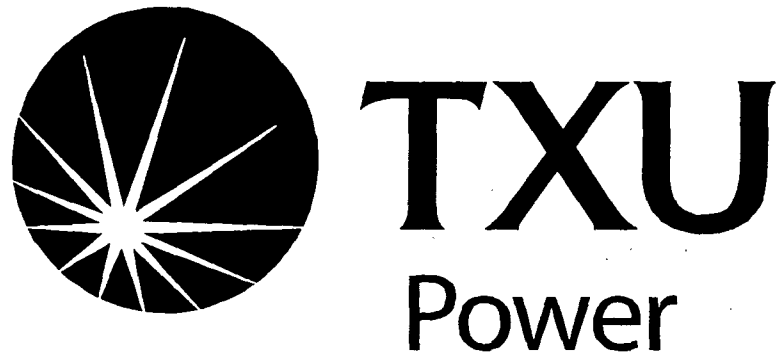


Enclosure 2

# ***Comanche Peak SES Transition to Westinghouse Safety Analysis Methodologies***



**May 23, 2007**

## CPSES Stretch Power Uprate

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***Purpose of meeting is to inform the NRC of our plans & schedule to facilitate resource allocation by the NRC***

3

## CPSES Stretch Power Uprate

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### **Meeting Agenda**

- CPSES Uprate History
- Project team
- Project objectives, milestones & overview
- NSSS analyses
- BOP analyses
- Plant modifications
- Testing
- Submittal format
- Active & planned licensing submittals
- Summary and Q&A

4

## Methodology Transition Project - What and Why?



Prior to 2006:

- TXU Power's Core Design & Safety Analysis organizations performed FSAR Chapter 4, 6, 7 and 15 analyses in-house, using vendor-independent methods

May of 2006:

- TXU Power transferred in-house Core Design & Safety Analysis and Risk & Reliability functions to Westinghouse Electric Co.
- Teaming arrangement developed, where Westinghouse performs these functions in the best interests of Comanche Peak
- Agreement includes application of latest NRC-approved Westinghouse analysis methodologies
  - Allows power uprates, longer cycles, more efficient core designs, newer technologies for more efficient plant operation

5

## Agenda



- Purpose of today's discussions
- Overview of the Accident Analysis Methodology Transition Project
- Specifics of Technical Specification Changes
  - Generic, enabling changes
    - Submitted for NRC review April 10, 2007
  - Conforming changes
    - Submitted for NRC review April 10, 2007
  - Cycle Specific Applications
    - To be submitted upon completion of analyses
- Licensing Action Submittal Schedules
- Concluding discussions

6

## Project Objectives



- Increase Rated Thermal Power from 3458 to 3612 MWt (~4.5%)
- Increase plant output
  - Unit 1                    49 MWe
  - Unit 2                    37 MWe
- Design and operating margin will be protected to assure continued safe and reliable plant operation
  - Modifications
  - Analyses

7

## Project Milestones



- August 2007 – LAR submittal to NRC (Units 1 & 2)
- Fall 2008 Outage – Unit 1 uprate implementation
- Fall 2009 Outage – Unit 2 uprate implementation

*NRC approval of proposed Uprate LAR requested to support the Fall 2008 Unit 1 outage*

8

## Methodology Transition Project Overview



- All adopted Westinghouse methodologies have been previously approved by the NRC on a generic basis for application at plants such as Comanche Peak

9

## Required Tech Spec Changes – enabling changes



- Tech Spec 5.6.5b, "Core Operating Limits"
- List of Methodologies used to establish Core Operating Limits expanded to include standard Westinghouse Methodologies
  - WCAP-11397-P-A, "Revised Thermal Design Procedure," April 1989.
  - WCAP-8745-P-A, "Design Bases for the Thermal Overpower  $\Delta T$  and Thermal Overtemperature  $\Delta T$  Trip Functions," September 1986.
    - Applicable to Comanche Peak's N-16-based OT/OP system
  - WCAP-14565-P-A, "VIPRE-01 Modeling and Qualification for Pressurized Water Reactor Non-LOCA Thermal-Hydraulic Safety Analysis," October 1999.
  - WCAP-12472-P-A, "BEACON Core Monitoring and Operations Support System," August 1994.

10

## NSSS Analyses - continued



- Component Evaluations
- System Evaluations
- Safety analyses include:
  - LOCA M&E / containment
  - SLB M&E / containment
  - Spent fuel pool criticality
  - Non-LOCA analyses
  - LB BELOCA w/ASTRUM <sup>(1)</sup>
  - SBLOCA <sup>(1)</sup>

<sup>(1)</sup> Addressed in Transition to Westinghouse Methodologies

11

## NSSS Uprate Modifications



### Nuclear Steam Supply System (NSSS)

- NSSS Control System changes
  - Steam Dump Control System
  - Rod Control System
  - SG Level Control System
  - Pressurizer Water Level Control System

12

## Required Tech Spec Changes – conforming changes



### •BEACON-TSM

- Replace the phrase “moveable incore detectors” to “core power distribution measurement information”
- Replace the phrase “flux map” to “power distribution measurements”
- Affects:
  - TS 3.1.7, “Rod Position Indication”
  - TS 3.2.1, “Heat Flux Hot Channel Factor”
  - TS 3.2.2, “Nuclear Enthalpy Rise Hot Channel Factor”
  - TS 3.2.4, “Quadrant Power Tilt Ratio”
  - TS 3.3.1, “Reactor Trip System (RTS) Instrumentation.”

13

## Required Tech Spec Changes – cycle-specific applications



- Tech Spec changes identified as cycle-specific analyses are completed
- Cycle-specific analyses are performed to bound operation at current Rated Thermal Power as well as planned uprated power
  - Current Rated Thermal Power = 3458 MWth
  - Planned Rated Thermal Power = 3612 MWth
- Permission to operate at uprated power is NOT part of this project

14



## Power Ascension Testing



- **Plant transient testing under evaluation**
- **NSSS & BOP systems** - monitoring during uprate power ascension
- **Flow-induced vibration monitoring of the following systems**
  - Main Steam, Feedwater (outside of containment)
  - Condensate / Heater Drains
  - Extraction Steam
- **Post-Modification Testing, including**
  - Heater Drain pump / system
  - Isophase bus duct cooling
- **Turbine Thermal Performance Testing**

15

## Submittal Format & Content



- **RS-001 EPU standard used to facilitate NRC review**
- **Recent NRC RAIs addressed**
- **Current licensing basis summarized**
- **Uses Westinghouse, Shaw, & Siemens uprate experience**

16

## Schedules



Mid-April 2008 First application of Westinghouse methodologies to Unit 2 Cycle 11  
NRC Approval Requested 2/15/08

### Supporting Milestones:

April 2007	Submitted Enabling & Conforming Tech Spec changes
July 2007	Submit SBLOCA & ASTRUM Evaluation Models
September 2007	Submit cycle-specific Tech Spec changes required to support methodology transition
	RTS & ESFAS Allowable Values
	- affected functions only
	Pressurizer Safety Valve set pressure and as-found tolerance

17

## General Discussion



Any Comments or Questions?

18

## Summary



- CPSES SPU will result in ~4.5% increase in RTP
- Dedicated, experienced project team
- Similar approach to other recent Westinghouse 4 loop SPU projects
- Design and operating margin will be protected to assure continued safe and reliable plant operation
- RS-001 EPU standard used to facilitate NRC review

19

## Summary



### Project Milestones

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**Questions?**

20