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JUN 07 2007

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Station OP1-17 Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION PROPOSED AMENDMENT NO. 291 TO LICENSE NPF-14 AND PROPOSED AMENDMENT NO. 260 TO LICENSE NPF-22: APPLICATION TO ELIMINATE LICENSE CONDITION 2.G REQUIRING REPORTING OF VIOLATIONS OF SECTIONS 2.C AND 2.F OF OPERATING LICENSE NO. NPF-14 AND TO ELIMINATE LICENSE CONDITION 2.E REQUIRING REPORTING OF VIOLATIONS OF SECTION 2.C OF OPERATING LICENSE NO. NPF-22 USING THE CONSOLIDATED LINE ITEM IMPROVEMENT PROCESS PLA-6135

Docket Nos. 50-387 and 50-388

In accordance with the provisions of 10 CFR 50.90, PPL Susquehanna, LLC is submitting a request for an amendment to the Operating Licenses for Susquehanna Steam Electric Station Units 1 and 2.

The proposed amendment would delete the license condition that requires reporting of violations of other requirements (e.g., conditions listed in Sections 2.C and 2.F for Unit 1 and Section 2.C for Unit 2) in the operating licenses. The change is consistent with the notice published in the *Federal Register* on November 4, 2005 as part of the consolidated line item improvement process (CLIIP).

These proposed changes have been reviewed by both the Plant Operations Review Committee and the Susquehanna Review Committee.

Attachment 1 provides a description of the proposed change and confirmation of applicability to Susquehanna SES. Attachment 2 provides the existing Operating License pages marked-up to show the proposed change. No new regulatory commitments are made in this submittal.

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PPL Susquehanna, LLC requests approval of the proposed License Amendment by November 1, 2007 with the amendment being implemented within 30 days following approval.

In accordance with 10 CFR 50.91(b), PPL Susquehanna, LLC is providing the Commonwealth of Pennsylvania with a copy of this proposed License Amendment request.

If you have any questions regarding this submittal, please contact Mr. Cornelius T. Coddington at (610) 774-4019.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on:

C. J. Gennon

Attachments:

Attachment 1 – Description and Assessment of the Proposed Change Attachment 2 - Proposed Operating License Changes Units 1 & 2, (Mark-ups)

cc: NRC Region I

Mr. A. Blamey, NRC Sr. Resident Inspector Mr. R. V. Guzman, NRC Sr. Project Manager Mr. R. Janati, DEP/BRP

Attachment 1 to PLA-6135

Description and Assessment of the Proposed Change

DESCRIPTION AND ASSESSMENT

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1.0 **DESCRIPTION**

The proposed amendments would delete Section 2.G of Facility Operating License NPF-14 for Susquehanna Steam Electric Station Unit 1, which requires reporting of violations of the requirements in Sections 2.C and 2.F of the facility operating license. The amendments would also delete Section 2.E of Facility Operating License NPF-22 for Susquehanna Steam Electric Station Unit 2, which requires reporting of violations of the requirements in Section 2.C of the facility operating license. The availability of these operating license improvements was announced in the *Federal Register* on November 4, 2005 as part of the consolidated line item improvement process (CLIIP).

2.0 DESCRIPTION OF PROPOSED AMENDMENT

Consistent with the CLIIP Notice of Availability (70 FR 67202), the proposed amendments consist of deleting Section 2.G of Facility Operating License NPF-14 for Unit 1 and Section 2.E of facility Operating License NPF-22 for Unit 2. The current requirements of the Unit 1 license condition are as follows:

- G. PPL Susquehanna, LLC shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through C(32), and F of this license within twenty-four (24) hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written follow-up in accordance with the procedures described in 10CFR50.73(b),(c), and (e).
 - (b) PPL Susquehanna, LLC shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

The Unit 1 existing conditions in Section 2.C that are subject to the current reporting requirement consist of the following:

(1) <u>Maximum Power Level</u>

PPL Susquehanna, LLC is authorized to operate the facility at reactor core power levels not in excess of 3489 megawatts thermal in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

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(3) Conduct of Work Activities During Fuel Load and Initial Startup

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The operating licensee shall review by committee all facility construction, Preoperational Testing, and System Demonstration activities performed concurrently with facility initial fuel loading or with the facility Startup Test Program to assure that the activity will not affect the safe performance of the facility fuel loading or the portion of the facility Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the facility fuel loading or the portion of the facility Startup Program being performed. The committee for the review shall be composed of a least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Assistant Superintendent of Plant's staff.

(4) <u>Thermal and Hydraulic Design (Section 4.4, SER)</u>

- (a) PPL Susquehanna, LLC is prohibited from power operation under natural circulation conditions.
- (b) Deleted per Amendment 42.
- (5) Qualification of Purge Valves (Section 6.2.4, SSER#1; 22, SSER#4)

Whenever the operational condition is other than cold shutdown or refueling, the operating licensee shall maintain each containment purge and vent isolation valve greater than 2-in. nominal diameter in one of the following conditions:

- (a) Closed and electrically prohibited from opening,
- (b) Blocked so as not to permit opening by more than 50 degrees, or
- (c) Operated to permit opening by more than 50 degrees after demonstrating that the valves are qualified to close from the full open position against peak LOCA pressure, and are also qualified per the criteria of Branch Technical Position CSB 6-4. Purge valve qualification documentation must be approved by the NRC prior to operating valves in this mode.

(6) Fire Protection Program (Section 9.5, SER, SSER#1, SSER#2, SSER#3)

PPL Susquehanna, LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Fire Protection Review Report for the facility and as approved in Fire Protection Program, Section 9.5, SER, SSER#1, SSER#2, SSER#3, SSER#4, SSER#6, Safety Evaluation of Fire Protection dated August 9, 1989, Safety Evaluation of Revision 4 to the Fire Protection Report dated March 29, 1993, Safety Evaluation of Fire Protection Program Issues, Safe Shutdown Methodology and Analysis of Associated Circuits dated October 21, 1997, and Safety Evaluation of the licensee's Amendment No. 177, dated June 24, 1998, to relocate the Fire Protection Program subject to the following provision:

The operating licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(7) Battery Room Area (Section 9.5.4, SER, SSER#1, SSER#3)

Prior to exceeding five percent of full power and subject to NRC review and approval, the operating licensee shall either conduct at an approved testing laboratory an ASTM E-119 test of the as-installed one-hour cable wrap configuration or install an automatic fire extinguishing system.

(8) Operation with Partial Feedwater Heating at End-of-Cycle (Section 15.1, SER, SSER# 1)

Prior to operation with partial feedwater heating, PPL Susquehanna, LLC shall provide for NRC review and approval, analyses which show a more limiting change does not occur in the minimum critical power ratio than that obtained using normal feedwater heating.

(9) Initial Test Program (Section 14, SER, SSER#1)

The operating licensee shall conduct the post-fuel-loading initial test program (set forth in Section 14 of the licensee's Final Safety Analysis Report, as amended through Amendment 50 and modified by the operating licensee's letter dated August 26, 1982, (PLA-1257)) without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- (a) Elimination of any test identified as essential in Section 14 of the licensees' Final Safety Analysis Report, as amended through Amendment 50 and modified by the operating licensee's letter dated August 26, 1982, (PLA-1257);
- (b) Modifications of test objectives, methods or acceptance criteria for any test identified as essential in Section 14 of the licensee's Final Safety Analysis Report, as amended through Amendment 50 and modified by the operating licensee's letter dated August 26, 1982, (PLA-1257);
- (c) Performance of any test at a power level different from that described in the program; and
- (d) Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).
- (10) Inservice Inspection Program (Section 5.2.4 and 6.6, SER, SSER#1, SSER#3)

By June 30, 1983, the operating licensee shall submit a revised inservice inspection program for NRC review and approval.

(11) Seismic System Analysis (Section 3.7.2, SSER#3)

By the dates indicated, the operating licensee shall provide documentation to the NRC for review which states the results of recheck of all calculations associated with calculating masses, section properties, and spring stiffnesses used in stick models for the following structures:

(a)	Containment	July 30, 1982
(b)	Reactor/Control Structure (Vertical model)	August 25, 1982
(c)	Diesel Generator Building	August 25, 1982
(d)	Engineering Safeguard Service Water Pumphouse	August 25, 1982

(12) Radon (ASLB Initial Decision, Paragraph 223)

This license will be subject to the ultimate outcome of the consolidated radon proceeding currently underway before the Appeal Boards in Docket Nos. 50-277, 50-278, 50-320, 50-354 and 50-355.

(13) Nearby Facilities (Section 2.2.2, SSER#3, SSER#4) (Closed)

- (a) The operating licensee shall submit a complete report for NRC review and approval delineating interim gas line flow restrictions to 39 cubic meters/sec of natural gas.
- (b) By December 31, 1982, the approved interim gas line flow restrictions and procedures addressing system configuration changes shall be implemented.
- (c) By February 28, 1983, the operating licensee shall submit a report for NRC review and approval describing either:
 - (1) Permanent modifications which limit flow to 39 cubic meters/sec, or
 - (2) Relocation of the pipeline to a safe distance from the facility.
- (d) By September 30, 1984, the selected modification or relocation of the pipeline shall be completed.

(14) Seismic and Loss-of-Coolant Accident Loads (Section 4.2.3, SSER #3)

By August 30, 1982, the operating licensee shall submit to NRC a complete description of the analytical methods along with analytical results with regard to fuel bundle liftoff. This submittal should contain information equivalent to that to be included in the General Electric Topical Report (NEDE-21175-P) regarding fuel bundle liftoff.

(15) Control Room Design Review (Appendix F, SER, SSER#3)

By September 1, 1982, the operating licensee shall complete correction of the following human engineering discrepancies as noted in Appendix F of the Safety Evaluation Report:

2.a.(3) Left/right convention on all controllers.

- 6.f. Unconventional labeling.
- (16) <u>Wetwell to Drywell Vacuum Breakers (Section 6.2.1.8, SSER#3,</u> <u>SSER#4</u>)

Prior to startup following the first refueling outage, the operating licensee shall implement design modification on the wetwell/drywell vacuum breaker valves that include:

- a) installation of new disc assemblies, new shaft bearing caps; and
- b) replacement of the shaft, keys and turnbuckle with stronger materials.

(17) <u>Scram Discharge System Piping (Section 4.6, SER, SSER#1, SSER#2,</u> <u>SSER#3)</u>

- (a) Within 60 days of the issuance of the BWR Owner's Group Report regarding modifications to the Emergency Procedure Guidelines, the operating licensee shall submit a report addressing the Emergency Procedure Guidelines with regard to Scram Discharge Volume (SDV) pipe breaks. The operating licensee shall implement any required system or procedural modifications on a schedule acceptable to the NRC staff.
- (b) Prior to startup following the first refueling outage, the operating licensee shall incorporate the following additional modifications into the scram discharge volume system:
 - (1) Redundant vent and drain valves, and
 - (2) Diverse and redundant SDV instrumentation for each instrumented volume, including both delta pressure sensors and float sensors.

(18) <u>Environmental Qualification (Section 3.11, SER, SSER#1, SSER#2,</u> <u>SSER#3, SSER#4)</u>

 (a) The operating licensee shall complete all actions related to environmental qualification of equipment on a schedule specified in Section 3.11 and Appendix 3.B of Supplement No. 3 of the Safety Evaluation Report with the exceptions of Section 3.11.5.(1) and Section 3.11.5.(2)(e).

- (b) Complete and auditable records must be available and maintained at a central location which describe the environmental qualification methods used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," Revision 1, dated July 1981. Such records shall be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document compliance with NUREG-0588.
- (c) Prior to startup following the first refueling outage, the operating licensee shall be in compliance with the provisions of NUREG-0588 for safety-related electrical equipment exposed to a harsh environment.
- (d) By April 15, 1983, the operating licensee shall implement the maintenance and surveillance schedule for components requiring initial maintenance and surveillance after the first year of operation.
- (e) Deleted by Amendment 37.
- (19) <u>Assurance of Proper Design and Construction (Section 17.6, SSER #3,</u> <u>SSER#4)</u>
 - (a) By December 31, 1982, the operating licensee shall review and categorize discrepancies on large pipe anchors outside containment.
 - (b) By December 31, 1982, the operating licensee shall restore to their original design requirements, discrepancies in large pipe anchors outside containment requiring more complex analysis than used in the original design.
- (20) <u>Emergency Preparedness (Appendix D, SSER #1, SSER #2; 13.3,</u> <u>SSER#4)</u>

By March 1, 1983, the operating licensee shall certify to the NRC staff the completion of the following offsite emergency preparedness items:

(a) Adequate supplies of KI for offsite emergency workers are obtained by the State of Pennsylvania to fulfill the existing State plan or a contingency plan is developed that reflects the inability to obtain supplies to support the existing State plan.

- (b) Adequate supplies of dosimetry for offsite emergency workers are obtained by the State of Pennsylvania to implement the existing State plan or the State plan is revised accordingly.
- (c) State and county plans are modified as necessary to account for the abandonment of the field Emergency Operations Center concept.

(21) School District Emergency Plans (ASLB Initial Decision, Paragraph 223)

This license will be subject to a finding (prior to operation at power levels exceeding five percent of full power) by the Director of Nuclear Reactor Regulation, in consultation with the Federal Emergency Management Agency, that all school districts within the plume exposure pathway emergency planning zone for the Susquehanna Steam Electric Station have completed written emergency plans to respond to fixed nuclear facility accidents.

(22) <u>Municipality Transportation Resources (ASLB Initial Decision,</u> Paragraph 223)

This license will be subject to a finding (prior to operation at power levels exceeding five percent of full power) by the Director of Nuclear Reactor Regulation, in consultation with the Federal Emergency Management Agency, that all municipalities within the plume exposure pathway emergency planning zone have completed their emergency response plans on the transportation resources and program.

- (23) <u>Seismic and Dynamic Qualification (Section 3.10, SER, SSER#1,</u> SSER#3, SSER#4)
 - (a) Prior to startup following the first refueling outage, the operating licensee shall complete any modifications or replacement of equipment found necessary as a result of the operating licensee's fatigue evaluation program. In the interim, the operating licensee shall document the occurrence of every safety relief valve discharge into the suppression pool; the associated cumulative damage factors shall be calculated for typical representative

equipment and kept up-to-date; and the operating licensee shall report to NRC any malfunction of equipment that occurs or should be suspected to have occurred due to any safety relief valve discharge.

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- (b) Prior to use the operating licensee shall complete qualification and documentation, as well as installation of the in-vessel rack.
- (c) By December 31, 1982, the operating licensee shall provide the completed final qualification report for Main Steam Isolation Value Actuator (HV-1F022A through D, HV-1F028 A through D) to the NRC staff for review.
- (d) The operating licensee shall implement the NRC staff's requirements after completion of the staff's review of the final qualification report for the Main Steam Isolation Valve Leakage Control System Heater (1E-203 A through D).
- (e) Prior to exceeding the 25-cycle operational limit, the operating licensee shall qualify the Recirculation Discharge Valve assemblies (HV-1F031 A and B) including new Limitorque actuators. The replacement actuators shall be wired for torque seating type operation.
- (f) Prior to startup following the first refueling outage, the operating licensee shall fully qualify the following items to the SQRT criteria and provide the final qualification reports to the NRC staff for review.
 - 1) CRD vent and drain valves (C12-F010/F011)
 - 2) Power Range Monitor Cabinet (H12-P608)
 - 3) Level Switch (E41-N014)
 - Level Switch Condensate Storage Tanks, Suppression Pool, HPCI Turbine Exhaust Drain Pot (E41-N002/N003, N015, N018)
 - 5) High Pressure Coolant Injection Turbine (15-211)

(24) Containment Purge System (Section 6.2.4, SER)

Prior to startup following the first refueling outage, the operating licensee shall install design features (e.g., screens) on the containment purge system to prevent blocking of the purge and vent valves by debris produced in an accident.

(25) <u>Additional Instrumentation and Control Concerns (Section 7.7.2, SER,</u> <u>SSER #2)</u>

Prior to startup following the first refueling outage, the operating licensee shall resolve the following concerns to the NRC's satisfaction:

- (a) whether common electrical power sources or sensor malfunctions may cause multiple control systems failures, and
- (b) whether high energy line breaks will result in unacceptable consequential control system failures.
- (26) Surveillance of Control Blade (Section 4.2.3, SER)

Deleted in Amendment 106.

(27) Emergency Diesel Engine Starting Systems (Section 9.6.3, SER)

Prior to startup following the first refueling outage, the operating licensee shall install air dryers upstream of the air receivers.

(28) NUREG-0737 Conditions (Section 22, SER)

The operating licensee shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2 and 3, NUREG-0776.

(a) <u>Nuclear Steam Supply System Vendor Review of Procedures</u> (I.C.7, SER, SSER #1)

Prior to beginning low-power testing, the operating licensee shall assure that the General Electric review of the power ascension test procedures has been completed and the General Electric recommendations have been incorporated.

(b) Special Low Power Testing and Training (I.G.1, SER, SSER#3)

(Deleted)

(c) Post Accident Sampling (II.B.3, SER, SSER#1, SSER#3)

Prior to startup following the first refueling outage, the operating licensee shall provide to NRC a revised procedure for core damage estimation to incorporate the requirements in Section 22.2, II.B.3 of Supplement No. 3 of the Safety Evaluation Report.

(d) Instrumentation for Detection of Inadequate Core Cooling (II.F.2, SER, SSER#1, SSER#3)

- (i) By August 31, 1982, the operating licensee shall submit a report addressing the analysis performed by the BWR Owners Group regarding additional instrumentation relative to inadequate core cooling and shall implement the staff's requirements after the completion of the staff's review of this report.
- (ii) By October 31, 1982, the operating licensee shall submit its proposal for conforming with item II.F.2 of NUREG-0737 in view of the BWR Owners Group report.

(e) <u>Modification of Automatic Depressurization System Logic</u> (II.K.3.18, SER, SSER#1, SSER#2, SSER#3)

- (a) By October 1, 1982, the operating licensee shall evaluate the alternative design modifications of the BWR Owners Group relative to the logic for the automatic depressurization system, submit such evaluation, and propose modifications to the NRC for review and approval.
- (b) Prior to startup following the first refueling outage, the operating licensee shall implement the approved alternative logic modification of the automatic depressurization system.
- (f) Effect of Loss of Power on Alternating Current Pump Seals (II.K.3.25, SER, SSER#1) (Closed)

Prior to startup after the first refueling, the operating licensee shall provide an emergency power supply to the cooling system for the recirculation pump seals. (g) <u>Upgrade Emergency Support Facilities (III.A.1.2, SER,</u> <u>SSER #1, SSER #2)</u> (Closed)

The operating licensee shall complete its Emergency Response Facilities as follows:

- (1) Safety Para. Display Sys. December 30, 1983
 (2) Emerg. Opera. Facility October 1, 1982
 (3) Tech. Support Cntr. October 1, 1982
- (29) SRV Inplant Test (Section 6.2.1.8, SER; 6.2.1.5, SSER#1)

Within 90 days following the staff receipt of the report providing the results of the inplant SRV test at the LaSalle, Unit 1 facility, the operating licensee shall furnish the results of its evaluation and application of the LaSalle data to assure that for Susquehanna Unit 1, the delta T between bulk and local pool temperatures will not exceed 10°F.

- (30) <u>Dynamic Testing and Analysis of Systems, Components, and Equipment</u> (Section 3.9.2, SSER#4)
 - (a) By April 1, 1983, the operating licensee shall provide to the NRC staff detailed analysis or testing results which demonstrate that the feedwater isolation valves can adequately perform their intended function and satisfy the requirements of General Design Criteria (GDC) 54 and 55 following a feedwater line break outside containment.
 - (b) Prior to exceeding five percent of full power, the operating licensee shall verify that all check valves relied upon for containment isolation, either within or outside containment, are dynamically qualified or the operating licensee shall provide a basis for continued operation and a program for qualifying such valves.
- (31) Control Room Design Review (Section 22, SSER #4)

Prior to startup following the first refueling outage, the operating licensee shall provide a report discussing the experience, including demonstrated reliability, of the Display Control System.

(32) Emergency Service Water System (Section 6.3.4, SSER #4) (Closed)

Prior to startup following the first refueling outage, the operating licensee shall complete design modifications to the emergency service water (ESW) system, approved by the staff, to eliminate single failure in the ESW system which leads to the need for an uncooled residual heat removal (RHR) pump.

The Unit 1 existing condition in Section 2.F that is subject to the current reporting requirement consists of the following:

F. This license is subject to the following additional condition for the protection of the environment:

Before engaging in additional construction or operational activities which may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement and its Addendum, PPL Susquehanna, LLC shall provide a written notification to the Director of the Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

The current requirements of the Unit 2 license condition are as follows:

2.E Reporting to the Commission:

PPL Susquehanna, LLC shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through C(16) of this license within twenty-four (24) hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written followup in accordance with the procedures described in 10 CFR 50.73 (b), (c), and (e).

The Unit 2 existing conditions in Section 2.C that are subject to the current reporting requirement consist of the following:

(1) <u>Maximum Power Level</u>

PPL Susquehanna, LLC is authorized to operate the facility at reactor core power levels not in excess of 3489 megawatts thermal (100% power) in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational test, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

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(3) <u>Fire Protection Program</u>

PPL Susquehanna, LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Fire Protection Review Report for the facility and as approved in Fire Protection Program, Section 9.5, SER, SSER#1, SSER#2, SSER#3, SSER#4, SSER#6, Safety Evaluation of Fire Protection dated August 9, 1989, Safety Evaluation of Revision 4 to the Fire Protection Report dated March 29, 1993, Safety Evaluation of Fire Protection Program Issues, Safe Shutdown Methodology and Analysis of Associated Circuits dated October 21, 1997, and Safety Evaluation of the licensee's Amendment No. 150, dated June 24, 1998, to relocate the Fire Protection Program subject to the following provision:

The operating licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(4) Operation with Partial Feedwater Heating at End-of-Cycle

PPL Susquehanna, LLC shall not operate with partial feedwater heating for the purpose of extending the normal fuel cycle unless acceptable justification is provided to and approved by the NRC staff prior to such operation.

(5) Initial Test Program

The operating licensee shall conduct the post-fuel-loading initial test program described in Section 14 of the Final Safety Analysis Report, as amended without making any major modifications unless such modifications have prior NRC approval. Major modifications are defined as:

- (a) Elimination of any safety-related test;*
- (b) Modifications of objective, test methods or acceptance criteria for any safety-related test;
- (c) Performance of any safety-related test at a power level different from that stated in the licensees' Final Safety Analysis Report by more than 5% of rated power;

- (d) Failure to satisfactorily complete the entire test program by the time core burnup equals 120 effective full power days;
- (e) Deviation from the initial test program administrative procedures or quality assurance controls as described in the licensees' Final Safety Analysis Report; and
- (f) Delays in the test program in excess of 30 days (14 days if power level exceeds 50%) concurrent with power operation. If continued power operation is desired during a delay, the operating licensee shall provide justification that adequate testing has been performed and evaluated to demonstrate that the facility can be operated at a planned power level with reasonable assurance that the health and safety of the public will not be endangered.
 - * Safety-related tests are those tests which verify the design, construction and operation of safety-related systems, structures and equipment.
- (6) Inservice Inspection Program

By March 1, 1985, the operating licensee shall submit a revised inservice inspection program for NRC review and approval.

(7) Environmental Qualification

Prior to March 31, 1985, the operating licensee shall environmentally qualify all electrical equipment according to the provisions of 10 CFR 50.49 except as follows:

- (a) All modifications of Unit 2 equipment which are common with Unit 1 shall be completed prior to the startup following the first refueling outage for Unit 1 which is prior to November 30, 1985.
- (b) Testing and qualification of conduit seals, silicone rubber insulated cable and NSIS cable shall be completed prior to November 30, 1985.
- (c) Modifications to the Target Rock Solenoid Valve SV-22651 shall be completed prior to November 30, 1985.

- (8) Seismic and Dynamic Qualification
 - (a) Prior to exceeding 5% of rated power, the operating licensee shall complete qualification and documentation, as well as installation for:
 - 1. RCIC back power supply and inverter.
 - 2. A/E added devices to NSSS panels.
 - (b) Prior use, the operating licensee shall complete qualification and documentation, as well as installation for the in-vessel panels.
- (9) <u>Surveillance of Control Blade</u>

Deleted in Amendment 74.

(10) Additional Instrumentation and Control Concerns

Prior to exceeding 5% of rated power, the operating licensee shall resolve the following concerns to the NRC's satisfaction.

- (a) whether common electrical power sources or sensor malfunctions may cause multiple control systems failures, and
- (b) whether high energy line breaks will result in unacceptable consequential control system failures.
- (11) Emergency Diesel Engine Starting Systems

Prior to September 1, 1985, the operating licensee shall install air dryers upstream of the air receivers.

(12) <u>NUREG-0737 Conditions</u>

The operating licensee shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2 "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2, 3, 4, 5, and 6, NUREG-0776.

BWROG reports. Within 90 days after the operating licensee is informed of staff requirements, the operating licensee shall submit for review and approval by the staff, a schedule for implementing any required modifications regarding upgrading of liquid level instrumentation or inclusion of additional instrumentation for detection of inadequate core cooling.

- (f) Modification of Automatic Depressurization System Logic -Feasibility for Increased Diversity For Some Event Sequences
 - (1) Prior to achieving initial criticality, the operating licensee shall:
 - (i) Install modifications to the ADS acceptable to the NRC, and
 - (ii) Propose Technical Specifications for the bypass timer setting and surveillance requirements for the bypass timer.
 - (2) Prior to September 1, 1985, the operating licensee shall:
 - (i) Incorporate into the Plant Emergency Procedures the usage of the manual inhibit switch, and
 - (ii) Propose the Technical Specifications for the manual inhibit switch.
 - (3) The operating licensee shall maintain the manual inhibit switch disabled until License Condition 2.C.(12)(F)(2) above is satisfied.
- (13) Emergency Service Water System

Prior to September 1, 1985, the operating licensee shall complete modifications to the emergency service water (ESW) system described in the operating licensee's letter dated May 16, 1983.

(14) Control of Heavy Loads (NUREG 0612)

Deleted per Amendment No. 21.

(a) Nuclear Steam Supply Vendor Review of Procedures

Prior to achieving initial criticality, the operating licensee shall assure that the General Electric review of the power ascension test procedures has been completed.

(b) Detailed Control Room Design Review

All human engineering deficiencies requiring correction as a result of the operating licensee's Detailed Control Room Design Review for Unit 1 shall be corrected in the Unit 2 control room. By March 1, 1985, the operating licensee should submit its schedule for implementing all engineering deficiency corrective action for review and approval by NRC staff.

(c) Post Accident Sampling

- Prior to exceeding 5% of rated power, the operating licensee shall install and have operational the Post-Accident Sampling System.
- (2) Prior to December 1, 1984, the operating licensee shall revise the interim core damage estimating procedure by submitting for staff review a final procedure which incorporates hydrogen levels, reactor vessel coolant level and containment radiation levels in addition to radionuclide data.

(d) Emergency Response Capabilities

The operating licensee shall complete emergency response facilities and capabilities as required in Attachment 2 of this license.

(e) Instrumentation for Detection of Inadequate Core Cooling

The operating licensee shall implement the staff's requirements regarding upgrading of liquid level instrumentation or inclusion of additional instrumentation for detection of inadequate core cooling necessary to comply with Commission regulations, based on the staff's review of the BWR Owner's Group Reports (SLI 8211 & SLI 8218) and the operating licensee's plant specific evaluation report addressing the recommendations of the

(15) <u>Radon (ASLB Initial Decision, Paragraph 223)</u>

This license will be subject to the ultimate outcome of the consolidated radon proceeding currently underway before the Appeal Boards in Docket Nos. 50-277, 50-278, 50-320, 50-354 and 50-355.

(16) Formal FEMA Finding

In the event the NRC finds that lack of progress in completion of procedures in FEMA final rule, 44 CFR 350, is an indication that a major substantial problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR 50.54(s)(2) will apply.

3.0 BACKGROUND

The background for this application is adequately addressed by NRC Notice of Availability published on November 4, 2005 (70 FR 67202) and the Notice of Opportunity to Comment published on August 29, 2005 (70 FR 51098).

4.0 <u>REGULATORY REQUIREMENTS AND GUIDANCE</u>

The applicable regulatory requirements and guidance associated with this application are adequately addressed by NRC Notice of Availability published on November 4, 2005 (70 FR 67202) and the Notice of Opportunity to Comment published on August 29, 2005 (70 FR 51098).

5.0 <u>TECHNICAL ANALYSIS</u>

PPL Susquehanna, LLC has reviewed the safety evaluation (SE) published on August 29, 2005, as part of the CLIIP Notice of Opportunity to Comment. PPL Susquehanna, LLC has concluded that the justifications presented in the SE prepared by the NRC staff are applicable to Susquehanna Steam Electric Station Units 1 and 2 and justify these proposed amendments of the facility operating licenses for Susquehanna Steam Electric Station Units 1 and 2.

6.0 **REGULATORY ANALYSIS**

A description of this proposed change and its relationship to applicable regulatory requirements and guidance was provided in the NRC Notice of Opportunity to Comment published on August 29, 2005 (70 FR 51098).

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7.0 NO SIGNIFICANT HAZARDS CONSIDERATION

PPL Susquehanna, LLC has reviewed the proposed no significant hazards consideration determination published on August 29, 2005 (70 FR 51098), as part of the CLIIP Notice of Opportunity to Comment. PPL Susquehanna, LLC has concluded that the proposed determination presented in the notice is applicable to Susquehanna Steam Electric Station and the determination is hereby incorporated by reference to satisfy the requirements of 10 CFR 50.91(a).

8.0 ENVIRONMENTAL EVALUATION

PPL Susquehanna, LLC has reviewed the environmental evaluation included in the model SE published on August 29, 2005 (70 FR 51098), as part of the CLIIP Notice of Opportunity to Comment. PPL Susquehanna, LLC has concluded that the NRC staff's findings presented in that evaluation are applicable to Susquehanna Steam Electric Station Units 1 and 2 and the evaluation is hereby incorporated by reference for this application.

9.0 PRECEDENT

This application is being made in accordance with the CLIIP. PPL Susquehanna, LLC is not proposing variations or deviations from the changes described in the NRC staff's model SE published on August 29, 2005 (70 FR 51098).

10.0 <u>REFERENCES</u>

- 1. *Federal Register* Notice of Opportunity to Comment on Model Safety Evaluation on Elimination of Typical License Condition Requiring Reporting of Violations of Section 2.C of Operating License Using the Consolidated Line Item Improvement Process, August 29, 2005 (70 FR 51098).
- 2. *Federal Register* Notice of Availability of Model Application Concerning Elimination of Typical License Condition Requiring Reporting of Violations of Section 2.C of Operating License Using the Consolidated Line Item Improvement Process, November 4, 2005 (70 FR 67202).

Attachment 2 to PLA-6135

Proposed Operating License Changes

Units 1 & 2

(Mark-ups)

Proposed Operating License Changes Unit 1 (Mark-up)

Addendum, PPL Susquehanna, LLC shall provide a written notification to the Director of the Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

- Defeted
 G. PPL-Susquehanna, LLC shall report any violations of the requirements contained in Soction 2, Items C(1), C(3) through C(32), and F of this license within twenty-four (24) hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written followup in accordance with the procedures described in 10 CFR 50.73 (b), (c), and (e).
 - (b) PPL Susquehanna, LLC shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- H. PPL Susquehanna, LLC shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

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Proposed Operating License Changes

Unit 2

(Mark-up)

- 2.D. The operating licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plan, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Physical Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Security and Contingency Plan for Independent Spent Fuel Storage Facility," and was submitted October 8, 2004.
- E. Reporting to the Commission: Deleted

PPL Susquehanna, LLG shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through C(16) of this license within twenty four (24) hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written followup in accordance with the procedures described in 10 CFR 50.73(b), (c), and (c).

- F. PPL Susquehanna, LLC shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- G. This license is effective as of the date of issuance and shall expire at midnight on March 23, 2024.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by Darrell G. Eisenhut For Harold R. Denton

Harold R. Denton, Director Office of Nuclear Reactor Regulation

Attachments:

- 1. Attachment 1
- 2. Attachment 2
- 3. Appendix A Technical
 - Specifications (NUREG-1042)
- 4. Appendix B Environmental Protection Plan

Date of Issuance: March 23, 1984

Amendment No. 102 Revised by letter dated October 28, 2004