



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
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June 27, 2007

R. T. Ridenoure, Vice President  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
P.O. Box 550  
Fort Calhoun, NE 68023-0550

SUBJECT: ASSESSMENT FOLLOW-UP LETTER - FORT CALHOUN STATION

Dear Mr. Ridenoure:

On June 26, 2007, the NRC staff completed an update of our assessment of safety performance at the Fort Calhoun Station. The assessment evaluated performance indicators (PIs) and inspection results since January 2007. The purpose of this letter is to inform you of the results of our assessment and our plans to conduct a future supplemental inspection at your facility. This letter supplements, but does not supersede, our annual assessment letter issued on March 2, 2007, ADAMS (ML070610601). Consistent with the guidance in NRC Manual Chapter 0305, "Operating Reactor Assessment Program," this assessment follow-up letter is being issued because a low to moderate safety significant (White) inspection finding was issued and a PI exceeded the established performance threshold during the second quarter of 2007. As a result, Fort Calhoun Station's safety performance has been assessed to be in the Degraded Cornerstone Column of the NRC's Action Matrix.

Here is a more detailed discussion of the PI and inspection finding. On April 21, 2007, you submitted your first quarter of 2007 PI data to the NRC. You reported that the Safety System Functional Failure (SSFF) PI, in the Mitigating Systems Cornerstone, was at the Green-White threshold of five failures. Subsequent to the end of the first quarter, on April 17, 2007, you submitted Licensee Event Report (LER) 07-003-00 (ADAMS ML071140029) regarding the failure of an emergency diesel generator (EDG) with an inoperable containment cooling fan from the opposite bus. In addition, on June 11, 2007, you submitted LER 07-004-00 regarding the inadvertent isolation of all containment spray due to an inadequate test procedure. As a result of these two recent reportable failures the current number of safety system functional failures is seven, indicating this PI will be White (Regulatory Response Band) for the second quarter of 2007. With regard to safety significant inspection findings on May 29, 2007, we also issued the final significance determination for a finding that involved the improper installation of the valve disk of Containment Spray Header Isolation Valve HCV-345 (NRC EA-07-047; ADAMS ML071500074). That finding was determined to be White (low to moderate safety significance) and also affected the Mitigating Systems Cornerstone.

As a result of Fort Calhoun Station's performance being in the Degraded Cornerstone Column of the NRC's Action Matrix because there are two White inputs in the Mitigating Systems Cornerstone, we plan to implement Supplemental Inspection Procedure 95002,

"Inspection for One Degraded Cornerstone or Any Three White Inputs in a Strategic Performance Area." This inspection will be performed to provide assurance that the root and contributing causes for the individual and collective risk significant performance issues are understood, to independently assess the extent of condition and the extent of cause, to independently determine whether safety culture components caused or significantly contributed to the individual or collective risk significant performance issues, and to provide assurance that the corrective actions are sufficient to address the root and contributing causes, as well as to prevent recurrence. In accordance with Inspection Procedure 95002, we will not schedule this inspection until you have completed your problem identification, evaluation, and corrective action plans. Therefore, we request that you notify us in writing when you are prepared for this inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Please contact Jeff Clark at (817) 860-8147 with any questions you may have regarding this letter.

Sincerely,

/RA/

Bruce S. Mallett  
Regional Administrator

Docket: 50-285  
License: DRP-40

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| 6/4/2007      | 6/4/2007 | 6/7/2007  | 6/18/2007 | 6/27/2007  |

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