BFB

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N. W. SUITE 818
ATLANTA, GEORGIA 30303

MAY 1 8 1976

In Reply Refer To:
IE:II:VLB
50-390/76-5
50-391/76-5

Tennessee Valley Authority
Attn: Mr. Godwin Williams, Jr.
Manager of Power
830 Power Building
Chattanooga, Tennessee 37401

Gentlemen:

This refers to the inspection conducted by Mr. V. L. Brownlee of this office on April 20-23, 1976, of activities authorized by NRC Construction Permit Nos. CPPR-91 and CPPR-92 for the Watts Bar Nuclear Plant facility, and to the discussion of our findings held with Mr. J. C. Killian at the conclusion of the inspection.

Areas examined during the inspection and our findings are discussed in the enclosed inspection report. Within these areas, the inspection consisted of selective examination of procedures and representative records, interviews with personnel, and observations by the inspector.

Two new unresolved items resulted from this inspection and are identified in Section III of the summary of the enclosed report. These items will be examined during subsequent inspections.

During the inspection, it was found that certain activities under your license appear to be in noncompliance with NRC requirements. These items and references to pertinent requirements are listed in Section I of the summary of the enclosed report. A reply to one of the items of noncompliance is not requested in that the item of noncompliance was identified by TVA's internal management system and corrective actions to correct the noncompliance and prevent recurrence were completed prior to this inspection.

This notice is sent to you pursuant to the provisions of Section 2.201 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations. Section 2.201 requires you to submit to this office, within 20 days of your receipt of this notice, a written statement or explanation in reply including: (1) corrective steps which have been taken by you, and the results achieved; (2) corrective steps which will be taken to avoid further noncompliance; and (3) the date when full compliance will be achieved.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room. If this report contains any information that you believe to be proprietary, it is necessary that you submit a written application to this office requesting that such information be withheld from public disclosure. If no proprietary information is identified, a written statement to that effect should be submitted. If an application is submitted, it must fully identify the bases for which information is claimed to be proprietary. The application should be prepared so that information sought to be withheld is incorporated in a separate paper and referenced in the application since the application will be placed in the Public Document Room. Your application, or written statement, should be submitted to us within 20 days. If we are not contacted as specified, the enclosed report and this letter may then be placed in the Public Document Room.

Should you have any questions concerning this letter, we will be glad to discuss them with you.

Very truly yours,

C. E. Murphy, Chief
Reactor Construction and
Nuclear Support Branch

Enclosure:
IE Inspection Report Nos.
50-390/76-5 and 50-391/76-5

cc w/encl: Mr. J. E. Gilleland
Assistant Manager of
Power

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N. W. SUITE 818
ATLANTA, GEORGIA 30303

IE Inspection Report Nos. 50-390/76-5 and 50-391/76-5

Licensee: Tennessee Valley Authority
830 Power Building
Chattanooga, Tennessee 37401

Facility Name: Watts Bar Nuclear Plant, Units 1 and 2
Docket Nos.: 50-390 and 50-391
License Nos.: CPPR-91 and CPPR-92
Category: A2/A2

Location: Spring City, Tennessee

Type of License: W PWR, 1160 Mwe

Type of Inspection: Unannounced, Construction

Dates of Inspection: April 20-23, 1973

Dates of Previous Inspection: March 8-9, 1976

Principal Inspector: V. L. Brownlee, Reactor Inspector
Projects Section
Reactor Construction and Engineering
Support Branch

Accompanying Inspectors: R. W. Wright, Reactor Inspector
Engineering Support Section No. 1
Reactor Construction and Engineering
Support Branch

C. R. McFarland, Reactor Inspector
Projects Section
Reactor Construction and Engineering
Support Branch

Other Accompanying Personnel: C. E. Murphy, Chief
Reactor Construction and Engineering
Support Branch

Principal Inspector: V. L. Brownlee 5/13/76
Date

V. L. Brownlee, Reactor Inspector
Projects Section
Reactor Construction and Engineering
Support Branch

Reviewed by: J. C. Bryant 5/13/76
Date

J. C. Bryant, Chief
Projects Section
Reactor Construction and Engineering
Support Branch

SUMMARY OF FINDINGS

I. Enforcement Items

Certain items appear to be in noncompliance with 10 CFR 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Processing Plants," as indicated below:

Infractions

76-5-A1(II) Lack of Procedures Implementation (Units 1 and 2)

Criterion V of Appendix B to 10 CFR 50 as implemented by Appendix A, paragraph A.2.5 of the PSAR specify that activities affecting quality are to be performed in accordance with established procedures.

Contrary to the requirement, TVA failed to implement established procedures as noted:

(1) Cement Testing-Frequency (Details III, paragraph 5)

(2) Water Testing-Frequency (Details III, paragraph 5)

This is an infraction.

One infraction which was identified through your internal management system and which was corrected is identified in Details I, paragraph 4 of this report. No reply to this item of noncompliance is required.

II. Licensee Action on Previously Identified Enforcement Matters

75-8-A1(II) Vendor QA Audits (Units 1 and 2)

TVA has submitted a letter of response dated November 10, 1975, which identified the corrective actions and plans. IE:II will examine the corrective actions and plans during subsequent inspections.

76-4-A1(II) QA Program Breakdown - Documentation of Radiographs and Weld History

CB&I failed to implement established procedures and TVA's program procedure for field surveillance failed to identify CB&I's nonconformance to established procedures. This item remains open.

III. New Unresolved Items

76-5/1 IE Bulletins and Licensee Responses

The following IEB's are identified for specific followup at the Engineering offices, Knoxville, during a subsequent inspection.

a. IE Bulletin No. 76-02 - Relay Coil Failure

b. IE Bulletin No. 76-03 Relay Malfunction

c. IE Bulletin No. 76-05 - Relay Failure

76-5/2 Documentation of Fabrication For Structural Steel Reactor Coolant System Supports (10 CFR 50-55(e))

TVA's initial report of this deficiency was made on March 24, 1976. The first interim report was submitted on April 23, 1976. IE:II will followup during subsequent inspections.

IV. Status of Previously Reported Unresolved Items

74-5/1 Valve Wall Thickness Verification Program (Units 1 and 2)

TVA informed IE:II personnel of preliminary plans relative to the valve wall thickness program. TVA will submit a formal valve wall thickness verification program that meets Region II letters of June 30, 1972, and February 16, 1973. This item remains open.

75-3/1 Regulatory Operations Bulletin and Licensee Response (Units 1 and 2)

ROB 74-9 - "Deficiency in General Electric Model 4KV Magne-Blast Circuit Breakers." This items remains open.

75-8/1 Charpy Impact Test Specimens (Units 1 and 2)

The licensee agreed to review the method of measuring Charpy impact test specimens presently being used to assure traceable calibration. This item remains open.

76-1/1 Containment Hold Down Anchor Bolt Nut (Units 1 and 2)

TVA informed Region II of a problem with anchor bolt nuts, and reported it as a 50.55(e) item. TVA submitted

the final report on March 9, 1976. Site work has not been completed to date. This item remains open. (Details I, paragraph 6)

76-2/1 Instrumentation Procedures (Units 1 and 2)

Specific procedures for receipt and handling of instruments have not been completed. TVA expects receipt of instrumentation in the near future which will require the use of these procedures. TVA has committed to develop applicable procedures prior to receipt of instrumentation on-site. This item remains open.

76-4/1 QA Program Breakdown - Documentation of Radiographs and Weld History Records (10 CFR 50.55(e)) (Units 1 and 2)

TVA informed Region II of the QA program breakdown problem and reported it as a 50.55(e) item. Site investigative work appears complete. Documentation and report effort continues. This item remains open. (Details I, paragraph 5)

V. Design Changes

None

VI. Unusual Occurrences

None

VII. Other Significant Findings

None

VIII. Management Interview

The exit interview was held on April 23, 1976, with Mr. J. C. Killian, Project Manager, members of his staff, and QA representatives of DED, DEC and OEDC. They were apprised of the findings of this inspection as noted in this report.

DETAILS I

Prepared by: V. L. Brownlee 5/11/76
Date

V. L. Brownlee, Reactor Inspector
Projects Section
Reactor Construction and Engineering
Support Branch

Dates of Inspection: April 20-23, 1976

Reviewed by: J. C. Bryant 5/13/76
Date

J. C. Bryant, Chief
Projects Section
Reactor Construction and Engineering
Support Branch

All information in Details I applies equally to Units 1 and 2 except where identified with a specific reactor.

1. Individuals Contacted

a. Tennessee Valley Authority (TVA)

J. C. Killian - Project Manager
T. B. Northern, Jr. - Construction Engineer
J. M. Lamb - Supervisor, Mechanical Engineering Unit
A. R. White - General Construction Superintendent
L. C. Northard - Supervisor, QA Unit
J. S. Colley - DED, QA Staff
J. C. Cofield - Supervisor, Materials Engineering Unit
R. F. Keck - OEDC, QA Staff
J. Reid - Cement Mason, Apprentice Instructor
D. Woods - General Foreman, Cement Finisher
W. J. Cobb - Cement Mason
S. Artman - Cement Mason
G. Saffels - Cement Mason
J. Ballard - Mechanical Engineer
D. Gilland - Cement Mason

b. Contractor Organization

Chicago Bridge and Iron (CB&I)

C. L. Spears - QA Supervisor

2. Scope

Inspection efforts included determining status of project, evaluation of concrete structural repairs and practices, followup of CB&I QA program breakdown - documentation of radiographs and weld history records, followup of containment hold down anchor bolt nuts problem and site QA unit audit activities.

3. Project Status

a. Unit 1

The reactor building crane wall is "topped out." Concreting operations continue for the reactor building shield wall and the refueling canal. CB&I is expected to start containment vessel erection during the first part of May 1976. Embed and support supplier problems are causing a significant delay in setting of the reactor vessel and steam generators. Cooling tower erection has been completed except for cleanup and miscellaneous items.

b. Unit 2

The containment vessel contractor, CB&I, has completed the first five rings of the vessel. TVA has started concrete operations within the containment building. Cooling tower work is nearing completion with a small percent of internal work, miscellaneous and cleanup remaining.

4. Evaluation of Concrete Structural Repairs and Practices

Input to this section is a combined effort of R. W. Wright and V. L. Brownlee.

During the entrance interview TVA informed the inspectors of an external inquiry relating to TVA's QA/QC program for control of concrete repair. The inquiry involved comments made by a former employee. TVA site personnel performed an investigation and concluded that, although one defective repair area was identified, this was not a matter of safety significance; that the structural integrity of the facilities was not being compromised; and that the QA/QC program for the control of concrete repair was providing quality concrete repairs.

The above isolated deficient area of repair was not inspected due to the inadequacies of the early version of the drilling or chipping release procedure, QCP-1.7. This procedure deficiency was subse-

quently identified and corrected by revising the procedure. The inadequacies of the original procedure which permitted the failure to inspect constituted an item of noncompliance of the infraction category. Since the procedural problems were identified and corrected within the TVA management control system and the specific area of concern is being re-repaired in accordance with established procedures TVA will not be cited for a noncompliance nor will a reply to this item of noncompliance be required.

IE inspector efforts relative to this matter included discussions with site management, engineers, general foreman, foreman, journeymen and apprentice cement finishers, physical observation of identified syspect areas of repair and review of site records and documentation. The inspector's findings are consistent with the licensee's conclusions. IE:II has no further questions regarding this matter at this time.

5. QA Program Breakdown - Documentation of Radiographs and Weld History Records (10 CFR 50.55(e))

CB&I has not gone back to work on containment vessel erection. The inspector was informed that all radiographic film relative to the containment vessel has been reviewed and accepted. Agreement has been reached that there is complete coverage of the vessel joints requiring radiography and that each film matches up with the adjacent film. The spot radiography comparisons confirm that the existing film represents the joints identified on the radiography reports.

Present plans are that CB&I will start Unit 1 containment vessel erection during the first part of May 1976.

6. Containment Hold Down Anchor Bolt Nuts (10 CFR 50.55(e))

TVA submitted the final report on March 9, 1976. The report appears to be clear and straight forward. Site examination of this matter indicates that Unit 2 corrective actions have been completed and fully documented. Unit 1 corrective actions and documentation of those actions have not been completed. This item remains open until Unit 1 corrective actions have been completed.

7. Site QA Unit - Audits

The scope of the site QA unit major activities include: (a) site auditing of DEC (internal), contracting, and/or service (external) organizations; (b) testing and certification of inspection personnel; (c) review and approval of site generated QA/QC documents; and (d) review of site purchase documents.

The inspector performed a general review of the unit audit program. The unit manpower consists of three civil, two electrical, and two mechanical auditors, a unit supervisor and a secretary. The period selected for audit review was from January 1975 through June 1976. The construction schedule is used as a basis for identifying activities to be inspected. From January-December 1975 there were thirteen general, six electrical, twelve mechanical, fifteen civil and four contractor audits performed.

The audit files were orderly and easily retrievable. The scope and frequency of audits performed provided good coverage of the general and discipline QC procedures. The coverage was consistent with the construction schedule. Selective examination of procedures for scope, depth and followup found the system to be functioning in accordance with the audit procedure DEC-QAP 18.01, "Auditing Construction Activities." No items of noncompliance were identified.

DETAILS II

Prepared by: C. R. McFarland 5-11-76
C. R. McFarland, Reactor Inspector — Date
Projects Section
Reactor Construction and Engineering
Support Branch

Dates of Inspection: April 20-22, 1976

Reviewed by: J. C. Bryant 5/12/76
J. C. Bryant, Chief — Date
Projects Section
Reactor Construction and Engineering
Support Branch

All information in Details II applies equally to Units 1 and 2 except where identified with a specific reactor at Watts Bar Nuclear Plant (WBNP)

1. Individuals Contacted

Tennessee Valley Authority (TVA)

Division of Engineering and Construction (DEC)

L. C. Northard - Supervisor, Site QA Staff
J. R. Inger - QA Engineer, Mechanical
J. M. Lamb - Supervisor, QC Mechanical Engineers
J. D. Shanlever - QC Engineer, Mechanicl
J. A. Holmes - QC Engineer, Instrumentation

2. Documents Reviewed

a. QA Manual for Design and Construction

(1) 2.0 QA Program (QAP)

(2) 2.01 QAP Description - All Plants except Browns Ferry

(3) 2.01.02 QAP Description - Applies to WBNP, Draft March 9, 1976

(4) 3.01 Field Change Requests

(5) 16.02 NRC-OIE Replies

(6) 15.01 Control of Nonconforming Items

(7) 18.01 Auditing of Construction Activities

b. Audit Reports

(1) M-75-03 Lifting and Transporting Major Components

(2) M-75-05 Receiving, Inspection and Storage of Steam Generators

(3) M-75-08 Lift and Transport of Steam Generators to Permanent Storage Area

(4) M-76-03 Handling, Storage and Maintenance of Permanent Mechanical Equipment

(5) E-76-02 Handling, Storage and Maintenance of Permanent Electrical Material

(6) E-76-03 Lifting and Transporting Major Components

(7) G-75-02 Receipt, Inspection, Storage, Withdrawal, and Transfer of Permanent Material

(8) G-75-07 Preparation and Documentation of Field Change Requests

(9) G-75-08 Engineering Change Notices

(10) G-75-09 Quality Assurance Records

(11) S-76-02 CBI Nuclear QA Manual, ASME Section III

c. Quality Control Procedures (QCP)

(1) 1.6R2 Receipt, Inspection, Storage, Withdrawal, and Transfer of Permanent Material

(2) 1.8RO Quality Assurance Records

(3) 1.9RO Disposition and Documentation of Engineering Change Notices

(4) 1.10RO Preparation and Control of WBNP QC Procedures

(5) 1.13RI Preparation and Documentation of Field Change Requests

(6) 1.18R2 Lifting and Transporting Major Components

(7) 3.1RO Handling, Storage, and Maintenance of Permanent Electrical Material

(8) 4.5R2 Handling, Storage and Maintenance of Permanent Mechanical Equipment

(9) 4.12RO Assembly and Installation of RPV Head and Control Rod Drive Mechanisms

d. Westinghouse Specifications

(1) "Procedures and Specifications Pertinent to Field Operations Involving Westinghouse Equipment or Systems, "Sent to WBNP March 17, 1976.

(2) Process Specification 597760, Rev. 4, "Cleanliness Requirements During Storage, Construction, Erection, and Startup Activities of Nuclear Power Systems"

(3) Process Specification 85310, Rev. 2, "Packaging and Preparing Nuclear Components and Spare Parts for Shipment and Storage"

(4) Process Specification 87114, "Steam Generator Preinstallation Maintenance Procedure"

(5) Technical Manual 1440-C255, "Pressurizer"

e. WBNF Test Procedures

(1) Test Procedures for Manitowoc Model 4600 Series IV Liftcrane, 60 Foot Diameter Series III

(2) Test Procedure for 4100 W Manitowoc Crane

(3) Test Procedure for 600 Ton Low Platform Trailer

3. Overall Review and Inspection of QA Program Implementation

The inspector reviewed the implementation of the licensee's QA program relative to site surveillance of design control, control of site originated design, audits of design control for site design, and audit records. Discussions were held with a site QA staff member and a QC engineer responsible for engineering change notices

(ECN), field change requests (FCR) and related QA records and documentation. The procedure for ECN's (QCP-1.9) and FCR's (QCP-1.13) and the QA Manual section for controlling nonconforming items (Section 15.01) were reviewed and discussed with the QA and QC engineers. Typical documentation was reviewed in the QA site files and in the QC and records files for mechanical and electrical design items. QA audits of ECN's and FCR's for related work were reviewed (audits G-75-07, G-75-08, M-76-03, and E-76-02). The QA Manual is being implemented. An effective system of site surveillance and audits of design control is being implemented and is consistent with the information reported in IE report numbers 50-390/75-5, 50-391/75-5, 50-390/75-6 and 50-391/75-6. No nonconformances nor unresolved items were identified.

4. QA/QC for Lifting and Handling Major Components

The inspector reviewed the implementation of the TVA QA program relative to the lifting and handling of the reactor vessels (RV), steam generators (SG) and pressurizers, the RV heads, and the RV internals. The Westinghouse requirements as defined in the specifications, paragraph 2.d, have been incorporated into the requirements of QCP 4.5 and QCP 4.12. A review of the documentation indicates that the equipment has been inspected routinely. Westinghouse site representatives are informed of the results of the inspections and audits of the functions associated with the major components have been performed as reported in audit reports (M-75-03, M-75-05, M-75-08, M-76-03, and E-76-03).

The inspector reviewed the procedures for testing the lifting and handling equipment and the records of tests prior to the handling of the major components to date.

The inspector observed a deadweight test of the Manitowoc Model 4600 that was being conducted April 21. The inspector had previously observed the off loading of a steam generator for Unit 2 (IE Reports 50-390/75-10 and 50-391/75-11) with the Manitowoc Model 4600.

The inspector observed the storage for reactor vessels (2), reactor vessel heads (2), reactor internals (2 sets of upper and lower assemblies), steam generators (6), and the pressurizer for Unit 1. The storage environment and protection, preservation of cleanliness, surveillance, seals, protective coatings and coverings, and support structures appear to comply with the WBNP and Westinghouse requirements. The RV's are provided physical protection by use of barricades and fencing and are stored in a relatively remote area. The steam generators (3) for Unit 2 are stored in the same area. The steam generators (3) for Unit 1 are stored adjacent to a busy construction

road. TVA is currently considering providing physical protection to the Unit 1 steam generators.

The reactor vessel heads for Units 1 and 2 are stored in the new Head Assembly Building. The RV heads were being prepared for having the control rod drive mechanisms (CRDM) welded to the CRDM nozzles on the head. Qualification tests of the weld procedure were in progress at the time of the inspection.

No nonconformance nor unresolved items were identified.

DETAILS III

Prepared by: R. W. Wright 5/6/76
R. W. Wright, Reactor Inspector Date
Engineering Support Section No. 1
Reactor Construction and
Engineering Support Branch

Dates of Inspection: April 20-23, 1976

Reviewed by: W. B. Smith Acting 5/6/76
S. D. Ebneter, Acting Section Chief Date
Engineering Support Section No. 1
Reactor Construction and
Engineering Support Branch

The details in this section apply to Unit 1 only unless noted otherwise.

1. Individuals Contacted

Tennessee Valley Authority (TVA)

a. Site

J. C. Killian - Project Manager
T. B. Northern, Jr. - Construction Engineer
A. R. White - General Construction Superintendent
R. L. Heatherly Supervisor, QC and Records Unit
J. C. Cofield - Supervisor, Materials Engineering Unit
H. S. Shepperd - Supervisor, Civil Engineering Unit
L. C. Northard - Supervisor, Site QA Unit, DEC Staff
J. E. Daniel - QA Engineer, DEC QA Staff, WB Unit
R. L. Young - QA Engineer, DEC QA Staff, WB Unit
C. O. Christopher - Civil Engineer, CEU
F. R. Gass - Concrete Placing Inspector, CEU
C. W. Willingham - Materials Testing Inspector, CEU
S. E. Brown - Batch Plant Inspector, CEU

b. Knoxville

R. F. Keck - OEDC - QA Staff
J. S. Colley - DED - QA Engineer

2. Scope

TVA apprised the IE inspectors at the management entrance interview of their investigative findings concerning an external inquiry into suspect deficient structual concrete repairs and practices at the Watts Bar site. Consequently, IE inspection priority was place on evaluation of the adequacy of the licensee's investigation. (Details I, paragraph 4)

The scheduled portion of this inspection was conducted to (a) observe a relatively large structual concrete wall placement to verify that proper quality control procedures were imposed in the preparation for the placement, throughout the placement itself, and for the protection and curing of the resulting concrete, and (b) to ascertain that containment structural concrete records reflect work accomplished consistent with applicable codes, SAR commitments and procedural requirements.

3. Observation of Work Activities

The IE inspector observed the preparation and partial placement of pour number RBI-E15 for Unit No. 1's reactor cavity and refuel canal wall. The above placement comprised approximately 170 cubic yards of 500.75 AFW mix (500.75 AFW-G for congested restricted areas), which was central mixed and delivered by bucket dump trucks. A pump and a 4 cubic yard bucket were employed to convey the mix to the placement area where controls were properly implemented limiting the concrete's free fall, layer depth and flow distance. The concrete pour card and inspector's report were found to be properly signed off and reflected the work performed. Observation revealed form work which was secure and clean, properly spaced reinforcement, proper vibration technique being applied and proper placement being accomplished. Visible supervision and QC inspection was evident during the placement. The exposed top of wall surface was scarified to receive a future lift and cured by ponding water and sheets of white plastic to protect against loss of moisture.

This inspector observed the testing of a batched mix for slump, air content, temperature, yield and the cylinder sampling as required by approved procedure. Calibration stickers examined revealed that all concrete test equipment used had current valid calibration dates. The placement and materials testing QC inspectors were interviewed for their familiarity with inspection requirements, specified testing frequency and QC records to be maintained.

The batch plant operation was observed and the facilities inspected for proper storage, segregation and protection of concrete ingredient materials.

No items of noncompliance were observed or identified in any of the above areas of inspection.

4. Concrete Record Review

The following records associated with concrete placements RBI-E7 (Reactor Shield Wall) and RBI-C6b (Polar Crane Wall) were examined to ascertain that the work was accomplished and documented in accordance with applicable codes, commitments and procedural requirements:

a. Applicable structural drawings

b. Concrete pour cards

c. Placement Inspectors Report

d. Concrete Mixing Plant Reports

e. In-process control testing and concrete cylinder frequency of testing and test results

f. Mixer efficiency tests

g. Concrete ingredient material records to include physical and chemical analysis of cement; fly ash testing for percent passing No. 325 sieve; specific gravity, absorption and aggregate gradations; testing of mixing water; and the physical testing of admixtures

h. Form removal and concrete curing records

i. Batch plant production and scale calibration records

j. Batch plant, laboratory and concrete placement QC inspector qualifications and training.

k. QA audits concerning splicing of rebar, laboratory and concrete placing activities.

l. Review of concrete related, "Conditions Adverse to Quality," reports

5. Item of Noncompliance (Units 1 and 2)

Within the above scope of records inspection, two examples of failure to follow procedure were identified by the IE inspector. Criterion V of Appendix B to 10 CFR 50 as implemented by commitments set forth in the PSAR, Appendix A, paragraph A.2.5, specifies, in part, that activities affecting quality shall be accomplished in accordance with instructions, procedures or drawings.

Contrary to the above requirements, the licensee's QC procedure WBNP QCP 2.2 RO, section 6.1.1.1 and section 6.1.4.2 was not complied with. Record review revealed that the bimonthly frequency for the chemical testing of water used in concrete production had been neglected for the months of November 1975 and March 1976. Similarly, the required rate of sampling cement for testing by Singleton Materials Laboratory has been remiss on four occasions between September 24, 1975 and January 9, 1976.

Failure to implement established procedures appears to be in noncompliance with Criterion V of 10 CFR 50, Appendix B. This item has been identified as an infraction level of noncompliance.