDS3c-L, CSSDS3c-R, CSSSB-7, CSSSB-45, CSSSB-88, CSSSB-93, CSSSB-135, CSSSB-172, CSSSB-187, CSSSB-225, CSSSB-267, *CSSSB-273, CSSSB-315, CSSSB-352.

Reference: 0-TI-365, Paragraph 7.6 and 7.11.2 and Appendix 9.2 and BWRVIP-18-A, BWR Core Spray Internals Inspection and Flaw Evaluation Guidelines, EPRI TR-1011469, February, 2005.

Examination Results: One indication observed.

* One Relevant indication was observed, a bent Upper Bracket at 273° (ref. INF-BFNP-2-07-007)

Core Plate Hold Down Bolts Visual (VT-3) Examinations: Total = 34

34 - Hold Down Bolts, CPHDB, 0 TO 90 Degrees, CPHDB, 90 to 180 Degrees, *CPHDB, 180 to 270 Degrees, and CPHDB, 270 to 360 Degrees.

Reference: 0-TI-365, Paragraph 7.14 and Appendix 9.2 and BWRVIP-25, BWR Core Plate Inspection and Flaw Evaluation Guidelines, EPRI TR-107284.

Examination Results: Recordable Indications noted.

* Three (3) Relevant Indications were observed. FME was located at 175°, 181° and 240° in annulus. (Ref. INF-BFNP2-2007-008)

Top Guide Visual (VT-1 and EVT-1) Examinations: Total = 8

Top Guide vertical Aligner Pin assemblies, Location at Azimuth 0 and 90 Degrees
Top Guide Rim Welds, Location 11 at Azimuth 0-90, 90-180, 180-270, and 270-360 Degrees
Socket/Bosses to Rim Welds, location 3 at Azimuth 0 and 90 Degrees
Debris Inspection of Top Guide Area, Periphery
Debris Inspection of Top Guide Area, General

Reference: 0-TI-365, Paragraph 7.9 and Appendix 9.2 and BWRVIP-26-A, BWR Top Guide Inspection and Flaw Evaluation Guidelines, EPRI TR-1009946, November 2004.

Examination Results: No indications of IGSCC cracking observed.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Guide Tube/Core Plate/Fuel Support Castings Visual (VT-3 and EVT-1) Examinations: Total = 41

Control Rod Guide Tube: 06-27, 06-35, 10-19, 10-43, 18-11, 18-51, 22-15, 26-27, 42-11, 42-51, 46-39, 50-19, 50-43, 54-35, 54-27, and 30-31 = 25 Visuals.

Fuel Support Castings and area of Core Plate: OFSC 06-27, OFSC 06-35, OFSC 10-19, OFSC 10-43, OFSC 18-11, OFSC 18-51, OFSC 22-15, OFSC 26-27, OFSC 42-11, OFSC 42-51, OFSC 46-39, OFSC 50-19, OFSC 50-43, OFSC 54-35, OFSC 54-27, and OFSC 30-31. = 16 Visuals

Reference: 0-TI-365, Paragraph 7.10 and Appendix 9.2 and BWRVIP-47-A, BWR Lower Plenum Inspection and Flaw Evaluation Guidelines, EPRI TR-1009947, November 2004.

Examination Results: No indications of IGSCC cracking observed.

Steam Dryer Visual (VT-1) Examination: Total = 13

Steam Dryer Welds: TB-1/2-01, TB-1/2-02, TB2/3-01, TB-2/3-02, TB-2/3-03; *TB-3/4-01, *TB-3/4-02, *TB-3/4-03, *TB-4/5-01, TB-4/5-02, TB-4/5-03, TB-5/6-01, TB-5/6-02,

Reference: 0-TI-365, Reactor Pressure Vessel Internals Inspection (RPVII) Units 1, 2 and 3, Paragraph 7.15 and Appendix 9.2 and GE-SIL-644, Revision 01 and BWRVIP-139, Steam Dryer Inspection and Flaw Evaluation Guidelines, EPRI TR-1011463, April 2005.

Examination Results: Recordable Indications noted.

* Two new Relevant Indications were observed. The Dryer Center Bank Divider and Tie Bar 4/5-1 were bent. (Ref. INF-BFNP-2-07-001)

Steam Separator Visual (VT-1) Examination: Total = 96

Steam Separator Shroud Bolts: *SHB 1-12, 0-90 Degrees, SHB 13-24, 90-180 Degrees, *SHB 25-36, 180-270 Degrees, and SHB 37-48, 270-360 Degrees.

Steam Separator Stand Pipes: 0-90 Degrees, 90-180 degrees, 180-270 Degrees, and 270-360 Degrees.

Reference: Extended Power Uprate PER # 74383

Examination Results: Recordable Indications noted.

* Two new Relevant Indications were observed. The lower portions of two Shroud Head Bolts showed greater than normal amounts of wear. (Ref. INF-BFNP-2-07-002, INF-BFNP-2-07-003)

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Jet Pump Assemblies and Riser Braces, Visual Inspections (VT-3, VT-1, and EVT-1) Examinations:

Total = 167

JP-1 JP-1 IN-4 JP-1 MX-2 *JP-1 WD-1 JP-1 AS-1&2 JP-1 RB-1a/c-0102 JP-1 RB-2a/c-0102 RS-1-0102 RS-2-0102 RS-3-0102 RS-6-0102 RS-8-0102 RS-9-0102 JP-1 DF-1 JP-1 DF-2 JP-1 AD-3a JP-1 AD-3b JP-1 AD-1 *JP-1 AD-2	JP-2 JP-2 IN-4 JP-2 MX-2 JP-2 WD-1 JP-2 AS-1&2 JP-2 RB-1b/d-0102 JP-2 RB-2b/d-0102 RS-7-0102 JP-2 DF-1 JP-2 DF-2 JP-2 AD-3a JP-2 AD-3b JP-2 AD-1 JP-2 AD-2	JP-3 JP-3 IN-4 JP-3 MX-2 *JP-3 WD-1 *JP-3 AS-1&2 JP-3 RB-1a/c-0304 JP-3 RB-2a/c-0304 RS-1-0304 RS-2-0304 RS-3-0304 RS-8-0304 RS-9-0304 RS-6-0304 JP-3 DF-1 JP-3 DF-2 JP-3 AD-3a JP-3 AD-3b JP-3 AD-1 *JP-3 AD-2	JP-4 JP-4 IN-4 JP-MX-2 *JP-4 WD-1 JP-4 AS-1&2 JP-4 RB1b/d-0304 JP-4 RB-2b/d-0304 RS-7-0304 JP-4 DF-1 JP-4 DF-2 JP-4 AD-3a JP-4 AD-3b JP-4 AD-1 *JP-4 AD-2	JP-5 JP-5 IN-4 JP-5 MX-2 *JP-5 WD-1 JP-5 AS-1&2 JP-5 RB1a/c-0506 JP-5 RB-2a/c-0506 RS-1-0506 RS-2-0506 RS-3-0506 RS-6-0506 RS-8-0506 RS-9-0506 JP-5 DF-1 JP-5 DF-2 JP-5 AD-3a JP-5 AD-3b JP-5 AD-1 *JP-5 AD-2	JP-6 JP-6 IN-4 JP-6 MX-2 JP-6 WD-1 JP-6 AS-1&2 JP-6 RB-1b/d-0506 JP-6 RB-2b/d-0506 RS-7-0506 JP-6 DF-1 JP-6 DF-2 JP-6 AD-3a JP-6 AD-3b JP-6 AD-1 JP-6 AD-2
JP-7 JP-7 IN-4 JP-7 MX-2 *JP-7 WD-1 *JP-7 AS-1&2 RS-1-0708 RS-2-0708 RS-3-0708 RS-6-0708 RS-8-0708 RS-9-0708 JP-7 DF-1 JP-7 DF-2 JP-7 AD-3a JP-7 AD-3b JP-7 AD-1 JP-7 AD-2	JP-8 JP-8 IN-4 JP-8 MX-2 JP-8 WD-1 JP-8 AS-1&2 RS-7-0708 JP-8 DF-1 JP-8 DF-2 JP-8 AD-3a JP-8 AD-3b JP-8 AD-1 *JP-8 AD-2	JP-9 JP-9 IN-4 JP-9 MX-2 JP-9 WD-1 *JP-9 AS-1&2 RS-1-0910 RS-2-0910 RS-3-0910 RS-6-0910 RS-8-0910 RS-9-0910 JP-9 DF-1 JP-9 DF-2 JP-9 AD-3a JP-9 AD-3b JP-9 AD-1 JP-9 AD-2	JP-10 JP-10 IN-4 JP-10 MX-2 JP-10 WD-1 JP-10 AS-1&2 RS-7-0910 JP-10 DF-1 JP-10 DF-2 JP-10 AD-3a JP-10 AD-3b JP-10 AD-1 *JP-10 AD-2 JP-16 **JP-16 WD-1 JP-16 AS-1&2	JP-11 JP-11 WD-1 JP-11 AS-1&2 JP-17 *JP-17 WD-1 JP-17 AS-1&2 JP-19 *JP-19 WD-1 JP-19 AS-1&2	JP-12 *JP-12 WD-1 JP-12 AS-1&2 JP-18 *JP-18 WD-1 *JP-18 AS-1&2 JP-20 *JP-20 WD-1 JP-20 AS-1&2
JP-13 *JP-13 WD-1 JP-13 AS-1&2	JP-14 *JP-14 WD-1 JP-14 AS-1&2	JP-15 *JP-15 WD-1 *JP-15 AS-1&2			

**** Recordable wear (service induced) was noted during U2C14 RFO inspection.**

Reference: 0-TI-365, Paragraph 7.8 and Appendix 9.2 and BWRVIP-41 revision 01, BWR Jet Pump Assembly Inspection and Flaw Evaluation Guidelines, EPRI TR-1012137, August, 2005.

Examination Results: Reference next page and ISI Report R-181 for results.

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1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.11.9 Core Plate delta/P Standby Liquid Control (SLC) Nozzle BWRVIP-27: 1 VT-2 (Visual) Examination

Perform an Enhanced VT-2 (EVT-2) Visual inspection of the Core Plate delta/P Standby Liquid Control (SLC), nozzle to safe end weld and safe end of the N10-SE in accordance with the recommendations of BWRVIP-27-A. The recommendations are to perform a visual leak check of the safe end to nozzle weld and safe end during the drywell leak check effort performed each outage. Insulation removal is required to perform the leak check. The implementation interval shall start with the Unit 2 Cycle 12 Refueling Outage.

Leakage inspections shall be performed as described during the Cycle 12 outage and during each subsequent refueling outage.

Reference: 0-TI-365, Paragraph 7.16 and Appendix 9.2 and BWRVIP-27-A, "BWR Standby Liquid Control System/Core Plate delta/P Inspection and Flaw Evaluation Guidelines" EPRI TR-1007279, August 2003.

Examination Results: A Enhanced VT-2 (EVT-2) examination was performed on the nozzle to safe-end weld and safe end during the system leakage test and revealed no leakage.

TVA/Browns Ferry Nuclear Plant, NRC Commitment in FSAR Section 10.10.5: 14 Welds

TVA/BFN to monitor the Unit 2 Emergency Equipment Cooling Water (EECW) System 067, stainless steel butt welds each cycle to ensure structural integrity of the system. Perform radiography in accordance with TVA Nondestructive Examination (NDE) Procedures, of designated welds to monitor for Microbiologically Induced Corrosion (MIC) progress. Reference procedure SPP-9.7, Appendix "C" and Work Order # 06-717852-000

Residual Heat Removal (RHR) Room Coolers B & D Welds:

T-EECW-2-BD-01B, T-EECW-2- BD-11B, T-EECW-2-BD-13B, T-EECW-2-BD-23B, T-EECW-2-BD-27B, and T-EECW-2-BD-33B.

Core Spray (CS) Room Coolers A & C Welds:

T-EECW-2-CAC-03B, T-EECW-2-CAC-04B, T-EECW-2-CAC-07B, T-EECW-2-CAC-08B, T-EECW-CAC-17B, T-EECW-2-CAC-19B, T-EECW-2-CAC-24B, and T-EECW-2-CAC-28B.

Examination Results: No new MIC growth recorded.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
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1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

SECTION 2

EXAMINATION SUMMARY

EXAMINATIONS PERFORMED DURING

UNIT 2 CYCLE 14 OUTAGE

Exam Requirements:

V01-02, B01-02,
B02-02, B07-02,
B12-02, 0T1365

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	FCV-01-037-BC	2-ISI-0222-C-02	V01-02	B-G-2	B7.70	VT-1		20061017	R-010	F	Voluntary examination. Examined one valve body bolt (position #17) stud per Engineering direction. Bolt rejected by Engineering and replaced with a new bolt. This closes this item. Reference report # R-009. W.O. 06-723923-001
RECIR	GR-2-15(OL)	2-ISI-0270-C-01	B02-02	E	NU0313	UT	BF-133	20070311	R-115	P	
RECIR	GR-2-38	2-ISI-0270-C-02	B02-02	C	NU0313	UT	ALTSS/ WB85	20070313	R-109	P	
RECIR	GR-2-41	2-ISI-0270-C-02	B02-02	C	NU0313	UT	ALTSS/S Q123	20070313	R-112	P	
RECIR	GR-2-48	2-ISI-0270-C-02	B02-02	C	NU0313	UT	ALTSS/S Q123	20070310	R-114	P	
RECIR	GR-2-53	2-ISI-0270-C-01	B02-02	E	NU0313	UT	ALTSS/B F133	20070310	R-106	P	SIZING OF PREVIOUS IGSCC INDICATION.
RHRS	DRHR-2-03	2-ISI-0221-C-01	B12-02	NU0313	D	UT	SQ-123	20070310	R-121	P	EXPANDED SCOPE
RHRS	DRHR-2-03B	2-ISI-0221-C-01	B02-02	G	NU0313	VT-2		20070008	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-09	2-ISI-0221-C-01	B02-02	E	NU0313	UT	ALTSS/B F133	20070310	R-108	P	SIZING OF EXISTING INDICATION.
RHRS	DRHR-2-11	2-MSG-0018-C-09	B12-02	D	NU0313	UT	BF-102	20070314	R-119	P	EXPANDED SCOPE
RHRS	DRHR-2-12	2-ISI-0221-C-01	B02-02	D	NU0313	UT	ALTSS/ WB85	20070307	R-086	P	
RHRS	DRHR-2-13B	2-ISI-0221-C-01	B02-02	G	NU0313	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-22	2-ISI-0221-C-01	B02-02	E	NU0313	UT	ALTSS/B F133	20070310	R-107	P	SIZING OF EXISTING INDICATION.
RHRS	DSRHR-2-04	2-ISI-0221-C-01	B02-02	C	NU0313	UT	ALTSS/S Q123	20070310	R-120	P	
RPV	N10-SE	2-ISI-0380-C-01	B07-02	BWRVIP-27	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.

Exam Requirements:

V01-02, B01-02,
B02-02, B07-02,
B12-02, OTI365

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	N11A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N11B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N12A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N12B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N-16A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N-16B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE
RPV	N4A-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	N4B-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	N4C-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	N4D-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	N4E-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	N4F-FW-SPARG	2-CHM-2046-C-02	B01-02	EPU74383	N/A	VT-1E		20070317	R-181	P	FOR EPU, END BRACKETS ONLY, REFERENCE PER# 74383.
RPV	RCRD-2-33	2-ISI-0272-C-01	B12-02	D	NU0313	UT	BF-131	20070315	R-116	P	EXPANDED SCOPE
RPV	RPV CORE PLATE	2-CHM-2046-C-02	OTI365	BWRVIP-25	N/A	VT-3		20070317	R-181	P	INCLUDES CORE PLATE BOLTING. 54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV CR GUIDE TUBES	2-CHM-2046-C-02	OTI365	BWRVIP-47	N/A	VT-1E		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV CR GUIDE TUBES	2-CHM-2046-C-02	OTI365	BWRVIP-47	N/A	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.

Exam Requirements:

V01-02, B01-02,
B02-02, B07-02,
B12-02, 0TI365

**OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000**

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV CS PIPING	2-CHM-2046-C-02	0TI365	BWRVIP-18	N/A	VT-1		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV CS PIPING	2-CHM-2046-C-02	0TI365	BWRVIP-18	N/A	VT-1E		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV CS PIPING	2-CHM-2046-C-02	0TI365	BWRVIP-18	N/A	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV JET PUMPS	2-CHM-2046-C-02	0TI365	BWRVIP-41	N/A	VT-1		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV JET PUMPS	2-CHM-2046-C-02	0TI365	BWRVIP-41	N/A	VT-1E		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV JET PUMPS	2-CHM-2046-C-02	0TI365	BWRVIP-41	N/A	VT-3		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV STEAM DRYER	2-CHM-2046-C-02	0TI365	BWRVIP139	N/A	VT-1		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV STEAM SEPARATOR	2-CHM-2046-C-02	0TI365	EPU74383	N/A	VT-1		20070317	R-181	P	FOR EPU REFERENCE PER# 74383. 54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV TOP GUIDE	2-CHM-2046-C-02	0TI365	BWRVIP-26	N/A	VT-1		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	RPV TOP GUIDE	2-CHM-2046-C-02	0TI365	BWRVIP-26	N/A	VT-1E		20070317	R-181	P	54-ISI-363-02 and SDCN# 30-5038911-02 & 03. REFERENCE REPORT R-181 FOR VENDOR M&TE AND EXAM RESULTS.
RPV	SHROUD WELD H-1	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070323	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD H-2	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070323	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD H-3	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		200703	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.

Exam Requirements:
 V01-02, B01-02,
 B02-02, B07-02,
 B12-02, 0TI365

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	SHROUD WELD H-4	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070304	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD H-5	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070304	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD H-6	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070304	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD H-7	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070303	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD V7	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070305	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RPV	SHROUD WELD V8	2-CHM-2046-C-02	0TI365	BWRVIP-76	N/A	UT		20070305	R-181	P	SEE REPORT R-181 FOR VENDOR M&TE.
RWCUS	CRD-2-005-003	2-ISI-0272-C-01	B02-02	D	NU0313	UT	SQ-116	20070401	R-171	P	W.O. 07-713160-000
RWCUS	DSRWC-2-03(OL)	2-ISI-0272-C-01	B02-02	E	NU0313	UT	BF-133	20070311	R113	P	
RWCUS	RCRD-2-49	2-ISI-0272-C-01	B02-02	D	NU0313	UT	BF-131/BF-132	20070228	R-126	P	
RWCUS	RCRD-2-50	2-ISI-0272-C-01	B12-02	NU0313	D	UT	BF-131/BF-132	20070311	R-118	P	EXPANDED SCOPE
RWCUS	RCRD-2-52	2-ISI-0272-C-01	B02-02	D	NU0313	UT	BF-131/BF-132	20070303	R-074	F	WELD CUT-OUT AND REPLACED WITH NEW VALVE AND PIPE, WELD CRD-2-005-003, W.O. 07-713160-000. Preservice UT exam performed on new weld on 04/10/07 Report R-171.
RWCUS	RWCU-2-003-069	2-ISI-0272-C-01	B02-02	D	NU0313	UT	SQ-116	20070330	R-153	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-070	2-ISI-0272-C-01	B02-02	D	NU0313	UT	SQ-116	20070330	R-154	P	DCN# 6888A W.O. 06-723036-001
RWCUS	RWCU-2-003-071	2-ISI-0272-C-01	B02-02	A	NU0313	UT	SQ-116	20070330	R-155	P	DCN#6888A W.O. 06-723035-001

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

BFN CONTAINMENT INSERVICE INSPECTION PROGRAM U2C14 REFUELING OUTAGE SUMMARY
REPORT

OWNER: TENNESSEE VALLEY AUTHORITY
OFFICE OF NUCLEAR POWER
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

BFN CONTAINMENT INSERVICE INSPECTION (CISI) PROGRAM U2C14 REFUELING OUTAGE
SUMMARY REPORT

Unit: BFN Unit 2
Refueling Outage: U2C14
Period/Interval: Third Period of the First Interval
Code of Record: ASME Section XI, 1992 Edition/1992 Addenda
Program Procedure: 0-TI-376, Revision 5.

Summary of Examinations

The records contained within the U2C14 Site Final Report comprise the Containment Inservice examinations performed to implement the requirements of ASME Section XI, Subsection IWE. The examinations are summarized as follows:

Table IWE-2500-1, Examination Category E-A, Containment Surfaces

Drywell liner, ECCS ring header, Suppression chamber interior above the water line,
Main Vent Headers and Downcomers

Table IWE-2500-1, Examination Category E-D, Seals, Gaskets and Moisture Barriers

During U2C14, 100% of the Drywell moisture barrier seal at elevation 550' was examined.

Table IWE-2500-1, Examination Category E-G, Pressure Retaining Bolting

Numerous flanges were examined (that were not previously examined during the interval) this outage.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

BFN CONTAINMENT INSERVICE INSPECTION PROGRAM

ASME SECTION XI SUBSECTION IWE

This information contained in this Appendix is provided in accordance with the requirements of 10CFR50.55a (b)(2)(ix)(A), evaluation of inaccessible areas, and 10CFR50.55a(b)(2)(ix)(D), evaluation for additional examinations, as they pertain to containment inservice examinations performed during the BFN Unit 2 Cycle 14 refueling outage.

These subject examinations were performed in accordance with ASME Section XI Subsection IWE, 1992 Edition/1992 Addenda. BFN Unit 2 is in the third period of the first examination interval.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

SUMMARY OF INDICATIONS

OWNER: TENNESSEE VALLEY AUTHORITY
OFFICE OF NUCLEAR POWER
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-001

Examination Report No. CISI-214-004 Component ID DW-HD-2-1

Condition / Indication: Paint flaking on interior of drywell head.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The flaking indication noted is the results of mechanical damage and or coating failure and not from a degradation in the Dry Well Head surface. There are no inaccessible area on the interior Drywell head.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-002

Examination Report No. CISI-214-011 Component ID PSC INT-2-B-11A

Condition / Indication: 1/4" flaking paint on inside bottom edge of manway due to mechanical damage; no pitting or physical damage to liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The 1/4 inch indication noted is the results of mechanical damage and not from a degradation in the liner surface.

The damage occurred from equipment being passed through the man-way during a previous outage. The damage is

generally small in size (1/4" to 3" in length) and is the results of moving and or securing equipment in place during

outage work. The extent of damage is to the coating. (2) Mechanical damage to the inaccessible areas located on

the inner radius of the TORUS will be on structural members (Ring Girders) and not on the shell. Mechanical

damage to the coating on the outer radius of the TORUS shell has occurred. The rate of corrosion is reduced from the

nitrogen atmosphere during operation and a light surface rust is all that develops. These areas are visible and are

monitored each outage. (3) No degradation exist that affects structural integrity or leak tightness of the

containment vessel. No corrective actions required at this time.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified

are coating related and not detrimental to the containment vessel. The areas are not suspect and additional

examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
UNIT: TWO	CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED	

NOI No. U2C14-003

Examination Report No. CISI-214-012 Component ID DW HD-2-1

Condition / Indication: 1/4" flaking paint on inside bottom edge of manway due to mechanical damage, no pitting or physical damage to liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The observations are expected conditions considering the environment, age, and service conditions of the drywell head. Mechanical damage on the drywell shell between each bolt hole, outer surface on flange. The bolt holes in the flange were observed to have light to medium rust / scaling around the circumference of the holes.

(2) The observations are judged to be minor in nature and have no adverse impact on the overall structural integrity of the drywell head. The drywell head is acceptable for continued service and the overall structural integrity of the drywell head is still maintained.

(3) No corrective actions required.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

(1) See Part II above.

(2) No similar components / the flaws evaluated as acceptable.

(3) No additional corrective actions.

(4) Only one drywell head per unit / no similar components.

OWNER: TENNESSEE VALLEY AUTHORITY
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1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-004

Examination Report No. CISI-214-015 Component ID ECCS RN 2-2

Condition / Indication: Mechanical damage to bare metal with light rust residue (no pitting).

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) A chain that is used for valve operation is scraping across the ECCS Ring Header causing mechanical damage to the coating and putting indentations in the pressure boundary surface. A 3" x 3" area is affected. Damage to the containment from a chain operated valve does not occur in inaccessible areas.

(2) The inaccessible areas on the TORUS exterior are free of damage from chain operated valves.

(3) WO #07-713009-000 initiated to repair the coating damage in the accessible areas.

No degradation of the containment exist. PER #120821 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-005

Examination Report No. CISI-214-016 Component ID ECCS RN 2-3

Condition / Indication: Areas of mechanical and coatings damage.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The mechanical damage to the coating is the results work activities (scaffolding, securing, lifting) in the area. Work activities of this type does not occur in inaccessible areas. The light rust in the crevices of a weld is the results of a thin layer of coating on the weld. This condition is a random occurrence that happens during coating application.

(2) TORUS shell welds do exist in the inaccessible areas. This degradation is slow and limited in development due to the existing coating and the amount of moisture. "Accessible" areas with this condition are monitored and if conditions warrant, access to the inaccessible areas will be obtained for inspection and repair.

(3) PER #120821 initiated to document the condition found. WO #07-713009-000 initiated to perform future repairs to the mechanical damage in the accessible areas.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-006

Examination Report No. CISI-214-018 Component ID ECCS RN 2-4

Condition / Indication: Blistering, mechanical damage, and coating damage.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The mechanical damage to the coating is the results work activities (scaffolding, securing, lifting) in the area.

Work activities of this type does not occur in inaccessible areas. The areas of minor blistering is the results of poor surface cleaning prior to coating application. This condition is a random in occurrence.

(2) Blistering could exist in the inaccessible areas. This is a coating degradation that will continue to provide some protection to the shell of the TORUS. "Accessible" areas with this condition are monitored and if conditions warrant, access to the inaccessible areas will be obtained for inspection and repair.

(3) PER #120821 initiated to document the condition found. WO #07-713009-000 initiated to perform future repairs the mechanical damage in the accessible areas.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area; and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-007

Examination Report No. CISI-214-019 Component ID ECCS RN 2-5

Condition / Indication: Chipped and mechanical damage to bare metal.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The mechanical damage and chips to the coating are the results work activities (scaffolding, securing, lifting) in the area. Work activities of this type does not occur in the inaccessible areas.

(2) The inaccessible areas on the TORUS exterior are free of mechanical damage from work activities.

The mechanical damage is to the coatings, not to the containment.

(3) PER #120821 initiated to document the condition found. WO #07-713009-000 initiated to perform future repairs to the mechanical damage in the accessible areas.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No: U2C14-008

Examination Report No: CISI-214-025 Component ID PCS INT 2-B-1A

Condition / Indication: Heavy rust with corrosion in penetration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indication identified is on the end of a 3/4" pipe that penetrates and extends into the TORUS for 8".

The rust is on the final 3" open end portion of the pipe that has never been coated. Rust does not exist at the penetration or on the shell surface. This is not a degradation to the Pressure Boundary.

(2) The pipe is inaccessible for a physical examination. Visually the rust appears to exist on the uncoated portion of the pipe (last 3"). The indication has been addressed in previous outage inspections.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

No corrective actions required. PER #120821 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-009

Examination Report No. CISI-214-026 Component ID PCS INT 2-B-2A

Condition / Indication: Scratches, light rust, chipped coatings, no damage to liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the liner surface. The damage is generally small in size (1/4" to 3" in length) and is the results of moving and or securing equipment in place during outage work. The extent of damage is to the coating.

(2) Mechanical damage to the inaccessible areas located on the inner radius of the TORUS will be on structural members (Ring Girders) and not on the shell. Mechanical damage to the coating on the outer radius of the TORUS shell has occurred. The rate of corrosion is reduced from the nitrogen atmosphere during operation and a light surface rust is all that develops. These areas are visible and are monitored each outage.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-010

Examination Report No. CISI-214-027 Component ID PCS INT 2-B-3A

Condition / Indication: Chips to coating with light rust.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the liner surface. The damage occurred from equipment being moved around the outer radius the TORUS. The damage is generally small in size (1/4" to 3" in length) and the extent of damage is to the coating. Work activities of this type does not occur in inaccessible areas. The light rust in the toe of the welds is the results of a thin layer of coating on the weld. This condition is a random occurrence that happens during coating application. This condition could exist in the inaccessible areas. (2) The rate of corrosion is reduced from the existing coating and the nitrogen atmosphere during operation. The identified accessible areas with this condition are monitored and if conditions warrant, access to the inaccessible areas will be obtained for inspection and repair. (3) No degradation exist that affects structural integrity or leak tightness of the containment vessel. No corrective actions required at this time.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-011

Examination Report No. CISI-214-028 Component ID PCS INT 2-B-4A

Condition / Indication: Chipped areas in coating, with light rust on liner; no pitting.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the liner

surface. The damage occurred from equipment being moved around the outer radius the TORUS. The damage is generally small in size (1/4" to 3" in length) and the extent of damage is to the coating.

Work activities of this type does not occur in inaccessible areas.

(2) The rate of corrosion is reduced from the nitrogen atmosphere that exist during operation. The identified areas with this condition are monitored and if conditions warrant, repairs are made.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel. No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-012

Examination Report No. CISI-214-030 Component ID PCS INT 2-B-14A

Condition / Indication: Numerous chips 1/8" to 3/8" with light rust and a 3/4" chipped area with light rust.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the liner surface. The damage occurred from equipment being moved around the outer radius the TORUS. The damage is generally small in size (1/4" to 3" in length) and the extent of damage is to the coating.

Work activities of this type does not occur in inaccessible areas.

(2) The rate of corrosion is reduced from the nitrogen atmosphere that exist during operation. The identified areas with this condition are monitored and if conditions warrant, repairs are made.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-013

Examination Report No. CISI-214-031 Component ID PCS INT 2-B-15A

Condition / Indication: Nicks and chips in coatings; no damage to surface with light rust.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the liner surface. The damage occurred from equipment being moved around the outer radius the TORUS. The damage is generally small in size (1/4" to 3" in length) and the extent of damage is to the coating.

Work activities of this type does not occur in inaccessible areas.

(2) The rate of corrosion is reduced from the nitrogen atmosphere that exist during operation. The identified areas with this condition are monitored and if conditions warrant, repairs are made.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No: U2C14-014

Examination Report No. CISI-214-032 Component ID PCS INT 2-B-16A

Condition / Indication: Light rust showing through coating and light rust bloom.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

The light rust in the toe of the welds and in the 2 sq ft area 1 foot from RG-16 is the results of a thin layer of coating on the weld and to the surface. This condition is a random occurrence that happens during coating application. This condition could exist in the inaccessible areas.

(2) The rate of corrosion is reduced from the existing coating and the nitrogen atmosphere during operation. The identified accessible areas with this condition are monitored and if conditions warrant, access to the inaccessible areas will be obtained for inspection and repair.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-015

Examination Report No. CISI-214-042 Component ID 2-X-205 Penetration

Condition / Indication: Penetration is to dirty to perform inspection.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

Penetration was cleaned and re-inspected acceptable.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

N/A

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-016

Examination Report No. CISI-214-041 Component ID PEN 2-X-232C

Condition / Indication: Discoloration with light rust on weld.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The light rust in the pipe welds is the results of a thin layer of coating on the weld. This condition is a random occurrence that happens during coating application. This condition could exist in the inaccessible areas.

(2) The rate of corrosion is reduced from the existing coating and the nitrogen atmosphere during operation. The identified accessible areas with this condition are monitored and if conditions warrant, access to the inaccessible areas will be obtained for inspection and repair.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel. No corrective actions required at this time. PER #120757 initiated to document the condition found.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-018

Examination Report No. CISI-214-050 Component ID VNT-HDR-2-1A

Condition / Indication: Flaking paint and peeling.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a

degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy

the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified

are coating related and not detrimental to the containment vessel. The areas are not suspect and additional

examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-019

Examination Report No. CISI-214-051 Component ID VNT-HDR-2-8A

Condition / Indication: Flaking paint and discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a

degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-020

Examination Report No. CISI-214-052 Component ID VNT-HDR-2-7A

Condition / Indication: Flaking paint and blisters.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-021

Examination Report No. CISI-214-053 Component ID VNT-HDR-2-2A

Condition / Indication: Loose flaking paint with light rust.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-022

Examination Report No. CISI-214-054 Component ID PSC-MVN-2-B-1

Condition / Indication: Scratches, light rust with no pitting on liner, surface.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-023

Examination Report No. CISI-214-055 Component ID PSC-MVN-2-B-2

Condition / Indication: Flaking paint and blisters.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-024

Examination Report No. CISI-214-056 Component ID PSC-MVN-2-B-3

Condition / Indication: Flaking paint.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No: U2C14-025

Examination Report No. CISI-214-057 Component ID PSC-MVN-2-B-4

Condition / Indication: Flaking paint and mechanical damage to coating.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-026

Examination Report No. CISI-214-058 Component ID PSC-MVH-2-B-13

Condition / Indication: Light to medium rust, chipped paint and flaking paint.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART-III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-027

Examination Report No. CISI-214-059 Component ID PSC-MVN-2-B-14

Condition / Indication: Medium rust in vent header.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a

degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy

the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO	CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED	

NOI No. U2C14-028

Examination Report No. CISI-214-060 Component ID PSC-MVN-2-B-15

Condition / Indication: Light rust, no pitting in header surface.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO	CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED	

NOI No. U2C14-029

Examination Report No. CISI-214-064 Component ID PSC-MVN-2-B-16

Condition / Indication: Chipped paint damage to coating, light rust, no pitting in liner surface.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-030

Examination Report No. CISI-214-068 Component ID DW LNR-2-3

Condition / Indication: Discoloration on DW liner, no pitting on DW liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) From Azimuth 220° to 240° at approx. El. 589' up to Core Spray Header black soot was found on the containment surface. The soot was from smoke that came up from a lower elevation. No other indications of degradation to the coating were found in this area or in the area of Azimuth 210° to 0° to 30°. Based on the current inspection, no adverse condition exist that may be present in inaccessible areas.

(2) The areas of exposed liner surface were inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER#121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-031

Examination Report No. CISI-214-069 Component ID PSC-MVH-2-B-1 thru PSC-MVH-2-B-16

Condition / Indication: Flaking paint, rust and rust blooms and discoloration on miter joint weld, for each downcomer.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior vapor region of the Downcomers.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel. PER #121542 was initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

OWNER: TENNESSEE VALLEY AUTHORITY
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1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-032

Examination Report No. CISI-214-070 Component ID VNT HDR-2-5A

Condition / Indication: Flaking paint, light to medium rust with rust blooms.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) - A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) - A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-033

Examination Report No. CISI-214-071 Component ID VNT HDR-2-3A

Condition / Indication: Discoloration, light rust, no pitting in liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-034

Examination Report No. CISI-214-072 Component ID VNT HDR-2-4A

Condition / Indication: Light rust, coloration, mechanical damage to the coating, no pitting in substrate.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-035

Examination Report No. CISI-214-073 Component ID DW LNR-2-5

Condition / Indication: Mechanical damage on substrate, DW liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A).
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The 2" x 1/2" area of damage coating at Az 132°, was evaluated and determined to be isolated case of mechanical damage. No other indications of degradation to the coating were found on the 616' EL. Based on the current inspection, no adverse condition exist that may be present in inaccessible areas.

(2) The areas of exposed liner surface were inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO	CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED	

NOI No. U2C14-036

Examination Report No. CISI-214-076 Component ID SHR-LG-2-3

Condition / Indication: Loose bolt, 1/8" gap between bolt head and flange.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

The inspection was re-performed. Technically, this flange is not in the IWE boundary. The connection identified was not loose and is not pressure retaining. No further actions required. All shear lugs inspected, therefore no inaccessible areas.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

N/A

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-037

Examination Report No. CISI-214-077 Component ID PSC-MVN-2-B-5

Condition / Indication: Nicks, scratches, and rust blooms.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-038

Examination Report No. CISI-214-078 Component ID DW LNR-2-3

Condition / Indication: Flaking paint, peeling with damage to coating.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The flaking and peeling coating on the 584' elevation of the drywell is the results of failure of the zinc primer coat.

This is a coating failure which could occur in inaccessible areas, however, the liner is still protected with a layer of zinc primer. No other indications of degradation to the coating were found. Based on the current inspection, no adverse condition exist that may be present in inaccessible areas.

(2) The areas of exposed liner surface were inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-039

Examination Report No. CISI-214-079 Component ID MSB-2-1, MSB-2-2, MSB-2-3

Condition / Indication: Separation to liner, physical damage and rust bloom.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) In spot locatings the moisture seal barrier has separated from the liner and has been physically been damaged which allows moisture, if present, to become entrapped between the seal and containment liner which can result in liner damage (rust and pitting corrosion). Degradation in the seal have occurred in both the accessible areas are the areas that are inaccessible fro repair. (2) The areas which are inaccessible for repairs are located under the air duct at azimuth 90 and 270 degrees. Inspections have identified that the seal barrier under the duct has some minor separation from the liner and some depressions in the center. Since during plant operation the atmosphere is inert with nitrogen and there is still some remaining ligament in the seal barrier, it is concluded that the seal barrier will perform its intended function. (3) the degradations in the seal which are accessible for repair shall be removed and repaired.

Following removal of the seal, a VT-3 examination of the liner shall be performed.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

See Part II above for description of flaw, extent, and conditions that led to degradation. The conditions have been evaluated as acceptable. There are no similar components with the IWE program (Category E-D).

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-040

Examination Report No. CISI-214-08-2 Component ID DW LNR-2-6

Condition / Indication: Flaking paint, chips to coatings.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The areas of damage coating at Az 30° & 22°, were evaluated and determined to be isolated case of mechanical damage. No other indications of degradation to the coating were found on the 633' EL. Based on the current inspection, no adverse condition exist that may be present in inaccessible areas.

(2) The areas of exposed liner surface were inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No.
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-041

Examination Report No. CISI-214-085 Component ID PSC-MVH-2-B-6

Condition / Indication: Light rust, rust bloom, discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-042

Examination Report No. CISI-214-083 Component ID VNT HDR-2-6A

Condition / Indication: Rust / coloration

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A).
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER.#121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

OWNER: TENNESSEE VALLEY AUTHORITY
OFFICE OF NUCLEAR POWER
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-043

Examination Report No. CISI-214-086 Component ID PSC-MVH-2-B-7

Condition / Indication: Nicks and scratches with light rust, discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-044

Examination Report No. CISI-214-092 Component ID PSC-MVH-2-B-9

Condition / Indication: Nicks, scratches with light rust.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a

degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy

the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified

are coating related and not detrimental to the containment vessel. The areas are not suspect and additional

examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOi No. U2C14-045

Examination Report No. CISI-214-094 Component ID PSC-MVH-2-B-11

Condition / Indication: Nicks, scratches with mechanical damage to coating. No damage to liner.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-046

Examination Report No. CISI-214-095 Component ID PSC-MVH-2-B-13

Condition / Indication: Light rust/ discoloration, mechanical damage, nicks, scratches to coating.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-047

Plant/Unit BFN/2

Examination Report No. CISI-214-093 Component ID PSC-MVH-2-B-10

Condition / Indication: Light rust bloom, with discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and not from a degradation in the containment surface. There are no inaccessible area in the interior Vent Line/Header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No

If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-048

Examination Report No. CISI-214-096 Component ID DW LNR-2-4

Condition / Indication: Light rust, mechanical damage to coating, scrapes with discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The areas of damage coating on the 604' EL. was evaluated and determined to be isolated case of mechanical damage. No other indications of degradation to the coating were found on the 604' EL. Based on the current inspection, no adverse condition exist that may be present in inaccessible areas.

(2) The areas of exposed liner surface were inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

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PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-051

Examination Report No. CISI-214-099 Component ID DW LNR-2-1

Condition / Indication: Rust and pitting in areas shown on sheet of N-VT-3 report CISI-214-099.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A)
(Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications were noted during the examination of the steel containment vessel surface in the area of the moisture seal barrier. No inaccessible areas exist with respect to acceptable moisture seal barrier inspections.

(2) The indications have been evaluated as acceptable. There is no degradation which affects the structural integrity or containment function of the steel containment vessel.

(3) No additional corrective actions required.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D) Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

Additional examinations not warranted at this time. The conditions are not considered to be a defective condition with respect to the steel containment vessel. The conditions noted occurred at the moisture seal barrier to-steel containment liner interface during moisture seal repair / replacement.

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DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

NOI No. U2C14-052

Examination Report No. CISI-214-087 Component ID PSC MVH 2-B-8

Condition / Indication: Light rust, mechanical damage to coating, scrapes with discoloration.

PART II - DISPOSITION (Supplemental Information)

Evaluation of inaccessible areas as required by 10CFR50.55a(b)(2)(ix)(A) (Include (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation; (2) An evaluation of each area, and the result of the evaluation; and (3) A description of necessary corrective actions) [additional separate continuation sheets may be attached, as necessary].

(1) The indications noted are the results of mechanical damage and or coating failure and not from a degradation in the containment surface. These are no inaccessible are in the interior vent line / header.

(2) Area was examined prior to and following coating removal. This examination was performed to satisfy the requirements of IWE 2500(b). The exposed surface was inspected and found to be in good condition.

(3) No degradation exist that affects structural integrity or leak tightness of the containment vessel.

PER #121542 initiated to document the condition found. Work Order #07-713904-000 was initiated for future application of coating to assist in preventing degradation of the exposed surface.

PART III - ADDITIONAL EXAMINATIONS (Supplemental Information)

Additional examinations required per 10CFR50.55a(b)(2)(ix)(D): Yes No
If Yes, provide (1) A description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation; (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components; (3) A description of the necessary corrective actions; and (4) The number and type of additional examinations to ensure detection of similar degradation in similar components [additional separate continuation sheets may be attached, as necessary].

The areas identified have been accepted by engineering evaluation (Part II above). The inspection findings identified are coating related and not detrimental to the containment vessel. The areas are not suspect and additional examinations are not required.

(1) See Part II, Item (1) above.

(2) The conditions identified have been accepted by engineering evaluation.

Coating examinations have been performed for similar components.

The evaluation results of similar flaws for similar components are expected to be the same as this component.

Therefore, no additional examinations are warranted.

(3) The conditions identified have been accepted by engineering evaluation. No additional corrective actions required.

(4) No additional examinations required to ensure detection of similar flaws in similar components.

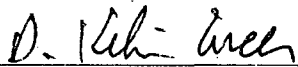
May 02, 2007

Sam Flood, ANI/ANII, PEC-1C, BFN

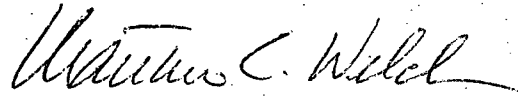
**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE CONTAINMENT INSERVICE INSPECTION (CISI) SCAN PLAN
REVISION 004**

Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage CISI Scan Plan, Revision 004, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, Subsection IWE, 1992 Edition and 1992 Addenda.

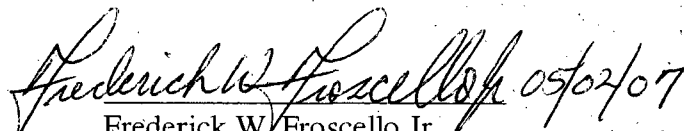
This document was prepared by D. Kelvin Green of BFN Components Engineering and forwarded to Matthew Welch and Fred Froscello of TVAN Inspection Services Organization (ISO) as notification of work scope.




D. Kelvin Green
BFN Components Engineering



Matthew C. Welch
BFN ISO, NDE Level III



Frederick W. Froscello Jr.
BFN ISO, NDE Specialist - ISI



Sam Flood ANI/ANII
Concurrence

cc: T. L. Shults, SAB-1B, BFN
M. L. Turnbow, STC-1I, SQN

Revision 004

05/02/2007

Total Examinations: 147

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
IWE EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC
SCV	2-X-205	BFN-CISI-002-3	14	N/A	N/A	V10-01	VT-3	N-VT-15
SCV	2-X-205	BFN-CISI-002-3	14	N/A	N/A	V10-01	VT-3	N-VT-15
SCV	2-X-232C	BFN-CISI-071	14	N/A	N/A	V10-01	VT-3	N-VT-15
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW HD-2-1	BFN-CISI-012	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	DW LNR-2-1	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-1	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-1	BFN-CISI-008-2	14	E1.12	E-A	92E-MS	VT-3	N-VT-15
SCV	DW LNR-2-1	BFN-CISI-008-2	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	DW LNR-2-2	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-3	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-3	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-3	BFN-CISI-008-2	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	DW LNR-2-4	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-5	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-6	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	DW LNR-2-6	BFN-CISI-008-2	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	ECCS RH 2-1	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-10	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-11	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-12	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-13	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-14	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-15	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-16	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-2	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-3	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-4	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-5	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-6	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-7	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-7	BFN-CISI-002	14	E1.12	E-A	P92-92	VT-3	N-VT-15

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC
SCV	ECCS RH 2-8	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	ECCS RH 2-9	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	FLG 2-X-35A	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35B	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35C	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35D	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35E	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35F	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-35G	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-4	BFN-CISI-012	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-47	BFN-CISI-032	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	FLG 2-X-6	BFN-CISI-044	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	GEN-VTE	2-47E872-1-ISI	14	E1.11	E-A	92E-92	VT-GEN	2-TI-173
SCV	MSB-2-1	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15
SCV	MSB-2-1	BFN-CISI-015	14	N/A	N/A	V01-01	VT-3	N-VT-15
SCV	MSB-2-2	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15
SCV	MSB-2-2	BFN-CISI-015	14	N/A	N/A	V01-01	VT-3	N-VT-15
SCV	MSB-2-3	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15
SCV	MSB-2-3	BFN-CISI-015	14	N/A	N/A	V01-01	VT-3	N-VT-15
SCV	NON-Code Piping		14	N/A	N/A	V10-01	VT-3	N-VT-15
SCV	PEN 2-X-213B	BFN-CISI-027	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	PEN 2-X-223	BFN-CISI-068-2	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	PEN 2-X-48	BFN-CISI-046	14	E1.12	E-A	P92-92	VT-3	N-VT-15
SCV	PEN 2-X-48	BFN-CISI-046	14	E8.10	E-G	P92-92	VT-1	N-VT-15
SCV	PEN 2-X-48	BFN-CISI-046	14	E8.10	E-G	P92-92	VT-1	N-VT-15
SCV	PSC EXT 2-B-1	BFN-CISI-015	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC EXT 2-B-1	BFN-CISI-015	14	E1.12	E-A	P92-01	VT-3	N-VT-15
SCV	PSC EXT 2-B-16	BFN-CISI-015	14	N/A	N/A	92E-CV	VT-3	N-VT-15
SCV	PSC EXT 2-B-16	BFN-CISI-015	14	E1.12	E-A	P92-01	VT-3	N-VT-15
SCV	PSC INT 2-B-10A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-11A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-12A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-13A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-14A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-15A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-16A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-1A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-2A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-3A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-4A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC
SCV	PSC INT 2-B-5A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-6A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-7A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-8A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC INT 2-B-9A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	N/A	N/A	92E-CV	VT-3	N-VT-15
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	N/A	N/A	92E-CV	VT-3	N-VT-15
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	N/A	N/A	92E-CV	VT-3	N-VT-15
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	N/A	N/A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	SHR LG-2-1	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-2	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-3	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-4	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-5	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-6	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-7	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	SHR LG-2-8	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15
SCV	VNT HDR-2-1A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-1A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-2A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-2A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-3A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-4A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-4A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-4A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-5A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-5A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-6A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-6A	BFN-CISI-018	14	N/A	N/A	92E-PC	VT-3	N-VT-15
SCV	VNT HDR-2-7A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-8A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15
SCV	VNT HDR-2-8A	BFN-CISI-018	14	N/A	N/A	92E-CV	VT-3	N-VT-15

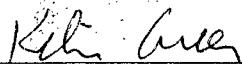
March 12, 2007

Sam Flood, ANI/ANII, PEC-1C, BFN

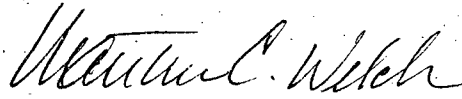
**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE CONTAINMENT INSERVICE INSPECTION (CISI) SCAN PLAN
REVISION 003**

Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage CISI Scan Plan, Revision 003, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, Subsection IWE, 1992 Edition and 1992 Addenda.

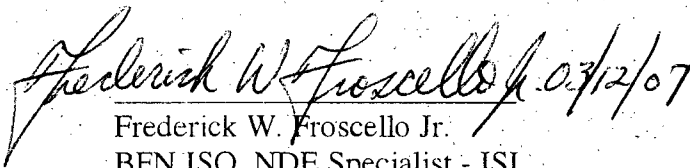
This document was prepared by D. Kelvin Green of BFN Components Engineering and forwarded to Matthew Welch and Fred Froscello of TVAN Inspection Services Organization (ISO) as notification of work scope.



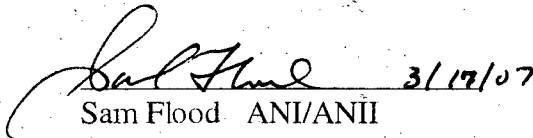
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Sam Flood ANI/ANII
Concurrence

cc: T. L. Shults, SAB-1B, BFN
M. L. Turnbow, STC-1I, SQN

Revision 001

03/12/2007

Total Examinations: 90

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
IWE EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	DW HD-2-1		BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-1		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-2		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-3		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-4		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-5		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-6		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-1		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-10		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-11		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-12		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-13		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-14		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-15		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-16		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-2		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-3		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-4		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-5		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-6		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-7		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-8		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-9		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	FLG 2-X-35A		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	FLG 2-X-35B		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35C		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35D		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35E		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35F		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35G		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-4		BFN-CISI-012	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-47		BFN-CISI-032	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-6		BFN-CISI-044	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	GEN-VTE		2-47E872-1-ISI	14	E1.11	E-A	92E-92	VT-GEN	2-TI-173	
SCV	MSB-2-1		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-2		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-3		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	NON-Code Piping			14	N/A	N/A	V10-01	VT-3	N-VT-15	See 0-TI-376 for scope.
SCV	PEN 2-X-213B		BFN-CISI-027	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-223		BFN-CISI-068-2	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-48		BFN-CISI-046	14	E1.12	E-A	P92-92	VT-3	N-VT-15	
SCV	PEN 2-X-48		BFN-CISI-046	14	E8.10	E-G	P92-92	VT-1	N-VT-15	
SCV	PSC INT 2-B-10A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-11A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-12A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-13A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-14A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-15A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-16A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-1A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-2A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	PSC INT 2-B-3A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-4A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-5A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-6A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-7A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-8A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-9A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-1		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-10		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-11		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-12		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-13		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-14		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-15		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-16		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-2		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-3		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-4		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-5		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-6		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-7		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-8		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-9		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	SHR LG-2-1		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-2		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-3		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-4		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	COMMENTS
SCV	SHR LG-2-5		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-6		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-7		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-8		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	VNT HDR-2-1A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-2A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-3A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-4A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-5A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-6A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-7A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-8A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	


February 25, 2007

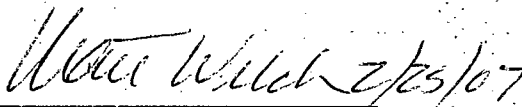
Sam Flood, ANI/ANII, PEC-1C, BFN


BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE CONTAINMENT INSERVICE INSPECTION (CISI) SCAN PLAN
REVISION 002

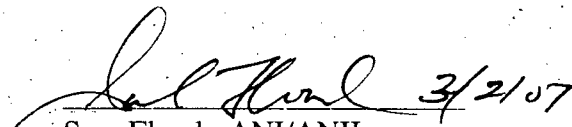
Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage CISI Scan Plan, Revision 002, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, Subsection IWE, 1992 Edition and 1992 Addenda.

This document was prepared by D. Kelvin Green of BFN Components Engineering and forwarded to Matthew Welch and Fred Froscello of TVAN Inspection Services Organization (ISO) as notification of work scope.


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Revision 2
 03/02/2007
 Total Examinations: 88

**TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT
 UNIT 2 IWE EXAMS SCHEDULED FOR CYCLE 14**

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-1	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-2	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-3	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-4	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-5	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-6	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-1	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-10	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-11	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-12	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-13	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-14	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-15	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-16	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-2	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-3	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-4	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-5	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-6	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-7	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-8	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-9	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	FLG 2-X-35A	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35B	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	COMMENTS
SCV	FLG 2-X-35C	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35D	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35E	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35F	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35G	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-4	BFN-CISI-012	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-47	BFN-CISI-032	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-6	BFN-CISI-044	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	GEN-VTE	2-47E872-1-ISI	14	E1.11	E-A	92E-92	VT-GEN	2-TI-173	
SCV	MSB-2-1	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-2	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-3	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	NON-Code Piping		14	N/A	N/A	V10-01	VT-3	N-VT-15	See 0-TI-376 for scope.
SCV	PEN 2-X-213B	BFN-CISI-027	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-223	BFN-CISI-068-2	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-48	BFN-CISI-046	14	E1.12	E-A	P92-92	VT-3	N-VT-15	
SCV	PEN 2-X-48	BFN-CISI-046	14	E8.10	E-G	P92-92	VT-1	N-VT-15	
SCV	PSC INT 2-B-10A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-11A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-12A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-13A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-14A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-15A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-16A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-1A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-2A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-3A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-4A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	WELDNO	ISONG	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	PSC INT 2-B-5A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-6A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-7A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-8A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-9A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	SHR LG-2-1	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-2	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-3	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-4	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-5	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-6	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	VNT HDR-2-1A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	VNT HDR-2-2A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-3A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-4A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-5A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-6A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-7A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-8A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	


February 16, 2007

Sam Flood, ANI/ANII, PEC-1C, BFN


**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE CONTAINMENT INSERVICE INSPECTION (CISI) SCAN PLAN
REVISION 001**

Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage CISI Scan Plan, Revision 001, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, Subsection IWE, 1992 Edition and 1992 Addenda.

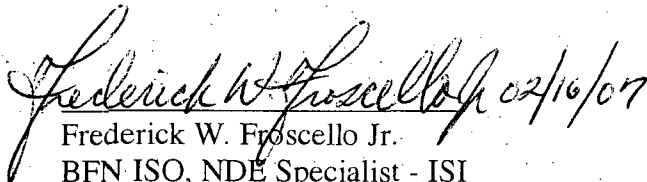
This document was prepared by D. Kelvin Green of BFN Components Engineering and forwarded to Richard Seals and Fred Froscello of TVAN Inspection Services Organization (ISO) as notification of work scope.



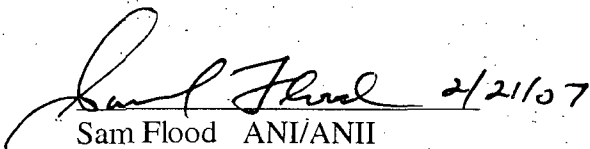
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M. L. Turnbow, STC-1I, SQN

Revision 1

02/20/2007

Total Examinations: 87

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT
UNIT 2 IWE EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	COMMENTS
SCV	DW HD-2-1	BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-1	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-2	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-3	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-4	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-5	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-6	BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-1	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-10	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-11	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-12	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-13	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-14	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-15	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-16	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-2	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-3	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-4	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-5	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-6	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-7	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-8	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-9	BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	FLG 2-X-35A	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35B	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	FLG 2-X-35C	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35D	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35E	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35F	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35G	BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-4	BFN-CISI-012	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-47	BFN-CISI-032	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-6	BFN-CISI-044	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	GEN-VTE	2-47E872-1-ISI	14	E1.11	E-A	92E-92	VT-GEN	2-TI-173	
SCV	MSB-2-1	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-2	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-3	BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	NON-Code Piping		14	N/A	N/A	V10-01	VT-3	N-VT-15	Reference SPP-9.1, Part E, Appendix A. Non code class piping that penetrates containment will be visually examined from the outside diameter (OD of the pipe) as non-code augmented examination as part of the VT-3 examination conducted at or near the end o
SCV	PEN 2-X-213B	BFN-CISI-027	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-223	BFN-CISI-068-2	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-48	BFN-CISI-046	14	E1.11	E-A	92E-92	VT-1	N-VT-15	
SCV	PSC INT 2-B-10A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-11A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-12A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-13A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-14A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-15A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-16A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-1A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-2A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-3A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	PSC INT 2-B-4A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-5A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-6A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-7A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-8A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-9A	BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	SHR LG-2-1	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-2	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-3	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-4	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-5	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-6	BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	VNT HDR-2-1A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-2A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-3A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-4A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-5A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-6A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-7A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-8A	BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	

November 28, 2006

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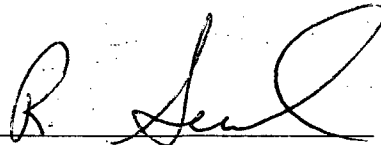
**BROWNS FERRY NUCLEAR PLANT (BFN) – UNIT 2 CYCLE 14 REFUELING
OUTAGE CONTAINMENT INSERVICE INSPECTION (CISI) SCAN PLAN
REVISION 000**

Attached for your review is the BFN Unit 2 Cycle 14 Refueling Outage CISI Scan Plan, Revision 000, for the examinations to be performed for the current Unit 2 outage by BFN Site Engineering/Components Engineering. These examinations are being performed to satisfy the requirements of ASME Section XI Code, Subsection IWE, 1992 Edition and 1992 Addenda.

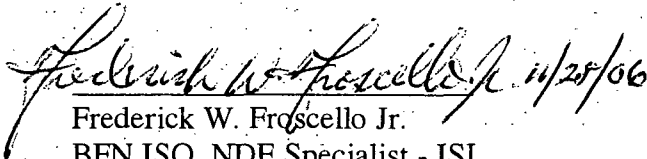
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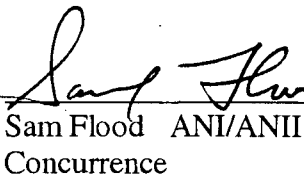
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M. L. Turnbow, STC-II, SQN

Revision 000

11/28/2006

Total Examinations: 102

**TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2**

IWE EXAMS SCHEDULED FOR CYCLE 14

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	DW HD-2-1		BFN-CISI-012	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-1		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-2		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-3		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-4		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-5		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	DW LNR-2-6		BFN-CISI-008-2	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-1		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-10		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-11		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-12		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-13		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-14		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-15		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-16		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-2		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-3		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-4		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-5		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-6		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-7		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-8		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	ECCS RH 2-9		BFN-CISI-002	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	FLG 2-X-35A		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	COMMENTS
SCV	FLG 2-X-35B		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35C		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35D		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35E		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35F		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-35G		BFN-CISI-043	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-4		BFN-CISI-012	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-47		BFN-CISI-032	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	FLG 2-X-6		BFN-CISI-044	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	GEN-VTE		2-47E872-1-ISI	14	E1.11	E-A	92E-92	VT-GEN	2-TI-173	
SCV	MSB-2-1		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-2		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	MSB-2-3		BFN-CISI-015	14	E5.30	E-D	92E-92	VT-3	N-VT-15	
SCV	NON-Code Piping			14	N/A	N/A	V10-01	VT-3	N-VT-15	Reference SPP-9.1,Part E, Appendix A. Non code class piping that penetrates containment will be visually examined from the outside diameter (OD of the pipe) as non-code augmented examination as part of the VT-3 examination conducted at or near the end of the interval. These examinations will include the accessible pipe surface between the isolation valves and the containment wall, the surface of the isolation valves, and any pressure retaining bolting within these boundaries. (Reference SPP 9.1, Appendix A, Part E.) Penetrations to be inspected include: X-22, -23, -24,-25, -26, -27(a-f), -35a, -35b, 35c, 35d, 35e, 35f, -50(a-f), -205, -207(a-h), -208A-L, -209a, -209d, -229b, -229d, -229g, -229j, -229k, -231.
SCV	PEN 2-X-213B		BFN-CISI-027	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-223		BFN-CISI-068-2	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	PEN 2-X-48		BFN-CISI-046	14	E1.11	E-A	92E-92	VT-1	N-VT-15	
SCV	PSC INT 2-B-10A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-10B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-11A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-11B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	PSC INT 2-B-12A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-12B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-13A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-13B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-14A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-14B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-15A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-15B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-16A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-16B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-1A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-1B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-2A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-2B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-3A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-3B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-4A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-4B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-5A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-5B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-6A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-6B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-7A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-7B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-8A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC INT 2-B-8B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC INT 2-B-9A		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	COMMENTS
SCV	PSC INT 2-B-9B		BFN-CISI-007	14	E1.12	E-A	92E-92	VT-3	VENDOR VT	
SCV	PSC MVH 2-B-1		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-10		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-11		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-12		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-13		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-14		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-15		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-2		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-3		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-4		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-5		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-6		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-7		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-8		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	PSC MVH 2-B-9		BFN-CISI-017&18	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	SHR LG-2-1		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-2		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-3		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-4		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-5		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	SHR LG-2-6		BFN-CISI-060	14	E8.10	E-G	92E-92	VT-1	N-VT-15	
SCV	VNT HDR-2-1A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-2A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-3A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-4A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-5A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	

SYSTEM	COMPONENT	SEGMENT	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	COMMENTS
SCV	VNT HDR-2-6A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-7A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	
SCV	VNT HDR-2-8A		BFN-CISI-018	14	E1.20	E-A	92E-92	VT-3	N-VT-15	

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

IWE Exams
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UNIT: TWO CYCLE 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	DW HD-2-1	BFN-CISI-012	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-004	P	NOI# U2C14-001 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-005. REFERENCE PER# 121542
SCV	DW HD-2-1	BFN-CISI-012	92E-92	E-A	E1.12	VT-3		20070406	CISI-214-107	P	UPPER FLANGE, 98.5% COVERAGE ON EXAM.
SCV	DW HD-2-1	BFN-CISI-012	92E-92	E-A	E1.12	VT-3		20070405	CISI-214-108	P	LOWER FLANGE
SCV	DW HD-2-1	BFN-CISI-012	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-012	P	NOI# U2C14-003 ACCEPTED BY ENGINEERING.
SCV	DW LNR-2-1	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070311	CISI-214-081	P	EXAM COVERS 180 TO 360 DEGREES
SCV	DW LNR-2-1	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070311	CISI-214-080	P	EXAM COVERS 0 TO 180 DEGREES AT 550' ELEV.
SCV	DW LNR-2-1	BFN-CISI-008-2	92E-MS	E-A	E1.12	VT-3		20070321	CISI-214-099	P	EXAMINER J. A. FERGERSON. INSPECTION AFTER REMOVAL OF MOISTURE SEAL BARRIER. NOI# U2C14-051 ACCEPTED BY ENGINEERING.
SCV	DW LNR-2-2	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070309	CISI-214-074	P	
SCV	DW LNR-2-3	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070307	CISI-214-068	P	NOI# U2C14-030 ACCEPTED BY ENGINEERING. EXAM 210 TO 0 AND 30 DEGREES.
SCV	DW LNR-2-3	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070308	CISI-214-078	P	NOI# U2C14-038 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-089.
SCV	DW LNR-2-4	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070308	CISI-214-096	P	NOI# U2C14-048 ACCEPTED BY ENGINEERING.
SCV	DW LNR-2-5	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070310	CISI-214-073	P	NOI# U2C14-035 ACCEPTED BY ENGINEERING.
SCV	DW LNR-2-6	BFN-CISI-008-2	92E-92	E-A	E1.12	VT-3		20070310	CISI-214-082	P	NOI# U2C14-040 ACCEPTED BY ENGINEERING.
SCV	ECCS RH 2-1	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-013	P	
SCV	ECCS RH 2-10	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-020	P	
SCV	ECCS RH 2-11	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070221	CISI-214-007	P	EXAMINER L. H. BREYER
SCV	ECCS RH 2-12	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070221	CISI-214-007	P	EXAMINER L. H. BREYER
SCV	ECCS RH 2-13	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070221	CISI-214-007	P	EXAMINER L. H. BREYER

OWNER: TENNESSEE VALLEY AUTHORITY

PLANT: BROWNS FERRY NUCLEAR PLANT

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NUCLEAR POWER GROUP

1101 MARKET STREET

CHATTANOOGA, TENNESSEE 37402

PO BOX 2000

DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	ECCS RH 2-14	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070221	CISI-214-007	P	EXAMINER L. H. BREYER
SCV	ECCS RH 2-15	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070221	CISI-214-007	P	
SCV	ECCS RH 2-16	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-014	P	EXAMINER L. H. BREYER
SCV	ECCS RH 2-2	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-015	P	EXAMINER L. H. BREYER. NOI# U2C14-004 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120821
SCV	ECCS RH 2-3	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-016	P	NOI# U2C14-005 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120821
SCV	ECCS RH 2-4	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-018	P	NOI# U2C14-006 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120821
SCV	ECCS RH 2-5	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070224	CISI-214-019	P	EXAMINER L. H. BREYER. NOI# U2C14-007 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120821
SCV	ECCS RH 2-6	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-020	P	
SCV	ECCS RH 2-7	BFN-CISI-002	P92-92	E-A	E1.12	VT-3		20070314	CISI-214-097	P	REWORK OF PEN 2-X-227A, HANGER R-25. W.O. 05-724223-001/DCN# 65786. REFERENCE PER# 120149.
SCV	ECCS RH 2-7	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-020	P	
SCV	ECCS RH 2-8	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-020	P	
SCV	ECCS RH 2-9	BFN-CISI-002	92E-92	E-A	E1.12	VT-3		20070222	CISI-214-020	P	
SCV	FLG 2-X-35A	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-35B	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-35C	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-35D	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-35E	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-35F	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	

OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

IWE Exams
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UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	FLG 2-X-35G	BFN-CISI-043	92E-92	E-G	E8.10	VT-1		20070303	CISI-214-047	P	
SCV	FLG 2-X-4	BFN-CISI-012	92E-92	E-G	E8.10	VT-1		20070223	CISI-214-036	P	
SCV	FLG 2-X-47	BFN-CISI-032	92E-92	E-G	E8.10	VT-1		20070316	CISI-214-110	P	
SCV	FLG 2-X-6	BFN-CISI-044	92E-92	E-G	E8.10	VT-1		20070221	CISI-214-003	P	
SCV	GEN-VTE	2-47E872-1-ISI	92E-92	E-A	E1.11	VT-GE		20070223	CISI-214-109	P	Exams from 02/23/07 to 03/11/07. See report CISI-214-109 FOR EXAMINERS AND M&E.
SCV	MSB-2-1	BFN-CISI-015	92E-92	E-D	E5.30	VT-3		20070311	CISI-214-079	P	EXAMINER J. A. FERGERSON. NOI# U2C14-039 ACCEPTED BY ENGINEERING. W.O. 06-716470-000
SCV	MSB-2-2	BFN-CISI-015	92E-92	E-D	E5.30	VT-3		20070311	CISI-214-079	P	EXAMINER J. A. FERGERSON. NOI# U2C14-039 ACCEPTED BY ENGINEERING. W.O. 06-716470-000
SCV	MSB-2-3	BFN-CISI-015	92E-92	E-D	E5.30	VT-3		20070311	CISI-214-079	P	EXAMINER J. A. FERGERSON. NOI# U2C14-039 ACCEPTED BY ENGINEERING. W.O. 06-716470-000
SCV	PEN 2-X-213B	BFN-CISI-027	92E-92	E-G	E8.10	VT-1		20070221	CISI-214-006	P	
SCV	PEN 2-X-223	BFN-CISI-068-2	92E-92	E-G	E8.10	VT-1		20070225	CISI-214-010	P	
SCV	PEN 2-X-48	BFN-CISI-046	P92-92	E-A	E1.12	VT-3		20070330	CISI-214-105	P	W.O. 04-712431-001. WELD# PNTSL-2-004-001 AND PNTSL-2-004-002.
SCV	PEN 2-X-48	BFN-CISI-046	P92-92	E-G	E8.10	VT-1		20070330	CISI-214-106	P	W.O. 04-712431-001. EXAM PERFORMED AFTER 0-TI-106 LLRT WAS COMPLETED.
SCV	PEN 2-X-48	BFN-CISI-046	P92-92	E-G	E8.10	VT-1		20070311	CISI-214-104	P	W.O. 04-712431-001. FLANGE AND BOLTING.
SCV	PSC EXT 2-B-1	BFN-CISI-015	P92-01	E-A	E1.12	VT-3		20070209	CISI-214-001	P	W.O. 05-713835-000
SCV	PSC EXT 2-B-16	BFN-CISI-015	P92-01	E-A	E1.12	VT-3		20070215	CISI-214-002	P	W.O. 05-713835-000
SCV	PSC INT 2-B-10A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC INT 2-B-11A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-011	P	NOI# U2C14-002 IS ACCEPTED BY ENGINEERING. REFERENCE PER3 120757

OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
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 DECATUR, ALABAMA 35609-2000

IWE Exams
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UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	PSC INT 2-B-12A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-035	P	
SCV	PSC INT 2-B-13A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-029	P	
SCV	PSC INT 2-B-14A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-030	P	NOI# U2C14-012 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-15A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-031	P	NOI# U2C14-013 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-16A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-032	P	NOI# U2C14-014 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-1A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-025	P	NOI# U2C14-008 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-2A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-026	P	NOI# U2C14-009 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-3A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-027	P	NOI# U2C14-010 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-4A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-028	P	NOI# U2C14-011 IS ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC INT 2-B-5A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC INT 2-B-6A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC INT 2-B-7A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC INT 2-B-8A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC INT 2-B-9A	BFN-CISI-007	92E-92	E-A	E1.12	VT-3		20070223	CISI-214-008	P	
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-054	P	NOI# U2C14-022 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-093	P	NOI# U2C14-047 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-094	P	NOI# U2C14-045 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE PER# 120757

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402.

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

IWE Exams
 92E-92
 92E-MS
 P92-92
 P92-01

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-095	P	NOI# U2C14-046 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-058	P	NOI# U2C14-026 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-066 AND CISI-214-069.
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-059	P	NOI# U2C14-027 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-060	P	NOI# U2C14-028 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-064	P	NOI# U2C14-029 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-055	P	NOI# U2C14-023 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-067 AND CISI-214-069.
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-056	P	NOI# U2C14-024 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-065.
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	92E-92	N/A	N/A	VT-3		20070307	CISI-214-048	P	PER# 121542. DOWNCOMERS 8, 14, 76, 89, and 90.
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-057	P	NOI# U2C14-025 AND U2C14-031 ACCEPTED BY ENGINEERING.
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-077	P	NOI# U2C14-037 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-085	P	NOI# U2C14-041 AND U2C14-031 ACCEPTED BY ENGINEERING.
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-086	P	NOI# U2C14-043 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069 AND CISI-214-091.
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-087	P	NOI# U2C14-052 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-092	P	NOI# U2C14-044 AND U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-069.

OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

IWE Exams
 92E-92
 92E-MS
 P92-92
 P92-01

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	SHR LG-2-1	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-2	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-3	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-076	P	NOI# U2C14-036 ACCEPTED BY ENGINEERING. CISI PROGRAM ENGINEER TO RE-EVALUATE CODE CATEGORY CLASSIFICATION OF BOLTED CONNECTION.
SCV	SHR LG-2-4	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-5	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-6	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-7	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	SHR LG-2-8	BFN-CISI-060	92E-92	E-G	E8.10	VT-1		20070310	CISI-214-075	P	
SCV	VNT HDR-2-1A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-050	P	NOI# U2C14-018 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-061
SCV	VNT HDR-2-2A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-053	P	NOI# U2C14-021 ACCEPTED BY ENGINEERING
SCV	VNT HDR-2-3A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-071	P	NOI# U2C14-033 ACCEPTED BY ENGINEERING
SCV	VNT HDR-2-4A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-072	P	NOI# U2C14-034 ACCEPTED BY ENGINEERING.
SCV	VNT HDR-2-5A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-070	P	NOI# U2C14-032 ACCEPTED BY ENGINEERING.
SCV	VNT HDR-2-6A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-083	P	NOI# U2C14-042 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-090.
SCV	VNT HDR-2-7A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-052	P	NOI# U2C14-020 ACCEPTED BY ENGINEERING.
SCV	VNT HDR-2-8A	BFN-CISI-018	92E-92	E-A	E1.20	VT-3		20070307	CISI-214-051	P	NOI# U21C4-019 IS ACCEPTED BY ENGINEERING. REFERENCE CISI-214-063

IWE Exams
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V01-01
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OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	2-X-205	BFN-CISI-002-3	V10-01	N/A	N/A	VT-3		20070225	CISI-214-042	P	NOI# U2C14-015 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-098.
SCV	2-X-205	BFN-CISI-002-3	V10-01	N/A	N/A	VT-3		20070316	CISI-214-098	P	VISUAL INSPECTION AFTER CLEANING. CLEARED NOI# U2C14-015. REPORT CISI-214-042.
SCV	2-X-232C	BFN-CISI-071	V10-01	N/A	N/A	VT-3		20070225	CISI-214-041	P	NOI# U2C14-016 ACCEPTED BY ENGINEERING. REFERENCE PER# 120757
SCV	DW HD-2-1	BFN-CISI-012	92E-PC	N/A	N/A	VT-3		20070222	CISI-214-005	P	REFERENCE CISI-214-004
SCV	DW LNR-2-1	BFN-CISI-008-2	92E-PC	N/A	N/A	VT-3		20070322	CISI-214-101	P	EXAMINER J. A. FERGERSON. REFERENCE CISI-214-099.
SCV	DW LNR-2-3	BFN-CISI-008-2	92E-PC	N/A	N/A	VT-3		20070308	CISI-214-089	P	REFERENCE CISI-214-078
SCV	DW LNR-2-6	BFN-CISI-008-2	92E-PC	N/A	N/A	VT-3		20070310	CISI-214-084	P	REFERENCE CISI-214-082.
SCV	MSB-2-1	BFN-CISI-015	V01-01	N/A	N/A	VT-3		20070325	CISI-214-103	P	W.O. 06-716470-000. VOLUNTARY EXAM.
SCV	MSB-2-2	BFN-CISI-015	V01-01	N/A	N/A	VT-3		20070325	CISI-214-103	P	W.O. 06-716470-000. VOLUNTARY EXAM.
SCV	MSB-2-3	BFN-CISI-015	V01-01	N/A	N/A	VT-3		20070325	CISI-214-103	P	W.O. 06-716470-000. VOLUNTARY EXAM.
SCV	NON-Code Piping		V10-01	N/A	N/A	VT-3		20070225	CISI-214-017*	P	See 0-TI-376 for scope. Various dates. *Reports: CISI-214-021, CISI-214-022, CISI-214-023, CICI-214-024, CISI-214-033, CISI-214-034, CISI-214-037, CISI-214-038, CISI-214-039, CISI-214-040, CISI-214-043, CISI-214-044, CISI-214-045, CISI-214-098, CISI-214-100, and CISI-214-102.
SCV	PSC EXT 2-B-1	BFN-CISI-015	92E-PC	N/A	N/A	VT-3		20070215	CISI-214-002	P	W.O. 05-713835-000
SCV	PSC EXT 2-B-16	BFN-CISI-015	92E-CV	N/A	N/A	VT-3		20070209	CISI-214-001	P	W.O. 05-713835-000
SCV	PSC MVH 2-B-1	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-054.
SCV	PSC MVH 2-B-10	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-093.
SCV	PSC MVH 2-B-11	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-094.
SCV	PSC MVH 2-B-12	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-095.

OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

IWE Exams
 92E-CV
 92E-PC
 V01-01
 V10-01

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

UNIT: TWO CYCLE: 14 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING.
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-049	P	W.O. 06-716564-000
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	92E-CV	N/A	N/A	VT-3		20070307	CISI-214-048	P	PER# 121542. DOWNCOMERS 8, 14, 76, 89, and 90.
SCV	PSC MVH 2-B-13	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-066	P	REFERENCE CISI-214-058
SCV	PSC MVH 2-B-14	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-059
SCV	PSC MVH 2-B-15	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-060.
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-064
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	92E-CV	N/A	N/A	VT-3		20070307	CISI-214-048	P	REFERENCE PER# 121542. REFERENCE CISI-214-049. DOWNCOMERS 8, 14, 76, 89, and 90.
SCV	PSC MVH 2-B-16	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-049	P	W.O. 06-716564-000
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-067	P	REFERENCE CISI-214-055
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-055.
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-049	P	W. O. 06-716564-000
SCV	PSC MVH 2-B-2	BFN-CISI-017&18	92E-CV	N/A	N/A	VT-3		20070307	CISI-214-048	P	REFERENCE PER# 121542. REFERENCE CISI-214-049. DOWNCOMERS 8, 14, 76, 89, and 90.
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-049	P	W.O. 06-716564-000. REFERENCE CISI-214-049.
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-056.
SCV	PSC MVH 2-B-3	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-065	P	REFERENCE CISI-214-056
SCV	PSC MVH 2-B-4	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-057
SCV	PSC MVH 2-B-5	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-077.
SCV	PSC MVH 2-B-6	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-085.

IWE Exams
92E-CV
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V01-01
V10-01

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
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PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

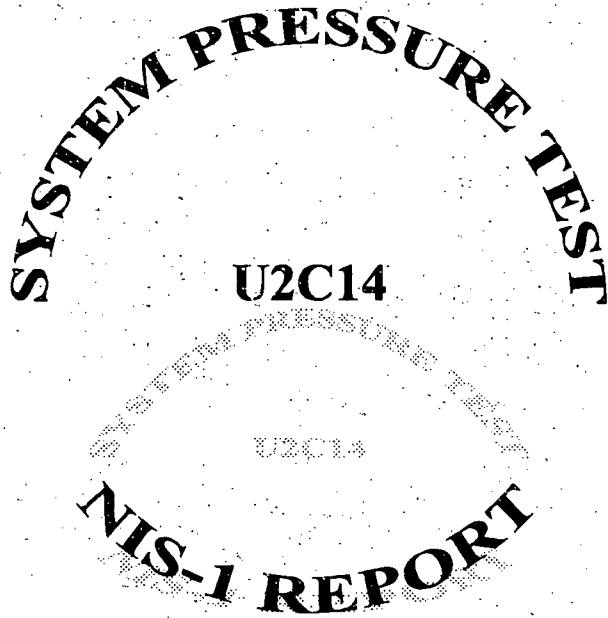
UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-091	P	NOI# U2C14-043 ACCEPTED BY ENGINEERING.
SCV	PSC MVH 2-B-7	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-086.
SCV	PSC MVH 2-B-8	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-087.
SCV	PSC MVH 2-B-9	BFN-CISI-017&18	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-069	P	NOI# U2C14-031 ACCEPTED BY ENGINEERING. REFERENCE CISI-214-092.
SCV	VNT HDR-2-1A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-061	P	REFERENCE CISI-214-050
SCV	VNT HDR-2-2A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-062	P	REFERENCE CISI-214-053
SCV	VNT HDR-2-4A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-072	P	
SCV	VNT HDR-2-4A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-088	P	REFERENCE CISI-214-072.
SCV	VNT HDR-2-5A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-070	P	
SCV	VNT HDR-2-6A	BFN-CISI-018	92E-PC	N/A	N/A	VT-3		20070307	CISI-214-090	P	REFERENCE CISI-214-083
SCV	VNT HDR-2-8A	BFN-CISI-018	92E-CV	N/A	N/A	VT-3		20070307	CISI-214-063	P	REFERENCE CISI-214-051

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT
SYSTEM PRESSURE TEST NIS-1 REPORT
UNIT 2 CYCLE 14



BROWNS FERRY
NUCLEAR PLANT

UNIT 2 CYCLE 14

ASME SECTION XI

FORM NIS-1, OWNER'S REPORT
ON
SYSTEM PRESSURE TESTS

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

SYSTEM PRESSURE TEST SUMMARY REPORT

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

FORM NIS-1 Sheet 2 of 5

APPENDIX I

Appendix I addresses item 13, Abstract of Examinations and Tests, on the Form NIS-1. Appendix I includes the next page which provide a list of the Class 1 and 2 System Pressure Tests performed on BFN Unit 2 during this operating cycle and refueling outage (U2C14).

All Class 1 and Class 2 System Pressure Tests have been performed for the Unit 2, 3rd Interval, 2nd Period.

Exam Requirements:
Pressure Test
Class 1 and 2

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CRDS	2-SI-3.3.1.D	N/A	96E-03	C-H	C7.10	VT-2		20070220	R-030	P	
CRDS	2-SI-3.3.1.D	N/A	96E-03	C-H	C7.30	VT-2		20070220	R-030	P	
CRDS	2-SI-3.3.1.D	N/A	96E-03	C-H	C7.70	VT-2		20070220	R-030	P	
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.30	VT-2		20060707	R-007	P	CORE SPRAY LOOP I
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.30	VT-2		20070105	R-011	P	CORE SPRAY LOOP II
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.50	VT-2		20060707	R-007	P	CORE SPRAY LOOP I
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.50	VT-2		20070105	R-011	P	CORE SPRAY LOOP II
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.70	VT-2		20060707	R-007	P	CORE SPRAY LOOP I
CSS	2-SI-3.3.6	N/A	96E-03	C-H	C7.70	VT-2		20070105	R-011	P	CORE SPRAY LOOP II
HPCIS	2-SI-3.3.9	N/A	96E-03	C-H	C7.30	VT-2		20060112	R-005	P	
HPCIS	2-SI-3.3.9	N/A	96E-03	C-H	C7.50	VT-2		20060112	R-005	P	
HPCIS	2-SI-3.3.9	N/A	96E-03	C-H	C7.70	VT-2		20060112	R-005	P	
MSS	2-SI-3.3.1.C	N/A	96E-03	C-H	C7.30	VT-2		20070220	R-029	P	
MSS	2-SI-3.3.1.C	N/A	96E-03	C-H	C7.30	VT-2		20070220	R-169	P	
MSS	2-SI-3.3.1.C	N/A	96E-03	C-H	C7.70	VT-2		20070407	R-169	P	
MSS	2-SI-3.3.1.C	N/A	96E-03	C-H	C7.70	VT-2		20070120	R-029	P	
RCICS	2-SI-3.3.10	N/A	96E-03	C-H	C7.30	VT-2		20051102	R-002	P	
RCICS	2-SI-3.3.10	N/A	96E-03	C-H	C7.50	VT-2		20051102	R-002	P	
RCICS	2-SI-3.3.10	N/A	96E-03	C-H	C7.70	VT-2		20051102	R-002	P	
RHR	2-SI-3.3.8.C	N/A	96E-03	C-H	C7.10	VT-2		20060106	R-004	P	
RHR	2-SI-3.3.8.C	N/A	96E-03	C-H	C7.30	VT-2		20060106	R-004	P	
RHR	2-SI-3.3.8.C	N/A	96E-03	C-H	C7.50	VT-2		20060106	R-004	P	

Exam Requirements:
Pressure Test
Class 1 and 2

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHR	2-SI-3.3.8.C	N/A	96E-03	C-H	C7.70	VT-2		20060106	R-004	P	
RHRS	2-SI-3.3.8.A	N/A	96E-03	C-H	C7.10	VT-2		20060901	R-008	P	
RHRS	2-SI-3.3.8.A	N/A	96E-03	C-H	C7.30	VT-2		20060901	R-008	P	
RHRS	2-SI-3.3.8.A	N/A	96E-03	C-H	C7.50	VT-2		20060901	R-008	P	
RHRS	2-SI-3.3.8.A	N/A	96E-03	C-H	C7.70	VT-2		20060901	R-008	P	
RHRS	2-SI-3.3.8.B	N/A	96E-03	C-H	C7.30	VT-2		20070220	R-041	P	
RHRS	2-SI-3.3.8.B	N/A	96E-03	C-H	C7.70	VT-2		20070220	R-041	P	
RHRS	DRHR-2-03B	2-ISI-0221-C-01	B02-02	G	NU0313	VT-2		20070008	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-03B	2-ISI-0221-C-01	96E-03	R-A	R1.16G	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-13B	2-ISI-0221-C-01	B02-02	G	NU0313	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	DRHR-2-13B	2-ISI-0221-C-01	96E-03	R-A	R1.16G	VT-2		20070408	R-163	P	REFERENCE REPORT R-163 FOR M&TE AND EXAM RESULTS.
RHRS	RHRG-2-05-A	2-ISI-0406-C-01	96E-03	C-B	C2.33	VT-2		20060901	R-008	P	SEE REPORT # R-008 FOR EXAM RESULTS AND M&TE.
RHRS	RHRG-2-06-A	2-ISI-0406-C-01	96E-03	C-B	C2.33	VT-2		20060901	R-008	P	SEE REPORT # R-008 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.10	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.50	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.60	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	B-P	B15.70	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.

Exam Requirements:
Pressure Test
Class 1 and 2

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
PO BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 14

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	2-SI-3.3.1.A	N/A	96E-03	C-H	C7.30	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.
RPV	2-SI-3.3.1.A	N/A	96E-03	C-H	C7.70	VT-2		20070408	R-163	P	SEE REPORT # R-163 FOR EXAM RESULTS AND M&TE.
RPV	N10-SE	2-ISI-0380-C-01	B07-02	BWRVIP-27	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N11A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N11B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N12A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N12B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N-16A-SE	2-ISI-0383-C-01	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
RPV	N-16B-SE	2-ISI-0383-C-02	B07-02	BWRVIP-49	N/A	VT-2		20070408	R-163	P	See report R-163 for M&TE.
SLCS	2-SI-3.3.4	N/A	96E-03	C-H	C7.10	VT-2		20070319	R-122	P	
SLCS	2-SI-3.3.4	N/A	96E-03	C-H	C7.30	VT-2		20070319	R-122	P	
SLCS	2-SI-3.3.4	N/A	96E-03	C-H	C7.50	VT-2		20070319	R-122	P	
SLCS	2-SI-3.3.4	N/A	96E-03	C-H	C7.70	VT-2		20070319	R-122	P	

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

FORM NIS-1 Sheet 4 of 5

APPENDIX II

Appendix II addresses item 14, Abstract of Results of Examinations and Tests, on the Form NIS-1.

Four (4) relevant leaks were identified during the system pressure tests covered by this report. The leaks are listed below.

Three (3) leaks were identified during the Class 1 primary system leak test, 2-SI-3.3.1.A

2-ISV-043-0599 leak at bolted connection

2-PMP-068-0060B leak at threaded connection on pump seal vent

2-CRDM-085-54-23 leak at bolted flangeconnection

One (1) leak was identified during the Class 2 Main Steam system leak test, 2-SI-3.3.1.C.

2-PCV-001-0146 significant leakage at pressure seal ring requiring corrective actions

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

FORM NIS-1 Sheet 5 of 5

APPENDIX III

Appendix III addresses item 15, Abstract of Corrective Measures, on the Form NIS-1.

Prior to April 8, 2005, for all leaks identified at bolted connections, an engineering evaluation of the bolted connection structural integrity, susceptibility of the bolting to corrosion and potential failure was conducted in accordance with Request for Relief 2-SPT-11, (Proposed Alternative to IWA-5250, Corrective Measures for Leakage at Bolted Connections, approved by the NRC in the letter dated April 8, 1999 (L44 990414 002)). As of April 8, 2005 BFN was given permission to use IWA-5250(a)(2) 1998 Edition, 1999 Addenda when addressing corrective actions for leakage at bolted connections and the engineering evaluation for leakage at bolted connections in non-borated systems is no longer required. (Note: All leakage listed below is from non-borated systems.)

2-ISV-043-0599 minor leak at body to bonnet bolted connection
Initiated corrective actions (corrective actions have not yet been completed).

2-PMP-068-0060B leak at threaded connection on pump seal vent
tightened threaded connection

2-CRDM-085-54-23 minor leak at bolted flange connection.
Evaluated leakage - this leaking condition has not degraded and will not degrade the fasteners, the structural integrity of the connection or any surrounding components. Since the leakage was below the rate described as typical by the vendor (GE), no further actions are planned.

2-PCV-001-0146 significant leakage at pressure seal ring requiring corrective action.
Disassembled valve and replaced pressure seal ring.

ENCLOSURE 2

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNITS 2
AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME),
SECTION XI, THIRD TEN-YEAR INSPECTION INTERVAL
REPAIRS AND REPLACEMENTS PROGRAM
SUMMARY REPORT (NIS-2) FOR CYCLE 14 OPERATION

(SEE ATTACHED)

BROWNS FERRY
NUCLEAR PLANT

UNIT 2 CYCLE 14

ASME SECTION XI

FORM NIS-2, OWNER'S REPORT

OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

APPENDIX I _____ **Summary of Repair and
Replacement Activities**

APPENDIX II _____ **Form NIS-2 Owner's Report
For Repairs or Replacements**

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

APPENDIX I

SUMMARY OF REPAIR AND REPLACEMENT ACTIVITIES

Owner: TENNESSEE VALLEY AUTHORITY
 1101 Market Street
 Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
 P. O. Box 2000
 Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
05-722579-000	075	MAINT	2	MODIFICATION
06-715787-000	075	MAINT	2	REPLACEMENT
06-717380-000 06-717380-001	069	MAINT	1	MODIFICATION
03-004267-000	001	MAINT	1	REPLACEMENT
04-723525-003	001	TVA	1	MODIFICATION
06-723923-001	001	MAINT	1	REPLACEMENT
06-718862-000	074	MAINT	2	REPLACEMENT
06-717668-000 06-717669-000 06-717671-000	073	MAINT	2	REPLACEMENT
06-717647-000 06-717705-000	074	MAINT	2	REPLACEMENT
05-715970-000 05-715971-000 05-715972-000 05-719562-000	074	MAINT	2	REPLACEMENT
05-715974-000 05-715975-000 05-719563-000 05-719564-000	075	MAINT	2	REPLACEMENT
06-711366-018	001	MAINT	1	REPLACEMENT
04-722413-031 thru -042, 04-722413-044 thru -052, 06-717703-000 06-717704-000	001	MAINT	2	REPLACEMENT

Owner: TENNESSEE VALLEY AUTHORITY
 1101 Market Street
 Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
 P. O. Box 2000
 Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
06-717672-000 07-711244-000 07-711245-000	003	MAINT	1	REPLACEMENT
02-011925-001	001	MAINT	?	MODIFICATION
06-719319-000	085	TVA	1	REPLACEMENT
06-716045-000 06-716045-001	001 & 006	TVA	2	REPLACEMENT
06-714856-000	001	MAINT	2	REPLACEMENT
06-712169-003	001	MAINT	2	REPLACEMENT
05-717805-000 05-717805-001	001	TVA	2	MODIFICATION
05-724223-001	075 & 100	TVA	2 & MC	MODIFICATION
06-718765-006 06-718735-006	068	TVA	1	REPLACEMENT
03-022883-001	010	TVA	2	REPLACEMENT
00-003350-000 00-003350-002 00-003350-003 00-003350-005	073	TVA	1	MODIFICATION
03-004270-000	001	MAINT	1	REPLACEMENT
06-711366-000 thru-007 06-711366-021 06-711366-026	001	TVA	1	MODIFICATION
06-723036-001	069	MAINT	1	MODIFICATION
03-015954-000	071	MAINT	2	REPAIR
04-718365-000	068	MAINT	1	MODIFICATION
06-710345-002	001	MAINT	2	MODIFICATION
06-711111-000	075	MAINT	2	MODIFICATION
06-725504-000	010	MAINT	1	REPLACEMENT
07-711826-001	069	MAINT	1	REPAIR

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

LEGEND

WID - Work Implementing Document

Example: A99999A or 50000A refers to a Design Change Notice
99-999999-999 refers to a Work Order

SYS- System

001 - Main Steam	069 - Reactor Water Cleanup
003 - Reactor Feedwater	071 - Reactor Core Isolation Cooling
006 - Heater Drains & Vents	073 - High Pressure Coolant Injection
008 - Turbine Drains	074 - Residual Heat Removal
010 - Reactor Drains, Vents and Blowdown	075 - Core Spray
063 - Standby Liquid Control	085 - Control Rod Drive
068 - Reactor Water Recirculation	092 - Neutron Monitoring

ORG - Organization which performed the WID

MAINT - TVA's Maintenance Organization
GE - General Electric Company
TVA - Work performed by Stone and Webster Engineering Corporation
or Framatome utilizing TVA's Quality Assurance Program and procedures

CLASS - Refers to ASME Code Class 1 or 2

ACTIVITY - Classifies work activity as being repair, replacement or modification

Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

APPENDIX II

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801

Date January 30, 2006

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order (WO) 05-722579-000
Repair/Replacement Organization P. O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 075, Core Spray System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Core Spray Pmp 2C Min Flow Shutoff V/V	Powell	N/A	N/A	2-SHV-075-0017	N/A	Corrected	No

7. Description of Work 1/4" packing leak-off line reattached.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7075 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: 05-722579-000

9. Remarks 1/4" packing leak-off line reattached.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI:

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] System Engineer

Owner or Owner's Designee, Title

Date 1-30 20 06

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 10/21/05 to 8/24/06 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Inspector's Signature

Commissions TN 4011

National Board, State, Province, and Endorsements

Date 8/24 20 06

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date October 12, 2006
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Order (WO) 06-715787-000
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000; Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 075, Core Spray System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Core Spray PSC Water Fill Isol Vlv	Edwards Valves Inc.	N/A	N/A	2-SHV-075-0649	N/A	‡	No
‡ - replaced disc assembly							
disc assembly	Edwards Valves Inc.	N/A	N/A	2-SHV-075-0649		Removed	No
disc assembly	Edwards Valves Inc.	N/A	N/A	2-SHV-075-0649		Installed	Yes

7. Description of Work Replaced valve disc assembly.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 77K53-820721-3 and Design Criteria BFN-50-7075 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: 06-715787-000

9. Remarks Replaced valve disc assembly.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed

[Signature]

System Engineer

Date

6-8 20 07

Owner or Owner's Designee Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 5/23/06 to 6/12/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]

Inspector's Signature

Commissions

TN4011

National Board, State, Province, and Endorsements

Date

6/12 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date October 16, 2006

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) 67211 and 67125,
 Work Orders (WOs) 06-717380-000 and 06-717380-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 069, Reactor Water Cleanup (RWCU) System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RWCU Chemical Decon. Shutoff Valve	VOGT	2-216919	N/A	2-SHV-069-0552	N/A	Removed	No
pipe	unknown	N/A	N/A	N/A	N/A	Removed	No
pipe	Energy & Process Corp.	N/A	N/A	N/A	N/A	Installed	No
pipe cap	Consolidated Power Supply	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Removed 2-SHV-069-0522, replaced some piping and installed pipe cap.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
 Other Pressure N/A psi Test Temp. N/A °F ** - Ref. Code Case N-416-3

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7069 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WOs) 06-717380-000 and 06-717380-001

9. Remarks Removed 2-SHV-069-0522, replaced some piping and installed pipe cap.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 1/26 20 07
Owner or Owner's Designee Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 5/23/06 to 2/2/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/2 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date October 16, 2006
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000 Work Order (WO) 03-004267-000
Address Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code ASME Section III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
ASME Section III, Article 9, 1965 and
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1073	N/A	2-PCV-001-0180	1968	Removed	Yes
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	205	N/A	2-PCV-001-0180	2004	Installed	Yes
							Yes
							Yes

7. Description of Work Replaced Main Steam Relief valve main body.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in GE P. O. 205AJ600, and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Work Order (WO) 03-004267-000

Applicable Manufacturer's Data Reports to be attached

The main valve body was replaced with a new valve body (same manufacturer/model number).

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] System Engineer

Owner or Owner's Designee, Title

Date 10-16 20 06

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 5-21-06 to 10-19-06, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions

TN 4511

National Board, State, Province, and Endorsements

Date 10/19 20 06

**FORM N-2 CERTIFICATE HOLDERS' DATA REPORT FOR IDENTICAL
NUCLEAR PARTS AND APPURTENANCES***

As Required by the Provisions of the ASME Code, Section III
Not to Exceed One Day's Production

1. Manufactured and certified by Target Rock, 1966E Broadhollow Rd., E. Farmingdale, NY 11735
(name and address of NPT Certificate Holder)
2. Manufactured for Tennessee Valley Authority, 1101 Market St., Chattanooga, TN
(name and address of Purchaser)
3. Location of installation Browns Ferry Nuclear Plant Unit 1, Intersection Shaw & Nuclear Plant, Athens, AL
(name and address)
4. Type PL7567F-100 See page 3 of 3 See page 3 of 3 N/A 2004
(drawing no.) (mat'l. spec. no.) (tensile strength) (CRN) (year built)
5. ASME Code, Section III, Division 1: 1968 Summer 1970 1 None
(edition) (addenda date) (class) (Code Case no.)
6. Fabricated in accordance with Const. Spec. (Div. 2 only) N/A Revision N/A Date N/A
(no.)
7. Remarks: Body/Base Assembly (MSSRV) Part No. 7567F-100-23
Spare parts for valve model no. 7567F (PL7567F-100) (TR Project No. 03S 039)
8. Nom. Thickness (in.) N/A Min. design thickness (in.) N/A Dia. ID (ft & in.) N/A Length overall (ft & in.) N/A
9. When applicable, Certificate Holders' Data Reports are attached for each item of this report:

Part or Appurtenance Serial Number	National Board No. In Numerical Order	Part or Appurtenance Serial Number	National Board No. In Numerical Order
(1) 205	N/A	(26)	
(2) ---		(27)	
(3)		(28)	
(4)		(29)	
(5)		(30)	
(6)		(31)	
(7)		(32)	
(8)		(33)	
(9)		(34)	
(10)		(35)	
(11)		(36)	
(12)		(37)	
(13)		(38)	
(14)		(39)	
(15)		(40)	
(16)		(41)	
(17)		(42)	
(18)		(43)	
(19)		(44)	
(20)		(45)	
(21)		(46)	
(22)		(47)	
(23)		(48)	
(24)		(49)	
(25)		(50)	

10. Design pressure N/A psi. Temp. N/A °F Hydro. Test pressure 2375 psig at ambient temp. °F
(when applicable)

* Supplemental information in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 x 11, (2) information in items 1 through 4 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

Certificate Holder's Serial Nos. 7567-100-23 s/n 205

CERTIFICATION OF DESIGN

Design specifications certified by R. P. Ghosh P.E. State CA Reg. No. 16371
(when applicable)

Design report* certified by D. M. Pattarini P.E. State NY Reg. No. 029841
(when applicable)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this (these) Parts
conforms to the rules for construction of the ASME Code, Section III, Division 1.

NPT Certificate of Authorization No. N-1948 Expires 12/12/2004

Date 11/14/2004 Name Target Rock Signed [Signature]
(NPT Certificate Holder) R. E. Glazier, QA Manager
(authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of New York and employed by OneBeacon America Insurance Co. of Boston, MA have inspected the pump, or valve, described in this Data Report on 11/14/2004 and state that to the best of my knowledge and belief, the Certificate Holder has fabricated these parts or appurtenances in accordance with the ASME Code, Section III, Division 1. Each part listed has been authorized for stamping on the date shown above.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 11/14/04 Signed [Signature] Commissions NY 5102
(Authorized Inspector) (Nat'l. Bd. (incl. endorsements) and state or prov. and no.)

FORM N-2 (Pg. 3 of 3)

Certificate Holder's Serial Nos. 7567F-100-23- s/n 205

1. Manufactured and certified by Target Rock, 1966E Broadhollow Rd., E. Farmingdale, NY 11735
(name and address of NPT Certificate Holder)
2. Manufactured for Tennessee Valley Authority, 1101 Market St., Chattanooga, TN
(name and address of Purchaser)
3. Location of installation Browns Ferry Nuclear Plant Unit 1, Intersection Shaw & Nuclear Plant, Athens, AL
(name and address)
4. Type PL7567F-100 See page 3 of 3 See page 3 of 3 N/A 2004
(drawing no.) (mat'l. spec. no.) (tensile strength) (CRN) (year built)

DESCRIPTION	PART No.	SERIAL No. OR HEAT No.	MATERIAL SPECIFICATION	TENSILE STRENGTH (ksi)
Body Assembly	600054-11	s/n 205	SA 105	70
Seat Insert	200847-1	s/n 255	SA 105	70
Base	300206-1	s/n 317	SA 105	70
Main Disc	200848-1	s/n 4728	SA 182 F304L	65
Stud	102049-5	8089528	SA 193 B16	125
Stud	102049-4	8067576	SA 193 B7	125
Stud	102049-3	D12678	SA 193 B16	125
12-point Bolt	204018-1	94615	SA 193 B7	125
Spline Nut	204041-1	8962767	SA 194 gr. 7	n/a

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date October 20, 2006
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Design Change Notices (DCNs) 63005 and 68112,
Work Order (WO) 04-723525-003
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Strain Gauge	HITEC Products Inc HBWAK-35-250-6-10FG	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Attached 64 strain gauges to the exterior of the Main Steam piping in the Drywell.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
** - welding did not penetrate pressure boundary.
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in DCNs 63005 and 68112 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

Work Order (WO) 04-723525-003

9. Remarks Installed 64 strain gauges on the exterior of the Main Steam piping.

Applicable Manufacturer's Data Reports to be attached

Welding did not penetrate the pressure boundary therefore this activity was exempt from a hydrostatic pressure test.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed

Stephen C. Williams
Owner or Owner's Designee, Title

System Engineer

Date

10/20 20 06

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 10/12/06 to 11/7/06 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Samuel Flood
Inspector's Signature

Commissions

TN 4011

National Board, State, Province, and Endorsements

Date

11/7 20 06

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date October 22, 2006
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801

2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Design Change Notice (DCN) T39905 and 68280,
 Work Order (WO) 06-723923-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Line C Inbd Isol Vlv (MSIV)	Atwood & Morrill 20851-H-26	N/A	N/A	2-FCV-001-0037	N/A	‡	No
‡ - Replaced poppet valve disc, stem and some of the bolting							
poppet valve disc	Atwood & Morrill	Unknown	N/A	2-FCV-001-0037	N/A	Removed	No
poppet valve disc	Atwood & Morrill	3	N/A	2-FCV-001-0037	N/A	Installed	No
stem	Atwood & Morrill	Unknown	N/A	2-FCV-001-0037	N/A	Removed	No
stem	Atwood & Morrill	1	N/A	2-FCV-001-0037	N/A	Installed	No
bolting	Atwood & Morrill	N/A	N/A	2-FCV-001-0037	N/A	Removed	No
bolting	NOVA	N/A	N/A	2-FCV-001-0037	N/A	Installed	No

7. Description of Work Replaced valve disc and stem with modified components.
Replaced some of the body to cover plate bolting.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91750, GE Purchase Spec 21A1062 Rev. 0 and 21A1062AL Rev. 6 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: 06-723923-001

9. Remarks Replaced valve disc, and stem with modified components. Replaced some of the body to cover plate bolting due to damage during disassembly. Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 1/31/07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 10/16/06 to 2/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011

Date 2/12/07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date January 24, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Orders (WO) 06-718862-000
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 074, Residual Heat Removal (RHR) System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code snubber - MSS-SP 58 - 1967
pipng - USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code-Stamped (Yes or No)
Support (Snubber) 2-47B452S0252	Pacific Scientific	7270	N/A	2-SNUB-074-5035	N/A	Removed	No
Support (Snubber) 2-47B452S0252	Pacific Scientific	10535	N/A	2-SNUB-074-5035	1981	Installed	Yes

7. Description of Work Replaced snubber with a like for like new snubber.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in P.O. 84-IP-0754 (ref. R40 060201 008) and Design Criteria BFN-50-7074 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks WO 06-718862-000 - (2-SNUB-074-5035)

Applicable Manufacturer's Data Reports to be attached

The original snubber (7270) was removed and tested as part of the 10 percent sample per the snubber program.

The replacement snubber (10535) is a new snubber and was functionally tested per 2-SI-4.6.H-2A prior to installation.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen L. Wilford* System Engineer
Owner or Owner's Designee Title

Date 1/26 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 1/27/06 to 1/29/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Paul Flinn
Inspector's Signature

Commissions TN4011
National Board, State, Province, and Endorsements

Date 1/29 20 07

FORM NF-1 NPT CER. CATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS
As Required by the Provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipe Support Corp. 74 C Commerce Way, Hobart, Massachusetts - 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Nat'l Board No.	(h) Year Built
(1) <u>10434-10617</u>	<u>None</u>	<u>1801103-07-H</u>	<u>DR-1416-REV. 0</u>	<u>Linear</u>	<u>1</u>	<u>None</u>	<u>1981</u>
(2)							
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks: Built in accordance with TVA/C.F. Brain Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
(Date)

Code Case No. 1644-5

Date 7/8/81 Signed Pacific Scientific by Ronald A. Nava
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific

Filed Per IA 3256

Design Specifications Certified by (1) Alex Valsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
PE State California Reg. No. 13533

(1) Last name only, signature not required

W 9/27/06

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2" x 11" (2) information in items 1, 2, 3, 4, 5 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSBI&I Co. of Hartford, CT

have inspected the component supports described in this Data Report on 7-8-81

is _____ and state that to the best of my knowledge and belief the NPT Certificate holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8-81

Signed William Meyer Commission Ca-1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commission _____
(Nat'l Bd., State, Prov., and No.)

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Date February 13, 2007
 Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2
 Work Orders (WO) 06-717668-000, 06-717669-000 and 06-717671-000
Repair/Replacement Organization P.O. No., Job No. etc

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 073, High Pressure Injection System (ASME Code Class 2 equivalent)

(snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda
 5. (a) Applicable Construction Code (piping sys) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47A455-390	Pacific Scientific	13117	N/A	2-SNUB-073-5016	N/A	Removed	No
Support (Snubber) 2-47A455-390	Pacific Scientific	10624	N/A	2-SNUB-073-5016	1980	Installed	Yes
Support (Snubber) 2-47B455S0042	Pacific Scientific	15619	N/A	2-SNUB-073-5009	N/A	Removed	No
Support (Snubber) 2-47B455S0042	Pacific Scientific	15493	N/A	2-SNUB-073-5009	1981	Installed	Yes
Support (Snubber) 2-47B455S0042	Pacific Scientific	10476	N/A	2-SNUB-073-5010	N/A	Removed	No
Support (Snubber) 2-47B455S0042	Pacific Scientific	16303	N/A	2-SNUB-073-5010	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure: Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE. Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7073 and BFN-50-C-7105.

FORM NIS-2 (Back)

Work Orders (WO) 06-717668-000, 06-717669-000 and 06-717671-000

9. Remarks The existing snubbers were replaced with new snubbers which were functionally tested per 2-SI-4.6.H-2A prior to installation. Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A
Certificate of Authorization No. N/A Expiration Date N/A
Signed [Signature] System Engineer Date 3-8 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 9/27/06 to 3/12/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 3/12 20 07

FORM NP-1 NPT CERTIFICATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS*
As Required by the Provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 100 Commerce Way, Hibernia, Massachusetts, 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) NPT Board No.	(h) Year Built
(1) <u>10483-</u>	<u>None</u>	<u>1801102-05-J</u>	<u>DR-1351-REV. A</u>	<u>Linear</u>	<u>1</u>	<u>None</u>	<u>1980</u>
(2) <u>10677</u>							
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
(Date)

Code Case No. 1644-5

Date 13 May 1980 Signed Pacific Scientific by Bill Jenkins
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design information on file at Pacific Scientific

Stress Report or Load Capacity Data Sheets on file at Pacific Scientific

Filed Per PA 3256

Design Specifications Certified by (1) Alex Valsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
PE State California Reg. No. 13533

(1) List name only, signature not required

Ch
9/28/86

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2" x 11", (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

000-375301

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSBI&I Co. of Bartford, CT

have inspected the component supports described in this Data Report on 5/16 19 80 and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed those component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 5/16/80

Signed William Meyer Commission Ca #1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed those component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commission _____
(Nat'l Bd., State, Prov., and No.)

0000373002

FORM NF-1 NPT CERTIFICATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS

As Required by ASME Code for Nuclear Power Plant Components, Section III, Division 1

000

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 Commerce Way, Woburn, Massachusetts, 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Nat'l Board No.	(h) Year Built
(1) 15403-15502	None	1801106-05-J	DR-1350-Rev. B	Linear	1	None	1981
(2)							
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
(Date)

Code Case No. 1644-5

Date 2/6/81 Signed Pacific Scientific by Perula A. Nava
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific

Filed Per IA 3256

Design Specifications Certified by (1) Alex Walsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
 PE State California Reg. No. 13533

OS 9/29/81

(1) List name only, signature not required.

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in., (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form

00005

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by ESBI&I Co. of Hartford, CT

Have inspected the component supports described in this Data Report on 2/6/81 and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2/6/81

Signed William May Commissions Ca. #1494 (Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commissions _____ (Nat'l Bd., State, Prov., and No.)

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 C Commerce Way, Hoburn, Massachusetts, 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) NPT Board No.	(h) Year Built
(1) <u>15678-15702</u>	<u>None</u>	<u>1801106-05-J</u>	<u>DR-1350-Rev. B</u>	<u>Linear</u>	<u>1</u>	<u>None</u>	<u>1981</u>
(2) <u>16291-16345</u>	<u>"</u>	<u>"</u>	<u>"</u>	<u>"</u>	<u>"</u>	<u>"</u>	<u>"</u>
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

0000873841

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conforms to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
(Date)

Code Case No. 1644-5
 Date 7/24/81 Signed Pacific Scientific by Rosalie A. Nava
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at:
Pacific Scientific
 Filed Per NA 3256

Design Specifications Certified by (1) Alex Nalsenko PE State California
 Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
 PE State California Reg. No. 13533

(1) List name only, signature not required. *OK 6/24/86*

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in., (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of the form.

Sheet 2 of 2

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSRI&I Co. of Hartford, CT

Have inspected the component supports described in this Data Report on 2/26/81 to 81 and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2/26/81

Signed William Mayer

Commission Ca #1494

(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commission _____

(Nat'l Bd., State, Prov., and No.)

000373342

S/N 16291-16345

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date February 26, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000 Work Orders (WO) 06-717647-000 and 06-717705-000
Address Repair/Replacement Organization P.O. No., Job No., etc
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System System 074, Residual Heat Removal System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B452S0157	Fronex A/DE	ADH-3000-780	N/A	2-SNUB-074-5008	N/A	Removed	No
Support (Snubber) 2-47B452S0157	Fronex A/DE	ADH-3000-867	N/A	2-SNUB-074-5008	N/A	Installed	No
Support (Snubber) R-92	Pacific Scientific	7272	N/A	2-SNUB-074-5041	N/A	Removed	No
Support (Snubber) R-92	Pacific Scientific	10695	N/A	2-SNUB-074-5041	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7074 and BFN-50-C-7105.

FORM NIS-2 (Back)

Work Orders (WO) 06-717647-000 and 06-717705-000

9. Remarks The existing snubbers were replaced with new snubbers.
Applicable Manufacturer's Data Reports to be attached
S/N ADH-3000-867 was functionally tested per 2-SI-4.6.H-2B prior to installation as 2-SNUB-074-5008.
S/N 10695 was functionally tested per 2-SI-4.6.H-2A prior to installation as 2-SNUB-074-5041.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams*, System Engineer Date 3-8 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee or Province of Connecticut and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 9/27/06 to 3/12/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

David Flood Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 3/12 20 07

FORM NPT CERTIFICATE HOLDERS DATA REPORT FOR COMPONENT SUPPORTS
 As Required by the provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 Commerce Way, Woburn, Massachusetts, 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Natl Board No.	(h) Year Built
(1) 10415-10424	None	1801103-07-H	DR-1416-Rev. 0	Linear	1	None	1981
(2) 10618-10699	"	"	"	"	"	"	"
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks: Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

*NEW SNUBBER USED 10695
 AHC*

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
(Date)

Code Case No. 1644-5

Date 7/1/81 Signed Pacific Scientific by Ronald A. Nava
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific

Filed Per NA 3256

Design Specifications Certified by (1) Alex Valsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
 PE State California Reg. No. 13533

1/8 9/27/81

(1) List name only, signature not required.

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in., (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered, and number of sheets is recorded at top of this form.

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspection and the State or Province of California and employed by HSBI&I Co. of Bartford, CT

_____ have inspected the component supports described in this Data Report on 7-8-81 19____ and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8-81

Signed William Meyer Commission Ca #1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commissions _____
(Nat'l Bd., State, Prov., and No.)

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date February 17, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Work Orders (WOs) 05-715970-000, 05-715971-000, 05-715972-000 and 05-719562-000, EDC 69128A
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 074, Residual Heat Removal (RHR) System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RHR System Fill Check Valve	Hancock	N/A	N/A	2-CKV-074-0792	N/A	‡	No
‡ - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-074-0792	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-34-0017	N/A	2-CKV-074-0792	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-074-0792	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-32-0013	N/A	2-CKV-074-0792	N/A	Installed	No

7. Description of Work Replaced bonnet/cap and disc

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7074 and BFN-50-C-7105.

FORM NIS-2 (Back)

WIDs Work Orders (WOs) 05-715970-000, 05-715971-000, 05-715972-000 and 05-719562-000

9. Remarks Replaced bonnet and disc

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the requirements of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed

[Signature]
Owner or Owner's Designee, Title

System Engineer

Date

7-13, 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 5/31/06 to 7/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions

TN4011

National Board, State, Province, and Endorsements

Date

7/13/ 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date February 17, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
4. Identification of System System 074, Residual Heat Removal (RHR) System (ASME Code Class 2 equivalent)
ASME Code Class 2 equivalent
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, Addenda, Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 .95 Edition, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RHR System Fill Check Valve	Hancock	N/A	N/A	2-CKV-074-0802	N/A	±	No
± - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-074-0802	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-34-0016	N/A	2-CKV-074-0802	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-074-0802	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-33-0014	N/A	2-CKV-074-0802	N/A	Installed	No
RHR System Fill Check Valve	Hancock	N/A	N/A	2-CKV-074-0803	N/A	±	No
± - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-074-0803	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-33-0013	N/A	2-CKV-074-0803	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-074-0803	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-33-0015	N/A	2-CKV-074-0803	N/A	Installed	No
RHR System Fill Check Valve	Hancock	N/A	N/A	2-CKV-074-0804	N/A	±	No
± - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-074-0804	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-34-0014	N/A	2-CKV-074-0804	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-074-0804	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-33-0016	N/A	2-CKV-074-0804	N/A	Installed	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date February 17, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Work Orders (WOs) 05-715974-000, 05-715975-000,
05-719563-000 and 05-719564-000, EDC 69128A
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 075, Core Spray (CS) System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
CS System Fill Check Valve	Hancock	N/A	N/A	2-CKV-075-0609	N/A	‡	No
‡ - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-075-0609	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-31-0008	N/A	2-CKV-075-0609	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-075-0609	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-31-0011	N/A	2-CKV-075-0609	N/A	Installed	No

7. Description of Work Replaced bonnet/cap and disc
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7075 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WOs) 05-715974-000, 05-715975-000, 05-719563-000 and 05-719564-000

9. Remarks Replaced bonnet and disc
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the requirements of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Stephen C. Willey, System Engineer Date 7-13, 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/9/06 to 7/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Stephen C. Willey Commissions TN4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 7/13/ 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date February 17, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Work Orders (WOs) 05-715974-000, 05-715975-000,
05-719563-000 and 05-719564-000, EDC 69128A
Repair/Replacement Organization P.O. No., Job No., etc.
 Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A
3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
4. Identification of System System 075, Core Spray (CS) System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition. N/A Addenda. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
CS System Fill Check Valve	Hancock	N/A	N/A	2-CKV-075-0610	N/A	‡	No
‡ - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-075-0610	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-31-0003	N/A	2-CKV-075-0610	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-075-0610	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-31-0007	N/A	2-CKV-075-0610	N/A	Installed	No
CS System Fill Check Valve	Hancock	N/A	N/A	2-CKV-075-0606		‡	No
‡ - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-075-0606	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-32-0010	N/A	2-CKV-075-0606	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-075-0606	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-31-0006	N/A	2-CKV-075-0606	N/A	Installed	No
CS System Fill Check Valve	Hancock	N/A	N/A	2-CKV-075-0607		‡	No
‡ - Replaced CKV cap and disc							
cap	Hancock	N/A	N/A	2-CKV-075-0607	N/A	Removed	No
cap	Anderson Greenwood Crosby	N900132-32-0011	N/A	2-CKV-075-0607	N/A	Installed	No
disc	Hancock	N/A	N/A	2-CKV-075-0607	N/A	Removed	No
disc	Anderson Greenwood Crosby	N900133-31-0001	N/A	2-CKV-075-0607	N/A	Installed	No

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date March 11, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order (WO) 06-711366-018
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Pipe Support Bolting	Unknown	N/A	N/A	2-47B400S0038	N/A	Removed	No
Main Steam Pipe Support Bolting	NOVA	N/A	N/A	2-47B400S0038	N/A	Installed	No
Main Steam Pipe Support Bolting	Unknown	N/A	N/A	2-47B400S0039	N/A	Removed	No
Main Steam Pipe Support Bolting	NOVA	N/A	N/A	2-47B400S0039	N/A	Installed	No
Main Steam Pipe Support Bolting	Unknown	N/A	N/A	2-47B400S0040	N/A	Removed	No
Main Steam Pipe Support Bolting	NOVA	N/A	N/A	2-47B400S0040	N/A	Installed	No
Main Steam Pipe Support Bolting	Unknown	N/A	N/A	2-47B400S0041	N/A	Removed	No
Main Steam Pipe Support Bolting	NOVA	N/A	N/A	2-47B400S0041	N/A	Installed	No

7. Description of Work Replaced bolting damaged during disassembly.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt

Other ** Pressure N/A psi Test Temp. N/A °F

** - see Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in AISC and Design Criteria BFN-50-7001, BFN-50-C-7105 and BFN-50-C-7107.

FORM NIS-2 (Back)

WID: Work Order (WO) 06-711366-018

9. Remarks Replaced bolting damaged during disassembly. Performed VT-3 exams of supports following installation.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed: [Signature] System Engineer
Owner or Owner's Designee, Title

Date 3-11, 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 3/4/07 to 3/13/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions TN4011
National Board, State, Province, and Endorsements

Date 3/13, 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner: Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Date May 18, 2007
 Sheet 1 of 5

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2
 Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code (snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda, 1644-5 Code Case
(piping sys) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) RSSG-3	Pacific Scientific	468	N/A	2-SNUB-001-5022	N/A	Removed	No
Support (Snubber) RSSG-3	Pacific Scientific	10580	N/A	2-SNUB-001-5022	1981	Installed	Yes
Support (Snubber) RSSG-4	Pacific Scientific	471	N/A	2-SNUB-001-5023	N/A	Removed	No
Support (Snubber) RSSG-4	Pacific Scientific	10581	N/A	2-SNUB-001-5023	1981	Installed	Yes
Support (Snubber) RSSG-5	Pacific Scientific	408	N/A	2-SNUB-001-5024	N/A	Removed	No
Support (Snubber) RSSG-5	Pacific Scientific	6809	N/A	2-SNUB-001-5024	1980	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000

9. Remarks The existing pre-NF snubbers were removed and discarded.
Applicable Manufacturer's Data Reports to be attached
New snubbers were functionally tested per 2-SI-4.6.H-2A and installed.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams* System Engineer Date 6-8 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
or Province of Tennessee and employed by HSB CT of
Connecticut have inspected the components described
in this Owner's Report during the period 9/13/06 to 6/13/07, and state that
to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's
Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in
any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Samuel Flood Commissions TN4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/13 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Name

Date May 18, 2007

1101 Market Street

Chattanooga, TN 37402-2801 Address

Sheet 2 of 5

2. Plant Browns Ferry Nuclear Plant (BFN) Name

Unit 2

P. O. Box 2000, Decatur, AL 35609-2000 Address

Work Orders (WOs) 04-722413-031 through 04-722413-042 and 04-722413-044 through 04-722413-052 and 06-717703-000 and 06-717704-000

3. Work Performed by TVA-BFN Address

Repair/Replacement Organization P.O. No., Job No., etc. Type Code Symbol Stamp N/A

P. O. Box 2000, Decatur, AL 35609-2000 Name

Authorization No. N/A

P. O. Box 2000, Decatur, AL 35609-2000 Address

Expiration Date N/A

4. Identification of System System 001 Main Steam System (ASME Code Class 2 equivalent)

(snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda, 1644-5 Code Case

5. (a) Applicable Construction Code (piping sys) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) RSSH-3	Pacific Scientific	439	N/A	2-SNUB-001-5027	N/A	Removed	No
Support (Snubber) RSSH-3	Pacific Scientific	10583	N/A	2-SNUB-001-5027	1981	Installed	Yes
Support (Snubber) RSSH-4	Pacific Scientific	438	N/A	2-SNUB-001-5028	N/A	Removed	No
Support (Snubber) RSSH-4	Pacific Scientific	10591	N/A	2-SNUB-001-5028	1981	Installed	Yes
Support (Snubber) RSSH-5	Pacific Scientific	420	N/A	2-SNUB-001-5029	N/A	Removed	No
Support (Snubber) RSSH-5	Pacific Scientific	10597	N/A	2-SNUB-001-5029	1981	Installed	Yes
Support (Snubber) RSSJ-3	Pacific Scientific	398	N/A	2-SNUB-001-5032	N/A	Removed	No
Support (Snubber) RSSJ-3	Pacific Scientific	10685	N/A	2-SNUB-001-5032	1981	Installed	Yes
Support (Snubber) RSSJ-4	Pacific Scientific	413	N/A	2-SNUB-001-5033	N/A	Removed	No
Support (Snubber) RSSJ-4	Pacific Scientific	10643	N/A	2-SNUB-001-5033	1981	Installed	Yes
Support (Snubber) RSSK-4	Pacific Scientific	466	N/A	2-SNUB-001-5037	N/A	Removed	No
Support (Snubber) RSSK-4	Pacific Scientific	10649	N/A	2-SNUB-001-5037	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt

Other: ** Pressure N/A psi Test Temp. N/A °F

** - See Remarks

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000

Remarks The existing pre-NF snubbers were removed and discarded.

New snubbers were functionally tested per 2-SI-4.6.H-2A and installed.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street

Date May 18, 2007

Chattanooga, TN 37402-2801
Address

Sheet 3 of 5

2. Plant Browns Ferry Nuclear Plant (BFN)
Name

Unit 2

Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000

3. Work Performed by TVA-BFN
Name

Repair/Replacement Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A

P. O. Box 2000, Decatur, AL 35609-2000
Address

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001 Main Steam System (ASME Code Class 2 equivalent)

(snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda, 1644-5 Code Case

5. (a) Applicable Construction Code (piping sys) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) RSSK-5	Pacific Scientific	467	N/A	2-SNUB-001-5071	N/A	Removed	No
Support (Snubber) RSSK-5	Pacific Scientific	10652	N/A	2-SNUB-001-5071	1981	Installed	Yes
Support (Snubber) RSSL-3	Pacific Scientific	434	N/A	2-SNUB-001-5040	N/A	Removed	No
Support (Snubber) RSSL-3	Pacific Scientific	10653	N/A	2-SNUB-001-5040	1981	Installed	Yes
Support (Snubber) RSSL-4	Pacific Scientific	473	N/A	2-SNUB-001-5041	N/A	Removed	No
Support (Snubber) RSSL-4	Pacific Scientific	10654	N/A	2-SNUB-001-5041	1981	Installed	Yes
Support (Snubber) RSSE-4	Pacific Scientific	407	N/A	2-SNUB-001-5017	N/A	Removed	No
Support (Snubber) RSSE-4	Pacific Scientific	10655	N/A	2-SNUB-001-5017	1981	Installed	Yes
Support (Snubber) RSSF-1	Pacific Scientific	402	N/A	2-SNUB-001-5018	N/A	Removed	No
Support (Snubber) RSSF-1	Pacific Scientific	10657	N/A	2-SNUB-001-5018	1981	Installed	Yes
Support (Snubber) RSSF-2	Pacific Scientific	7277	N/A	2-SNUB-001-5019	N/A	Removed	No
Support (Snubber) RSSF-2	Pacific Scientific	10660	N/A	2-SNUB-001-5019	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000

Remarks The existing pre-NF snubbers were removed and discarded.

New snubbers were functionally tested per 2-SI-4.6.H-2A and installed.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date May 18, 2007
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 4 of 5
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Unit 2
 Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 001 Main Steam System (ASME Code Class 2 equivalent)
(snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda, 1644-5 Code Case
5. (a) Applicable Construction Code (piping sys) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) RSSG-2	Pacific Scientific	433	N/A	2-SNUB-001-5021	N/A	Removed	No
Support (Snubber) RSSG-2	Pacific Scientific	10664	N/A	2-SNUB-001-5021	1981	Installed	Yes
Support (Snubber) RSSH-2	Pacific Scientific	470	N/A	2-SNUB-001-5026	N/A	Removed	No
Support (Snubber) RSSH-2	Pacific Scientific	10667	N/A	2-SNUB-001-5026	1981	Installed	Yes
Support (Snubber) RSSJ-1	Pacific Scientific	513	N/A	2-SNUB-001-5030	N/A	Removed	No
Support (Snubber) RSSJ-1	Pacific Scientific	10670	N/A	2-SNUB-001-5030	1981	Installed	Yes
Support (Snubber) RSSJ-2	Pacific Scientific	431	N/A	2-SNUB-001-5031	N/A	Removed	No
Support (Snubber) RSSJ-2	Pacific Scientific	10686	N/A	2-SNUB-001-5031	1981	Installed	Yes
Support (Snubber) RSSL-1	Pacific Scientific	423	N/A	2-SNUB-001-5038	N/A	Removed	No
Support (Snubber) RSSL-1	Pacific Scientific	10675	N/A	2-SNUB-001-5038	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street

Date May 18, 2007

Chattanooga, TN 37402-2801
Address

Sheet 5 of 5

2. Plant Browns Ferry Nuclear Plant (BFN)
Name

Unit 2

Work Orders (WOs) 04-722413-031 through 04-722413-042
and 04-722413-044 through 04-722413-052 and
06-717703-000 and 06-717704-000

3. Work Performed by TVA-BFN
Name

Repair/Replacement Organization P.O. No., Job No., etc.
 Type Code Symbol Stamp N/A

P. O. Box 2000, Decatur, AL 35609-2000
Address

Authorization No. N/A

4. Identification of System System 001 Main Steam System (ASME Code Class 2 equivalent)
Expiration Date N/A

(snubber) ASME Section III, NF, 1977 Edition, Summer 1977 Addenda, 1644-5 Code Case

5. (a) Applicable Construction Code (piping sys) USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) RSSL-2	Pacific Scientific	428	N/A	2-SNUB-001-5039	N/A	Removed	No
Support (Snubber) RSSL-2	Pacific Scientific	10684	N/A	2-SNUB-001-5039	1981	Installed	Yes
Support (Snubber) MSS-9E	Pacific Scientific	4248	N/A	2-SNUB-001-5062	N/A	Removed	No
Support (Snubber) MSS-9E	Pacific Scientific	10693	N/A	2-SNUB-001-5062	1981	Installed	Yes
Support (Snubber) MSS-9W	Pacific Scientific	5213	N/A	2-SNUB-001-5063	N/A	Removed	No
Support (Snubber) MSS-9W	Pacific Scientific	10692	N/A	2-SNUB-001-5063	1981	Installed	Yes

7. Description of Work Replaced snubbers with new snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F ** - See Remarks

FORM NF-1 NPT CERTIFICATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS*
As Required by the Provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S-. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 C Commerce Way, Woburn, Mass. 01801
(Name and address of purchaser or owner)

3. Location of Installation: Unknown

4. Identification

(a)	(b)	(c)	(d)	(e)	(f)	(g)	(h)
Component Support I.D. No.	Canadian Registration No.	Applicable Drawings with Last Rev. & Date	Stress Report or Load Capacity Data Sheet	Type of Component Support	Class	Nat'l. Board No.	Year Built
(1) 6791-6836	None	1801103-07-H	DR-1352-Rev. B	Linear	1	None	1990
(2)							
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks Built TVA/C.F. Braun Design Spec. No. 400-20

000873746

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977 Addenda Summer 1977
Code Case No. 1644-5 (Date)

Date 2-8-80 Signed Pacific Scientific by Ed Yager
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. N-1198 to use the Component Support
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific
Filed Per NA 3256

Design Specifications Certified by (1) Alex Walsenko PE State California
Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) LEO E. Ay
PE State California Reg. No. 13533

(1) List name only, signature not required

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in. (2) information in items 1, 2 & 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

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07/3/07

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B35403/Bell

FORM NF-1 NPT CERTIFICATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS*
As Required by the Provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 C Commerce Way, Woburn, Mass. 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

	(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Nat'l. Board No.	(h) Year Built
(1)	6791-6836	None	1801103-07-H	DR-1352-Rev. B	Linear	1	None	1980
(2)								
(3)								
(4)								
(5)								
(6)								
(7)								
(8)								
(9)								
(10)								

5. Remarks Built TVA/C.F. Braun Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
Code Case No. 1644-5 (Date)

Date 2-8-80 Signed Pacific Scientific by Ed Yager
(NPT Certificate Holder)

Our ASME Certificate of Authorization No. N-1198 to use the Component Support
(NPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific
Filed Per NA 3256

Design Specifications Certified by (1) Alex Walsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay

PE State California Reg. No. 13533

(1) List name only, signature not required

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8" x 11", (2) information in items 1, 2 & 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

Handwritten signatures and dates:
12/8/80
07/31/81

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035403/Bell

FORM NPT CERTIFICATE HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS

As Required by the Provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803

(Name and address of NPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 C Commerce Way, Woburn, Massachusetts, 01801

(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Nat'l Board No.	(h) Year Built
(1) 10415-10424	None	1801103-07-B	DR-1416-Rev. 0	Linear	1	None	1981
(2) 10618-10699	"	"	"	"	"	"	"
(3)							
(4)							
(5)							
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks: Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977

Code Case No. 1644-5

Date 7/1/81 Signed Pacific Scientific by Roalve A. Nava

(NPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports

(NPT)

Symbol expires Aug. 4, 1981

(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at:

Pacific Scientific

Filed Per IA 3256

Design Specifications Certified by (1) Alex Walsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay

PE State California Reg. No. 13533

(1) List name only, signature not required.

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in., (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSBI&I Co. of Hartford, CT

have inspected the component supports described in this Data Report on 7-8-81

and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8-81

Signed William Meyer Commissions Co #1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commissions _____
(Nat'l Bd., State, Prov., and No.)

INPT CERTIFICATE HOLDERS DATA REPORT FOR COMPONENT SUPPORTS

As Required by the Provisions of the ASME Code Rules, Section III, Division 1, Edition 1977 *10/13*

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803

(Name and address of Certificate Holder)

2. Manufacturer for Bergen Paterson Pipe Support Corp. 74 C Commerce Way, North Attleboro, Massachusetts 01901

(Name and address of purchaser or owner)

3. Location of installation Unknown

4. Identification

(a)	(b)	(c)	(d)	(e)	(f)	(g)	(h)
Component Support I.D. No.	Canadian Registration No.	Applicable Drawings with Last Rev. & Date	Stress Report or Load Capacity Data Sheet	Type of Component Support	Class	Material Board No.	Year Built

(1) 10434-10617 None 1801103-07-B DR-1416-Rev. 0 Linear ? None 1981

(2) _____

(3) _____

(4) _____

(5) _____

(6) _____

(7) _____

(8) _____

(9) _____

(10) _____

5. Built in accordance with TVA/C.F. Basin Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977

Code Case No. 1544-5 (Date)

Date 7/8/81 Signed Pacific Scientific by Ronald A. Nava

(INPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports

(INPT)

Symbol expires Aug. 4, 1981 (Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at Pacific Scientific

Filed Per IA 3256

Design Specifications Certified by (1) Alex Val'senko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay

State California Reg. No. 13533

(1) List name only, signature not required

9/13/81

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2" x 11", (2) information in items 1, 2, 4, 5 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of
 Province of California and employed by RSBISI Co. of Hartford, CT

have inspected the component supports described in this Data Report on 7-8-81
 and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance
 with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component
 supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any
 personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8-81
 Signed William Meyer Commissions Ca # 1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

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I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of
 Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state
 that the parts referred to as data items _____, not included in the certificate of shop inspection, have been
 inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accord-
 ance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component
 supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any
 personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____
 Signed _____ Commissions _____
(Nat'l Bd., State, Prov., and No.)

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date May 18, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Orders (WOs) 06-717672-000, 07-711244-000 and 07-711245-000
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 003, Feedwater System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B415S0019	Pacific Scientific	7242	N/A	2-SNUB-003-5030	N/A	Removed	Yes
Support (Snubber) 2-47B415S0019	Pacific Scientific	10694	N/A	2-SNUB-003-5030	1981	Installed	Yes
Support (Snubber) 2-47B415S0002	Bergen Patterson/ Anchor Darling	ADH-1000-3416	N/A	2-SNUB-003-5016	N/A	Removed	No
Support (Snubber) 2-47B415S0002	Bergen Patterson	TVA Serial # M0365	N/A	2-SNUB-003-5016	N/A	Installed	No
Support (Snubber) 2-47B415S0010	Bergen Patterson	TVA Serial # M0253	N/A	2-SNUB-003-5025	N/A	Removed	No
Support (Snubber) 2-47B415S0010	Bergen Patterson	TVA Serial # M0176	N/A	2-SNUB-003-5025	N/A	Installed	No

7. Description of Work Replaced 2-SNUB-003-5030 with like for like new snubber.
Replaced 2-SNUB-003-5016 and 2-SNUB-003-5025 with rebuilt snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7003 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

WO 06-717672-000 - (2-SNUB-003-5030) -

The original snubber (7242) was removed and tested as part of the 10 percent sample per the snubber program.

The replacement snubber (10694) is a new snubber and was functionally tested per 2-SI-4.6.H-2A prior to installation.

WO 07-711244-000 - (2-SNUB-003-5016) -

The original snubber (ADH-1000-3416) was removed due to a Part 21 concern (PER 117785).

The replacement snubber (TVA Serial # M0366) was removed from 3-SNUB-003-5028 under WO 03-014819-000 and rebuilt under this WO.

Rebuild included replacement of the main cylinder and functional testing per 2-SI-4.6.H-2B prior to installation.

WO 07-711245-000 - (2-SNUB-003-5025) -

The original snubber (TVA Serial # M0253) was removed due to a Part 21 concern (PER 117785).

The replacement snubber (TVA Serial # M0176) was removed from 3-SNUB-003-5026 under WO 03-014912-000 and rebuilt under this WO.

Rebuild included replacement of the main cylinder and functional testing per 2-SI-4.6.H-2B prior to installation.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen C. Wilford* System Engineer

Date 5-18 20 07

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 9/27/06 to 6/11/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Jan F. Hurl
Inspector's Signature

Commissions

 TN4011

National Board, State, Province, and Endorsements

Date 6/11/ 20 07

FORM NP-1 INPT CERTIFIED HOLDERS' DATA REPORT FOR COMPONENT SUPPORTS
As Required by the provisions of the ASME Code Rules, Section III, Division 1

1. Manufactured by Pacific Scientific 1346 S. State College Blvd. Anaheim, Ca. 92803
(Name and address of INPT Certificate Holder)

2. Manufacturer for Bergen Paterson Pipesupport Corp. 74 C. Commerce Way, Ibburn, Massachusetts, 01801
(Name and address of purchaser or owner)

3. Location of Installation Unknown

4. Identification

(a) Component Support I.D. No.	(b) Canadian Registration No.	(c) Applicable Drawings with Last Rev. & Date	(d) Stress Report or Load Capacity Data Sheet	(e) Type of Component Support	(f) Class	(g) Nat'l Board No.	(h) Year Built
(1) 10415-10424	None	1801103-07-H	DR-1416-Rev. 0	Linear	1	None	1981
(2) 10618-10699	"	"	"	"	"	"	"
(3)							
(4)							
(5)		<u>NEW S/N 10694 USED</u>					
(6)							
(7)							
(8)							
(9)							
(10)							

5. Remarks Built in accordance with TVA/C.F. Braun Design Spec. No. 400-20

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that these components supports conform to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Division 1, Edition 1977, Addenda Summer 1977
Code Case No. 1644-5 (Date)

Date 7/1/81 Signed Pacific Scientific by Rosalie A. Nava
(INPT Certificate Holder)

Our ASME Certificate of Authorization No. 1198 to use the Component Supports
(INPT)

Symbol expires Aug. 4, 1981
(Date)

CERTIFICATION OF DESIGN

Design Information on File at Pacific Scientific

Stress Report or Load Capacity Data Sheets on File at:
Pacific Scientific

Filed Per NA 3256

Design Specifications Certified by (1) Alex Valsenko PE State California

Reg. No. C22,109

Stress Analysis Report or Load Capacity Data Sheets Certified by (1) Leo E. Ay
PE State California Reg. No. 13533

of 9/27/86

(1) List name only, signature not required.

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8 1/2 in., (2) information in items 1, 2, 4c, 4g on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded at top of this form.

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSBI&I Co. of Hartford, CT

have inspected the component supports described in this Data Report on 7-8-81

and state that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8-81

Signed William Meyer Commissions Ca #1494
(Nat'l Bd., State, Prov., and No.)

CERTIFICATION OF FIELD INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have compared the statements in this Data Report with the described component supports and state that the parts referred to as data items _____, not included in the certificate of shop inspection, have been inspected by me and that to the best of my knowledge and belief the NPT Certificate Holder has constructed these component supports in accordance with the ASME Code for Nuclear Power Plant Components.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the component supports described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____

Signed _____ Commissions _____
(Nat'l Bd., State, Prov., and No.)

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date May 19, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
Work Order (WO) 02-011925-001
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Attachment) 2-47B400S0110	TVA	N/A	N/A	2-SNUB-001-5060	N/A	Corrected	No

7. Description of Work Modified pipe support 2-47B400S0110. Added additional members and extended some welds on existing members.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WO 02-011925-001

9: Remarks Modified pipe support 2-47B400S0110. Added additional members and extended some welds on existing members.

Applicable Manufacturer's Data Reports to be attached

Performed surface exams on the integral attachment welds and a visual of the modified support.

Multiple horizontal lines for additional remarks or data entry.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 6-7 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/1/07 to 6/12/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/12 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 11, 2007
Name
1:101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
Work Order (WO) 06-719319-000
Repair/Replacement Organization P. O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 085, Control Rod Drive (CRD) System (ASME Code Class.1 equivalent)
5. (a) Applicable Construction Code ASME Section III 19 74 Edition, Winter 1975 Addenda, N207-1361-2 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements, 19 95 Edition, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 18-55	General Electric Nuclear Energy	A4700	N/A	2-CRDM-085-18-55	1988	Removed	Yes
Control Rod Drive Mechanism 18-55	General Electric Nuclear Energy	A4227	N/A	2-CRDM-085-18-55	1992	Installed	Yes
Control Rod Drive Mechanism 34-47	General Electric Nuclear Energy	A5539	N/A	2-CRDM-085-34-47	1992	Removed	Yes
Control Rod Drive Mechanism 34-47	General Electric Nuclear Energy	A5437	N/A	2-CRDM-085-34-47	1992	Installed	Yes
Control Rod Drive Mechanism 46-39	General Electric Nuclear Energy	A4481	N/A	2-CRDM-085-46-39	1988	Removed	Yes
Control Rod Drive Mechanism 46-39	General Electric Nuclear Energy	A3611	N/A	2-CRDM-085-46-39	1992	Installed	Yes

Identification of Components continued on Page 2

7. Description of Work Replaced 24 Control Rod Drives (CRD); used 22 refurbished and 2 new CRDs.
Replaced two CRD flange bolts on 2-CRDM-085-30-27
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

WID: Work Order (WO) 06-719319-000

9. Remarks Replaced 24 Control Rod Drives (CRD); used 22 refurbished and 2 new CRDs.
Applicable Manufacturer's Data Reports to be attached
Replaced two CRD flange bolts on 2-CRDM-085-30-27

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Stephen C. Wilford System Engineer Date 6-14 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/23/07 to 6/15/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Sam Flood Commissions TN4011
National Board, State, Province, and Endorsements

Date 6/23/07 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY

SUPPLEMENTAL SHEET

- Owner Tennessee Valley Authority (TVA) Date June 11, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 4
Name Unit 2
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Order (WO) 06-719319-000
3. Work Performed by TVA-BFN Repair/Replacement Organization P.O. No., Job No., etc.
Name Type Code Symbol Stamp N/A
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 085, Control Rod Drive System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code ASME Section III 19 74 Edition, Winter 1975 Addenda, N207 1361-2 Code Case
 Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 06-35	General Electric Nuclear Energy	A5508	N/A	2-CRDM-085-06-35	1988	Removed	Yes
Control Rod Drive Mechanism 06-35	General Electric Nuclear Energy	A4833	N/A	2-CRDM-085-06-35	1992	Installed	Yes
Control Rod Drive Mechanism 30-31	General Electric Nuclear Energy	A5196	N/A	2-CRDM-085-30-31	1988	Removed	Yes
Control Rod Drive Mechanism 30-31	General Electric Nuclear Energy	A5406	N/A	2-CRDM-085-30-31	1992	Installed	Yes
Control Rod Drive Mechanism 30-27 and flange bolts	General Electric Nuclear Energy	A3741	N/A	2-CRDM-085-30-27	1995	Removed	Yes
Control Rod Drive Mechanism 30-27 and 2 flange bolts	General Electric Nuclear Energy	A4155	N/A	2-CRDM-085-30-27	1992	Installed	Yes
Control Rod Drive Mechanism 54-27	General Electric Nuclear Energy	A4268	N/A	2-CRDM-085-54-27	1988	Removed	Yes
Control Rod Drive Mechanism 54-27	General Electric Nuclear Energy	A6845	N/A	2-CRDM-085-54-27	2007	Installed	Yes
Control Rod Drive Mechanism 22-15	General Electric Nuclear Energy	A4100	N/A	2-CRDM-085-22-15	1988	Removed	Yes
Control Rod Drive Mechanism 22-15	General Electric Nuclear Energy	A4325	N/A	2-CRDM-085-22-15	1992	Installed	Yes
Control Rod Drive Mechanism 42-11	General Electric Nuclear Energy	A5077	N/A	2-CRDM-085-42-11	1988	Removed	Yes
Control Rod Drive Mechanism 42-11	General Electric Nuclear Energy	A2151	N/A	2-CRDM-085-42-11	1992	Installed	Yes
Control Rod Drive Mechanism 26-59	General Electric Nuclear Energy	A3919	N/A	2-CRDM-085-26-59	1988	Removed	Yes
Control Rod Drive Mechanism 26-59	General Electric Nuclear Energy	A3872	N/A	2-CRDM-085-26-59	1992	Installed	Yes
Control Rod Drive Mechanism 30-55	General Electric Nuclear Energy	A4412	N/A	2-CRDM-085-30-55	1988	Removed	Yes
Control Rod Drive Mechanism 30-55	General Electric Nuclear Energy	A3759	N/A	2-CRDM-085-30-55	1992	Installed	Yes

Identification of Components continued on Page 3

7. Description of Work Replaced 24 Control Rod Drives (CRD); used 22 refurbished and 2 new CRDs. Replaced two CRD flange bolts on 2-CRDM-085-30-27
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

- Owner Tennessee Valley Authority (TVA) Date June 11, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 3 of 4
Name Unit 2
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Order (WO) 06-719319-000
Address Repair/Replacement Organization P. O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name Authorization No. N/A
P. O. Box 2000, Decatur, AL 35609-2000
Address Expiration Date N/A
4. Identification of System System 085, Control Rod Drive System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code ASME Section III 19 74 Edition, Winter 1975 Addenda, N207 1361-2 Code Case
 Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 10-47	General Electric Nuclear Energy	A5666	N/A	2-CRDM-085-10-47	1988	Removed	Yes
Control Rod Drive Mechanism 10-47	General Electric Nuclear Energy	A4002	N/A	2-CRDM-085-10-47	1992	Installed	Yes
Control Rod Drive Mechanism 26-47	General Electric Nuclear Energy	A4719	N/A	2-CRDM-085-26-47	1988	Removed	Yes
Control Rod Drive Mechanism 26-47	General Electric Nuclear Energy	A5573	N/A	2-CRDM-085-26-47	1992	Installed	Yes
Control Rod Drive Mechanism 50-47	General Electric Nuclear Energy	A5310	N/A	2-CRDM-085-50-47	1988	Removed	Yes
Control Rod Drive Mechanism 50-47	General Electric Nuclear Energy	A4756	N/A	2-CRDM-085-50-47	1992	Installed	Yes
Control Rod Drive Mechanism 02-43	General Electric Nuclear Energy	A5279	N/A	2-CRDM-085-02-43	1988	Removed	Yes
Control Rod Drive Mechanism 02-43	General Electric Nuclear Energy	A5576	N/A	2-CRDM-085-02-43	1992	Installed	Yes
Control Rod Drive Mechanism 50-39	General Electric Nuclear Energy	A4788	N/A	2-CRDM-085-50-39	1988	Removed	Yes
Control Rod Drive Mechanism 50-39	General Electric Nuclear Energy	A5604	N/A	2-CRDM-085-50-39	1992	Installed	Yes
Control Rod Drive Mechanism 54-39	General Electric Nuclear Energy	A5120	N/A	2-CRDM-085-54-39	1988	Removed	Yes
Control Rod Drive Mechanism 54-39	General Electric Nuclear Energy	A5418	N/A	2-CRDM-085-54-39	1992	Installed	Yes
Control Rod Drive Mechanism 34-35	General Electric Nuclear Energy	A4816	N/A	2-CRDM-085-34-35	1988	Removed	Yes
Control Rod Drive Mechanism 34-35	General Electric Nuclear Energy	A5231	N/A	2-CRDM-085-34-35	1992	Installed	Yes
Control Rod Drive Mechanism 50-31	General Electric Nuclear Energy	A5528	N/A	2-CRDM-085-50-31	1988	Removed	Yes
Control Rod Drive Mechanism 50-31	General Electric Nuclear Energy	A2134	N/A	2-CRDM-085-50-31	1992	Installed	Yes

Identification of Components continued on Page 4

7. Description of Work Replaced 24 Control Rod Drives (CRD); used 22 refurbished and 2 new CRDs. Replaced two CRD flange bolts on 2-CRDM-085-30-27
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

- Owner Tennessee Valley Authority (TVA) Date June 11, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 4 of 4
Name
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Address
 Work Order (WO) 06-719319-000
3. Work Performed by TVA-BFN Repair/Replacement Organization P.O. No., Job No., etc.
Name Type Code Symbol Stamp N/A
P. O. Box 2000, Decatur, AL 35609-2000 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System System 085: Control Rod Drive System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code ASME Section III 19 74 Edition, Winter 1975 Addenda, N207 1361-2 Code Case
 Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 38-27	General Electric Nuclear Energy	A5037	N/A	2-CRDM-085-38-27	1988	Removed	Yes
Control Rod Drive Mechanism 38-27	General Electric Nuclear Energy	A6873	N/A	2-CRDM-085-38-27	2007	Installed	Yes
Control Rod Drive Mechanism 06-23	General Electric Nuclear Energy	A5073	N/A	2-CRDM-085-06-23	1988	Removed	Yes
Control Rod Drive Mechanism 06-23	General Electric Nuclear Energy	A5678	N/A	2-CRDM-085-06-23	1992	Installed	Yes
Control Rod Drive Mechanism 50-23	General Electric Nuclear Energy	A5030	N/A	2-CRDM-085-50-23	1988	Removed	Yes
Control Rod Drive Mechanism 50-23	General Electric Nuclear Energy	A5709	N/A	2-CRDM-085-50-23	1992	Installed	Yes
Control Rod Drive Mechanism 54-23	General Electric Nuclear Energy	A5126	N/A	2-CRDM-085-54-23	1988	Removed	Yes
Control Rod Drive Mechanism 54-23	General Electric Nuclear Energy	A5433	N/A	2-CRDM-085-54-23	1992	Installed	Yes
Control Rod Drive Mechanism 42-19	General Electric Nuclear Energy	A4780	N/A	2-CRDM-085-42-19	1988	Removed	Yes
Control Rod Drive Mechanism 42-19	General Electric Nuclear Energy	A4445	N/A	2-CRDM-085-42-19	1992	Installed	Yes

7. Description of Work Replaced 24 Control Rod Drives (CRD); used 22 refurbished and 2 new CRDs. Replaced two CRD flange bolts on 2-CRDM-085-30-27
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

RD-176334

G06022

1798

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A2151 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 11/19/92

Signed GE-NEBG-NF & CM-OA
(NPT Certificate Holder)

By [Signature]
(SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPT N-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/27/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any persons' injury or property damage or a loss of any kind arising from or connected with this inspection.

11/19/92
Date

[Signature]
Inspector's Signature

NO. 1151-0001-100 3588 PA
Assigned Shop State, Province and/or

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS".

OK
11/21/92

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A2151 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson

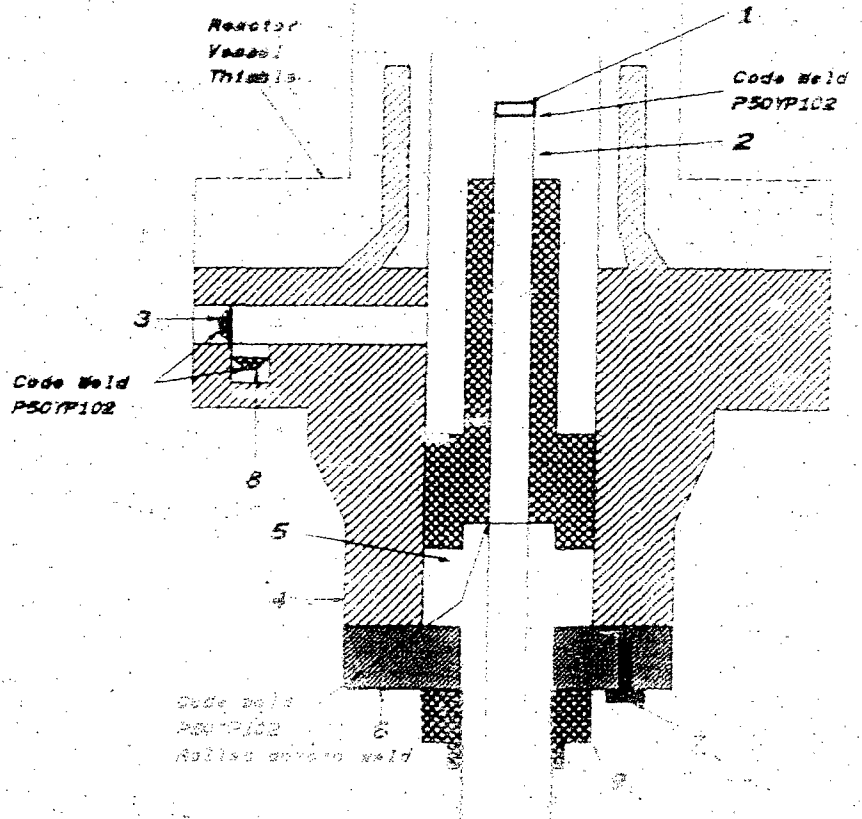
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 3.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
5 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7951P001
SA182 - F304
0.38" thick x 1.207" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.00" thick x 2.62" dia.



RD-176334

G06022

2128

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/M of Part: A2134 Nat'l Bd. No. N/A

(c) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(d) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(e) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 12/02/92

Signed GE-NEBG-NF&CM-OA By [Signature]

(NPT Certificate Holder)

(QC Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPT N-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1

Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15579

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018846

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 11/16, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

12/02/1992

[Signature]

NO 1231 Ohio RD 3626 PA

Date

Inspector's Signature

National Board, State, Province and City

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in Item 3 of this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS".

at
9/27/07

RD-176334
G06022

1776

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part: A3611 Nat'l Bd. No. N/A

Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Paterson

Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of services for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance, as defined in the code, conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report, if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 11/19/92 Signed GE-NEBG-NF & CM-OA by [Signature]
(NPT Certificate Holder) SC OR Representative

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No: NPT N-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/28, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

11/19 1992 [Signature]
Date Inspector's Signature

NO 1201, CHIC, NC 27695 PA
National Board, State, Province and City

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information is on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS".

CA
H23/67

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A3611 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G003 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

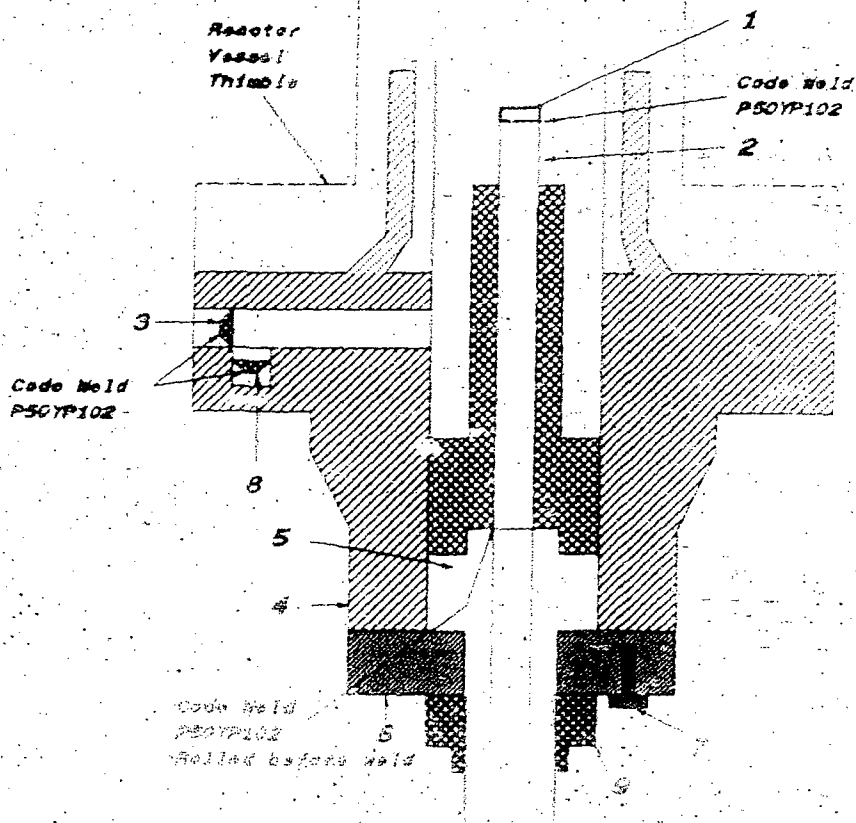
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 918D810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" ood circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.30" dia.
9. Nut 137C53934P001
QA - 19 SA473
1.30" thick x 2.52" dia.



RD-776334
G06022

1754

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A3759 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207.1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 11/19/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 45570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/28, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

11/19 1992 [Signature] NC 1231 Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

157/901

11/29/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A3759 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson

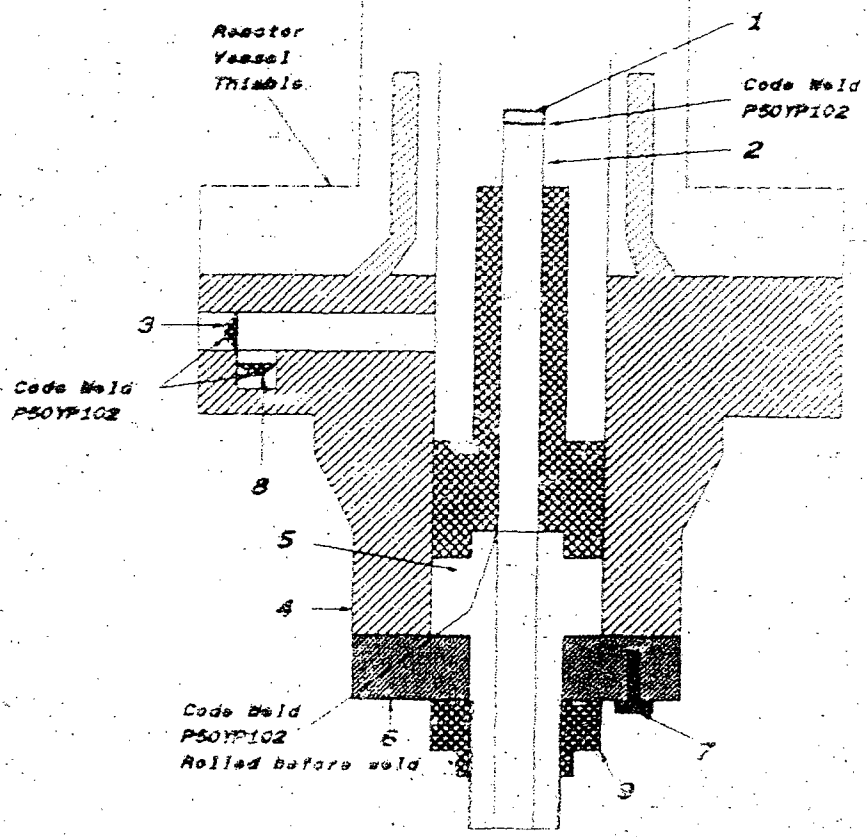
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.085" max dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 3.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-116334
G06022 3699

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA : Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part : A3872 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 03/13/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (N/A Representative)

Certificate of Authorization Expires: 6/18/93 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22AE253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 3/10, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

3/13, 1992 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS"

(07/90)
04123107

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A3872 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

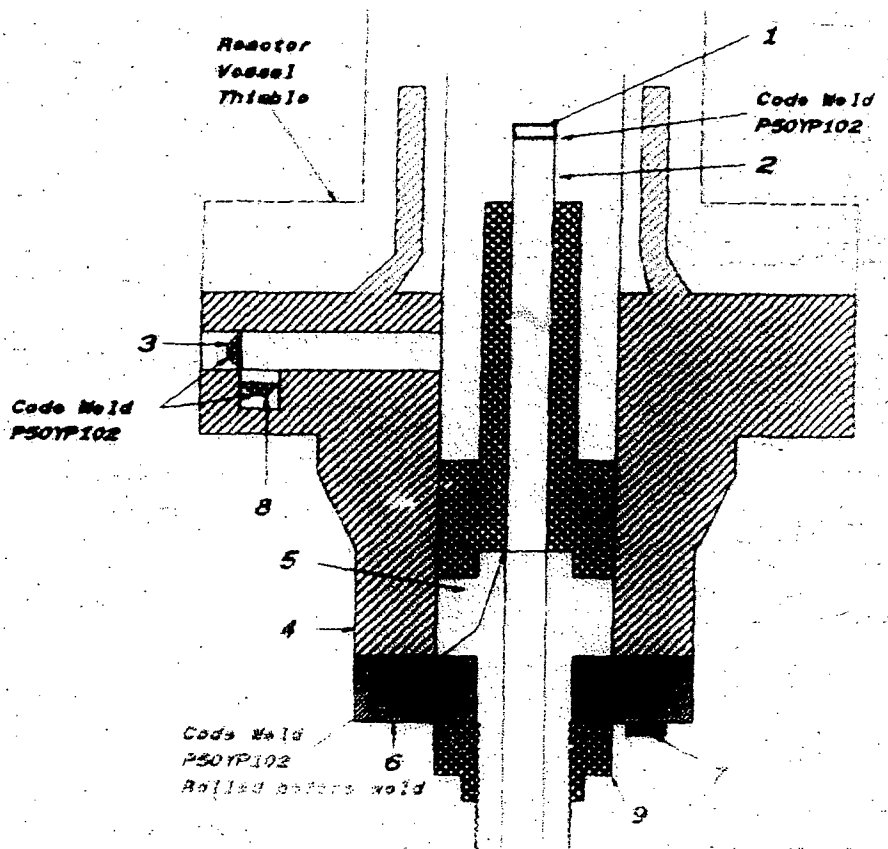
(b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
0.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B7
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334

G06022

1278

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

2. Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of NPT Certificate Holder for completed nuclear component)

3. Identification - Certificate Holder's S/N of Part: A4002 Nat'l Bd. No. N/A

4. Constructed According to Drawing No. 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson

Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

Applicable ASME Code: Section III, Edition 1974 Addenda Date W/75 Case No. N207 1361-2 Class: 1

5. MARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed.)

Sheet 1 of 1

We certify that the statements in this report are correct and that vessel, part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.

Date: 06/25/92

Signed: GE-NEBG-NF & CM-OA

(NPT Certificate Holder)

By: [Signature]
(SC or Representative)

Certificate of Authorization Expires: 5/16/93 Certification of Authorization No. NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A5253 Rev. 1

Design specification certified by Bjorn Naaberg Prof. Eng. State Calif. Reg. No. 15670

DC22A5254 Rev. 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor, State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/7/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of life, limb or time connected with this inspection.

[Signature]
Date: 5/7/92

[Signature]
Inspector

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 of this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "REMARKS".

A4002
04/23/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A4002 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

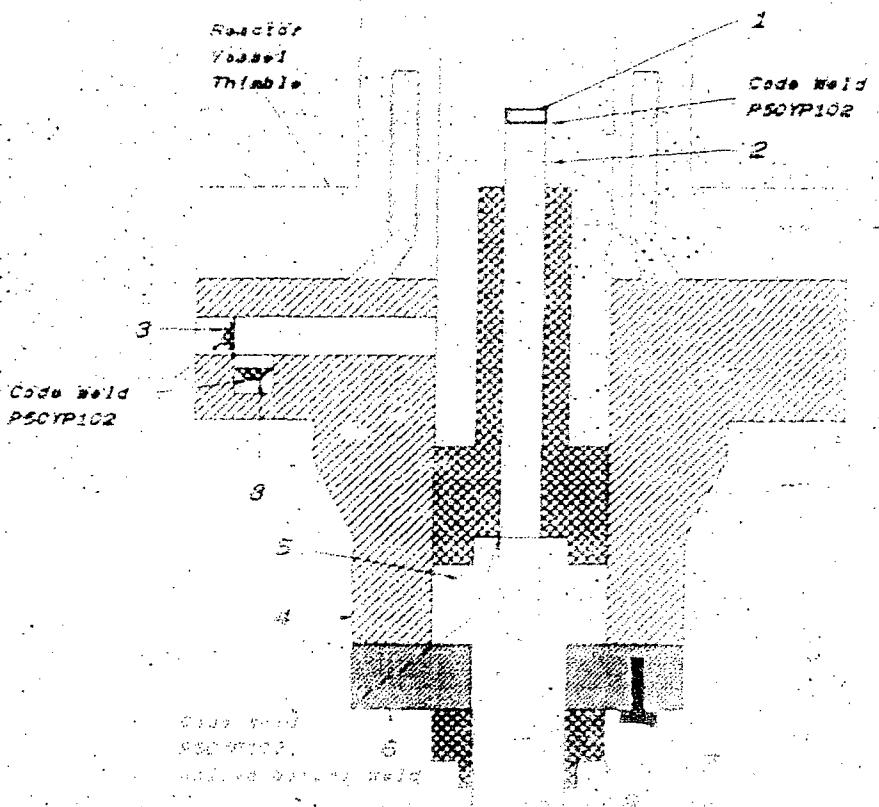
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974 Addenda Date W'75, Case No. N207.1361-2 Class. I

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 156B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B5
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 125A7581P001
SA182 - F304
3/8" thick x 1.257" dia.
9. Nut 137C5314P001
SA193 - B5
1.30" thick x 2.87" dia.



RD-176334
006022 3501

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part : A4155 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III ; Edition 1974 , Addenda Date W7F , Case No. N207 1361-2 Class 1

REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 03/13/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (GC Of Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 3/26, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

3/13, 1992 [Signature] NB 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province, and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

OT 2/23/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A4155 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

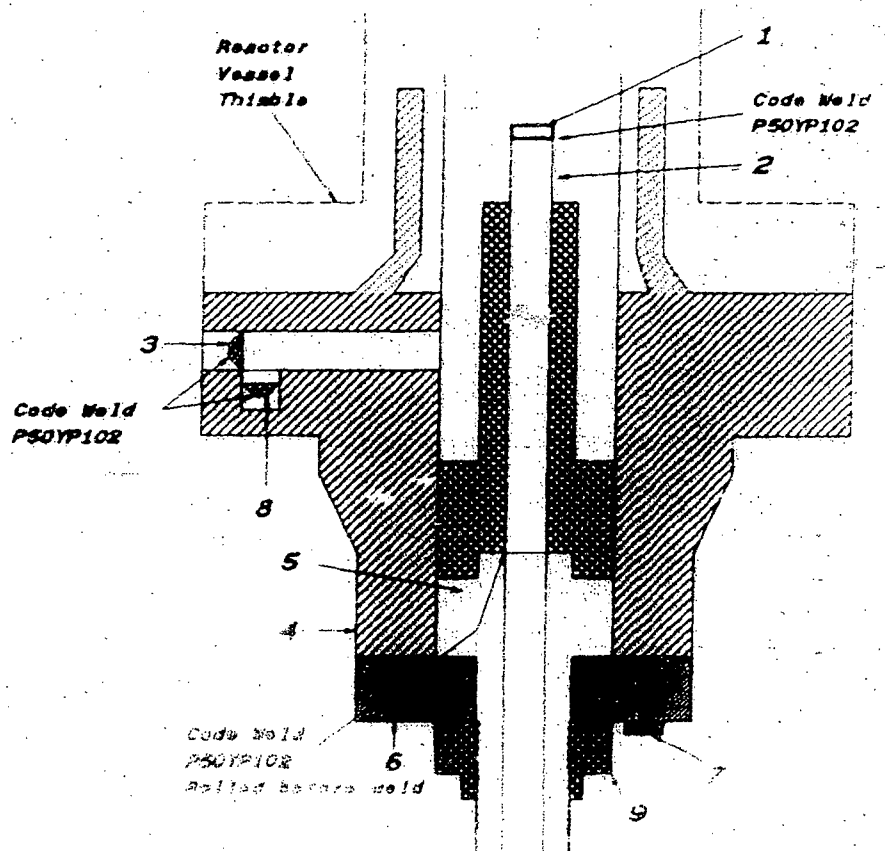
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
KM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334

G06022-1068

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for: IVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part: A4227 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 06/25/92 Signed: GE - NEBG - NF & CM - QA (NPT Certificate Holder) By: [Signature] (SC 22 Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPTN-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/27/1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or loss of any kind arising out of or connected with this inspection.

Date: 6/25/1992 Inspector's Signature: [Signature] National Board, State, Province, and C. NC 1221 Ohio, WC 3656 PA

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in 1-3 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 1. "REMARKS".

2/23/92

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE'NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part A4227 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534-G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class I

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief Description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.

3. Plug 15SA1176P001
 SA182 - F304
 1/4" thick x 0.812" OD

4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD

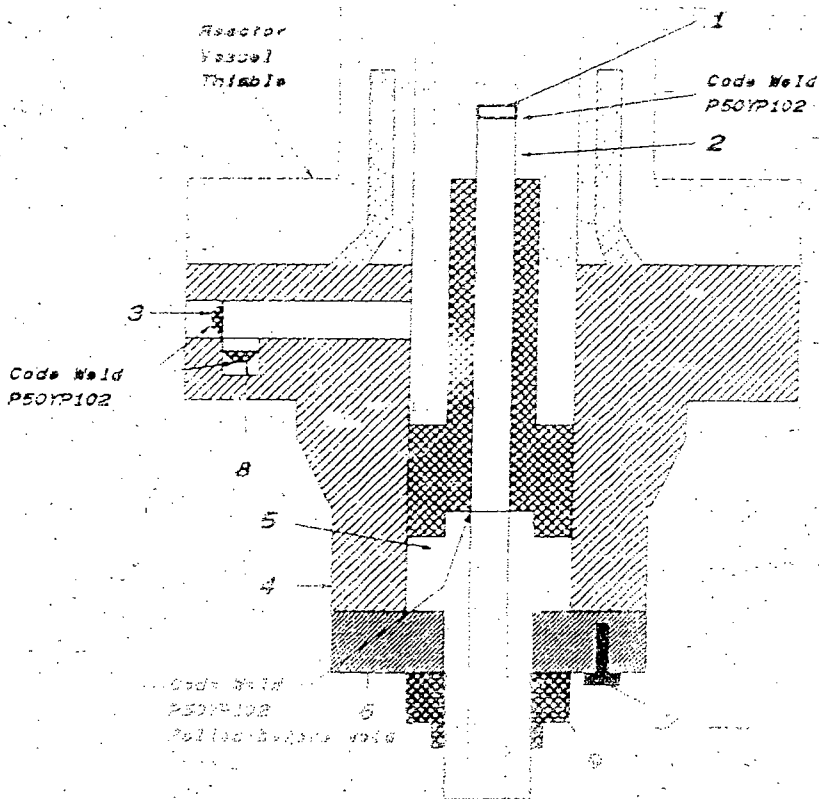
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.

6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
 SA193 - B6
 5 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.

9. Base 137C5934P001
 SA182 - F304
 1.10" thick x 2.62" dia.



RD-176334
G-06022 4034

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4325 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class I
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of services for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certificate Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 03/26/92 Signed GE-NEBG-NF & CM-QA By [Signature]
(NPT Certificate Holder) (QC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 3/18, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

3/26, 1992 [Signature] NC 1231 Ohio, WC 3536 PA
Date Inspector's Signature National Board State, Province, etc.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS".

04-2123/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28407
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A4325 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

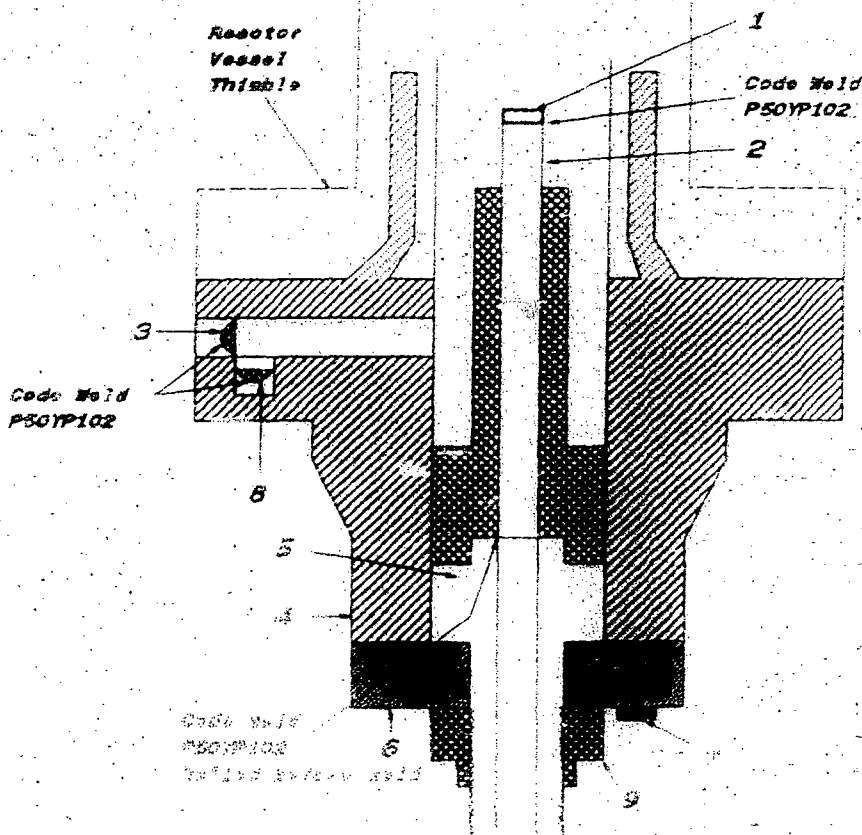
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 156B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9373P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D510P001 (719E474)
SA182 - F304
3.37" thick x 9.5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 177C4518P002
SA193 - B6
6 ea. 1/2" dia. on 1 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Plug 137C5634P001
XM - 19 SA479
1.20" thick x 2.52" dia.



RD-176334

G06022

4509

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured/Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A4445 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 9/14/92

Signed GE - NEBG - NF & CM - QA
(NPT Certificate Holder)

By [Signature]
(SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1

Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/12/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/14/92
Date

[Signature]
Inspector's Signature

NBC 201 Ohio, Inc. 1000
National Board, State, Province, etc.

*Supplemental sheets in form of lists, sketches or drawings are, if used, provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "REMARKS".

of 423/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part : A4445 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 156B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD

2. Indicator Tube 165B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.

3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD

4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD

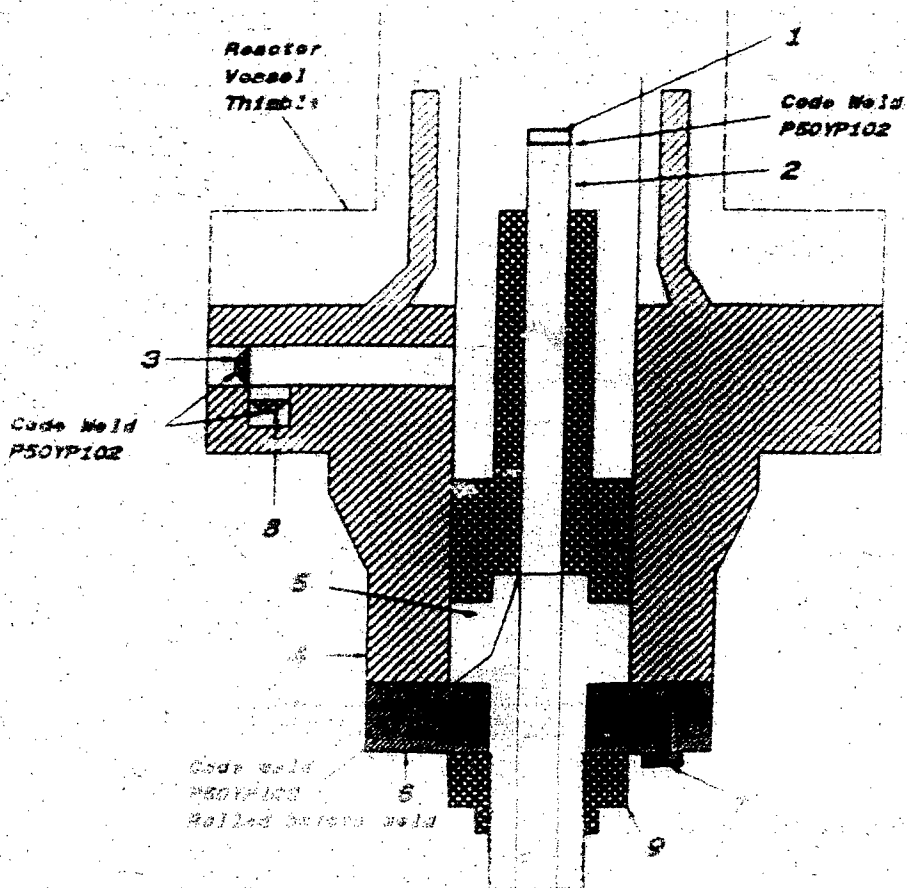
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.

6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4116P002
 SA193 - B6
 3 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.312" dia.

9. Nut 137C5934P001
 XM-19 SA479
 1.00" thick x 2.62" dia.



RD-176334

G-06022

1256

FORM NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A4756 Nat'l Bd. No. N/A

Constructed According to Drawing No: 768E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson

(c) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.

Date: 06/25/92

Signed: GE - NEBG - NF & CM - QA

(NPT Certificate Holder)

(SCQA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. NPT N - 1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1

Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1

Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/2 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer takes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

6/25 1992

James P. ...

NO 1221 Ohio WC 2588 PA

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in Part of Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3 of "REMARKS".

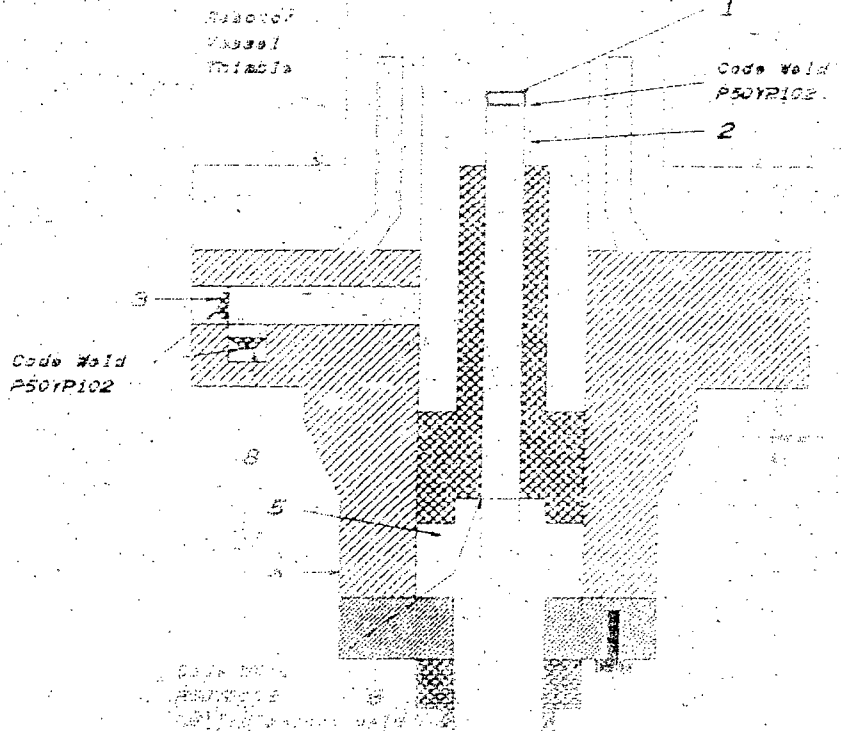
A4756
A-2/23/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part: A4756 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class 1
3. REMARKS Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
1/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/8" dia. or 1/8" bolt size
8. Plug 173A7951P001
SA182 - F304
3/8" thick x 1.062" dia.
9. Cap 137C5311P001
SA182 - F304
1/8" thick x 2.875" dia.



RD-176334
G06022 3061

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4833 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No: N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report, if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 07/29/92 Signed: GE-NEBG-NF & CM-QA By: [Signature]
(NPT Certificate Holder) (ASME Code Representative)

Certificate of Authorization Expires: 5/16/93 Certification of Authorization No. NPT-N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Biorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor or State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/29/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/29/92 [Signature] NC 1231 Ohio, WC 3685 PA
Date Inspector's Signature National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "REMARKS".

07212307

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE-NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2 Identification - Certificate Holder's S/N of Part: A4833 Nat'l Bd. No. N/A

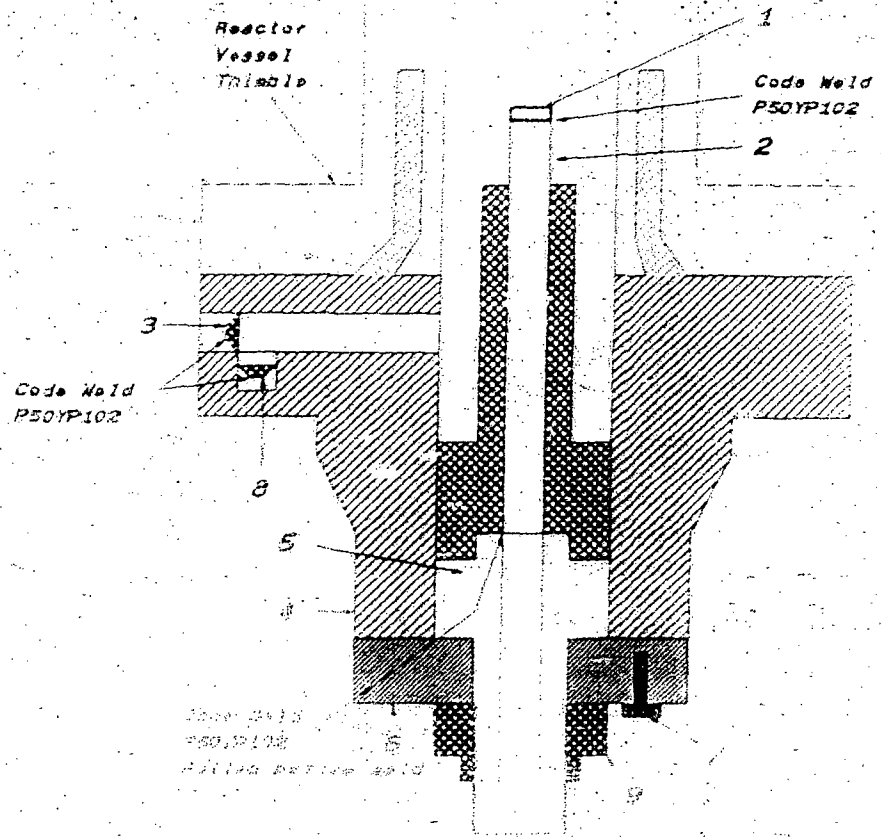
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class I

3 REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1525 psi min.
 (Brief description of service for which component was designed)

1. Cap 158B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B5
Dia. 1/2" dia. on 1 1/8" bolt circle
8. Plug 17SA7981P001
SA182 - F304
0.38" thick x 1.102" dia.
9. Nut 117C3531P001
SA193 - B5
1.30" thick x 2.52" dia.



RD-176334
G06022 675

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. 1

- 1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (a) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of NPT Certificate Holder for completed nuclear component)
- Identification - Certificate Holder's S/N of Part: A5201 Part Bd. No.: N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev. 9 Eng. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Allowable Data W75, Case No. N207 1351-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and that the part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. The Design, Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certificate holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.

Date: 04/24/92 Signed GE - NEFG - NF & CM - CA (NPT Certificate Holder) [Signature] (QC Representative)

Certificate of Authorization Expires: 6/16/93 Certificate of Authorization No. NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State CA, Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State CA, Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 4/10/1992 and state that to the best of my knowledge and belief, the NPT Certificate holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer, expressed or implied, shall be liable in any manner for any accidents or injuries or property damage, past or future, of any kind or kind from or connected with this inspection.

4/10/1992 [Signature] 100 West Ohio, P.O. Box 3836 PA
State PA Reg. No. [blank]

Supplemental sheets to this report, if needed, or reworking may be used provided (1) size is 8 1/2" x 11" (2) this report is included on each sheet, and (3) work order is numbered and number of sheets is recorded.

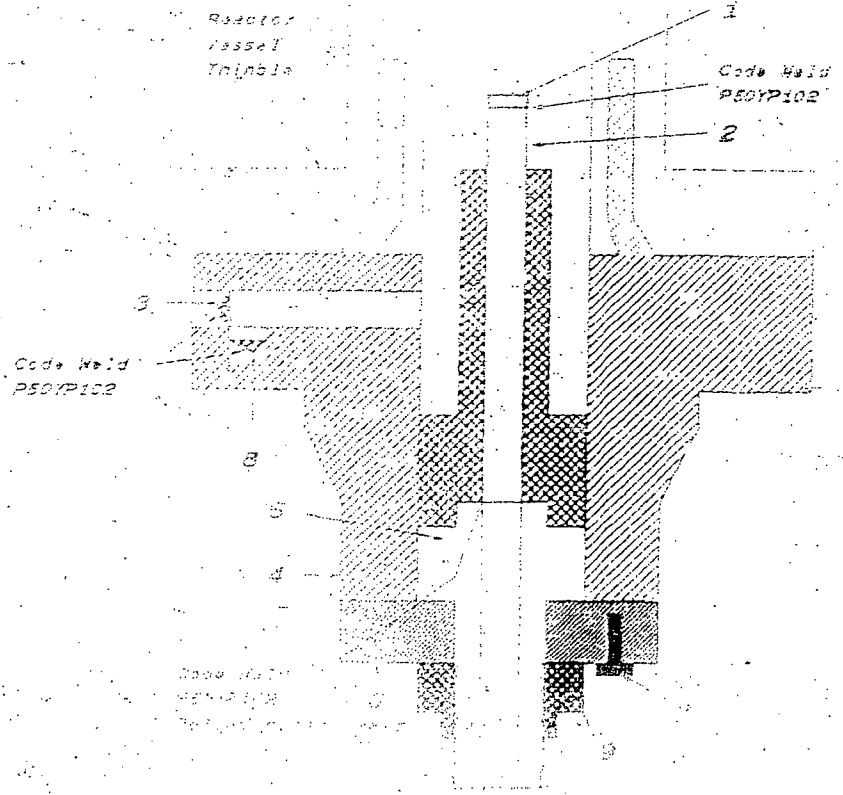
[Signature] 2/2/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (a) Manufactured for: TVA Chattanooga, Tennessee 37402-8127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part: A5231 Nat'l Bd. No.: N/A
- (a) Constructed According to Drawing No.: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Appendix Code W75 Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min
 (Brief description of service for which component was designed)

Sheet 2 of 3

1. Cap 165B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166BS313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E174)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B7
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 172A7561P001
SA182 - F304
1/8" thick x 1.312" dia.
9. Wt. 137C0505P001
SA 304 S187
1/8" thick x 1.312" dia.



RD-176334
G06022 2692

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
- Identification - Certificate Holder's S/N of Part: A5406 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W7, Case No. N207 1361-2 Class 1
2. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 06/12/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

UC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/23 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

5/12 1992 [Signature] NC 1231 Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS"

PA 2/23/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b). Manufactured for : TVA Chattanooga, Tennessee 37402-2127

(Name and Address of R Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/M of Part : A5406 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.

3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD

4. Flange 919CC10P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD

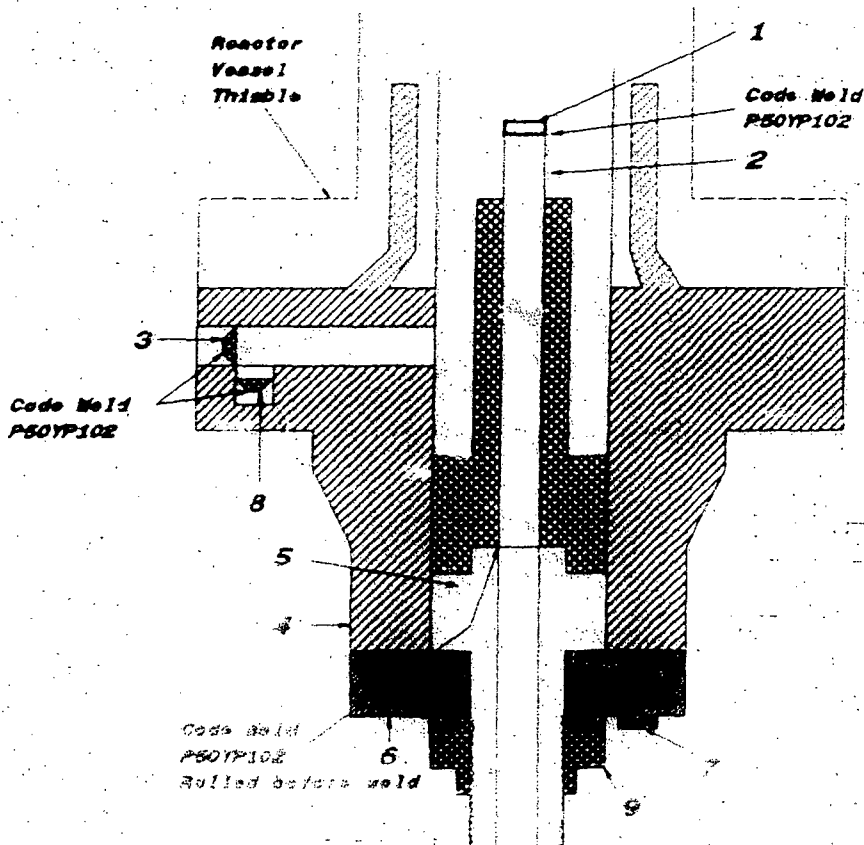
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.

6. Ring-Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.

9. Nut 137C5924P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334
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FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
- Identification - Certificate Holder's S/N of Part: A5418 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed.)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 06/12/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 4/28, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

6/12 1992 James P. Farris NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province and No.

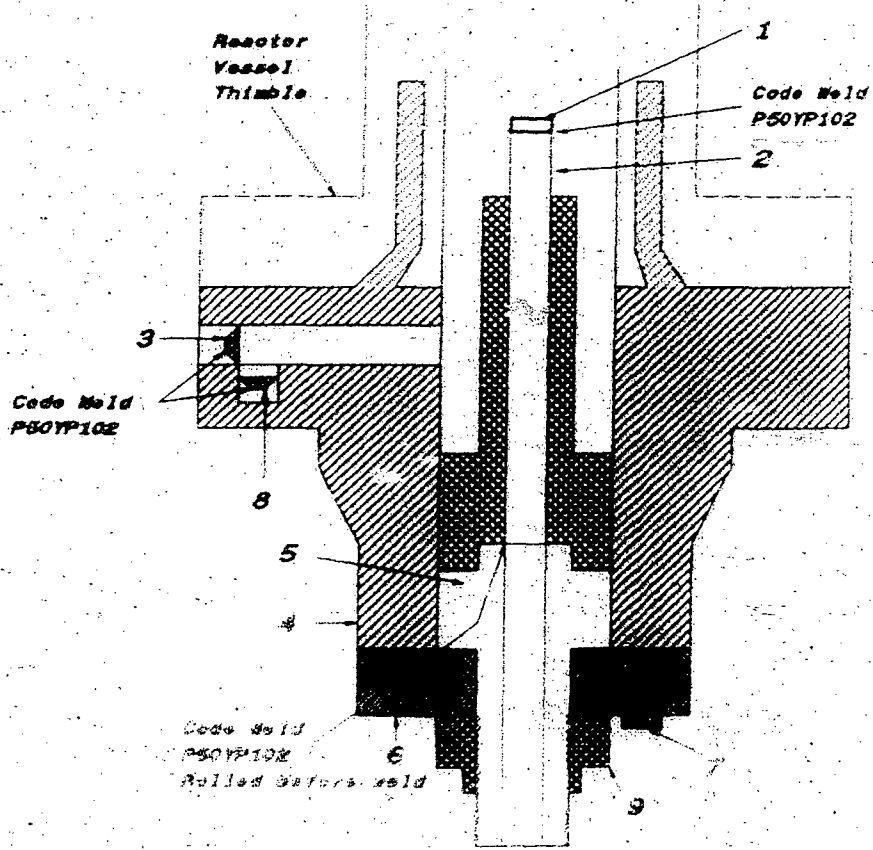
*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(17/90)
2/23/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5418 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by: D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD.
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - F15
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334
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1410

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TV4 Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A5433 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Cwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class I

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and on a vessel, part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. The applicable Design Load, Location and Stress Report are not the responsibility of the NPT Certificate holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.

Date: 06/25/92 Signed: GE-NEBG-NF & CM-DA by [Signature]
(NPT Certificate Holder) (SC or Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. NPT N-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. M018846

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/11/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages of a loss of any kind insofar as it is connected with this inspection.

6/25/92 [Signature] Wilmington, North Carolina
(Inspector) (Inspector's Name, State, Province, etc.)

Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this data report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "REMARKS".

127, 001
[Signature]
2/23/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div.

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A5433 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.

3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD

4. Flange 919U610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD

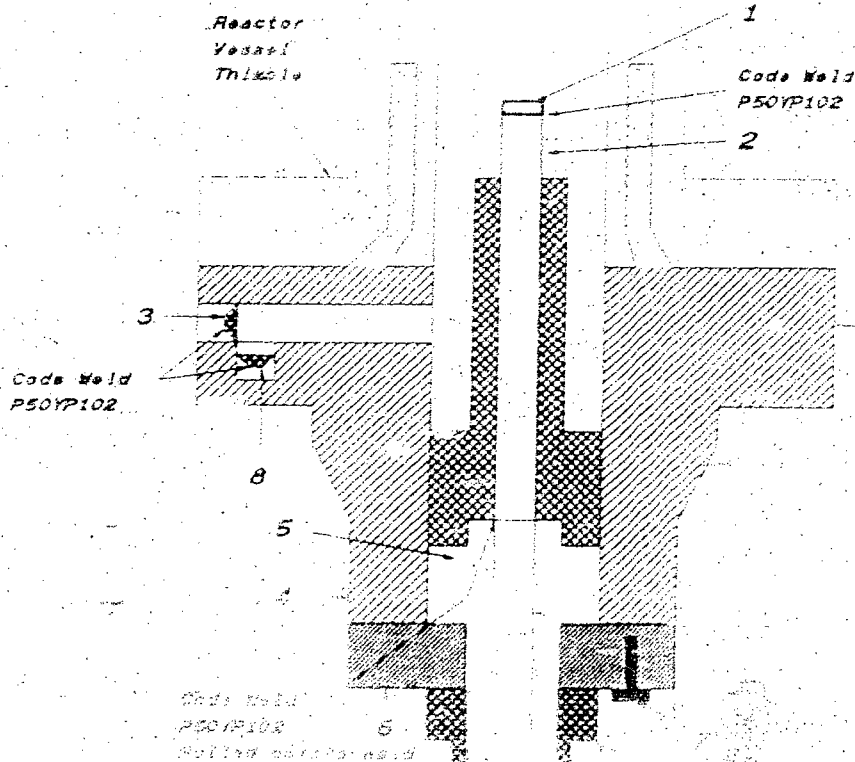
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.

6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7921P001
SA182 - F304
0.38" thick x 1.307" dia.

9. Plug 137C5311P001
AM - 15 SA-179
1.70" thick x 2.62" dia.



RD-176334

G-06022

3040

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NFR Co.)
2114 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
2. Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of NPT Certificate Holder for completed nuclear component)
3. Identification - Certificate Holder's S/N of Part: A5437 Nat'l. Id. No. N/A
4. (a) Constructed According to Drawing No: 768E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N2Q7 1361-2 Class. 1
5. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenance is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 07/29/92 Signed GE - NEBG - NF & CM - CA By [Signature]
(NPT Certificate Holder) (SC OR Representative)

Certificate of Authorization Expires: 5/10/93 Certification of Authorization No.: NPT N - 1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by: Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by: Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/23, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 7-23-92 Inspector's Signature: [Signature] National Board, State, Province And No.: NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS".

(07/90)

07
2/23/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A5437 Nat'l Bd. No. N/A

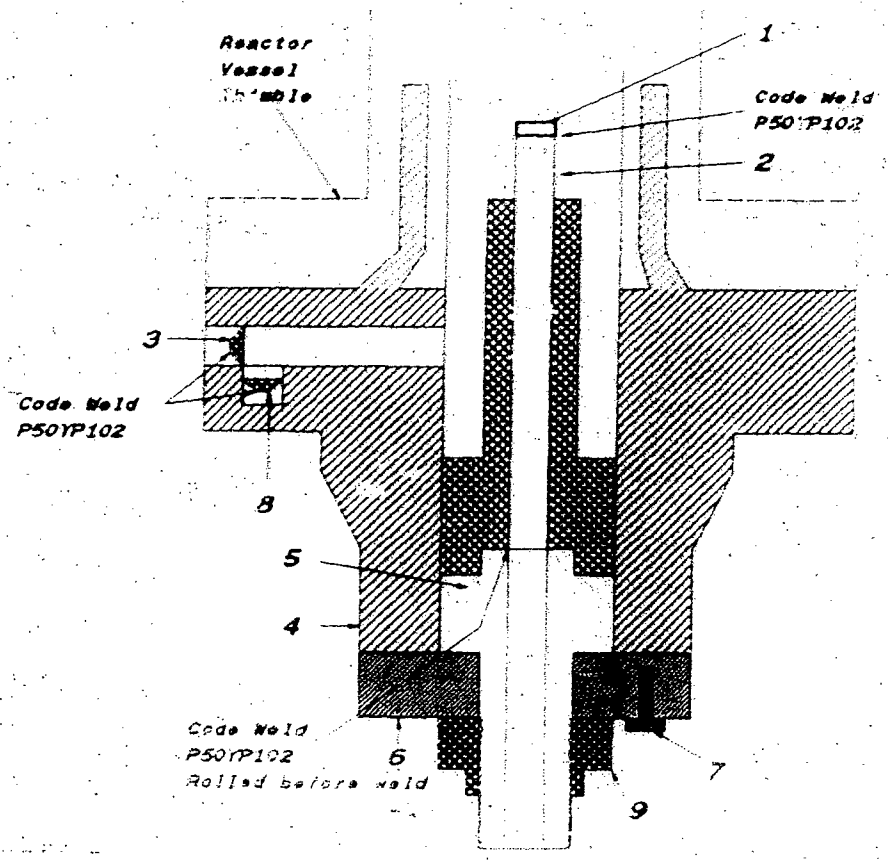
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Stands part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334
G-06022 2494

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of NPT Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part: A5573 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 08/12/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/28, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

6/12, 1992 [Signature]
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

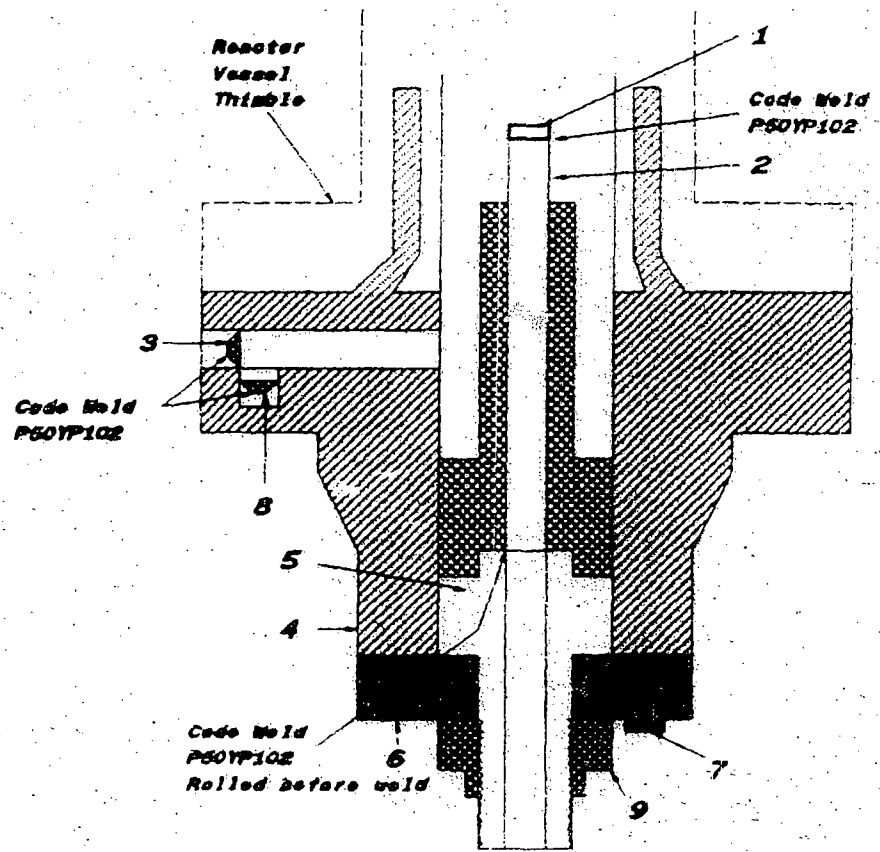
2/23/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : A5573 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176334
606022 4684

FORM N-7 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of NPT Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part: A5576 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the Code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 9/14/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No.: NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/11/1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/14/92 Inspector's Signature: [Signature] National Board, State, Province And No.: NC 1231, Ohio, WC 3686 FA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/80) 2/23/07

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification: Certificate Holder's S/N of Part: A5576 Nat'l Bd. No. N/A

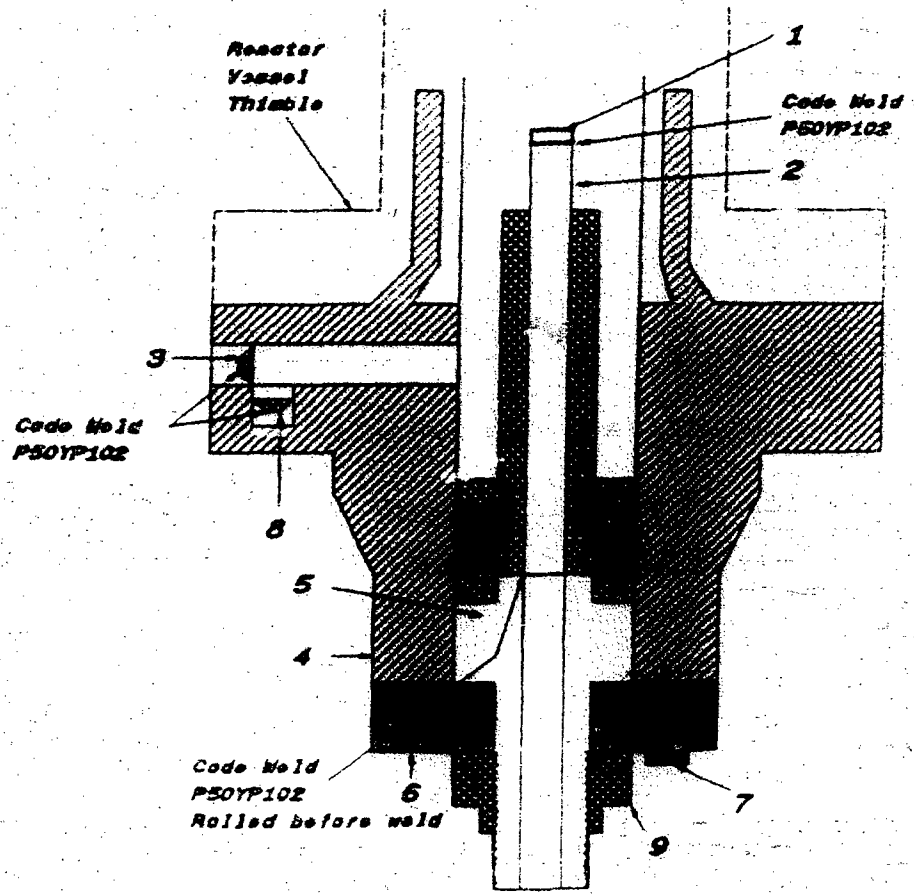
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - stainless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7861P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



RD-176324 4421
G06022

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part: A5604 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 9/14/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/17/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/14/92 [Signature]
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/00)
07/23/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A2151 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

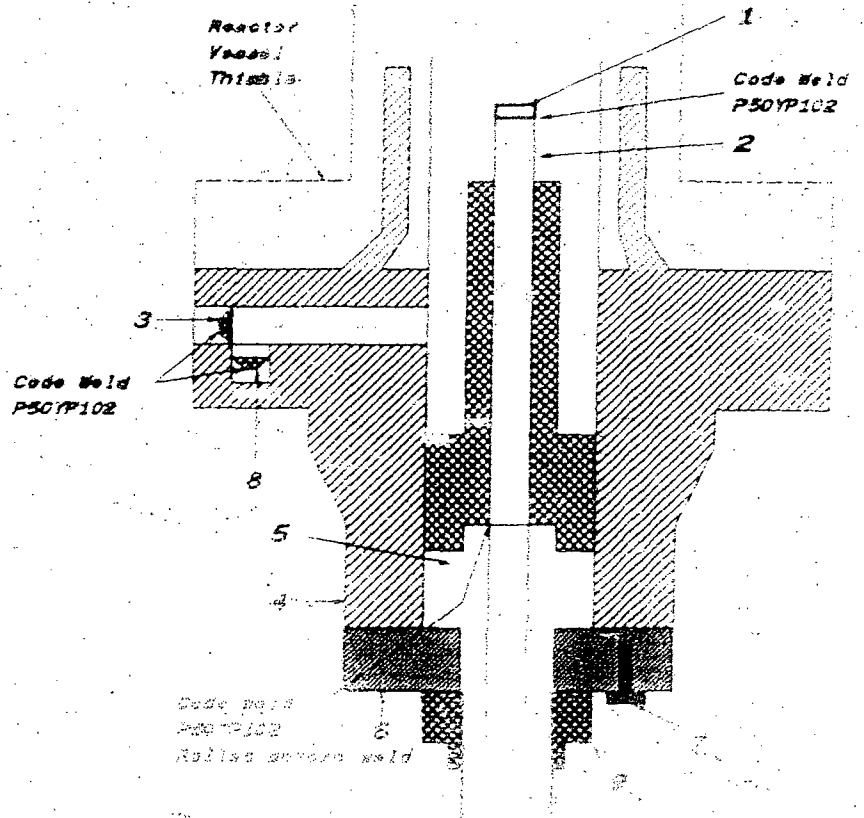
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
5 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7951P001
SA182 - F304
3/8" thick x 1.307" dia.
9. Nut 137D5934P001
XM - 19 SA479
1.00" thick x 2.52" dia.



RD-176334
G-06022 4223

FORM N NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5678 Nat'l Bd. No. N/A
Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (c) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 9/14/92 Signed GE - NEBG - NF & CM - QA By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/14/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/14/92 Inspector's Signature: [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

[Signature] (87/90)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127

(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/W of Part : A5678 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.063" max. dia.

3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD

4. Flange 19D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD

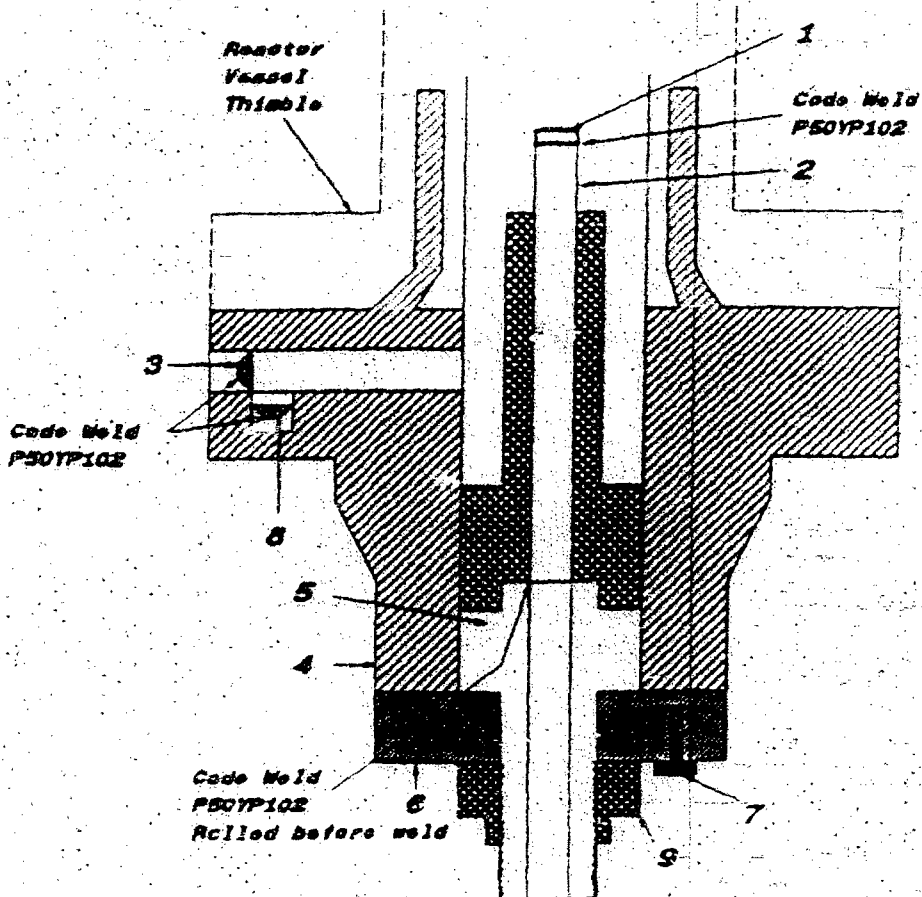
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.

6. Ring Flange 114B5122P002, P003
 137C815" P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.

9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.



RD-176334
G06022

1631

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

Identification - Certificate Holder's S/N of Part: A5709 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 06/25/92 Signed GE-NEBG-NF & CM-QA By [Signature]
(NPT Certificate Holder) (SC OR Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. NPT N-1151

Certification of Design for Appurtenance

Design information on file at: GE Company, San Jose, California

Stress analysis report on file at: GE Company, San Jose, California

DC22A6253 Rev. 1

Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev. 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/11/92 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

6/25 1992 [Signature]
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

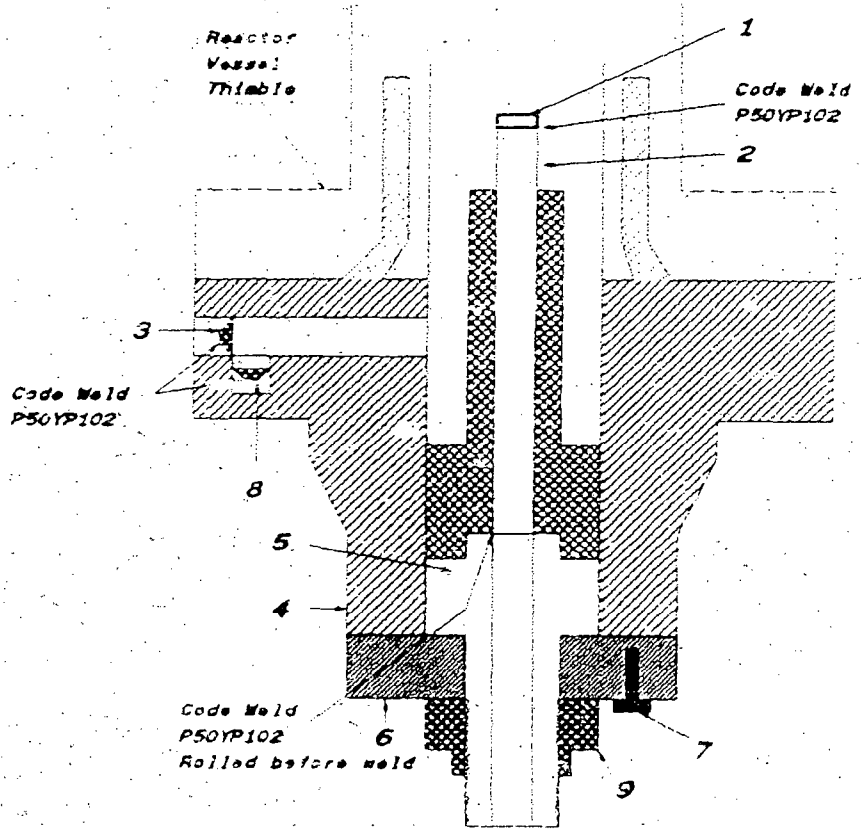
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12/3/97

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 AS required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5709 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 766E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114E5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
8 ea. 1/2" dia. on 4.1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM N-2 CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Division I

1. Manufactured and certified by General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and address of NPT Certificate Holder)
- (b) Manufactured for BROWNS FERRY INTERSECTION SHAW & NUCLEAR PLANT ATHENS, ALA 35611
 (Name and address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part A6845 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No. 919D258G003, Rev. 19 Drawing Prepared by P. RODRIGUEZ
- (b) Description of Part Inspected CYLINDER-TUBE AND FLANGE
- (c) Applicable ASME Code: Section III, Edition 1974 Addenda Date W' 75 Case No. 1361-2 Class 1
3. Remarks: STANDARD PART FOR USE WITH REACTOR. HYDROSTATICALLY TESTED AT 1820 PSI. MIN.
 (Brief description of service for which component was designed)

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenances is not included in the component Design Specification and Stress Report).

Date 02/15/2007 Signed GE-NE By *M. D. Johnson*
 (NPT Certificate Holder) (SCQ Representative)

Certificate of Authorization Expires 06/16/2008 Certificate of Authorization No. NPT N-1151

CERTIFICATION OF DESIGN FOR APPURTENANCE

Design information on file at GE COMPANY, SAN JOSE, CALIFORNIA
 Stress analysis report on file at GE COMPANY, SAN JOSE, CALIFORNIA
 DC22A6253, Rev. 02
 Design specification certified by B.N. SRIDHAR Prof. Eng. State CALIF. Reg. No. 18345
 DC22A6254, Rev. 01
 Stress analysis report certified by E.Y. GIBO Prof. Eng. State CALIF. Reg. No. M18646

CERTIFICATION OF SHOP INSPECTION

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 2/15, 2007 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/15 2007 *James E. ...* NB8214-IBNA NC1231
 Date Inspector's Signature National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8 1/2" x 11", (2) information in 1-2 on the Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3 "Remarks".

08
2/23/07

FORM N-2 CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Division I

1. Manufactured and certified by: General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and address of NPT Certificate Holder)
- (b) Manufactured for BROWNS FERRY INTERSECTION SHAW & NUCLEAR PLANT ATHENS, ALA 35611
 (Name and address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part A6873 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No. 919D258G003, Rev. 19 Drawing Prepared by P. RODRIGUEZ
- (b) Description of Part Inspected CYLINDER-TUBE AND FLANGE
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W' 75, Case No. 1361-2, Class I
3. Remarks: STANDARD PART FOR USE WITH REACTOR. HYDROSTATICALLY TESTED AT 1820 PSI. MIN.
 (Brief description of service for which component was designed)

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenances is not included in the component Design Specification and Stress Report).

Date 02/15/2007 Signed GE-NE By *M. E. Johnson*
 (NPT Certificate Holder) (SCQ Representative)

Certificate of Authorization Expires 06/16/2008 Certificate of Authorization No. NPT N-1151

CERTIFICATION OF DESIGN FOR APPURTENANCE

Design information on file at GE COMPANY, SAN JOSE, CALIFORNIA
 Stress analysis report on file at GE COMPANY, SAN JOSE, CALIFORNIA

DC22A6253, Rev. 02
 Design specification certified by B.N. SRIDHAR Prof. Eng. State CALIF. Reg. No. 18345

DC22A6254, Rev. 01
 Stress analysis report certified by E.Y. GIBO Prof. Eng. State CALIF. Reg. No. M18646

CERTIFICATION OF SHOP INSPECTION

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 2/15, 2007 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/15 2007 *James E. ...* NB8214 IBNA NC1231
 Date Inspector's Signature National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8 1/2 " x 11", (2) information in 1-2 on the Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3 "Remarks".

Handwritten initials and date: 2/23/07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 15, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Orders (WOs) 06-716045-000 and 06-716045-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)
and System 006, Heater Drains and Vents System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
SJAE drain piping	unknown	N/A	N/A	N/A	N/A	Removed	No
SJAE drain piping	Consolidated Power Supply	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Replaced Steam Jet Air Ejector(SJAE) drain piping

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
** - Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WOs) 06-716045-000 and 06-716045-001

9. Remarks Replaced Steam Jet Air Ejector(SJAE) drain piping
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Wilford* System Engineer Date 6-15 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee or Province of Connecticut and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 2/5/07 to 6/19/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sanford Commissions TN4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/19/07 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 15, 2007

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Sheet 1 of 1

Unit 2

Design Change Notice (DCN) 67470A & 68966A
 Work Order (WO) 06-714856-000
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code (valve) ASME Section III, Class 2, 1989 Edition, less "N" stamp
(piping) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Steam to SJA E A Stages 1 & 2 Isol Valve	Anchor Darling	N/A	N/A	2-FCV-001-0155		Removed	No
Steam to SJA E A Stages 1 & 2 Isol Valve	Flowserve Corp	EZ983-1-1	N/A	2-FCV-001-0155		Installed	No
pipe	unknown	N/A	N/A	N/A	N/A	Removed	No
pipe	United States Steel	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Replaced valve and a section of pipe.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
** - Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 67470A & 68966A
Work Order (WO) 06-714856-000

9. Remarks Replaced valve and a section of pipe.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] System Engineer
Owner or Owner's Designer Title

Date 6-15 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 4/2/07 to 6/22/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions TN 4011
National Board, State, Province, and Endorsements

Date 6/22 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 15, 2007

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Sheet 1 of 1

Unit 2

Work Orders (WOs) 07-712169-003
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam piping	unknown	N/A	N/A	N/A	N/A	Removed	No
Main Steam piping	United States Steel	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Replaced a section of Main Steam piping which is an open discharge to the condenser.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt **
** - open ended discharge piping
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WOs) 07-712169-003

9. Remarks Replaced a section of Main Steam piping which is an open discharge to the condenser.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Wilford* System Engineer Date 6-15 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 3/30/07 to 6/20/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Samuel Flood Commissions TN401
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/20 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 15, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) 64390A Stage 2
 Work Order (WO) 05-717805-000 and 05-717805-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam to Off Gas Preheater Check Valve	Flowserve	31BBM	N/A	2-CKV-001-0742	2005	Removed	No
Main Steam to Off Gas Preheater Check Valve	Flowserve	32BBM	N/A	2-CKV-001-0744	2005	Removed	No
Main Steam to Off Gas Preheater Isol Valve	Flowserve	E969A-1-3	N/A	2-SHV-001-0741	2000	Removed	No
Main Steam to Off Gas Preheater Isol Valve	Flowserve	E969A-1-4	N/A	2-SHV-001-0743	2000	Removed	No
piping	unknown	N/A	N/A	N/A	N/A	Removed	No
piping	Consolidated Power	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Removed four 2" valves and 2" piping and replaced them with 3/4" components.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
** - Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 64390A Stage 2
Work Order (WO) 05-717805-000 and 05-717805-001

9. Remarks Removed four 2" valves and 2" piping and replaced them with 3/4" components.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Stephen C. Williams System Engineer
Owner or Owner's Designee, Title

Date 6-15 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 11/8/06 to 6/21/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sam Stone
Inspector's Signature

Commissions TN4011
National Board, State, Province, and Endorsements

Date 6/21 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Date June 16, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) 65786
 Work Order (WO) 05-724223-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 075, Core Spray System. (ASME Code Class 2 equivalent)
System 100, Primary Containment Penetrations (ASME Code Class MC equivalent)

5. (a) Applicable Construction Code ASME Section III, Class B 19 65 Edition, N/A Addend N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Stiffener ring	TVA	N/A	N/A	N/A	2007	Installed	No

7. Description of Work Added a stiffener ring on one of the Core Spray suction branch connections from the primary containment torus ring header.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt

Other ** Pressure N/A psi Test Temp. N/A °F

** - Deferred Appendix J testing. Reference WO 06-722457-000.

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 65786,
Work Order (WO) 05-724223-001

9. Remarks Added a stiffener ring on one of the Core Spray suction branch connections from the primary containment torus ring header.

Applicable Manufacturer's Data Reports to be attached

This is a modification for Extended Power Uprate.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] System Engineer
Owner or Owner's Designee, Title

Date 6-16 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 11/16/06 to 6/21/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions TN4011
National Board, State, Province, and Endorsements

Date 6/21 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 16, 2007
Name
1101 Market Street
Address
Chattanooga, TN: 37402-2801
Address

2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Orders (WOs) 06-718765-006 and 06-718735-006
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 068, Reactor Recirculation System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code ASME Section III 19 70 Edition, N/A Addenda, N/A Code Case.
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Recirc Pump 2A	Byron Jackson	671-S-1023	N/A	2-PMP-068-0060A	1970	‡	Yes
‡ - Replaced pump cover and nuts							
pump cover	Byron Jackson	unknown	N/A	2-PMP-068-0060A	1970	Removed	No
pump cover	Flowserve	RLSA05841	N/A	2-PMP-068-0060A	2004	Installed	No
nuts	Byron Jackson	N/A	N/A	2-PMP-068-0060A	N/A	Removed	No
nuts	NOVA	N/A	N/A	2-PMP-068-0060A	N/A	Installed	No

7. Description of Work Replaced pump cover with a new design from the pump vendor. Replaced pump cover nuts.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in PEG Package 050517-CHF086GG0, GE P.O. 205-H0297 and Design Criteria BFN-50-7068 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WOs) 06-718765-006 and 06-718735-006

9. Remarks Replaced pump cover with a new design from the pump vendor. Replaced pump cover nuts.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Wilford* System Engineer Date 6-28 20 07
Owner or Owner's Designee Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 1/23/07 to 6/30/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Lawrence Howard Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/30 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

- Owner Tennessee Valley Authority (TVA) Date June 16, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
3. Work Performed by TVA-BFN Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
4. Identification of System System 068 Reactor Recirculation System (ASME Code Class 1 equivalent)
ASME Code Class 1 equivalent
5. (a) Applicable Construction Code ASME Section III 19 70 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Recirc Pump 2B	Byron Jackson	671-S-1024	N/A	2-PMP-068-0060B	1970	‡	Yes
‡ - Replaced pump cover and nuts							
pump cover	Byron Jackson	unknown	N/A	2-PMP-068-0060B	1970	Removed	No
pump cover	Flowsolve	RLSA05842	N/A	2-PMP-068-0060B	2004	Installed	No
nuts	Byron Jackson	N/A	N/A	2-PMP-068-0060B	N/A	Removed	No
nuts	NOVA	N/A	N/A	2-PMP-068-0060B	N/A	Installed	No

7. Description of Work Replaced pump cover with a new design from the pump vendor. Replaced pump cover nuts.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure: Exempt
 Other Pressure N/A psi Test Temp. N/A °F

**FORM N-2 CERTIFICATE HOLDERS' DATA REPORT FOR IDENTICAL
NUCLEAR PARTS AND APPURTENANCES***

As Required by the Provisions of the ASME Code, Section III
Not to Exceed One Day's Production

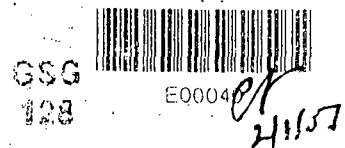
1. Manufactured and certified by FLOWERVE CORP., PUMP DIVISION, 2300 E. VERNON AVE., VERNON, CA 90058
(name and address of NPT Certificate Holder)
2. Manufactured for TENNESSEE VALLEY AUTHORITY, P.O. BOX 15500, KNOXVILLE TN 37901-550
(name and address of purchaser)
3. Location of installation BROWNS FERRY UNIT 2, NUCLEAR STORES INTERSECTION SHAW & NUCLEAR PLANT
ATHENS, AL 35611
(name and address)
4. Type DWG L002086 REV. B SEE REMARKS SEE REMARKS N/A 2004
(drawing no.) (mat'l. spec. no.) (tensile strength) (CRN) (year built)
5. ASME Code, Section III, Division 1: 1989 No Addenda 1 N/A
(edition) (addenda date) (class) (Code Case no.)
6. Fabricated in accordance with Const. Spec. (Div. 2 only) N/A Revision N/A Date N/A
(no.)
7. Remarks: PART NOMENCLATURE: 4th GENERATION DESIGN, NAMEPLATE ATTACHED WITH DRIVE PINS
MATERIAL SPECIFICATION NO /TENSILE STRENGTH:
ASME SA 351 GR.CF8M/70,000PSI, ASME SA 540 GR.B23 CL.4/120,000PSI, ASME SA 182 GR.F3 16/75,000PSI
8. Nom. thickness (in.) 2.020" Min. design thickness (in.) 0.940" Dia. ID (ft & in.) 1'-1.970" Length overall (ft & in.) 1'-7.430"
9. When applicable, Certificate Holders' Data Reports are attached for each item of this report:

Part or Appurtenance Serial Number	National Board No. in Numerical Order
(1) <u>RLSA06385</u>	<u>N/A</u>
(2)	
(3)	
(4)	
(5)	
(6)	
(7)	
(8)	
(9)	
(10)	
(11)	
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(16)	
(17)	
(18)	
(19)	
(20)	
(21)	
(22)	
(23)	
(24)	
(25)	

Part or Appurtenance Serial Number	National Board No. in Numerical Order
(26)	
(27)	
(28)	
(29)	
(30)	
(31)	
(32)	
(33)	
(34)	
(35)	
(36)	
(37)	
(38)	
(39)	
(40)	
(41)	
(42)	
(43)	
(44)	
(45)	
(46)	
(47)	
(48)	
(49)	
(50)	

ign pressure 1500 psi. Temp. 575 °F. Hydro. test pressure 1878-1950PSI@50-90 at temp. °F
(when applicable)

* Supplemental information in the form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 x 11, (2) information in items 2 and 3 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



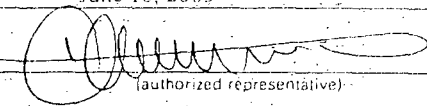
Certificate Holder's Serial Nos. RLSA06385 through RLSA06385

CERTIFICATION OF DESIGN

Design specifications certified by N/A P.E. State N/A Reg. no. N/A
(when applicable)
 Design report* certified by CARL F. REIMERS P.E. State CA Reg. no. M018283
(when applicable)

CERTIFICATE OF COMPLIANCE

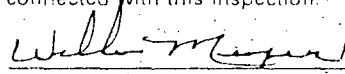
We certify that the statements made in this report are correct and that this (these) 4th Generation Design conforms to the rules of construction of the ASME Code, Section III, Division 1.

NPT Certificate of Authorization No. N-1131 Expires June 10, 2005
 Date 09/20/2004 Name Flowserve Corp., Pump Division Signed 
(NPT Certificate Holder) (authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSB-CT of Hartford, CT have inspected these items described in this Data Report on 9/20/04 and state that to the best of my knowledge and belief, the Certificate Holder has fabricated these parts or appurtenances in accordance with the ASME Code, Section III, Division 1. Each part listed has been authorized for stamping on the date shown above.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 9/20/04 Signed  Commissions CA 1494 CA-1969
(Authorized Nuclear Inspector) [Nat'l Bd. (incl. endorsements) and state or prov. and no.]

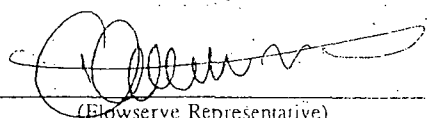
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FORM N-2 CERTIFICATE HOLDER'S DATA REPORT
SUPPLEMENTARY SHEET

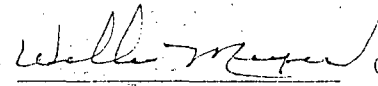
1. Manufactured and certified by FLOWSERVE CORPORATION, ROTATING EQUIPMENT DIVISION, NUCLEAR PRODUCTS DIVISION
2300 EAST VERNON AVE., VERNON, CA. 90058
(name and address of Certificate Holder)
2. Manufactured for TENESSE VALLEY AUTHORITY, P. O. BOX 15500, KNOXVILLE TN 37901-550
(name and address of Purchaser)
3. Location of Installation BROWNS FERRY UNIT 2, NUCLEAR STORES INTERSECTION SHAW & NUCLEAR PLANT,
ATHENS, AL. 35611
(name and address)
7. Remarks: FLOWSERVE JOB NO. RLCA01915 - 4TH GENERATION DESIGN Drawing: L002086 REV. B

Note: The items listed below are being supplied as individual parts.

SERIAL NO. (LOT NO.)	REF. NO.	PART NAME	PART NO.	MATERIAL
RLSA05841	2-1	COVER, PUMP	7011609	ASME SA 351 GR. CF8M
---	2-1-1	UNION ASSEMBLY	---	CONSISTING OF:
RLSA05290	---	UNION, MALE END, SOC WELD	7002211	ASME SA 182 GR. F316
RLSA05706	---	NUT, UNION	7002213	ASME SA 182 GR. F316
RLSA05658	---	UNION-SPCL FEMALE END	7011702	ASME SA 182 GR. F316
RLSA06019	8	COOLER ASSEMBLY	7012617	CONSISTING OF:
RLSA05676	8-1	HOUSING, COOLER	7011597	ASME SA 182 GR. F316
RLSA05659	8-1-1	CYLINDER, OUTER	7011595	ASME SA 182 GR. F316
RLSA05674	8-1-3	CYLINDER, ENCLOSING	7011591	ASME SA 182 GR. F316
---	8-1-7	UNION, ASSEMBLY, 0.750-3000LB	---	CONSISTING OF:
RLSA05549-7 / 8	---	UNION-SPCL FEMALE END	7012900	ASME SA 182 GR. F316
RLSA05290	---	UNION, MALE END, SOC WELD	7002211	ASME SA 182 GR. F316
RLSA05706	---	NUT, UNION .750	7002213	ASME SA 182 GR. F316
---	8-1-9	UNION, ASSEMBLY, 1.000-3000LB	---	CONSISTING OF:
RLSA05724	---	UNION-SPCL FEMALE END	7011694	ASME SA 182 GR. F316
04-006145	8-6	STUD, COOLER M36 X 201.3MM	7011613	ASME SA 540 GR. B23 CL. 4
04-005419	8-7	NUT, COOLER M36	7011614	ASME SA 194 GR. 7

DATE: 09/20/2004 NAME: FLOWSERVE CORPORATION SIGNED: 
(Certificate Holder) (Flowserve Representative)

Certificate of Authorization No. N-1131 Certificate of Authorization Expires JUNE 10, 2005

DATE: 9/20/04 SIGNED:  COMMISSIONS: CA 1494
Authorized Inspector National Board, State, Province and Number

**FORM N-2 CERTIFICATE HOLDERS' DATA REPORT FOR IDENTICAL
NUCLEAR PARTS AND APPURTENANCES***

As Required by the Provisions of the ASME Code, Section III
Not to Exceed One Day's Production

1. Manufactured and certified by FLOWSERVE CORP., PUMP DIVISION, 2300 E. VERNON AVE., VERNON, CA 90058
(name and address of NPT Certificate Holder)
2. Manufactured for TENNESSEE VALLEY AUTHORITY, P.O. BOX 15500, KNOXVILLE TN 37901-550
(name and address of purchaser)
3. Location of installation BROWNS FERRY UNIT 2, NUCLEAR STORES INTERSECTION SHAW & NUCLEAR PLANT
ATHENS, AL 35611
(name and address)
4. Type DWG L002086 REV. B SEE REMARKS SEE REMARKS N/A 2004
(drawing no.) (mat'l spec. no.) (tensile strength) (CRN) (year built)
5. ASME Code, Section III, Division 1: 1989 No Addenda 1 N/A
(edition) (addenda date) (class) (Code Case no.)
6. Fabricated in accordance with Const. Spec. (Div. 2 only) N/A Revision N/A Date N/A
(no.)
7. Remarks: PART NOMENCLATURE: 4th GENERATION DESIGN NAMEPLATE ATTACHED WITH DRIVE PINS.
MATERIAL SPECIFICATION NO / TENSILE STRENGTH:
ASME SA 351 GR. CF8M/70,000PSI, ASME SA 540 GR. B23 CL. 4/120,000PSI, ASME SA 182 GR. F316/75,000PSI
8. Nom. thickness (in.) 2.020" Min. design thickness (in.) 0.940" Dia. ID (ft & in.) 1'-1.970" Length overall (ft & in.) 1'-7.430"
9. When applicable, Certificate Holders' Data Reports are attached for each item of this report:

Part or Appurtenance Serial Number	National Board No. in Numerical Order
(1) <u>RLSA06386</u>	<u>N/A</u>
(2)	
(3)	
(4)	
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(22)	
(23)	
(24)	
(25)	

Part or Appurtenance Serial Number	National Board No. in Numerical Order
(26)	
(27)	
(28)	
(29)	
(30)	
(31)	
(32)	
(33)	
(34)	
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(45)	
(46)	
(47)	
(48)	
(49)	
(50)	

Design pressure 1500 psi. Temp. 575 °F. Hydro. test pressure 1878-1950PSI@50.90 at temp. °F
(when applicable)

* Supplemental information in the form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 x 11, (2) information in items 2 and 3 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

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2/1/07

Certificate Holder's Serial Nos. RLSA06386 through RLSA06386

CERTIFICATION OF DESIGN

Design specifications certified by N/A P.E. State N/A Reg. no. N/A
(when applicable)
Design report* certified by CARL F. REIMERS P.E. State CA Reg. no. M018283
(when applicable)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this (these) 4th Generation Design conforms to the rules of construction of the ASME Code, Section III, Division 1.

NPT Certificate of Authorization No. N-1131 Expires June 10, 2005
Date 09/24/2004 Name Flowserve Corp., Pump Division Signed [Signature]
(NPT Certificate Holder) (authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by HSB-CT of Hartford, CT have inspected these items described in this Data Report on 9-24-2004 and state that to the best of my knowledge and belief, the Certificate Holder has fabricated these parts or appurtenances in accordance with the ASME Code, Section III, Division 1. Each part listed has been authorized for stamping on the date shown above.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 9/24/04 Signed [Signature] Commissions CA 1494
(Authorized Nuclear Inspector) (Nat'l. Bd. (incl. endorsements) and state or prov. and no.)

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SERIAL NO- RLSA06386

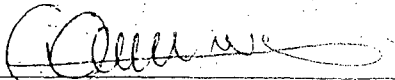
**FORM N-2 CERTIFICATE HOLDER'S DATA REPORT
SUPPLEMENTARY SHEET**

FLOWSERVE CORPORATION, ROTATING EQUIPMENT DIVISION, NUCLEAR PRODUCTS DIVISION

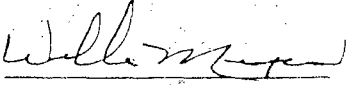
1. Manufactured and certified by 2300 EAST VERNON AVE., VERNON, CA. 90058
(name and address of Certificate Holder)
2. Manufactured for TENESSE VALLEY AUTHORITY, P.O. BOX 15500, KNOXVILLE TN 37901-550
(name and address of Purchaser)
3. Location of Installation BROWNS FERRY UNIT1, NUCLEAR STORES INTERSECTION SHAW & NUCLEAR PLANT,
ATHENS, AL. 35611
(name and address)
7. Remarks: FLOWSERVE JOB NO:- RLCA01915 - 4TH GENERATION DESIGN Drawing: L002086 REV. B.

Note: The items listed below are being supplied as individual parts.

SERIAL NO. (LOT NO.)	REF. NO.	PART NAME	PART NO.	MATERIAL
RLSA05842	2-1	COVER PUMP	7011609	ASME SA 351 GR. CF8M
---	2-1-1	UNION ASSEMBLY	---	CONSISTING OF:
RLSA05290	---	UNION, MALE END, SOC WELD	7002211	ASME SA 182 GR. F316
RLSA05706	---	NUT, UNION	7002213	ASME SA 182 GR. F316
RLSA05658	---	UNION-SPCL FEMALE END	7011702	ASME SA 182 GR. F316
RLSA06020	8	COOLER ASSEMBLY	7012617	CONSISTING OF:
RLSA05686	8-1	HOUSING, COOLER	7011597	ASME SA 182 GR. F316
RLSA05677	8-1-1	CYLINDER, OUTER	7011595	ASME SA 182 GR. F316
RLSA05656	8-1-3	CYLINDER, ENCLOSING	7011591	ASME SA 182 GR. F316
---	8-1-7	UNION, ASSEMBLY, 0.750-3000LB	---	CONSISTING OF:
RLSA05549-3 / 4	---	UNION-SPCL FEMALE END	7012900	ASME SA 182 GR. F316
RLSA05290	---	UNION, MALE END, SOC WELD	7002211	ASME SA 182 GR. F316
RLSA05706	---	NUT, UNION .750	7002213	ASME SA 182 GR. F316
---	8-1-9	UNION, ASSEMBLY, 1.000-3000LB	---	CONSISTING OF:
RLSA05724	---	UNION-SPCL FEMALE END	7011694	ASME SA 182 GR. F316
03-006657	8-6	STUD, COOLER M36 X 201.3MM	7011613	ASME SA 540 GR. B23 CL.4
03-006539	8-7	NUT, COOLER M36	7011614	ASME SA 194 GR. 7

DATE: 09/24/2004 NAME: FLOWSERVE CORPORATION SIGNED: 
(Certificate Holder) (Flowserve Representative)

Certificate of Authorization No. N-1131 Certificate of Authorization Expires JUNE 10, 2005

DATE: 9/24/04 SIGNED:  COMMISSIONS: CA424
Authorized Inspector National Board, State, Province and Number

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 16, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) 63374A
 Work Order (WO) 03-022883-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 010, Boiler Drains and Vents and Blowdown System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RPV Head Vent to CRW	Hancock 5500	XMY4? †	N/A	2-VTV-010-0500	N/A	Removed	No
RPV Head Vent to CRW	Flowserve 1878	85BBK	N/A	2-VTV-010-0500-	2005	Installed	No
RPV Head Vent to CRW	Hancock 5500	XMY4? †	N/A	2-VTV-010-0501	N/A	Removed	No
RPV Head Vent to CRW	Flowserve 1878	86BBK	N/A	2-VTV-010-0501	2005	Installed	No
RPV Head Vent	Hancock 5500	XMY32	N/A	2-VTV-010-0517	N/A	Removed	No
RPV Head Vent	Flowserve 1878	84BBK	N/A	2-VTV-010-0517	2005	Installed	No
pipe	unknown	N/A	N/A	N/A	N/A	Removed	No
pipe	United States Steel	N/A	N/A	N/A	N/A	Installed	No

† - last digit illegible

7. Description of Work Replaced 2-VTV-010-500, -501, -517 and associated piping.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure. ** Exempt
 Other Pressure N/A psi Test Temp. N/A °F ** - Code Case N-416-3

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 20077-0333 and Design Criteria BFN-50-7010 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 63374A
Work Order (WO) 03-022883-001

9. Remarks Replaced 2-VTV-010-500, -501, -517 and associated piping.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Wilford* System Engineer Date 6-16 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/14/07 to 6/29/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sam Flood
Inspector's Signature

Commissions TN 4011
National Board, State, Province, and Endorsements

Date 6/29 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner: Tennessee Valley Authority (TVA) Date June 18, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant: Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Design Change Notice (DCN) 50287 Stg 1
 Work Orders (WOs) 00-003350-000, -002, -003 and -005
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by: TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 073, High Pressure Coolant Injection (HPCI) System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
HPCI Steam Line Inbd Isolation Valve	Crane	N/A	N/A	2-FCV-073-0002	N/A	Removed	No
HPCI Steam Line Inbd Isolation Valve	Flowserve	E-125T-1-2	N/A	2-FCV-073-0002	2001	Installed	No
pipe	unknown	N/A	N/A	N/A	N/A	Removed	No
pipe	United States Steel	N/A	N/A	N/A	N/A	Installed	No
pipe support 2-47B455S0023	Bergen Patterson	15620	N/A	2-SNUB-073-5008	N/A	Removed	No
pipe support 2-47B455-2124	Lisega	614863-076 614863-077	N/A	2-SNUB-073-5006 2-SNUB-073-5007	N/A	Installed	No
pipe support 2-47B455-2123	Lisega	614863-073	N/A	2-SNUB-073-5005	N/A	Installed	No

7. Description of Work Replaced 2-FCV-73-2 with a new double disc gate valve. Replaced a section of pipe.
Removed pipe support 2-47B455S0023. Installed new supports 2-47B455-2123 and 2-47B455-2124.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
 Other Pressure N/A psi Test Temp. N/A °F ** - Code Case N-416-3

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7073, BFN-50-C-7105 and BFN-50-C-7107.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 50287 Stg 1
Work Orders (WOs) 00-003350-000, -002, -003 and -005

9. Remarks Replaced 2-FCV-73-2 with a new double disc gate valve. Replaced a section of pipe.
Applicable Manufacturer's Data Reports to be attached
Removed pipe support 2-47B455S0023. Installed new supports 2-47B455-2123 and 2-47B455-2124.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed: *Stephen C. Wilford* System Engineer Date 6-18 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 1/13/07 to 6/29/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

James Flunk Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/29 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 20, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit -2

Work Order (WO) 03-004270-000
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)

ASME Section III, Article 9, 1965 and

5. (a) Applicable Construction Code ASME Section III 19 68 Edition, Summer 1970 Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1063	N/A	2-PCV-001-0034	1968	Removed	Yes
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1019	N/A	2-PCV-001-0034	1968	Installed	Yes

7. Description of Work Replaced Main Steam Relief valve main body.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in GE P. O. 205AJ600, and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Order (WO) 03-004270-000

9. Remarks The main valve body was replaced with rebuilt valve body previously used on Unit 3 (same manufacturer/model number).
Applicable Manufacturer's Data Reports to be attached.
As a part of the Tech Spec required valve inspections WO 03-004270-000 replaced 2-PCV-001-0034 with a rebuilt valve previously used
in BFN Unit 3 (3-PCV-001-0042, S/N 1019). The replacement valve was removed from Unit 3 by WO 03-004244-000 and refurbished by
WO 03-004244-001. No pressure retaining parts were replaced during the refurbishment.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 6-20 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/13/07 to 6/29/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/29 20 07

FORM N-2 MANUFACTURERS DATA REPORT FOR PRESSURE VESSEL PART AND APPURTENANCES

13

As required by the Provision of the ASME Code Rules

1. (a) Manufactured by Target Rock Corp., 1966th Broadhollow Rd., E. Farmingdale, NY
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co., 175 Curtner Ave., San Jose, Calif.
(Name and address of Manufacturer of complete machine component)

2. Identification-Manufacturer's Serial No. of Part 1019 Part Id. No. ---

(a) Constructed According to Drawing No. 7567F-000-22 Drawing Prepared by Target Rock Corp.

(b) Description of Part Inspected Base Assembly

(c) Applicable ASME Code Section III, Edition 1968, Addenda date Summer 1970, Case No. -- Class 1

Remarks Base assembly is the subassembly component of TRC model 7567F
(Brief description of service for which component was designed)
Safety/Relief valve which serves as the control element. Application
of S/R valve is for BWR (Steam) service.

We certify that the statements made in this report are correct and this vessel part or appurtenance as defined in the Code conforms to the rules of construction of the ASME Code Section III.
 (The applicable Design Specification and Stress Report are not the responsibility of the part Manufacturer. An appurtenance Manufacturer is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date 9-14-78 Signed Target Rock Corp. By [Signature]
(Manufacturer)

Certificate of Authorization Expires 12/9/80 Certificate of Authorization No. 1948

CERTIFICATION OF DESIGN FOR APPURTENANCE (when applicable)

Design information on file at Target Rock Corp.

Stress analysis report on file at Target Rock Corporation

Design specifications certified by R.R. Ghosh Prof. Eng. State Calif. Reg. No. 16371

Stress analysis report certified by D.M. Patarini Prof. Eng. State N.Y. Reg. No. 029841

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of New York and employed by Commercial Union Ins. Co. of Boston, Mass. have inspected the part of a pressure vessel described in this Manufacturer's Partial Data Report on 9/26 1978 and state that to the best of my knowledge and belief, the Manufacturer has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this Manufacturer's Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9/26 1978 William J. Ireland NEW YORK STATE COMMISSION NO: 2288
Inspector's Signature ALSO COMMISSIONED IN: Fechl, Ohio & Conn.
Commission No. National Board, State, Province and No.

Supplemental sheets in form of lists, sketches or drawings may be used provided: (1) size is 8 1/2" x 11"; (2) information in items 1-2 on this data report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "Remarks".

Rebuilt on WO 03-004244-001, Installed on
 2-PCR-001-0034 on WO 03-004270-000
 8020410-07

[Signature]
 9-28-78
 NY 71315

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 16, 2007
 Sheet 1 of 5

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2
 Design Change Notice (DCN) 66314A,
 Work Orders (WOs) 06-711366-000 through 06-711366-007
 and 06-711366-021 and 06-711366-026
Repair/Replacement Organization P.O. No., Job No., etc

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 001, Main Steam System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Line A Inboard Isolation Valve	Atwood & Morrill	N/A	N/A	2-FCV-001-0014	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	4	N/A	2-FCV-001-0014	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	1	N/A	2-FCV-001-0014	N/A	Installed	No
poppet valve disc	Atwood & Morrill	1	N/A	2-FCV-001-0014	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-1	N/A	2-FCV-001-0014	N/A	Installed	No
cover plate	Atwood & Morrill	1	N/A	2-FCV-001-0014	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-5	N/A	2-FCV-001-0014	N/A	Installed	No

7. Description of Work Replaced pilot poppet valve discs, poppet valve discs and cover plates. Buildup on rib guide on 2-FCV-001-0037.
Replaced fillet weld on pilot poppet valve disc on 2-FCV-001-0027.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 90744, GE Purchase Spec 21A1062 Rev. 0 and 21A1062AL Rev. 6 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 66314A,
Work Orders (WOs) 06-711366-000 through 06-711366-007
and 06-711366-021 and 06-711366-026

- 9. Remarks Replaced pilot poppet valve discs, poppet valve discs and cover plates. Buildup on rib guide on 2-FCV-001-0037.
Replaced fillet weld on pilot poppet valve disc on 2-FCV-001-0027.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 6-29 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
or Province of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 1/25/07 to 6/30/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/30 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Date June 16, 2007

Sheet 2 of 5

Unit 2

Design Change Notice (DCN) 66314A,
 Work Orders (WOs) 06-711366-000 through 06-711366-007
 and 06-711366-021 and 06-711366-026

Repair/Replacement Organization P.O. No., Job No., etc.
 Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System: System 001 Main Steam System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, Addenda, Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Line B Inboard Isolation Valve	Atwood & Morrill	N/A	N/A	2-FCV-001-0026	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0026	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	4	N/A	2-FCV-001-0026	N/A	Installed	No
poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0026	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-7	N/A	2-FCV-001-0026	N/A	Installed	No
cover plate	Atwood & Morrill	N/A	N/A	2-FCV-001-0026	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-7	N/A	2-FCV-001-0026	N/A	Installed	No
Main Steam Line B Outboard Isolation Valve	Atwood & Morrill	N/A	N/A	2-FCV-001-0027	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0027	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	3	N/A	2-FCV-001-0027	N/A	Installed	No
fillet weld to pilot poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0027	N/A	Installed	No
poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0027	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-6	N/A	2-FCV-001-0027	N/A	Installed	No
cover plate	Atwood & Morrill	N/A	N/A	2-FCV-001-0027	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-3	N/A	2-FCV-001-0027	N/A	Installed	No

7. Description of Work Replaced pilot poppet valve discs, poppet valve discs and cover plates.
Replaced fillet weld on pilot poppet valve disc on 2-FCV-001-0027.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

Owner <u>Tennessee Valley Authority (TVA)</u> <small>Name</small> <u>1101 Market Street</u> <small>Address</small> <u>Chattanooga, TN 37402-2801</u> <small>Address</small> 2. Plant <u>Browns Ferry Nuclear Plant (BFN)</u> <small>Name</small> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small> 3. Work Performed by <u>TVA-BFN</u> <small>Name</small> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small> 4. Identification of System <u>System 001, Main Steam System (ASME Code Class 1 equivalent)</u> 5. (a) Applicable Construction Code <u>USAS B31.1.0</u> 19 <u>67</u> Edition, <u> </u> Addenda, <u> </u> Code Case (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 <u>95, 1996 Addenda</u> 6. Identification of Components	Date <u>June 16, 2007</u> Sheet <u>3</u> of <u>5</u> Unit <u>2</u> Design Change Notice (DCN) 66314A, Work Orders (WOs) 06-711366-000 through 06-711366-007 and 06-711366-021 and 06-711366-026 Repair/Replacement Organization P.O. No., Job No., etc. Type Code Symbol Stamp <u>N/A</u> Authorization No. <u>N/A</u> Expiration Date <u>N/A</u>
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Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Main Steam Line D Inboard Isolation Valve	Atwood & Morrill	N/A	N/A	2-FCV-001-0037	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	1	N/A	2-FCV-001-0037	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	2	N/A	2-FCV-001-0037	N/A	Installed	No
poppet valve disc	Atwood & Morrill	3	N/A	2-FCV-001-0037	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-4	N/A	2-FCV-001-0037	N/A	Installed	No
cover plate	Atwood & Morrill	N/A	N/A	2-FCV-001-0037	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-6	N/A	2-FCV-001-0037	N/A	Installed	No
weld buildup on rib guide	Atwood & Morrill	N/A	N/A	2-FCV-001-0037	N/A	Corrected	No
Main Steam Line D Outboard Isolation Valve	Atwood & Morrill	N/A	N/A	2-FCV-001-0051	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0051	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	3	N/A	2-FCV-001-0051	N/A	Installed	No
poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0051	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-8	N/A	2-FCV-001-0051	N/A	Installed	No
cover plate	Atwood & Morrill	N/A	N/A	2-FCV-001-0051	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-8	N/A	2-FCV-001-0051	N/A	Installed	No

7. Description of Work Replaced pilot poppet valve discs, poppet valve discs and cover plates. Buildup on rib guide on 2-FCV-001-0037.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

FORM NIS-2 OWNER'S REPORT FOR REPAIRS/REPLACEMENT ACTIVITY SUPPLEMENTAL SHEET

Owner Tennessee Valley Authority (TVA) Date June 16, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 4 of 5
Name Unit 2
Address
P. O. Box 2000, Decatur, AL 35609-2000
Address

3. Work Performed by TVA-BFN Design Change Notice (DCN) 66314A,
Name Work Orders (WOs) 06-711366-000 through 06-711366-007
P. O. Box 2000, Decatur, AL 35609-2000 and 06-711366-021 and 06-711366-026
Address Repair/Replacement Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A

4. Identification of System System 001 Main Steam System (ASME Code Class 1 equivalent) Authorization No. N/A
Address Expiration Date N/A

5. (a) Applicable Construction Code USAS B31.10 19 67 Edition, Addenda, Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
	Atwood & Morrill	N/A	N/A	2-FCV-001-0052	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0052	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	4	N/A	2-FCV-001-0052	N/A	Installed	No
poppet valve disc	Atwood & Morrill	N/A	N/A	2-FCV-001-0052	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-2	N/A	2-FCV-001-0052	N/A	Installed	No
cover plate	Atwood & Morrill	N/A	N/A	2-FCV-001-0052	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-4	N/A	2-FCV-001-0052	N/A	Installed	No
	Atwood & Morrill	N/A	N/A	2-FCV-001-0015	N/A	‡	No
‡ - replaced pilot poppet valve disc, poppet valve disc and cover plate							
pilot poppet valve disc	Atwood & Morrill	1	N/A	2-FCV-001-0015	N/A	Removed	No
pilot poppet valve disc	Atwood & Morrill	1	N/A	2-FCV-001-0015	N/A	Installed	No
poppet valve disc	Atwood & Morrill	2	N/A	2-FCV-001-0015	N/A	Removed	No
poppet valve disc	Atwood & Morrill	F0678-3	N/A	2-FCV-001-0015	N/A	Installed	No
cover plate	Atwood & Morrill	4	N/A	2-FCV-001-0015	N/A	Removed	No
cover plate	Atwood & Morrill	F0677-1	N/A	2-FCV-001-0015	N/A	Installed	No

7. Description of Work Replaced pilot poppet valve discs, poppet valve discs and cover plates.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
Other Pressure N/A psi Test Temp. N/A °F

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Date June 16, 2007

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) 68888A,
 Work Order (WO) 06-723036-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 069, Reactor Water Cleanup (RWCU) System (ASME Code Class 1 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case.

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RWCU Suction Isol. from Recirc Loop A	Velan	N/A	N/A	2-ISV-069-0500	N/A	Removed	No
RWCU piping	BF Shaw	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Removed 2-ISV-069-0500 and installed pipe in it's place.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
** - ref. Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7069 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 68888A,
Work Order (WO) 06-723036-001

9. Remarks Removed 2-ISV-069-0500 and installed pipe in it's place.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed: *Stephen Wilford* System Engineer Date 6-21 2007
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 3/18/07 to 6/29/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sam Flavel
Inspector's Signature

Commissions TN4011
National Board, State, Province, and Endorsements

Date 6/29 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 21, 2007
 Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2
 Work Order (WO) 03-015954-000
Repair/Replacement Organization P.O. No., Job No., etc

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 071, Reactor Core Isolation Cooling (RCIC) System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RCIC Turbine Steam Supply Valve	Walworth	N/A	N/A	2-FCV-071-0008	N/A	Corrected	No

7. Description of Work Weld build up on disc where the tack welds were cut off. Replaced the stem. Did not replace the wedge.
Replaced tack welds

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
** - ref. Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7071 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Order (WO) 03-015954-000

9. Remarks Weld build up on disc where the tack welds were cut off. Replaced the stem. Did not replace the wedge. Replaced tack welds
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed: [Signature] System Engineer Date: 6-25 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 3/9/07 to 6/27/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/27 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 25, 2007
Name
1101 Market Street
Address
Chattanooga, TN. 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Work Orders (WO) 04-718365-000
 Design Change-Notice (DCN) 65250A
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 068, Reactor Water Recirculation System (ASME Code Class 1 equivalent)
(supports) AISC 8th Edition and MSS-SP-58-1967
5. (a) Applicable Construction Code (piping system) USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements: 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S069	Grinnell	TVA Serial # G011	N/A	2-SNUB-068-5005	N/A	Removed	No
Support (Snubber) 2-47B408S069	Pacific Scientific	5498	N/A	2-SNUB-068-5005	N/A	Installed	No

7. Description of Work Modified the support (including one pin) and replaced the hydraulic snubber with a mechanical snubber.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other ** Pressure N/A psi Test Temp. N/A °F ** - See Remarks

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contracts 79KA2-825011 and 94N75-108205-000 and Design Criteria BFN-50-7068 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Orders (WO) 04-718365-000
Design Change Notice (DCN) 65250A

9. Remarks 2-SNUB-068-5005

Applicable Manufacturer's Data Reports to be attached

Modified the support (including a pin) and replaced the hydraulic snubber with a mechanical snubber.

The replacement snubber (5498) is a new snubber and was functionally tested per 2-SI-4.6.H-2A prior to installation.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed

Stephen C. Williams
Owner or Owner's Designee, Title

System Engineer

Date

6-25 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 3/25/05 to 6/26/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Samuel Howard
Inspector's Signature

Commissions

TN4011

National Board, State, Province, and Endorsements

Date

6/26 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 25, 2007
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Design Change Notice (DCN) 68707A,
 Work Order (WO) 06-710345-002
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 001, Main Steam System (ASME Code Class 2 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 -19 67 * Edition, N/A Addenda, N/A Code Case,
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
HP Steam to RFPT A Isol Valve	Wm. Powell Co.	N/A	N/A	2-FCV-001-0125	N/A	Installed	No

7. Description of Work Seal welded a plug in an injection port in the gland packing area of the valve which had been used for Furmanite injection.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
** - ref. Code Case N-416-3.
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Design Change Notice (DCN) 68707A,
Work Order (WO) 06-710345-002

9. Remarks Seal welded a plug in an injection port in the gland packing area of the valve which had been used for Furmanite injection.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Stephen C. Wilford System Engineer
Owner or Owner's Designee, Title

Date 6-25 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described in this Owner's Report during the period 2/23/07 to 6/27/07; and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sam F. Ford
Inspector's Signature

Commissions: TN 4011
National Board, State, Province, and Endorsements

Date 6/27 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Date June 25, 2007
 Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2
 Engineering Document Change (EDC) 51601,
 Work Order (WO) 06-711111-000
Repair/Replacement Organization, P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 075, Core Spray (CS) System (ASME Code Class 2 equivalent)

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
CS System I Test Bypass Valve	Walworth	N/A	N/A	2-FCV-075-0022	N/A	‡	No
‡ - Removed seal leak off line and installed plug							
seal leak off line	Walworth	N/A	N/A	N/A	N/A	Removed	No
seal leak off valve	Hancock	N/A	N/A	2-LOV-075-0022	N/A	Removed	No
plug	TVA	N/A	N/A	N/A	N/A	Installed	No

7. Description of Work Removed packing leak off line and 2-LOV-075-0022 from 2-FCV-075-0022, seal welded plug in port.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
** - ref. Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in G.E. PO 205H090 and Design Criteria BFN-50-7075 and BFN-50-C-7 f05.

FORM NIS-2 (Back)

WID: Engineering Document Change (EDC) 51601,
Work Order (WO) 06-711111-000

9. Remarks Removed packing leak off line and 2-LOV-075-0022 from 2-FCV-075-0022; seal welded plug in port.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen C. Williams* System Engineer
Owner or Owner's Designee Title

Date 6-25 20 07

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 2/12/07 to 6/26/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

San Hurd
Inspector's Signature

Commissions TN4011

National Board, State, Province, and Endorsements

Date 6/26 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY
-As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date: June 25, 2007
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Order (WO) 06-725504-000
Repair/Replacement Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 010, Boiler Drains and Vents and Blowdown System (ASME Code Class 1 equivalent)
5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RPV Drain to RWCU	Anchor/Darling	E3265-37-8	N/A	2-DRV-010-0505		Removed	No
RPV Drain to RWCU	Anchor/Darling	E3265-37-9	N/A	2-DRV-010-0505		Installed	No
pipe	Unknown	N/A	N/A	N/A		Removed	No
pipe	PEXCO	N/A	N/A	N/A		Installed	No
coupling	Alloy Stainless Products Co.	N/A	N/A	N/A		Installed	No

7. Description of Work Replaced valve and a section of piping
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** Exempt
** - ref. Code Case N-416-3
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 80633E and Design Criteria BFN-50-7010 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Order (WO) 06-725504-000

9. Remarks Replaced valve and a section of piping
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 6-25 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut

have inspected the components described in this Owner's Report during the period 1/31/07 to 6/29/07 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/29 20 07

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner: Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date: June 25, 2007
 Sheet 1 of 1

2. Plant: Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit: 2
 Work Order (WO) 07-711826-001
Repair/Replacement Organization P.O. No., Job No., etc.

3. Work Performed by: TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp: N/A
 Authorization No.: N/A
 Expiration Date: N/A

4. Identification of System: System 069, Reactor Water Cleanup (RWCU) System (ASME Code Class 1, equivalent)

5. (a) Applicable Construction Code: USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements: 19 95 Edition, 1996 Addenda

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped (Yes or No)
RWCU weld	TVA	N/A	N/A	RWCU-2-003-038	N/A	Corrected	No
RWCU weld	TVA	N/A	N/A	RWCU-2-003-039	N/A	Corrected	No

7. Description of Work: Built up two socket welds on the RWCU chemical decon line.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Exempt
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7069 and BFN-50-C-7105.

FORM NIS-2 (Back)

WID: Work Order (WO) 07-711826-001

9. Remarks Built up two socket welds on the RWCU chemical decon line.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen Wilford* System Engineer Date 6-25 20 07
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Tennessee and employed by HSB CT of Connecticut have inspected the components described

in this Owner's Report during the period 3/27/07 to 6/27/07, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Sam Flavel Commissions TW4011
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/27 20 07