



10 CFR 50.90

LR-N07-0145
LCR H06-02

JUN 21 2007

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Hope Creek Generating Station
Facility Operating License No. NPF-57
NRC Docket No. 50-354

Subject: Supplement to License Amendment Request to Relocate Component Lists
for Primary Containment Isolation Valves from Technical Specifications

Reference: 1) Letter from George P. Barnes (PSEG Nuclear LLC) to USNRC,
November 15, 2006

In Reference 1, PSEG Nuclear LLC (PSEG) requested an amendment to Facility
Operating License NPF-57 and the Technical Specifications (TS) for the Hope Creek
Generating Station (HCGS) to remove valve component lists and references to the lists
from the Technical Specifications (TS).

PSEG determined that Reference 1 inadvertently omitted the deletion of references to
TS Table 3.6.3-1 from TS Actions 3.6.1.2.d and 3.6.1.2.e and from TS restoration
Actions 3.6.1.2.d and 3.6.1.2.e. These additional TS changes are consistent with the
proposed changes to Limiting Conditions for Operation (LCO) 3.6.1.2.d and 3.6.1.2.e in
Reference 1. Attachment 1 to this letter provides a revised marked up TS page that
incorporates the additional required changes.

PSEG has determined that the information contained in this letter and attachments does
not alter the conclusions reached in the 10CFR50.92 no significant hazards analysis
previously submitted.

There are no regulatory commitments contained within this letter.

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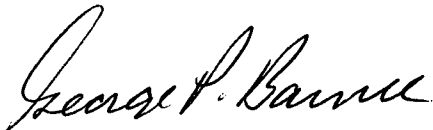
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Should you have any questions regarding this submittal, please contact Mr. Paul Duke at 856-339-1466.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 6/21/07
(date)

Sincerely,



George P. Barnes
Site Vice President
Hope Creek Generating Station

cc: S. Collins, Regional Administrator – NRC Region I
R. Ennis, Project Manager - USNRC
NRC Senior Resident Inspector - Hope Creek
K. Tosch, Manager IV, NJBNE

**Hope Creek Generating Station
Facility Operating License NPF-57
Docket No. 50-354**

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Primary Containment Isolation Valves from Technical Specifications**

Revised Markup of Technical Specification Page

TS Page

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CONTAINMENT SYSTEMS
LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- d. The measured combined leakage rate for all containment isolation valves which form the boundary for the long-term seal of the feedwater lines ~~in Table 3.6.3-1~~ exceeding 10 gpm, or
- e. The measured combined leakage rate for all other penetrations and containment isolation valves in hydrostatically tested lines ~~in Table 3.6.3-1~~ which penetrate the primary containment exceeding 10 gpm,

restore:

INSERT C
PSEG LETTER LR-N06-0437, ATT. 2

- a. The overall integrated leakage rate(s) (Type A test) to be in accordance with the Primary Containment Leakage Rate Testing Program, and
- b. ~~The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, valves which form the boundary for the long-term seal of the feedwater lines, and other valves which are hydrostatically tested per Table 3.6.3-1, subject to Type B and C tests to be in accordance with the Primary Containment Leakage Rate Testing Program, and~~
- c. The leakage rate to less than or equal to 150 scfh per main steam line and less than or equal to 250 scfh combined through all four main steam lines, and
- d. The combined leakage rate for all containment isolation valves which form the boundary for the long-term seal of the feedwater lines ~~in Table 3.6.3-1~~ to less than or equal to 10 gpm, and
- e. The combined leakage rate for all other penetrations and containment isolation valves in hydrostatically tested lines ~~in Table 3.6.3-1~~ which penetrate the primary containment to less than or equal to 10 gpm,

prior to increasing reactor coolant system temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2.a The primary containment leakage rates shall be demonstrated in accordance with the Primary Containment Leakage Rate Testing Program for the following:

- 1. Type A test.
- 2. Type B and C tests (including air locks).
- b. DELETED.
- c. DELETED.

* Exemption to Appendix "J" of 10 CFR 50.