

5928-07-20130
June 5, 2007

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Three Mile Island, Unit 1
Facility Operating License No. DPR-50
NRC Docket No. 50-289

Subject: Response to Request for Additional Information Concerning Technical Specifications Change Request No. 334 - One-Time Type A Test Interval Extension

- References:
- 1) Letter from P. B. Cowan (AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, dated September 15, 2006
 - 2) Letter from P. B. Cowan (AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, dated February 26, 2007
 - 3) Letter from R. G. West (AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, dated May 22, 2007

In the Reference 1 letter, AmerGen Energy Company, LLC (AmerGen) requested an amendment to Appendix A, Technical Specifications, of Facility Operating License No. DPR-50. The proposed change modifies Technical Specifications (TS) 6.8.5, "Reactor Building Leakage Rate Testing Program." Specifically, the proposed change will revise TS 6.8.5 to reflect a one-time extension to the Three Mile Island, Unit 1 Type A Integrated Leak Rate Test (ILRT) interval as currently specified in the Technical Specifications. In References 2 and 3, AmerGen provided additional information.

As a result of additional discussions with the U. S. Nuclear Regulatory Commission staff, the attached page 6-11c supersedes page 6-11c previously submitted in Reference 1.

This additional information is bounded by the Reference 1 No Significant Hazards Consideration. There are no commitments contained in this letter.

If any additional information is needed, please contact Tom Loomis at (610) 765-5510.

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I declare under penalty of perjury that the foregoing is true and correct. Executed on the 5th day of June, 2007.

Respectfully,

9-874 

Pamela B. Cowan
Director - Licensing & Regulatory Affairs
AmerGen Energy Company, LLC

Attachment: TMI Unit 1 TS Page 6-11c

cc: R. R. Janati, Commonwealth of Pennsylvania
S. J. Collins, Administrator, Region 1, USNRC
D. M. Kern, USNRC Senior Resident Inspector
P. J. Bamford, Project Manager, USNRC
File No. 06050

Revised Technical Specifications Page 6-11c

6.8.5 Reactor Building Leakage Rate Testing Program

The Reactor Building Leakage Rate Testing Program shall be established, implemented, and maintained as follows:

A program shall be established to implement the leakage rate testing of the Reactor Building as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995, as modified by the following exception to NEI 94-01, Rev. 0, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J":

- a. Section 9.2.3: The first Type A test performed after the September 1993 Type A test shall be performed prior to startup from the T1R18 refueling outage. The T1R18 refueling outage will begin no later than November 1, 2009.

The peak calculated Reactor Building internal pressure for the design basis loss of coolant accident, P_{ac} , is 50.6 psig.

The maximum allowable Reactor Building leakage rate, L_a , shall be 0.1 weight percent of containment atmosphere per 24 hours at P_{ac} .

Reactor Building leakage rate acceptance criteria is $\leq 1.0 L_a$. During the first plant startup following each test performed in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the Type B and Type C tests and $\leq 0.75 L_a$ for the Type A tests.