

June 27, 2007

LICENSEE: Wolf Creek Nuclear Operating Corporation

FACILITY: Wolf Creek Generating Station, Unit 1

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON MAY 15, 2007, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND WOLF CREEK NUCLEAR OPERATING CORPORATION, CONCERNING REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE WOLF CREEK GENERATING STATION, UNIT 1, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Wolf Creek Nuclear Operating Corporation held a telephone conference call on May 15, 2007, to discuss and clarify the staff's requests for additional information (RAIs) concerning the Wolf Creek Generating Station, Unit 1, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's RAIs.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary. A mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-3703 or e-mail VMR1@nrc.gov.

/RA/

Verónica M. Rodríguez, Project Manager
License Renewal Branch B
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures:

1. List of Participants
2. List of Requests for Additional Information

cc w/encls: See next page

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TELEPHONE CONFERENCE CALL
WOLF CREEK GENERATING STATION, UNIT 1
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS
MAY 15, 2007

PARTICIPANTS

Devender Reddy

Billy Rogers

Yaira Diaz Sanabria

Evelyn Gettys

Lorrie Bell

Eric Blocher

AFFILIATIONS

U.S. Nuclear Regulatory Commission (NRC)

NRC

NRC

NRC

Wolf Creek Nuclear Operating Corporation

Strategic Teaming and Resource Sharing Alliance

REQUESTS FOR ADDITIONAL INFORMATION
WOLF CREEK GENERATING STATION, UNIT 1
LICENSE RENEWAL APPLICATION

MAY 15, 2007

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Wolf Creek Nuclear Operating Corporation held a telephone conference call on May 15, 2007, as a followup discussion and request for clarification to the applicant's response to the following request for additional information (RAI) concerning the Wolf Creek Generating Station (WCGS), Unit 1, license renewal application (LRA). The applicant's response to this RAI was submitted by letter dated May 2, 2007.

RAI 2.1-2

NRC Regulatory Guide (RG) 1.188, "Standard Format and Content for Applications to Renew Nuclear Power Plant Operating Licenses," Revision 1, dated September 2005, provides endorsement on the use of Nuclear Energy Institute (NEI) 95-10, "Industry Guidelines for Implementing the Requirements of 10 CFR Part 54 - The License Renewal Rule," Revision 6, dated June 2005. RG 1.188 indicates that NEI 95-10, Revision 6, provides methods that the staff considers acceptable for complying with the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 54 for preparing an LRA.

NEI 95-10, Appendix F, states in part, that nonsafety-related SSCs that are not directly connected to safety-related SSCs, or that are connected downstream of the first equivalent anchor, may be within the scope of license renewal if their failure could prevent the performance of the system safety function for which the safety-related SSC is required. NEI 95-10, Appendix F, describes two options for determining which nonsafety-related SSCs may be within the scope of license renewal. The applicant's methodology for scoping of nonsafety-related components affecting safety-related components is briefly described in LRA Section 2.1.2.2.

The staff requests that the applicant provide the following information related to its evaluation of the 10 CFR 54.4(a)(2) criteria:

- (1) Explain which option (mitigative or preventive) was used for nonsafety-related SSCs not directly connected to safety-related SSCs. In addition, describe the process for scoping portions of nonsafety-related systems in rooms or building, level, or areas (BLA) that contain safety-related components.
- (2) Define a "room" or BLA as used to determine the location of safety-related equipment. In addition, explain how portions of nonsafety-related systems were scoped for spatial interaction.
- (3) Discuss how interactions between adjacent rooms and/or BLAs were evaluated for the purposes of 10 CFR 54.4(a)(2). For example, describe how the effects of a pipe break in a room that may not contain a safety-related component were evaluated for interaction with an adjacent room that may contain a safety-related component.

Enclosure 2

- (4) During the audit, the applicant indicated that portions of piping systems containing an insignificant amount of liquid that would not typically be replenished, such as small isolated drain lines, were not considered to be fluid filled, and as such, were not included within the scope of 10 CFR 54.4(a)(2). Provide the technical justification and extent of condition for the exclusion of such portions of systems from the scope of license renewal.
- (5) Similarly, roof drain piping was also considered not to be filled with fluid. However, the applicant has identified that some roof drain piping passes through portions of the auxiliary, control, and diesel generator buildings where safety-related equipment may be located. During the audit, the applicant stated that the design and installation of roof drain piping precludes spatial interaction concerns. Provide the technical justification and extent of condition for the exclusion of the roof drain piping that passes through the portions of the above buildings where safety-related equipment is located.

Discussion: Based on the discussion with the applicant, the staff indicated that the response to Item (4) of this RAI requires clarification. The staff requested that the applicant clarify:

- (1) if the location of the drain pipes was verified,
- (2) if the location of the electrical equipment in the vicinity of the identified drain pipes which might be susceptible to leakage from the drain pipes was verified, and
- (3) how WCGS verified that the electrical equipment would not be affected by leakage from the drain pipes (e.g., is the equipment designed for a potentially wetted environment)

The applicant agreed to clarify and provide its response in writing.

Letter to Wolf Creek Nuclear Operating Corporation, from V. Rodriguez, dated June 27, 2007

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CREEK NUCLEAR OPERATING CORPORATION, CONCERNING REQUESTS
FOR ADDITIONAL INFORMATION PERTAINING TO THE WOLF CREEK
GENERATING STATION, UNIT 1, LICENSE RENEWAL APPLICATION

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