

June 27, 2007

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Limerick Generating Station, Units 1 and 2
Facility Operating License Nos. NPF-39 and NPF-85
NRC Docket Nos. 50-352 and 50-353

SUBJECT: License Amendment Request
Proposed Changes to Accident Monitoring Instrumentation Technical
Specifications

Pursuant to 10 CFR 50.90, "Application for amendment of license or construction permit," Exelon Generation Company, LLC (Exelon), proposes changes to the Technical Specifications (TS), Appendix A of Operating License Nos. NPF-39 and NPF-85 for Limerick Generating Station (LGS), Units 1 and 2, respectively.

The proposed changes relocate the operability and surveillance requirements for the drywell air temperature and suppression chamber air temperature instrumentation from LGS TS Section 3.3.7.5, "Accident Monitoring Instrumentation," to the LGS Technical Requirements Manual (TRM).

The proposed changes conform to 10 CFR 50.36 for the contents of TS, and to the improved Standard Technical Specifications approved by the NRC in NUREG-1433, "Standard Technical Specifications – General Electric Plants, BWR/4." A similar change, to relocate the operability and surveillance requirements for the safety/relief valve position indication instrumentation from TS Section 3.3.7.5 to the TRM, was approved by the NRC by issuance of Amendment Nos. 179 and 141 for LGS, Units 1 and 2, respectively, by letter dated September 27, 2005.

Exelon has concluded that the proposed changes present no significant hazards consideration under the standards set forth in 10CFR 50.92.

This amendment request contains no regulatory commitments.

Exelon requests approval of the proposed amendment by June 27, 2008. Upon NRC approval, the amendment shall be implemented within 60 days of issuance.

These proposed changes have been reviewed by the Plant Operations Review Committee.

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We are notifying the State of Pennsylvania of this application for changes to the Technical Specifications by transmitting a copy of this letter and its attachments to the designated State Official.

If you have any questions or require additional information, please contact Glenn Stewart at 610-765-5529.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 27th day of June, 2007.

Respectfully,



Pamela B. Cowan
Director - Licensing & Regulatory Affairs
Exelon Generation Company, LLC

- Attachments:
1. Evaluation of Proposed Changes
 2. Markup of Proposed Technical Specifications Pages

cc:	Regional Administrator - NRC Region I	w/ attachments
	NRC Senior Resident Inspector - Limerick Generating Station	"
	NRC Project Manager, NRR - Limerick Generating Station	"
	Director, Bureau of Radiation Protection - Pennsylvania Department of Environmental Protection	"

ATTACHMENT 1

License Amendment Request

Limerick Generating Station, Units 1 and 2

Docket Nos. 50-352 and 50-353

EVALUATION OF PROPOSED CHANGES

**Subject: Proposed Changes to Accident Monitoring Instrumentation
Technical Specifications**

- 1.0 DESCRIPTION**
- 2.0 PROPOSED CHANGES**
- 3.0 BACKGROUND**
- 4.0 TECHNICAL ANALYSIS**
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- 6.0 ENVIRONMENTAL CONSIDERATION**
- 7.0 REFERENCES**

1.0 DESCRIPTION

Pursuant to 10 CFR 50.90, "Application for amendment of license or construction permit," Exelon Generation Company, LLC (Exelon), proposes changes to the Technical Specifications (TS), Appendix A of Operating License Nos. NPF-39 and NPF-85 for Limerick Generating Station (LGS), Units 1 and 2, respectively.

The proposed changes relocate the operability and surveillance requirements for the drywell air temperature and suppression chamber air temperature instrumentation from the Accident Monitoring Instrumentation section of TS, i.e., from LGS TS 3.3.7.5 and 4.3.7.5 to the LGS Technical Requirements Manual (TRM). The TRM is incorporated by reference into the LGS Updated Final Safety Analysis Report (UFSAR) and is subject to the controls of 10 CFR 50.59. Accordingly, any future changes to the drywell air temperature and suppression chamber air temperature instrumentation operability and surveillance requirements will be performed pursuant to 10 CFR 50.59.

The proposed changes conform to 10 CFR 50.36 (Reference 1) for the contents of TS, and to the improved Standard Technical Specifications approved by the NRC in NUREG-1433, "Standard Technical Specifications – General Electric Plants, BWR/4" (Reference 2). A similar change, to relocate the operability and surveillance requirements for the safety/relief valve position indication instrumentation from TS Section 3.3.7.5 to the TRM, was approved by the NRC by issuance of Amendment Nos. 179 and 141 for LGS, Units 1 and 2, respectively (Reference 3).

2.0 PROPOSED CHANGES

The changes requested by this amendment application are described below.

1. Delete Item 5, "Suppression Chamber Air Temperature," from TS Table 3.3.7.5-1, "Accident Monitoring Instrumentation," on TS page 3/4 3-85. This table identifies the operability requirements for this instrumentation.
2. Delete Item 7, "Drywell Air Temperature," from TS Table 3.3.7.5-1, "Accident Monitoring Instrumentation," on TS page 3/4 3-85. This table identifies the operability requirements for this instrumentation.
3. Delete Item 5, "Suppression Chamber Air Temperature," from TS Table 4.3.7.5-1, "Accident Monitoring Instrumentation Surveillance Requirements," on TS page 3/4 3-87. This table identifies the surveillance requirements for this instrumentation, i.e., a channel check and channel calibration are specified. The frequency for performing these surveillance requirements is specified in the LGS Surveillance Frequency Control Program (SFCP), which will also be revised to delete the associated surveillance frequencies from the SFCP upon implementation of the approved amendment. The surveillance requirements from TS and the frequency for performing the surveillance requirements from the SFCP will be relocated to the LGS TRM.

4. Delete Item 7, "Drywell Air Temperature," from TS Table 4.3.7.5-1, "Accident Monitoring Instrumentation Surveillance Requirements," on TS page 3/4 3-87. This table identifies the surveillance requirements for this instrumentation, i.e., a channel check and channel calibration are specified. The frequency for performing these surveillance requirements is specified in the LGS SFCP, which will also be revised to delete the associated surveillance frequencies from the SFCP upon implementation of the approved amendment. The surveillance requirements from TS and the frequency for performing the surveillance requirements from the SFCP will be relocated to the LGS TRM.

Prior to implementation of the amendment, operability and surveillance requirements for the drywell air temperature and suppression chamber air temperature instrumentation will be incorporated into the LGS TRM. The operability requirements will identify compensatory measures and completion times for these instruments. Any subsequent changes to the TRM requirements will be performed in accordance with 10 CFR 50.59.

There are no changes to the TS Bases section proposed by this amendment application since the drywell air temperature and suppression chamber air temperature instrumentation is not specifically identified in the TS Bases.

3.0 BACKGROUND

The NRC provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (58 FR 39132, July 22, 1993; Reference 4). In particular, the NRC indicated that certain items could be relocated from the TS to licensee-controlled documents. The Final Policy Statement identified future criteria to be used in determining whether particular safety functions are required to be included in the TS, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of, or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of, or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. The NRC adopted amendments to 10 CFR 50.36 (60 FR 36953, July 19, 1995; Reference 5) to codify and incorporate these criteria.

The NRC's policy statement provides that those existing TS requirements which do not satisfy these four specified criteria may be relocated to licensee-controlled documents, such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Subsequently, the nuclear steam supply system owners' groups and the NRC staff developed improved standard technical specifications (STS) that would establish models of the Commission's policy for each primary reactor type. The NRC issued the improved STS for General Electric BWR/4 plants as NUREG-1433, which was developed utilizing the guidance and criteria in the Commission's policy statement.

4.0 TECHNICAL ANALYSIS

The proposed license amendment relocates the drywell air temperature and suppression chamber air temperature instrumentation operability and surveillance requirements from the LGS TS to the LGS TRM. The TRM is referenced in the LGS UFSAR and is subject to the controls of 10 CFR 50.59. The TRM has been used to capture and control other requirements associated with previous LGS license amendments.

As discussed in the Background section above, an NRC policy statement concluded that those existing TS requirements which do not satisfy the screening criteria specified in 10 CFR 50.36 may be deleted from the TS, and the requirements established in licensee-controlled documents that are subject to the controls of 10 CFR 50.59. The NRC position on application of the screening criteria to accident monitoring instrumentation is documented in correspondence dated May 9, 1988, T. E. Murley (NRC) to R. F. Janecek (BWR Owners' Group) (Reference 6). The NRC position is that Regulatory Guide 1.97 (Reference 7), Type A, and Category 1, accident monitoring instrumentation should be incorporated into the plant's TS. The requirements for those instruments not meeting these criteria may be removed from the TS and established in a licensee-controlled document subject to the controls of 10 CFR 50.59. Regulatory Guide 1.97, Revision 2, defines Type A instruments as those that monitor primary information required to permit the control room operator to take specific manually controlled actions for which no automatic control is provided and that are required for safety systems to accomplish their safety functions for design basis accident events. Category 1 instruments are designed for full qualification, redundancy, continuous real-time display, and onsite (standby) power.

Regulatory Guide 1.97, Revision 2, designates drywell air temperature instrumentation as Type D, Category 2, instrumentation. Type D instruments provide information to indicate the operation of individual safety systems and other systems important to safety. Category 2 instruments are designed to less stringent qualifications that do not require seismic qualification, redundancy, or continuous display, and require only a high reliability power source (not necessarily standby power). From a plant-specific perspective, LGS UFSAR Section 7.5 and UFSAR Table 7.5-3 also identify the drywell air temperature instrumentation as Type D, Category 2. Relocating drywell air temperature instrumentation from the TS to a licensee-controlled document conforms with this NRC position on application of the screening criteria to accident monitoring instrumentation.

Regulatory Guide 1.97, Revision 2, does not include suppression chamber air temperature as an accident monitoring variable. Likewise, the LGS UFSAR does not specifically identify suppression chamber air temperature by itself as a Regulatory Guide 1.97 variable. However, LGS UFSAR Section 7.5 does describe the use of suppression chamber air temperature for monitoring the effectiveness of suppression chamber spray flow (a Type D, Category 2 variable). Relocating suppression chamber air temperature instrumentation from the TS to a licensee-controlled document conforms with this NRC position on application of the screening criteria to accident monitoring instrumentation.

NUREG-1433 identifies improved TS that were developed based on the screening criteria in the "Final Commission Policy Statement on Technical Specifications Improvement for Nuclear Power Reactors," that were subsequently codified in 10 CFR 50.36. According to NUREG-1433, accident monitoring instrumentation that satisfies the definition of Type A in Regulatory Guide 1.97 meets Criterion 3 of 10 CFR 50.36(c)(2)(ii). Also, Category I, non-Type A instrumentation is retained in TS because they are intended to assist operators in minimizing the consequences of accidents. Therefore, Category I, non-Type A variables are important for reducing public risk. However, as stated previously, the LGS UFSAR indicates that the drywell air temperature instrumentation is Type D, Category 2 and suppression chamber air temperature is used for monitoring the effectiveness of suppression chamber spray flow (a Type D, Category 2 variable).

An assessment of the subject accident monitoring instrumentation against the four criterion of 10 CFR 50.36 is provided below.

Criterion 1: The drywell air temperature and suppression chamber air temperature instrumentation does not provide the primary information used to detect or indicate a significant abnormal degradation of the reactor coolant pressure boundary considered by Criterion 1. This is consistent with the Commission's Final Policy Statement which indicated that the first criterion was intended to assure that TS controlled those instruments specifically installed to detect reactor coolant leakage.

Criterion 2: The Commission's Final Policy Statement indicates that the basic concept in adequate protection of the public health and safety is that the plant be operated within the bounds of the initial conditions assumed in the existing design basis accident and transient analyses, and that the plant will be operated to preclude unanalyzed transients and accidents. Therefore, Criterion 2 applies to process variables that are parameters for which specific values or ranges of values have been chosen as reference bounds in the design basis accident or transient analyses, and which are monitored and controlled during normal power operation such that process values remain within the analysis bounds. According to Regulatory Guide 1.97, Revision 2, accident monitoring instrumentation is provided to monitor plant variables and systems during and following design basis accidents or transients. Therefore, Criterion 2 does not apply to accident monitoring instrumentation since accident monitoring instrumentation provides indication of plant variables required by operators during accident conditions to take preplanned manual actions to accomplish safety functions rather than instrumentation that provides indication of process variables that are monitored and controlled by operators during normal plant operations to ensure that the initial conditions for accident analyses are met. As such, the drywell air temperature and suppression chamber air temperature instrumentation does not provide the primary information used to monitor process variables, design features or operating restrictions that are an initial condition of a design basis accident or transient analysis considered in Criterion 2.

Criterion 3: Drywell air temperature and suppression chamber air temperature instrumentation does not initiate any automatic safety function. Drywell air temperature and suppression chamber air temperature instrumentation is not used as the primary information required to permit operators to take specific manually controlled actions for which no automatic control is provided, and that are required for safety systems to accomplish their safety functions for design

basis accident events. The drywell air temperature indication does indicate the performance of safety systems and other systems important to safety. As such, the drywell air temperature instrumentation is considered Type D, Category 2 (refer to Table 7.5-3 of the LGS UFSAR). Suppression chamber air temperature is used for monitoring the effectiveness of suppression chamber spray flow (a Type D, Category 2 variable). Hence, drywell air temperature and suppression chamber air temperature instrumentation is not part of the primary success path used to mitigate a design basis accident or transient involving a failure of or challenge to the integrity of a fission product barrier considered in Criterion 3.

Criterion 4: The loss of the subject accident monitoring instrumentation has negligible effect on the probabilistic safety assessment, and has not been shown to be significant to public health and safety as considered in Criterion 4.

Consequently, the drywell air temperature and suppression chamber air temperature instrumentation does not meet any of the screening criteria contained in the Commission's Final Policy Statement and 10 CFR 50.36. This conclusion is supported by the NRC screening criteria for accident monitoring instrumentation required to be in TS as documented in Reference 6 and NUREG-1433 which indicate that only Regulatory Guide 1.97, Type A and Category 1, non-Type A instrumentation as defined by plant-specific analyses are required to be listed in the Accident Monitoring Instrumentation section of TS. As indicated previously, LGS UFSAR Section 7.5 designates the drywell air temperature and suppression chamber air temperature instrumentation as other than Type A or Category 1, non-Type A instrumentation. Accordingly, the drywell air temperature and suppression chamber air temperature instrumentation requirements can be established in a licensee-controlled document. Future changes to drywell air temperature and suppression chamber air temperature instrumentation requirements will be subject to the controls of 10 CFR 50.59.

5.0 REGULATORY ANALYSIS

5.1 No Significant Hazards Consideration

Exelon has evaluated whether or not a significant hazards consideration is involved with the proposed amendment by focusing on the three standards set forth in 10 CFR 50.92, "Issuance of amendment," as discussed below:

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No. The failure of the drywell air temperature or suppression chamber air temperature instrumentation is not assumed to be an initiator of any analyzed event in the UFSAR. The proposed changes do not alter the physical design of this instrumentation or any other plant structure, system, or component. The proposed changes relocate the drywell air temperature and suppression chamber air temperature instrumentation operability and surveillance requirements from the Limerick Generating Station (LGS) Technical

Specifications (TS) to a licensee-controlled document under the control of 10 CFR 50.59.

The proposed changes conform to NRC regulatory requirements regarding the content of plant TS as identified in 10 CFR 50.36, and also the guidance as approved by the NRC in NUREG-1433, "Standard Technical Specifications-General Electric Plants, BWR/4."

Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No. The proposed changes relocate the drywell air temperature and suppression chamber air temperature instrumentation operability and surveillance requirements from the LGS TS to a licensee-controlled document under the control of 10 CFR 50.59. The proposed changes do not alter the physical design, safety limits, or safety analysis assumptions associated with the operation of the plant. Accordingly, the proposed changes do not introduce any new accident initiators, nor do they reduce or adversely affect the capabilities of any plant structure, system, or component in the performance of their safety function.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Do the proposed changes involve a significant reduction in a margin of safety?

Response: No. The subject instrumentation does not provide primary information required to permit operators to take specific manually controlled actions for which no automatic control is provided, and that are required for safety systems to accomplish their safety functions for design basis accident events. The instrumentation provides only drywell air temperature indication and suppression chamber air temperature indication, and does not provide an input to any automatic safety function. Operability and surveillance requirements will be established in a licensee-controlled document to ensure the reliability of drywell air temperature and suppression chamber air temperature instrumentation capability. Changes to these requirements will be subject to the controls of 10 CFR 50.59, providing the appropriate level of regulatory control.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based on the above, Exelon concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of “no significant hazards consideration” is justified.

5.2 Applicable Regulatory Requirements/Criteria

The NRC provided guidance for the contents of TS in its “Final Policy Statement on Technical Specifications Improvement for Nuclear Power Reactors” (58 FR 39132, July 22, 1993). In particular, the NRC indicated that certain items could be relocated from the TS to licensee-controlled documents, and identified criteria to be used to determine the functions to be included in the TS. The NRC adopted revisions to 10 CFR 50.36 to codify and incorporate these criteria. The NRC published in NUREG-1433 improved standard technical specifications indicating that Regulatory Guide 1.97, Type A and Category 1, non-Type A variables should be included in TS. The TS requirements proposed for relocation do not meet this criteria based on plant-specific analysis described in LGS UFSAR Section 7.5, and accordingly, are not required to be in the TS. Operability and surveillance requirements for the drywell air temperature and suppression chamber air temperature instrumentation will be maintained in the LGS TRM, subject to the controls of 10 CFR 50.59.

In conclusion, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission’s regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 ENVIRONMENTAL CONSIDERATION

A review has determined that the proposed amendment would change a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or would change an inspection or surveillance requirement. However, the proposed amendment does not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluent that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the proposed amendment.

7.0 REFERENCES

1. 10 CFR 50.36, “Technical Specifications.”
2. NUREG-1433, “Standard Technical Specifications-General Electric Plants, BWR/4,” Revision 3.1, dated December 1, 2005.

3. Letter dated September 27, 2005, from T. Tate (USNRC) to C. Crane (Exelon Nuclear), "Limerick Generating Station, Units 1 and 2 - Issuance of Amendments RE: Relocation of Operability and Surveillance Requirements for the Safety/Relief Valve Position Instrumentation (TAC Nos. MC3454 and MC3455)."
4. NRC "Final Policy Statement on Technical Specifications Improvement for Nuclear Power Reactors," 58 FR 39132, dated July 22, 1993.
5. NRC Final Rule, 10 CFR 50.36, "Technical Specifications," 60 FR 36953 (July 19, 1995).
6. Letter, T. E. Murley (USNRC) to R. F. Janecek (BWR Owners' Group), dated May 9, 1988.
7. Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2, dated December 1980.

ATTACHMENT 2

License Amendment Request

**Limerick Generating Station, Units 1 and 2
Docket Nos. 50-352 and 50-353**

**Proposed Changes to Accident Monitoring
Instrumentation Technical Specifications**

Markup of Proposed Technical Specifications Pages

Unit 1 TS Pages

3/4 3-85

3/4 3-87

Unit 2 TS Pages

3/4 3-85

3/4 3-87

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1,2	80
2. Reactor Vessel Water Level	2	1	1,2	80
3. Suppression Chamber Water Level	2	1	1,2	80
4. Suppression Chamber Water Temperature	8, 6 locations	6, 1/location	1,2	80
5. Suppression Chamber Air Temperature Deleted	1	1	1,2	80
6. Drywell Pressure	2	1	1,2	80
7. Drywell Air Temperature Deleted	1	1	1,2	80
8. Deleted				
9. Deleted				
10. Deleted				
11. Primary Containment Post-LOCA Radiation Monitors	4	2	1,2,3	81
12. North Stack Wide Range Accident Monitor**	3*	3*	1,2,3	81
13. Neutron Flux	2	1	1,2	80

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK(a)</u>	<u>CHANNEL CALIBRATION(a)</u>
1. Reactor Vessel Pressure		
2. Reactor Vessel Water Level		
3. Suppression Chamber Water Level		
4. Suppression Chamber Water Temperature		
5. Suppression Chamber Air Temperature Deleted		
6. Primary Containment Pressure		
7. Drywell Air Temperature Deleted		
8. Deleted		
9. Deleted		
10. Deleted		
11. Primary Containment Post LOCA Radiation Monitors		**
12. North Stack Wide Range Accident Monitor***		
13. Neutron Flux		

(a) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.

**CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/h and a one point calibration check of the detector below 10 R/h with an installed or portable gamma source.

***High range noble gas monitors.

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1,2	80
2. Reactor Vessel Water Level	2	1	1,2	80
3. Suppression Chamber Water Level	2	1	1,2	80
4. Suppression Chamber Water Temperature	8, 6 locations	6, 1/location	1,2	80
5. Suppression Chamber Air Temperature Deleted	1	1	1,2	80
6. Drywell Pressure	2	1	1,2	80
7. Drywell Air Temperature Deleted	1	1	1,2	80
8. Deleted				
9. Deleted				
10. Deleted				
11. Primary Containment Post-LOCA Radiation Monitors	4	2	1,2,3	81
12. North Stack Wide Range Accident Monitor**	3*	3*	1,2,3	81
13. Neutron Flux	2	1	1,2	80

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK (a)</u>	<u>CHANNEL CALIBRATION (a)</u>
1. Reactor Vessel Pressure		
2. Reactor Vessel Water Level		
3. Suppression Chamber Water Level		
4. Suppression Chamber Water Temperature		
5. Suppression Chamber Air Temperature Deleted		
6. Primary Containment Pressure		
7. Drywell Air Temperature Deleted		
8. Deleted		
9. Deleted		
10. Deleted		
11. Primary Containment Post LOCA Radiation Monitors		**
12. North Stack Wide Range Accident Monitor***		
13. Neutron Flux		

(a) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.

**CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/h and a one point calibration check of the detector below 10 R/h with an installed or portable gamma source.

***High range noble gas monitors.