Facility: LaSalle Co. Station Date of Examination: November 2006
Examination Level (circle one): RO / SRO Operating Test Number: 2006-301

Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations	D S	Write an Out of Service
Conduct of Operations	D S	Interpret Core Thermal Limits
Equipment Control	N S	Perform Degraded Voltage Actions Per LOA-GRID-001
Radiation Control	N S	Initiate Surveillance LOS PC-M1, Primary Containment Integrity for Modes 1, 2, 3
Emergency Plan	N S	Determine Whether KI Should Be Issued to Emergency Workers.

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.

* Type Codes & Criteria:
(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes)
(N)ew or (M)odified from bank (≥ 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

Facility: LaSalle County Station — Date of Examination: November 13, 2006
Exam Level (circle one): RO / SRO-I / SRO-U — Operating Test No.: 2006-301

Control Room Systems@ (8 for RO; 7 for SRO-I; 2 or 3 for SRO-U, including 1 ESF)			
	System / JPM Title	Type Code*	Safety Function
a.	RMCS/Respond to Loss of RPIS Displays	A, N, S	1
b.	HPCS/Initiate HPCS Following a Failure of the Manual Initiation Push-Button	A, D, S	2
c.	Main Turbine/Conduct LOS-TG-M4, Turbine Bypass Valve Surveillance	D, S	3
d.	RR/Upshift Reactor Recirc Pumps	D, L, S	4
e.	RHR/Start one Loop of RHR and Establish Max. Suppression Pool Cooling with the Hard Card	A, D, S	5
f.	RWM/Rod Worth Minimizer/Perform RWM Operability Check for decreasing power below 10% in MODE 1	L, N, S	7
g.	CR HVAC/Shift CR HVAC from U2 to U1	A, N, S	9
h.	n/a	n/a	n/a
In-Plant Systems@ (3 for RO; 3 for SRO-I; 3 or 2 for SRO-U)			
i.	F/C / Respond to a Loss of Normal Level Control on the Fuel Pool	A, E, R	9
j.	DC/Place a Standby Battery Charger in Service	D	6
k.	SA/IA / Respond to Air Compressor Running Unloaded Due to Excessive Surge Condition.	E, R	8

@ All control room (and in-plant) systems must be different and serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

* Type Codes	Criteria for RO / SRO-I / SRO-U
(A)lternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)
(R)CA	≥ 1 / ≥ 1 / ≥ 1
(S)imulator	

Facility: LaSalle County Station | Date of Examination: November 13, 2006
Exam Level (circle one): RO / SRO-I / SRO-U | Operating Test No.: 2006-301

Control Room Systems@ (8 for RO; 7 for SRO-I; 2 or 3 for SRO-U, including 1 ESF)			
	System / JPM Title	Type Code*	Safety Function
a.	RMCS/Respond to Loss of RPIS Displays	A, N, S	1
b.	HPCS/Initiate HPCS Following a Failure of the Manual Initiation Push-Button	A, D, S	2
c.	Main Turbine/Conduct LOS-TG-M4, Turbine Bypass Valve Surveillance	D, S	3
d.	n/a		
e.	n/a		
f.	n/a		
g.	n/a		
h.	n/a		
In-Plant Systems@ (3 for RO; 3 for SRO-I; 3 or 2 for SRO-U)			
i.	F/C / Respond to a Loss of Normal Level Control on the Fuel Pool	A, E, R	9
j.	DC/Place a Standby Battery Charger in Service	D	6
k.	n/a		

@ All control room (and in-plant) systems must be different and serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

* Type Codes	Criteria for RO / SRO-I / SRO-U
(A)lternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)
(R)CA	≥ 1 / ≥ 1 / ≥ 1
(S)imulator	

ES-301 **Simulator Scenario Quality Checklist** **Form ES-301-4**

Facility: LaSalle County U1/U2 Date of Exam: 11/14/06 Scenario Numbers: 1 / 2 / 3 Operating Test No.: 2006301

QUALITATIVE ATTRIBUTES		Initials		
		a	b*	c#
1.	The initial conditions are realistic, in that some equipment and/or instrumentation may be out of service, but it does not cue the operators into expected events.	dr	na	dm
2.	The scenarios consist mostly of related events.	dr	na	dm
3.	Each event description consists of • the point in the scenario when it is to be initiated • the malfunction(s) that are entered to initiate the event • the symptoms/cues that will be visible to the crew • the expected operator actions (by shift position) • the event termination point (if applicable)	dr	na	dm
4.	No more than one non-mechanistic failure (e.g., pipe break) is incorporated into the scenario without a credible preceding incident such as a seismic event.	dr	na	dm
5.	The events are valid with regard to physics and thermodynamics.	dr	na	dm
6.	Sequencing and timing of events is reasonable, and allows the examination team to obtain complete evaluation results commensurate with the scenario objectives.	dr	na	dm
7.	If time compression techniques are used, the scenario summary clearly so indicates. Operators have sufficient time to carry out expected activities without undue time constraints. Cues are given.	dr	na	dm
8.	The simulator modeling is not altered.	dr	na	dm
9.	The scenarios have been validated. Pursuant to 10 CFR 55.46(d), any open simulator performance deficiencies or deviations from the referenced plant have been evaluated to ensure that functional fidelity is maintained while running the planned scenarios.	dr	na	dm
10.	Every operator will be evaluated using at least one new or significantly modified scenario. All other scenarios have been altered in accordance with Section D.5 of ES-301.	dr	na	dm
11.	All individual operator competencies can be evaluated, as verified using Form ES-301-6 (submit the form along with the simulator scenarios).	dr	na	dm
12.	Each applicant will be significantly involved in the minimum number of transients and events specified on Form ES-301-5 (submit the form with the simulator scenarios).	dr	na	dm
13.	The level of difficulty is appropriate to support licensing decisions for each crew position.	dr	na	dm

Target Quantitative Attributes (Per Scenario; See Section D.5.d)		Actual Attributes	--	--	--
1.	Total malfunctions (5–8)	5 / 5 / 6	dr	na	dm
2.	Malfunctions after EOP entry (1–2)	2 / 2 / 2	dr	na	dm
3.	Abnormal events (2–4)	3 / 3 / 4	dr	na	dm
4.	Major transients (1–2)	1 / 1 / 1	dr	na	dm
5.	EOPs entered/requiring substantive actions (1–2)	1 / 1 / 2	dr	na	dm
6.	EOP contingencies requiring substantive actions (0–2)	1 / 1 / 1	dr	na	dm
7.	Critical tasks (2–3)	2 / 2 / 3	dr	na	dm

Facility: LaSalle Co Station Date of Examination: Nov 14, 2006 Operating Test No.:2006301

Competencies	APPLICANTS								
	RO ☐ SRO-I ☒ SRO-☐			RO ☐ SRO-I ☒ SRO-☐			RO ☐ SRO-☐ SRO-U ☒		
	SCENARIO			SCENARIO			SCENARIO		
	1	2	3	1	2	3	1	2	3
Interpret/Diagnose Events and Conditions	1.3.4 5.6	2.4.6 7.8	1.2.4 5.6.8	1,3,4, 5,6	2.4.6 7.8	1.2.4 5.6.8	1,3,4, 5,6	2.4.6 7.8	n/a
Comply With and Use Procedures (1)	All	All	All	All	All	All	All	All	n/a
Operate Control Boards (2)	2.3.4	1.3.4 6.8	1.2.4 5.7	2,3,4	1.3.4 6.8	1.2.4 5.7	2,3,4	1.3.4 6.8	n/a
Communicate and Interact	1.5.7	1.3.4 6.7	1.2.4 6.7	1,5,7	1.3.4 6.7	1.2.4 6.7	1,5,7	1.3.4 6.7	n/a
Demonstrate Supervisory Ability (3)	2.3.4 5.8	1.2.3 4.6	2.4.6 7.8	2,3,4 5,8	1.2.3 4.6	2.4.6 7.8	2,3,45 ,8	1.2.3 4.6	n/a
Comply With and Use Tech. Specs. (3)	1.3.4	2.4	1.2.3	1,3,4	2.4	1.2.3	1,3,4	2.4	n/a

Notes:
(1) Includes Technical Specification compliance for an RO.
(2) Optional for an SRO-U.
(3) Only applicable to SROs.

Instructions:

Check the applicants' license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

Facility: LaSalle Date of Exam: November 2006

Tier	Group	RO K/A Category Points											SRO-Only Points		
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A2	G*	Total
1. Emergency & Abnormal Plant Evolutions	1	4	4	3	N/A			2	3	N/A		4	20	3	4	7
	2	1	1	1				2	1			1	7	2	1	3
	Tier Totals	5	5	4				4	4			5	27	6	4	10
2. Plant Systems	1	3	2	3	3	2	2	2	2	2	2	3	26	3	2	5
	2	1	0	2	1	2	1	1	1	1	1	1	12	2	1	3
	Tier Totals	4	2	5	4	4	3	3	3	3	3	4	38	5	3	8
3. Generic Knowledge and Abilities Categories		1		2		3		4					10	1	2	3	4	7
		2		3		2		3						2	2	1	2	

Note:

1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).
2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements.
4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.

e. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.

6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.

7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.

8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401	BWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO)							Form ES-401-1	
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	03						Knowledge of the operational implications on thermal limits as it applies to	3.6	1/0
295003 Partial or Complete Loss of AC / 6					02		Ability to determine and/or interpret reactor power/pressure/and level as they apply to ...	4.2	1/0
295004 Partial or Total Loss of DC Pwr / 6						X	2.3.11 - Ability to control radiation releases.	2.7	1/0
295005 Main Turbine Generator Trip / 3			06		01		K3.06 – Knowledge of the reasons for realignment of electrical distribution response as it applies to ... A2.01(SRO Only) – Ability to determine and/or interpret turbine speed as it applies to ...	3.3 2.7	1/1
295006 SCRAM / 1				05		X	A1.05 – Ability to operate and/or monitor the Neutron Monitoring System it applies to ... 2.4.27(SRO Only) – Knowledge of fire in the plant procedures.	4.2 3.5	1/1
295016 Control Room Abandonment / 7		01					Knowledge of the interrelations between ..... and the Remote Shutdown Panel.	4.4	1/0
295018 Partial or Total Loss of CCW / 8		02					Knowledge of the interrelations between .... and plant operatons.	3.4	1/0
295019 Partial or Total Loss of Inst. Air / 8			02				Knowledge of the reasons for standby air compressor response as it applies to	3.5	1/0
295021 Loss of Shutdown Cooling / 4				01			Ability to operate and/or monitor the RWCU system as it applies to ...	3.4	1/0
295023 Refueling Acc / 8					01		Ability to determine and/or interpret area radiation levels as they apply to	3.6	1/0
295024 High Drywell Pressure / 5						X	2.4.11 – Knowledge of abnormal condition procedures.	3.4	1/0
295025 High Reactor Pressure / 3					06 / 04		A2.06 – Ability to determine and/or interpret reactor water level as it applies to .... A2.04(SRO Only) – Ability to determine and/or interpret suppression pool level as it applies to	3.7 3.9	1/1
295026 Suppression Pool High Water Temp. / 5						X	2.4.3 - Ability to identify post-accident instrumentation. 2.4.14(SRO Only) – Knowledge of general guidelines for EOP flowchart use.	3.5 3.9	1/1
295027 High Containment Temperature / 5							SUPPRESSED – LaSalle has a Mark II Containment		
295028 High Drywell Temperature / 5					04	X	2.1.1 – Knowledge of the conduct of operations. A2.04(SRO Only) – Ability to determine and/or interpret drywell pressure as it applies to	3.7 4.2	1/1

ES-401	BWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO)							Form ES-401-1	
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295030 Low Suppression Pool Wtr Lvl / 5	02					X	K1.02 – Knowledge of the operational implications on pump NPSH as it applies to .... 2.1.12(SRO Only) – Ability to apply technical specifications for a system.	3.5 4.0	1/1
295031 Reactor Low Water Level / 2	01	01				X	K1.01 – Knowledge of the operational implications on adequate core cooling as it applies to .... K2.01 – Knowledge of the interrelations between .... and reactor water level indication. 2.1.25(SRO Only) – Ability to obtain and interpret station reference material such as graphs/ monographs/and tables which contain performance data.	4.6 4.4 3.1	2/1
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1	05						Knowledge of the operational implications of cold shutdown boron weight as it applies to	3.4	1/0
295038 High Off-site Release Rate / 9		03					Knowledge of the interrelations between .... and the Plant VentilationSystems.	3.6	1/0
600000 Plant Fire On Site / 8			04				Knowledge of the reasons for the actions contained in the abnormal procedure for	2.8	1/0
K/A Category Totals:	4	4	3	2	3 / 3	4 / 4	Group Point Total:		20/7

ES-401	BWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (RO / SRO)							Form ES-401-1	
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									
295007 High Reactor Pressure / 3									
295008 High Reactor Water Level / 2									
295009 Low Reactor Water Level / 2									
295010 High Drywell Pressure / 5									
295011 High Containment Temp / 5							SUPPRESSED – LaSalle has a Mark II Containment		
295012 High Drywell Temperature / 5				02			Ability to operate and/or monitor the Drywell Cooling system as it applies to	3.8	1/0
295013 High Suppression Pool Temp. / 5					01		Ability to determine and/or interpret suppression pool temperature as it applies to :....	3.8	1/0
295014 Inadvertent Reactivity Addition / 1						X	2.2.33 – Knowledge of control rod programming.	2.5	1/0
295015 Incomplete SCRAM / 1					02		A2.02(SRO Only) – Ability to determine and/or interpret control rod position as it applies to	4.2	0/1
295017 High Off-site Release Rate / 9						X	2.4.6(SRO Only) – Knowledge of symptom-based EOP mitigation stategies.	4.0	0/1
295020 Inadvertent Cont. Isolation / 5 & 7	05						Knowledge of the operational implications of a loss of dywell/containment cooling as it applies to	3.3	1/0
295022 Loss of CRD Pumps / 1		07					Knowledge of the interrelations between .... and reactor pressure(SCRAM assist)	3.4	1/0
295029 High Suppression Pool Wtr Lvl / 5			01				Knowledge of the reasons for emergency depressurization as it applies to	3.5	1/0
295032 High Secondary Containment Area Temperature / 5				04			Ability to operate and/or monitor the fire protection system as it applies to	3.3	1/0
295033 High Secondary Containment Area Radiation Levels / 9									
295034 Secondary Containment Ventilation High Radiation / 9									
295035 Secondary Containment High Differential Pressure / 5					01		A2.01(SRO Only) – Ability to determine and/or interpret secondary containment pressure as it applies to	3.9	0/1
295036 Secondary Containment High Sump/Area Water Level / 5									
500000 High CTMT Hydrogen Conc. / 5									
K/A Category Point Totals:	1	1	1	2	1 / 2	1 / 1	Group Point Total:		7/3

ES-401	BWR Examination Outline Plant Systems - Tier 2/Group 1 (RO / SRO)												Form ES-401-1	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection Mode				07						03		K4.07 – Knowledge of .... design feature(s) and/or interlocks which provide for emergency generator load sequencing A4.03 – Ability to manually operate and/or monitor the keep fill system.	3.5 3.4	2/0
205000 Shutdown Cooling			01					07			X	K3.01 – Knowledge of the effect that a loss or malfunction of the .... will have on reactor pressure. 2.2.1 – Ability to perform pre-startup procedures including those controls that could affect reactivity. A2.07(SRO Only) - Ability to predict the impacts of a loss of motor cooling on the ....; and use procedures to correct,control, or mitigate the consequences.	3.3 3.7 2.7	2/0
206000 HPCI												SUPPRESSED – LaSalle does not have a HPCI system.		
207000 Isolation (Emergency) Condenser												SUPPRESSED – LaSalle does not have an Isolation Condenser		
209001 LPCS				08				05				K4.08 – Knowledge of .... design feature(s) and/or interlocks which provide for automatic system initiation. A2.05(SRO Only) – Ability to predict the impacts of a core spray line break on the ....; and use procedures to correct,control, or mitigate the consequences.	3.8 3.2	1/1
209002 HPCS					01							Knowledge of the operational implications of an indication of pump cavitation as it applies to	2.5	1/0
211000 SLC						01						Knowledge of the effect that a loss or malfunction of the plant air systems will have on the .... Plant Obj. # 028.00.16	2.4	1/0
212000 RPS							03					Ability to predict and/or monitor changes in RPS motor-generator output frequency. Plant Obj. # 049.00.05/06	2.4	1/0
215003 IRM								05				Ability to predict the impacts of faulty or erratic operation of detectors/system on the ....; and use procedures to correct,control, or mitigate the consequences.	3.3	1/0
215004 Source Range Monitor									02			Ability to monitor automatic operation of the .... including annunciator and alarm signals.	3.4	1/0
215005 APRM / LPRM										06		Ability to manually operate and/or monitor verification of proper functioning/operability.	3.6	1/0
217000 RCIC											X	2.1.30 – Ability to locate and operate components/including local controls.	3.9	1/0

ES-401	BWR Examination Outline Plant Systems - Tier 2/Group 1 (RO / SRO)												Form ES-401-1	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
218000 ADS	04											K1.04 – Knowledge of the physical connections and/or cause-effect relationships between .... and drywell/containment pressure.	3.9	1/1
223002 PCIS/Nuclear Steam Supply Shutoff		01									X	K2.01 – Knowledge of the electrical power supplies to the logic power supplies. Plant Obj # 091.00.16 2.4.16(SRO Only) – Knowledge of EOP implementation hierarchy and coordination with other support procedures..	2.4 4.0	1/1
239002 SRVs	08		01									K1.08 – Knowledge of the physical connections and/or cause-effect relationships between .... and ADS. K3.01 – Knowledge of the effect that a loss or malfunction of the .... will have on reactor pressure control.	4.0 3.9	2/0
259002 Reactor Water Level Control		01										Knowledge of the electrical power supplies to reactor water level control system circuits Plant Obj # 078.00.16	2.4	1/0
261000 SGTS			05									Knowledge of the effect that a loss or malfunction of .... will have on secondary containment radiation/ contamination levels.	3.2	1/0
262001 AC Electrical Distribution				01								Knowledge of .... design feature(s) and/or interlocks which provide for bus lockouts.	3.0	1/0
262002 UPS (AC/DC)						02						Knowledge of the effect that a loss or malfunction of D.C. power will have on....	2.8	1/0
263000 DC Electrical Distribution					01						X	K5.01 – Knowledge of the operational implications of Hydrogen generation during battery charging as it applies to .... 2.4.26 – Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.	2.6 2.9	2/0
264000 EDGs							03	01				A1.03 Ability to predict and/or monitor changes in operating parameters. A2.01(SRO Only) – Ability to predict the impacts of parallel generator operation on the ....; and use procedures to correct, control, or mitigate the consequences.	2.8 3.5	1/1
300000 Instrument Air								01			X	A2.01 – Ability to predict the impacts of air dryer and filter malfunctions on the ....; and use procedures to correct,control, or mitigate the consequences. 2.4.8(SRO Only) – Knowledge of how event-based emergency/abnormal operating procedures are used in conjunction with symptom-based EOPs.	2.9 3.7	1/1

ES-401	BWR Examination Outline Plant Systems - Tier 2/Group 1 (RO / SRO)												Form ES-401-1	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
400000 Component Cooling Water	0 2								0 1			K1.02 – Knowledge of the physical connections and/or cause-effect relationships between .... and loads cooled. A3.01 – Ability to monitor automatic operation of the .... including setpoints on instrument signal levels for normal ops, warnings, and trips that are applicable to	3.2 3.0	2/0
K/A Category Point Totals:	3	2	3	3	2	2	2	2 / 3	2	2	3 / 2	Group Point Total:		26/5

ES-401	BWR Examination Outline Plant Systems - Tier 2/Group 2 (RO / SRO)												Form ES-401-1	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
201001 CRD Hydraulic														
201002 RMCS														
201003 Control Rod and Drive Mechanism														
201004 RSCS												SUPPRESSED – RSCS has been abandoned in place at LaSalle.		
201005 RCIS												SUPPRESSED – LaSalle does not have RCIS.		
201006 RWM														
202001 Recirculation														
202002 Recirculation Flow Control														
204000 RWCU														
214000 RPIS					01							Knowledge of the operational implications of reed switches as they applies to	2.7	1/0
215001 Traversing In-core Probe			01									Knowledge of the effect that a loss or malfunction of the .... will have on Local Power Range Monitor Calibration Plant Obj. # 046.00.18	2.3	1/0
215002 RBM				03								Knowledge of .... design feature(s) and/or interlocks which provide for initiation point (30%)	2.9	1/0
216000 Nuclear Boiler Inst.					10							Knowledge of the operational implications of vessel level measurement as it applies to	3.1	1/0
219000 RHR/LPCI: Torus/Pool Cooling Mode														
223001 Primary CTMT and Aux.							02					Ability to predict and/or monitor changes in Drywell pressure associated with operation of	3.6	1/0
226001 RHR/LPCI: CTMT Spray Mode								10				Ability to predict the impacts of Nuclear Boiler Instrument failures on the ....; and use procedures to correct,control, or mitigate the consequences.	3.0	1/0
230000 RHR/LPCI: Torus/Pool Spray Mode									01			Ability to monitor automatic operation of the .... including valve operation.	3.4	1/0
233000 Fuel Pool Cooling/Cleanup										10		Ability to manually operate and/or monitor tank levels.	2.5	1/0
234000 Fuel Handling Equipment											X	2.2.26(SRO Only) – Knowledge of refueling administrative requirements.	3.7	0/1

ES-401	BWR Examination Outline Plant Systems - Tier 2/Group 2 (RO / SRO)												Form ES-401-1	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
239001 Main and Reheat Steam						01		01				K6.01 - Knowledge of the effect that a loss or malfunction of electrical power will have on.... A2.01(SRO Only) – Ability to predict the impacts of a malfunction of reactor turbine pressure regulating system on the ....; and use procedures to correct,control, or mitigate the consequences.	3.1	0/1
239003 MSIV Leakage Control												SUPPRESSED – MSIV Leakage Control abandoned in place at LaSalle.		
241000 Reactor/Turbine Pressure Regulator											X	2.1.7 – Ability to evaluate plant performance and make operational judgements based on operating characteristics/reactor behavior/and instrument interpretation.	3.7	1/0
245000 Main Turbine Gen. / Aux.	06											Knowledge of the physical connections and/or cause-effect relationships between .... and component cooling water system.	2.6	1/0
256000 Reactor Condensate			02									Knowledge of the effect that a loss or malfunction of the .... will have on CRD Hydraulic system.	3.2	1/0
259001 Reactor Feedwater								09				A2.09(SRO Only) – Ability to predict the impacts of TDRFP steam inlet pressure flow on the ....; and use procedures to correct,control, or mitigate the consequences.	2.6	0/1
268000 Radwaste														
271000 Offgas														
272000 Radiation Monitoring														
286000 Fire Protection														
288000 Plant Ventilation														
290001 Secondary CTMT														
290003 Control Room HVAC														
290002 Reactor Vessel Internals														
K/A Category Point Totals:	1	0	2	1	2	1	1	1/2	1	1	1/1	Group Point Total:		12/3

Operating Test Comments:

Admin JPM a - Before validation it was determined that a more common practice for operators would be to verify a tagout. The JPM was replaced with an Administrative JPM to verify a tagout.

Admin JPM c - Upon validation it was determined that the JPM had been used during one of the applicant's certification examinations and was replaced.

Admin JPM d - Upon validation it was determined that the JPM did not provide an adequate evaluation tool and was replaced.

Control Room JPM c Before validation it was determined that the actions for this JPM took place outside the control room at LaSalle Co. Station. The JPM was replaced.

Control Room JPM d Before validation it was determined that the actions required to complete this JPM were lengthy, causing the JPM to take >1 hour to complete. The JPM was replaced.

Control Room JPM e Before validation it was determined that the actions required to complete this JPM were lengthy and contained a significant number of NLO activities that did not contribute to applicant evaluation. The JPM was replaced.

Control Room JPM g Before validation it was determined that the actions required to complete the JPM were mostly conducted by in-plant operators. The JPM was replaced.

In-Plant JPM i - Upon validation it was determined that some of the controls were in contaminated or high radiation areas. Use of the JPM was not in accordance with the station's ALARA program. The JPM was replaced.

Scenario 1, Event 5 During validation it was determined that the full core display was already lost due to the power failure associated with the scenario. The event was replaced with a new event 4, Loss of Hydrogen Seal Oil.

Scenario 1, Event 6 During validation it was determined that for purposes of evaluation the failed Divisional Emergency Diesel Generators would be changed. The Division 2 EDG would be restored and the Division 1 EDG would fail. This would allow lighting to be restored in the control room.

Scenario 2, IC During validation it was determined that the plant configuration could not be as outlined in the initial conditions if station procedures were followed. Changes were made to the initial conditions to make the simulator conditions conform to what would the plant configuration would be during a normal plant shutdown.

Scenario 2, Event 2 Deleted because the plant's power level would already be below the setpoint for this event to occur.

Scenario 2, Event 3 Because the TDRFPs were already secured per procedure, the normal evolution was changed to complete the next step in the shutdown procedure - shift electrical busses to their normal shutdown lineup.

Scenario 2, Event 4 New event 4 added to replace deleted event 2.

Scenario 2, Event 5 Modified to obtain the desired loss of feedwater per the original scenario. With the

	change in initial conditions, the MDRFP was running, hence, the loss of oil and ultimate trip of the MDRFP.
Scenario 2, Event 7	Modified because TCVs would shut automatically with lowering reactor pressure and desired overcooling would not occur. Stuck open TCV was added to obtain overcooling event.
Scenario 3, Event 6	Modified due to the low probability of both pump's seals failing concurrently. A small break LOCA was also inserted to assist the pressure rise in the drywell.