From: "Beth Murphy" <Beth.Murphy@state.tn.us>
To: <swm@nrc.gov>
Date: 05/30/2007 9:26:38 AM
Subject: State of Tennessee final reg submittal

Mr. Moore,
The State of Tennessee had some rules go into effect on May 26, 2007. Here is our final submittal letter and attachments. If you have any questions please let me know.

Thanks,
Beth Murphy
Division of radiological Health
State of Tennessee
(615)532-0364

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Subject:	State of Tennessee final reg submittal
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From:	"Beth Murphy" <Beth.Murphy@state.tn.us>
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STATE OF TENNESSEE
Department of Environment and Conservation
Division of Radiological Health
L & C Annex, Third Floor
401 Church Street
Nashville, TN 37243–1532

May 30, 2007

Scott W. Moore, Deputy Director
Division of Materials Safey and State Agreements
Office of Federal and State Materials and Environmental Management Programs
11555 Rockville Pike, O3-C10
Washington, DC 20555

Dear Mr. Moore:

Enclosed is a copy of final regulations to Tennessee "State Regulations for Protection Against Radiation" (SRPAR). The comments received on the proposed regulations were made and were officially adopted on May 26, 2007. Any changes to Tennessee's Rules are identified by red-line/strike out text and correspond to the following equivalent amendments to NRC's regulations.

	RATS ID	Title	State Section
•	1994-3	Timeliness in Decommissioning Material Facilities–Parts 30, 40, and 70	1200-2-10-.17;
•	2001–1	Requirements for Certain Generally Licensed Industrial Devices Containing Byproduct Material–Part 30, 31, 32	1200-2-10-.08, .10, .13;
•	2002–1	Revision of the Skin Dose Limit–Part 20	1200-2-5-.32, .50

We believe that adoption of these revisions satisfies the compatibility and health and safety categories established in the Office of State and Tribal Programs (STP) Procedure SA–200 for these amendments.

If you have any questions, please feel free to contact me at 615–532–0364 or Beth Murphy of my staff at 615–532–0392 or Beth.Murphy@state.tn.us.

Sincerely,

Lawrence E. Nanney, Director

Attachments

Revised Chapters 1200-2-5 and 1200-2-10 of Tennessee Regulations that incorporate equivalent NRC amendments into State regulations

NRC review letters dated March 25, 2004 and August 31, 2006

RULES
OF
DEPARTMENT OF ENVIRONMENT AND CONSERVATION
DIVISION OF RADIOLOGICAL HEALTH

CHAPTER 1200–2–5
STANDARDS FOR PROTECTION AGAINST RADIATION

TABLE OF CONTENTS

1200–2–5–.01 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.02 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.03 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.04 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.05 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.06 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.07 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.08 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.09 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.10 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.11 REPEALED.

Authority: *T.C.A. §§4–5–202; 68–202–206 and 68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed May 5, 1988; effective August 29, 1988. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.12 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.13 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.14 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.15 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.16 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed January 8, 1990; effective May 1, 1990. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.17 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed July 11, 1988; effective August 25, 1988. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.18 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed May 9, 1990; effective August 29, 1990. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.19 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.20 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed March 9, 1990; effective June 26, 1990. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.21 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.22 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.23 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.24 REPEALED.

Authority: T.C.A. §68–202–101 et seq. ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.25 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.26 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.27 REPEALED.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.28 REPEALED.

Authority: *T.C.A. §68–202–203; 68–202–206 and 4–5–201 et seq.* ***Administrative History:*** *Original rule filed March 22, 1990; effective June 2, 1990. Repeal filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.29 REPEALED.

1200–2–5–.30 PURPOSE.

(1) The regulations in 1200–2–5–.30 through 1200–2–5–.162 establish standards for protection against ionizing radiation. These standards are issued under Tennessee Code Annotated (T.C.A.) 4–5–201 et seq. and 68–202–203 and 206, as amended. These standards are also issued to meet the Nuclear Regulatory Commission's requirements for compatibility as set out in 42 United States Code Annotated (USCA) Section 2021(d)(2) and 10 CFR 20. It is the intent of the Division of Radiological Health of the Tennessee Department of Conservation that these rules enable the State of Tennessee to maintain its compatibility as an Agreement State. This principle should be considered, when relevant, in any interpretation of these rules. To that end, judicial or administrative interpretation of corresponding rules in other jurisdictions should be given persuasive authority.

(2) The purpose of these standards is to control the receipt, possession, use, transfer and disposal of sources of radiation by any person. This is done so that the total dose to an individual from all sources of radiation other than background radiation does not exceed these standards. However, nothing in these standards shall be construed as limiting a licensee's or registrant's actions that may be necessary to protect health and safety during an emergency.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.31 SCOPE.

These standards apply to all persons who receive, possess, use, transfer, or dispose of sources of radiation within the jurisdiction of the State of Tennessee. The limits in these standards do not apply to doses due to background radiation or to exposure of patients to radiation for medical diagnosis or therapy.

Authority: T.C.A. §§4–5–201 et seq. and 68–202–201 et seq. ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.32 DEFINITIONS.

(1) Absorbed dose means the energy imparted by ionizing radiation per unit mass of irradiated material. The units of absorbed dose are the rad and the gray (Gy).

(2) Act means the Tennessee Code Annotated Chapter 202, as amended.

(3) Activity is the rate of disintegration (transformation) or decay of radioactive material. The units of activity are the curie (Ci) and the becquerel (Bq).

(4) Adult means an individual 18 or more years of age.

(5) Airborne radioactive material means radioactive material dispersed in the air in the form of dusts, fumes, particulates, mists, vapors or gases.

(6) Airborne radioactivity area means a room, enclosure, or area in which airborne radioactive materials, composed wholly or partly of licensed material, exist in concentrations:

(a) In excess of the derived air concentrations (DACs) specified in Schedule RHS 8–30; or

(b) To such a degree that an individual present in the area without respiratory protective equipment could exceed, during the hours an individual is present in a week, an intake of 0.6 percent of the annual limit on intake (ALI) or 12 DAC–hours.

(7) ALARA (acronym for 'as low as is reasonably achievable') means making every reasonable effort to maintain exposures to radiation as far below the dose limits in these standards as is practical consistent with the purpose for which the activity is undertaken and taking into account:

(a) The state of technology;

(b) The economics of improvements in relation to:

1. The state of technology;

2. Benefits to public health and safety, and other societal and socioeconomic considerations; and

3. Utilization of radiation and radioactive materials in the public interest.

(8) Annual limit on intake (ALI) means the derived limit for the amount of radioactive material taken into the body of an adult worker by inhalation or ingestion in a year. ALI is the smaller value of intake of a given radionuclide in a year by the reference man that would result in a committed effective dose equivalent of 5 rems (0.05 Sv) or a committed dose equivalent of 50 rems (0.5 Sv) to any individual organ or tissue. ALI values for intake by ingestion and by inhalation of selected radionuclides are given in Schedule RHS 8–30.

(9) Background radiation means radiation from cosmic sources, naturally occurring radioactive materials, including radon (except as a decay product of source or special nuclear material) and global fallout as it exists in the environment from the testing of nuclear explosive devices

or from past nuclear accidents such as Chernobyl that contribute to background radiation and are not under the control of the licensee. "Background radiation" does not include radiation from sources of radiation subject to licensing or registering by the Division.

(10) Bioassay (or radiobioassay) means the determination of kinds, quantities or concentrations, and, in some cases, the locations of radioactive material in the human body, whether by direct measurement (in vivo counting) or by analysis and evaluation of materials excreted or removed from the human body.

(11) Byproduct material refers to any radioactive material (except special nuclear material) yielded in or made radioactive by exposure to the radiation incident to the process of producing or utilizing special nuclear material.

(12) Class (or lung class or inhalation class) means a classification scheme for inhaled material according to its rate of clearance from the pulmonary region of the lung. Materials are classified as D, W, or Y, which applies to a range of clearance half–times: for Class D (Days) of less than 10 days, for Class W (Weeks) from 10 to 100 days, and for Class Y (Years) of greater than 100 days.

(13) Collective dose is the sum of the individual doses received in a given period of time by a specific population from exposure to a specific source of radiation.

(14) Committed dose equivalent (CDE) ($H_{T,50}$) is the dose equivalent to organs or tissues of reference (T) that will be received from an intake of radioactive material by an individual during the 50–year period following the intake.

(15) Committed effective dose equivalent (CEDE) ($H_{E,50}$) is the sum of the products of the weighting factors applicable to each of the body organs or tissues that are irradiated and the committed dose equivalent to these organs or tissues ($H_{E,50}=(\Sigma W_T H_{T,50})$.

(16) Declared pregnant woman means a woman who has voluntarily informed her employer, in writing, of her pregnancy and the estimated date of conception. The declaration remains in effect until the declared pregnant woman withdraws the declaration in writing or is no longer pregnant.

(17) Deep–dose equivalent (DDE) (H_d), which applies to external whole–body exposure, is the dose equivalent at a tissue depth of 1 cm (1000 mg/cm^2).

(18) Department refers to the Tennessee Department of Environment and Conservation.

(19) Derived air concentration (DAC) means the concentration of a given radionuclide in air which, if breathed by the reference man for a working year of 2,000 hours under conditions of light work (inhalation rate 1.2 cubic meters of air per hour), results in an intake of one ALI. DAC values are given in Schedule RHS 8–30.

(20) Derived air concentration–hour (DAC–hour) is the product of the concentration of radioactive material in air (expressed as a fraction or multiple of the derived air concentration for each radionuclide) and the time of exposure to that radionuclide, in hours. A licensee may take 2,000 DAC–hours to represent one ALI, equivalent to a committed effective dose equivalent of 5 rems (0.05 Sv).

(21) Division means the Division of Radiological Health of the Tennessee Department of Environment and Conservation.

(22) Dose or radiation dose is a generic term that means absorbed dose, dose equivalent, effective dose equivalent, committed dose equivalent, committed effective dose equivalent, or total effective dose equivalent, as defined in other paragraphs of this rule.

(23) Dose equivalent (H_T) means the product of the absorbed dose in tissue, the quality factor, and all other necessary modifying factors at the location of interest. The units of dose equivalent are the rem and sievert (Sv).

(24) Dosimetry processor means an individual or an organization that processes and evaluates individual monitoring equipment in order to determine the radiation dose delivered to the equipment ($\Sigma H_{E,50}=(W_T H_{T,50})$.

(25) Effective dose equivalent (EDE) (H_E) is the sum of the products of the dose equivalent to the organ or tissue (H_T) and the weighting factors (W_T) applicable to each of the body organs or tissues that are irradiated ($H_E=(\Sigma W_T H_T)$.

(26) Embryo/fetus means the developing human organism from conception until the time of birth.

(27) Entrance or access point means any location through which an individual could gain access to radiation areas or to sources of radiation. This includes entry or exit portals of sufficient size to permit human entry, irrespective of their intended use.

(28) Exposure means being exposed to ionizing radiation or to radioactive material.

(29) External dose means that portion of the dose equivalent received from sources of radiation outside the body.

(30) Extremity means hand, elbow, arm below the elbow, foot, knee, or leg below the knee.

(31) Generally applicable environmental radiation standards means standards issued by the Environmental Protection Agency (EPA) under the authority of the Atomic Energy Act of 1954, as amended, that impose limits on radiation exposures or levels, or concentrations or quantities of radioactive material, in the general environment outside the boundaries of locations under the control of persons possessing or using sources of radiation.

(32) Government agency means any executive department, commission, independent establishment, corporation wholly or partly owned by the United States of America, which is an instrumentality of the United States, or any board, bureau, division, service, office, officer, authority, administration, or other establishment in the executive branch of the Government.

(33) Gray (See 1200–2–5–.33(1)(a)).

(34) High radiation area means an area, accessible to individuals, in which radiation levels from radiation sources external to the body could result in an individual receiving a dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour at 30 centimeters from the source of radiation or from any surface that the radiation penetrates.

(35) Individual means any human being.

(36) Individual monitoring means:

(a) The assessment of dose equivalent by the use of devices designed to be worn by an individual;

(b) The assessment of committed effective dose equivalent by bioassay (see Bioassay) or by determination of the time–weighted air concentrations to which an individual has been exposed, i.e., DAC–hours; or

(c) The assessment of dose equivalent by the use of survey data.

(37) Individual monitoring devices (individual monitoring equipment) means devices designed to be worn by a single individual for the assessment of dose equivalent, such as film badges, thermoluminescence dosimeters (TLDs), pocket ionization chambers, and personal ("lapel") air sampling devices.

(38) Internal dose means that portion of the dose equivalent received from radioactive material taken into the body.

(39) Lens dose equivalent applies to the external exposure of the lens of the eye and is taken as the dose equivalent at a tissue depth of 0.3 centimeter (300 mg/cm^2).

(40) License means a license issued under the regulations in Chapter 1200–2–10.

(41) Licensed material means radioactive material received, possessed, used, transferred or disposed of under a general or specific license issued by the Division.

(42) Licensee means the holder of a license.

(43) Limits (or dose limits) means the permissible upper bounds of radiation doses.

(44) Lost or missing radioactive material means radioactive material whose location is unknown. It includes material that has been shipped but has not reached its destination and whose location cannot be readily traced in the transportation system.

(45) Member of the public means any individual except when that individual is receiving an occupational dose.

(46) Minon means an individual less than 18 years of age.

(47) Monitoring (or radiation monitoring, radiation protection monitoring) means the measurement of radiation levels, concentrations, surface area concentrations or quantities of radioactive material and the use of the results of these measurements to evaluate potential exposures and doses.

(48) Nonstochastic effects means health effects, the severity of which vary with the dose and for which a threshold is believed to exist. Radiation–induced cataract formation is an example of a nonstochastic effect (also called a deterministic effect).

(49) NRC means the Nuclear Regulatory Commission or its duly authorized representatives.

(50) Occupational dose means the dose received by an individual in the course of employment in which the individual's assigned duties involve exposure to sources of radiation from registered, unregistered, licensed and unlicensed sources of radiation, whether in the possession of the licensee, registrant or other person. Occupational dose does not include dose received from background radiation, from any medical administration the individual has received, from exposure to individuals administered radioactive material and released in accordance with subparagraph 1200–2–10–.14(2)(e), from voluntary participation in medical research programs, or as a member of the general public.

(51) Person means an individual, trust, firm, joint stock company, corporation (including a government corporation), partnership, association, state, municipality, commission, political subdivision of a state, any interstate body, any governmental agency of this state and any department, agency or instrumentality of the federal government.

(52) Planned special exposure (PSE) means an infrequent exposure to radiation, separate from and in addition to the annual dose limits.

(53) Public dose means the dose received by a member of the public from exposure to radiation and radioactive material released by a licensee, or another source of radiation in a licensee's or registrant's unrestricted areas. It does not include occupational dose or doses received from background radiation, from any medical administration the individual has received, from exposure to individuals administered radioactive material and released in accordance with subparagraph 1200–2–10–.14(2)(e), or from voluntary participation in medical research programs.

(54) Quality factor (Q) means the modifying factor (see Tables RHS 5–1 and RHS 5–2) that is used to derive dose equivalent from absorbed dose.

(55) Quarter means a period of time equal to one–fourth of the year observed by the licensee or registrant (approximately 13 consecutive weeks), providing that the beginning of the first quarter in a year coincides with the starting date of the year and that no day is omitted or duplicated in consecutive quarters.

(56) Rad (See 1200–2–5–.33(1)(b)).

(57) Radiation includes all ionizing electromagnetic waves and corpuscular emissions such as, but not necessarily limited to, gamma rays and x–rays, alpha and beta particles, electrons, neutrons, and protons, and other nuclear particles, but not radio waves or visible, infrared, or ultraviolet light.

(58) Radiation area means an area, accessible to individuals, in which radiation levels could result in an individual receiving a dose equivalent in excess of 0.005 rem (0.05 mSv) in 1 hour at 30 centimeters from the source of radiation or from any surface that the radiation penetrates.

(59) Reference man means a hypothetical aggregation of human physical and physiological characteristics arrived at by the Division after considering among others data and information published by the International Commission on Radiation Protection and the National Council on Radiation Protection and Measurements.

(60) Rem (See 1200–2–5–.33(1)(c)).

(61) Respiratory protective device means an apparatus, such as a respirator, used to reduce the individual's intake of airborne radioactive materials.

(62) Restricted area means an area, access to which is limited by the licensee or registrant for the purpose of protecting individuals against undue risks from exposure to radiation and radioactive materials. Restricted area does not include areas used as residential quarters, but separate rooms in a residential building may be set apart as a restricted area.

(63) Sanitary sewerage means a system of public sewers for carrying off waste water and refuse, but excluding sewage treatment facilities, septic tanks, and leach fields owned or operated by the licensee.

(64) Shallow–dose equivalent (H_s), which applies to the external exposure of the skin of the whole body or the skin of an extremity, is taken as the dose equivalent at a tissue depth of 0.007 centimeter (7 mg/cm^2).

(65) Sievert (See 1200–2–5–.33(1)(d)).

(66) Site boundary means that line beyond which the land or property is not owned, leased or otherwise controlled by the licensee or registrant.

(67) Source material refers to:

(a) Uranium or thorium, or any combination thereof, in any physical or chemical form; or

(b) Ores that contain by weight, 1/20 of one percent (0.05 %) or more of: uranium, thorium or any combinations thereof. Source material does not include special nuclear material.

(68) Stochastic effects means health effects that occur randomly and for which the probability of the effect occurring, rather than its severity, is assumed to be a linear function of dose without threshold. Hereditary effects and cancer incidence are examples of stochastic effects.

(69) Survey means an evaluation of the radiological conditions and potential hazards incident to the production, use, transfer, release, disposal, or presence of radioactive material or other sources of radiation. When appropriate, such an evaluation includes a physical survey of the location of a source of radiation and measurements or calculations of levels of radiation or concentrations or quantities of radioactive material present.

(70) Total effective dose equivalent (TEDE) means the sum of the deep–dose equivalent (for external exposures) and the committed effective dose equivalent (for internal exposures).

(71) Unrestricted area means an area, access to which is neither limited nor controlled by the licensee or registrant.

(72) Very high radiation area means an area, accessible to individuals, in which radiation levels from radiation sources external to the body could result in an individual receiving an absorbed dose in excess of 500 rads (5 grays) in 1 hour at 1 meter from a source of radiation or 1 meter from any surface that the radiation penetrates.

(Note: At very high doses received at high dose rates, units of absorbed dose (e.g., rads and grays) are appropriate, rather than units of dose equivalent (e.g., rems and sieverts)).

(73) Week means seven (7) consecutive days starting on Sunday.

(74) Weighting factor (W_T), for an organ or tissue (T) is the proportion of the risk of stochastic effects resulting from irradiation of the organ or tissue to the total risk of stochastic effects when the whole body is irradiated uniformly. For calculating the effective dose equivalent, the values of W_T are:

ORGAN DOSE WEIGHTING FACTORS

Organ or Tissue	W_T
Gonads	0.25
Breasts	0.15
Red Bone Marrow	0.12

ORGAN DOSE WEIGHTING FACTORS

Organ or Tissue	W_T
Lung	0.12
Thyroid	0.03
Bone Surfaces	0.03
Remainder	[1]0.30
Whole Body	[2]1.00

[1] 0.30 results from 0.06 for each of 5 "remainder" organs (excluding the skin and the lens of the eye) that receive the highest doses.

[2] For the purpose of weighting the external whole body dose (for adding it to the internal dose), a single weighting factor, W_T=1.0, has been specified. The use of other weighting factors for external exposure will be approved on a case–by–case basis until such time as specific guidance is issued.

(75) Whole body means, for purposes of external exposure, head, trunk (including male gonads), arms above the elbow, or legs above the knee.

(76) Working level (WL) is any combination of short–lived radon daughters (for radon–222: polonium–218, lead–214, bismuth–214, and polonium–214; and for radon–220: polonium–216, lead–212, bismuth–212, and polonium–212) in 1 liter of air that will result in the ultimate emission of 1.3 (E+5) MeV of potential alpha particle energy.

(77) Working level month (WLM) means an exposure to 1 working level for 170 hours (2,000 working hours per year/12 months per year = approximately 170 hours per month).

(78) Year means the period of time beginning in January used to determine compliance with the provisions of these standards. The licensee or registrant may change the starting date of the year used to determine compliance by the licensee or registrant provided that the change is made at the beginning of the year and that no day is omitted or duplicated in consecutive years.

(79) Misadministration means the administration of:

(a) A radiopharmaceutical dosage greater than 30 microcuries of either sodium iodide I–125 or I–131:

1. Involving the wrong individual, or wrong radiopharmaceutical; or

2. When both:

(i) The administered dosage differs from the prescribed dosage by more than 20 percent (20%) of the prescribed dosage and

(ii) The administered dosage differs from the prescribed dosage by more than 30 microcuries.

(b) A therapeutic radiopharmaceutical dosage, other than sodium iodide I–125 or I–131:

1. Involving the wrong individual, wrong radiopharmaceutical, or wrong route of administration; or

2. When the administered dosage differs from the prescribed dosage by more than 20 percent (20%) of the prescribed dosage.

(c) A gamma stereotactic radiosurgery radiation dose:

1. Involving the wrong individual, or wrong treatment site; or

2. When the calculated total administered dose differs from the total prescribed dose by more than 10 percent (10%) of the total prescribed dose.

(d) A teletherapy radiation dose:

1. Involving the wrong individual, wrong mode of treatment or wrong treatment site;

2. When the treatment consists of three (3) or fewer fractions and the calculated total administered dose differs from the total prescribed dose by more than 10 percent (10%) of the total prescribed dose;

3. When the calculated weekly administered dose exceeds the weekly prescribed dose by 30 percent (30%) or more of the weekly prescribed dose; or

4. When the calculated total administered dose differs from the total prescribed dose by more than 20 percent (20%) of the total prescribed dose.

(e) A brachytherapy radiation dose:

1. Involving the wrong individual, wrong radioisotope, or wrong treatment site (excluding, for permanent implants, seeds that were implanted in the correct site but migrated outside the treatment site);

2. Involving a sealed source that is leaking;

3. When, for a temporary implant, one or more sealed sources are not removed upon completion of the procedure; or

4. When the calculated administered dose differs from the prescribed dose by more than 20 percent (20%) of the prescribed dose.

(f) A diagnostic radiopharmaceutical dosage, other than quantities greater than 30 microcuries of either sodium iodide I–125 or I–131:

1. Involving the wrong individual; or

2. When both:

(i) The exposure involves the wrong radiopharmaceutical or wrong route of administration, or when the administered dosage differs from the prescribed dosage; and

(ii) The dose to the individual exceeds 5 rems effective dose equivalent or 50 rems dose equivalent to any individual organ.

(g) A therapeutic radiation machine dose:

1. Involving the wrong individual, wrong mode of treatment or wrong treatment site,

2. When the treatment consists of three (3) or fewer fractions and the calculated total administered dose differs from the total prescribed dose by more than 10 percent (10%) of the total prescribed dose,

3. When the calculated weekly administered dose exceeds the weekly prescribed dose by 30 percent (30%) or more of the weekly prescribed dose, or

4. When the calculated total administered dose differs from the total prescribed dose by more than 20 percent (20%) of the total prescribed dose.

(h) A diagnostic x–ray radiation machine exposure involving the wrong individual.

(80) Constraint (or dose constraint) means a value above which specified licensee actions are required.

(81) Air-purifying respirator means a respirator with an air-purifying filter, cartridge or canister that removes specific air contaminants by passing ambient air through the air-purifying element.

(82) Assigned protection factor (APF) means the expected workplace level of respiratory protection that would be provided by a properly functioning respirator or a class of respirators to properly fitted and trained users. Operationally, the inhaled concentration can be estimated by dividing the ambient airborne concentration by the APF.

(83) Atmosphere-supplying respirator means a respirator that supplies the respirator user with breathing air from a source independent of the ambient atmosphere, and includes supplied-air respirators (SARs) and self-contained breathing apparatus (SCBA) units.

(84) Demand respirator means an atmosphere-supplying respirator that admits breathing air to the facepiece only when a negative pressure is created inside the facepiece by inhalation.

(85) Disposable respirator means a respirator for which maintenance is not intended and that is designed to be discarded after excessive breathing resistance, sorbent exhaustion, physical damage, or end-of-service-life renders it unsuitable for use. Examples of this type of respirator are a disposable half-mask respirator or a disposable escape-only self-contained breathing apparatus (SCBA).

(86) Filtering facepiece (dust mask) means a negative pressure particulate respirator with a filter as an integral part of the facepiece or with the entire facepiece composed of the filtering medium, not equipped with elastomeric sealing surfaces and adjustable straps.

(87) Fit factor means a quantitative estimate of the fit of a particular respirator to a specific individual, and typically estimates the ratio of the concentration of a substance in ambient air to its concentration inside the respirator when worn.

(88) Fit test means the use of a protocol to evaluate qualitatively or quantitatively the fit of a respirator on an individual.

(89) Helmet means a rigid respiratory inlet covering that also provides head protection against impact and penetration.

(90) Hood means a respiratory inlet covering that completely covers the head and neck and may also cover portions of the shoulders and torso.

(91) Loose-fitting facepiece means a respiratory inlet covering that is designed to form a partial seal with the face.

(92) Negative pressure respirator (tight fitting) means a respirator in which the air pressure inside the facepiece is negative during inhalation with respect to the ambient air pressure outside the respirator.

(93) Positive pressure respirator means a respirator in which the pressure inside the respiratory inlet covering exceeds the ambient air pressure outside the respirator.

(94) Powered air-purifying respirator (PAPR) means an air-purifying respirator that uses a blower to force the ambient air through air-purifying elements to the inlet covering.

(95) Pressure demand respirator means a positive pressure atmosphere-supplying respirator that admits breathing air to the facepiece when the positive pressure is reduced inside the facepiece by inhalation.

(96) Qualitative fit test (QLFT) means a pass/fail fit test to assess the adequacy of respirator fit that relies on the individual's response to the test agent.

(97) Quantitative fit test (QNFT) means an assessment of the adequacy of respirator fit by numerically measuring the amount of leakage into the respirator.

(98) Self-contained breathing apparatus (SCBA) means an atmosphere-supplying respirator for which the breathing air source is designed to be carried by the user.

(99) Supplied-air respirator (SAR) or airline respirator means an atmosphere-supplying respirator for which the source of breathing air is not designed to be carried by the user.

(100) Tight-fitting facepiece means a respiratory inlet covering that forms a complete seal with the face.

(101) User seal check (fit check) means an action conducted by the respirator user to determine if the respirator is properly seated to the face. Examples include negative pressure check, positive pressure check, irritant smoke check or isoamyl acetate check.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–5–.33 UNITS OF RADIATION DOSE.

(1) Definitions. As used in these standards the units of radiation dose are:

(a) Gray (Gy) is the SI unit of absorbed dose. One gray is equal to an absorbed dose of 1 joule/kilogram (100 rads).

(b) Rad is the special unit of absorbed dose. One rad is equal to an absorbed dose of 100 ergs/gram or 0.01 joule/kilogram (0.01 gray).

(c) Rem is the special unit of any of the quantities expressed as dose equivalent. The dose equivalent in rems is equal to the absorbed dose in rads multiplied by the quality factor (1 rem = 0.01 sievert).

(d) Sievert is the SI unit of any of the quantities expressed as dose equivalent. The dose equivalent in sieverts is equal to the absorbed dose in grays multiplied by the quality factor (1 Sv = 100 rems).

(2) As used in these standards the quality factors for converting absorbed dose to dose equivalent are shown in Table RHS 5–1.

Table RHS 5–1 QUALITY FACTORS AND ABSORBED DOSE EQUIVALENCIES

Type of Radiation	Quality Factor (Q)	Absorbed dose equal to a unit dose equivalent[1]
X–, gamma, or beta radiation	1	1
Alpha particles, multiple charged particles, fission fragments and heavy particles of unknown charge	20	0.05
Neutrons of unknown energy	10	0.1
High–energy protons	10	0.1

[1] Absorbed dose in rad equal to 1 rem or the absorbed dose in gray equal to 1 sievert.

If measuring the neutron fluence rate is more convenient than determining the neutron dose equivalent rate as provided in this paragraph, 1 rem (0.01 Sv) of neutron radiation of unknown energies may be assumed to result from a total fluence of 25 million neutrons per square centimeter incident upon the body. If sufficient information exists to estimate the approximate energy distribution of the neutrons, the licensee or registrant may use the fluence rate per unit dose equivalent or the appropriate Q value from Table RHS 5–2 to convert a measured tissue dose in rads to dose equivalent in rems.

Table RHS 5–2 MEAN QUALITY FACTORS, Q, AND FLUENCE PER UNIT DOSE EQUIVALENT FOR MONOENERGETIC NEUTRONS

	Neutron Energy (MeV)	Quality Factor [a] (Q)	Fluence per unit dose equivalent [b] (neutrons cm^{-2} rem^{-1})
(Thermal).	2.5 (E–8)	2	980 (E+6)
	1 (E–7)	2	980 (E+6)
	1 (E–6)	2	810 (E+6)
	1 (E–5)	2	810 (E+6)
	1 (E–4)	2	840 (E+6)
	1 (E–3)	2	980 (E+6)
	1 (E–2)	2.5	1010 (E+6)
	1 (E–1)	7.5	170 (E+6)
	5 (E–1)	11	39 (E+6)
	1	11	27 (E+6)
	2.5	9	29 (E+6)
	5	8	23 (E+6)

Neutron Energy (MeV)	Quality Factor [a] (Q)	Fluence per unit dose equivalent [b] (neutrons cm^{-2} rem^{-1})
7	7	24 (E+6)
10	6.5	24 (E+6)
14	7.5	17 (E+6)
20	8	16 (E+6)
40	7	14 (E+6)
60	5.5	16 (E+6)
1 (E+2)	4	20 (E+6)
2 (E+2)	3.5	19 (E+6)
3 (E+2)	3.5	16 (E+6)
4 (E+2)	3.5	14 (E+6)

[a] Value of quality factor (Q) at the point where the dose equivalent is maximum in a 30 cm diameter cylinder tissue–equivalent phantom.

[b] Monoenergetic neutrons incident normally on a 30 cm diameter cylinder tissue–equivalent phantom.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203 and 206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.34 UNITS OF RADIOACTIVITY.

(1) For the purposes of these standards, activity is expressed in the special unit of curies (Ci) or in the SI unit of becquerels (Bq), or their multiples, or disintegrations (transformations) per unit of time.

(a) One becquerel = 1 disintegration per second (s^{-1}).

(b) One curie = 3.7 x 10^{10} disintegrations per second = 3.7 (E+10) becquerels = 2.22 (E+12) disintegrations per minute.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.35 COMMUNICATIONS.

Unless otherwise specified, communications or reports concerning the regulations should be addressed to the Director, Division of Radiological Health, L&C Annex, 3rd Floor, 401 Church Street, Nashville, TN 37243–1523.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.36 REPEALED.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.37 THROUGH 1200–2–5–.39 RESERVED.

1200–2–5–.40 RADIATION PROTECTION PROGRAMS.

(1) Each licensee and registrant shall develop, document and implement a radiation protection program for a licensee's or registrant's activities that ensures compliance with these standards. See 1200–2–5–.131 for recordkeeping requirements relating to these programs.

(2) The licensee's or registrant's procedures and engineering controls shall be based on sound radiation protection principles and shall achieve occupational doses and doses to members of the public that are ALARA.

(3) The licensee or registrant shall periodically (at least annually) review radiation protection program content and implementation.

(4) To implement the ALARA requirements of paragraph 1200–2–5–.40(2) and notwithstanding the requirements in Rule 1200–2–5–.70, licensees shall establish a constraint on air emissions of radioactive material to the environment, excluding radon–222 and its daughters. The constraint shall ensure that the individual member of the public likely to receive the highest dose shall not be expected to receive a total effective dose equivalent in excess of 10 millirems (0.1 millisievert) per year from these emissions. If a licensee exceeds this dose constraint, the licensee shall report the occurrence as provided in Rule 1200–2–5–.143 and take prompt, appropriate corrective action to ensure against recurrence.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.41 THROUGH 1200–2–5–.49 RESERVED.

1200–2–5–.50 OCCUPATIONAL DOSE LIMITS FOR ADULTS.

(1) Except for planned special exposures under 1200-2-5-.54, the licensee or registrant shall limit the occupational dose to individual adults to the following annual dose limits:

(a) An annual limit that is t~~T~~he lesser of:

1. a~~A~~ total effective dose equivalent of 5 rems (0.05 Sv) or

2. the~~The~~ sum of the deep–dose equivalent and the committed dose equivalent to any individual organ or tissue other than the lens of the eye equal to 50 rems (0.5 Sv).

(b) The annual limits to the lens of the eye, to the skin of the whole body and to the skin of the extremities:

1. a~~A~~ lens–dose equivalent to 15 rems (0.15 Sv)~~.~~, and

~~(c)~~2. A shallow–dose equivalent of 50 rems (0.50 Sv) to the skin of the whole body or to the skin of any extremity.

~~(c) A shallow-dose equivalent of 50 rems (0.50 Sv) to the skin or to any extremity.~~

(2) The amount by which occupational dose from all sources exceeds an individual's annual

limits shall be subtracted from the individual's limit for planned special exposures for the current year and for lifetime exposure. See 1200-2-5-.54(1)(f)1 and 2.

(3) The assigned deep–dose equivalent shall be for the part of the body receiving the highest exposure. The assigned shallow–dose equivalent shall be the dose averaged over the contiguous 10 cm^2 of skin receiving the highest exposure. Deep–dose, lens–dose and shallow–dose equivalents may be assessed from surveys or other radiation measurements to demonstrate compliance with occupational dose limits. However, this may be done only if the individual monitoring device was not subject to the highest potential exposure, or the individual monitoring results are unavailable.

(4) Derived air concentration (DAC) and annual limit on intake (ALI) values are presented in Schedule RHS 8–30 and may be used to determine the individual's dose and demonstrate compliance with the occupational dose limits.

(5) In addition to the annual dose limits, the licensee shall limit the soluble uranium intake by an individual to 10 milligrams in a week in consideration of chemical toxicity (see footnote 3 of Schedule RHS 8–30).

(6) The licensee or registrant shall reduce the dose that an individual may be allowed to receive in the current year by the amount of occupational dose received while employed by any other person.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–5–.51 COMPLIANCE WITH REQUIREMENTS FOR SUMMATION OF EXTERNAL AND INTERNAL DOSES.

(1) If the licensee is required to monitor under both 1200–2–5–.71(1) and (2), the licensee shall demonstrate compliance with the dose limits by summing external and internal doses. If the licensee or registrant is required to monitor only under 1200–2–5–.71(1) or only under 1200–2–5–.71(2) then summation is not required to demonstrate compliance with the dose limits. The licensee may demonstrate compliance with the requirements for summation of external and internal doses by meeting one of the conditions specified in (2) of this rule and the conditions in (3) and (4) of this rule.

(Note: The dose equivalents for the lens of the eye, the skin, and the extremities are not included in the summation, but are subject to separate limits.)

(2) Intake by inhalation. If the only intake of radionuclides is by inhalation, the total effective dose equivalent limit is not exceeded if the sum of the deep–dose equivalent divided by the total effective dose equivalent limit, and one of the following, does not exceed unity:

(a) The sum of the fractions of the inhalation ALI for each radionuclide; or

(b) The total number of derived air concentration–hours (DAC–hours) for all radionuclides divided by 2,000; or

(c) The sum of the calculated committed effective dose equivalents to all significantly irradiated organs or tissues (T) [1] where the organ dose is expressed as a fraction of the annual limit. This sum shall be calculated from bioassay data using appropriate biological models.

(3) Intake by oral ingestion. The licensee shall account for oral ingestion of radionuclides and include it in demonstrating compliance with the limits when:

(a) The occupationally exposed individual intakes radionuclides by ingestion; and

(b) The oral ingestion exceeds 10 percent (10%) of the applicable oral ALI.

(4) Intake through wounds or absorption through skin. The licensee shall evaluate and, to the extent practical, account for intakes through wounds or skin absorption.

(Note: The intake through intact skin has been included in the calculation of DAC for hydrogen–3 and does not need to be further evaluated.)

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.52 DETERMINATION OF EXTERNAL DOSE FROM AIRBORNE RADIOACTIVE MATERIAL.

In determining the dose from airborne radioactive material, the licensee shall include the contribution to the deep–dose equivalent, lens–dose equivalent, and shallow–dose equivalent from external exposure to the radioactive cloud (see Schedule RHS 8–30 footnotes 1 and 2).

(Note: Airborne radioactivity measurements and DAC values should not be used as the primary means to assess the deep–dose equivalent when the airborne radioactive material includes radionuclides other than noble gases or if the cloud of airborne radioactive material is not relatively uniform. The determination of the deep–dose equivalent to an individual should be based upon measurements using instruments or individual monitoring devices.)

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.53 DETERMINATION OF INTERNAL EXPOSURE.

(1) To assess the dose used to determine compliance with occupational dose equivalent limits, and when required by 1200–2–5–.71, the licensee shall take suitable and timely measurements of:

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[1] *An organ or tissue is considered significantly irradiated if the product of the weighting factors, W_T, and the committed dose equivalent, H_{T50}, per unit intake for that organ or tissue is greater than 10 percent of the maximum weighted value of H_{T50} (i.e., $W_T H_{T50}$) per unit intake for any organ or tissue.*

(a) Concentrations of radioactive materials in air in work areas; or

(b) Quantities of radionuclides in the body; or

(c) Quantities of radionuclides excreted from the body; or

(d) Combinations of these measurements.

(2) The licensee shall assume that the concentration of airborne radioactive material inhaled by an individual is equal to the concentration in the individual's ambient air unless:

(a) Respiratory protective equipment is used, as provided in 1200-2-5-.92; or

(b) The assessment of intake is based on bioassays.

(3) When specific information is known about the physical and biochemical properties of the radionuclides taken into the body or the behavior of the material in an individual, the licensee may:

(a) Use that information to calculate the committed effective dose equivalent, and if used, the licensee shall document that information in the individual's record; and

(b) Upon prior approval of the Division adjust the DAC or ALI values to reflect the actual physical and chemical characteristics of airborne radioactive material (e.g., aerosol size distribution or density); and

(c) Separately assess the contribution of fractional intakes of Class D, W or Y compounds of a given radionuclide (see Schedule RHS 8-30) to the committed effective dose equivalent.

(4) If the licensee uses the measurements in 1200-2-5-.53(1)(b) or (c) to assess intakes of Class Y material, the licensee may delay recording and reporting the assessments for up to 7 months. This delay is allowed only if:

(a) It is necessary to make additional measurements basic to the assessments;

(b) Recording and reporting are not otherwise required by 1200-2-5-.141 or 1200-2-5-.143.

(5) If the identity and concentration of each radionuclide in a mixture are known, the fraction of the DAC applicable to the mixture for use in calculating DAC-hours must be either:

(a) The sum of the ratios of the concentration to the appropriate DAC value (e.g., D, W, Y) from Schedule RHS 8-30 for each radionuclide in the mixture; or

(b) The ratio of the total concentration for all radionuclides in the mixture to the most restrictive DAC value for any radionuclide in the mixture.

(6) If the identity of each radionuclide in the mixture is known, but the concentration of one or more of the radionuclides in the mixture is not known, the DAC for the mixture must be the most restrictive DAC of any radionuclide in the mixture.

(7) When a mixture of radionuclides in air exists, licensees may disregard certain radionuclides in the mixture if:

(a) The licensee uses the total activity of the mixture in demonstrating compliance with the dose limits in 1200–2–5–.50 and in complying with the monitoring requirements in 1200–2–5–.71(1)(b);

(b) The concentration of any radionuclide disregarded is less than 10 percent (10%) of its DAC; and

(c) The sum of the percentages for all disregarded radionuclides does not exceed 30 percent (30%).

(8) To calculate the committed effective dose equivalent, the licensee may assume that the inhalation of one ALI, or an exposure of 2,000 DAC–hours, results in a committed effective dose equivalent of 5 rems (0.05 Sv). This assumption may only be made for radionuclides that have their ALIs or DACs based on the committed effective dose equivalent.

(9) When the ALI (and the associated DAC) is determined by the nonstochastic organ dose limit of 50 rems (0.5 Sv), the intake of radionuclides that would result in a committed effective dose equivalent of 5 rems (0.05 Sv) (the stochastic ALI) is listed in parentheses in Table 1 of Schedule RHS 8–30. In this case, the licensee may, as a simplifying assumption, use the stochastic ALIs to determine committed effective dose equivalent. However, if the licensee uses the stochastic ALIs, the licensee must also demonstrate that the limit in 1200–2–5–.50(1)(a) is met.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.54 PLANNED SPECIAL EXPOSURES.

(1) A licensee or registrant may authorize an adult worker to receive doses in addition to the doses received under the limits specified in 1200–2–5–.50. Additional doses are allowed only if the following conditions are satisfied:

(a) The additional doses are accounted for separately from the doses received under the limits in 1200–2–5–.50.

(b) The licensee or registrant authorizes a planned special exposure only in an exceptional situation when alternatives that might avoid the dose estimated to result from the planned special exposure are unavailable or impractical.

(c) The licensee or registrant (and employer if different from the licensee or registrant) gives specific written authorization before the planned special exposure occurs.

(d) Before a planned special exposure, the licensee or registrant ensures that the individuals involved are:

1. Informed of the purpose of the planned operation;

2. Informed of the estimated doses and associated potential risks and specific radiation levels or other conditions that might be involved in performing the task; and

3. Instructed in the measures to be taken to keep the dose ALARA considering other risks that may be present.

(e) Prior to permitting an individual to participate in a planned special exposure, the licensee or registrant ascertains prior doses during the lifetime of the individual for each individual involved, as required by 1200–2–5–.133(2).

(f) Subject to 1200–2–5–.50(2) the licensee or registrant does not authorize a planned special exposure that would cause an individual to receive a dose from all planned special exposures and all doses in excess of the limits to exceed:

1. The numerical values of any of the dose limits in 1200–2–5–.50(1) in any year; and

2. Five (5) times the annual dose limits in 1200–2–5–.50(1) during the individual's lifetime.

(g) The licensee or registrant maintains records of the conduct of a planned special exposure in accordance with 1200–2–5–.134 and submits a written report in accordance with 1200–2–5.144.

(h) The licensee or registrant records in the individual's record the best estimate of the dose resulting from the planned special exposure. The dose from planned special exposures is not to be considered in controlling future occupational dose of the individual under 1200–2–5–.50(1) but is to be included in evaluations required by (5) and (6) of this rule.

(i) The licensee or registrant gives the individual written notice of the estimated dose within 30 days after the date of the planned special exposure.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.55 OCCUPATIONAL DOSE LIMITS FOR MINORS.

The annual occupational dose limits for minors are 10 percent (10%) of the annual dose limits specified for adult workers in 1200–2–5–.50.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.56 DOSE EQUIVALENT TO AN EMBRYO/FETUS.

(1) The licensee or registrant shall ensure that the dose equivalent to an embryo/fetus during the entire pregnancy, due to occupational exposure of a declared pregnant woman, does not exceed 0.5 rem (5 mSv). (For recordkeeping requirements see 1200–2–5–.135).

(2) Using ALARA the licensee or registrant shall make efforts to avoid substantial variation above a uniform monthly exposure rate to a declared pregnant woman.

(3) The dose equivalent to an embryo/fetus shall be taken as the sum of:

(a) The deep–dose equivalent to the declared pregnant woman; and

(b) The dose equivalent to the embryo/fetus from radionuclides in the embryo/fetus and radionuclides in the declared pregnant woman.

(4) If when a woman declares her pregnancy to the licensee or registrant the dose equivalent to the embryo/fetus is found to be 0.45 rem (4.5 mSv) or greater, the embryo/fetus is permitted an additional dose not exceeding 0.05 rem (0.5 mSv) during the remainder of the pregnancy.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.57 THROUGH 1200–2–5–.58 RESERVED.

1200–2–5–.59 ORDER REQUIRING FURNISHING OF BIOASSAY SERVICES.

Where necessary to ascertain the extent of an individual's exposure to concentrations of radioactive material, the Division may require a licensee to make available to the individual bioassay services and to furnish a copy of the reports of such services to the Division.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.60 DOSE LIMITS FOR INDIVIDUAL MEMBERS OF THE PUBLIC.

(1) Each licensee and registrant shall conduct operations so that:

(a) The total effective dose equivalent received by any individual member of the public from the licensed or registered operation does not exceed 0.1 rem (1 mSv) in a year. This limit is exclusive of the dose contribution from background radiation, from any medical administration the individual has received, from exposure to individuals administered radioactive material and released in accordance with subparagraph 1200–2–10–.14(2)(e), from voluntary participation in medical research programs, and from the licensee's disposal of radioactive material into sanitary sewerage in accordance with 1200–2–5–.122; and

(b) The dose in any unrestricted area from external sources does not exceed 0.002 rem (0.02 mSv) in any one hour.

(2) A licensee, registrant or applicant may apply for prior authorization to operate up to an annual dose limit of 0.5 rem (5 mSv) for an individual member of the public. This application by the licensee, registrant or applicant shall include the following:

(a) Demonstration of the need for and the expected duration of operations in excess of the limit in (1) of this rule;

(b) The licensee's or registrant's program to assess and control dose within the 0.5 rem (5 mSv) annual limit; and

(c) The procedures to be followed to maintain the dose as low as is reasonably achievable (ALARA).

(3) In addition to the requirements of this chapter, a licensee subject to the provisions of EPA's generally applicable environmental radiation standards in 40 CFR Part 190 shall comply with those standards.

(4) The Division may impose additional restrictions on radiation levels in unrestricted areas and on the total quantity of radionuclides that a licensee may release in effluents in order to restrict the collective dose.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.61 COMPLIANCE WITH DOSE LIMITS FOR INDIVIDUAL MEMBERS OF THE PUBLIC.

(1) The licensee or registrant shall demonstrate compliance with the dose limits in 1200–2–5–.60 by making or causing to be made surveys of:

(a) Radiation levels in unrestricted and restricted areas; and

(b) Radiation levels and radioactive materials in effluents released to unrestricted areas.

(2) A licensee or registrant shall show compliance with the annual dose limit in 1200–2–5–.60 by:

(a) Demonstrating by measurement or calculation that the individual likely to receive the highest dose from the licensee's or registrant's operation does not receive a total effective dose equivalent exceeding the annual dose limit; or

(b) Demonstrating that:

1. The annual average concentrations of radioactive material released in gaseous and liquid effluents at the boundary of the unrestricted area do not exceed the values specified in Table 2 of Schedule RHS 8–30; and

2. If an individual were continually present in an unrestricted area, the dose from external sources would not exceed 0.002 rem (0.02 mSv) in an hour and 0.05 rem (0.5 mSv) in a year.

(3) Upon approval from the Division, the licensee may adjust the effluent concentration values in Schedule RHS 8–30, Table 2, for members of the public, to take into account the actual physical and chemical characteristics of the effluents (e.g., aerosol size distribution, solubility, density, radioactive decay equilibrium, chemical form).

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.62 THROUGH 1200–2–5–.69 RESERVED.

1200–2–5–.70 GENERAL SURVEY AND MONITORING REQUIREMENTS.

(1) Each licensee and registrant shall make or cause to be made, surveys that:

(a) May be necessary for the licensee or registrant to comply with the standards in this chapter; and

(b) Are reasonable under the circumstances to evaluate:

1. The magnitude and extent of radiation levels;

2. Concentrations or quantities of radioactive material; and

3. The potential radiological hazards.

(2) The licensee or registrant shall ensure that instruments and equipment used for quantitative radiation measurements (e.g., dose rate and effluent monitoring) are calibrated periodically for the radiation measured.

(3) Except for direct and indirect reading pocket ionization chambers and those dosimeters used to measure the dose to the extremities, all personnel dosimeters for determining the dose and used to comply with these standards or with conditions specified in a license or registration shall be processed and evaluated by a dosimetry processor:

(a) Holding current personnel dosimetry accreditation from the National Voluntary Laboratory Accreditation Program (NVLAP) of the National Institute of Standards and Technology; and

(b) Approved for processing and evaluating dosimeters exposed to the type of radiation(s) included in the NVLAP program that most closely approximates the type of radiation(s) being monitored by the dosimeter.

***Authority**: T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, ans 68-202-206.* ***Administrative History**: Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.71 CONDITIONS REQUIRING INDIVIDUAL MONITORING OF EXTERNAL AND INTERNAL OCCUPATIONAL DOSE.

(1) Each licensee and registrant shall monitor exposures to radiation and radioactive material at levels sufficient to demonstrate compliance with the occupational dose limits of this chapter:

(a) Each licensee and registrant shall monitor occupational exposure to radiation from licensed or unlicensed and registered or unregistered radiation sources under the control of the licensee and registrant and shall supply and require the use of individual monitoring devices by:

1. Adults likely to receive, in one (1) year from sources external to the body, a dose in excess of 10 percent (10%) of the limits in 1200–2–5–.50;

2. Minors likely to receive, in one (1) year from radiation sources external to the body, a deep dose equivalent in excess of 0.1 rem (1 mSv), a lens dose equivalaent in excess of 0.15 rem (1.5 mSv), or a shallow dose equivalent to the skin or to the extremities in excess of 0.5 rem (5 mSv);

3. Declared pregnant women likely to receive during the entire pregnancy, from radiation sources external to the body, a deep dose equivalent in excess of 0.1 rem (1 mSv) [2]; and

4. Individuals entering a high or very high radiation area.

(2) Each licensee shall monitor (see 1200–2–5–.53) the occupational intake of radioactive material by, and assess the committed effective dose equivalent to:

[2] *All of the occupational dose in 1200–2–5–.50 continue to be applicable to the declared pregnant woman as long as the embryo/fetus dose equivalent limit is not exceeded.*

(a) Adults likely to receive, in one (1) year, an intake in excess of 10 percent (10%) of the applicable ALI(s) in Table 1, Columns 1 and 2, of Schedule RHS 8–30;

(b) Minors likely to receive, in one (1) year, a committed effective dose equivalent in excess of 0.1 rem (1 mSv); and

(c) Declared pregnant women likely to receive, during the entire pregnancy, a committed effective dose equivalent in excess of 0.1 rem (1 mSv).

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.72 THROUGH 1200–2–5–.79 RESERVED.

1200–2–5–.80 CONTROL OF ACCESS TO HIGH RADIATION AREAS.

(1) The licensee or registrant shall ensure that each access to a high radiation area has one or more of the following control features:

(a) A device that, upon an attempt at entry and before any opening into the area occurs, reduces the level of radiation. Before an opening occurs the level of radiation shall be below that at which an individual could receive a deep–dose equivalent of 0.1 rem (1 mSv) in 1 hour at 30 centimeters from the source of radiation or any surface that the radiation penetrates;

(b) A device that emits a conspicuously visible or audible alarm so the individual entering the high radiation area and the supervisor of the activity are made aware of the entry; or

(c) Locked entryways, except when access to the area is required, with positive control over each individual entry.

(2) In the case of a high radiation area established for a period of 30 days or less, the licensee or registrant may substitute continuous direct or electronic surveillance to prevent unauthorized entry for the controls required in (1) of this rule.

(3) A licensee or registrant may apply to the Division for approval of alternative methods for controlling access to high radiation areas.

(4) No control required by (1) through (3) of this rule shall prevent individuals from leaving a high radiation area.

(5) Control is not required for each entrance or access point to a room or other area that is a high radiation area solely because of the presence of radioactive materials prepared for transport and packaged and labeled in accordance with the regulations of the U.S. Department of Transportation provided that:

(a) The packages do not remain in the area longer than 3 days; and

(b) The dose rate at 1 meter from the external surface of any package does not exceed 0.01 rem (0.1 mSv) per hour.

(6) Control of areas in hospitals is not required solely because of the presence of patients containing radioactive material, provided:

(a) There are personnel in attendance who will take necessary precautions to prevent exposure of individuals to radiation or radioactive material in excess of the limits in these standards; and

(b) The licensee operates within the ALARA provisions of its radiation protection program.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 69-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.81 CONTROL OF ACCESS TO VERY HIGH RADIATION AREAS.

In addition to the requirements in 1200–2–5–.80, the licensee or registrant shall institute additional measures to ensure that an individual is not able to gain unauthorized or inadvertent access to areas in which radiation levels could be encountered at 500 rads (5 grays) or more in 1 hour at 1 meter from a source of radiation or any surface through which the radiation penetrates.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.82 CONTROL OF ACCESS TO VERY HIGH RADIATION AREAS–IRRADIATORS.

(1) Each area in which there may exist radiation levels in excess of 500 rads (5 grays) in 1 hour at 1 meter from a radiation source [3] that is used to irradiate materials shall meet the following requirements:

(a) At least one authorized person who is familiar with the activity of the facility and is prepared to render or summon assistance shall be physically present when radiation is produced.

(b) Each installation shall have primary barriers and/or secondary barriers sufficient to assure compliance with 1200–2–5–.50, 1200–2–5–.55, 1200–2–5–.56 and 1200–2–5–.60 of these standards.

(c) Each irradiation area shall be constructed so that persons within the area shall at all times be able to leave. Access control devices required by 1200–2–5–.82(1)(h)2 through 4 shall not prevent an individual from leaving the area.

(d) Devices and administrative procedures shall control each area to ensure that the area is clear of individuals prior to irradiation.

(e) After any use of the radiation source and prior to the first individual's entry into the area, the area shall be surveyed to ensure that the radiation level in the area from the radiation source is below that at which an individual could receive a deep–dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour.

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[3] *This rule applies to radiation from radiation sources that are used in non–self–shielded configuration. This rule does not apply to sources of radiation that are used in teletherapy, in radiography, or in completely self–shielded irradiators in which the source is both stored and operated within the same shielding radiation barrier and, in the designed configuration of the equipment, is always physically inaccessible to any individual and cannot create high levels of radiation in an area that is accessible to any individual.*

(f) Control Panel:

1. Only the operator at the control panel shall be able to activate an irradiator to create a radiation field in any area.

2. The irradiator control panel shall be provided with a locking device to prevent unauthorized use. The locking device shall, when locked, make the irradiator incapable of creating a radiation field.

3. The control panel and each entrance to an irradiation area shall have a device that gives a continuous indication of the radiation levels present in the area(s).

4. All meters and controls on the irradiator control panel shall be identified and discernible.

5. The operator shall have at the control panel a copy of operating and emergency procedures specific for that facility.

(g) Warning Devices:

1. Each area shall have devices that automatically generate conspicuously visible and audible alarm signals for at least five (5) seconds before irradiation begins. Following activation of these warning devices, there shall be a delay of not less than thirty (30) seconds before the irradiation may begin. The alarm signals shall be discernible in all irradiation areas. The alarm signals shall be sufficient to alert personnel in the area and to allow any individual in the area to reach and to operate the clearly identified emergency shut–off switches required in 1200–2–5–.82(1)(h)1.

2. Each area shall have visible flashing or rotating warning lights that operate when, and only when, radiation is being produced. Each entrance shall have a visible warning device that need not be flashing or rotating, but which operates when, and only when, radiation is being produced.

(h) Control Devices:

1. Each area shall contain accessible emergency shut–off switches. Operation of an emergency shut–off switch shall prevent irradiation from occurring. These switches and their mode of operation shall be identified by a conspicuously posted sign adjacent to each switch. Shut–off switches shall include a manual reset at each switch that must be reset at the switch before the irradiator may be reactivated by the operator at the control panel.

2. Each entrance or access point shall be equipped with interlocks. When any interlock is interrupted, broken, or tripped and before any opening into the area occurs, either:

   (i) The irradiator shall shut off automatically; or

   (ii) The radiation level within the area from the radiation source shall be reduced below that at which an individual could receive a deep–dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour.

   After shut–off or reduction in output, restoring the irradiator to full operation shall be possible only from the control panel.

3. Additional control devices shall be provided so that, upon failure of the interlocks to function as required by 1200–2–5–.82(1)(h)2:

   (i) The radiation level within the area from the radiation source shall be reduced below that at which an individual could receive a deep–dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour; and

   (ii) Conspicuously visible and audible alarm signals shall be generated that make the following persons aware of the hazard and of the failure of the interlocks:

      (I) Any individual attempting to enter the area; and

      (II) The individual required to be present in (1)(a) of this rule.

4. Interlocks shall not be used to shut off the irradiator except in an emergency or during testing.

5. Interlocks shall be bypassed only to test, adjust, maintain, and/or rearrange equipment. A conspicuous indication of the bypassed condition shall be made at the control panel. This subparagraph does not authorize the operation of an irradiator with warning devices, interlocks, emergency shut–off switches or other control devices that are incapable of proper operation.

6. Activities in which interlocks are bypassed as permitted under 1200–2–5–.82(1)(h)5 shall be:

   (i) Authorized only by the radiation safety officer;

   (ii) Performed only for a specified time;

   (iii) Recorded, showing:

      (I) Date,

      (II) Length of time bypassed,

      (III) Reason for bypassing, and

      (IV) Signature of the individual installing and removing the bypass.

      These records shall be maintained for inspection by the Division; and

   (iv) Performed at low power and current, if possible.

7. No individual shall be permitted to enter an area, the access of which is controlled by interlocks, while such interlocks are bypassed as permitted in 1200–2–5–.82(1)(h)5., unless such individual is utilizing personnel monitoring equipment that shall give an audible indication when a dose rate of .015 rem (.15 mSv) per hour is exceeded. The personnel monitoring equipment referred to in this paragraph is in addition to that required elsewhere in these standards. Calibration requirements in 1200–2–5–.70(2) shall also apply to such personnel monitoring equipment.

8. The licensee or registrant shall provide control devices so that, upon failure or removal of physical radiation barriers other than a sealed source's shielded storage container:

(i) The radiation level within the area from the radiation source shall be reduced below that at which an individual could receive a deep–dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour; and

(ii) Conspicuously visible and audible alarm signals shall be generated that make the following persons aware of the hazard and of the failure or removal of the physical barrier:

(I) Any individual attempting to enter the area; and

(II) The individual required to be present in (1)(a) of this rule.

9. When the shield for the stored sealed source(s) is a liquid, the licensee shall provide means to monitor the integrity of the shield and to signal, automatically, loss of adequate shielding.

10. Physical radiation barriers that comprise permanent structural components, such as walls, which have no credible probability of failure or removal in ordinary circumstances need not meet the requirements of (1)(h)8 of this rule.

(i) There shall be available at each facility portable radiation monitoring equipment that is operable and has been calibrated for the radiations being produced by the facility. Such equipment shall be tested for operation and calibrated at intervals not to exceed three (3) months and after each instrument servicing or repair. A note shall be attached to each instrument showing the latest calibration date. Records of calibration shall be maintained for inspection by the Division.

(j) The interlock and emergency shut–off systems required in (1)(h) of this rule shall be separate electrical circuits and/or mechanical systems.

(k) Electrical circuit diagrams of the irradiator and the associated interlock and emergency shut–off systems shall be kept current and on file at each irradiator facility.

(l) The access control and warning devices required in 1200–2–5–.82(1)(g) and (h) shall have been tested for proper functioning (see 1200–2–5–.138 for recordkeeping requirements).

1. Unless irradiation was continued uninterrupted from the previous day, testing shall be conducted prior to daily initiation of irradiation;

2. After any unintended interruption, testing shall be conducted prior to resumption of irradiation; and

3. The licensee or registrant shall submit and adhere to a schedule for periodic tests of the access control and warning systems.

(m) The licensee or registrant shall not conduct operations, other than those necessary to place the radiation source in safe condition or to effect repairs on controls, unless control and warning devices are functioning properly.

(n) Portals used in transporting only materials to and from the irradiation area shall be controlled by devices and administrative procedures that warn and physically protect individuals from inadvertent entry. Exit portals shall be equipped to:

1. Detect and signal the presence of any loose radiation sources being carried toward such an exit; and

2. Automatically prevent loose radiation sources from being carried out of the area.

(o) Licensees, registrants or applicants may apply to the Division for approval of alternative safety measures for irradiators, provided:

1. The irradiator is within the purview of this rule;

2. The irradiator will be used in a variety of positions or locations (such as open fields or forests) that make it impractical to comply with certain requirements of (1)(h) of this rule (such as automatic control of radiation levels);

3. Any alternative safety measures shall provide a degree of personnel protection at least equivalent to those specified in this rule;

4. At least one of the alternative measures shall include an access–preventing interlock control based on a measurement of the radiation. This interlock control shall ensure that no individual can gain access to the area in which an individual could receive a deep–dose equivalent in excess of 0.1 rem (1 mSv) in 1 hour at 30 centimeters from the radiation source or any surface that the radiation penetrates.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.83 THROUGH 1200–2–5–.89 RESERVED.

1200–2–5–.90 USE OF PROCESS OR OTHER ENGINEERING CONTROLS.

The licensee shall use, to the extent practicable, process or other engineering controls (e.g., containment, decontamination or ventilation) to control the concentrations of radioactive material in air.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-2020206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.91 USE OF OTHER CONTROLS.

(1) The licensee shall maintain the total effective dose equivalent ALARA by limiting intakes and increased monitoring if process or other engineering controls are not practical to control airborne radioactive materials concentration below those contained in the definition of airborne radioactivity area in 1200–2–5–.32. The limitation of intakes and increased monitoring shall be by one or more of the following means:

(a) Control of access;

(b) Limitation of exposure times;

(c) Use of respiratory protection equipment; or

(d) Other mechanisms specifically approved by the Division.

(2) If the licensee performs an ALARA analysis to determine whether respirators should be used, the licensee may consider safety factors other than radiological factors. The licensee should also consider the impact of respirator use on workers' industrial health and safety.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. November 17, 2005; effective January 31, 2006.*

1200–2–5–.92 USE OF INDIVIDUAL RESPIRATORY PROTECTION EQUIPMENT.

(1) If the licensee assigns or permits the use of respiratory protection equipment to limit intakes pursuant to 1200–2–5–.91:

(a) The licensee shall use only respiratory protection equipment that is tested and certified or had certification extended by the National Institute for Occupational Safety and Health/Mine Safety and Health Administration (NIOSH/MSHA), except as otherwise noted in this chapter.

(b) A licensee desiring to use equipment that has not been tested or certified by NIOSH, or for which there is no schedule for testing or certification, shall apply for authorization except as provided in this chapter. The application shall demonstrate by licensee testing or on the basis of reliable test information, that the equipment's material and performance characteristics provide protection equivalent to that of the equipment in paragraph (1)(a) of this rule under anticipated conditions of use.

(c) The licensee shall implement and maintain a respiratory protection program that includes:

1. Air sampling sufficient to identify the potential hazard, permit proper equipment selection and estimate doses;

2. Surveys and bioassays, as appropriate, to evaluate actual intakes;

3. Testing of respirators for operability (user seal check for face sealing devices and functional check for other) immediately before each use;

4. Written procedures regarding:

(i) The routine, non–routine and emergency use of respirators,

(ii) Respirator selection,

(iii) Fit testing,

(iv) Limitations on periods of respirator use and relief from respirator use.

(v) Storage, issuance, maintenance, repair, testing and quality assurance of respiratory protection equipment, including testing for operability immediately before each use,

(vi) Supervision and training of respirator users,

(vii) Monitoring, including air sampling and bioassays,

(viii) Breathing air quality,

(ix) Inventory and control,

(x) Record keeping, and

(xi) The use of process or other engineering controls, instead of respirators;

5. Determination by a physician that the individual user is medically fit to use the respiratory protection equipment before:

(i) The initial fitting of a face–sealing respirator,

(ii) The first field use of non–face–sealing respirators, and

(iii) Either every 12 months thereafter or periodically at a frequency determined by a physician;

6. Fit testing, with fit factor ≥ 10 times the APF for negative pressure devices, and a fit factor ≥ 500 for any positive pressure, continuous flow, and pressure–demand devices, before the first field use of tight fitting, face–sealing respirators and periodically thereafter at a frequency not to exceed 1 year. Fit testing must be performed with the facepiece operating in the negative pressure mode.

(d) The licensee shall advise each respirator user that the user may leave the area at any time for relief from respirator use in the event of equipment malfunction, physical or psychological distress, procedural or communication failure, significant deterioration of operating conditions or any other conditions that might require such relief.

(e) The licensee's use of the equipment shall not exceed the equipment's specifications. The licensee shall provide proper visual, communication and other special capabilities (such as adequate skin protection) when needed.

(f) The licensee shall also consider limitations appropriate to the type and mode of use. When selecting respiratory devices the licensee shall provide for vision correction, adequate communication, low temperature work environments and the concurrent use of other safety or radiological protection equipment. The licensee shall use equipment in such a way as not to interfere with the proper operation of the respirator.

(g) Standby rescue persons are required whenever one-piece atmosphere-supplying suits, or any combination of supplied air respiratory protection device and personnel protective equipment are used from which an unaided individual would have difficulty extricating himself or herself. The standby persons shall be equipped with respiratory protection devices or other apparatus appropriate for the potential hazards. The standby rescue persons shall observe or otherwise maintain continuous communication with the workers (visual, voice, signal line, telephone, radio, or other suitable means), and be immediately available to assist them in case of a failure of the air supply or for any other reason that requires relief from distress. A sufficient

number of standby rescue persons shall be immediately available to assist all users of this type of equipment and to provide effective emergency rescue if needed.

(h) Atmosphere-supplying respirators shall be supplied with respirable air of grade D quality or better as defined by the Compressed Gas Association in publication G-7.1, ``Commodity Specification for Air,'' 1997 and included in the regulations of the Occupational Safety and Health Administration (29 CFR 1910.134(i)(1)(ii)(A) through (E). Grade D quality air criteria include:

1. Oxygen content (v/v) of 19.5-23.5%;

2. Hydrocarbon (condensed) content of 5 milligrams per cubic meter of air or less;

3. Carbon monoxide (CO) content of 10 ppm or less;

4. Carbon dioxide content of 1,000 ppm or less; and

5. Lack of noticeable odor.

(i) The licensee shall ensure that no objects, materials or substances, such as facial hair, or any conditions that interfere with the face--facepiece seal or valve function, and that are under the control of the respirator wearer, are present between the skin of the wearer's face and the sealing surface of a tight-fitting respirator facepiece.

(j) In estimating the dose to individuals from intake of airborne radioactive materials, the concentration of radioactive material in the air that is inhaled when respirators are worn is initially assumed to be the ambient concentration in air without respiratory protection, divided by the assigned protection factor. If the dose is later found to be greater than the estimated dose, the corrected value shall be used. If the dose is later found to be less than the estimated dose, the corrected value may be used.

(2) In estimating an individual's exposure to airborne radioactive materials, the licensee may make allowance for respiratory protection equipment used to limit intakes pursuant to 1200–2–5–.91. To make such an allowance the following conditions, in addition to those in 1200–2–5–.92(1) shall be satisfied:

(a) The licensee selects respiratory protection equipment that provides a protection factor (see Schedule RHS 8–32) greater than the multiple by which peak concentrations of airborne radioactive materials in the working area are expected to exceed the values specified in Schedule RHS 8–30, Table 1, Column 3. If the selection of a respiratory protection device with a protection factor greater than the peak concentrations is inconsistent with the goal specified in 1200–2–5–.91 of keeping the total effective dose equivalent ALARA, the licensee may select respiratory protection equipment with a lower protection factor only if such a selection would result in keeping the total effective dose equivalent ALARA. The concentration of radioactive material inhaled when respirators are used may be initially estimated by dividing the average concentration in air, during each period of uninterrupted respirator use, by the protection factor. If the exposure is later found to exceed the estimate, the corrected value shall be used; if the exposure is later found to be less than the estimate, the corrected value may be used.

(b) The licensee shall obtain authorization from the Division before assigning respiratory protection factors in excess of those specified in Schedule RHS 8–32.

The Division may authorize a licensee to use higher protection factors on receipt of an application that:

1. Describes the situation for which a need exists for higher protection factors; and

2. Demonstrates that the respiratory protection equipment provides these higher protection factors under the proposed conditions of use.

(c) The licensee shall use as emergency devices only respiratory protection equipment that has been specifically certified or had certification extended for emergency use by NIOSH/MSHA.

(d) The licensee shall notify, in writing, the Division at least 30 days before the date that respiratory protection equipment is first used under the provisions of either 1200–2–5–.92 (1) or (2).

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994 Amendment filed July 18, 2002; effective October 1, 2002. Repeal and new rule filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.93 FURTHER RESTRICTIONS ON THE USE OF RESPIRATORY PROTECTION EQUIPMENT.

(1) The Division may impose restrictions in addition to those in 1200–2–5–.91, 1200–2–5–.92 and Schedule RHS 8–32 to:

(a) Ensure that the respiratory protection program of the licensee is adequate to limit doses of individuals from intakes of airborne radioactive materials consistent with maintaining total effective dose equivalent ALARA; and

(b) Limit the extent to which a licensee may use respiratory protection equipment instead of process or other engineering controls.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.94 APPLICATION FOR USE OF HIGHER ASSIGNED PROTECTION FACTORS.

(1) The licensee shall obtain authorization from the Division before using assigned respiratory protection factors in excess of those specified in Schedule RHS 8–32. The Division may authorize a licensee to use higher protection factors on receipt of an application that:

(a) Describes the situation for which a need exists for higher protection factors; and

(b) Demonstrates that the respiratory protection equipment provides these higher protection factors under the proposed conditions of use.

(2) Reserved.

Authority: T.C.A. §§4–5–201 et seq. and 68–202–201 et seq. **Administrative History**: Otriginal rule filed *November 17, 2005; effective January 31, 2006.*

1200–2–5–.95 THROUGH 1200–2–5–.99 RESERVED.

1200–2–5–.100 SECURITY OF STORED MATERIAL.

The licensee or registrant shall secure stored radiation sources against unauthorized access or removal.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.101 CONTROL OF MATERIAL NOT IN STORAGE.

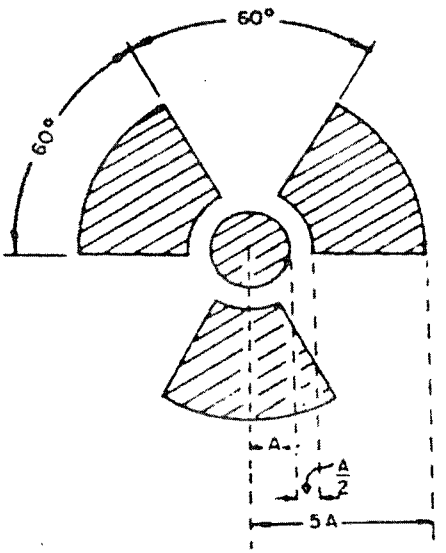
The licensee shall control and maintain constant surveillance of radioactive material that is not in storage.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.102 THROUGH 1200–2–5–.109 RESERVED.

1200–2–5–.110 CAUTION SIGNS.

(1) Unless otherwise authorized by the Division, the standard radiation symbol prescribed by this chapter shall use the colors magenta, or purple, or black on yellow background. The symbol prescribed by this chapter is the three–bladed design:



RADIATION SYMBOL

(a) Cross–hatched area is to be magenta, or purple, or black; and

(b) The background is to be yellow.

(2) The color requirements of (1) do not apply to licensees and registrants who use conspicuously etched or stamped radiation symbols to label sources, source holders or device components containing sources of radiation that are subjected to high temperatures.

(3) On or near the required signs and labels, the licensee or registrant may provide additional information to make individuals aware of potential radiation exposures and to minimize the exposures.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.111 POSTING REQUIREMENTS.

(1) The licensee or registrant shall post each radiation area with a conspicuous sign or signs bearing the radiation symbol and the words "CAUTION, RADIATION AREA."

(2) The licensee or registrant shall post each high radiation area with a conspicuous sign or signs bearing the radiation symbol and the words "CAUTION, HIGH RADIATION AREA" or "DANGER, HIGH RADIATION AREA."

(3) The licensee or registrant shall post each very high radiation area with a conspicuous sign or signs bearing the radiation symbol and words "GRAVE DANGER, VERY HIGH RADIATION AREA."

(4) The licensee shall post each airborne radioactivity area with a conspicuous sign or signs bearing the radiation symbol and the words "CAUTION, AIRBORNE RADIOACTIVITY AREA" or "DANGER, AIRBORNE RADIOACTIVITY AREA."

(5) Each area where radioactive material is used or stored in amounts exceeding 10 times that specified in Schedule RHS 8–31 shall be posted by the licensee with conspicuous sign(s) bearing the radiation symbol and the words "CAUTION, RADIOACTIVE MATERIAL(S)" or "DANGER, RADIOACTIVE MATERIAL(S)."

(6) A licensee is not required to post caution signs in areas or rooms containing radioactive materials for periods of less than 8 hours, if each of the following conditions is met:

(a) The materials are constantly attended during these periods by an individual who takes the precautions necessary to prevent the exposure of individuals to radiation or radioactive materials in excess of the limits established in this chapter; and

(b) The area or room is subject to the licensee's control.

(7) Rooms or other areas in hospitals that are occupied by patients are not required to be posted with caution signs pursuant to 1200–2–5–.111 provided that:

(a) The patient is being treated with sealed sources or has been treated with unsealed radioactive material in quantities less than 30 millicuries (110 MBq) or the measured dose rate at 1 meter from the patient is less than 0.005 rem (0.05 mSv) per hour; and

(b) There are personnel in attendance who will take the necessary precautions to:

1. Prevent the exposure of individuals to radiation and radioactive material in excess of these standards; and

2. Operate within the ALARA provisions of the licensee's radiation protection program.

(8) A room or area is not required to be posted with a caution sign because of the presence of a sealed source provided the radiation level at 30 centimeters from the surface of the source container or housing does not exceed 0.005 rem (0.05 mSv) per hour.

(9) A room containing medical or dental diagnostic x–ray equipment, restricted to use within the room, need not be posted as noted in 1200–2–5–.111(1) and (2) provided:

(a) The registrant exercises control to ensure the patient will be the only person exposed to radiation levels exceeding the limits in these standards; and

(b) Each room entrance is identified as an "X–ray Room".

(10) Provided a room or area is not otherwise required to be posted under paragraphs (1) or (2) of this rule, a room or area will not have to be so posted because mobile or portable medical or dental diagnostic x–ray equipment is intermittently used between rooms and/or areas.

(11) All radiation machines shall be clearly labeled at the control panel near the switch that energizes the apparatus, and at any remote switched that energize the apparatus, with the words "CAUTION – RADIATION – THIS EQUIPMENT PRODUCES RADIATION WHEN ENERGIZED" or "DANGER – RADIATION – THIS EQUIPMENT PRODUCES RADIATION WHEN ENERGIZED"

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.112 RESERVED.

1200–2–5–.113 LABELING CONTAINERS.

(1) The licensee shall ensure that each container of radioactive material bears a durable, clearly visible label bearing the radiation symbol and the words "CAUTION, RADIOACTIVE MATERIAL" or "DANGER, RADIOACTIVE MATERIAL." The label shall also provide sufficient information to permit individuals handling, using or in the vicinity of the containers to take precautions to avoid or minimize exposures. Such information may need to include, without limitation, the radionuclide(s) present, an estimate of the quantity of radioactivity, the date for which the activity is estimated, radiation levels, the kinds of material and the mass enrichment.

(2) Prior to removal or disposal of empty uncontaminated containers to unrestricted areas, the licensee shall:

(a) Remove or deface the radioactive material label; or

(b) Otherwise clearly indicate that the container no longer contains radioactive materials.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.114 EXEMPTIONS TO LABELING REQUIREMENTS.

(1) A licensee is not required to label:

(a) Containers holding radioactive material in quantities less than the quantities listed in Schedule RHS 8–31;

(b) Containers holding radioactive material in concentrations less than those specified in Table 2 of Schedule RHS 8–30;

(c) Containers attended by an individual who takes the precautions necessary to prevent the exposure of individuals in excess of the limits established by this chapter;

(d) Containers when they are in transport and packaged and labeled in accordance with the regulations of the U.S. Department of Transportation [4];

(e) Containers that are accessible only to individuals authorized to handle, use or be in the vicinity of the containers, if the contents are identified to these individuals by a readily available written record. Examples of containers of this type are containers in locations such as water–filled canals, storage vaults or hot cells. The record shall be retained as long as the containers are in use for the purpose indicated on the record; or

(f) Installed manufacturing or process equipment, such as reactor components, piping, and tanks.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.115 PROCEDURES FOR RECEIVING AND OPENING PACKAGES.

(1) Each licensee who expects to receive a package containing quantities of radioactive material in excess of a Type A quantity, as defined in subparagraph 1200–2–4–.04(1)(iii), shall arrange to receive:

(a) The package when the carrier offers it for delivery; or

(b) Notification of the arrival of the package at the carrier's terminal and to take possession of the package expeditiously.

(2) Each licensee shall:

[4] *Labeling of packages containing radioactive materials is required by the U.S. Department of Transportation (DOT) if the amount and type of radioactive material exceeds the limits for an excepted quantity or article as defined and limited by DOT regulations 49 C.F.R. 173.403 (m) and (w) and 173.421–424.*

(a) Monitor the external surfaces of a labeled [5] package for radioactive contamination unless the package contains only radioactive material in the form of a gas or in special form as defined in subparagraph 1200–2–4–.04(1)(bbb);

(b) Monitor the external surfaces of a labeled package for radiation levels unless the package contains quantities of radioactive material that are less than or equal to the Type A quantity, as defined in subparagraph 1200–2–4–.04(1)(iii) and Rule 1200–2–10–.37, Schedule RHS 10–6; and

(c) Monitor all packages known to contain radioactive material for radioactive contamination and radiation levels if there is evidence of degradation of package integrity, such as packages that are crushed, wet or damaged.

(3) The licensee shall monitor as soon as practical after receipt of the package. A package received at the licensee's facility during the licensee's normal working hours or showing evidence of package degradation shall be monitored within three (3) hours. A package not received during the licensee's normal working hours and not showing evidence of package degradation shall be monitored no later than three (3) hours after the beginning of the next working day.

(4) The licensee shall immediately notify the final delivery carrier and the Division by telephone, telegram, mailgram or facsimile when either removable radioactive surface contamination or external radiation levels exceed the following:

(a) Removable radioactive surface contamination limits:

1. The level of removable (non–fixed) radioactive contamination on the external surfaces of each package offered for transport shall be kept ALARA. The level of removable radioactive contamination may be determined by wiping an area of 300 square centimeters of the surface concerned with an absorbent material, using moderate pressure, and measuring the activity on the wiping material. Sufficient measurements shall be taken in the most appropriate locations to yield a representative assessment of the removable contamination levels. Except as provided in part 1200–2–5–.115(4)(a)2, the amount of radioactivity measured on any single wiping material, when averaged over the surface wiped, shall not exceed the limits set forth in Table RHS 5–3 at any time during transport. Other methods of assessment of equal or greater efficiency may be used. When other methods are used, the detection efficiency of the method used shall be taken into account and in no case shall the removable contamination on the external surfaces of the package exceed ten (10) times the limits set forth in Table RHS 5–3.

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[5] *Labeled means labeled with a Radioactive White I, Yellow II or Yellow III label as specified in U.S. Department of Transportation (DOT) regulations in 49 CFR §§172.403 and 172.436–440, as published October 1, 1993.*

Table RHS 5–3 REMOVABLE EXTERNAL RADIOACTIVE CONTAMINATION WIPE LIMITS

Contaminant	Maximum Permissible Limits		
	Bq/cm^2	$\mu Ci/cm^2$	dpm/cm^2
Beta and gamma emitters and low toxicity alpha emitters; all radionuclides with half–lives less than 10 days; natural uranium; natural thorium; uranium–235; uranium–238; thorium–232; thorium–228; and thorium–230 when contained in ores or physical concentrates	0.37	1 (E–5)	22
All other alpha emitting radionuclides	0.037	1 (E–6)	2.2

2. For packages transported as exclusive use shipments by rail or highway only, the removable contamination at any time during transport shall not exceed ten (10) times the levels prescribed in Table RHS 5–1. The levels at the beginning of transport shall not exceed the levels prescribed in Table RHS 5–1.

(b) External radiation limits:

1. The external radiation levels around the package and around the vehicle, if applicable, shall not exceed 200 millirems (2 millisieverts) per hour at any point on the external surface of the package at any time during transportation. The transport index shall not exceed 10.

2. A package that exceeds the radiation level limits specified in part 1200–2–5–.115(4)(b)1. shall be transported as exclusive use by rail, highway, or water, and the radiation levels external to the package shall not exceed the following during transportation:

(i) 200 millirems (2 millisieverts) per hour on the accessible external surface of the package, unless the following conditions are met, in which case the limit is 1,000 millirems (10 millisieverts) per hour:

(I) The shipment is made in a closed transport vehicle;

(II) The package is secured within the vehicle so that its position remains fixed during transportation; and

(III) There are no loading or unloading operations between the beginning and end of the transportation;

(ii) Two hundred (200) millirems (2 millisieverts) per hour at any point on the outer surface of the vehicle, including the top and underside of the vehicle, or in the case of a flat–bed style vehicle, at any point on the vertical planes projected from the outer edges of the vehicle, on the upper surface of the load, or enclosure, if used, and on the lower external surface of the vehicle; and

(iii) Ten (10) millirems (0.1 millisievert) per hour at any point 2 meters (6.6 feet) from the outer lateral surfaces of the vehicle (excluding the top and underside of the vehicle); or in the case of a flat–bed style vehicle, at any

point 2 meters from the vertical planes projected from the outer edges of the vehicle (excluding the top and underside of the vehicle); and

(iv) Two (2) millirems (0.02 millisievert) per hour in any normally–occupied space of the vehicle, except that this provision does not apply to private motor carriers if persons occupying these spaces wear radiation monitoring devices in accordance with Rule 1200–2–5–.71.

(5) Each licensee shall:

(a) Establish, maintain and retain written procedures for safely opening packages in which radioactive material is received; and

(b) Ensure that the procedures are followed and that due consideration is given to special instructions for the type of package being opened.

(6) Licensees transferring special form sources to or from a work site in licensee owned or operated vehicles are exempt from the contamination monitoring requirements of paragraph (2) of this rule. Licensees are not exempt from the requirement in paragraph (2) for surveying radiation levels to ensure that the source is still properly secured in its shield.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.116 THROUGH 1200–2–5–.119 RESERVED.

1200–2–5–.120 GENERAL DISPOSAL REQUIREMENTS.

(1) A licensee shall dispose of radioactive material only:

(a) By transfer to an authorized recipient as provided in other chapters of these regulations;

(b) By decay in storage;

(c) By release in effluents within the limits in 1200–2–5–.60; or

(d) As authorized under 1200–2–5–.121, 1200–2–5–.122, 1200–2–5–.123 or 1200–2–5–.124.

(2) A person shall be specifically licensed to receive waste containing licensed material from other persons for:

(a) Treatment prior to disposal;

(b) Treatment or disposal by incineration;

(c) Decay in storage; or

(d) Disposal at a land disposal facility licensed under Chapter 1200–2–11.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.121 METHOD FOR GRANTING APPROVAL OF ALTERNATIVE DISPOSAL PROCEDURES.

(1) A licensee or applicant for a license may apply to the Division for approval of alternative procedures for disposal of radioactive material generated in the licensee's activities. Each application shall include:

(a) A description of the waste that contains the radioactive material to be disposed, including the physical and chemical properties important to risk evaluation;

(b) The proposed manner and conditions of waste disposal;

(c) An analysis and evaluation of pertinent information about the environment of the disposal site;

(d) The nature and location of other potentially affected licensed and unlicensed facilities; and

(e) Analyses and procedures to ensure that doses are maintained ALARA and within the dose limits in this chapter.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.122 DISPOSAL BY RELEASE INTO SANITARY SEWERAGE.

(1) A licensee may release radioactive material into sanitary sewerage if each of the following conditions is satisfied:

(a) The material is readily soluble in water or is a readily dispersible biological material; and

(b) The quantity of radioactive material the licensee releases into the sewer in any one month divided by the average monthly volume of water released into the sewer by the licensee does not exceed the concentration listed in Table III of Schedule RHS 8–30; and

(c) If more than one radionuclide is released, the following conditions shall also be satisfied:

1. The licensee shall determine the fraction of the limit in Table III of Schedule RHS 8–30 represented by its releases into sanitary sewerage. This shall be done by dividing the actual monthly average concentration of each radionuclide released by the licensee into the sewer by the concentration of that radionuclide listed in Table III of Schedule RHS 8–30; and

2. The sum of the fractions for each radionuclide required by (1)(c)1. of this rule does not exceed unity; and

(d) The total quantity of licensed and other radioactive material that the licensee releases into the sanitary sewerage system in a year does not exceed:

1. 5 curies (185 GBq) of hydrogen–3;

2. 1 curie (37 GBq) of carbon–14; and

3. 1 curie (37 GBq) of all other radioactive materials combined.

(2) Excreta from individuals undergoing medical diagnosis or therapy with radioactive material are not subject to the limitations contained in (1) of this rule.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed April18, 2006 ; effective July 2, 2006.*

1200–2–5–.123 TREATMENT OR DISPOSAL BY INCINERATION.

A licensee may treat or dispose of radioactive material by incineration only in the amounts and forms specified in 1200–2–5–.124 or as specifically approved by the Division pursuant to 1200–2–5–.121.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.124 DISPOSAL OF SPECIFIC WASTES.

(1) A licensee may dispose of the following radioactive material as if it were not radioactive:

(a) 0.05 microcurie (1.85 kBq), or less, of hydrogen–3 or carbon–14 per gram of medium used for liquid scintillation counting; and

(b) 0.05 microcurie (1.85 kBq), or less, of hydrogen–3 or carbon–14 per gram of animal tissue, averaged over the weight of the entire animal.

(2) A licensee may not dispose of tissue under paragraph (1)(b) of this rule in a manner that would permit its use either as food for humans or as animal feed.

(3) The licensee shall maintain records in accordance with 1200–2–5–.137.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.125 TRANSFER FOR DISPOSAL AND MANIFESTS.

(1) This rule and Schedule RHS 8–33 concern low level radioactive waste and are to:

(a) Control transfers of low–level radioactive waste by any waste generator, waste collector or waste processor licensee, as defined in Schedule RHS 8–33 of Rule 1200–2–5–.161, who ships low–level waste either directly, or indirectly through a waste collector or waste processor, to a licensed low–level waste land disposal facility as defined in Chapter 1200–2–11;

(b) Establish a manifest tracking system; and

(c) Supplement existing requirements concerning transfers and recordkeeping for those wastes.

(2) Any licensee shipping radioactive waste intended for ultimate disposal at a licensed land disposal facility shall document the information required on U.S. NRC Uniform Low–Level

Radioactive Waste Manifest and transfer this recorded manifest information to the intended consignee as specified in Section I of Schedule RHS 8–33.

(3) Each shipment manifest shall include a certification by the waste generator as specified in Section II of Schedule RHS 8–33.

(4) The waste generator, collector, processor, disposal facility operator, and each person involved in the transfer and disposal shall comply with the requirements specified in Section III of Schedule RHS 8–33.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.126 COMPLIANCE WITH ENVIRONMENTAL AND HEALTH PROTECTION REGULATIONS.

Nothing in these standards relieves the licensee from complying with other federal, state, and local regulations governing toxic or hazardous properties of waste materials.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.127 THROUGH 1200–2–5–.129 RESERVED.

1200–2–5–.130 GENERAL RECORDS PROVISIONS.

(1) Each licensee and registrant shall use the units: curie, rad, rem, including multiples and subdivisions, and shall clearly indicate the units of all quantities on records required by these standards.

(2) In the records required by this part, the licensee may record quantities in SI units in parentheses following each of the units specified in paragraph (1). However, all quantities must be recorded as stated in paragraph (1).

(3) Notwithstanding the requirements above in paragraph (1), when recording information on shipment manifests, as required in paragraph 1200–2–5–.125(2), information shall be recorded in the International System of Units (SI) or in SI and units as specified in paragraph (1).

(4) The licensee or registrant shall make a clear distinction among the quantities entered on the records required by this chapter (e.g., total effective dose equivalent, shallow–dose equivalent, lens dose equivalent, deep–dose equivalent, committed effective dose equivalent).

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., and 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.131 RECORDS OF RADIATION PROTECTION PROGRAMS.

(1) Each licensee and registrant shall maintain records of the radiation protection program, including:

(a) The provisions of the program; and

(b) Audits and other reviews of program content and implementation.

(2) The licensee or registrant shall retain the records required by (1)(a) of this rule until the Division terminates each pertinent license or registration requiring the record. The licensee or registrant shall retain the records required by (1)(b) of this rule for 3 years after the record is made.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.132 RECORDS OF SURVEYS.

(1) Each licensee and registrant shall maintain records showing the results of surveys and calibrations required by 1200–2–5–.70 and 1200–2–5–.115(2). The licensee or registrant shall retain these records for 3 years after the record is made.

(2) The licensee or registrant shall retain each of the following records until the Division terminates each pertinent license or registration requiring the record:

(a) Survey results used to determine the dose from external sources and to assess individual dose equivalents with or without individual monitoring data;

(b) Results of measurements and calculations used to:

1. Determine individual intakes of radioactive material;

2. Assess internal intakes of radioactive material; and

3. Assess internal dose;

(c) Results of air sampling, surveys and bioassays required pursuant to 1200–2–5–.92(1)(c)1. and 2.; and

(d) Results of measurements and calculations used to evaluate the release of radioactive effluents to the environment.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.133 DETERMINATION OF PRIOR OCCUPATIONAL DOSE.

(1) For each individual who is likely to receive, in a year, an occupational dose requiring monitoring pursuant to 1200–2–5–.71, the licensee or registrant shall:

(a) Determine the occupational radiation dose received during the current year; and

(b) Attempt to obtain the records of lifetime cumulative occupational radiation dose.

(2) Prior to permitting an individual to participate in a planned special exposure, the licensee or registrant shall determine:

(a) The internal and external doses from all previous planned special exposures; and

(b) All doses in excess of the limits (including doses received during accidents and emergencies) received during the lifetime of the individual.

(3) In complying with the requirements of (1) of this rule, a licensee or registrant may:

(a) Accept, as a record of the individual's occupational dose for the current year, a written statement disclosing the nature and the amount of any occupational dose the individual may have received during the current year. Such statement shall be signed by the individual or the individual's most recent employer for work involving radiation exposure.

(b) Accept, as the record of lifetime cumulative radiation dose, an up–to–date Form RHS 8–1H, or equivalent. Such form shall be signed by the individual and countersigned by an appropriate official of the most recent employer for work involving radiation exposure. If the individual is employed by a person other than the licensee or registrant, the countersignature shall be from the current employer.

(c) From the most recent employer obtain reports of the individual's dose equivalent(s) for work involving radiation exposure. If the individual is employed by a person other than the licensee or registrant the report shall be from the individual's current employer. Reports may be obtained by telephone, telegram, electronic media or letter. The licensee or registrant shall request a written verification of the dose data if the authenticity of the transmitted report cannot be established.

(4) The licensee or registrant shall record the exposure history together with all information required by (1) of this rule on Form RHS 8–1H [6], or other clear and legible record. The form or record shall show each period in which the individual received occupational exposure and be signed by the individual receiving the exposure.

For each period for which the licensee or registrant obtains reports, the licensee or registrant shall use the dose shown in the report in preparing Form RHS 8–1H. For any period in which the licensee or registrant does not obtain a report, the licensee or registrant shall place a notation on Form RHS 8–1H indicating the periods of time for which data are not available.

(5) If the licensee or registrant is unable to obtain a complete record of an individual's current and previously accumulated occupational dose, the licensee or registrant shall:

(a) In establishing administrative controls under 1200–2–5–.50(6) for the current year, reduce the individual's allowable dose limit by 1.25 rems (12.5 mSv) for each quarter for which records were unavailable and the individual could have received occupational exposure; and

(b) Not allow the individual to be available for planned special exposures.

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[6] *Licensees or registrants are not required to reevaluate the separate external dose equivalents and internal committed dose equivalents or intakes of radionuclides assessed under 1200–2–5–.01 through 1200–2–5–.28. Futher, occupational exposure histories obtained and recorded on Form RHS 8–1 before January 1, 1994, would not have included effective dose equivalent, but may be used in the absence of specific information on the intake of radionuclides by the individual.*

(6) The licensee or registrant shall retain the records on Form RHS 8–1H or equivalent until the Division terminates each pertinent license or registration requiring this record. The licensee or registrant shall retain records used in preparing Form RHS 8–1H for three (3) years after the record is made.

Authority: *T.C.A. §§4–5–201 et seq., 68-202-101, 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.134 RECORDS OF PLANNED SPECIAL EXPOSURES.

(1) For each use of the provisions of 1200–2–5–.54 for planned special exposures, the licensee or registrant shall maintain records that describe:

(a) The exceptional circumstances requiring the use of a planned special exposure;

(b) The name of the management official who authorized the planned special exposure and a copy of the signed authorization;

(c) What actions were necessary;

(d) Why the actions were necessary;

(e) How doses were maintained ALARA; and

(f) What individual and collective doses were expected to result, and the doses actually received in the planned special exposure.

(2) The licensee or registrant shall retain the records until the Division terminates each pertinent license or registration requiring these records.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.135 RECORDS OF INDIVIDUAL MONITORING RESULTS.

(1) Each licensee and registrant shall maintain records of doses received:

(a) By all individuals for whom monitoring was required pursuant to 1200–2–5–.71 and

(b) During the planned special exposures, accidents and emergency conditions.

(2) These records shall include [7], when applicable:

(a) The deep–dose equivalent to the whole body, lens–dose equivalent, shallow–dose equivalent to the skin and shallow–dose equivalent to the extremities;

(b) The estimated intake of radionuclides (see 1200–2–5–.51);

(c) The committed effective dose equivalent assigned to the intake of radionuclides;

[7] *Assessments of dose equivalent and records made using units in effect before the licensee's or registrant's adoption of 1200–2–5–.30 through 1200–2–5–.160 need not be changed.*

(d) The specific information used to assess the committed effective dose equivalent pursuant to 1200–2–5–.53(3) and when required by 1200–2–5–.71;

(e) The total effective dose equivalent when required by 1200–2–5–.51; and

(f) The total of the deep–dose equivalent and the committed dose to the organ receiving the highest total dose.

(3) The licensee or registrant shall make entries of the records specified in (1) of this rule at least annually.

(4) The licensee or registrant shall maintain the records:

(a) On Form RHS 8–2C and in accordance with its instructions, or

(b) In clear and legible form containing all information required by Form RHS 8–2C.

(5) The records required under this rule should be protected from public disclosure because of their personal privacy nature. These records are protected when transferred to the Division under the regulations in 1200–2–4–.10.

(6) The licensee or registrant shall maintain the records of dose to an embryo/fetus with the records of dose to the declared pregnant woman. The declaration of pregnancy shall also be kept on file, but may be maintained separately from the dose records.

(7) The licensee or registrant shall retain each required form or record until the Division terminates each pertinent license or registration requiring the record.

Authority: *T.C.A. §§4–5–201 et seq., 68-202-101 et seq., 68–202–201 et seq. 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.136 RECORDS OF DOSE TO INDIVIDUAL MEMBERS OF THE PUBLIC.

(1) Each licensee and registrant shall maintain records sufficient to demonstrate compliance with the dose limit for individual members of the public (see 1200–2–5–.60).

(2) The licensee or registrant shall retain the records required by (1) of this rule until the Division terminates each pertinent license or registration requiring the record.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.137 RECORDS OF WASTE DISPOSAL.

(1) Each licensee shall maintain records of the disposal of radioactive materials made under 1200–2–5–.121, 1200–2–5–.122, 1200–2–5–.123, 1200–2–5–.124, Chapter 1200–2–11 and disposal by burial in soil, including burials authorized before May 12, 1986 [8].

[8] *A previous 1200–2–5–.19 permitted burial of small quantities of radioactive materials in soil before May 12, 1986 without specific Division Authorization.*

(2) The licensee shall retain the records required by (1) of this rule until the Division terminates each pertinent license requiring the record.

Authority: T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206. ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.138 RECORDS OF TESTING ENTRY CONTROL DEVICES FOR VERY HIGH RADIATION AREAS.

(1) Each licensee and registrant shall maintain records of tests made under 1200–2–5–.82(1)(l)1., 2., and 3. on entry control devices for very high radiation areas. These records shall include the date, time, and results of each such test of function.

(2) The licensee or registrant shall retain the records required by paragraph (1) for three (3) years after the record is made.

Authority: T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206. ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.139 FORM OF RECORDS.

Each record required by this chapter shall remain legible throughout the retention period. The record may be the original or a reproduced copy or a microform provided that the copy or microform is authenticated by authorized personnel. The microform shall be capable of producing a clear copy throughout the retention period. The record may also be stored in electronic media capable of producing legible, accurate, and complete records during the retention period. Records such as letters, drawings, and specifications shall include all pertinent information, such as stamps, initials, and signatures. The licensee or registrant shall maintain adequate safeguards against tampering with and loss of records.

Authority: T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206. ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.140 REPORTS OF THEFT OR LOSS OF LICENSED MATERIAL.

(1) Telephone reports.

(a) Each licensee shall report:

1. Immediately after learning of any lost, stolen or missing radioactive material:

(i) In an aggregate quantity equal to or greater than 1,000 times the quantity specified in Schedule RHS 8–31; and

(ii) Under such circumstances that it appears to the licensee that an exposure could result to persons in unrestricted areas; or

2. Within 30 days after learning of any lost, stolen or missing radioactive material:

(i) In a quantity greater than 10 times the quantity specified in Schedule RHS 8–31; and

(ii) That is still missing at this time.

(b) Reports shall be made to the Division, telephone (615) 532–0364, during the hours of 7:00 a.m. Central Time to 4:30 p.m. Central Time except weekends and holidays. At all other times, reports can be made through the Tennessee Emergency Management Agency (615) 741–0001.

(2) Written reports

(a) Each licensee required to make a report under (1) of this rule shall, within 30 days after making the telephone report, make a written report setting forth the following information:

1. A description of the radioactive material involved, including kind, quantity and chemical and physical form;

2. A description of the circumstances under which the loss, theft or misplacement occurred;

3. A statement of disposition, or probable disposition, of the radioactive material involved;

4. Exposures of individuals to radiation and the circumstances under which the exposures occurred;

5. The possible total effective dose equivalent to persons in unrestricted areas;

6. Actions that have been taken, or will be taken, to recover the material; and

7. Procedures or measures that have been, or will be, adopted to ensure against a recurrence of the loss, theft or misplacement of radioactive material.

(b) Reports shall be made to the Division of Radiological Health, L&C Annex, 3rd Floor, 401 Church Street, Nashville, TN 37243–1532.

(3) If after filing the written report, the licensee learns of additional substantive information the licensee shall report such additional information within 30 days.

(4) Each report filed with the Division shall list for each individual exposed: the name, Social Security account number, and date of birth. The report shall be prepared so that this information is stated in a separate and detachable part.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.141 NOTIFICATION OF INCIDENTS.

(1) Immediate notification. Notwithstanding other requirements for notification the requirements of this rule are controlling. Licensees and registrants shall notify the Division as soon as possible but not later than four (4) hours after discovery that a source of radiation possessed by the licensee or registrant has caused, may have caused or threatens to cause any of the following:

(a) An individual to receive:

1. A total effective dose equivalent of 25 rems (0.25 Sv) or more;

2. A lens–dose equivalent of 75 rems (0.75 Sv) or more; or

3. A shallow–dose equivalent to the skin or extremities of 250 rads (2.5 Gy) or more;

(b) The release of radioactive material that could cause an individual present for 24 hours to receive five (5) times or more the annual occupational limit on intake. This does not apply to locations where personnel are not normally stationed during routine operations, such as hot–cells or specific process enclosures; or

(c) Prevention of immediate protective actions necessary to avoid exposure to radiation or releases that could exceed regulatory limits (events may include fires, explosions, toxic gas releases, etc.).

(2) Twenty–four hour notification. Licensees and registrants shall notify the Division within 24 hours after discovery that a source of radiation possessed by the licensee or registrant may have caused or threatens to cause any of the following:

(a) An individual to receive, in a period of 24 hours:

1. A total effective dose equivalent exceeding 5 rems (0.05 Sv),

2. A lens–dose equivalent exceeding 15 rems (0.15 Sv), or

3. A shallow–dose equivalent to the skin or extremities exceeding 50 rems (0.5 Sv);

(b) The release of radioactive material that could cause an individual present for 24 hours to receive an intake exceeding one annual occupational limit on intake. This does not apply to locations where personnel are not normally stationed during routine operations, such as hot–cells or specific process enclosures; or

(c) Any of the following events involving licensable material:

1. An unplanned contamination event that:

(i) Requires restricted access to the contaminated area for more than 24 hours. Restriction may be by imposing additional radiological controls or by prohibiting entry into the area;

(ii) Involves a quantity of material greater than five (5) times the lowest annual limit on intake specified for the material in Schedule RHS 8–30 of 1200–2–5; and

(iii) Restricts access to the area for a reason other than to allow isotopes with a half–life of less than 24 hours to decay prior to decontamination.

2. An event in which equipment is disabled or fails to function as designed when:

(i) The equipment is required by regulation or license condition to:

(I) Prevent releases exceeding regulatory limits,

(II) Prevent exposures to radiation exceeding regulatory limits, or

(III) Mitigate the consequences of an accident;

(ii) The equipment is required to be available and operable when it is disabled or fails to function; and

(iii) No equipment meeting the same performance standards is immediately available, operable and capable of performing the required safety function.

3. An event that requires unplanned medical treatment at a medical facility of an individual with spreadable radioactive contamination on the individual's clothing or body.

4. An unplanned fire or explosion damaging any licensable material or any device, container or equipment containing licensable material when:

(i) The quantity of material involved exceeds five (5) times the lowest annual limit on intake specified for the material in Schedule RHS 8–30 of 1200–2–5, and

(ii) The damage affects the integrity of the licensable material or any device, container or equipment containing licensable material.

(3) Preparation and submission of reports. Licensees and registrants shall make reports in response to the requirements of this section as follows:

(a) Licensees and registrants shall make reports required by paragraphs (1) and (2) of this rule by telephone to the Division.

1. The telephone number for the Division is:

(615) 532–0364 7:00 a.m. Central Time to 4:30 p.m. Central Time except weekends and holidays

(615) 741–0001 Tennessee Emergency Management Agency at all other times.

2. To the extent that the information is available at the time of notification, the information provided in these reports shall include:

(i) The caller's name and call back telephone number;

(ii) A description of the event, including date and time;

(iii) The exact location of the event;

(iv) The isotopes, quantities, and chemical and physical form of the licensable material involved; and

(v) Any personnel radiation exposure data available.

(b) Written report. Licensees and registrants who make a report required by paragraph (1) or (2) of this rule shall submit a written follow–up report within 30 days of the initial report. This requirement may be satisfied by submitting written reports prepared under other regulations that contain all necessary information and are appropriately

distributed. Licensees and registrants shall send these written reports to the Division at the address given in 1200–2–4–.07. The reports shall include the following:

1. A description of the event, including the probable cause and the manufacturer and model number (if applicable) of any equipment that failed or malfunctioned;

2. The exact location of the event;

3. The isotopes, quantities, and chemical and physical forms of the licensable material involved;

4. Date and time of the event;

5. Corrective actions taken or planned and the results of any evaluations or assessments; and

6. For each individual exposed:

   (i) The name, Social Security number and date of birth. The report shall be prepared so that this information is stated in a separate and detachable part, and

   (ii) The extent of exposure of each individual without identification of individuals by name.

(4) This rule does not include doses that result from, and are within the limits for, planned special exposures reported under 1200–2–5–.144.

Authority: *T.C.A. §§4–5–201 et seq., 68-202-101 et seq., 68–202–203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.142 REPORTS TO INDIVIDUALS OF EXPOSURE TO RADIATION.

(1) Licensees and registrants shall report radiation exposure data for an individual, including the results of any measurements, analyses and calculations of radioactive material deposited or retained in the body of an individual, as specified in this rule.

(2) Each licensee or registrant, at the request of any worker, shall advise such worker annually of the worker's exposure to sources of radiation as shown in records maintained by the licensee or registrant pursuant to Rule 1200–2–5–.135.

(3) Each licensee or registrant, at the request of a worker formerly engaged in licensed or registered activities controlled by the licensee or registrant, shall furnish to the worker a report of the individual's exposure to sources of radiation:

   (a) 1. As shown in records maintained by the licensee or registrant pursuant to Rule 1200–2–5–.135 for each year the worker was required to be monitored under the provisions of Rule 1200–2–5–.41; and

   2. For each year the worker was required to be monitored under the requirements in effect before January 2, 1993.

   (b) This report shall:

1. Be furnished within 30 days from the time the request is made or within 30 days after the exposure of the individual has been determined by the licensee, whichever is later;

2. Cover the period that the worker's activities involved exposure to sources of radiation licensed or registered by the Division; and

3. Include the dates and locations of licensed or registered activities in which the worker participated during this period.

(c) The worker's request shall include social security number, dates and location of employment or association and other appropriate identifying data.

(4) When a licensee or registrant is required under Rule 1200–2–5–.143 to report to the Division any exposure of an identified occupationally exposed individual or an identified member of the public to sources of radiation, the licensee or registrant shall also provide a copy of the report submitted to the Division to the individual. Such report shall be transmitted at a time not later than the transmittal to the Division.

(5) At the request of a worker who is terminating employment with the licensee or registrant that involved radiation dose, or of a worker who, while employed by another person, is terminating assignment to work involving radiation dose in the licensee's or registrant's facility during the current year, each licensee or registrant shall provide at termination to each worker, or to the worker's designee, a written report regarding the radiation dose received by that worker from operations of the licensee or registrant during the current year or fraction thereof. If the most recent monitoring results are not available at that time, the licensee or registrant shall provide a written estimate of the dose. Estimated doses shall be clearly indicated as such.

(6) Reports submitted under this rule shall:

(a) Be in writing;

(b) Include appropriate identifying data such as the name of the licensee or registrant, the name of the individual and the individual's social security number;

(c) Include the individual's radiation exposure information; and

(d) Include data and results obtained under Division regulations, or conditions, as shown in records maintained by the licensee or registrant under Division regulations

(e) Contain the following statement:

This report is furnished to you under the provisions of the Division of Radiological Health of the Tennessee Department of Environment and Conservation regulations entitled "State Regulations for Protection Against Radiation." You should preserve this report for future reference.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203, and 68-202-206.* ***Administrative History:*** *Original rule filed July 18, 2002; effectiveOctober 1, 2002.*

1200–2–5–.143 REPORTS OF EXPOSURES, RADIATION LEVELS, AND CONCENTRATIONS OF RADIOACTIVE MATERIAL EXCEEDING THE LIMITS.

(1) In addition to the notification required by 1200–2–5–.141, each licensee and registrant shall submit a written report within 30 days after learning of any of the following occurrences:

(a) Any incident for which notification is required by 1200–2–5–.141;

(b) Doses in excess of any of the following:

1. The occupational dose limits for adults in 1200–2–5–.50;

2. The occupational dose limits for minors in 1200–2–5–.55;

3. The limits for an embryo/fetus of a declared pregnant woman in 1200–2–5–.56;

4. The limits for an individual member of the public in 1200–2–5–.60;

5. Any applicable limit in the license or registration; or

6 The ALARA constraints for air emissions established under paragraph 1200–2–5–.40(4); or

(c) Levels of radiation or concentrations of radioactive material in:

1. A restricted area in excess of any applicable limit in the license or registration; or

2. An unrestricted area in excess of 10 times any limit set forth in these standards, the license or registration; whether or not there is exposure of any individual in excess of the limits in 1200–2–5–.60).

(d) Levels of radiation or releases of radioactive material exceeding EPA's generally applicable environmental standards in 40 C.F.R. 190, or license or registration conditions. This applies only if the licensee or registrant is subject to the standards.

(2) Contents of reports.

(a) Each report required by (1) of this rule shall describe the extent of exposure of individuals to radiation and radioactive material, including, as appropriate:

1. Estimates of each individual's dose;

2. The levels of radiation and concentrations of radioactive material involved;

3. The cause of the elevated exposures, dose rates or concentrations; and

4. Corrective steps taken or planned to ensure against a recurrence, including the schedule for achieving conformance with applicable limits, ALARA constraints, generally applicable environmental standards, and associated license or registration conditions.

(b) Each report filed under paragraph 1200–2–5–.143(1) shall include for each occupationally overexposed individual [9]: the name, Social Security account number, and date of birth. The report shall be prepared so that this information is stated in a separate and detachable part.

(3) All licensees or registrants who make reports under (1) of this rule shall submit the report in writing to the Division of Radiological Health, L&C Annex, 3rd Floor, 401 Church Street, Nashville, TN 37243–1532.

Authority: *T.C.A. §§4–5–201 et seq., 68-202-101 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed October 16, 1996; effective December 30, 1996. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.144 REPORTS OF PLANNED SPECIAL EXPOSURES.

The licensee or registrant shall submit a written report to the Division of Radiological Health, L&C Annex, 3rd Floor, 401 Church Street, Nashville, TN 37243–1532 within 30 days following any planned special exposure. The report shall inform the Division that a planned special exposure occurred and provide the information required by 1200–2–5–.134.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.145 NOTIFICATIONS, RECORDS AND REPORTS OF MISADMINISTRATION.

(1) For a misadministration:

(a) The licensee shall notify by telephone the Division at the number given in Rule 1200–2–4–.07 no later than the next calendar day after discovery of the misadministration.

(b) The licensee shall submit a written report to the Division at the address given in Rule 1200–2–4–.07 within 15 days after discovery of the misadministration.

1. The written report shall include:

(i) The licensee's name,

(ii) The prescribing physician's name,

(iii) A brief description of the event,

(iv) Why the event occurred,

(v) The effect on the individual who received the misadministration,

(vi) What improvements are needed to prevent recurrence,

(vii) Actions taken to prevent recurrence,

[9] *With respect to the limit for the embryo/fetus (1200–2–5–.56), the identifiers should be those of the declared pregnant woman..*

(viii) Whether the licensee notified the individual (or the individual's responsible relative or guardian) and if not, why not, and if there was notification, what information was provided.

2. The report shall not contain the individual's name or any other information that could lead to identification of the individual.

3. To meet the requirements of this rule, the notification of the individual receiving the misadministration may be made instead to that individual's responsible relative or guardian, when appropriate.

(c) The licensee shall notify the referring physician and also notify the individual receiving the misadministration of the misadministration no later than 24 hours after its discovery, unless the referring physician personally informs the licensee either that he will inform the individual or that, based on medical judgment, telling the individual would be harmful. The licensee is not required to notify the individual without first consulting the referring physician. If the referring physician or the individual receiving the misadministration cannot be reached within 24 hours, the licensee shall notify the individual as soon as possible thereafter. The licensee may not delay any appropriate medical care for the individual, including any necessary remedial care because of the misadministration, because of any delay in notification.

(d) If the individual was notified, the licensee shall also furnish, within 15 days after discovery of the misadministration, a written report to the individual by sending either:

1. A copy of the report that was submitted to the Division; or

2. A brief description of both the event and the consequences as they may affect the individual, provided a statement is included that the report submitted to the Division can be obtained from the licensee.

(2) Each licensee shall retain a record of each misadministration for five (5) years. The record shall contain:

(a) The names of all individuals involved (including the prescribing physician, allied health personnel, the individual who received the misadministration and that individual's referring physician, if applicable),

(b) The individual's social security number or other identification number if one has been assigned,

(c) A brief description of the misadministration, why it occurred, the effect on the individual, improvements needed to prevent recurrence and the actions taken to prevent recurrence.

(3) Aside from the notification requirement, nothing in this section affects any rights or duties of licensees and physicians in relation to each other, to individuals receiving misadministrations, or to that individual's responsible relatives or guardians.

***Authority**: T.C.A. §§4–5–201 et seq. and 68–202–203 and 68-202-206.* ***Administrative History:** Original rule filed July 18, 2002; effective October 1, 2002.*

1200–2–5–.146 THROUGH 1200–2–5–.149 RESERVED.

1200–2–5–.150 APPLICATIONS FOR EXEMPTIONS.

The Division may, upon application by a licensee or registrant or upon its own initiative, grant a specific written exemption from these standards if the Division determines the exemption is authorized by law and would not result in undue hazard to life or property.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq., 68-202-203 and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.151 ADDITIONAL REQUIREMENTS.

The Division may, by rule, regulation, or order, impose requirements on a licensee or registrant, in addition to those established in these regulations, as it deems appropriate or necessary to protect health or to minimize danger to life or property.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–203 and 68–202–206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994.*

1200–2–5–.152 VACATING PREMISES.

Each specific licensee shall, no less than 30 days before vacating or relinquishing possession or control of premises, notify the Division in writing of intent to vacate. If the premises have been contaminated with radioactive material as a result of his activities, the Department may require that the licensee decontaminate or have decontaminated the location to a level for use as an unrestricted area, the details to be specified in each case by the Division.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.153 THROUGH 1200–2–5–.159 RESERVED.

1200–2–5–.160 VIOLATIONS.

A violation of any of these standards subjects the violator to possible civil and criminal penalties.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–5–.161 SCHEDULES.

RHS 8–30

ANNUAL LIMITS ON INTAKE (ALI) AND DERIVED AIR CONCENTRATIONS (DAC) OF RADIONUCLIDES FOR OCCUPATIONAL EXPOSURE; EFFLUENT CONCENTRATIONS; CONCENTRATIONS FOR RELEASE TO SANITARY SEWERAGE

Introduction

For each radionuclide, Table I indicates the chemical form which is to be used for selecting the appropriate ALI or DAC value. The ALIs and DACs for inhalation are given for an aerosol with an activity median aerodynamic diameter (AMAD) of 1 μm, micron, and for three classes (D,W,Y) of radioactive material, which refer to their retention (approximately days, weeks, or years) in the pulmonary region of the lung. This classification applies to a range of clearance half-times for D if less than 10 days, for W from 10 to 100 days, and for Y greater than 100 days. The class (D, W, or Y) given in the column headed "Class" applies only to the inhalation ALIs and DACs given in Table I, columns 2 and 3. Table II provides concentration limits for airborne and liquid effluents released to the general environment. Table III provides concentration limits for discharges to sanitary sewerage systems.

Note:

The values in Tables I, II, and III are presented in the computer "E" notation. In this notation a value of 6E-02 represents a value of 6×10^{-2} or 0.06, 6E+2 represents 6×10^{2} or 600, and 6E+0 represents 6×10^{0} or 6.

Table I "Occupational Values"

Note that the columns in Table I of this schedule captioned, "Oral Ingestion ALI," "Inhalation," "ALI," and "DAC," are applicable to occupational exposure to radioactive material.

The ALIs in this schedule are the annual intakes of a given radionuclide by the reference man, which would result in either a committed effective dose equivalent (CEDE) of 0.05 Sv (5 rem), stochastic ALI, or a committed dose equivalent of 0.5 Sv (50 rem) to an organ or tissue, non-stochastic ALI. The stochastic ALIs were derived to result in a risk, due to irradiation of organs and tissues, comparable to the risk associated with deep dose equivalent to the whole body of 0.05 Sv (5 rem). The derivation includes multiplying the committed dose equivalent to an organ or tissue by a weighting factor, w_T. This weighting factor is the proportion of the risk of stochastic effects resulting from irradiation of the organ or tissue, T, to the total risk of stochastic effects when the whole body is irradiated uniformly. The values of w_T are listed under the definition of weighting factor in 1200–2–5–.32. The non-stochastic ALIs were derived to avoid non-stochastic effects, such as prompt damage to tissue or reduction in organ function.

A value of $w_T = 0.06$ is applicable to each of the five organs or tissues in the "remainder" category receiving the highest dose equivalents, and the dose equivalents of all other remaining tissues may be disregarded. The following portions of the GI tract—stomach, small intestine, upper large intestine, and lower large intestine—are to be treated as four separate organs.

Note that the dose equivalents for an extremity, skin, and lens of the eye are not considered in computing the CEDE but are subject to limits that must be met separately.

When an ALI is defined by the stochastic dose limit, this value alone is given. When an ALI is determined by the non-stochastic dose limit to an organ, the organ or tissue to which the limit applies is shown, and the ALI for the stochastic limit is shown in parentheses. Abbreviated organ or tissue designations are used:

1. LLI wall = lower large intestine wall;
2. St wall = stomach wall;
3. Blad wall = bladder wall; and
4. Bone surf = bone surface.

The use of the ALIs listed first, the more limiting of the stochastic and non-stochastic ALIs, will ensure that non-stochastic effects are avoided and that the risk of stochastic effects is limited to an acceptably low value. If, in a particular situation involving a radionuclide for which the non-stochastic ALI is limiting, the use of that non-stochastic ALI is considered unduly conservative, the licensee or registrant may use the stochastic ALI to determine the committed effective dose equivalent. However, the licensee or registrant shall also ensure that the 0.5 Sv (50 rem) dose equivalent limit for any organ or tissue is not exceeded by the sum of the external deep dose equivalent plus the internal committed dose equivalent to that organ, not the effective dose. For the case where there is no external dose contribution, this would be demonstrated if the sum of the fractions of the nonstochastic ALIs (ALI_{ns}) that contribute to the committed dose equivalent to the organ receiving the highest dose does not exceed unity, that is, Σ (intake [in μCi] of each radionuclide/ALI_{ns}) $\leq$ 1.0. If there is an external deep dose equivalent contribution of H_d, then this sum must be less than 1 - (H_d/50), instead of $\leq$ 1.0.

Note that the dose equivalents for an extremity, skin, and lens of the eye are not considered in computing the committed effective dose equivalent but are subject to limits that must be met separately.

The derived air concentration (DAC) values are derived limits intended to control chronic occupational exposures. The relationship between the DAC and the ALI is given by:

$$DAC = \frac{\text{ALI (in } \mu C_i)}{(2000\ hrs\ /\ working\ \ yr\ \text{X 60 min/hr X 2 x } 10^4\ ml\ /\ \text{min})}$$

$$= \frac{ALI}{2.4\ x\ 10^9}\ \mu\text{Ci/ml}$$

where 2 x 10^4 ml is the volume of air breathed per minute at work by the reference man under working conditions of light work.

The DAC values relate to one of two modes of exposure: either external submersion or the internal committed dose equivalents resulting from inhalation of radioactive materials. DACs based upon submersion are for immersion in a semi-infinite cloud of uniform concentration and apply to each radionuclide separately.

The ALI and DAC values include contributions to exposure by the single radionuclide named and any ingrowth of daughter radionuclides produced in the body by decay of the parent. However, intakes that include both the parent and daughter radionuclides should be treated by the general method appropriate for mixtures.

The values of ALI and DAC do not apply directly when the individual both ingests and inhales a radionuclide, when the individual is exposed to a mixture of radionuclides by either inhalation or ingestion or both, or when the individual is exposed to both internal and external irradiation. See 1200–2–5–.51. When an individual is exposed to radioactive materials, which fall under several of the translocation classifications of the same radionuclide (such as Class D, Class W, or Class Y), the exposure may be evaluated as if it were a mixture of different radionuclides.

It should be noted that the classification of a compound as Class D, W, or Y is based on the chemical form of the compound and does not take into account the radiological half-life of different radioisotopes. For this reason, values are given for Class D, W, and Y compounds, even for very short-lived radionuclides.

Table II "Effluent Concentrations"

The columns in Table II of this schedule captioned "Air" and "Water" are applicable to the assessment and control of dose to the public, particularly in the implementation of the provisions of 1200–2–5–.61. The concentration values given in Columns 1 and 2 of Table II are equivalent to the radionuclide concentrations, which, if inhaled or ingested continuously over the course of a year, would produce a total effective dose equivalent of 0.5 mSv (0.05 rem).

Consideration of non-stochastic limits has not been included in deriving the air and water effluent concentration limits because non-stochastic effects are presumed not to occur at or below the dose levels established for individual members of the public. For radionuclides, where the non-stochastic limit was governing in deriving the occupational DAC, the stochastic ALI was used in deriving the corresponding airborne effluent limit in Table II. For this reason, the DAC and airborne effluent limits are not always proportional, as was the case in the previous Schedule RHS 8–1.

The air concentration values listed in Table II, Column 1 were derived by one of two methods. For those radionuclides for which the stochastic limit is governing, the occupational stochastic inhalation ALI was divided by 2.4×10^9, relating the inhalation ALI to the DAC, as explained above, and then divided by a factor of 300. The factor of 300 includes the following components: a factor of 50 to relate the 0.05 Sv (5 rem) annual occupational dose limit to the 1mSv (0.1 rem) limit for members of the public; a factor of three to adjust for the difference in exposure time and the inhalation rate for a worker and that for members of the public; and a factor of two to adjust the occupational values, derived for adults, so that they are applicable to other age groups.

For those radionuclides for which submersion, that is external dose, is limiting, the occupational DAC in Table I, Column 3 was divided by 219. The factor of 219 is composed of a factor of 50, as described above, and a factor of 4.38 relating occupational exposure for 2,000 hours per year to full-time exposure (8,760 hours per year). Note that an additional factor of two for age considerations is not warranted in the submersion case.

The water concentrations were derived by taking the most restrictive occupational stochastic oral ingestion ALI and dividing by 7.3×10^7. The factor of 7.3×10^7 (ml) includes the following components: the factors of 50 and 2 described above and a factor of 7.3×10^5 (ml), which is the annual water intake of the reference man.

Note 2 of this schedule provides groupings of radionuclides, which are applicable to unknown mixtures of radionuclides. These groupings, including occupational inhalation ALIs and DACs, air and water effluent concentrations and releases to sewer, require demonstrating that the most limiting radionuclides in successive classes are absent. The limit for the unknown mixture is defined when the presence of one of the listed radionuclides cannot be definitely excluded as being present, either from knowledge of the radionuclide composition of the source or from actual measurements.

Table III "Releases to Sewers"

The monthly average concentrations for release to sanitary sewerage are applicable to the provisions in 1200–2–5–.122. The concentration values were derived by taking the most restrictive occupational stochastic oral ingestion ALI and dividing by 7.3×10^6 (ml). The factor of 7.3×10^6 (ml) is composed of a factor of 7.3×10^5 (ml), the annual water intake by a reference man, and a factor of 10, such that the concentrations, if the sewage released by the licensee were the only source of water ingested by a reference man during a year, would result in a committed effective dose equivalent of 5 mSv (0.5 rem).

LIST OF ELEMENTS

Name	Symbol	Atomic Number
Actinium	Ac	89
Aluminum	Al	13
Americium	Am	95
Antimony	Sb	51
Argon	Ar	18
Arsenic	As	33
Astatine	At	85
Barium	Ba	56
Berkelium	Bk	97
Beryllium	Be	4
Bismuth	Bi	83
Bromine	Br	35
Cadmium	Cd	48
Calcium	Ca	20
Californium	Cf	98
Carbon	C	6
Cerium	Ce	58
Cesium	Cs	55
Chlorine	Cl	17
Chromium	Cr	24
Cobalt	Co	27
Copper	Cu	29
Curium	Cm	96
Dysprosium	Dy	66
Einsteinium	Es	99
Erbium	Er	68
Europium	Eu	63
Fermium	Fm	100
Fluorine	F	9
Francium	Fr	87
Gadolinium	Gd	64
Gallium	Ga	31
Germanium	Ge	32
Gold	Au	79
Hafnium	Hf	72
Holmium	Ho	67
Hydrogen	H	1
Indium	In	49
Iodine	I	53
Iridium	Ir	77
Iron	Fe	26
Krypton	Kr	36
Lanthanum	La	57
Lead	Pb	82
Lutetium	Lu	71
Magnesium	Mg	12
Manganese	Mn	25
Mendelevium	Md	101
Mercury	Hg	80
Molybdenum	Mo	42
Neodymium	Nd	60
Neptunium	Np	93
Nickel	Ni	28
Niobium	Nb	41
Osmium	Os	76
Palladium	Pd	46
Phosphorus	P	15
Platinum	Pt	78
Plutonium	Pu	94
Polonium	Po	84
Potassium	K	19
Praseodymium	Pr	59
Promethium	Pm	61
Protactinium	Pa	91
Radium	Ra	88
Radon	Rn	86
Rhenium	Re	75
Rhodium	Rh	45
Rubidium	Rb	37
Ruthenium	Ru	44
Samarium	Sm	62
Scandium	Sc	21
Selenium	Se	34
Silicon	Si	14
Silver	Ag	47
Sodium	Na	11
Strontium	Sr	38
Sulfur	S	16
Tantalum	Ta	73
Technetium	Tc	43
Tellurium	Te	52
Terbium	Tb	65
Thallium	Tl	81
Thorium	Th	90
Thulium	Tm	69
Tin	Sn	50
Titanium	Ti	22
Tungsten	W	74
Uranium	U	92
Vanadium	V	23
Xenon	Xe	54
Ytterbium	Yb	70
Yttrium	Y	39
Zinc	Zn	30
Zirconium	Zr	40

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
1	Hydrogen-3	Water, DAC includes skin absorption	8E+4	8E+4	2E-5	1E-7	1E-3	1E-2
		Gas (HT or T_2) Submersion[1]: Use above values as HT and T2 oxidize in air and in the body to HTO.						
4	Beryllium-7	W, all compounds except those given for Y	4E+4	2E+4	9E-6	3E-8	6E-4	6E-3
		Y, oxides, halides, and nitrates	-	2E+4	8E-6	3E-8	-	-
4	Beryllium-10	W, see ^{7}Be	1E+3	2E+2	6E-8	2E-10	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{7}Be	-	1E+1	6E-9	2E-11	-	-
6	Carbon-11[2]	Monoxide	-	1E+6	5E-4	2E-6	-	-
		Dioxide	-	6E+5	3E-4	9E-7	-	-
		Compounds	4E+5	4E+5	2E-4	6E-7	6E-3	6E-2
6	Carbon-14	Monoxide	-	2E+6	7E-4	2E-6	-	-
		Dioxide	-	2E+5	9E-5	3E-7	-	-
		Compounds	2E+3	2E+3	1E-6	3E-9	3E-5	3E-4
9	Fluorine-18[2]	D, fluorides of H, Li, Na, K, Rb, Cs, and Fr	5E+4	7E+4	3E-5	1E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
		W, fluorides of Be, Mg, Ca, Sr, Ba, Ra, Al, Ga, In, Tl, As, Sb, Bi, Fe, Ru, Os, Co, Ni, Pd, Pt, Cu, Ag, Au, Zn, Cd, Hg, Sc, Y, Ti, Zr, V, Nb, Ta, Mn, Tc, and Re	-	9E+4	4E-5	1E-7	-	-
		Y, lanthanum fluoride	-	8E+4	3E-5	1E-7	-	-
11	Sodium-22	D, all compounds	4E+2	6E+2	3E-7	9E-10	6E-6	6E-5
11	Sodium-24	D, all compounds	4E+3	5E+3	2E-6	7E-9	5E-5	5E-4
12	Magnesium-28	D, all compounds except those given for W	7E+2	2E+3	7E-7	2E-9	9E-6	9E-5
		W, oxides, hydroxides, carbides, halides, and nitrates	-	1E+3	5E-7	2E-9	-	-
13	Aluminum-26	D, all compounds except those given for W	4E+2	6E+1	3E-8	9E-11	6E-6	6E-5
		W, oxides, hydroxides, carbides, halides, and nitrates	-	9E+1	4E-8	1E-10	-	-
14	Silicon-31	D, all compounds except those given for W and Y	9E+3	3E+4	1E-5	4E-8	1E-4	1E-3
		W, oxides, hydroxides, carbides, and nitrates	-	3E+4	1E-5	5E-8	-	-
		Y, aluminosilicate glass	-	3E+4	1E-5	4E-8	-	-
14	Silicon-32	D, see ^{31}Si	2E+3	2E+2	1E-7	3E-10	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{31}Si	-	1E+2	5E-8	2E-10	-	-
		Y, see ^{31}Si	-	5E+0	2E-9	7E-12	-	-
15	Phosphorus-32	D, all compounds except phosphates given for W	6E+2	9E+2	4E-7	1E-9	9E-6	9E-5
		W, phosphates of Zn^{2+}, S^{3+}, Mg^{2+}, Fe^{3+}, Bi^{3+}, and lanthanides	-	4E+2	2E-7	5E-10	-	-
15	Phosphorus-33	D, see ^{32}P	6E+3	8E+3	4E-6	1E-8	8E-5	8E-4
		W, see ^{32}P	-	3E+3	1E-6	4E-9	-	-
16	Sulfur-35	Vapor	-	1E+4	6E-6	2E-8	-	-
		D, sulfides and sulfates except those given for W	1E+4	2E+4	7E-6	2E-8	-	-
			LLI wall (8E+3)	-	-	-	1E-4	1E-3
		W, elemental sulfur, sulfides of Sr, Ba, Ge, Sn, Pb, As, Sb, Bi, Cu, Ag, Au, Zn, Cd, Hg, W, and Mo. Sulfates of Ca, Sr, Ba, Ra, As, Sb, and Bi	6E+3	-	-	-	-	-
			-	2E+3	9E-7	3E-9	-	-
17	Chlorine-36	D, chlorides of H, Li, Na, K, Rb, Cs, and Fr	2E+3	2E+3	1E-6	3E-9	2E-5	2E-4
		W, chlorides of lanthanides, Be, Mg, Ca, Sr, Ba, Ra, Al, Ga, In, Tl, Ge, Sn, Pb, As, Sb, Bi, Fe, Ru, Os, Co, Rh, Ir, Ni, Pd, Pt, Cu, Ag, Au, Zn, Cd, Hg, Sc, Y, Ti, Zr, Hf, V, Nb, Ta, Cr, Mo, W, Mn, Tc, and Re	-	2E+2	1E-7	3E-10	-	-
17	Chlorine-38[2]	D, see ^{36}Cl	2E+4	4E+4	2E-5	6E-8	-	-
			St wall (3E+4)	-	-	-	3E-4	3E-3
		W, see ^{36}Cl	-	5E+4	2E-5	6E-8	-	-
17	Chlorine-39[2]	D, see ^{36}Cl	2E+4	5E+4	2E-5	7E-8	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3
		W, see ^{36}Cl	-	6E+4	2E-5	8E-8	-	-
18	Argon-37	Submersion[1]	-	-	1E+0	6E-3	-	-
18	Argon-39	Submersion[1]	-	-	2E-4	8E-7	-	-
18	Argon-41	Submersion[1]	-	-	3E-6	1E-8	-	-
19	Potassium-40	D, all compounds	3E+2	4E+2	2E-7	6E-10	4E-6	4E-5
19	Potassium-42	D, all compounds	5E+3	5E+3	2E-6	7E-9	6E-5	6E-4
19	Potassium-43	D, all compounds	6E+3	9E+3	4E-6	1E-8	9E-5	9E-4
19	Potassium-44[2]	D, all compounds	2E+4	7E+4	3E-5	9E-8	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3
19	Potassium-45[2]	D, all compounds	3E+4	1E+5	5E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
20	Calcium-41	W, all compounds	3E+3	4E+3	2E-6	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			Bone surf (4E+3)	Bone surf (4E+3)	-	5E-9	6E-5	6E-4
20	Calcium-45	W, all compounds	2E+3	8E+2	4E-7	1E-9	2E-5	2E-4
20	Calcium-47	W, all compounds	8E+2	9E+2	4E-7	1E-9	1E-5	1E-4
21	Scandium-43	Y, all compounds	7E+3	2E+4	9E-6	3E-8	1E-4	1E-3
21	Scandium-44m	Y, all compounds	5E+2	7E+2	3E-7	1E-9	7E-6	7E-5
21	Scandium-44	Y, all compounds	4E+3	1E+4	5E-6	2E-8	5E-5	5E-4
21	Scandium-46	Y, all compounds	9E+2	2E+2	1E-7	3E-10	1E-5	1E-4
21	Scandium-47	Y, all compounds	2E+3	3E+3	1E-6	4E-9	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
21	Scandium-48	Y, all compounds	8E+2	1E+3	6E-7	2E-9	1E-5	1E-4
21	Scandium-49[2]	Y, all compounds	2E+4	5E+4	2E-5	8E-8	3E-4	3E-3
22	Titanium-44	D, all compounds except those given for W and Y	3E+2	1E+1	5E-9	2E-11	4E-6	4E-5
		W, oxides, hydroxides, carbides, halides, and nitrates	-	3E+1	1E-8	4E-11	-	-
		Y, $SrTiO_3$	-	6E+0	2E-9	8E-12	-	-
22	Titanium-45	D, see ^{44}Ti	9E+3	3E+4	1E-5	3E-8	1E-4	1E-3
		W, see ^{44}Ti	-	4E+4	1E-5	5E-8	-	-
		Y, see ^{44}Ti	-	3E+4	1E-5	4E-8	-	-
23	Vanadium-47[2]	D, all compounds except those given for W	3E+4	8E+4	3E-5	1E-7	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		W, oxides, hydroxides, carbides, and halides	-	1E+5	4E-5	1E-7	-	-
23	Vanadium-48	D, see ^{47}V	6E+2	1E+3	5E-7	2E-9	9E-6	9E-5
		W, see ^{47}V	-	6E+2	3E-7	9E-10	-	-
23	Vanadium-49	D, see ^{47}V	7E+4	3E+4	1E-5	-	-	-
			LLI wall (9E+4)	Bone surf (3E+4)	-	5E-8	1E-3	1E-2
		W, see ^{47}V	-	2E+4	8E-6	2E-8	-	-
24	Chromium-48	D, all compounds except those given for W and Y	6E+3	1E+4	5E-6	2E-8	8E-5	8E-4
		W, halides and nitrates	-	7E+3	3E-6	1E-8	-	-
		Y, oxides and hydroxides	-	7E+3	3E-6	1E-8	-	-
24	Chromium-49[2]	D, see ^{48}Cr	3E+4	8E+4	4E-5	1E-7	4E-4	4E-3
		W, see ^{48}Cr	-	1E+5	4E-5	1E-7	-	-
		Y, see ^{48}Cr	-	9E+4	4E-5	1E-7	-	-
24	Chromium-51	D, see ^{48}Cr	4E+4	5E+4	2E-5	6E-8	5E-4	5E-3
		W, see ^{48}Cr	-	2E+4	1E-5	3E-8	-	-
		Y, see ^{48}Cr	-	2E+4	8E-6	3E-8	-	-
25	Manganese-51[2]	D, all compounds except those given for W	2E+4	5E+4	2E-5	7E-8	3E-4	3E-3
		W, oxides, hydroxides, halides, and nitrates	-	6E+4	3E-5	8E-8	-	-
25	Manganese-52m[2]	D, see ^{51}Mn	3E+4	9E+4	4E-5	1E-7	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{51}Mn	-	1E+5	4E-5	1E-7	-	-
25	Manganese-52	D, see ^{51}Mn	7E+2	1E+3	5E-7	2E-9	1E-5	1E-4
		W, see ^{51}Mn	-	9E+2	4E-7	1E-9	-	-
25	Manganese-53	D, see ^{51}Mn	5E+4	1E+4	5E-6	-	7E-4	7E-3
			-	Bone surf (2E+4)	-	3E-8	-	-
		W, see ^{51}Mn	-	1E+4	5E-6	2E-8	-	-
25	Manganese-54	D, see ^{51}Mn	2E+3	9E+2	4E-7	1E-9	3E-5	3E-4
		W, see ^{51}Mn	-	8E+2	3E-7	1E-9	-	-
25	Manganese-56	D, see ^{51}Mn	5E+3	2E+4	6E-6	2E-8	7E-5	7E-4
		W, see ^{51}Mn	-	2E+4	9E-6	3E-8	-	-
26	Iron-52	D, all compounds except those given for W	9E+2	3E+3	1E-6	4E-9	1E-5	1E-4
		W, oxides, hydroxides, and halides	-	2E+3	1E-6	3E-9	-	-
26	Iron-55	D, see ^{52}Fe	9E+3	2E+3	8E-7	3E-9	1E-4	1E-3
		W, see ^{52}Fe	-	4E+3	2E-6	6E-9	-	-
26	Iron-59	D, see ^{52}Fe	8E+2	3E+2	1E-7	5E-10	1E-5	1E-4
		W, see ^{52}Fe	-	5E+2	2E-7	7E-10	-	-
26	Iron-60	D, see ^{52}Fe	3E+1	6E+0	3E-9	9E-12	4E-7	4E-6
		W, see ^{52}Fe	-	2E+1	8E-9	3E-11	-	-
27	Cobalt-55	W, all compounds except those given for Y	1E+3	3E+3	1E-6	4E-9	2E-5	2E-4
		Y, oxides, hydroxides, halides, and nitrates	-	3E+3	1E-6	4E-9	-	-
27	Cobalt-56	W, see ^{55}Co	5E+2	3E+2	1E-7	4E-10	6E-6	6E-5
		Y, see ^{55}Co	4E+2	2E+2	8E-8	3E-10	-	-
27	Cobalt-57	W, see ^{55}Co	8E+3	3E+3	1E-6	4E-9	6E-5	6E-4
		Y, see ^{55}Co	4E+3	7E+2	3E-7	9E-10	-	-
27	Cobalt-58m	W, see ^{55}Co	6E+4	9E+4	4E-5	1E-7	8E-4	8E-3
		Y, see ^{55}Co	-	6E+4	3E-5	9E-8	-	-
27	Cobalt-58	W, see ^{55}Co	2E+3	1E+3	5E-7	2E-9	2E-5	2E-4
		Y, see ^{55}Co	1E+3	7E+2	3E-7	1E-9	-	-
27	Cobalt-60m[2]	W, see ^{55}Co	1E+6	4E+6	2E-3	6E-6	-	-
			St wall (1E+6)	-	-	-	2E-2	2E-1
		Y, see ^{55}Co	-	3E+6	1E-3	4E-6	-	-
27	Cobalt-60	W, see ^{55}Co	5E+2	2E+2	7E-8	2E-10	3E-6	3E-5
		Y, see ^{55}Co	2E+2	3E+1	1E-8	5E-11	-	-
27	Cobalt-61[2]	W, see ^{55}Co	2E+4	6E+4	3E-5	9E-8	3E-4	3E-3
		Y, see ^{55}Co	2E+4	6E+4	2E-5	8E-8	-	-
27	Cobalt-62m[2]	W, see ^{55}Co	4E+4	2E+5	7E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
		Y, see ^{55}Co	-	2E+5	6E-5	2E-7	-	-
28	Nickel-56	D, all compounds except those given for W	1E+3	2E+3	8E-7	3E-9	2E-5	2E-4
		W, oxides, hydroxides, and carbides	-	1E+3	5E-7	2E-9	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		Vapor	-	1E+3	5E-7	2E-9	-	-
28	Nickel-57	D, see ^{56}Ni	2E+3	5E+3	2E-6	7E-9	2E-5	2E-4
		W, see ^{56}Ni	-	3E+3	1E-6	4E-9	-	-
		Vapor	-	6E+3	3E-6	9E-9	-	-
28	Nickel-59	D, see ^{56}Ni	2E+4	4E+3	2E-6	5E-9	3E-4	3E-3
		W, see ^{56}Ni	-	7E+3	3E-6	1E-8	-	-
		Vapor	-	2E+3	8E-7	3E-9	-	-
28	Nickel-63	D, see ^{56}Ni	9E+3	2E+3	7E-7	2E-9	1E-4	1E-3
		W, see ^{56}Ni	-	3E+3	1E-6	4E-9	-	-
		Vapor	-	8E+2	3E-7	1E-9	-	-
28	Nickel-65	D, see ^{56}Ni	8E+3	2E+4	1E-5	3E-8	1E-4	1E-3
		W, see ^{56}Ni	-	3E+4	1E-5	4E-8	-	-
		Vapor	-	2E+4	7E-6	2E-8	-	-
28	Nickel-66	D, see ^{56}Ni	4E+2	2E+3	7E-7	2E-9	-	-
			LLI wall (5E+2)	-	-	-	6E-6	6E-5
		W, see ^{56}Ni	-	6E+2	3E-7	9E-10	-	-
		Vapor	-	3E+3	1E-6	4E-9	-	-
29	Copper-60[2]	D, all compounds except those given for W and Y	3E+4	9E+4	4E-5	1E-7	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		W, sulfides, halides, and nitrates	-	1E+5	5E-5	2E-7	-	-
		Y, oxides and hydroxides	-	1E+5	4E-5	1E-7	-	-
29	Copper-61	D, see ^{60}Cu	1E+4	3E+4	1E-5	4E-8	2E-4	2E-3
		W, see ^{60}Cu	-	4E+4	2E-5	6E-8	-	-
		Y, see ^{60}Cu	-	4E+4	1E-5	5E-8	-	-
29	Copper-64	D, see ^{60}Cu	1E+4	3E+4	1E-5	4E-8	2E-4	2E-3
		W, see ^{60}Cu	-	2E+4	1E-5	3E-8	-	-
		Y, see ^{60}Cu	-	2E+4	9E-6	3E-8	-	-
29	Copper-67	D, see ^{60}Cu	5E+3	8E+3	3E-6	1E-8	6E-5	6E-4
		W, see ^{60}Cu	-	5E+3	2E-6	7E-9	-	-
		Y, see ^{60}Cu	-	5E+3	2E-6	6E-9	-	-
30	Zinc-62	Y, all compounds	1E+3	3E+3	1E-6	4E-9	2E-5	2E-4
30	Zinc-63[2]	Y, all compounds	2E+4	7E+4	3E-5	9E-8	-	-
			St wall (3E+4)	-	-	-	3E-4	3E-3
30	Zinc-65	Y, all compounds	4E+2	3E+2	1E-7	4E-10	5E-6	5E-5
30	Zinc-69m	Y, all compounds	4E+3	7E+3	3E-6	1E-8	6E-5	6E-4
30	Zinc-69[2]	Y, all compounds	6E+4	1E+5	6E-5	2E-7	8E-4	8E-3
30	Zinc-71m	Y, all compounds	6E+3	2E+4	7E-6	2E-8	8E-5	8E-4
30	Zinc-72	Y, all compounds	1E+3	1E+3	5E-7	2E-9	1E-5	1E-4
31	Gallium-65[2]	D, all compounds except those given for W	5E+4	2E+5	7E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	9E-4	9E-3
		W, oxides, hydroxides, carbides, halides, and nitrates	-	2E+5	8E-5	3E-7	-	-
31	Gallium-66	D, see ^{65}Ga	1E+3	4E+3	1E-6	5E-9	1E-5	1E-4
		W, see ^{65}Ga	-	3E+3	1E-6	4E-9	-	-
31	Gallium-67	D, see ^{65}Ga	7E+3	1E+4	6E-6	2E-8	1E-4	1E-3
		W, see ^{65}Ga	-	1E+4	4E-6	1E-8	-	-
31	Gallium-68[2]	D, see ^{65}Ga	2E+4	4E+4	2E-5	6E-8	2E-4	2E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{65}Ga	-	5E+4	2E-5	7E-8	-	-
31	Gallium-70[2]	D, see ^{65}Ga	5E+4	2E+5	7E-5	2E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
		W, see ^{65}Ga	-	2E+5	8E-5	3E-7	-	-
31	Gallium-72	D, see ^{65}Ga	1E+3	4E+3	1E-6	5E-9	2E-5	2E-4
		W, see ^{65}Ga	-	3E+3	1E-6	4E-9	-	-
31	Gallium-73	D, see ^{65}Ga	5E+3	2E+4	6E-6	2E-8	7E-5	7E-4
		W, see ^{65}Ga	-	2E+4	6E-6	2E-8	-	-
32	Germanium-66	D, all compounds except those given for W	2E+4	3E+4	1E-5	4E-8	3E-4	3E-3
		W, oxides, sulfides, and halides	-	2E+4	8E-6	3E-8	-	-
32	Germanium-67[2]	D, see ^{66}Ge	3E+4	9E+4	4E-5	1E-7	-	-
			St wall (4E+4)	-	-	-	6E-4	6E-3
		W, see ^{66}Ge	-	1E+5	4E-5	1E-7	-	-
32	Germanium-68	D, see ^{66}Ge	5E+3	4E+3	2E-6	5E-9	6E-5	6E-4
		W, see ^{66}Ge	-	1E+2	4E-8	1E-10	-	-
32	Germanium-69	D, see ^{66}Ge	1E+4	2E+4	6E-6	2E-8	2E-4	2E-3
		W, see ^{66}Ge	-	8E+3	3E-6	1E-8	-	-
32	Germanium-71	D, see ^{66}Ge	5E+5	4E+5	2E-4	6E-7	7E-3	7E-2
		W, see ^{66}Ge	-	4E+4	2E-5	6E-8	-	-
32	Germanium-75[2]	D, see ^{66}Ge	4E+4	8E+4	3E-5	1E-7	-	-
			St wall (7E+4)	-	-	-	9E-4	9E-3
		W, see ^{66}Ge	-	8E+4	4E-5	1E-7	-	-
32	Germanium-77	D, see ^{66}Ge	9E+3	1E+4	4E-6	1E-8	1E-4	1E-3
		W, see ^{66}Ge	-	6E+3	2E-6	8E-9	-	-
32	Germanium-78[2]	D, see ^{66}Ge	2E+4	2E+4	9E-6	3E-8	-	-
			St wall (2E+4)	-	-	-	3E-4	3E-3
		W, see ^{66}Ge	-	2E+4	9E-6	3E-8	-	-
33	Arsenic-69[2]	W, all compounds	3E+4	1E+5	5E-5	2E-7	-	-
			St wall (4E+4)	-	-	-	6E-4	6E-3
33	Arsenic-70[2]	W, all compounds	1E+4	5E+4	2E-5	7E-8	2E-4	2E-3
33	Arsenic-71	W, all compounds	4E+3	5E+3	2E-6	6E-9	5E-5	5E-4
33	Arsenic-72	W, all compounds	9E+2	1E+3	6E-7	2E-9	1E-5	1E-4
33	Arsenic-73	W, all compounds	8E+3	2E+3	7E-7	2E-9	1E-4	1E-3
33	Arsenic-74	W, all compounds	1E+3	8E+2	3E-7	1E-9	2E-5	2E-4
33	Arsenic-76	W, all compounds	1E+3	1E+3	6E-7	2E-9	1E-5	1E-4
33	Arsenic-77	W, all compounds	4E+3	5E+3	2E-6	7E-9	-	-
			LLI wall (5E+3)	-	-	-	6E-5	6E-4
33	Arsenic-78[2]	W, all compounds	8E+3	2E+4	9E-6	3E-8	1E-4	1E-3
34	Selenium-70[2]	D, all compounds except those given for W	2E+4	4E+4	2E-5	5E-8	1E-4	1E-3
		W, oxides, hydroxides, carbides, and elemental Se	1E+4	4E+4	2E-5	6E-8	-	-
34	Selenium-73m[2]	D, see ^{70}Se	6E+4	2E+5	6E-5	2E-7	4E-4	4E-3
		W, see ^{70}Se	3E+4	1E+5	6E-5	2E-7	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
34	Selenium-73	D, see ^{70}Se	3E+3	1E+4	5E-6	2E-8	4E-5	4E-4
		W, see ^{70}Se	-	2E+4	7E-6	2E-8	-	-
34	Selenium-75	D, see ^{70}Se	5E+2	7E+2	3E-7	1E-9	7E-6	7E-5
		W, see ^{70}Se	-	6E+2	3E-7	8E-10	-	-
34	Selenium-79	D, see ^{70}Se	6E+2	8E+2	3E-7	1E-9	8E-6	8E-5
		W, see ^{70}Se	-	6E+2	2E-7	8E-10	-	-
34	Selenium-81m[2]	D, see ^{70}Se	4E+4	7E+4	3E-5	9E-8	3E-4	3E-3
		W, see ^{70}Se	2E+4	7E+4	3E-5	1E-7	-	-
34	Selenium-81[2]	D, see ^{70}Se	6E+4	2E+5	9E-5	3E-7	-	-
			St wall (8E+4)	-	-	-	1E-3	1E-2
		W, see ^{70}Se	-	2E+5	1E-4	3E-7	-	-
34	Selenium-83[2]	D, see ^{70}Se	4E+4	1E+5	5E-5	2E-7	4E-4	4E-3
		W, see ^{70}Se	3E+4	1E+5	5E-5	2E-7	-	-
35	Bromine-74m[2]	D, bromides of H, Li, Na, K, Rb, Cs, and Fr	1E+4	4E+4	2E-5	5E-8	-	-
			St wall (2E+4)	-	-	-	3E-4	3E-3
		W, bromides of lanthanides, Be, Mg, Ca, Sr, Ba, Ra, Al, Ga, In, Tl, Ge, Sn, Pb, As, Sb, Bi, Fe, Ru, Os, Co, Rh, Ir, Ni, Pd, Pt, Cu, Ag, Au, Zn, Cd, Hg, Sc, Y, Ti, Zr, Hf, V, Nb, Ta, Mn, Tc, and Re	-	4E+4	2E-5	6E-8	-	-
35	Bromine-74[2]	D, see ^{74m}Br	2E+4	7E+4	3E-5	1E-7	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3
		W, see ^{74m}Br	-	8E+4	4E-5	1E-7	-	-
35	Bromine-75[2]	D, see ^{74m}Br	3E+4	5E+4	2E-5	7E-8	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3
		W, see ^{74m}Br	-	5E+4	2E-5	7E-8	-	-
35	Bromine-76	D, see ^{74m}Br	4E+3	5E+3	2E-6	7E-9	5E-5	5E-4
		W, see ^{74m}Br	-	4E+3	2E-6	6E-9	-	-
35	Bromine-77	D, see ^{74m}Br	2E+4	2E+4	1E-5	3E-8	2E-4	2E-3
		W, see ^{74m}Br	-	2E+4	8E-6	3E-8	-	-
35	Bromine-80m	D, see ^{74m}Br	2E+4	2E+4	7E-6	2E-8	3E-4	3E-3
		W, see ^{74m}Br	-	1E+4	6E-6	2E-8	-	-
35	Bromine-80[2]	D, see ^{74m}Br	5E+4	2E+5	8E-5	3E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
		W, see ^{74m}Br	-	2E+5	9E-5	3E-7	-	-
35	Bromine-82	D, see ^{74m}Br	3E+3	4E+3	2E-6	6E-9	4E-5	4E-4
		W, see ^{74m}Br	-	4E+3	2E-6	5E-9	-	-
35	Bromine-83	D, see ^{74m}Br	5E+4	6E+4	3E-5	9E-8	-	-
			St wall (7E+4)	-	-	-	9E-4	9E-3
		W, see ^{74m}Br	-	6E+4	3E-5	9E-8	-	-
35	Bromine-84[2]	D, see ^{74m}Br	2E+4	6E+4	2E-5	8E-8	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		W, see ^{74m}Br	-	6E+4	3E-5	9E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
36	Krypton-74[2]	Submersion[1]	-	-	3E-6	1E-8	-	-
36	Krypton-76	Submersion[1]	-	-	9E-6	4E-8	-	-
36	Krypton-77[2]	Submersion[1]	-	-	4E-6	2E-8	-	-
36	Krypton-79	Submersion[1]	-	-	2E-5	7E-8	-	-
36	Krypton-81	Submersion[1]	-	-	7E-4	3E-6	-	-
36	Krypton-83m[2]	Submersion[1]	-	-	1E-2	5E-5	-	-
36	Krypton-85m	Submersion[1]	-	-	2E-5	1E-7	-	-
36	Krypton-85	Submersion[1]	-	-	1E-4	7E-7	-	-
36	Krypton-87[2]	Submersion[1]	-	-	5E-6	2E-8	-	-
36	Krypton-88	Submersion[1]	-	-	2E-6	9E-9	-	-
37	Rubidium-79[2]	D, all compounds	4E+4	1E+5	5E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	8E-4	8E-3
37	Rubidium-81m[2]	D, all compounds	2E+5	3E+5	1E-4	5E-7	-	-
			St wall (3E+5)	-	-	-	4E-3	4E-2
37	Rubidium-81	D, all compounds	4E+4	5E+4	2E-5	7E-8	5E-4	5E-3
37	Rubidium-82m	D, all compounds	1E+4	2E+4	7E-6	2E-8	2E-4	2E-3
37	Rubidium-83	D, all compounds	6E+2	1E+3	4E-7	1E-9	9E-6	9E-5
37	Rubidium-84	D, all compounds	5E+2	8E+2	3E-7	1E-9	7E-6	7E-5
37	Rubidium-86	D, all compounds	5E+2	8E+2	3E-7	1E-9	7E-6	7E-5
37	Rubidium-87	D, all compounds	1E+3	2E+3	6E-7	2E-9	1E-5	1E-4
37	Rubidium-88[2]	D, all compounds	2E+4	6E+4	3E-5	9E-8	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
37	Rubidium-89[2]	D, all compounds	4E+4	1E+5	6E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	9E-4	9E-3
38	Strontium-80[2]	D, all soluble compounds except $SrTiO_3$	4E+3	1E+4	5E-6	2E-8	6E-5	6E-4
		Y, all insoluble compounds and $SrTiO_3$	-	1E+4	5E-6	2E-8	-	-
38	Strontium-81[2]	D, see ^{80}Sr	3E+4	8E+4	3E-5	1E-7	3E-4	3E-3
		Y, see ^{80}Sr	2E+4	8E+4	3E-5	1E-7	-	-
38	Strontium-82	D, see ^{80}Sr	3E+2	4E+2	2E-7	6E-10	-	-
			LLI wall (2E+2)	-	-	-	3E-6	3E-5
		Y, see ^{80}Sr	2E+2	9E+1	4E-8	1E-10	-	-
38	Strontium-83	D, see ^{80}Sr	3E+3	7E+3	3E-6	1E-8	3E-5	3E-4
		Y, see ^{80}Sr	2E+3	4E+3	1E-6	5E-9	-	-
38	Strontium-85m[2]	D, see ^{80}Sr	2E+5	6E+5	3E-4	9E-7	3E-3	3E-2
		Y, see ^{80}Sr	-	8E+5	4E-4	1E-6	-	-
38	Strontium-85	D, see ^{80}Sr	3E+3	3E+3	1E-6	4E-9	4E-5	4E-4
		Y, see ^{80}Sr	-	2E+3	6E-7	2E-9	-	-
38	Strontium-87m	D, see ^{80}Sr	5E+4	1E+5	5E-5	2E-7	6E-4	6E-3
		Y, see ^{80}Sr	4E+4	2E+5	6E-5	2E-7	-	-
38	Strontium-89	D, see ^{80}Sr	6E+2	8E+2	4E-7	1E-9	-	-
			LLI wall (6E+2)	-	-	-	8E-6	8E-5
		Y, see ^{80}Sr	5E+2	1E+2	6E-8	2E-10	-	-
38	Strontium-90	D, see ^{80}Sr	3E+1	2E+1	8E-9	-	-	-
			Bone surf (4E+1)	Bone surf (2E+1)	-	3E-11	5E-7	5E-6
		Y, see ^{80}Sr	-	4E+0	2E-9	6E-12	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
38	Strontium-91	D, see ^{80}Sr	2E+3	6E+3	2E-6	8E-9	2E-5	2E-4
		Y, see ^{80}Sr	-	4E+3	1E-6	5E-9	-	-
38	Strontium-92	D, see ^{80}Sr	3E+3	9E+3	4E-6	1E-8	4E-5	4E-4
		Y, see ^{80}Sr	-	7E+3	3E-6	9E-9	-	-
39	Yttrium-86m[2]	W, all compounds except those given for Y	2E+4	6E+4	2E-5	8E-8	3E-4	3E-3
		Y, oxides and hydroxides	-	5E+4	2E-5	8E-8	-	-
39	Yttrium-86	W, see ^{86m}Y	1E+3	3E+3	1E-6	5E-9	2E-5	2E-4
		Y, see ^{86m}Y	-	3E+3	1E-6	5E-9	-	-
39	Yttrium-87	W, see ^{86m}Y	2E+3	3E+3	1E-6	5E-9	3E-5	3E-4
		Y, see ^{86m}Y	-	3E+3	1E-6	5E-9	-	-
39	Yttrium-88	W, see ^{86m}Y	1E+3	3E+2	1E-7	3E-10	1E-5	1E-4
		Y, see ^{86m}Y	-	2E+2	1E-7	3E-10	-	-
39	Yttrium-90m	W, see ^{86m}Y	8E+3	1E+4	5E-6	2E-8	1E-4	1E-3
		Y, see ^{86m}Y	-	1E+4	5E-6	2E-8	-	-
39	Yttrium-90	W, see ^{86m}Y	4E+2	7E+2	3E-7	9E-10	-	-
			LLI wall (5E+2)	-	-	-	7E-6	7E-5
		Y, see ^{86m}Y	-	6E+2	3E-7	9E-10	-	-
39	Yttrium-91m[2]	W, see ^{86m}Y	1E+5	2E+5	1E-4	3E-7	2E-3	2E-2
		Y, see ^{86m}Y	-	2E+5	7E-5	2E-7	-	-
39	Yttrium-91	W, see ^{86m}Y	5E+2	2E+2	7E-8	2E-10	-	-
			LLI wall (6E+2)	-	-	-	8E-6	8E-5
		Y, see ^{86m}Y	-	1E+2	5E-8	2E-10	-	-
39	Yttrium-92	W, see ^{86m}Y	3E+3	9E+3	4E-6	1E-8	4E-5	4E-4
		Y, see ^{86m}Y	-	8E+3	3E-6	1E-8	-	-
39	Yttrium-93	W, see ^{86m}Y	1E+3	3E+3	1E-6	4E-9	2E-5	2E-4
		Y, see ^{86m}Y	-	2E+3	1E-6	3E-9	-	-
39	Yttrium-94[2]	W, see ^{86m}Y	2E+4	8E+4	3E-5	1E-7	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		Y, see ^{86m}Y	-	8E+4	3E-5	1E-7	-	-
39	Yttrium-95[2]	W, see ^{86m}Y	4E+4	2E+5	6E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
		Y, see ^{86m}Y	-	1E+5	6E-5	2E-7	-	-
40	Zirconium-86	D, all compounds except those given for W and Y	1E+3	4E+3	2E-6	6E-9	2E-5	2E-4
		W, oxides, hydroxides, halides, and nitrates	-	3E+3	1E-6	4E-9	-	-
		Y, carbide	-	2E+3	1E-6	3E-9	-	-
40	Zirconium-88	D, see ^{86}Zr	4E+3	2E+2	9E-8	3E-10	5E-5	5E-4
		W, see ^{86}Zr	-	5E+2	2E-7	7E-10	-	-
		Y, see ^{86}Zr	-	3E+2	1E-7	4E-10	-	-
40	Zirconium-89	D, see ^{86}Zr	2E+3	4E+3	1E-6	5E-9	2E-5	2E-4
		W, see ^{86}Zr	-	2E+3	1E-6	3E-9	-	-
		Y, see ^{86}Zr	-	2E+3	1E-6	3E-9	-	-
40	Zirconium-93	D, see ^{86}Zr	1E+3	6E+0	3E-9	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			Bone surf (3E+3)	Bone surf (2E+1)	-	2E-11	4E-5	4E-4
		W, see ^{86}Zr	-	2E+1	1E-8	-	-	-
			-	Bone surf (6E+1)	-	9E-11	-	-
		Y, see ^{86}Zr	-	6E+1	2E-8	-	-	-
			-	Bone surf (7E+1)	-	9E-11	-	-
40	Zirconium-95	D, see ^{86}Zr	1E+3	1E+2	5E-8	-	2E-5	2E-4
			-	Bone surf (3E+2)	-	4E-10	-	-
		W, see ^{86}Zr	-	4E+2	2E-7	5E-10	-	-
		Y, see ^{86}Zr	-	3E+2	1E-7	4E-10	-	-
40	Zirconium-97	D, see ^{86}Zr	6E+2	2E+3	8E-7	3E-9	9E-6	9E-5
		W, see ^{86}Zr	-	1E+3	6E-7	2E-9	-	-
		Y, see ^{86}Zr	-	1E+3	5E-7	2E-9	-	-
41	Niobium-88[2]	W, all compounds except those given for Y	5E+4	2E+5	9E-5	3E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
		Y, oxides and hydroxides	-	2E+5	9E-5	3E-7	-	-
41	Niobium-89[2] (66 min)	W, see ^{88}Nb	1E+4	4E+4	2E-5	6E-8	1E-4	1E-3
		Y, see ^{88}Nb	-	4E+4	2E-5	5E-8	-	-
41	Niobium-89 (122 min)	W, see ^{88}Nb	5E+3	2E+4	8E-6	3E-8	7E-5	7E-4
		Y, see ^{88}Nb	-	2E+4	6E-6	2E-8	-	-
41	Niobium-90	W, see ^{88}Nb	1E+3	3E+3	1E-6	4E-9	1E-5	1E-4
		Y, see ^{88}Nb	-	2E+3	1E-6	3E-9	-	-
41	Niobium-93m	W, see ^{88}Nb	9E+3	2E+3	8E-7	3E-9	-	-
			LLI wall (1E+4)	-	-	-	2E-4	2E-3
		Y, see ^{88}Nb	-	2E+2	7E-8	2E-10	-	-
41	Niobium-94	W, see ^{88}Nb	9E+2	2E+2	8E-8	3E-10	1E-5	1E-4
		Y, see ^{88}Nb	-	2E+1	6E-9	2E-11	-	-
41	Niobium-95m	W, see ^{88}Nb	2E+3	3E+3	1E-6	4E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
		Y, see ^{88}Nb	-	2E+3	9E-7	3E-9	-	-
41	Niobium-95	W, see ^{88}Nb	2E+3	1E+3	5E-7	2E-9	3E-5	3E-4
		Y, see ^{88}Nb	-	1E+3	5E-7	2E-9	-	-
41	Niobium-96	W, see ^{88}Nb	1E+3	3E+3	1E-6	4E-9	2E-5	2E-4
		Y, see ^{88}Nb	-	2E+3	1E-6	3E-9	-	-
41	Niobium-97[2]	W, see ^{88}Nb	2E+4	8E+4	3E-5	1E-7	3E-4	3E-3
		Y, see ^{88}Nb	-	7E+4	3E-5	1E-7	-	-
41	Niobium-98[2]	W, see ^{88}Nb	1E+4	5E+4	2E-5	8E-8	2E-4	2E-3
		Y, see ^{88}Nb	-	5E+4	2E-5	7E-8	-	-
42	Molybdenum-90	D, all compounds except those given for Y	4E+3	7E+3	3E-6	1E-8	3E-5	3E-4
		Y, oxides, hydroxide, and MoS_2	2E+3	5E+3	2E-6	6E-9	-	-
42	Molybdenum-93m	D, see ^{90}Mo	9E+3	2E+4	7E-6	2E-8	6E-5	6E-4
		Y, see ^{90}Mo	4E+3	1E+4	6E-6	2E-8	-	-
42	Molybdenum-93	D, see ^{90}Mo	4E+3	5E+3	2E-6	8E-9	5E-5	5E-4
		Y, see ^{90}Mo	2E+4	2E+2	8E-8	2E-10	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
42	Molybdenum-99	D, see ^{90}Mo	2E+3	3E+3	1E-6	4E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{90}Mo	1E+3	1E+3	6E-7	2E-9	-	-
42	Molybdenum-101[2]	D, see ^{90}Mo	4E+4	1E+5	6E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
		Y, see ^{90}Mo	-	1E+5	6E-5	2E-7	-	-
43	Technetium-93m[2]	D; all compounds except those given for W	7E+4	2E+5	6E-5	2E-7	1E-3	1E-2
		W, oxides, hydroxides, halides, and nitrates	-	3E+5	1E-4	4E-7	-	-
43	Technetium-93	D, see ^{93m}Tc	3E+4	7E+4	3E-5	1E-7	4E-4	4E-3
		W, see ^{93m}Tc	-	1E+5	4E-5	1E-7	-	-
43	Technetium-94m[2]	D, see ^{93m}Tc	2E+4	4E+4	2E-5	6E-8	3E-4	3E-3
		W, see ^{93m}Tc	-	6E+4	2E-5	8E-8	-	-
43	Technetium-94	D, see ^{93m}Tc	9E+3	2E+4	8E-6	3E-8	1E-4	1E-3
		W, see ^{93m}Tc	-	2E+4	1E-5	3E-8	-	-
43	Technetium-95m	D, see ^{93m}Tc	4E+3	5E+3	2E-6	8E-9	5E-5	5E-4
		W, see ^{93m}Tc	-	2E+3	8E-7	3E-9	-	-
43	Technetium-95	D, see ^{93m}Tc	1E+4	2E+4	9E-6	3E-8	1E-4	1E-3
		W, see ^{93m}Tc	-	2E+4	8E-6	3E-8	-	-
43	Technetium-96m[2]	D, see ^{93m}Tc	2E+5	3E+5	1E-4	4E-7	2E-3	2E-2
		W, see ^{93m}Tc	-	2E+5	1E-4	3E-7	-	-
43	Technetium-96	D, see ^{93m}Tc	2E+3	3E+3	1E-6	5E-9	3E-5	3E-4
		W, see ^{93m}Tc	-	2E+3	9E-7	3E-9	-	-
43	Technetium-97m	D, see ^{93m}Tc	5E+3	7E+3	3E-6	-	6E-5	6E-4
			-	St Wall (7E + 3)	-	1E-8	-	-
		W, see ^{93m}Tc	-	1E+3	5E-7	2E-9	-	-
43	Technetium-97	D, see ^{93m}Tc	4E+4	5E+4	2E-5	7E-8	5E-4	5E-3
		W, see ^{93m}Tc	-	6E+3	2E-6	8E-9	-	-
43	Technetium-98	D, see ^{93m}Tc	1E+3	2E+3	7E-7	2E-9	1E-5	1E-4
		W, see ^{93m}Tc	-	3E+2	1E-7	4E-10	-	-
43	Technetium-99m	D, see ^{93m}Tc	8E+4	2E+5	6E-5	2E-7	1E-3	1E-2
		W, see ^{93m}Tc	-	2E+5	1E-4	3E-7	-	-
43	Technetium-99	D, see ^{93m}Tc	4E+3	5E+3	2E-6	-	6E-5	6E-4
			-	St wall (6E+3)	-	8E-9	-	-
		W, see ^{93m}Tc	-	7E+2	3E-7	9E-10	-	-
43	Technetium-101[2]	D, see ^{93m}Tc	9E+4	3E+5	1E-4	5E-7	-	-
			St wall (1E+5)	-	-	-	2E-3	2E-2
		W, see ^{93m}Tc	-	4E+5	2E-4	5E-7	-	-
43	Technetium-104[2]	D, see ^{93m}Tc	2E+4	7E+4	3E-5	1E-7	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		W, see ^{93m}Tc	-	9E+4	4E-5	1E-7	-	-
44	Ruthenium-94[2]	D, all compounds except those given for W and Y	2E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, halides	-	6E+4	3E-5	9E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation		Air (μCi/ml)	Water (μCi/ml)	
				ALI (μCi)	DAC (μCi/ml)			
		Y, oxides and hydroxides	-	6E+4	2E-5	8E-8	-	-
44	Ruthenium-97	D, see ^{94}Ru	8E+3	2E+4	8E-6	3E-8	1E-4	1E-3
		W, see ^{94}Ru	-	1E+4	5E-6	2E-8	-	-
		Y, see ^{94}Ru	-	1E+4	5E-6	2E-8	-	-
44	Ruthenium-103	D, see ^{94}Ru	2E+3	2E+3	7E-7	2E-9	3E-5	3E-4
		W, see ^{94}Ru	-	1E+3	4E-7	1E-9	-	-
		Y, see ^{94}Ru	-	6E+2	3E-7	9E-10	-	-
44	Ruthenium-105	D, see ^{94}Ru	5E+3	1E+4	6E-6	2E-8	7E-5	7E-4
		W, see ^{94}Ru	-	1E+4	6E-6	2E-8	-	-
		Y, see ^{94}Ru	-	1E+4	5E-6	2E-8	-	-
44	Ruthenium-106	D, see ^{94}Ru	2E+2	9E+1	4E-8	1E-10	-	-
			LLI wall (2E+2)	-	-	-	3E-6	3E-5
		W, see ^{94}Ru	-	5E+1	2E-8	8E-11	-	-
		Y, see ^{94}Ru	-	1E+1	5E-9	2E-11	-	-
45	Rhodium-99m	D, all compounds except those given for W and Y	2E+4	6E+4	2E-5	8E-8	2E-4	2E-3
		W, halides	-	8E+4	3E-5	1E-7	-	-
		Y, oxides and hydroxides	-	7E+4	3E-5	9E-8	-	-
45	Rhodium-99	D, see ^{99m}Rh	2E+3	3E+3	1E-6	4E-9	3E-5	3E-4
		W, see ^{99m}Rh	-	2E+3	9E-7	3E-9	-	-
		Y, see ^{99m}Rh	-	2E+3	8E-7	3E-9	-	-
45	Rhodium-100	D, see ^{99m}Rh	2E+3	5E+3	2E-6	7E-9	2E-5	2E-4
		W, see ^{99m}Rh	-	4E+3	2E-6	6E-9	-	-
		Y, see ^{99m}Rh	-	4E+3	2E-6	5E-9	-	-
45	Rhodium-101m	D, see ^{99m}Rh	6E+3	1E+4	5E-6	2E-8	8E-5	8E-4
		W, see ^{99m}Rh	-	8E+3	4E-6	1E-8	-	-
		Y, see ^{99m}Rh	-	8E+3	3E-6	1E-8	-	-
45	Rhodium-101	D, see ^{99m}Rh	2E+3	5E+2	2E-7	7E-10	3E-5	3E-4
		W, see ^{99m}Rh	-	8E+2	3E-7	1E-9	-	-
		Y, see ^{99m}Rh	-	2E+2	6E-8	2E-10	-	-
45	Rhodium-102m	D, see ^{99m}Rh	1E+3	5E+2	2E-7	7E-10	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		W, see ^{99m}Rh	-	4E+2	2E-7	5E-10	-	-
		Y, see ^{99m}Rh	-	1E+2	5E-8	2E-10	-	-
45	Rhodium-102	D, see ^{99m}Rh	6E+2	9E+1	4E-8	1E-10	8E-6	8E-5
		W, see ^{99m}Rh	-	2E+2	7E-8	2E-10	-	-
		Y, see ^{99m}Rh	-	6E+1	2E-8	8E-11	-	-
45	Rhodium-103m[2]	D, see ^{99m}Rh	4E+5	1E+6	5E-4	2E-6	6E-3	6E-2
		W, see ^{99m}Rh	-	1E+6	5E-4	2E-6	-	-
		Y, see ^{99m}Rh	-	1E+6	5E-4	2E-6	-	-
45	Rhodium-105	D, see ^{99m}Rh	4E+3	1E+4	5E-6	2E-8	-	-
			LLI wall (4E+3)	-	-	-	5E-5	5E-4
		W, see ^{99m}Rh	-	6E+3	3E-6	9E-9	-	-
		Y, see ^{99m}Rh	-	6E+3	2E-6	8E-9	-	-
45	Rhodium-106m	D, see ^{99m}Rh	8E+3	3E+4	1E-5	4E-8	1E-4	1E-3
		W, see ^{99m}Rh	-	4E+4	2E-5	5E-8	-	-
		Y, see ^{99m}Rh	-	4E+4	1E-5	5E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
45	Rhodium-107[2]	D, see ^{99m}Rh	7E+4	2E+5	1E-4	3E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
		W, see ^{99m}Rh	-	3E+5	1E-4	4E-7	-	-
		Y, see ^{99m}Rh	-	3E+5	1E-4	3E-7	-	-
46	Palladium-100	D, all compounds except those given for W and Y	1E+3	1E+3	6E-7	2E-9	2E-5	2E-4
		W, nitrates	-	1E+3	5E-7	2E-9	-	-
		Y, oxides and hydroxides	-	1E+3	6E-7	2E-9	-	-
46	Palladium-101	D, see ^{100}Pd	1E+4	3E+4	1E-5	5E-8	2E-4	2E-3
		W, see ^{100}Pd	-	3E+4	1E-5	5E-8	-	-
		Y, see ^{100}Pd	-	3E+4	1E-5	4E-8	-	-
46	Palladium-103	D, see ^{100}Pd	6E+3	6E+3	3E-6	9E-9	-	-
			LLI wall (7E+3)	-	-	-	1E-4	1E-3
		W, see ^{100}Pd	-	4E+3	2E-6	6E-9	-	-
		Y, see ^{100}Pd	-	4E+3	1E-6	5E-9	-	-
46	Palladium-107	D, see ^{100}Pd	3E+4	2E+4	9E-6	-	-	-
			LLI wall (4E+4)	Kidneys (2E+4)	-	3E-8	5E-4	5E-3
		W, see ^{100}Pd	-	7E+3	3E-6	1E-8	-	-
		Y, see ^{100}Pd	-	4E+2	2E-7	6E-10	-	-
46	Palladium-109	D, see ^{100}Pd	2E+3	6E+3	3E-6	9E-9	3E-5	3E-4
		W, see ^{100}Pd	-	5E+3	2E-6	8E-9	-	-
		Y, see ^{100}Pd	-	5E+3	2E-6	6E-9	-	-
47	Silver-102[2]	D, all compounds except those given for W and Y	5E+4	2E+5	8E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	9E-4	9E-3
		W, nitrates and sulfides	-	2E+5	9E-5	3E-7	-	-
		Y, oxides and hydroxides	-	2E+5	8E-5	3E-7	-	-
47	Silver-103[2]	D, see ^{102}Ag	4E+4	1E+5	4E-5	1E-7	5E-4	5E-3
		W, see ^{102}Ag	-	1E+5	5E-5	2E-7	-	-
		Y, see ^{102}Ag	-	1E+5	5E-5	2E-7	-	-
47	Silver-104m[2]	D, see ^{102}Ag	3E+4	9E+4	4E-5	1E-7	4E-4	4E-3
		W, see ^{102}Ag	-	1E+5	5E-5	2E-7	-	-
		Y, see ^{102}Ag	-	1E+5	5E-5	2E-7	-	-
47	Silver-104[2]	D, see ^{102}Ag	2E+4	7E+4	3E-5	1E-7	3E-4	3E-3
		W, see ^{102}Ag	-	1E+5	6E-5	2E-7	-	-
		Y, see ^{102}Ag	-	1E+5	6E-5	2E-7	-	-
47	Silver-105	D, see ^{102}Ag	3E+3	1E+3	4E-7	1E-9	4E-5	4E-4
		W, see ^{102}Ag	-	2E+3	7E-7	2E-9	-	-
		Y, see ^{102}Ag	-	2E+3	7E-7	2E-9	-	-
47	Silver-106m	D, see ^{102}Ag	8E+2	7E+2	3E-7	1E-9	1E-5	1E-4
		W, see ^{102}Ag	-	9E+2	4E-7	1E-9	-	-
		Y, see ^{102}Ag	-	9E+2	4E-7	1E-9	-	-
47	Silver-106[2]	D, see ^{102}Ag	6E+4	2E+5	8E-5	3E-7	-	-
			St. wall (6E+4)	-	-	-	9E-4	9E-3
		W, see ^{102}Ag	-	2E+5	9E-5	3E-7	-	-
		Y, see ^{102}Ag	-	2E+5	8E-5	3E-7	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
47	Silver-108m	D, see ^{102}Ag	6E+2	2E+2	8E-8	3E-10	9E-6	9E-5
		W, see ^{102}Ag	-	3E+2	1E-7	4E-10	-	-
		Y, see ^{102}Ag	-	2E+1	1E-8	3E-11	-	-
47	Silver-110m	D, see ^{102}Ag	5E+2	1E+2	5E-8	2E-10	6E-6	6E-5
		W, see ^{102}Ag	-	2E+2	8E-8	3E-10	-	-
		Y, see ^{102}Ag	-	9E+1	4E-8	1E-10	-	-
47	Silver-111	D, see ^{102}Ag	9E+2	2E+3	6E-7	-	-	-
			LLI wall (1E+3)	Liver (2E+3)	-	2E-9	2E-5	2E-4
		W, see ^{102}Ag	-	9E+2	4E-7	1E-9	-	-
		Y, see ^{102}Ag	-	9E+2	4E-7	1E-9	-	-
47	Silver-112	D, see ^{102}Ag	3E+3	8E+3	3E-6	1E-8	4E-5	4E-4
		W, see ^{102}Ag	-	1E+4	4E-6	1E-8	-	-
		Y, see ^{102}Ag	-	9E+3	4E-6	1E-8	-	-
47	Silver-115[2]	D, see ^{102}Ag	3E+4	9E+4	4E-5	1E-7	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		W, see ^{102}Ag	-	9E+4	4E-5	1E-7	-	-
		Y, see ^{102}Ag	-	8E+4	3E-5	1E-7	-	-
48	Cadmium-104[2]	D, all compounds except those given for W and Y	2E+4	7E+4	3E-5	9E-8	3E-4	3E-3
		W, sulfides, halides, and nitrates	-	1E+5	5E-5	2E-7	-	-
		Y, oxides and hydroxides	-	1E+5	5E-5	2E-7	-	-
48	Cadmium-107	D, see ^{104}Cd	2E+4	5E+4	2E-5	8E-8	3E-4	3E-3
		W, see ^{104}Cd	-	6E+4	2E-5	8E-8	-	-
		Y, see ^{104}Cd	-	5E+4	2E-5	7E-8	-	-
48	Cadmium-109	D, see ^{104}Cd	3E+2	4E+1	1E-8	-	-	-
			Kidneys (4E+2)	Kidneys (5E+1)	-	7E-11	6E-6	6E-5
		W, see ^{104}Cd	-	1E+2	5E-8	-	-	-
			-	Kidneys (1E+2)	-	2E-10	-	-
		Y, see ^{104}Cd	-	1E+2	5E-8	2E-10	-	-
48	Cadmium-113m	D, see ^{104}Cd	2E+1	2E+0	1E-9	-	-	-
			Kidneys (4E+1)	Kidneys (4E+0)	-	5E-12	5E-7	5E-6
		W, see ^{104}Cd	-	8E+0	4E-9	-	-	-
			-	Kidneys (1E+1)	-	2E-11	-	-
		Y, see ^{104}Cd	-	1E+1	5E-9	2E-11	-	-
48	Cadmium-113	D, see ^{104}Cd	2E+1	2E+0	9E-10	-	-	-
			Kidneys (3E+1)	Kidneys (3E+0)	-	5E-12	4E-7	4E-6
		W, see ^{104}Cd	-	8E+0	3E-9	-	-	-
			-	Kidneys (1E+1)	-	2E-11	-	-
		Y, see ^{104}Cd	-	1E+1	6E-9	2E-11	-	-
48	Cadmium-115m	D, see ^{104}Cd	3E+2	5E+1	2E-8	-	4E-6	4E-5
			-	Kidneys (8E+1)	-	1E-10	-	-
		W, see ^{104}Cd	-	1E+2	5E-8	2E-10	-	-
		Y, see ^{104}Cd	-	1E+2	6E-8	2E-10	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
48	Cadmium-115	D, see ^{104}Cd	9E+2	1E+3	6E-7	2E-9	-	-
			LLI wall (1E+3)	-	-	-	1E-5	1E-4
		W, see ^{104}Cd	-	1E+3	5E-7	2E-9	-	-
		Y, see ^{104}Cd	-	1E+3	6E-7	2E-9	-	-
48	Cadmium-117m	D, see ^{104}Cd	5E+3	1E+4	5E-6	2E-8	6E-5	6E-4
		W, see ^{104}Cd	-	2E+4	7E-6	2E-8	-	-
		Y, see ^{104}Cd	-	1E+4	6E-6	2E-8	-	-
48	Cadmium-117	D, see ^{104}Cd	5E+3	1E+4	5E-6	2E-8	6E-5	6E-4
		W, see ^{104}Cd	-	2E+4	7E-6	2E-8	-	-
		Y, see ^{104}Cd	-	1E+4	6E-6	2E-8	-	-
49	Indium-109	D, all compounds except those given for W	2E+4	4E+4	2E-5	6E-8	3E-4	3E-3
		W, oxides, hydroxides, halides, and nitrates	-	6E+4	3E-5	9E-8	-	-
49	Indium-110[2] (69.1 min)	D, see ^{109}In	2E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{109}In	-	6E+4	2E-5	8E-8	-	-
49	Indium-110 (4.9 h)	D, see ^{109}In	5E+3	2E+4	7E-6	2E-8	7E-5	7E-4
		W, see ^{109}In	-	2E+4	8E-6	3E-8	-	-
49	Indium-111	D, see ^{109}In	4E+3	6E+3	3E-6	9E-9	6E-5	6E-4
		W, see ^{109}In	-	6E+3	3E-6	9E-9	-	-
49	Indium-112[2]	D, see ^{109}In	2E+5	6E+5	3E-4	9E-7	2E-3	2E-2
		W, see ^{109}In	-	7E+5	3E-4	1E-6	-	-
49	Indium-113m[2]	D, see ^{109}In	5E+4	1E+5	6E-5	2E-7	7E-4	7E-3
		W, see ^{109}In	-	2E+5	8E-5	3E-7	-	-
49	Indium-114m	D, see ^{109}In	3E+2	6E+1	3E-8	9E-11	-	-
			LLI wall (4E+2)	-	-	-	5E-6	5E-5
		W, see ^{109}In	-	1E+2	4E-8	1E-10	-	-
49	Indium-115m	D, see ^{109}In	1E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{109}In	-	5E+4	2E-5	7E-8	-	-
49	Indium-115	D, see ^{109}In	4E+1	1E+0	6E-10	2E-12	5E-7	5E-6
		W, see ^{109}In	-	5E+0	2E-9	8E-12	-	-
49	Indium-116m[2]	D, see ^{109}In	2E+4	8E+4	3E-5	1E-7	3E-4	3E-3
		W, see ^{109}In	-	1E+5	5E-5	2E-7	-	-
49	Indium-117m[2]	D, see ^{109}In	1E+4	3E+4	1E-5	5E-8	2E-4	2E-3
		W, see ^{109}In	-	4E+4	2E-5	6E-8	-	-
49	Indium-117[2]	D, see ^{109}In	6E+4	2E+5	7E-5	2E-7	8E-4	8E-3
		W, see ^{109}In	-	2E+5	9E-5	3E-7	-	-
49	Indium-119m[2]	D, see ^{109}In	4E+4	1E+5	5E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	7E-4	7E-3
		W, see ^{109}In	-	1E+5	6E-5	2E-7	-	-
50	Tin-110	D, all compounds except those given for W	4E+3	1E+4	5E-6	2E-8	5E-5	5E-4
		W, sulfides, oxides, hydroxides, halides, nitrates, and stannic phosphate	-	1E+4	5E-6	2E-8	-	-
50	Tin-111[2]	D, see ^{110}Sn	7E+4	2E+5	9E-5	3E-7	1E-3	1E-2
		W, see ^{110}Sn	-	3E+5	1E-4	4E-7	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
50	Tin-113	D, see ^{110}Sn	2E+3	1E+3	5E-7	2E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
		W, see ^{110}Sn	-	5E+2	2E-7	8E-10	-	-
50	Tin-117m	D, see ^{110}Sn	2E+3	1E+3	5E-7	-	-	-
			LLI wall (2E+3)	Bone surf (2E+3)	-	3E-9	3E-5	3E-4
		W, see ^{110}Sn	-	1E+3	6E-7	2E-9	-	-
50	Tin-119m	D, see ^{110}Sn	3E+3	2E+3	1E-6	3E-9	-	-
			LLI wall (4E+3)	-	-	-	6E-5	6E-4
		W, see ^{110}Sn	-	1E+3	4E-7	1E-9	-	-
50	Tin-121m	D, see ^{110}Sn	3E+3	9E+2	4E-7	1E-9	-	-
			LLI wall (4E+3)	-	-	-	5E-5	5E-4
		W, see ^{110}Sn	-	5E+2	2E-7	8E-10	-	-
50	Tin-121	D, see ^{110}Sn	6E+3	2E+4	6E-6	2E-8	-	-
			LLI wall (6E+3)	-	-	-	8E-5	8E-4
		W, see ^{110}Sn	-	1E+4	5E-6	2E-8	-	-
50	Tin-123m[2]	D, see ^{110}Sn	5E+4	1E+5	5E-5	2E-7	7E-4	7E-3
		W, see ^{110}Sn	-	1E+5	6E-5	2E-7	-	-
50	Tin-123	D, see ^{110}Sn	5E+2	6E+2	3E-7	9E-10	-	-
			LLI wall (6E+2)	-	-	-	9E-6	9E-5
		W, see ^{110}Sn	-	2E+2	7E-8	2E-10	-	-
50	Tin-125	D, see ^{110}Sn	4E+2	9E+2	4E-7	1E-9	-	-
			LLI wall (5E+2)	-	-	-	6E-6	6E-5
		W, see ^{110}Sn	-	4E+2	1E-7	5E-10	-	-
50	Tin-126	D, see ^{110}Sn	3E+2	6E+1	2E-8	8E-11	4E-6	4E-5
		W, see ^{110}Sn	-	7E+1	3E-8	9E-11	-	-
50	Tin-127	D, see ^{110}Sn	7E+3	2E+4	8E-6	3E-8	9E-5	9E-4
		W, see ^{110}Sn	-	2E+4	8E-6	3E-8	-	-
50	Tin-128[2]	D, see ^{110}Sn	9E+3	3E+4	1E-5	4E-8	1E-4	1E-3
		W, see ^{110}Sn	-	4E+4	1E-5	5E-8	-	-
51	Antimony-115[2]	D, all compounds except those given for W	8E+4	2E+5	1E-4	3E-7	1E-3	1E-2
		W, oxides, hydroxides, halides, sulfides, sulfates, and nitrates	-	3E+5	1E-4	4E-7	-	-
51	Antimony-116m[2]	D, see ^{115}Sb	2E+4	7E+4	3E-5	1E-7	3E-4	3E-3
		W, see ^{115}Sb	-	1E+5	6E-5	2E-7	-	-
51	Antimony-116[2]	D, see ^{115}Sb	7E+4	3E+5	1E-4	4E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
		W, see ^{115}Sb	-	3E+5	1E-4	5E-7	-	-
51	Antimony-117	D, see ^{115}Sb	7E+4	2E+5	9E-5	3E-7	9E-4	9E-3
		W, see ^{115}Sb	-	3E+5	1E-4	4E-7	-	-
51	Antimony-118m	D, see ^{115}Sb	6E+3	2E+4	8E-6	3E-8	7E-5	7E-4
		W, see ^{115}Sb	5E+3	2E+4	9E-6	3E-8	-	-
51	Antimony-119	D, see ^{115}Sb	2E+4	5E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{115}Sb	2E+4	3E+4	1E-5	4E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
51	Antimony-120[2] (16 min)	D, see ^{115}Sb	1E+5	4E+5	2E-4	6E-7	-	-
			St wall (2E+5)	-	-	-	2E-3	2E-2
		W, see ^{115}Sb	-	5E+5	2E-4	7E-7	-	-
51	Antimony-120 (5.76 d)	D, see ^{115}Sb	1E+3	2E+3	9E-7	3E-9	1E-5	1E-4
		W, see ^{115}Sb	9E+2	1E+3	5E-7	2E-9	-	-
51	Antimony-122	D, see ^{115}Sb	8E+2	2E+3	1E-6	3E-9	-	-
			LLI wall (8E+2)	-	-	-	1E-5	1E-4
		W, see ^{115}Sb	7E+2	1E+3	4E-7	2E-9	-	-
51	Antimony-124m[2]	D, see ^{115}Sb	3E+5	8E+5	4E-4	1E-6	3E-3	3E-2
		W, see ^{115}Sb	2E+5	6E+5	2E-4	8E-7	-	-
51	Antimony-124	D, see ^{115}Sb	6E+2	9E+2	4E-7	1E-9	7E-6	7E-5
		W, see ^{115}Sb	5E+2	2E+2	1E-7	3E-10	-	-
51	Antimony-125	D, see ^{115}Sb	2E+3	2E+3	1E-6	3E-9	3E-5	3E-4
		W, see ^{115}Sb	-	5E+2	2E-7	7E-10	-	-
51	Antimony-126m[2]	D, see ^{115}Sb	5E+4	2E+5	8E-5	3E-7	-	-
			St wall (7E+4)	-	-	-	9E-4	9E-3
		W, see ^{115}Sb	-	2E+5	8E-5	3E-7	-	-
51	Antimony-126	D, see ^{115}Sb	6E+2	1E+3	5E-7	2E-9	7E-6	7E-5
		W, see ^{115}Sb	5E+2	5E+2	2E-7	7E-10	-	-
51	Antimony-127	D, see ^{115}Sb	8E+2	2E+3	9E-7	3E-9	-	-
			LLI wall (8E+2)	-	-	-	1E-5	1E-4
		W, see ^{115}Sb	7E+2	9E+2	4E-7	1E-9	-	-
51	Antimony-128[2] (10.4 min)	D, see ^{115}Sb	8E+4	4E+5	2E-4	5E-7	-	-
			St wall (1E+5)	-	-	-	1E-3	1E-2
		W, see ^{115}Sb	-	4E+5	2E-4	6E-7	-	-
51	Antimony-128 (9.01 h)	D, see ^{115}Sb	1E+3	4E+3	2E-6	6E-9	2E-5	2E-4
		W, see ^{115}Sb	-	3E+3	1E-6	5E-9	-	-
51	Antimony-129	D, see ^{115}Sb	3E+3	9E+3	4E-6	1E-8	4E-5	4E-4
		W, see ^{115}Sb	-	9E+3	4E-6	1E-8	-	-
51	Antimony-130[2]	D, see ^{115}Sb	2E+4	6E+4	3E-5	9E-8	3E-4	3E-3
		W, see ^{115}Sb	-	8E+4	3E-5	1E-7	-	-
51	Antimony-131[2]	D, see ^{115}Sb	1E+4	2E+4	1E-5	-	-	-
			Thyroid (2E+4)	Thyroid (4E+4)	-	6E-8	2E-4	2E-3
		W, see ^{115}Sb	-	2E+4	1E-5		-	-
			-	Thyroid (4E+4)	-	6E-8	-	-
52	Tellurium-116	D, all compounds except those given for W	8E+3	2E+4	9E-6	3E-8	1E-4	1E-3
		W, oxides, hydroxides, and nitrates	-	3E+4	1E-5	4E-8	-	-
52	Tellurium-121m	D, see ^{116}Te	5E+2	2E+2	8E-8	-	-	-
			Bone surf (7E+2)	Bone surf 4E+2)	-	5E-10	1E-5	1E-4
		W, see ^{116}Te	-	4E+2	2E-7	6E-10	-	-
52	Tellurium-121	D, see ^{116}Te	3E+3	4E+3	2E-6	6E-9	4E-5	4E-4
		W, see ^{116}Te	-	3E+3	1E-6	4E-9	-	-
52	Tellurium-123m	D, see ^{116}Te	6E+2	2E+2	9E-8	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			Bone surf (1E+3)	Bone surf (5E+2)	-	8E-10	1E-5	1E-4
		W, see ^{116}Te	-	5E+2	2E-7	8E-10	-	-
52	Tellurium-123	D, see ^{116}Te	5E+2	2E+2	8E-8	-	-	-
			Bone surf (1E+3)	Bone surf (5E+2)	-	7E-10	2E-5	2E-4
		W, see ^{116}Te	-	4E+2	2E-7	-	-	-
			-	Bone surf (1E+3)	-	2E-9	-	-
52	Tellurium-125m	D, see ^{116}Te	1E+3	4E+2	2E-7	-	-	-
			Bone surf (1E+3)	Bone surf 1E+3)	-	1E-9	2E-5	2E-4
		W, see ^{116}Te	-	7E+2	3E-7	1E-9	-	-
52	Tellurium-127m	D, see ^{116}Te	6E+2	3E+2	1E-7	-	9E-6	9E-5
			-	Bone surf (4E+2)	-	6E-10	-	-
		W, see ^{116}Te	-	3E+2	1E-7	4E-10	-	-
52	Tellurium-127	D, see ^{116}Te	7E+3	2E+4	9E-6	3E-8	1E-4	1E-3
		W, see ^{116}Te	-	2E+4	7E-6	2E-8	-	-
52	Tellurium-129m	D, see ^{116}Te	5E+2	6E+2	3E-7	9E-10	7E-6	7E-5
		W, see ^{116}Te	-	2E+2	1E-7	3E-10	-	-
52	Tellurium-129[2]	D, see ^{116}Te	3E+4	6E+4	3E-5	9E-8	4E-4	4E-3
		W, see ^{116}Te	-	7E+4	3E-5	1E-7	-	-
52	Tellurium-131m	D, see ^{116}Te	3E+2	4E+2	2E-7	-	-	-
			Thyroid (6E+2)	Thyroid (1E+3)	-	2E-9	8E-6	8E-5
		W, see ^{116}Te	-	4E+2	2E-7	-	-	-
			-	Thyroid (9E+2)	-	1E-9	-	-
52	Tellurium-131[2]	D, see ^{116}Te	3E+3	5E+3	2E-6	-	-	-
			Thyroid (6E+3)	Thyroid (1E+4)	-	2E-8	8E-5	8E-4
		W, see ^{116}Te	-	5E+3	2E-6	-	-	-
			-	Thyroid (1E+4)	-	2E-8	-	-
52	Tellurium-132	D, see ^{116}Te	2E+2	2E+2	9E-8	-	-	-
			Thyroid (7E+2)	Thyroid (8E+2)	-	1E-9	9E-6	9E-5
		W, see ^{116}Te	-	2E+2	9E-8	-	-	-
			-	Thyroid (6E+2)	-	9E-10	-	-
52	Tellurium-133m[2]	D, see ^{116}Te	3E+3	5E+3	2E-6	-	-	-
			Thyroid (6E+3)	Thyroid (1E+4)	-	2E-8	9E-5	9E-4
		W, see ^{116}Te	-	5E+3	2E-6	-	-	-
			-	Thyroid (1E+4)	-	2E-8	-	-
52	Tellurium-133[2]	D, see ^{116}Te	1E+4	2E+4	9E-6	-	-	-
			Thyroid (3E+4)	Thyroid (6E+4)	-	8E-8	4E-4	4E-3
		W, see ^{116}Te	-	2E+4	9E-6	-	-	-
			-	Thyroid (6E+4)	-	8E-8	-	-
52	Tellurium-134[2]	D, see ^{116}Te	2E+4	2E+4	1E-5	-	-	-
			Thyroid (2E+4)	Thyroid (5E+4)	-	7E-8	3E-4	3E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{116}Te	-	2E+4	1E-5	-	-	-
			-	Thyroid (5E+4)	-	7E-8	-	-
53	Iodine-120m[2]	D, all compounds	1E+4	2E+4	9E-6	3E-8	-	-
			Thyroid (1E+4)	-	-	-	2E-4	2E-3
53	Iodine-120[2]	D, all compounds	4E+3	9E+3	4E-6	-	-	-
			Thyroid (8E+3)	Thyroid (1E+4)	-	2E-8	1E-4	1E-3
53	Iodine-121	D, all compounds	1E+4	2E+4	8E-6	-	-	-
			Thyroid (3E+4)	Thyroid (5E+4)	-	7E-8	4E-4	4E-3
53	Iodine-123	D, all compounds	3E+3	6E+3	3E-6	-	-	-
			Thyroid (1E+4)	Thyroid (2E+4)	-	2E-8	1E-4	1E-3
53	Iodine-124	D, all compounds	5E+1	8E+1	3E-8	-	-	-
			Thyroid (2E+2)	Thyroid (3E+2)	-	4E-10	2E-6	2E-5
53	Iodine-125	D, all compounds	4E+1	6E+1	3E-8	-	-	-
			Thyroid (1E+2)	Thyroid (2E+2)	-	3E-10	2E-6	2E-5
53	Iodine-126	D, all compounds	2E+1	4E+1	1E-8	-	-	-
			Thyroid (7E+1)	Thyroid (1E+2)	-	2E-10	1E-6	1E-5
53	Iodine-128[2]	D, all compounds	4E+4	1E+5	5E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	8E-4	8E-3
53	Iodine-129	D, all compounds	5E+0	9E+0	4E-9	-	-	-
			Thyroid (2E+1)	Thyroid (3E+1)	-	4E-11	2E-7	2E-6
53	Iodine-130	D, all compounds	4E+2	7E+2	3E-7	-	-	-
			Thyroid (1E+3)	Thyroid (2E+3)	-	3E-9	2E-5	2E-4
53	Iodine-131	D, all compounds	3E+1	5E+1	2E-8	-	-	-
			Thyroid (9E+1)	Thyroid (2E+2)	-	2E-10	1E-6	1E-5
53	Iodine-132m[2]	D, all compounds	4E+3	8E+3	4E-6	-	-	-
			Thyroid (1E+4)	Thyroid (2E+4)	-	3E-8	1E-4	1E-3
53	Iodine-132	D, all compounds	4E+3	8E+3	3E-6	-	-	-
			Thyroid (9E+3)	Thyroid (1E+4)	-	2E-8	1E-4	1E-3
53	Iodine-133	D, all compounds	1E+2	3E+2	1E-7	-	-	-
			Thyroid (5E+2)	Thyroid (9E+2)	-	1E-9	7E-6	7E-5
53	Iodine-134[2]	D, all compounds	2E+4	5E+4	2E-5	6E-8	-	-
			Thyroid (3E+4)	-	-	-	4E-4	4E-3
53	Iodine-135	D, all compounds	8E+2	2E+3	7E-7	-	-	-
			Thyroid (3E+3)	Thyroid (4E+3)	-	6E-9	3E-5	3E-4
54	Xenon-120[2]	Submersion[1]	-	-	1E-5	4E-8	-	-
54	Xenon-121[2]	Submersion[1]	-	-	2E-6	1E-8	-	-
54	Xenon-122	Submersion[1]	-	-	7E-5	3E-7	-	-
54	Xenon-123	Submersion[1]	-	-	6E-6	3E-8	-	-
54	Xenon-125	Submersion[1]	-	-	2E-5	7E-8	-	-
54	Xenon-127	Submersion[1]	-	-	1E-5	6E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
54	Xenon-129m	Submersion[1]	-	-	2E-4	9E-7	-	-
54	Xenon-131m	Submersion[1]	-	-	4E-4	2E-6	-	-
54	Xenon-133m	Submersion[1]	-	-	1E-4	6E-7	-	-
54	Xenon-133	Submersion[1]	-	-	1E-4	5E-7	-	-
54	Xenon-135m[2]	Submersion[1]	-	-	9E-6	4E-8	-	-
54	Xenon-135	Submersion[1]	-	-	1E-5	7E-8	-	-
54	Xenon-138[2]	Submersion[1]	-	-	4E-6	2E-8	-	-
55	Cesium-125[2]	D, all compounds	5E+4	1E+5	6E-5	2E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
55	Cesium-127	D, all compounds	6E+4	9E+4	4E-5	1E-7	9E-4	9E-3
55	Cesium-129	D, all compounds	2E+4	3E+4	1E-5	5E-8	3E-4	3E-3
55	Cesium-130[2]	D, all compounds	6E+4	2E+5	8E-5	3E-7	-	-
			St wall (1E+5)	-	-	-	1E-3	1E-2
55	Cesium-131	D, all compounds	2E+4	3E+4	1E-5	4E-8	3E-4	3E-3
55	Cesium-132	D, all compounds	3E+3	4E+3	2E-6	6E-9	4E-5	4E-4
55	Cesium-134m	D, all compounds	1E+5	1E+5	6E-5	2E-7	-	-
			St wall (1E+5)	-	-	-	2E-3	2E-2
55	Cesium-134	D, all compounds	7E+1	1E+2	4E-8	2E-10	9E-7	9E-6
55	Cesium-135m[2]	D, all compounds	1E+5	2E+5	8E-5	3E-7	1E-3	1E-2
55	Cesium-135	D, all compounds	7E+2	1E+3	5E-7	2E-9	1E-5	1E-4
55	Cesium-136	D, all compounds	4E+2	7E+2	3E-7	9E-10	6E-6	6E-5
55	Cesium-137	D, all compounds	1E+2	2E+2	6E-8	2E-10	1E-6	1E-5
55	Cesium-138[2]	D, all compounds	2E+4	6E+4	2E-5	8E-8	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
56	Barium-126[2]	D, all compounds	6E+3	2E+4	6E-6	2E-8	8E-5	8E-4
56	Barium-128	D, all compounds	5E+2	2E+3	7E-7	2E-9	7E-6	7E-5
56	Barium-131m[2]	D, all compounds	4E+5	1E+6	6E-4	2E-6	-	-
			St wall (5E+5)	-	-	-	7E-3	7E-2
56	Barium-131	D, all compounds	3E+3	8E+3	3E-6	1E-8	4E-5	4E-4
56	Barium-133m	D, all compounds	2E+3	9E+3	4E-6	1E-8	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
56	Barium-133	D, all compounds	2E+3	7E+2	3E-7	9E-10	2E-5	2E-4
56	Barium-135m	D, all compounds	3E+3	1E+4	5E-6	2E-8	4E-5	4E-4
56	Barium-139[2]	D, all compounds	1E+4	3E+4	1E-5	4E-8	2E-4	2E-3
56	Barium-140	D, all compounds	5E+2	1E+3	6E-7	2E-9	-	-
			LLI wall (6E+2)	-	-	-	8E-6	8E-5
56	Barium-141[2]	D, all compounds	2E+4	7E+4	3E-5	1E-7	3E-4	3E-3
56	Barium-142[2]	D, all compounds	5E+4	1E+5	6E-5	2E-7	7E-4	7E-3
57	Lanthanum-131[2]	D, all compounds except those given for W	5E+4	1E+5	5E-5	2E-7	6E-4	6E-3
		W, oxides and hydroxides	-	2E+5	7E-5	2E-7	-	-
57	Lanthanum-132	D, see ^{131}La	3E+3	1E+4	4E-6	1E-8	4E-5	4E-4
		W, see ^{131}La	-	1E+4	5E-6	2E-8	-	-
57	Lanthanum-135	D, see ^{131}La	4E+4	1E+5	4E-5	1E-7	5E-4	5E-3
		W, see ^{131}La	-	9E+4	4E-5	1E-7	-	-
57	Lanthanum-137	D, see ^{131}La	1E+4	6E+1	3E-8	-	2E-4	2E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			-	Liver (7E+1)	-	1E-10	-	-
		W, see ^{131}La	-	3E+2	1E-7	-	-	-
			-	Liver (3E+2)	-	4E-10	-	-
57	Lanthanum-138	D, see ^{131}La	9E+2	4E+0	1E-9	5E-12	1E-5	1E-4
		W, see ^{131}La	-	1E+1	6E-9	2E-11	-	-
57	Lanthanum-140	D, see ^{131}La	6E+2	1E+3	6E-7	2E-9	9E-6	9E-5
		W, see ^{131}La	-	1E+3	5E-7	2E-9	-	-
57	Lanthanum-141	D, see ^{131}La	4E+3	9E+3	4E-6	1E-8	5E-5	5E-4
		W, see ^{131}La	-	1E+4	5E-6	2E-8	-	-
57	Lanthanum-142[2]	D, see ^{131}La	8E+3	2E+4	9E-6	3E-8	1E-4	1E-3
		W, see ^{131}La	-	3E+4	1E-5	5E-8	-	-
57	Lanthanum-143[2]	D, see ^{131}La	4E+4	1E+5	4E-5	1E-7	-	-
			St wall (4E+4)	-	-	-	5E-4	5E-3
		W, see ^{131}La	-	9E+4	4E-5	1E-7	-	-
58	Cerium-134	W, all compounds except those given for Y	5E+2	7E+2	3E-7	1E-9	-	-
			LLI wall (6E+2)	-	-	-	8E-6	8E-5
		Y, oxides, hydroxides, and fluorides	-	7E+2	3E-7	9E-10	-	-
58	Cerium-135	W, see ^{134}Ce	2E+3	4E+3	2E-6	5E-9	2E-5	2E-4
		Y, see ^{134}Ce	-	4E+3	1E-6	5E-9	-	-
58	Cerium-137m	W, see ^{134}Ce	2E+3	4E+3	2E-6	6E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
		Y, see ^{134}Ce	-	4E+3	2E-6	5E-9	-	-
58	Cerium-137	W, see ^{134}Ce	5E+4	1E+5	6E-5	2E-7	7E-4	7E-3
		Y, see ^{134}Ce	-	1E+5	5E-5	2E-7	-	-
58	Cerium-139	W, see ^{134}Ce	5E+3	8E+2	3E-7	1E-9	7E-5	7E-4
		Y, see ^{134}Ce	-	7E+2	3E-7	9E-10	-	-
58	Cerium-141	W, see ^{134}Ce	2E+3	7E+2	3E-7	1E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
		Y, see ^{134}Ce	-	6E+2	2E-7	8E-10	-	-
58	Cerium-143	W, see ^{134}Ce	1E+3	2E+3	8E-7	3E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{134}Ce	-	2E+3	7E-7	2E-9	-	-
58	Cerium-144	W, see ^{134}Ce	2E+2	3E+1	1E-8	4E-11	-	-
			LLI wall (3E+2)	-	-	-	3E-6	3E-5
		Y, see ^{134}Ce	-	1E+1	6E-9	2E-11	-	-
59	Praseodymium-136[2]	W, all compounds except those given for Y	5E+4	2E+5	1E-4	3E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
		Y, oxides, hydroxides, carbides, and fluorides	-	2E+5	9E-5	3E-7		-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
59	Praseodymium-137[2]	W, see ^{136}Pr	4E+4	2E+5	6E-5	2E-7	5E-4	5E-3
		Y, see ^{136}Pr	-	1E+5	6E-5	2E-7	-	-
59	Praseodymium-138m	W, see ^{136}Pr	1E+4	5E+4	2E-5	8E-8	1E-4	1E-3
		Y, see ^{136}Pr	-	4E+4	2E-5	6E-8	-	-
59	Praseodymium-139	W, see ^{136}Pr	4E+4	1E+5	5E-5	2E-7	6E-4	6E-3
		Y, see ^{136}Pr	-	1E+5	5E-5	2E-7	-	-
59	Praseodymium-142m[2]	W, see ^{136}Pr	8E+4	2E+5	7E-5	2E-7	1E-3	1E-2
		Y, see ^{136}Pr	-	1E+5	6E-5	2E-7	-	-
59	Praseodymium-142	W, see ^{136}Pr	1E+3	2E+3	9E-7	3E-9	1E-5	1E-4
		Y, see ^{136}Pr	-	2E+3	8E-7	3E-9	-	-
59	Praseodymium-143	W, see ^{136}Pr	9E+2	8E+2	3E-7	1E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{136}Pr	-	7E+2	3E-7	9E-10	-	-
59	Praseodymium-144[2]	W, see ^{136}Pr	3E+4	1E+5	5E-5	2E-7	-	-
			St wall (4E+4)	-	-	-	6E-4	6E-3
		Y, see ^{136}Pr	-	1E+5	5E-5	2E-7	-	-
59	Praseodymium-145	W, see ^{136}Pr	3E+3	9E+3	4E-6	1E-8	4E-5	4E-4
		Y, see ^{136}Pr	-	8E+3	3E-6	1E-8	-	-
59	Praseodymium-147[2]	W, see ^{136}Pr	5E+4	2E+5	8E-5	3E-7	-	-
			St wall (8E+4)	-	-	-	1E-3	1E-2
		Y, see ^{136}Pr	-	2E+5	8E-5	3E-7	-	-
60	Neodymium-136[2]	W, all compounds except those given for Y	1E+4	6E+4	2E-5	8E-8	2E-4	2E-3
		Y, oxides, hydroxides, carbides, and fluorides	-	5E+4	2E-5	8E-8	-	-
60	Neodymium-138	W, see ^{136}Nd	2E+3	6E+3	3E-6	9E-9	3E-5	3E-4
		Y, see ^{136}Nd	-	5E+3	2E-6	7E-9	-	-
60	Neodymium-139m	W, see ^{136}Nd	5E+3	2E+4	7E-6	2E-8	7E-5	7E-4
		Y, see ^{136}Nd	-	1E+4	6E-6	2E-8	-	-
60	Neodymium-139[2]	W, see ^{136}Nd	9E+4	3E+5	1E-4	5E-7	1E-3	1E-2
		Y, see ^{136}Nd	-	3E+5	1E-4	4E-7	-	-
60	Neodymium-141	W, see ^{136}Nd	2E+5	7E+5	3E-4	1E-6	2E-3	2E-2
		Y, see ^{136}Nd	-	6E+5	3E-4	9E-7	-	-
60	Neodymium-147	W, see ^{136}Nd	1E+3	9E+2	4E-7	1E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{136}Nd	-	8E+2	4E-7	1E-9	-	-
60	Neodymium-149[2]	W, see ^{136}Nd	1E+4	3E+4	1E-5	4E-8	1E-4	1E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		Y, see ^{136}Nd	-	2E+4	1E-5	3E-8	-	-
60	Neodymium-151[2]	W, see ^{136}Nd	7E+4	2E+5	8E-5	3E-7	9E-4	9E-3
		Y, see ^{136}Nd	-	2E+5	8E-5	3E-7	-	-
61	Promethium-141[2]	W, all compounds except those given for Y	5E+4	2E+5	8E-5	3E-7	-	-
			St wall (6E+4)	-	-	-	8E-4	8E-3
		Y, oxides, hydroxides, carbides, and fluorides	-	2E+5	7E-5	2E-7	-	-
61	Promethium-143	W, see ^{141}Pm	5E+3	6E+2	2E-7	8E-10	7E-5	7E-4
		Y, see ^{141}Pm	-	7E+2	3E-7	1E-9	-	-
61	Promethium-144	W, see ^{141}Pm	1E+3	1E+2	5E-8	2E-10	2E-5	2E-4
		Y, see ^{141}Pm	-	1E+2	5E-8	2E-10	-	-
61	Promethium-145	W, see ^{141}Pm	1E+4	2E+2	7E-8	-	1E-4	1E-3
			-	Bone surf (2E+2)	-	3E-10	-	-
		Y, see ^{141}Pm	-	2E+2	8E-8	3E-10	-	-
61	Promethium-146	W, see ^{141}Pm	2E+3	5E+1	2E-8	7E-11	2E-5	2E-4
		Y, see ^{141}Pm	-	4E+1	2E-8	6E-11	-	-
61	Promethium-147	W, see ^{141}Pm	4E+3	1E+2	5E-8	-	-	-
			LLI wall (5E+3)	Bone surf (2E+2)	-	3E-10	7E-5	7E-4
		Y, see ^{141}Pm	-	1E+2	6E-8	2E-10	-	-
61	Promethium-148m	W, see ^{141}Pm	7E+2	3E+2	1E-7	4E-10	1E-5	1E-4
		Y, see ^{141}Pm	-	3E+2	1E-7	5E-10	-	-
61	Promethium-148	W, see ^{141}Pm	4E+2	5E+2	2E-7	8E-10	-	-
			LLI wall (5E+2)	-	-	-	7E-6	7E-5
		Y, see ^{141}Pm	-	5E+2	2E-7	7E-10	-	-
61	Promethium-149	W, see ^{141}Pm	1E+3	2E+3	8E-7	3E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{141}Pm	-	2E+3	8E-7	2E-9	-	-
61	Promethium-150	W, see ^{141}Pm	5E+3	2E+4	8E-6	3E-8	7E-5	7E-4
		Y, see ^{141}Pm	-	2E+4	7E-6	2E-8	-	-
61	Promethium-151	W, see ^{141}Pm	2E+3	4E+3	1E-6	5E-9	2E-5	2E-4
		Y, see ^{141}Pm	-	3E+3	1E-6	4E-9	-	-
62	Samarium-141m[2]	W, all compounds	3E+4	1E+5	4E-5	1E-7	4E-4	4E-3
62	Samarium-141[2]	W, all compounds	5E+4	2E+5	8E-5	2E-7	-	-
			St wall (6E+4)	-	-	-	8E-4	8E-3
62	Samarium-142[2]	W, all compounds	8E+3	3E+4	1E-5	4E-8	1E-4	1E-3
62	Samarium-145	W, all compounds	6E+3	5E+2	2E-7	7E-10	8E-5	8E-4
62	Samarium-146	W, all compounds	1E+1	4E+2	1E-11	-	-	-
			Bone surf (3E+1)	Bone surf (6E-2)	-	9E-14	3E-7	3E-6
62	Samarium-147	W, all compounds	2E+1	4E-2	2E-11	-	-	-
			Bone surf (3E+1)	Bone surf (7E-2)	-	1E-13	4E-7	4E-6
62	Samarium-151	W, all compounds	1E+4	1E+2	4E-8	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			LLI wall (1E+4)	Bone surf (2E+2)	-	2E-10	2E-4	2E-3
62	Samarium-153	W, all compounds	2E+3	3E+3	1E-6	4E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
62	Samarium-155[2]	W, all compounds	6E+4	2E+5	9E-5	3E-7	-	-
			St wall (8E+4)	-	-	-	1E-3	1E-2
62	Samarium-156	W, all compounds	5E+3	9E+3	4E-6	1E-8	7E-5	7E-4
63	Europium-145	W, all compounds	2E+3	2E+3	8E-7	3E-9	2E-5	2E-4
63	Europium-146	W, all compounds	1E+3	1E+3	5E-7	2E-9	1E-5	1E-4
63	Europium-147	W, all compounds	3E+3	2E+3	7E-7	2E-9	4E-5	4E-4
63	Europium-148	W, all compounds	1E+3	4E+2	1E-7	5E-10	1E-5	1E-4
63	Europium-149	W, all compounds	1E+4	3E+3	1E-6	4E-9	2E-4	2E-3
63	Europium-150 (12.62 h)	W, all compounds	3E+3	8E+3	4E-6	1E-8	4E-5	4E-4
63	Europium-150 (34.2 y)	W, all compounds	8E+2	2E+1	8E-9	3E-11	1E-5	1E-4
63	Europium-152m	W, all compounds	3E+3	6E+3	3E-6	9E-9	4E-5	4E-4
63	Europium-152	W, all compounds	8E+2	2E+1	1E-8	3E-11	1E-5	1E-4
63	Europium-154	W, all compounds	5E+2	2E+1	8E-9	3E-11	7E-6	7E-5
63	Europium-155	W, all compounds	4E+3	9E+1	4E-8	-	5E-5	5E-4
			-	Bone surf (1E+2)	-	2E-10	-	-
63	Europium-156	W, all compounds	6E+2	5E+2	2E-7	6E-10	8E-6	8E-5
63	Europium-157	W, all compounds	2E+3	5E+3	2E-6	7E-9	3E-5	3E-4
63	Europium-158[2]	W, all compounds	2E+4	6E+4	2E-5	8E-8	3E-4	3E-3
64	Gadolinium-145[2]	D, all compounds except those given for W	5E+4	2E+5	6E-5	2E-7	-	-
			St wall (5E+4)	-	-	-	6E-4	6E-3
		W, oxides, hydroxides, and fluorides	-	2E+5	7E-5	2E-7	-	-
64	Gadolinium-146	D, see ^{145}Gd	1E+3	1E+2	5E-8	2E-10	2E-5	2E-4
		W, see ^{145}Gd	-	3E+2	1E-7	4E-10	-	-
64	Gadolinium-147	D, see ^{145}Gd	2E+3	4E+3	2E-6	6E-9	3E-5	3E-4
		W, see ^{145}Gd	-	4E+3	1E-6	5E-9	-	-
64	Gadolinium-148	D, see ^{145}Gd	1E+1	8E-3	3E-12	-	-	-
			Bone surf (2E+1)	Bone surf (2E+2)	-	2E-14	3E-7	3E-6
		W, see ^{145}Gd	-	3E-2	1E-11	-	-	-
			-	Bone surf (6E-2)	-	8E-14	-	-
64	Gadolinium-149	D, see ^{145}Gd	3E+3	2E+3	9E-7	3E-9	4E-5	4E-4
		W, see ^{145}Gd	-	2E+3	1E-6	3E-9	-	-
64	Gadolinium-151	D, see ^{145}Gd	6E+3	4E+2	2E-7	-	9E-5	9E-4
			-	Bone surf (6E+2)	-	9E-10	-	-
		W, see ^{145}Gd	-	1E+3	5E-7	2E-9	-	-
64	Gadolinium-152	D, see ^{145}Gd	2E+1	1E-2	4E-12	-	-	-
			Bone surf (3E+1)	Bone surf (2E-2)	-	3E-14	4E-7	4E-6
		W, see ^{145}Gd	-	4E-2	2E-11	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			-	Bone surf (8E-2)	-	1E-13	-	-
64	Gadolinium-153	D, see ^{145}Gd	5E+3	1E+2	6E-8	-	6E-5	6E-4
			-	Bone surf (2E+2)	-	3E-10	-	-
		W, see ^{145}Gd	-	6E+2	2E-7	8E-10	-	-
64	Gadolinium-159	D, see ^{145}Gd	3E+3	8E+3	3E-6	1E-8	4E-5	4E-4
		W, see ^{145}Gd	-	6E+3	2E-6	8E-9	-	-
65	Terbium-147[2]	W, all compounds	9E+3	3E+4	1E-5	5E-8	1E-4	1E-3
65	Terbium-149	W, all compounds	5E+3	7E+2	3E-7	1E-9	7E-5	7E-4
65	Terbium-150	W, all compounds	5E+3	2E+4	9E-6	3E-8	7E-5	7E-4
65	Terbium-151	W, all compounds	4E+3	9E+3	4E-6	1E-8	5E-5	5E-4
65	Terbium-153	W, all compounds	5E+3	7E+3	3E-6	1E-8	7E-5	7E-4
65	Terbium-154	W, all compounds	2E+3	4E+3	2E-6	6E-9	2E-5	2E-4
65	Terbium-155	W, all compounds	6E+3	8E+3	3E-6	1E-8	8E-5	8E-4
65	Terbium-156m (5.0 h)	W, all compounds	2E+4	3E+4	1E-5	4E-8	2E-4	2E-3
65	Terbium-156m (24.4 h)	W, all compounds	7E+3	8E+3	3E-6	1E-8	1E-4	1E-3
65	Terbium-156	W, all compounds	1E+3	1E+3	6E-7	2E-9	1E-5	1E-4
65	Terbium-157	W, all compounds	5E+4	3E+2	1E-7	-	-	-
			LLI wall (5E+4)	Bone surf (6E+2)	-	8E-10	7E-4	7E-3
65	Terbium-158	W, all compounds	1E+3	2E+1	8E-9	3E-11	2E-5	2E-4
65	Terbium-160	W, all compounds	8E+2	2E+2	9E-8	3E-10	1E-5	1E-4
65	Terbium-161	W, all compounds	2E+3	2E+3	7E-7	2E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
66	Dysprosium-155	W, all compounds	9E+3	3E+4	1E-5	4E-8	1E-4	1E-3
66	Dysprosium-157	W, all compounds	2E+4	6E+4	3E-5	9E-8	3E-4	3E-3
66	Dysprosium-159	W, all compounds	1E+4	2E+3	1E-6	3E-9	2E-4	2E-3
66	Dysprosium-165	W, all compounds	1E+4	5E+4	2E-5	6E-8	2E-4	2E-3
66	Dysprosium-166	W, all compounds	6E+2	7E+2	3E-7	1E-9	-	-
			LLI wall (8E+2)	-	-	-	1E-5	1E-4
67	Holmium-155[2]	W, all compounds	4E+4	2E+5	6E-5	2E-7	6E-4	6E-3
67	Holmium-157[2]	W, all compounds	3E+5	1E+6	6E-4	2E-6	4E-3	4E-2
67	Holmium-159[2]	W, all compounds	2E+5	1E+6	4E-4	1E-6	3E-3	3E-2
67	Holmium-161	W, all compounds	1E+5	4E+5	2E-4	6E-7	1E-3	1E-2
67	Holmium-162m[2]	W, all compounds	5E+4	3E+5	1E-4	4E-7	7E-4	7E-3
67	Holmium-162[2]	W, all compounds	5E+5	2E+6	1E-3	3E-6	-	-
			St wall (8E+5)	-	-	-	1E-2	1E-1
67	Holmium-164m[2]	W, all compounds	1E+5	3E+5	1E-4	4E-7	1E-3	1E-2
67	Holmium-164[2]	W, all compounds	2E+5	6E+5	3E-4	9E-7	-	-
			St wall (2E+5)	-	-	-	3E-3	3E-2
67	Holmium-166m	W, all compounds	6E+2	7E+0	3E-9	9E-12	9E-6	9E-5
67	Holmium-166	W, all compounds	9E+2	2E+3	7E-7	2E-9	-	-
			LLI wall (9E+2)	-	-	-	1E-5	1E-4
67	Holmium-167	W, all compounds	2E+4	6E+4	2E-5	8E-8	2E-4	2E-3
68	Erbium-161	W, all compounds	2E+4	6E+4	3E-5	9E-8	2E-4	2E-3
68	Erbium-165	W, all compounds	6E+4	2E+5	8E-5	3E-7	9E-4	9E-3
68	Erbium-169	W, all compounds	3E+3	3E+3	1E-6	4E-9	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			LLI wall (4E+3)	-	-	-	5E-5	5E-4
68	Erbium-171	W, all compounds	4E+3	1E+4	4E-6	1E-8	5E-5	5E-4
68	Erbium-172	W, all compounds	1E+3	1E+3	6E-7	2E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
69	Thulium-162[2]	W, all compounds	7E+4	3E+5	1E-4	4E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
69	Thulium-166	W, all compounds	4E+3	1E+4	6E-6	2E-8	6E-5	6E-4
69	Thulium-167	W, all compounds	2E+3	2E+3	8E-7	3E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
69	Thulium-170	W, all compounds	8E+2	2E+2	9E-8	3E-10	-	-
			LLI wall (1E+3)	-	-	-	1E-5	1E-4
69	Thulium-171	W, all compounds	1E+4	3E+2	1E-7	-	-	-
			LLI wall (1E+4)	Bone surf (6E+2)	-	8E-10	2E-4	2E-3
69	Thulium-172	W, all compounds	7E+2	1E+3	5E-7	2E-9	-	-
			LLI wall (8E+2)	-	-	-	1E-5	1E-4
69	Thulium-173	W, all compounds	4E+3	1E+4	5E-6	2E-8	6E-5	6E-4
69	Thulium-175[2]	W, all compounds	7E+4	3E+5	1E-4	4E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
70	Ytterbium-162[2]	W, all compounds except those given for Y	7E+4	3E+5	1E-4	4E-7	1E-3	1E-2
		Y, oxides, hydroxides, and fluorides	-	3E+5	1E-4	4E-7	-	-
70	Ytterbium-166	W, see ^{162}Yb	1E+3	2E+3	8E-7	3E-9	2E-5	2E-4
		Y, see ^{162}Yb	-	2E+3	8E-7	3E-9	-	-
70	Ytterbium-167[2]	W, see ^{162}Yb	3E+5	8E+5	3E-4	1E-6	4E-3	4E-2
		Y, see ^{162}Yb	-	7E+5	3E-4	1E-6	-	-
70	Ytterbium-169	W, see ^{162}Yb	2E+3	8E+2	4E-7	1E-9	2E-5	2E-4
		Y, see ^{162Yb}	-	7E+2	3E-7	1E-9	-	-
70	Ytterbium-175	W, see ^{162}Yb	3E+3	4E+3	1E-6	5E-9	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
		Y, see ^{162}Yb	-	3E+3	1E-6	5E-9	-	-
70	Ytterbium-177[2]	W, see ^{162}Yb	2E+4	5E+4	2E-5	7E-8	2E-4	2E-3
		Y, see ^{162}Yb	-	5E+4	2E-5	6E-8	-	-
70	Ytterbium-178[2]	W, see ^{162}Yb	1E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		Y, see ^{162}Yb	-	4E+4	2E-5	5E-8	-	-
71	Lutetium-169	W, all compounds except those given for Y	3E+3	4E+3	2E-6	6E-9	3E-5	3E-4
		Y, oxides, hydroxides, and fluorides	-	4E+3	2E-6	6E-9	-	-
71	Lutetium-170	W, see ^{169}Lu	1E+3	2E+3	9E-7	3E-9	2E-5	2E-4
		Y, see ^{169}Lu	-	2E+3	8E-7	3E-9	-	-
71	Lutetium-171	W, see ^{169}Lu	2E+3	2E+3	8E-7	3E-9	3E-5	3E-4
		Y, see ^{169}Lu	-	2E+3	8E-7	3E-9	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
71	Lutetium-172	W, see ^{169}Lu	1E+3	1E+3	5E-7	2E-9	1E-5	1E-4
		Y, see ^{169}Lu	-	1E+3	5E-7	2E-9	-	-
71	Lutetium-173	W, see ^{169}Lu	5E+3	3E+2	1E-7	-	7E-5	7E-4
			-	Bone surf (5E+2)	-	6E-10	-	-
		Y, see ^{169}Lu	-	3E+2	1E-7	4E-10	-	-
71	Lutetium-174m	W, see ^{169}Lu	2E+3	2E+2	1E-7	-	-	-
			LLI wall (3E+3)	Bone surf (3E+2)	-	5E-10	4E-5	4E-4
		Y, see ^{169}Lu	-	2E+2	9E-8	3E-10	-	-
71	Lutetium-174	W, see ^{169}Lu	5E+3	1E+2	5E-8	-	7E-5	7E-4
			-	Bone surf (2E+2)	-	3E-10	-	-
		Y, see ^{169}Lu	-	2E+2	6E-8	2E-10	-	-
71	Lutetium-176m	W, see ^{169}Lu	8E+3	3E+4	1E-5	3E-8	1E-4	1E-3
		Y, see ^{169}Lu	-	2E+4	9E-6	3E-8	-	-
71	Lutetium-176	W, see ^{169}Lu	7E+2	5E+0	2E-9	-	1E-5	1E-4
			-	Bone surf (1E+1)	-	2E-11	-	-
		Y, see ^{169}Lu	-	8E+0	3E-9	1E-11	-	-
71	Lutetium-177m	W, see ^{169}Lu	7E+2	1E+2	5E-8	-	1E-5	1E-4
			-	Bone surf (1E+2)	-	2E-10	-	-
		Y, see ^{169}Lu	-	8E+1	3E-8	1E-10	-	-
71	Lutetium-177	W, see ^{169}Lu	2E+3	2E+3	9E-7	3E-9	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
		Y, see ^{169}Lu	-	2E+3	9E-7	3E-9	-	-
71	Lutetium-178m[2]	W, see ^{169}Lu	5E+4	2E+5	8E-5	3E-7	-	-
			St. wall (6E+4)	-	-	-	8E-4	8E-3
		Y, see ^{169}Lu	-	2E+5	7E-5	2E-7	-	-
71	Lutetium-178[2]	W, see ^{169}Lu	4E+4	1E+5	5E-5	2E-7	-	-
			St wall (4E+4)	-	-	-	6E-4	6E-3
		Y, see ^{169}Lu	-	1E+5	5E-5	2E-7	-	-
71	Lutetium-179	W, see ^{169}Lu	6E+3	2E+4	8E-6	3E-8	9E-5	9E-4
		Y, see ^{169}Lu	-	2E+4	6E-6	3E-8	-	-
72	Hafnium-170	D, all compounds except those given for W	3E+3	6E+3	2E-6	8E-9	4E-5	4E-4
		W, oxides, hydroxides, carbides, and nitrates	-	5E+3	2E-6	6E-9	-	-
72	Hafnium-172	D, see ^{170}Hf	1E+3	9E+0	4E-9	-	2E-5	2E-4
			-	Bone surf (2E+1)	-	3E-11	-	-
		W, see ^{170}Hf	-	4E+1	2E-8	-	-	-
			-	Bone surf (6E+1)	-	8E-11	-	-
72	Hafnium-173	D, see ^{170}Hf	5E+3	1E+4	5E-6	2E-8	7E-5	7E-4
		W, see ^{170}Hf	-	1E+4	5E-6	2E-8	-	-
72	Hafnium-175	D, see ^{170}Hf	3E+3	9E+2	4E-7	-	4E-5	4E-4
			-	Bone surf (1E+3)	-	1E-9	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{170}Hf	-	1E+3	5E-7	2E-9	-	-
72	Hafnium-177m[2]	D, see ^{170}Hf	2E+4	6E+4	2E-5	8E-8	3E-4	3E-3
		W, see ^{170}Hf	-	9E+4	4E-5	1E-7	-	-
72	Hafnium-178m	D, see ^{170}Hf	3E+2	1E+0	5E-10	-	3E-6	3E-5
			-	Bone surf (2E+0)	-	3E-12	-	-
		W, see ^{170}Hf	-	5E+0	2E-9	-	-	-
			-	Bone surf (9E+0)	-	1E-11	-	-
72	Hafnium-179m	D, see ^{170}Hf	1E+3	3E+2	1E-7	-	1E-5	1E-4
			-	Bone surf (6E+2)	-	8E-10	-	-
		W, see ^{170}Hf	-	6E+2	3E-7	8E-10	-	-
72	Hafnium-180m	D, see ^{170}Hf	7E+3	2E+4	9E-6	3E-8	1E-4	1E-3
		W, see ^{170}Hf	-	3E+4	1E-5	4E-8	-	-
72	Hafnium-181	D, see ^{170}Hf	1E+3	2E+2	7E-8	-	2E-5	2E-4
			-	Bone surf (4E+2)	-	6E-10	-	-
		W, see ^{170}Hf	-	4E+2	2E-7	6E-10	-	-
72	Hafnium-182m[2]	D, see ^{170}Hf	4E+4	9E+4	4E-5	1E-7	5E-4	5E-3
		W, see ^{170}Hf	-	1E+5	6E-5	2E-7	-	-
72	Hafnium-182	D, see ^{170}Hf	2E+2	8E-1	3E-10	-	-	-
			Bone surf (4E+2)	Bone surf (2E+0)	-	2E-12	5E-6	5E-5
		W, see ^{170}Hf	-	3E+0	1E-9	-	-	-
			-	Bone surf (7E+0)	-	1E-11	-	-
72	Hafnium-183[2]	D, see ^{170}Hf	2E+4	5E+4	2E-5	6E-8	3E-4	3E-3
		W, see ^{170}Hf	-	6E+4	2E-5	8E-8	-	-
72	Hafnium-184	D, see ^{170}Hf	2E+3	8E+3	3E-6	1E-8	3E-5	3E-4
		W, see ^{170}Hf	-	6E+3	3E-6	9E-9	-	-
73	Tantalum-172[2]	W, all compounds except those given for Y	4E+4	1E+5	5E-5	2E-7	5E-4	5E-3
		Y, elemental Ta, oxides, hydroxides, halides, carbides, nitrates, and nitrides	-	1E+5	4E-5	1E-7	-	-
73	Tantalum-173	W, see ^{172}Ta	7E+3	2E+4	8E-6	3E-8	9E-5	9E-4
		Y, see ^{172}Ta	-	2E+4	7E-6	2E-8	-	-
73	Tantalum-174[2]	W, see ^{172}Ta	3E+4	1E+5	4E-5	1E-7	4E-4	4E-3
		Y, see ^{172}Ta	-	9E+4	4E-5	1E-7	-	-
73	Tantalum-175	W, see ^{172}Ta	6E+3	2E+4	7E-6	2E-8	8E-5	8E-4
		Y, see ^{172}Ta	-	1E+4	6E-6	2E-8	-	-
73	Tantalum-176	W, see ^{172}Ta	4E+3	1E+4	5E-6	2E-8	5E-5	5E-4
		Y, see ^{172}Ta	-	1E+4	5E-6	2E-8	-	-
73	Tantalum-177	W, see ^{172}Ta	1E+4	2E+4	8E-6	3E-8	2E-4	2E-3
		Y, see ^{172}Ta	-	2E+4	7E-6	2E-8	-	-
73	Tantalum-178	W, see ^{172}Ta	2E+4	9E+4	4E-5	1E-7	2E-4	2E-3
		Y, see ^{172}Ta	-	7E+4	3E-5	1E-7	-	-
73	Tantalum-179	W, see ^{172}Ta	2E+4	5E+3	2E-6	8E-9	3E-4	3E-3
		Y, see ^{172}Ta	-	9E+2	4E-7	1E-9	-	-
73	Tantalum-180m	W, see ^{172}Ta	2E+4	7E+4	3E-5	9E-8	3E-4	3E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		Y, see ^{172}Ta	-	6E+4	2E-5	8E-8	-	-
73	Tantalum-180	W, see ^{172}Ta	1E+3	4E+2	2E-7	6E-10	2E-5	2E-4
		Y, see ^{172}Ta	-	2E+1	1E-8	3E-11	-	-
73	Tantalum-182m[2]	W, see ^{172}Ta	2E+5	5E+5	2E-4	8E-7	-	-
			St wall (2E+5)	-	-	-	3E-3	3E-2
		Y, see ^{172}Ta	-	4E+5	2E-4	6E-7	-	-
73	Tantalum-182	W, see ^{172}Ta	8E+2	3E+2	1E-7	5E-10	1E-5	1E-4
		Y, see ^{172}Ta	-	1E+2	6E-8	2E-10	-	-
73	Tantalum-183	W, see ^{172}Ta	9E+2	1E+3	5E-7	2E-9	-	-
			LLI wall (1E+3)	-	-	-	2E-5	2E-4
		Y, see ^{172}Ta	-	1E+3	4E-7	1E-9	-	-
73	Tantalum-184	W, see ^{172}Ta	2E+3	5E+3	2E-6	8E-9	3E-5	3E-4
		Y, see ^{172}Ta	-	5E+3	2E-6	7E-9	-	-
73	Tantalum-185[2]	W, see ^{172}Ta	3E+4	7E+4	3E-5	1E-7	4E-4	4E-3
		Y, see ^{172}Ta	-	6E+4	3E-5	9E-8	-	-
73	Tantalum-186[2]	W, see ^{172}Ta	5E+4	2E+5	1E-4	3E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
		Y, see ^{172}Ta	-	2E+5	9E-5	3E-7	-	-
74	Tungsten-176	D, all compounds	1E+4	5E+4	2E-5	7E-8	1E-4	1E-3
74	Tungsten-177	D, all compounds	2E+4	9E+4	4E-5	1E-7	3E-4	3E-3
74	Tungsten-178	D, all compounds	5E+3	2E+4	8E-6	3E-8	7E-5	7E-4
74	Tungsten-179[2]	D, all compounds	5E+5	2E+6	7E-4	2E-6	7E-3	7E-2
74	Tungsten-181	D, all compounds	2E+4	3E+4	1E-5	5E-8	2E-4	2E-3
74	Tungsten-185	D, all compounds	2E+3	7E+3	3E-6	9E-9	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
74	Tungsten-187	D, all compounds	2E+3	9E+3	4E-6	1E-8	3E-5	3E-4
74	Tungsten-188	D, all compounds	4E+2	1E+3	5E-7	2E-9	-	-
			LLI wall (5E+2)	-	-	-	7E-6	7E-5
75	Rhenium-177[2]	D, all compounds except those given for W	9E+4	3E+5	1E-4	4E-7	-	-
			St wall (1E+5)	-	-	-	2E-3	2E-2
		W, oxides, hydroxides, and nitrates	-	4E+5	1E-4	5E-7	-	-
75	Rhenium-178[2]	D, see ^{177}Re	7E+4	3E+5	1E-4	4E-7	-	-
			St wall (1E+5)	-	-	-	1E-3	1E-2
		W, see ^{177}Re	-	3E+5	1E-4	4E-7	-	-
75	Rhenium-181	D, see ^{177}Re	5E+3	9E+3	4E-6	1E-8	7E-5	7E-4
		W, see ^{177}Re	-	9E+3	4E-6	1E-8	-	-
75	Rhenium-182 (12.7 h)	D, see ^{177}Re	7E+3	1E+4	5E-6	2E-8	9E-5	9E-4
		W, see ^{177}Re	-	2E+4	6E-6	2E-8	-	-
75	Rhenium-182 (64.0 h)	D, see ^{177}Re	1E+3	2E+3	1E-6	3E-9	2E-5	2E-4
		W, see ^{177}Re	-	2E+3	9E-7	3E-9	-	-
75	Rhenium-184m	D, see ^{177}Re	2E+3	3E+3	1E-6	4E-9	3E-5	3E-4
		W, see ^{177}Re	-	4E+2	2E-7	6E-10	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
75	Rhenium-184	D, see ^{177}Re	2E+3	4E+3	1E-6	5E-9	3E-5	3E-4
		W, see ^{177}Re	-	1E+3	6E-7	2E-9	-	-
75	Rhenium-186m	D, see ^{177}Re	1E+3	2E+3	7E-7	-	-	-
			St wall (2E+3)	St wall (2E+3)	-	3E-9	2E-5	2E-4
		W, see ^{177}Re	-	2E+2	6E-8	2E-10	-	-
75	Rhenium-186	D, see ^{177}Re	2E+3	3E+3	1E-6	4E-9	3E-5	3E-4
		W, see ^{177}Re	-	2E+3	7E-7	2E-9	-	-
75	Rhenium-187	D, see ^{177}Re	6E+5	8E+5	4E-4	-	8E-3	8E-2
			-	St wall (9E+5)	-	1E-6	-	-
		W, see ^{177}Re	-	1E+5	4E-5	1E-7	-	-
75	Rhenium-188m[2]	D, see ^{177}Re	8E+4	1E+5	6E-5	2E-7	1E-3	1E-2
		W, see ^{177}Re	-	1E+5	6E-5	2E-7	-	-
75	Rhenium-188	D, see ^{177}Re	2E+3	3E+3	1E-6	4E-9	2E-5	2E-4
		W, see ^{177}Re	-	3E+3	1E-6	4E-9	-	-
75	Rhenium-189	D, see ^{177}Re	3E+3	5E+3	2E-6	7E-9	4E-5	4E-4
		W, see ^{177}Re	-	4E+3	2E-6	6E-9	-	-
76	Osmium-180[2]	D, all compounds except those given for W and Y	1E+5	4E+5	2E-4	5E-7	1E-3	1E-2
		W, halides and nitrates	-	5E+5	2E-4	7E-7	-	-
		Y, oxides and hydroxides	-	5E+5	2E-4	6E-7	-	-
76	Osmium-181[2]	D, see ^{180}Os	1E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{180}Os	-	5E+4	2E-5	6E-8	-	-
		Y, see ^{180}Os	-	4E+4	2E-5	6E-8	-	-
76	Osmium-182	D, see ^{180}Os	2E+3	6E+3	2E-6	8E-9	3E-5	3E-4
		W, see ^{180}Os	-	4E+3	2E-6	6E-9	-	-
		Y, see ^{180}Os	-	4E+3	2E-6	6E-9	-	-
76	Osmium-185	D, see ^{180}Os	2E+3	5E+2	2E-7	7E-10	3E-5	3E-4
		W, see ^{180}Os	-	8E+2	3E-7	1E-9	-	-
		Y, see ^{180}Os	-	8E+2	3E-7	1E-9	-	-
76	Osmium-189m	D, see ^{180}Os	8E+4	2E+5	1E-4	3E-7	1E-3	1E-2
		W, see ^{180}Os	-	2E+5	9E-5	3E-7	-	-
		Y, see ^{180}Os	-	2E+5	7E-5	2E-7	-	-
76	Osmium-191m	D, see ^{180}Os	1E+4	3E+4	1E-5	4E-8	2E-4	2E-3
		W, see ^{180}Os	-	2E+4	8E-6	3E-8	-	-
		Y, see ^{180}Os	-	2E+4	7E-6	2E-8	-	-
76	Osmium-191	D, see ^{180}Os	2E+3	2E+3	9E-7	3E-9	-	-
			LLI wall (3E+3)	-	-	-	3E-5	3E-4
		W, see ^{180}Os	-	2E+3	7E-7	2E-9	-	-
		Y, see ^{180}Os	-	1E+3	6E-7	2E-9	-	-
76	Osmium-193	D, see ^{180}Os	2E+3	5E+3	2E-6	6E-9	-	-
			LLI wall (2E+3)	-	-	-	2E-5	2E-4
		W, see ^{180}Os	-	3E+3	1E-6	4E-9	-	-
		Y, see ^{180}Os	-	3E+3	1E-6	4E-9	-	-
76	Osmium-194	D, see ^{180}Os	4E+2	4E+1	2E-8	6E-11	-	-
			LLI wall (6E+2)	-	-	-	8E-6	8E-5

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{180}Os	-	6E+1	2E-8	8E-11	-	-
		Y, see ^{180}Os	-	8E+0	3E-9	1E-11	-	-
77	Iridium-182[2]	D, all compounds except those given for W and Y	4E+4	1E+5	6E-5	2E-7	-	-
			St wall (4E+4)	-	-	-	6E-4	6E-3
		W, halides, nitrates, and metallic iridium	-	2E+5	6E-5	2E-7	-	-
		Y, oxides and hydroxides	-	1E+5	5E-5	2E-7	-	-
77	Iridium-184	D, see ^{182}Ir	8E+3	2E+4	1E-5	3E-8	1E-4	1E-3
		W, see ^{182}Ir	-	3E+4	1E-5	5E-8	-	-
		Y, see ^{182}Ir	-	3E+4	1E-5	4E-8	-	-
77	Iridium-185	D, see ^{182}Ir	5E+3	1E+4	5E-6	2E-8	7E-5	7E-4
		W, see ^{182}Ir	-	1E+4	5E-6	2E-8	-	-
		Y, see ^{182}Ir	-	1E+4	4E-6	1E-8	-	-
77	Iridium-186	D, see ^{182}Ir	2E+3	8E+3	3E-6	1E-8	3E-5	3E-4
		W, see ^{182}Ir	-	6E+3	3E-6	9E-9	-	-
		Y, see ^{182}Ir	-	6E+3	2E-6	8E-9	-	-
77	Iridium-187	D, see ^{182}Ir	1E+4	3E+4	1E-5	5E-8	1E-4	1E-3
		W, see ^{182}Ir	-	3E+4	1E-5	4E-8	-	-
		Y, see ^{182}Ir	-	3E+4	1E-5	4E-8	-	-
77	Iridium-188	D, see ^{182}Ir	2E+3	5E+3	2E-6	6E-9	3E-5	3E-4
		W, see ^{182}Ir	-	4E+3	1E-6	5E-9	-	-
		Y, see ^{182}Ir	-	3E+3	1E-6	5E-9	-	-
77	Iridium-189	D, see ^{182}Ir	5E+3	5E+3	2E-6	7E-9	-	-
			LLI wall (5E+3)	-	-	-	7E-5	7E-4
		W, see ^{182}Ir	-	4E+3	2E-6	5E-9	-	-
		Y, see ^{182}Ir	-	4E+3	1E-6	5E-9	-	-
77	Iridium-190m[2]	D, see ^{182}Ir	2E+5	2E+5	8E-5	3E-7	2E-3	2E-2
		W, see ^{182}Ir	-	2E+5	9E-5	3E-7	-	-
		Y, see ^{182}Ir	-	2E+5	8E-5	3E-7	-	-
77	Iridium-190	D, see ^{182}Ir	1E+3	9E+2	4E-7	1E-9	1E-5	1E-4
		W, see ^{182}Ir	-	1E+3	4E-7	1E-9	-	-
		Y, see ^{182}Ir	-	9E+2	4E-7	1E-9	-	-
77	Iridium-192m	D, see ^{182}Ir	3E+3	9E+1	4E-8	1E-10	4E-5	4E-4
		W, see ^{182}Ir	-	2E+2	9E-8	3E-10	-	-
		Y, see ^{182}Ir	-	2E+1	6E-9	2E-11	-	-
77	Iridium-192	D, see ^{182}Ir	9E+2	3E+2	1E-7	4E-10	1E-5	1E-4
		W, see ^{182}Ir	-	4E+2	2E-7	6E-10	-	-
		Y, see ^{182}Ir	-	2E+2	9E-8	3E-10	-	-
77	Iridium-194m	D, see ^{182}Ir	6E+2	9E+1	4E-8	1E-10	9E-6	9E-5
		W, see ^{182}Ir	-	2E+2	7E-8	2E-10	-	-
		Y, see ^{182}Ir	-	1E+2	4E-8	1E-10	-	-
77	Iridium-194	D, see ^{182}Ir	1E+3	3E+3	1E-6	4E-9	1E-5	1E-4
		W, see ^{182}Ir	-	2E+3	9E-7	3E-9	-	-
		Y, see ^{182}Ir	-	2E+3	8E-7	3E-9	-	-
77	Iridium-195m	D, see ^{182}Ir	8E+3	2E+4	1E-5	3E-8	1E-4	1E-3
		W, see ^{182}Ir	-	3E+4	1E-5	4E-8	-	-
		Y, see ^{182}Ir	-	2E+4	9E-6	3E-8	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
77	Iridium-195	D, see ^{182}Ir	1E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{182}Ir	-	5E+4	2E-5	7E-8	-	-
		Y, see ^{182}Ir	-	4E+4	2E-5	6E-8	-	-
78	Platinum-186	D, all compounds	1E+4	4E+4	2E-5	5E-8	2E-4	2E-3
78	Platinum-188	D, all compounds	2E+3	2E+3	7E-7	2E-9	2E-5	2E-4
78	Platinum-189	D, all compounds	1E+4	3E+4	1E-5	4E-8	1E-4	1E-3
78	Platinum-191	D, all compounds	4E+3	8E+3	4E-6	1E-8	5E-5	5E-4
78	Platinum-193m	D, all compounds	3E+3	6E+3	3E-6	8E-9	-	-
			LLI wall (3E+4)	-	-	-	4E-5	4E-4
78	Platinum-193	D, all compounds	4E+4	2E+4	1E-5	3E-8	-	-
			LLI wall (5E+4)	-	-	-	6E-4	6E-3
78	Platinum-195m	D, all compounds	2E+3	4E+3	2E-6	6E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
78	Platinum-197m[2]	D, all compounds	2E+4	4E+4	2E-5	6E-8	2E-4	2E-3
78	Platinum-197	D, all compounds	3E+3	1E+4	4E-6	1E-8	4E-5	4E-4
78	Platinum-199[2]	D, all compounds	5E+4	1E+5	6E-5	2E-7	7E-4	7E-3
78	Platinum-200	D, all compounds	1E+3	3E+3	1E-6	5E-9	2E-5	2E-4
79	Gold-193	D, all compounds except those given for W and Y	9E+3	3E+4	1E-5	4E-8	1E-4	1E-3
		W, halides and nitrates	-	2E+4	9E-6	3E-8	-	-
		Y, oxides and hydroxides	-	2E+4	8E-6	3E-8	-	-
79	Gold-194	D, see ^{193}Au	3E+3	8E+3	3E-6	1E-8	4E-5	4E-4
		W, see ^{193}Au	-	5E+3	2E-6	8E-9	-	-
		Y, see ^{193}Au	-	5E+3	2E-6	7E-9	-	-
79	Gold-195	D, see ^{193}Au	5E+3	1E+4	5E-6	2E-8	7E-5	7E-4
		W, see ^{193}Au	-	1E+3	6E-7	2E-9	-	-
		Y, see ^{193}Au	-	4E+2	2E-7	6E-10	-	-
79	Gold-198m	D, see ^{193}Au	1E+3	3E+3	1E-6	4E-9	1E-5	1E-4
		W, see ^{193}Au	-	1E+3	5E-7	2E-9	-	-
		Y, see ^{193}Au	-	1E+3	5E-7	2E-9	-	-
79	Gold-198	D, see ^{193}Au	1E+3	4E+3	2E-6	5E-9	2E-5	2E-4
		W, see ^{193}Au	-	2E+3	8E-7	3E-9	-	-
		Y, see ^{193}Au	-	2E+3	7E-7	2E-9	-	-
79	Gold-199	D, see ^{193}Au	3E+3	9E+3	4E-6	1E-8	-	-
			LLI wall (3E+3)	-	-	-	4E-5	4E-4
		W, see ^{193}Au	-	4E+3	2E-6	6E-9	-	-
		Y, see ^{193}Au	-	4E+3	2E-6	5E-9	-	-
79	Gold-200m	D, see ^{193}Au	1E+3	4E+3	1E-6	5E-9	2E-5	2E-4
		W, see ^{193}Au	-	3E+3	1E-6	4E-9	-	-
		Y, see ^{193}Au	-	2E+4	1E-6	3E-9	-	-
79	Gold-200[2]	D, see ^{193}Au	3E+4	6E+4	3E-5	9E-8	4E-4	4E-3
		W, see ^{193}Au	-	8E+4	3E-5	1E-7	-	-
		Y, see ^{193}Au	-	7E+4	3E-5	1E-7	-	-
79	Gold-201[2]	D, see ^{193}Au	7E+4	2E+5	9E-5	3E-7	-	-
			St wall (9E+4)	-	-	-	1E-3	1E-2
		W, see ^{193}Au	-	2E+5	1E-4	3E-7	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		Y, see ^{193}Au	-	2E+5	9E-5	3E-7	-	-
80	Mercury-193m	Vapor	-	8E+3	4E-6	1E-8	-	-
		Organic D	4E+3	1E+4	5E-6	2E-8	6E-5	6E-4
		D, sulfates	3E+3	9E+3	4E-6	1E-8	4E-5	4E-4
		W, oxides, hydroxides, halides, nitrates, and sulfides	-	8E+3	3E-6	1E-8	-	-
80	Mercury-193	Vapor	-	3E+4	1E-5	4E-8	-	-
		Organic D	2E+4	6E+4	3E-5	9E-8	3E-4	3E-3
		D, see ^{193m}Hg	2E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{193m}Hg	-	4E+4	2E-5	6E-8	-	-
80	Mercury-194	Vapor	-	3E+1	1E-8	4E-11	-	-
		Organic D	2E+1	3E+1	1E-8	4E-11	2E-7	2E-6
		D, see ^{193m}Hg	8E+2	4E+1	2E-8	6E-11	1E-5	1E-4
		W, see ^{193m}Hg	-	1E+2	5E-8	2E-10	-	-
80	Mercury-195m	Vapor	-	4E+3	2E-6	6E-9	-	-
		Organic D	3E+3	6E+3	3E-6	8E-9	4E-5	4E-4
		D, see ^{193m}Hg	2E+3	5E+3	2E-6	7E-9	3E-5	3E-4
		W, see ^{193m}Hg	-	4E+3	2E-6	5E-9	-	-
80	Mercury-195	Vapor	-	3E+4	1E-5	4E-8	-	-
		Organic D	2E+4	5E+4	2E-5	6E-8	2E-4	2E-3
		D, see ^{193m}Hg	1E+4	4E+4	1E-5	5E-8	2E-4	2E-3
		W, see ^{193m}Hg	-	3E+4	1E-5	5E-8	-	-
80	Mercury-197m	Vapor	-	5E+3	2E-6	7E-9	-	-
		Organic D	4E+3	9E+3	4E-6	1E-8	5E-5	5E-4
		D, see ^{193m}Hg	3E+3	7E+3	3E-6	1E-8	4E-5	4E-4
		W, see ^{193m}Hg	-	5E+3	2E-6	7E-9	-	-
80	Mercury-197	Vapor	-	8E+3	4E-6	1E-8	-	-
		Organic D	7E+3	1E+4	6E-6	2E-8	9E-5	9E-4
		D, see ^{193m}Hg	6E+3	1E+4	5E-6	2E-8	8E-5	8E-4
		W, see ^{193m}Hg	-	9E+3	4E-6	1E-8	-	-
80	Mercury-199m[2]	Vapor	-	8E+4	3E-5	1E-7	-	-
		Organic D	6E+4	2E+5	7E-5	2E-7	-	-
			St wall (1E+5)	-	-	-	1E-3	1E-2
		D, see ^{193m}Hg	6E+4	1E+5	6E-5	2E-7	8E-4	8E-3
		W, see ^{193m}Hg	-	2E+5	7E-5	2E-7	-	-
80	Mercury-203	Vapor	-	8E+2	4E-7	1E-9	-	-
		Organic D	5E+2	8E+2	3E-7	1E-9	7E-6	7E-5
		D, see ^{193m}Hg	2E+3	1E+3	5E-7	2E-9	3E-5	3E-4
		W, see ^{193m}Hg	-	1E+3	5E-7	2E-9	-	-
81	Thallium-194m[2]	D, all compounds	5E+4	2E+5	6E-5	2E-7	-	-
			St wall (7E+4)	-	-	-	1E-3	1E-2
81	Thallium-194[2]	D, all compounds	3E+5	6E+5	2E-4	8E-7	-	-
			St wall (3E+5)	-	-	-	4E-3	4E-2
81	Thallium-195[2]	D, all compounds	6E+4	1E+5	5E-5	2E-7	9E-4	9E-3
81	Thallium-197	D, all compounds	7E+4	1E+5	5E-5	2E-7	1E-3	1E-2
81	Thallium-198m[2]	D, all compounds	3E+4	5E+4	2E-5	8E-8	4E-4	4E-3
81	Thallium-198	D, all compounds	2E+4	3E+4	1E-5	5E-8	3E-4	3E-3
81	Thallium-199	D, all compounds	6E+4	8E+4	4E-5	1E-7	9E-4	9E-3
81	Thallium-200	D, all compounds	8E+3	1E+4	5E-6	2E-8	1E-4	1E-3
81	Thallium-201	D, all compounds	2E+4	2E+4	9E-6	3E-8	2E-4	2E-3
81	Thallium-202	D, all compounds	4E+3	5E+3	2E-6	7E-9	5E-5	5E-4

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
81	Thallium-204	D, all compounds	2E+3	2E+3	9E-7	3E-9	2E-5	2E-4
82	Lead-195m[2]	D, all compounds	6E+4	2E+5	8E-5	3E-7	8E-4	8E-3
82	Lead-198	D, all compounds	3E+4	6E+4	3E-5	9E-8	4E-4	4E-3
82	Lead-199[2]	D, all compounds	2E+4	7E+4	3E-5	1E-7	3E-4	3E-3
82	Lead-200	D, all compounds	3E+3	6E+3	3E-6	9E-9	4E-5	4E-4
82	Lead-201	D, all compounds	7E+3	2E+4	8E-6	3E-8	1E-4	1E-3
82	Lead-202m	D, all compounds	9E+3	3E+4	1E-5	4E-8	1E-4	1E-3
82	Lead-202	D, all compounds	1E+2	5E+1	2E-8	7E-11	2E-6	2E-5
82	Lead-203	D, all compounds	5E+3	9E+3	4E-6	1E-8	7E-5	7E-4
82	Lead-205	D, all compounds	4E+3	1E+3	6E-7	2E-9	5E-5	5E-4
82	Lead-209	D, all compounds	2E+4	6E+4	2E-5	8E-8	3E-4	3E-3
82	Lead-210	D, all compounds	6E-1	2E-1	1E-10	-	-	-
			Bone surf (1E+0)	Bone surf (4E-1)	-	6E-13	1E-8	1E-7
82	Lead-211[2]	D, all compounds	1E+4	6E+2	3E-7	9E-10	2E-4	2E-3
82	Lead-212	D, all compounds	8E+1	3E+1	1E-8	5E-11	-	-
			Bone surf (1E+2)	-	-	-	2E-6	2E-5
82	Lead-214[2]	D, all compounds	9E+3	8E+2	3E-7	1E-9	1E-4	1E-3
83	Bismuth-200[2]	D, nitrates	3E+4	8E+4	4E-5	1E-7	4E-4	4E-3
		W, all other compounds	-	1E+5	4E-5	1E-7	-	-
83	Bismuth-201[2]	D, see ^{200}Bi	1E+4	3E+4	1E-5	4E-8	2E-4	2E-3
		W, see ^{200}Bi	-	4E+4	2E-5	5E-8	-	-
83	Bismuth-202[2]	D, see ^{200}Bi	1E+4	4E+4	2E-5	6E-8	2E-4	2E-3
		W, see ^{200}Bi	-	8E+4	3E-5	1E-7	-	-
83	Bismuth-203	D, see ^{200}Bi	2E+3	7E+3	3E-6	9E-9	3E-5	3E-4
		W, see ^{200}Bi	-	6E+3	3E-6	9E-9	-	-
83	Bismuth-205	D, see ^{200}Bi	1E+3	3E+3	1E-6	3E-9	2E-5	2E-4
		W, see ^{200}Bi	-	1E+3	5E-7	2E-9	-	-
83	Bismuth-206	D, see ^{200}Bi	6E+2	1E+3	6E-7	2E-9	9E-6	9E-5
		W, see ^{200}Bi	-	9E+2	4E-7	1E-9	-	-
83	Bismuth-207	D, see ^{200}Bi	1E+3	2E+3	7E-7	2E-9	1E-5	1E-4
		W, see ^{200}Bi	-	4E+2	1E-7	5E-10	-	-
83	Bismuth-210m	D, see ^{200}Bi	4E+1	5E+0	2E-9	-	-	-
			Kidneys (6E+1)	Kidneys (6E+0)	-	9E-12	8E-7	8E-6
		W, see ^{200}Bi	-	7E-1	3E-10	9E-13	-	-
83	Bismuth-210	D, see ^{200}Bi	8E+2	2E+2	1E-7	-	1E-5	1E-4
			-	Kidneys (4E+2)	-	5E-10	-	-
		W, see ^{200}Bi	-	3E+1	1E-8	4E-11	-	-
83	Bismuth-212[2]	D, see ^{200}Bi	5E+3	2E+2	1E-7	3E-10	7E-5	7E-4
		W, see ^{200}Bi	-	3E+2	1E-7	4E-10	-	-
83	Bismuth-213[2]	D, see ^{200}Bi	7E+3	3E+2	1E-7	4E-10	1E-4	1E-3
		W, see ^{200}Bi	-	4E+2	1E-7	5E-10	-	-
83	Bismuth-214[2]	D, see ^{200}Bi	2E+4	8E+2	3E-7	1E-9	-	-
			St wall (2E+4)	-	-	-	3E-4	3E-3
		W, see ^{200}Bi	-	9E-2	4E-7	1E-9	-	-
84	Polonium-203[2]	D, all compounds except those given for W	3E+4	6E+4	3E-5	9E-8	3E-4	3E-3

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, oxides, hydroxides, and nitrates	-	9E+4	4E-5	1E-7	-	-
84	Polonium-205[2]	D, see ^{203}Po	2E+4	4E+4	2E-5	5E-8	3E-4	3E-3
		W, see ^{203}Po	-	7E+4	3E-5	1E-7	-	-
84	Polonium-207	D, see ^{203}Po	8E+3	3E+4	1E-5	3E-8	1E-4	1E-3
		W, see ^{203}Po	-	3E+4	1E-5	4E-8	-	-
84	Polonium-210	D, see ^{203}Po	3E+0	6E-1	3E-10	9E-13	4E-8	4E-7
		W, see ^{203}Po	-	6E-1	3E-10	9E-13	-	-
85	Astatine-207[2]	D, halides	6E+3	3E+3	1E-6	4E-9	8E-5	8E-4
		W	-	2E+3	9E-7	3E-9	-	-
85	Astatine-211	D, halides	1E+2	8E+1	3E-8	1E-10	2E-6	2E-5
		W	-	5E+1	2E-8	8E-11	-	-
86	Radon-220	With daughters removed	-	2E+4	7E-6	2E-8	-	-
		With daughters present	-	2E+1	9E-9	3E-11	-	-
				(or 12 working level months)	(or 1.0 working level)			
86	Radon-222	With daughters removed	-	1E+4	4E-6	1E-8	-	-
		With daughters present	-	1E+2	3E-8	1E-10	-	-
				(or 4 working level months)	(or 0.33 working level)			
87	Francium-222[2]	D, all compounds	2E+3	5E+2	2E-7	6E-10	3E-5	3E-4
87	Francium-223[2]	D, all compounds	6E+2	8E+2	3E-7	1E-9	8E-6	8E-5
88	Radium-223	W, all compounds	5E+0	7E-1	3E-10	9E-13	-	-
			Bone surf (9E+0)	-	-	-	1E-7	1E-6
88	Radium-224	W, all compounds	8E+0	2E+0	7E-10	2E-12	-	-
			Bone surf (2E+1)	-	-	-	2E-7	2E-6
88	Radium-225	W, all compounds	8E+0	7E-1	3E-10	9E-13	-	-
			Bone surf (2E+1)	-	-	-	2E-7	2E-6
88	Radium-226	W, all compounds	2E+0	6E-1	3E-10	9E-13	-	-
			Bone surf (5E+0)	-	-	-	6E-8	6E-7
88	Radium-227[2]	W, all compounds	2E+4	1E+4	6E-6	-	-	-
			Bone surf (2E+4)	Bone surf (2E+4)	-	3E-8	3E-4	3E-3
88	Radium-228	W, all compounds	2E+0	1E+0	5E-10	2E-12	-	-
			Bone surf (4E+0)	-	-	-	6E-8	6E-7
89	Actinium-224	D, all compounds except those given for W and Y	2E+3	3E+1	1E-8	-	-	-
			LLI wall (2E+3)	Bone surf (4E+1)	-	5E-11	3E-5	3E-4
		W, halides and nitrates	-	5E+1	2E-8	7E-11	-	-
		Y, oxides and hydroxides	-	5E+1	2E-8	6E-11	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
89	Actinium-225	D, see ^{224}Ac	5E+1	3E-1	1E-10	-	-	-
			LLI wall (5E+1)	Bone surf (5E-1)	-	7E-13	7E-7	7E-6
		W, see ^{224}Ac	-	6E-1	3E-10	9E-13	-	-
		Y, see ^{224}Ac	-	6E-1	3E-10	9E-13	-	-
89	Actinium-226	D, see ^{224}Ac	1E+2	3E+0	1E-9	-	-	-
			LLI wall (1E+2)	Bone surf (4E+0)	-	5E-12	2E-6	2E-5
		W, see ^{224}Ac	-	5E+0	2E-9	7E-12	-	-
		Y, see ^{224}Ac	-	5E+0	2E-9	6E-12	-	-
89	Actinium-227	D, see ^{224}Ac	2E-1	4E-4	2E-13	-	-	-
			Bone surf (4E-1)	Bone surf (8E-4)	-	1E-15	5E-9	5E-8
		W, see ^{224}Ac	-	2E-3	7E-13	-	-	-
			-	Bone surf (3E-3)	-	4E-15	-	-
		Y, see ^{224}Ac	-	4E-3	2E-12	6E-15	-	-
89	Actinium-228	D, see ^{224}Ac	2E+3	9E+0	4E-9	-	3E-5	3E-4
			-	Bone surf (2E+1)	-	2E-11	-	-
		W, see ^{224}Ac	-	4E+1	2E-8	-	-	-
			-	Bone surf (6E+1)	-	8E-11	-	-
		Y, see ^{224}Ac	-	4E+1	2E-8	6E-11	-	-
90	Thorium-226[2]	W, all compounds except those given for Y	5E+3	2E+2	6E-8	2E-10	-	-
			St wall (5E+3)	-	-	-	7E-5	7E-4
		Y, oxides and hydroxides	-	1E+2	6E-8	2E-10	-	-
90	Thorium-227	W, see ^{226}Th	1E+2	3E-1	1E-10	5E-13	2E-6	2E-5
		Y, see ^{226}Th	-	3E-1	1E-10	5E-13	-	-
90	Thorium-228	W, see ^{226}Th	6E+0	1E-2	4E-12	-	-	-
			Bone surf (1E+1)	Bone surf (2E-2)	-	3E-14	2E-7	2E-6
		Y, see ^{226}Th	-	2E-2	7E-12	2E-14	-	-
90	Thorium-229	W, see ^{226}Th	6E-1	9E-4	4E-13	-	-	-
			Bone surf (1E+0)	Bone surf (2E-3)	-	3E-15	2E-8	2E-7
		Y, see ^{226}Th	-	2E-3	1E-12	-	-	-
			-	Bone surf (3E-3)	-	4E-15	-	-
90	Thorium-230	W, see ^{226}Th	4E+0	6E-3	3E-12	-	-	-
			Bone surf (9E+0)	Bone surf (2E-2)	-	2E-14	1E-7	1E-6
		Y, see ^{226}Th	-	2E-2	6E-12	-	-	-
			-	Bone surf (2E-2)	-	3E-14	-	-
90	Thorium-231	W, see ^{226}Th	4E+3	6E+3	3E-6	9E-9	5E-5	5E-4
		Y, see ^{226}Th	-	6E+3	3E-6	9E-9	-	-
90	Thorium-232	W, see ^{226}Th	7E-1	1E-3	5E-13	-	-	-
			Bone surf (2E+0)	Bone surf (3E-3)	-	4E-15	3E-8	3E-7
		Y, see ^{226}Th	-	3E-3	1E-12	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			-	Bone surf (4E-3)	-	6E-15	-	-
90	Thorium-234	W, see ^{226}Th	3E+2	2E+2	8E-8	3E-10	-	-
			LLI wall (4E+2)	-	-	-	5E-6	5E-5
		Y, see ^{226}Th	-	2E+2	6E-8	2E-10	-	-
91	Protactinium-227[2]	W, all compounds except those given for Y	4E+3	1E+2	5E-8	2E-10	5E-5	5E-4
		Y, oxides and hydroxides	-	1E+2	4E-8	1E-10	-	-
91	Protactinium-228	W, see ^{227}Pa	1E+3	1E+1	5E-9	-	2E-5	2E-4
			-	Bone surf (2E+1)	-	3E-11	-	-
		Y, see ^{227}Pa	-	1E+1	5E-9	2E-11	-	-
91	Protactinium-230	W, see ^{227}Pa	6E+2	5E+0	2E-9	7E-12	-	-
			Bone surf (9E+2)	-	-	-	1E-5	1E-4
		Y, see ^{227}Pa	-	4E+0	1E-9	5E-12	-	-
91	Protactinium-231	W, see ^{227}Pa	2E-1	2E-3	6E-13	-	-	-
			Bone surf (5E-1)	Bone surf (4E-3)	-	6E-15	6E-9	6E-8
		Y, see ^{227}Pa	-	4E-3	2E-12	-	-	-
			-	Bone surf (6E-3)	-	8E-15	-	-
91	Protactinium-232	W, see ^{227}Pa	1E+3	2E+1	9E-9	-	2E-5	2E-4
			-	Bone surf (6E+1)	-	8E-11	-	-
		Y, see ^{227}Pa	-	6E+1	2E-8	-	-	-
			-	Bone surf (7E+1)	-	1E-10	-	-
91	Protactinium-233	W, see ^{227}Pa	1E+3	7E+2	3E-7	1E-9	-	-
			LLI wall (2E+3)	-	-	-	2E-5	2E-4
		Y, see ^{227}Pa	-	6E+2	2E-7	8E-10	-	-
91	Protactinium-234	W, see ^{227}Pa	2E+3	8E+3	3E-6	1E-8	3E-5	3E-4
		Y, see ^{227}Pa	-	7E+3	3E-6	9E-9	-	-
92	Uranium-230	D, UF_6, UO_2F_2, $UO_2(NO_3)_2$	4E+0	4E-1	2E-10	-	-	-
			Bone surf (6E+0)	Bone surf (6E-1)	-	8E-13	8E-8	8E-7
		W, UO_3, UF_4, UCl_4	-	4E-1	1E-10	5E-13	-	-
		Y, UO_2, U_3O_8	-	3E-1	1E-10	4E-13	-	-
92	Uranium-231	D, see ^{230}U	5E+3	8E+3	3E-6	1E-8	-	-
			LLI wall (4E+3)	-	-	-	6E-5	6E-4
		W, see ^{230}U	-	6E+3	2E-6	8E-9	-	-
		Y, see ^{230}U	-	5E+3	2E-6	6E-9	-	-
92	Uranium-232	D, see ^{230}U	2E+0	2E-1	9E-11	-	-	-
			Bone surf (4E+0)	Bone surf (4E-1)	-	6E-13	6E-8	6E-7
		W, see ^{230}U	-	4E-1	2E-10	5E-13	-	-
		Y, see ^{230}U	-	8E-3	3E-12	1E-14	-	-
92	Uranium-233	D, see ^{230}U	1E+1	1E+0	5E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
		W, see ^{230}U	-	7E-1	3E-10	1E-12	-	-
		Y, see ^{230}U	-	4E-2	2E-11	5E-14	-	-
92	Uranium-234[3]	D, see ^{230}U	1E+1	1E+0	5E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6
		W, see ^{230}U	-	7E-1	3E-10	1E-12	-	-
		Y, see ^{230}U	-	4E-2	2E-11	5E-14	-	-
92	Uranium-235[3]	D, see ^{230}U	1E+1	1E+0	6E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6
		W, see ^{230}U	-	8E-1	3E-10	1E-12	-	-
		Y, see ^{230}U	-	4E-2	2E-11	6E-14	-	-
92	Uranium-236	D, see ^{230}U	1E+1	1E+0	5E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6
		W, see ^{230}U	-	8E-1	3E-10	1E-12	-	-
		Y, see ^{230}U	-	4E-2	2E-11	6E-14	-	-
92	Uranium-237	D, see ^{230}U	2E+3	3E+3	1E-6	4E-9	-	-
			LLI wall (2E+3)	-	-	-	3E-5	3E-4
		W, see ^{230}U	-	2E+3	7E-7	2E-9	-	-
		Y, see ^{230}U	-	2E+3	6E-7	2E-9	-	-
92	Uranium-238[3]	D, see ^{230}U	1E+1	1E+0	6E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6
		W, see ^{230}U	-	8E-1	3E-10	1E-12	-	-
		Y, see ^{230}U	-	4E-2	2E-11	6E-14	-	-
92	Uranium-239[2]	D, see ^{230}U	7E+4	2E+5	8E-5	3E-7	9E-4	9E-3
		W, see ^{230}U	-	2E+5	7E-5	2E-7	-	-
		Y, see ^{230}U	-	2E+5	6E-5	2E-7	-	-
92	Uranium-240	D, see ^{230}U	1E+3	4E+3	2E-6	5E-9	2E-5	2E-4
		W, see ^{230}U	-	3E+3	1E-6	4E-9	-	-
		Y, see ^{230}U	-	2E+3	1E-6	3E-9	-	-
92	Uranium-natural[3]	D, see ^{230}U	1E+1	1E+0	5E-10	-	-	-
			Bone surf (2E+1)	Bone surf (2E+0)	-	3E-12	3E-7	3E-6
		W, see ^{230}U	-	8E-1	3E-10	9E-13	-	-
		Y, see ^{230}U	-	5E-2	2E-11	9E-14	-	-
93	Neptunium-232[2]	W, all compounds	1E+5	2E+3	7E-7	-	2E-3	2E-2
			-	Bone surf (5E+2)	-	6E-9	-	-
93	Neptunium-233[2]	W, all compounds	8E+5	3E+6	1E-3	4E-6	1E-2	1E-1
93	Neptunium-234	W, all compounds	2E+3	3E+3	1E-6	4E-9	3E-5	3E-4
93	Neptunium-235	W, all compounds	2E+4	8E+2	3E-7	-	-	-
			LLI wall (2E+4)	Bone surf (1E+3)	-	2E-9	3E-4	3E-3
93	Neptunium-236 (1.15E+5 y)	W, all compounds	3E+0	2E-2	9E-12	-	-	-
			Bone surf (6E+0)	Bone surf (5E-2)	-	8E-14	9E-8	9E-7
93	Neptunium-236m (22.5 h)	W, all compounds	3E+3	3E+1	1E-8	-	-	-
			Bone surf (4E+3)	Bone surf (7E+1)	-	1E-10	5E-5	5E-4

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
93	Neptunium-237	W, all compounds	5E-1	4E-3	2E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	1E-14	2E-8	2E-7
93	Neptunium-238	W, all compounds	1E+3	6E+1	3E-8	-	2E-5	2E-4
			-	Bone surf (2E+2)	-	2E-10	-	-
93	Neptunium-239	W, all compounds	2E+3	2E+3	9E-7	3E-9	-	-
			LLI wall (2E+3)	-	-	-	2E-5	2E-4
93	Neptunium-240[2]	W, all compounds	2E+4	8E+4	3E-5	1E-7	3E-4	3E-3
94	Plutonium-234	W, all compounds except PuO_2	8E+3	2E+2	9E-8	3E-10	1E-4	1E-3
		Y, PuO_2	-	2E+2	8E-8	3E-10	-	-
94	Plutonium-235[2]	W, see ^{234}Pu	9E+5	3E+6	1E-3	4E-6	1E-2	1E-1
		Y, see ^{234}Pu	-	3E+6	1E-3	3E-6	-	-
94	Plutonium-236	W, see ^{234}Pu	2E+0	2E-2	8E-12	-	-	-
			Bone surf (4E+0)	Bone surf (4E-2)	-	5E-14	6E-8	6E-7
		Y, see ^{234}Pu	-	4E-2	2E-11	6E-14	-	-
94	Plutonium-237	W, see ^{234}Pu	1E+4	3E+3	1E-6	5E-9	2E-4	2E-3
		Y, see ^{234}Pu	-	3E+3	1E-6	4E-9	-	-
94	Plutonium-238	W, see ^{234}Pu	9E-1	7E-3	3E-12	-	-	-
			Bone surf (2E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
		Y, see ^{234}Pu	-	2E-2	8E-12	2E-14	-	-
94	Plutonium-239	W, see ^{234}Pu	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
		Y, see ^{234}Pu	-	2E-2	7E-12	-	-	-
			-	Bone surf (2E-2)	-	2E-14	-	-
94	Plutonium-240	W, see ^{234}Pu	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
		Y, see ^{234}Pu	-	2E-2	7E-12	-	-	-
			-	Bone surf (2E-2)	-	2E-14	-	-
94	Plutonium-241	W, see ^{234}Pu	4E+1	3E-1	1E-10	-	-	-
			Bone surf (7E+1)	Bone surf (6E-1)	-	8E-13	1E-6	1E-5
		Y, see ^{234}Pu	-	8E-1	3E-10	-	-	-
			-	Bone surf (1E+0)	-	1E-12	-	-
94	Plutonium-242	W, see ^{234}Pu	8E-1	7E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
		Y, see ^{234}Pu	-	2E-2	7E-12	-	-	-
			-	Bone surf (2E-2)	-	2E-14	-	-
94	Plutonium-243	W, see ^{234}Pu	2E+4	4E+4	2E-5	5E-8	2E-4	2E-3
		Y, see ^{234}Pu	-	4E+4	2E-5	5E-8	-	-
94	Plutonium-244	W, see ^{234}Pu	8E-1	7E-3	3E-12	-	-	-
			Bone surf (2E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
		Y, see ^{234}Pu	-	2E-2	7E-12	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			-	Bone surf (2E-2)	-	2E-14	-	-
94	Plutonium-245	W, see ^{234}Pu	2E+3	5E+3	2E-6	6E-9	3E-5	3E-4
		Y, see ^{234}Pu	-	4E+3	2E-6	6E-9	-	-
94	Plutonium-246	W, see ^{234}Pu	4E+2	3E+2	1E-7	4E-10	-	-
			LLI wall (4E+2)	-	-	-	6E-6	6E-5
		Y, see ^{234}Pu	-	3E+2	1E-7	4E-10	-	-
95	Americium-237[2]	W, all compounds	8E+4	3E+5	1E-4	4E-7	1E-3	1E-2
95	Americium-238[2]	W, all compounds	4E+4	3E+3	1E-6	-	5E-4	5E-3
			-	Bone surf (6E+3)	-	9E-9	-	-
95	Americium-239	W, all compounds	5E+3	1E+4	5E-6	2E-8	7E-5	7E-4
95	Americium-240	W, all compounds	2E+3	3E+3	1E-6	4E-9	3E-5	3E-4
95	Americium-241	W, all compounds	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
95	Americium-242m	W, all compounds	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
95	Americium-242	W, all compounds	4E+3	8E+1	4E-8	-	5E-5	5E-4
			-	Bone surf (9E+1)	-	1E-10	-	-
95	Americium-243	W, all compounds	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
95	Americium-244m[2]	W, all compounds	6E+4	4E+3	2E-6	-	-	-
			St wall (8E+4)	Bone surf (7E+3)	-	1E-8	1E-3	1E-2
95	Americium-244	W, all compounds	3E+3	2E+2	8E-8	-	4E-5	4E-4
			-	Bone surf (3E+2)	-	4E-10	-	-
95	Americium-245	W, all compounds	3E+4	8E+4	3E-5	1E-7	4E-4	4E-3
95	Americium-246m[2]	W, all compounds	5E+4	2E+5	8E-5	3E-7	-	-
			St wall (6E+4)	-	-	-	8E-4	8E-3
95	Americium-246[2]	W, all compounds	3E+4	1E+5	4E-5	1E-7	4E-4	4E-3
96	Curium-238	W, all compounds	2E+4	1E+3	5E-7	2E-9	2E-4	2E-3
96	Curium-240	W, all compounds	6E+1	6E-1	2E-10	-	-	-
			Bone surf (8E+1)	Bone surf (6E-1)	-	9E-13	1E-6	1E-5
96	Curium-241	W, all compounds	1E+3	3E+1	1E-8	-	2E-5	2E-4
			-	Bone surf (4E+1)	-	5E-11	-	-
96	Curium-242	W, all compounds	3E+1	3E-1	1E-10	-	-	-
			Bone surf (5E+1)	Bone surf (3E-1)	-	4E-13	7E-7	7E-6
96	Curium-243	W, all compounds	1E+0	9E-3	4E-12	-	-	-
			Bone surf (2E+0)	Bone surf (2E-2)	-	2E-14	3E-8	3E-7
96	Curium-244	W, all compounds	1E+0	1E-2	5E-12	-	-	-
			Bone surf (3E+0)	Bone surf (2E-2)	-	3E-14	3E-8	3E-7
96	Curium-245	W, all compounds	7E-1	6E-3	3E-12	-	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
96	Curium-246	W, all compounds	7E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
96	Curium-247	W, all compounds	8E-1	6E-3	3E-12	-	-	-
			Bone surf (1E+0)	Bone surf (1E-2)	-	2E-14	2E-8	2E-7
96	Curium-248	W, all compounds	2E-1	2E-3	7E-13	-	-	-
			Bone surf (4E-1)	Bone surf (3E-3)	-	4E-15	5E-9	5E-8
96	Curium-249[2]	W, all compounds	5E+4	2E+4	7E-6	-	7E-4	7E-3
			-	Bone surf (3E+4)	-	4E-8	-	-
96	Curium-250	W, all compounds	4E-2	3E-4	1E-13	-	-	-
			Bone surf (6E-2)	Bone surf (5E-4)	-	8E-16	9E-10	9E-9
97	Berkelium-245	W, all compounds	2E+3	1E+3	5E-7	2E-9	3E-5	3E-4
97	Berkelium-246	W, all compounds	3E+3	3E+3	1E-6	4E-9	4E-5	4E-4
97	Berkelium-247	W, all compounds	5E-1	4E-3	2E-12	-	-	-
			Bone surf (1E+0)	Bone surf (9E-3)	-	1E-14	2E-8	2E-7
97	Berkelium-249	W, all compounds	2E+2	2E+0	7E-10	-	-	-
			Bone surf (5E+2)	Bone surf (4E+0)	-	5E-12	6E-6	6E-5
97	Berkelium-250	W, all compounds	9E+3	3E+2	1E-7	-	1E-4	1E-3
			-	Bone surf (7E+2)	-	1E-9	-	-
98	Californium-244[2]	W, all compounds except those given for Y	3E+4	6E+2	2E-7	8E-10	-	-
			St wall (3E+4)	-	-	-	4E-4	4E-3
		Y, oxides and hydroxides	-	6E+2	2E-7	8E-10	-	-
98	Californium-246	W, see ^{244}Cf	4E+2	9E+0	4E-9	1E-11	5E-6	5E-5
		Y, see ^{244}Cf	-	9E+0	4E-9	1E-11	-	-
98	Californium-248	W, see ^{244}Cf	8E+0	6E-2	3E-11	-	-	-
			Bone surf (2E+1)	Bone surf (1E-1)	-	2E-13	2E-7	2E-6
		Y, see ^{244}Cf	-	1E-1	4E-11	1E-13	-	-
98	Californium-249	W, see ^{244}Cf	5E-1	4E-3	2E-12	-	-	-
			Bone surf (1E+0)	Bone surf (9E-3)	-	1E-14	2E-8	2E-7
		Y, see ^{244}Cf	-	1E-2	4E-12	-	-	-
				Bone surf				
			-	(1E-2)	-	2E-14	-	-
98	Californium-250	W, see ^{244}Cf	1E+0	9E-3	4E-12	-	-	-
			Bone surf (2E+0)	Bone surf (2E-2)	-	3E-14	3E-8	3E-7
		Y, see ^{244}Cf	-	3E-2	1E-11	4E-14	-	-
98	Californium-251	W, see ^{244}Cf	5E-1	4E-3	2E-12	-	-	-
			Bone surf (1E+0)	Bone surf (9E-3)	-	1E-14	2E-8	2E-7
		Y, see ^{244}Cf	-	1E-2	4E-12	-	-	-
			-	Bone surf (1E-2)	-	2E-14	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
98	Californium-252	W, see ^{244}Cf	2E+0	2E-2	8E-12	-	-	-
			Bone surf (5E+0)	Bone surf (4E-2)	-	5E-14	7E-8	7E-7
		Y, see ^{244}Cf	-	3E-2	1E-11	5E-14	-	-
98	Californium-253	W, see ^{244}Cf	2E+2	2E+0	8E-10	3E-12	-	-
			Bone surf (4E+2)	-	-	-	5E-6	5E-5
		Y, see ^{244}Cf	-	2E+0	7E-10	2E-12	-	-
98	Californium-254	W, see ^{244}Cf	2E+0	2E-2	9E-12	3E-14	3E-8	3E-7
		Y, see ^{244}Cf	-	2E-2	7E-12	2E-14	-	-
99	Einsteinium-250	W, all compounds	4E+4	5E+2	2E-7	-	6E-4	6E-3
			-	Bone surf (1E+3)	-	2E-9	-	-
99	Einsteinium-251	W, all compounds	7E+3	9E+2	4E-7	-	1E-4	1E-3
			-	Bone surf (1E+3)	-	2E-9	-	-
99	Einsteinium-253	W, all compounds	2E+2	1E+0	6E-10	2E-12	2E-6	2E-5
99	Einsteinium-254m	W, all compounds	3E+2	1E+1	4E-9	1E-11	-	-
			LLI wall (3E+2)	-	-	-	4E-6	4E-5
99	Einsteinium-254	W, all compounds	8E+0	7E-2	3E-11	-	-	-
			Bone surf (2E+1)	Bone surf (1E-1)	-	2E-13	2E-7	2E-6
100	Fermium-252	W, all compounds	5E+2	1E+1	5E-9	2E-11	6E-6	6E-5
100	Fermium-253	W, all compounds	1E+3	1E+1	4E-9	1E-11	1E-5	1E-4
100	Fermium-254	W, all compounds	3E+3	9E+1	4E-8	1E-10	4E-5	4E-4
100	Fermium-255	W, all compounds	5E+2	2E+1	9E-9	3E-11	7E-6	7E-5
100	Fermium-257	W, all compounds	2E+1	2E-1	7E-11	-	-	-
			Bone surf (4E+1)	Bone surf (2E-1)	-	3E-13	5E-7	5E-6
101	Mendelevium-257	W, all compounds	7E+3	8E+1	4E-8	-	1E-4	1E-3
			-	Bone surf (9E+1)	-	1E-10	-	-
101	Mendelevium-258	W, all compounds	3E+1	2E-1	1E-10	-	-	-
			Bone surf (5E+1)	Bone surf (3E-1)	-	5E-13	6E-7	6E-6
-	Any single radionuclide not listed above with decay mode other than alpha emission or spontaneous fission and with radioactive half-life less than 2 hours	Submersion[1]	-	2E+2	1E-7	1E-9	-	-

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	Inhalation ALI (μCi)	Inhalation DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
-	Any single radionuclide not listed above with decay mode other than alpha emission or spontaneous fission and with radioactive half-life greater than 2 hours		-	2E-1	1E-10	1E-12	1E-8	1E-7
-	Any single radionuclide not listed above that decays by alpha emission or spontaneous fission, or any mixture for which either the identity or the concentration of any radionuclide in the mixture is not known	-	-	4E-4	2E-13	1E-15	2E-9	2E-8

ENDNOTES:

[1] "Submersion" means that values given are for submersion in a hemispherical semi-infinite cloud of airborne material.

[2] These radionuclides have radiological half-lives of less than 2 hours. The total effective dose equivalent received during operations with these radionuclides might include a significant contribution from external exposure. The DAC values for all radionuclides, other than those designated Class "Submersion," are based upon the committed effective dose equivalent due to the intake of the radionuclide into the body and do not include potentially significant contributions to dose equivalent from external exposures. The licensee may substitute 1E-7 μCi/ml for the listed DAC to account for the submersion dose prospectively, but should use individual monitoring devices or other radiation measuring instruments that measure external exposure to demonstrate compliance with the limits. (See 1200-2-5-.52)

[3] For soluble mixtures of U-238, U-234, and U-235 in air, chemical toxicity may be the limiting factor (see 1200-2-5-.50(4)). If the percent by weight (enrichment) of U-235 is not greater than 5, the concentration value for a 40-hour workweek is 0.2 milligrams uranium per cubic meter of air average. For any enrichment, the product of the average concentration and time of exposure during a 40-hour workweek shall not exceed 8E-3 (SA) μCi-hr/ml, where SA is the specific activity of the uranium inhaled. The specific activity for natural uranium is 6.77E-7 curies per gram U. The specific activity for other mixtures of U-238, U-235, and U-234, if not known, shall be:

$$SA = 3.6E\text{-}7 \text{ curies/gram U} \quad \text{U-depleted}$$

$$SA = [0.4 + 0.38 \text{ (enrichment)} + 0.0034 \text{ (enrichment)}^2] \text{ E} - 6, \qquad \text{enrichment} \geq 0.72$$

where enrichment is the percentage by weight of U-235, expressed as percent.

NOTE:

1. If the identity of each radionuclide in a mixture is known but the concentration of one or more of the radionuclides in the mixture is not known, the DAC for the mixture shall be the most restrictive DAC of any radionuclide in the mixture.

2. If the identity of each radionuclide in the mixture is not known, but it is known that certain radionuclides specified in this schedule are not present in the mixture, the inhalation ALI, DAC, and effluent and sewage concentrations for the mixture are the lowest values specified in this schedule for any radionuclide that is not known to be absent from the mixture; or

Atomic No.	Radionuclide	Class	Table I Occupational Values			Table II Effluent Concentrations		Table III Releases to Sewers
			Col. 1	*Col. 2*	*Col. 3*	*Col. 1*	*Col. 2*	
			Inhalation					*Monthly Average Concentration (μCi/ml)*
			Oral Ingestion ALI (μCi)	ALI (μCi)	DAC (μCi/ml)	Air (μCi/ml)	Water (μCi/ml)	
If it is known that Ac-227-D and Cm-250-W are not present			-	7E-4	3E-13	-	-	-
If, in addition, it is known that Ac-227-W, Y, Th-229-W, Y, Th-230-W, Th-232-W, Y, Pa-231-W, Y, Np-237-W, Pu-239-W, Pu-240-W, Pu-242-W, Am-241-W, Am-242m-W, Am-243-W, Cm-245-W, Cm-246-W, Cm-247-W, Cm-248-W, Bk-247-W, Cf-249-W, and Cf-251-W are not present			-	7E-3	3E-12	-	-	-
If, in addition, it is known that Sm-146-W, Sm-147-W, Gd-148-D, W, Gd-152-D, W, Th-228-W, Y, Th-230-Y, U-232-Y, U-233-Y, U-234-Y, U-235-Y, U-236-Y, U-238-Y, Np-236-W, Pu-236-W, Y, Pu-238-W, Y, Pu-239-Y, Pu-240-Y, Pu-242-Y, Pu-244-W, Y, Cm-243-W, Cm-244-W, Cf-248-W, Cf-249-Y, Cf-250-W, Y, Cf-251-Y, Cf-252-W, Y, and Cf-254-W, Y are not present			-	7E-2	3E-11	-	-	-
If, in addition, it is known that Pb-210-D, Bi-210m-W, Po-210-D, W, Ra-223-W, Ra-225-W, Ra-226-W, Ac-225-D, W, Y, Th-227-W, Y, U-230-D, W, Y, U-232-D, W, Pu-241-W, Cm-240-W, Cm-242-W, Cf-248-Y, Es-254-W, Fm-257-W, and Md-258-W are not present			-	7E-1	3E-10	-	-	-
If, in addition, it is known that Si-32-Y, Ti-44-Y, Fe-60-D, Sr-90-Y, Zr-93-D, Cd-113m-D, Cd-113-D, In-115-D, W, La-138-D, Lu-176-W, Hf-178m-D, W, Hf-182-D, W, Bi-210m-D, Ra-224-W, Ra-228-W, Ac-226-D, W, Y, Pa-230-W, Y, U-233-D, W, U-234-D, W, U-235-D, W, U-236-D, W, U-238-D, W, Pu-241-Y, Bk-249-W, Cf-253-W, Y, and Es-253-W are not present			-	7E+0	3E-9	-	-	-
If it is known that Ac-227-D, W, Y, Th-229-W, Y, Th-232-W, Y, Pa-231-W, Y, Cm-248-W, and Cm-250-W are not present			-	-	-	1E-14	-	-
If, in addition, it is known that Sm-146-W, Gd-148-D, W, Gd-152-D, Th-228-W, Y, Th-230-W, Y, U-232-Y, U-233-Y, U-234-Y, U-235-Y, U-236-Y, U-238-Y, U-Nat-Y, Np-236-W, Np-237-W, Pu-236-W, Y, Pu-238-W, Y, Pu-239-W, Y, Pu-240-W, Y, Pu-242-W, Y, Pu-244-W, Y, Am-241-W, Am-242m-W, Am-243-W, Cm-243-W, Cm-244-W, Cm-245-W, Cm-246-W, Cm-247-W, Bk-247-W, Cf-249-W, Y, Cf-250-W, Y, Cf-251-W, Y, Cf-252-W, Y, and Cf-254-W, Y are not present			-	-	-	1E-13	-	-
If, in addition, it is known that Sm-147-W, Gd-152-W, Pb-210-D, Bi-210m-W, Po-210-D, W, Ra-223-W, Ra-225-W, Ra-226-W, Ac-225-D, W, Y, Th-227-W, Y, U-230-D, W, Y, U-232-D, W, U-Nat-W, Pu-241-W, Cm-240-W, Cm-242-W, Cf-248-W, Y, Es-254-W, Fm-257-W, and Md-258-W are not present			-	-	-	1E-12	-	-
If, in addition it is known that Fe-60, Sr-90, Cd-113m, Cd-113, In-115, I-129, Cs-134, Sm-145, Sm-147, Gd-148, Gd-152, Hg-194 (organic), Bi-210m, Ra-223, Ra-224, Ra-225, Ac-225, Th-228, Th-230, U-233, U-234, U-235, U-236, U-238, U-Nat, Cm-242, Cf-248, Es-254, Fm-257, and Md-258 are not present			-	-	-	-	1E-6	1E-5

3. If a mixture of radionuclides consists of uranium and its daughters in ore dust (10 μm AMAD particle distribution assumed) prior to chemical separation of the uranium from the ore, the following values may be used for the DAC of the mixture: 6E-

11 μCi of gross alpha activity from uranium-238, uranium-234, thorium-230, and radium-226 per milliliter of air; 3E-11 μCi of natural uranium per milliliter of air; or 45 micrograms of natural uranium per cubic meter of air.

4. If the identity and concentration of each radionuclide in a mixture are known, the limiting values should be derived as follows: determine, for each radionuclide in the mixture, the ratio between the concentration present in the mixture and the concentration otherwise established in Schedule RHS 8–30 for the specific radionuclide when not in a mixture. The sum of such ratios for all of the radionuclides in the mixture may not exceed "1" (i.e., "unity").

Example: If radionuclides "A," "B," and "C" are present in concentrations C_A, C_B, and C_C, and if the applicable DACs are DAC_A, DAC_B, and DAC_C, respectively, then the concentrations shall be limited so that the following relationship exists:

$$\frac{C_A}{DAC_A} + \frac{C_B}{DAC_B} + \frac{C_C}{DAC_C} \leq 1$$

Authority: T.C.A. §§4–5–201 et seq., 68–202–201.

SCHEDULE RHS 8–31

QUANTITIES [a] OF LICENSED MATERIAL REQUIRING LABELING

Radionuclide	Quantity (μCi) [*a]	Radionuclide	Quantity (μCi) [*a]
Hydrogen–3	1,000	Manganese–54	100
Beryllium–7	1,000	Manganese–56	1,000
Beryllium–10	1	Iron–52	100
Carbon–11	1,000	Iron–55	100
Carbon–14	1,000	Iron–59	10
Fluorine–18	1,000	Iron–60	1
Sodium–22	10	Cobalt–55	100
Sodium–24	100	Cobalt–56	10
Magnesium–28	100	Cobalt–57	100
Aluminum–26	10	Cobalt–58m	1,000
Silicon–31	1,000	Cobalt–58	100
Silicon–32	1	Cobalt–60m	1,000
Phosphorus–32	10	Cobalt–60	1
Phosphorus–33	100	Cobalt–61	1,000
Sulfur–35	100	Cobalt–62m	100
Chlorine–36	10	Nickel–56	100
Chlorine–38	1,000	Nickel–57	100
Chlorine–39	1,000	Nickel–59	100
Argon–39	1,000	Nickel–63	100
Argon–41	1,000	Nickel–65	1,000
Potassium–40	100	Nickel–66	10
Potassium–42	1,000	Copper–60	1,000
Potassium–43	1,000	Copper–61	1,000
Potassium–44	1,000	Copper–64	1,000
Potassium–45	1,000	Copper–67	1,000
Calcium–41	100	Zinc–62	100
Calcium–45	100	Zinc–63	1,000
Calcium–47	100	Zinc–65	10
Scandium–43	1,000	Zinc–69m	100
Scandium–44m	100	Zinc–69	1,000
Scandium–44	100	Zinc–71m	1,000
Scandium–46	10	Zinc–72	100
Scandium–47	100	Gallium–65	1,000
Scandium–48	100	Gallium–66	100
Scandium–49	1,000	Gallium–67	1,000
Titanium–44	1	Gallium–68	1,000
Titanium–45	1,000	Gallium–70	1,000
Vanadium–47	1,000	Gallium–72	100
Vanadium–48	100	Gallium–73	1,000
Vanadium–49	1,000	Germanium–66	1,000
Chromium–48	1,000	Germanium–67	1,000
Chromium–49	1,000	Germanium–68	10
Chromium–51	1,000	Germanium–69	1,000
Manganese–51	1,000	Germanium–71	1,000
Manganese–52m	1,000	Germanium–75	1,000
Manganese–52	100	Germanium–77	1,000
Manganese–53	1,000	Germanium–78	1,000
		Arsenic–69	1,000
		Arsenic–70	1,000
		Arsenic–71	100

[*a] To Convert μCi to KBq, multiply the μCi value by 37.

Radionuclide	Quantity (μCi) [*a]	Radionuclide	Quantity (μCi) [*a]
Arsenic–72	100	Yttrium–86m	1,000
Arsenic–73	100	Yttrium–86	100
Arsenic–74	100	Yttrium–87	100
Arsenic–76	100	Yttrium–88	10
Arsenic–77	100	Yttrium–90m	1,000
Arsenic–78	1,000	Yttrium–90	10
Selenium–70	1,000	Yttrium–91m	1,000
Selenium–73m	1,000	Yttrium–91	10
Selenium–73	100	Yttrium–92	100
Selenium–75	100	Yttrium–93	100
Selenium–79	100	Yttrium–94	1,000
Selenium–81m	1,000	Yttrium–95	1,000
Selenium–81	1,000	Zirconium–86	100
Selenium–83	1,000	Zirconium–88	10
Bromine–74m	1,000	Zirconium–89	100
Bromine–74	1,000	Zirconium–93	1
Bromine–75	1,000	Zirconium–95	10
Bromine–76	100	Zirconium–97	100
Bromine–77	1,000	Niobium–88	1,000
Bromine–80m	1,000	Niobium–89m	
Bromine–80	1,000	(66 min)	1,000
Bromine–82	100	Niobium–89	
Bromine–83	1,000	(122 min)	1,000
Bromine–84	1,000	Niobium–90	100
Krypton–74	1,000	Niobium–93m	10
Krypton–76	1,000	Niobium–94	1
Krypton–77	1,000	Niobium–95m	100
Krypton–79	1,000	Niobium–95	100
Krypton–81	1,000	Niobium–96	100
Krypton–83m	1,000	Niobium–97	1,000
Krypton–85m	1,000	Niobium–98	1,000
Krypton–85	1,000	Molybdenum–90	100
Krypton–87	1,000	Molybdenum –93m	100
Krypton–88	1,000	Molybdenum–93	10
Rubidium–79	1,000	Molybdenum–99	100
Rubidium–81m	1,000	Molybdenum–101	1,000
Rubidium–81	1,000	Technetium–93m	1,000
Rubidium–82m	1,000	Technetium–93	1,000
Rubidium–83	100	Technetium–94m	1,000
Rubidium–84	100	Technetium–94	1,000
Rubidium–86	100	Technetium–96m	1,000
Rubidium–87	100	Technetium–96	100
Rubidium–88	1,000	Technetium–97m	100
Rubidium–89	1,000	Technetium–97	1,000
Strontium–80	100	Technetium–98	10
Strontium–81	1,000	Technetium–99m	1,000
Strontium–83	100	Technetium–99	100
Strontium–85m	1,000	Technetium–101	1,000
Strontium–85	100	Technetium–104	1,000
Strontium–87m	1,000	Ruthenium–94	1,000
Strontium–89	10	Ruthenium–97	1,000
Strontium–90	0.1	Ruthenium–103	100
Strontium–91	100	Ruthenium–105	1,000
Strontium–92	100	Ruthenium–106	1
		Rhodium–99m	1,000

Radionuclide	Quantity (μCi) *a	Radionuclide	Quantity (μCi) *a
Rhodium–99	100	Tin–121m	100
Rhodium–100	100	Tin–121	1,000
Rhodium–101m	1,000	Tin–123m	1,000
Rhodium –101	10	Tin–123	10
Rhodium–102m	10	Tin–125	10
Rhodium–102	10	Tin–126	10
Rhodium–103m	1,000	Tin–127	1,000
Rhodium–105	100	Tin–128	1,000
Rhodium–106m	1,000	Antimony–115	1,000
Rhodium–107	1,000	Antimony–116m	1,000
Palladium–100	100	Antimony–116	1,000
Palladium–101	1,000	Antimony–117	1,000
Palladium–103	100	Antimony–118m	1,000
Palladium–107	10	Antimony–119	1,000
Palladium–109	100	Antimony–120	
Silver–102	1,000	(16m)	1,000
Silver–103	1,000	Antimony–120	
Silver–104m	1,000	(5.76d)	100
Silver–104	1,000	Antimony–122	100
Silver–105	100	Antimony–124m	1,000
Silver–106m	100	Antimony–124	10
Silver–106	1,000	Antimony–125	100
Silver–108m	1	Antimony–126m	1,000
Silver–110m	10	Antimony–130	1,000
Silver–111	100	Antimony–131	1,000
Silver–112	100	Tellurium–116	1,000
Silver–115	1,000	Tellurium–121m	10
Cadmium–104	1,000	Tellurium–121	100
Cadmium–107	1,000	Tellurium–123m	10
Cadmium–109	1	Tellurium–123	100
Cadmium–113m	0.1	Tellurium–125m	10
Cadmium–113	100	Antimony–126	100
Cadmium–115m	10	Antimony–127	100
Cadmium–115	100	Tellurium–127m	10
Cadmium–117m	1,000	Tellurium–127	1,000
Cadmium–117	1,000	Antimony–128 (10.4 m)	1,000
Indium–109	1,000	Antimony–128 (9.01 h)	100
Indium–110m		Antimony–129	100
(69.1m)	1,000	Tellurium–129m	10
Indium–110		Tellurium–129	1,000
(4.9h)	1,000	Tellurium–131m	10
Indium–111	100	Tellurium–131	100
Indium–112	1,000	Tellurium–132	10
Indium–113m	1,000	Tellurium–133m	100
Indium–114m	10	Tellurium–133	1,000
Indium–115m	1,000	Tellurium–134	1,000
Indium–115	100	Iodine–120m	1,000
Indium–116m	1,000	Iodine–120	100
Indium–117m	1,000	Iodine–121	1,000
Indium–117	1,000	Iodine–123	100
Indium–119m	1,000	Iodine–124	10
Tin–110	100	Iodine–125	1
Tin–111	1,000	Iodine–126	1
Tin–113	100	Iodine–128	1,000
Tin–117m	100	Iodine–129	1
Tin–119m	100	Iodine–130	10

Radionuclide	Quantity (μCi) *a	Radionuclide	Quantity (μCi) *a
Iodine–131	1	Cerium–139	100
Iodine–132m	100	Cerium–141	100
Iodine–132	100	Cerium–143	100
Iodine–133	10	Cerium–144	1
Iodine–134	1,000	Praseodymium–136	1,000
Iodine–135	100	Praseodymium–137	1,000
Xenon–120	1,000	Praseodymium–138m	1,000
Xenon–121	1,000	Praseodymium–139	1,000
Xenon–122	1,000	Praseodymium–142m	1,000
Xenon–123	1,000	Praseodymium–142	100
Xenon–125	1,000	Praseodymium–143	100
Xenon–127	1,000	Praseodymium–144	1,000
Xenon–129m	1,000	Praseodymium–145	100
Xenon–131m	1,000	Praseodymium–147	1,000
Xenon–133m	1,000	Neodymium–136	1,000
Xenon–133	1,000	Neodymium–138	100
Xenon–135m	1,000	Neodymium–139m	1,000
Xenon–135	1,000	Neodymium–139	1,000
Xenon–138	1,000	Neodymium–141	1,000
Cesium–125	1,000	Neodymium–147	100
Cesium–127	1,000	Neodymium–149	1,000
Cesium–129	1,000	Neodymium–151	1,000
Cesium–130	1,000	Promethium–141	1,000
Cesium–131	1,000	Promethium–143	100
Cesium–132	100	Promethium–144	10
Cesium–134m	1,000	Promethium–145	10
Cesium–134	10	Promethium–146	1
Cesium–135m	1,000	Promethium–147	10
Cesium–135	100	Promethium–148m	10
Cesium–136	10	Promethium–148	10
Cesium–137	10	Promethium–149	100
Cesium–138	1,000	Promethium–150	1,000
Barium–126	1,000	Promethium–151	100
Barium–128	100	Samarium–141m	1,000
Barium–131m	1,000	Samarium–141	1,000
Barium–131	100	Samarium–142	1,000
Barium–133m	100	Samarium–145	100
Barium–133	100	Samarium–146	1
Barium–135m	100	Samarium–147	100
Barium–139	1,000	Samarium–151	10
Barium–140	100	Samarium–153	100
Barium–141	1,000	Samarium–155	1,000
Barium–142	1,000	Samarium–156	1,000
Lanthanum–131	1,000	Europium–145	100
Lanthanum–132	100	Europium–146	100
Lanthanum–135	1,000	Europium–147	100
Lanthanum–137	10	Europium–148	10
Lanthanum–138	100	Europium–149	100
Lanthanum–140	100	Europium–150 (12.62h)	100
Lanthanum–141	100	Europium–150 (34.2y)	1
Lanthanum–142	1,000	Europium–152m	100
Lanthanum–143	1,000	Europium–152	1
Cerium–134	100	Europium–154	1
Cerium–135	100	Europium–155	10
Cerium–137m	100	Europium–156	100
Cerium–137	1,000	Europium– 157	100

Radionuclide	Quantity (μCi) *a	Radionuclide	Quantity (μCi) *a
Europium–158	1,000	Ytterbium–169	100
Gadolinium–145	1,000	Ytterbium–175	100
Gadolinium–146	10	Ytterbium–177	1,000
Gadolinium–147	100	Ytterbium–178	1,000
Gadolinium–148	0.001	Lutetium–169	100
Gadolinium–149	100	Lutetium–170	100
Gadolinium–151	10	Lutetium–171	100
Gadolinium–152	100	Lutetium–172	100
Gadolinium–153	10	Lutetium–173	10
Gadolinium–159	100	Lutetium–174m	10
Terbium–147	1,000	Lutetium–174	10
Terbium–149	100	Lutetium–176m	1,000
Terbium– 150	1,000	Lutetium–176	100
Terbium–151	100	Lutetium–177m	10
Terbium–153	1,000	Lutetium–177	100
Terbium–154	100	Lutetium–178m	1,000
Terbium–155	1,000	Lutetium–178	1,000
Terbium–156m (5.0 h)	1,000	Lutetium–179	1,000
Terbium–156m (24.4h)	1,000	Hafnium–170	100
Terbium–156	100	Hafnium–172	1
Terbium–157	10	Hafnium–173	1,000
Terbium–158	1	Hafnium–175	100
Terbium–160	10	Hafnium–177m	1,000
Terbium–161	100	Hafnium–178m	0.1
Dysprosium–155	1,000	Hafnium–179m	10
Dysprosium–157	1,000	Hafnium–180m	1,000
Dysprosium–159	100	Hafnium–181	10
Dysprosium–165	1,000	Hafnium–182m	1,000
Dysprosium–166	100	Hafnium–182	0.1
Holmium–155	1,000	Hafnium–183	1,000
Holmium–157	1,000	Hafnium–184	100
Holmium–159	1,000	Tantalum–172	1,000
Holmium–161	1,000	Tantalum–173	1,000
Holmium–162m	1,000	Tantalum–174	1,000
Holmium–162	1,000	Tantalum–175	1,000
Holmium–164m	1,000	Tantalum–176	100
Holmium–164	1,000	Tantalum–177	1,000
Holmium–166m	1	Tantalum–178	1,000
Holmium–166	100	Tantalum–179	100
Holmium–167	1,000	Tantalum–180m	1,000
Erbium–161	1,000	Tantalum–180	100
Erbium–165	1,000	Tantalum–182m	1,000
Erbium–169	100	Tantalum–182	10
Erbium–171	100	Tantalum–183	100
Erbium–172	100	Tantalum–184	100
Thulium–162	1,000	Tantalum–185	1,000
Thulium–166	100	Tantalum–186	1,000
Thulium–167	100	Tungsten–176	1,000
Thulium–170	10	Tungsten–177	1,000
Thulium–171	10	Tungsten–178	1,000
Thulium–172	100	Tungsten–179	1,000
Thulium–173	100	Tungsten–181	1,000
Thulium–175	1,000	Tungsten–185	100
Ytterbium–162	1,000	Tungsten–187	100
Ytterbium–166	100	Tungsten–188	10
Ytterbium–167	1,000	Rhenium–177	1,000

Radionuclide	Quantity (μCi) *a	Radionuclide	Quantity (μCi) *a
Rhenium–178	1,000	Mercury–193m	100
Rhenium–181	1,000	Mercury–193	1,000
Rhenium–182 (12.7h)	1,000	Mercury–194	1
Rhenium–182 (64.0 h)	100	Mercury–195m	100
Rhenium–184m	10	Mercury–195	1,000
Rhenium–184	100	Mercury–197m	100
Rhenium–186m	10	Mercury–197	1,000
Rhenium–186	100	Mercury–199m	1,000
Rhenium–187	1,000	Mercury–203	100
Rhenium–188m	1,000	Thallium–194m	1,000
Rhenium–188	100	Thallium–194	1,000
Rhenium–189	100	Thallium–195	1,000
Osmium–180	1,000	Thallium–197	1,000
Osmium–181	1,000	Thallium–198m	1,000
Osmium–182	100	Thallium–198	1,000
Osmium–185	100	Thallium–199	1,000
Osmium–189m	1,000	Thallium–200	1,000
Osmium–191m	1,000	Thallium–201	1,000
Osmium–191	100	Thallium–202	100
Osmium–193	100	Thallium–204	100
Osmium–194	1	Lead–195m	1,000
Iridium–182	1,000	Lead–198	1,000
Iridium–184	1,000	Lead–199	1,000
Iridium–185	1,000	Lead–200	100
Iridium–186	100	Lead–201	1,000
Iridium–187	1,000	Lead–202m	1,000
Iridium–188	100	Lead–202	10
Iridium–189	100	Lead–203	1,000
Iridium–190m	1,000	Lead–205	100
Iridium–190	100	Lead–209	1,000
Iridium–192m (1.4m)	10	Lead–210	0.01
Iridium–192 (73.8d)	1	Lead–211	100
Iridium–194m	10	Lead–212	1
Iridium–194	100	Lead–214	100
Iridium–195m	1,000	Bismuth–200	1,000
Iridium–195	1,000	Bismuth–201	1,000
Platinum–186	1,000	Bismuth–202	1,000
Platinum–188	100	Bismuth–203	100
Platinum –189	1,000	Bismuth–205	100
Platinum–191	100	Bismuth–206	100
Platinum–193m	100	Bismuth–207	10
Platinum–193	1,000	Bismuth–210m	0.1
Platinum–195m	100	Bismuth–210	1
Platinum–197m	1,000	Bismuth–212	10
Platinum–197	100	Bismuth–213	10
Platinum–199	1,000	Bismuth–214	100
Platinum–200	100	Polonium–203	1,000
Gold–193	1,000	Polonium–205	1,000
Gold–194	100	Polonium–207	1,000
Gold–195	10	Polonium–210	0.1
Gold–198m	100	Astatine–207	100
Gold–198	100	Astatine–211	10
Gold–199	100	Radon–220	1
Gold–200m	100	Radon–222	1
Gold–200	1,000	Francium–222	100
Gold–201	1,000	Francium–223	100

Radionuclide	Quantity (μCi) *a	Radionuclide	Quantity (μCi) *a
Radium–223	0.1	Plutonium–241	0.01
Radium–224	0.1	Plutonium–242	0.001
Radium–225	0.1	Plutonium–243	1,000
Radium–226	0.1	Plutonium–244	0.001
Radium–227	1,000	Plutonium–245	100
Radium–228	0.1	Americium–237	1,000
Actinium–224	1	Americium–238	100
Actinium–225	0.01	Americium–239	1,000
Actinium–226	0.1	Americium–240	100
Actinium–227	0.001	Americium–241	0.001
Actinium–228	1	Americium–242m	0.001
Thorium–226	10	Americium–242	10
Thorium–227	0.01	Americium–243	0.001
Thorium–228	0.001	Americium–244m	100
Thorium–229	0.001	Americium–244	10
Thorium–230	0.001	Americium–245	1,000
Thorium–231	100	Americium–246m	1,000
Thorium–232	100	Americium–246	1,000
Thorium–234	10	Curium–238	100
Thorium–natural	100	Curium–240	0.1
Protactinium–227	10	Curium–241	1
Protactinium–228	1	Curium–242	0.01
Protactinium–230	0.1	Curium–243	0.001
Protactinium–231	0.001	Curium–244	0.001
Protactinium–232	1	Curium–245	0.001
Protactinium–233	100	Curium–246	0.001
Protactinium–234	100	Curium–247	0.001
Uranium–230	0.01	Curium–248	0.001
Uranium–231	100	Curium–249	1,000
Uranium–232	0.001	Berkelium–245	100
Uranium–233	0.001	Berkelium–246	100
Uranium–234	0.001	Berkelium–247	0.001
Uranium–235	0.001	Berkelium–249	0.1
Uranium–236	0.001	Berkelium–250	10
Uranium–237	100	Californium–244	100
Uranium–238	100	Californium–246	1
Uranium–239	1,000	Californium–248	0.01
Uranium–240	100	Californium–249	0.001
Uranium–natural	100	Californium–250	0.001
Neptunium–232	100	Californium–251	0.001
Neptunium–233	1,000	Californium–252	0.001
Neptunium–234	100	Californium–253	0.1
Neptunium–235	100	Californium–254	0.001
Neptunium–236 (1.15E+5)	0.001	Einsteinium–250	100
Neptunium–236 (22.5h)	1	Einsteinium–250	100
Neptunium–237	0.001	Einsteinium–251	100
Neptunium–238	10	Einsteinium–253	0.1
Neptunium–239	100	Einsteinium–254m	1
Neptunium–240	1,000	Einsteinium–254	0.01
Plutonium–234	10	Fermium–252	1
Plutonium–235	1,000	Fermium–253	1
Plutonium–236	0.001	Fermium–254	10
Plutonium–237	100	Fermium–255	1
Plutonium–238	0.001	Fermium–257	0.01
Plutonium–239	0.001	Mendelevium–257	10
Plutonium–240	0.001	Mendelevium–258	0.01

Radionuclide	Quantity (μCi) [*a]
Any alpha–emitting radionuclide not listed above or mixtures of alpha emitters of unknown composition	0.001
Any radionuclide other than alpha emitting radionuclides not listed above, or mixtures of beta emitters of unknown composition	0.01

[a] The quantities listed above were derived by taking 1/10 of the most restrictive ALI listed in Table 1 Columns 1 and 2 of Schedule RHS 8–30 of this chapter, rounding to the nearest factor of 10, and arbitrarily constraining the values listed between 0.001 and 1,000 μCi (37 Bq and 37 MBq). Values of 100 μCi (3.7 MBq) have been assigned for radionuclides having a radioactive half–life in excess of 109 years, except rhenium, 1,000 μCi (37 MBq), to take into account their low specific activity.

NOTE: For purposes of 1200–2–5–.111, 1200–2–5–.114, and 1200–2–5–.140, where there is involved a combination of radionuclides in known amounts, the limit for the combination shall be derived as follows: determine, for each radionuclide in the combination, the ratio between the quantity present in the combination and the limit otherwise established for the specific radionuclide when not in combination. The sum of such ratios for all radionuclides in the combination may not exceed "1" — that is, unity.

[*a] To Convert μCi to KBq, multiply the μCi value by 37.

[*a] To Convert μCi to KBq, multiply the μCi value by 37.

SCHEDULE RHS 8–32

ASSIGNED PROTECTION FACTORS FOR RESPIRATORS [a]

	Operating Mode [c]	Assigned Protection Factors
I. Air–Purifying Respirators [Particulate [b] only] [c]:		
Filtering facepiece disposable [d]	Negative Pressure	[d]
Facepiece, half [e]	Negative Pressure	10
Facepiece, full	Negative Pressure	100
Facepiece, half	Powered air–purifying respirators	50
Facepiece, full	Powered air–purifying respirators	1000
Helmet/hood	Powered air–purifying respirators	1000
Facepiece, loose–fitting	Powered air–purifying respirators	25
II. Atmosphere–Supplying Respirators [Particulate, gases and vapors [f]]:		
1. Air–line respirator:		
Facepiece, half	Demand	10
Facepiece, half	Continuous Flow	50
Facepiece, half	Pressure Demand	50
Facepiece, full	Demand	100
Facepiece, full	Continuous Flow	1000
Facepiece, full	Pressure Demand	1000
Helmet/hood	Continuous Flow	1000
Facepiece, loose–fitting	Continuous Flow	25
Suit	Continuous Flow	[g]
2. Self–contained breathing apparatus (SCBA):		
Facepiece, full	Demand	[h] 100
Facepiece, full	Pressure Demand	[i] 10,000
Facepiece, full	Demand, Recirculating	[h] 100
Facepiece, full	Positive Pressure Recirculating	[i] 10,000
III. Combination Respirators		
Any combination of air–purifying and atmosphere–supplying respirators	Assigned protection factor for type and mode of operation as listed above	

a These assigned protection factors apply only in a respiratory protection program that meets the requirements of this chapter. They are applicable only to airborne radiological hazards and may not be appropriate to circumstances when chemical or other respiratory hazards exist instead of, or in addition to, radioactive hazards. Selection and use of respirators for such circumstances must also comply with U. S. Department of Labor regulations. Radioactive contaminants for which the concentration values in Table 1, Column 3 of schedule RHS 8–32 in Rule 1200–2–5–.161 are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under these circumstances, limitations on occupancy may have to be governed by external dose limits.

b Air purifying respirators with APF <100 shall be equipped with particulate filters that are at least 95 percent (95%) efficient. Air purifying respirators with APF = 100 shall be equipped with particulate filters that are at least 99 percent (99%) efficient. Air purifying respirators with APFs >100 shall be equipped with particulate filters that are at least 99.97 percent (99.97%) efficient.

c The licensee may apply to the Division for the use of an APF greater than 1 for sorbent cartridges as protection against airborne radioactive gases and vapors (e.g., radioiodine).

d Licensees may permit individuals to use this type of respirator who have not been medically screened or fit tested on the device provided that no credit be taken for their use in estimating intake or dose. It is also recognized that it is difficult to perform an effective positive or negative pressure pre-use user seal check on this type of device. All other respiratory protection program requirements listed in Rule 1200–2–5–.92 apply. An assigned protection factor has not been assigned for these devices. However, an APF equal to 10 may be used if the licensee can demonstrate a fit factor of at least 100 by use of a validated or evaluated, qualitative or quantitative fit test.

e Under-chin type only. No distinction is made in this Schedule between elastomeric half-masks with replaceable cartridges and those designed with the filter medium as an integral part of the facepiece (e.g., disposable or reusable disposable). Both types are acceptable so long as the seal area of the latter contains some substantial type of seal-enhancing material such as rubber or plastic, the two or more suspension straps are adjustable, the filter medium is at least 95 percent (95%) efficient and all other requirements of this chapter are met.

f The assigned protection factors for gases and vapors are not applicable to radioactive contaminants that present an absorption or submersion hazard. For tritium oxide vapor, approximately one-third of the intake occurs by absorption through the skin so that an overall protection factor of 3 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Exposure to radioactive noble gases is not considered a significant respiratory hazard, and protective actions for these contaminants should be based on external (submersion) dose considerations.

g No NIOSH approval schedule is currently available for atmosphere supplying suits. This equipment may be used in an acceptable respiratory protection program as long as all the other minimum program requirements, with the exception of fit testing, are met (i.e., Rule 1200–2–5–.92).

h The licensee should implement institutional controls to assure that these devices are not used in areas immediately dangerous to life or health (IDLH).

i This type of respirator may be used as an emergency device in unknown concentrations for protection against inhalation hazards. External radiation hazards and other limitations to permitted exposure such as skin absorption shall be taken into account in these circumstances. This device may not be used by any individual who experiences perceptible outward leakage of breathing gas while wearing the device.

SCHEDULE RHS 8–33

REQUIREMENTS FOR TRANSFER OF LOW–LEVEL RADIOACTIVE WASTE FOR DISPOSAL AT LAND DISPOSAL FACILITIES AND MANIFESTS

I. Manifest

A waste generator, collector, or processor who transports, or offers for transportation, low–level radioactive waste intended for ultimate disposal at a licensed low–level radioactive waste land disposal facility shall prepare a manifest. The manifest shall contain the information requested on applicable NRC Forms 540 (Uniform Low–Level Radioactive Waste Manifest (Shipping Paper)) and 541 (Uniform Low–Level Radioactive Waste Manifest (Container and Waste Description)) and, if necessary, on an applicable NRC Form 542 (Uniform Low–Level Radioactive Waste Manifest (Manifest Index and Regional Compact Tabulation)). NRC Forms 540 and 540A shall be completed and shall physically accompany the pertinent low–level waste shipment. Upon agreement between shipper and consignee, NRC Forms 541 and 541A and 542 and 542A may be completed, transmitted and stored in electronic media with the capability for producing legible, accurate and complete records of the respective forms. Licensees are not required to comply with the manifesting requirements of this rule when they ship:

1. LLW for processing and expect its return (i.e., for storage under their license) prior to disposal at a licensed land disposal facility;

2. LLW that is being returned to the licensee who is the 'waste generator' or 'generator,' as defined in this rule; or

3. Radioactively contaminated material to a 'waste processor' that becomes the processor's 'residual waste.'

For guidance in completing these forms, refer to the instructions that accompany the forms. Copies of manifests required by this appendix may be legible carbon copies, photocopies or computer printouts that reproduce the data in the format of the uniform manifest.

NRC Forms 540, 540A, 541, 541A, 542 and 542A and the accompanying instructions, in hard copy, may be obtained from the Information and Records Management Branch, Office of Information Resources Management, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone (301) 415–7232.

This appendix includes information requirements of the Department of Transportation, as codified in 49 CFR part 172. Information on hazardous, medical, or other waste, required to meet Environmental Protection Agency regulations, as codified in 40 CFR parts 259, 261 or elsewhere, is not addressed in this section and must be provided on the required EPA forms. However, the required EPA forms shall accompany the Uniform Low–Level Radioactive Waste Manifest required by this chapter.

As used in this appendix, the following definitions apply:

1. Chelating agent has the same meaning as that given in Rule 1200–2–11–.03.

2. Chemical description means a description of the principal chemical characteristics of a low–level radioactive waste.

3. Computer–readable medium means that the regulatory agency's computer can transfer the information from the medium into its memory.

4. Consignee means the designated receiver of the shipment of low–level radioactive waste.

5. Decontamination facility means a facility operating under a license issued by the Division, the U.S. Nuclear Regulatory Commission, or another Agreement State, whose principal purpose is decontamination of equipment or materials to accomplish recycle, reuse or other waste management objectives and, for purposes of this rule, is not considered to be a consignee for LLW shipments.

6. Disposal container means a container principally used to confine low–level radioactive waste during disposal operations at a land disposal facility (also see 'high integrity container'). Note that for some shipments, the disposal container may be the transport package.

7. EPA identification number means the number received by a transporter following application to the Administrator of EPA as required by 40 CFR 263.

8. Generator means a licensee operating under a license issued by the Division, the U.S. Nuclear Regulatory Commission, or another Agreement State who:

   a. Is a waste generator as defined in this rule, or

   b. Is the licensee to whom waste can be attributed within the context of the Low–Level Radioactive Waste Policy Amendments Act of 1985 (e.g., waste generated as a result of decontamination or recycle activities).

9. High integrity container (HIC) means a container commonly designed to meet the structural stability requirements of paragraph 1200–2–11–.17(7) and to meet Department of Transportation requirements for a Type A package.

10. Land disposal facility has the same meaning as that given in Rule 1200–2–11–.03.

11. NRC Forms 540, 540A, 541, 541A, 542 and 542A means official NRC Forms referenced in this appendix. Licensees need not use originals of these NRC Forms as long as any substitute forms are equivalent to the original documentation in respect to content, clarity, size and location of information. Upon agreement between the shipper and consignee, NRC Forms 541 (and 541A) and NRC Forms 542 (and 542A) may be completed, transmitted and stored in electronic media. The electronic media shall have the capability for producing legible, accurate and complete records in the format of the uniform manifest.

12. Package means the assembly of components necessary to ensure compliance with the packaging requirements of U.S. DOT regulations, together with its radioactive contents, as presented for transport.

13. Physical description means the items called for on NRC Form 541 to describe a low–level radioactive waste.

14. Residual waste means low–level radioactive waste resulting from processing or decontamination activities that cannot be easily separated into distinct batches attributable to specific waste generators. This waste is attributable to the processor or decontamination facility, as applicable.

15. Shipper means the licensed entity (i.e., the waste generator, waste collector, or waste processor) who offers low–level radioactive waste for transportation, typically consigning this type of waste to a licensed waste collector, waste processor, or land disposal facility operator.

16. Shipping paper means NRC Form 540 and, if required, NRC Form 540A which includes the information required by U.S. DOT in 49 CFR 172.

17. Source material has the same meaning as that given in subparagraph 1200–2–5–.32.

18. Special nuclear material has the same meaning as that given in T.C.A. §68–202–202(1).

19. Uniform Low–Level Radioactive Waste Manifest (or uniform manifest) means the combination of NRC Forms 540, 541 and, if necessary, 542 and their respective continuation sheets as needed, or equivalent.

20. Waste collector means an entity, operating under a license issued by the Division, the U.S. NRC or another Agreement State, whose principal purpose is to collect and consolidate waste generated by others and to transfer this waste, without processing or repackaging the collected waste, to another licensed waste collector, licensed waste processor or licensed land disposal facility.

21. Waste description means the physical, chemical and radiological description of a low–level radioactive waste as called for on NRC Form 541.

22. Waste generator means an entity, operating under a license issued by the Division, the U.S. NRC or another Agreement State, who:

    a. Possesses any material or component that contains radioactivity or is radioactively contaminated for which the licensee foresees no further use, and:

    b. Transfers this material or component to a licensed land disposal facility or to a licensed waste collector or processor for handling or treatment before disposal. A licensee performing processing or decontamination services may be a 'waste generator' if the transfer of low–level radioactive waste from its facility is defined as 'residual waste.'

23. Waste processor means an entity, operating under a license issued by the Division, the U.S. NRC or another Agreement State, whose principal purpose is to process, repackage or otherwise treat low–level radioactive material or waste generated by others before eventual transfer of waste to a licensed low–level radioactive waste land disposal facility.

24. Waste type means a waste within a disposal container having a unique physical description (i.e., a specific waste descriptor code or description; or a waste sorbed on or solidified in a specifically defined media).

Information Requirements

A. General Information

The shipper of the radioactive waste shall provide the following information on the uniform manifest:

1. The name, facility address, and telephone number of the licensee shipping the waste.

2. An explicit declaration indicating whether the shipper is acting as a waste generator, collector, processor, or a combination of these identifiers for the purposes of the manifested shipment; and

3. The name, address, and telephone number or the name and U.S. EPA hazardous waste identification number for the carrier transporting the waste to the land disposal facility.

B. Shipment Information

The shipper of the radioactive waste shall provide the following information regarding the waste shipment on the uniform manifest:

1. The date of the waste shipment;

2. The total number of packages/disposal containers;

3. The total disposal volume and disposal weight in the shipments;

4. The total radionuclide activity in the shipment;

5. The activity of each of the radionuclides H-3, C-14, Tc-99, and I-129 contained in the shipment; and

6. The total masses of U-233, U-235, and plutonium in special nuclear material, and the total mass of uranium and thorium in source material.

C. Disposal Container and Waste Information.

The shipper of the radioactive waste shall provide the following information on the uniform manifest regarding the waste and each disposal container of waste in the shipment:

1. An alphabetic or numeric identification that uniquely identifies each disposal container in the shipment;

2. A physical description of the disposal container, including the manufacturer and model of any high integrity container;

3. The volume displaced by the disposal container;

4. The gross weight of the disposal container, including the waste;

5. For waste consigned to a disposal facility, the maximum radiation level at the surface of each disposal container;

6. A physical and chemical description of the waste;

7. The total weight percentage of chelating agent for any waste containing more than 0.1% chelating agent by weight, plus the identity of the principal chelating agent;

8. The approximate volume of waste within a container;

9. The sorbing or solidification media, if any, and the identity of the solidification media vendor and brand name;

10. The identities and activities of individual radionuclides contained in each container, the masses of U–233, U–235 and plutonium in special nuclear material, and the masses of uranium and thorium in source material. For discrete waste types (i.e., activated materials, contaminated equipment, mechanical filters, sealed source/devices and wastes in solidification/stabilization media), the identities and activities of individual radionuclides associated with or contained on these waste types within a disposal container shall be reported;

11. The total radioactivity within each container; and

12. For wastes consigned to a disposal facility, the classification of the waste under paragraph 1200–2–11–.17(6). Waste not meeting the structural stability requirements of subparagraph 1200–2–11–.17(7)(b) shall be identified.

D. Uncontainerized Waste Information.

The shipper of the radioactive waste shall provide the following information on the uniform manifest regarding a waste shipment delivered without a disposal container:

1. The approximate volume and weight of the waste;

2. A physical and chemical description of the waste;

3. The total weight percentage of chelating agent if the chelating agent exceeds 0.1% by weight, plus the identity of the principal chelating agent;

4. For waste consigned to a disposal facility, the classification of the waste under paragraph 1200–2–11–.17(6). Waste not meeting the structural stability requirements of subparagraph 1200–2–11–.17(7)(b) shall be identified;

5. The identities and activities of individual radionuclides contained in the waste, the masses of U–233, U–235 and plutonium in special nuclear material, and the masses of uranium and thorium in source material; and

6. For wastes consigned to a disposal facility, the maximum radiation levels at the surface of the waste.

E. Multi–Generator Disposal Container Information.

This section applies to disposal containers enclosing mixtures of waste originating from different generators. (Note: The origin of the LLW resulting from a processor's activities may be attributable to one or more 'generators' (including 'waste generators') as defined in this rule). It also applies to mixtures of wastes shipped in an uncontainerized form, for which portions of the mixture within the shipment originate from different generators.

1. For homogeneous mixtures of waste, such as incinerator ash, provide the waste description applicable to the mixture and the volume of the waste attributed to each generator.

2. For heterogeneous mixtures of waste, such as the combined products from a large compactor, identify each generator contributing waste to the disposal container and, for discrete waste types (i.e., activated materials, contaminated equipment, mechanical filters, sealed source/devices and wastes in solidification/stabilization media), the identities and activities of individual radionuclides contained on these waste types within the disposal container. For each generator, provide the following:

    a. The volume of waste within the disposal container;

    b. A physical and chemical description of the waste, including the solidification agent, if any;

    c. The total weight percentage of chelating agents for any disposal container containing more than 1/10 of one percent (0.1%) chelating agent by weight, plus the identity of the principal chelating agent;

    d. The sorbing or solidification media, if any, and the identity of the solidification media vendor and brand name if the media is claimed to meet stability requirements in subparagraph 1200–2–11–.17(7)(b); and

    e. Radionuclide identities and activities contained in the waste, the masses of U–233, U–235 and plutonium in special nuclear material, and the masses of uranium and thorium in source material if contained in the waste.

II. Certification

An authorized representative of the waste generator, processor, or collector shall certify by signing and dating the shipment manifest that the transported materials are properly classified, described, packaged, marked, and labeled and are in proper condition for transportation according to the applicable regulations of the U.S. Department of Transportation, the U.S. Nuclear Regulatory Commission and the Division of Radiological Health. A collector, in signing the certification, is certifying that nothing has been done to the collected waste that would invalidate the waste generator's certification.

III. Control and Tracking

A. Any licensee who transfers radioactive waste to a land disposal facility or a licensed waste collector shall comply with the requirements in paragraphs A.1through 9 of this section. Any licensee who transfers waste to a licensed waste processor for waste treatment or repackaging shall comply with the requirements of paragraphs A.4 through 9. A licensee shall:

1. Prepare all waste so that the waste is classified according to paragraph 1200–2–11–.17(6) and meets the waste characteristics requirements in paragraph 1200–2–11–.17(7);

2. Label each disposal container (or transport package if potential radiation hazards preclude labeling of the individual disposal container) of waste to identify whether it is Class A waste, Class B waste, Class C waste, or greater than Class C waste, in accordance with paragraph 1200–2–11–.17(6);

3. Conduct a quality assurance program to assure compliance with paragraph 1200–2–11–.17(6) and paragraph 1200–2–11–.17(7); the program shall include management evaluation of audits;

4. Prepare the NRC Uniform Low–Level Radioactive Waste Manifest as required by this appendix;

5. Forward a copy or electronically transfer the Uniform Low–Level Radioactive Waste Manifest to the intended consignee so that either:

   a. Receipt of the manifest precedes the LLW shipment, or

   b. The manifest is delivered to the consignee with the waste at the time the waste is transferred to the consignee.

   c. Using both a. and b. is also acceptable;

6. Include NRC Form 540 (and NRC Form 540A, if required) with the shipment regardless of the option chosen in part 5 of this subparagraph;

7. Receive acknowledgement of the receipt of the shipment in the form of a signed copy of NRC Form 540 or equivalent documentation;

8. Retain a copy of or electronically store the Uniform Low–Level Radioactive Waste Manifest and documentation of acknowledgment of receipt as the record of transfer of licensed material as required by Chapter 1200–2–10; and

9. For any shipments or any part of a shipment for which acknowledgment of receipt has not been received within the times set forth in this appendix, conduct an investigation in accordance with paragraph E of this appendix.

B. Any waste collector licensee who handles only prepackaged waste shall:

1. Acknowledge receipt of the waste from the shipper within one (1) week of receipt by returning a signed copy of NRC Form 540 or equivalent documentation;

2. Prepare a new manifest to reflect consolidated shipments that meet the requirements of this appendix. The waste collector shall ensure that, for each container of waste in the shipment, the manifest identifies the generator of that container of waste;

3. Forward a copy or electronically transfer the Uniform Low–Level Radioactive Waste Manifest to the intended consignee so that either;

   a. Receipt of the manifest precedes the LLW shipment, or

   b. The manifest is delivered to the consignee with the waste at the time the waste is transferred to the consignee.

   c. Using both (i) and (ii) is also acceptable;

4. Include NRC Form 540 (and NRC Form 540A, if required) with the shipment regardless of the option chosen in part 3 of this subparagraph;

5. Receive acknowledgement of the receipt of the shipment in the form of a signed copy of NRC Form 540;

6. Retain a copy of or electronically store the Uniform Low–Level Radioactive Waste Manifest and documentation of acknowledgment of receipt as the record of transfer of licensed material as required by Chapter 1200–2–10;

7. For any shipments or any part of a shipment for which acknowledgment of receipt has not received within the times set forth in this appendix, conduct an investigation in accordance with paragraph E of this appendix; and

8. Notify the shipper and the Director, Division of Radiological Health, when any shipment, or part of a shipment, has not arrived within 60 days after receipt of an advance manifest, unless notified by the shipper that the shipment has been cancelled.

C. Any licensed waste processor who treats or repackages wastes shall:

1. Acknowledge receipt of the waste from the shipper within one (1) week of receipt by returning a signed copy of NRC Form 540 or equivalent documentation;

2. Prepare a new manifest that meets the requirements of this appendix. Preparation of the new manifest reflects that the processor is responsible for meeting these requirements. For each container of waste in the shipment, the manifest shall identify the waste generators, the preprocessed waste volume and the other information required in paragraph I.E. of this appendix;

3. Prepare all wastes so that the waste is classified according to paragraph 1200–2–11–.17(6) and meets the waste characteristics requirements in paragraph 1200–2–11–.17(7);

4. Label each package of waste to identify whether it is Class A waste, Class B waste, or Class C waste, in accordance with paragraphs 1200–2–11–.17(6) and 1200–2–11–.17(8);

5. Conduct a quality assurance program to ensure compliance with paragraphs 1200–2–11–.17(6) and 1200–2–11–.17(7). The program shall include management evaluation of audits;

6. Forward a copy or electronically transfer the Uniform Low–Level Radioactive Waste Manifest to the intended consignee so that either:

   a. Receipt of the manifest precedes the LLW shipment, or

   b. The manifest is delivered to the consignee with the waste at the time the waste is transferred to the consignee.

   c. Using both (i) and (ii) is also acceptable;

7. Include NRC Form 540 (and NRC Form 540A, if required) with the shipment regardless of the option chosen in paragraph C.6 of this section;

8. Receive acknowledgement of the receipt of the shipment in the form of a signed copy of NRC Form 540;

9. Retain a copy of or electronically store the Uniform Low–Level Radioactive Waste Manifest and documentation of acknowledgment of receipt as the record of transfer of licensed material as required by Chapter 1200–2–10;

10. For any shipment or any part of a shipment for which acknowledgment of receipt has not been received within the times set forth in this appendix, conduct an investigation in accordance with paragraph E of this appendix; and

11. Notify the shipper and the Director, Division of Radiological Health, when any shipment, or part of a shipment, has not arrived within 60 days after receipt of an advance manifest, unless notified by the shipper that the shipment has been cancelled.

D. The land disposal facility operator shall:

1. Acknowledge receipt of the waste within one (1) week of receipt by returning, as a minimum, a signed copy of NRC Form 540 or equivalent documentation to the shipper. The shipper to be notified is the licensee who last possessed the waste and transferred the waste to the operator. If any discrepancy exists between materials listed on the Uniform Low–Level Radioactive Waste Manifest and materials received, copies or electronic transfer of the affected forms must be returned indicating the discrepancy;

2. Maintain copies of all completed manifests or equivalent documentation and electronically store the information required by paragraph 1200–2–11–.19(1) until the Division terminates the license; and

3. Notify the shipper and the Director, Division of Radiological Health, when any shipment or part of a shipment has not arrived within 60 days after receipt of an advance manifest, unless notified by the shipper that the shipment has been cancelled.

E. Any shipment or portion of a shipment for which acknowledgment is not received within the times set forth in this section shall:

1. Be investigated by the shipper if the shipper has not received notification or receipt within 20 days after transfer; and

2. Be traced and reported. The investigation shall include tracing the shipment and filing a report with the Division of Radiological Health, at the address given in Rule 120-2-4-.07. Each licensee who conducts a trace investigation shall file a written report with the Division within two (2) weeks of completion of the investigation.

Authority: *T.C.A. §§4–5–201 et seq., 68-202-101 et seq., 68–202–201 et seq., 68-202-203, and 68-202-206.* ***Administrative History:*** *Original rule filed October 19, 1993; effective January 2, 1994. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006. Amendment filed _____; effective_____.*

1200–2–5–.162 TYPE X QUANTITIES AND TRANSPORT GROUPS.

(1) Transport group as used in this rule means any one of seven groups into which radionuclides in normal form are classified, according to their toxicity and their relative potential hazard in transport, in Table RHS 2–3.

(a) Any radionuclide, not specifically listed in one of the groups in Table RHS 2–3 shall be assigned to one of the groups in accordance with Table RHS 2–2.

(b) For mixtures of radionuclides the following shall apply:

1. If the identity and respective activity of each radionuclide are known, the permissible activity of each radionuclide shall be such that the sum, for all groups present, of the ratio between the total activity for each group to the permissible activity for each group will not be greater than unity.

2. If the groups of the radionuclides are known, but the activity in each group cannot be reasonably determined, the mixture shall be assigned to the most restrictive group present.

3. If the identity of all or some of the radionuclides cannot be reasonably determined, each of the unidentified radionuclides shall be considered as belonging to the most restrictive group which cannot be positively excluded.

4. Mixtures consisting of a single radioactive decay chain where the radionuclides are in the naturally occurring proportions shall be considered as consisting of a single radionuclide. The group and activity shall be that of the first member present in the chain, except that if a radionuclide "x" has a half–life longer than that of the first member and an activity greater than that of any other member, including the first, at any time during transportation, the group of the nuclide "x" and the activity of the mixture shall be the maximum activity of that nuclide "x" during transportation.

TABLE RHS 2–1 TYPE X QUANTITIES

Transport Group	Type X Quantity limit (in curies)
I	0.001
II	0.050
III	3
IV	20
V	20
VI	1,000
VII	1,000
Special Form	20

TABLE RHS 2–2

Radionuclide	Radioactive half–life 0 to 1,000 days	1,000 days to one million years	over one million years
Atomic Number 1–81	Group III	Group II	Group III
Atomic Number 82 and Over	Group I	Group I	Group III

TABLE RHS 2–3 TRANSPORT GROUPING OF RADIONUCLIDES

Element *	Radionuclide ***	Group
Actinium (89)	Ac–227	I
	Ac–228	I
Americium (95)	Am–241	I
	Am–243	I
Antimony (51)	Sb–122	IV
	Sb–124	III
	Sb–125	III
Argon (18)	Ar–37	VI
	Ar–41	II
	Ar–41 (uncompressed) **	V
Arsenic (33)	As–73	IV
	As–74	IV
	As–76	IV
	As–77	IV
Astatine (85)	At–211	III
Barium (56)	Ba–131	IV
	Ba–133	II
	Ba–140	III
Berkelium (97)	Bk–249	I
Beryllium (4)	Be–7	IV
Bismuth (83)	Bi–206	IV
	Bi–207	III
	Bi–210	II
	Bi–212	III
Bromine (35)	Br –82	IV
Cadmium (48)	Cd–109	IV
	Cd–115 m	III
	Cd–115	IV
Calcium (20)	Ca–45	IV
	Ca–47	IV
Californium (98)	Cf–249	I
	Cf–250	I
	Cf–252	I
Carbon (6)	C–14	IV
Cerium (58)	Ce–141	IV
	Ce–143	IV
	Ce–144	III
Cesium (55)	Cs–131	IV

* *Atomic number shown in parentheses.*

*** *Atomic weight shown after the radionuclide symbol.*

** *Uncompressed means at a pressure not exceeding one atmosphere.*

m *Metastable state.*

Element *	Radionuclide ***	Group
	Cs–134 m	III
	Cs–134	III
	Cs–135	IV
	Cs–136	IV
	Cs–137	III
Chlorine (17)	Cl–36	III
	Cl–38	IV
Chromium (24)	Cr–51	IV
Cobalt (27)	Co–56	III
	Co–57	IV
	Co–58 m	IV
	Co–58	IV
	Co–60	III
Copper (29)	Cu–64	IV
Curium (96)	Cm–242	I
	Cm–243	I
	Cm–244	I
	Cm–245	I
	Cm–246	I
Dysprosium (66)	Dy–154	III
	Dy–165	IV
	Dy–166	IV
Erbium (68)	Er–169	IV
	Er–171	IV
Europium (63)	Eu–130	III
	Eu–152 m	IV
	Eu–152	III
	Eu–154	II
	Eu–155	IV
Fluorine(9)	F–18	IV
Gadolinium (64)	Gd–153	IV
	Gd–159	IV
Gallium (31)	Ga–67	III
	Ga–72	IV
Germanium (32)	Ge–71	IV
Gold (79)	Au–193	III
	Au–194	III
	Au–195	III
	Au–196	IV
	Au–198	IV
	Au–199	IV
Hafnium (72)	Hf–181	IV
Holmium (67)	Ho–166	IV
Hydrogen (1)	H–3 (see tritium)	
Indium (49)	In–113 m	IV
	In–114 m	III
	In–115 m	IV
	In–115	IV
Iodine (53)	I–124	IV
	I–125	III
	I–126	III
	I–129	III

m *Metastable state.*

Element *	Radionuclide ***	Group
	I–131	III
	I–132	IV
	I–133	III
	I–134	IV
	I–135	IV
Iridium (77)	Ir–190	IV
	Ir–192	III
	Ir–194	IV
Iron (26)	Fe–55	IV
	Fe–59	IV
Krypton (36)	Kr–85 ^{m}	III
	Kr–85 ^{m} (uncompressed) **	V
	Kr–85	III
	Kr–85 (uncompressed) **	II
	Kr–87 (uncompressed) **	V
Lanthanum (57)	La–140	IV
Lead (82)	Pb–203	IV
	Pb–210	II
	Pb–212	II
Lutetium (71)	Lu–172	IV
	Lu–177	IV
Magnesium (12)	Mg–28	IV
Manganese (25)	Mn–52	IV
	Mn–54	IV
	Mn–56	IV
Mercury (80)	Hg–197 ^{m}	IV
	Hg–197	IV
	Hg–203	IV
Mixed fission products(MFP)		II
Molybdenum (42)	Mo–99	IV
Neodymium (60)	Nd–147	IV
	Nd–149	IV
Neptunium (93)	Np–237	I
	Np–239	I
Nickel (28)	Ni–56	III
	Ni–59	IV
	Ni–63	IV
	Ni–65	IV
Niobium (41)	Nb–93 ^{m}	IV
	Nb–95	IV
	Nb–97	IV
Osmium (76)	Os–185	IV
	Os–191 ^{m}	IV
	Os–191	IV
	Os–193	IV
Palladium (46)	Pd–103	IV
	Pd–109	IV
Phosphorus (15)	P–32	IV
Platinum (78)	Pt–191	IV
	Pt–193	IV

** *Uncompressed means at a pressure not exceeding one atmosphere.*

^{m} *Metastable state.*

Element *	Radionuclide ***	Group
	Pt–193 m	I
	Pt–197 m	IV
	Pt–197	IV
Plutonium (94)	Pu–238 (F)	I
	Pu–239 (F)	I
	Pu–240	I
	Pu–241 (F)	I
	Pu–242	I
Polonium (84)	Po–210	I
Potassium (19)	K–42	IV
	K–43	III
Praseodymium (59)	Pr–142	IV
	Pr–143	IV
Promethium (61)	Pm–147	IV
	Pm–149	IV
Protactinium (91)	Pa–230	I
	Pa–231	I
	Pa–233	II
Radium (88)	Ra–223	II
	Ra–224	II
	Ra–226	I
	Ra–228	I
Radon (86)	Rn–220	IV
	Rn–222	II
Rhenium (75)	Re–183	IV
	Re–186	IV
	Re–187	IV
	Re–188	IV
	Re Natural	IV
Rhodium (45)	Rh–103 m	IV
	Rh–105	IV
Rubidium (37)	Rb–86	IV
	Rb–87	IV
	Rb Natural	IV
Ruthenium (44)	Ru–97	IV
	Ru–103	IV
	Ru–105	IV
	Ru–106	III
Samarium (62)	Sm–145	III
	Sm–147	III
	Sm–151	IV
	Sm–153	IV
Scandium (21)	Sc–46	III
	Sc–47	IV
	Sc–48	IV
Selenium (34)	Se–75	IV
Silicon (14)	Si–31	IV
Silver (47)	Ag–105	IV
	Ag–110 m	III
	Ag–111	IV
Sodium (11)	Na–22	III

(F) Fissile material.

m Metastable State.

Element *	Radionuclide ***	Group
	Na–24	IV
Strontium (38)	Sr–85 ^{m}	IV
	Sr–85	IV
	Sr–89	III
	Sr–90	II
	Sr–91	III
	Sr–92	IV
Sulphur (16)	S–35	IV
Tantalum (73)	Ta–182	III
Technetium (43)	Tc–96^{m}	IV
	Tc–96	IV
	Tc–97 ^{m}	IV
	Tc–97	IV
	Tc–99 ^{m}	IV
	Tc–99	IV
Tellurium (52)	Te–125 ^{m}	IV
	Te–127 ^{m}	IV
	Te–127	IV
	Te–129 ^{m}	III
	Te–129	IV
	Te–131 ^{m}	III
	Te–132	IV
Terbium (65)	Tb–160	III
Thallium (81)	Tl–200	IV
	Tl–201	IV
	Tl–202	IV
	Tl–204	III
Thorium (90)	Th–227	II
	Th–228	I
	Th–230	I
	Th–231	I
	Th–232	III
	Th–234	II
	Th Natural	III
Thulium (69)	Tm–168	III
	Tm–170	III
	Tm–171	IV
Tin (50)	Sn–113	IV
	Sn–117 ^{m}	III
	Sn–121	III
	Sn–125	IV
Tritium (1)	H–3	IV
	H–3 (as a gas, as luminous paint, or absorbed on solid material)	VII
Tungsten (74)	W–181	IV
	W–185	IV
	W–187	IV
Uranium (92)	U–230	II
	U–232	I
	U–233 (F)	II

^{m} *Metastable State.*
(F) Fissile material.

Element *	Radionuclide ***	Group
	U–234	II
	U–235 (F)	III
	U–236	II
	U–238	III
	U Natural	III
	U Enriched (F)	III
	U Depleted	III
Vanadium (23)	V–48	IV
	V–49	III
Xenon (54)	Xe–125	III
	Xe–131 m	III
	Xe–131 m (uncompressed) **	V
	Xe–133	III
	Xe–133 (uncompressed) **	VI
	Xe–135	II
	Xe–135 (uncompressed) **	V
Ytterbium (70)	Yb–175	IV
Yttrium (39)	Y–88	III
	Y–90	IV
	Y–91 m	III
	Y–91	III
	Y–92	IV
	Y–93	IV
Zinc (30)	Zn–65	IV
	Zn–69 m	IV
	Zn–68	IV
	Zn–69	IV
Zirconium (40)	Zr–93	IV
	Zr–95	III
	Zr–97	IV

Authority: T.C.A. *§§4–5–201 et seq. and 68–202–201 et seq. Original rule filed November 17, 2005; effective January 31, 2006.*

(F) Fissile material.

** *Uncompressed means at a pressure not exceeding one atmosphere.*

m *Metastable State.*

RULES
OF
DEPARTMENT OF ENVIRONMENT AND CONSERVATION
DIVISION OF RADIOLOGICAL HEALTH

CHAPTER 1200–2–10
LICENSING AND REGISTRATION

TABLE OF CONTENTS

1200–2–10–.01 PURPOSE.

This chapter establishes requirements for the licensing and registration of sources of radiation.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.02 SCOPE.

Except as otherwise specifically provided, no person shall receive, possess, use, transfer, own, or acquire radioactive material unless authorized in a specific or general license issued pursuant to this chapter. All other sources of radiation, registered inspectors, and x–ray installations and services unless exempt from this chapter under 1200–2–10–.03, 1200–2–10–.04, 1200–2–10–.06, 1200–2–10–.07 or 1200–2–10–.30 shall be registered with the Division in accordance with the requirements of 1200–2–10–.24 of this chapter.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.03 EXEMPTIONS: SOURCE MATERIAL.

(1) Any person is exempt from this chapter to the extent that such person receives, possesses, uses, or transfers source material in any chemical mixture, compound, solution or alloy in which the source material is by weight less than 1/20 of 1 percent (0.05%) of the mixture, compound, solution, or alloy.

(2) Any person is exempt from this chapter to the extent that such person receives, possesses, uses or transfers unrefined and unprocessed ore containing source material; provided that, except as authorized in a specific license, such person shall not refine or process such ore.

(3) Any person is exempt from this chapter to the extent that such person receives, possesses, uses or transfers:

(a) Any quantities of thorium contained in: a. incandescent gas mantles; b. vacuum tubes; c. welding rods; d. electric lamps for illuminating purposes provided that each lamp does not contain more than 50 milligrams of thorium; e. germicidal lamps, sunlamps, and lamps for outdoor or industrial lighting provided that each lamp does not contain more than 2 grams of thorium; f. rare earth metals and compounds, mixtures, and products containing not more than ¼ percent (0.25%) by weight of thorium, uranium, or any combination of these; or g. personnel neutron dosimeters provided that each dosimeter does not contain more than 50 milligrams of thorium.

(b) Source material contained in the following products: a. glazed ceramic tableware, provided that the glaze contains not more than 20 percent (20%) by weight source material; b. piezoelectric ceramic glassware containing not more than ten percent (10%) by weight source material; but not including commercially manufactured glass brick, pane glass, ceramic tile, or other glass or ceramic used in construction; d. glass enamel or glass enamel frit containing not more than ten percent (10%) by weight source material imported or ordered for importation into the United States, or initially distributed by manufacturers in the United States, before July 25, 1983.

(c) Photographic film, negatives, and prints containing uranium or thorium

(d) Any finished product or part fabricated of, or containing tungsten or magnesium–thorium alloys, provided that the thorium content of the alloy does not exceed four percent (4%) by weight and that the exemption contained in this subparagraph shall not be deemed to authorize the chemical, physical, or metallurgical treatment or processing of any such product or part.

(e) Uranium contained in counterweights installed in aircraft, rockets, projectiles and missiles or stored or handled in connection with installation or removal of such counterweights, provided that:

1. The counterweights are manufactured in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission authorizing distribution by the licensee pursuant to 10 CFR 40;

2. Each counterweight has been impressed with the following legend clearly legible through any plating or other covering: "DEPLETED URANIUM" [1].

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[1] *The requirements specified in 2. and 3. of this subparagraph need not be met by counterweights manufactured prior to December 31, 1969; provided that such counterweights are impressed with the legend, "CAUTION – RADIOACTIVE MATERIAL – URANIUM" as previously required by the regulations.*

Depleting uranium means the source material uranium in which the isotope uranium–235 is less than 0.711 weight percent of the total uranium present;

3. Each counterweight is durably and legibly labeled or marked with the identification of manufacturer and the statement: "UNAUTHORIZED ALTERATIONS PROHIBITED" [2] and

Deleted: PROHIBITED"[1];

4. The exemption contained in this subparagraph shall not be deemed authorize the chemical, physical, or metallurgical treatment or processing of any such counterweights other than repair or restoration of any plating or other covering.

(f) Uranium used as shielding constituting part of any shipping container that is conspicuously and legibly impressed with the legend "CAUTION – RADIOACTIVE SHIELDING – URANIUM" and that is encased in mild steel or equally fire resistant metal or minimum wall thickness of 1/8 inch.

(g) Thorium contained in finished optical lenses, provided that each lens does not contain more than 30 percent (30%) by weight of thorium; and that the exemption contained in this subparagraph shall not be deemed to authorize either:

1. The shaping, grinding, or polishing of such lens or manufacturing processes other than the assembly of such lens into optical systems and devices without any alteration of the lens; or

2. The receipt, possession, use or transfer of thorium contained in contact lenses, in spectacles, or in eyepieces in binoculars or other optical instruments.

(h) Uranium contained in detector heads for use in fire detection units, provided that each detector head contains no more than 0.005 microcurie of uranium.

(i) Thorium contained in any finished aircraft engine part containing nickel–thoria alloy, provided that:

1. The thorium is dispersed in the nickel–thoria alloy in the form of finely divided thoria (thorium dioxide); and

2. The thorium content in the nickel–thoria alloy does not exceed four percent (4%) by weight.

(4) The exemptions in (3) of this rule do not authorize the manufacture of any of the products described.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

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[2] *Authority to transfer possession or control by the manufacturer, processor or producer of any equipment, device, commodity or other product containing byproduct material, whose subsequent possession, use, transfer and disposal by all other persons are exempted from regulatory requirements may be obtained only from the U.S. Nuclear Regulatory Commission, Washington, DC 20555.*

1200–2–10–.04 EXEMPTIONS: RADIOACTIVE MATERIALS OTHER THAN SOURCE MATERIAL.

(1) Exempt concentrations.

(a) Except as provided in 1200–2–10–.04(1)(b), any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns, or acquires products containing radioactive material introduced in concentrations not in excess of those listed in Schedule RHS 8–4.

(b) No person may introduce radioactive material into a product or material knowing or having reason to believe that it will be transferred to persons exempt under 1200–2–10–.04(1)(a) or equivalent regulations of the U.S. Nuclear Regulatory Commission, any Agreement State or Licensing State except in accordance with a license issued pursuant to 1200–2–10–.13(8) or the general license provided in 1200–2–10–.29.

(2) Exempt products. Except for persons who apply radioactive materials to or persons who incorporate radioactive material into the products listed in this paragraph, any person is exempt from these regulations to the extent that he receives, possesses, uses, transfers, owns or acquires the following products [3];

(a) Time pieces, hands or dials containing not more than the following quantities of radioactive material and not exceeding the following specified levels of radiation:

1. 25 millicuries of tritium per timepiece;

2. 5 millicuries of tritium per hand;

3. 15 millicuries of tritium per dial (bezels when used shall be considered as part of the dial);

4. 100 microcuries of promethium–147 per watch or 200 microcuries of promethium–147 per any other timepiece;

5. 20 microcuries of promethium–147 per watch hand or 40 microcuries of promethium–147 per other timepiece hand;

6. 60 microcuries of promethium–147 per watch dial or 120 microcuries of promethium–147 per other timepiece dial (bezels when used shall be considered part of the dial);

7. The levels of radiation from hands and dials containing radioactive materials will not exceed when measured through 50 milligrams per square centimeter of absorber:

(i) For wrist watches, 0.1 millirad per hour at 10 centimeters from any surface;

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[3] *Authority to transfer possession or control by the manufacturer, processor or producer of any equipment, device, commodity or other product containing byproduct material, whose subsequent possession, use, transfer and disposal by all other persons are exempted from regulatory requirements may be obtained only from the U.S. Nuclear Regulatory Commission, Washington, DC 20555.*

(ii) For pocket watches, 0.1 millirad per hour at 1 centimeter from any surface;

(iii) For any other timepiece, 0.2 millirad per hour at 10 centimeters from any surface.

8. One (1) microcuries of radium–226 per timepiece in timepieces acquired prior to the effective date of this regulation.

(b) Lock illuminators containing not more than 15 millicuries of tritium or not more than 2 millicuries of promethium–147 installed in automobile locks. The levels of radiation from each lock illuminator containing promethium–147 will not exceed 1 millirad per hour at 1 centimeter from any surface when measured through 50 milligrams per square centimeter of absorber.

(c) Balances of precision containing not more than 1 millicurie of tritium per balance or not more than 0.5 millicurie of tritium per balance part.

(d) Automobile shift quadrants containing not more than 25 millicuries of tritium.

(e) Marine compasses containing not more than 750 millicuries of tritium gas and other marine navigational instruments containing not more than 250 millicuries of tritium gas.

(f) Thermostat dials and pointers containing not more than 25 millicuries of tritium per thermostat.

(g) Electron tubes [4] containing not more than one of the following specified quantities of radioactive material per tube:

1. 150 millicuries of tritium per microwave receiver protector tube or 10 millicuries of tritium per any other electron tube;

2. 1 microcurie of cobalt–60;

3. 5 microcuries of nickel–63;

4. 30 microcuries of krypton–85;

5. 5 microcuries of cesium–137;

6. 30 microcuries of promethium–147;

provided, the levels of radiation from each electron tube containing radioactive material do not exceed 1 millirad per hour at 1 centimeter from any surface when measured through 7 milligrams per square centimeter of absorber.

(h) Resins containing scandium–46 and designed for sand consolidation in oil wells.

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[4] *"Electron tubes", as used in this subparagraph, include spark gap tubes, power tubes, gas tubes, including glow lamps, receiving tubes, microwave tubes, indicator tubes, pickup tubes, radiation detection tubes and any other completely sealed tube that is designed to conduct or control electrical currents.*

1. Any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns or acquires synthetic plastic resins containing scandium–46 that are designed for sand consolidation in oil wells.

2. Such resins shall have been manufactured or imported in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission, or shall have been manufactured in accordance with the specifications contained in a specific license issued by the Division or any Agreement State to the manufacturer of such resins pursuant to licensing requirements equivalent to those in Sections 32.16 and 32.17 of 10 CFR (Code of Federal Regulations) Part 32 of the regulations of the U.S. Nuclear Regulatory Commission.

3. This exemption does not authorize the manufacture of any resins containing scandium–46.

(i) Gas and aerosol detectors containing radioactive material.

1. Except for persons who manufacture, process or produce gas and aerosol detectors containing radioactive material, any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns, or acquires radioactive material in gas and aerosol detectors designed to protect life or property from fires and airborne hazards provided that detectors containing radioactive material shall have been manufactured, imported, or transferred [5] in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission pursuant to Section 32.26 of 10 CFR Part 32 or a Licensing State pursuant to regulations equivalent to 1200–2–10–.13(15) that authorizes the transfer of the detectors to persons who are exempt from regulatory requirements.

Deleted: transferred [2]

2. Gas and aerosol detectors previously manufactured and distributed to general licensees in accordance with a specific license issued by an Agreement State shall be considered exempt under 1200–2–10–.04(2)(i)1., provided that the device is labeled in accordance with the specific license authorizing distribution of the generally licensed device, and provided further that they meet the requirements of 1200–2–10–.13(15).

3. Gas and aerosol detectors containing NARM previously manufactured and distributed in accordance with a specific license issued by a Licensing State shall be considered exempt under 1200–2–1–.04(2)(i)1., provided that the device is labeled in accordance with the specific license authorizing distribution, and provided further that they meet the requirements of 1200–2–10–.13(15).

(j) Self–luminous products containing radioactive material.

1. Except for persons who manufacture, process, or produce self–luminous products containing tritium, krypton–85, or promethium–147, any person is

[5] *Authority to transfer possession or control by the manufacturer, processor or producer of any equipment, device, commodity or other product containing byproduct material, whose subsequent possession, use, transfer and disposal by all other persons are exempted from regulatory requirements may be obtained only from the U.S. Nuclear Regulatory Commission, Washington, DC 20555.*

exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns or acquires tritium, krypton–85, promethium–147 in self luminous products manufactured, processed, produced, imported, or transferred in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission pursuant to Section 32.22 of 10 CFR Part 32, which license authorizes the transfer of the product to persons who are exempt from regulatory requirements.

2. The exemption in 1200–2–10–.04(2)(j)1. does not apply to tritium, krypton–85, or promethium–147 used in products for frivolous purposes or in toys or adornments.

3. Any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers or owns self–luminous products containing less than 0.1 microcurie of radium–226 that were acquired prior to the effective date of this regulation.

(k) Ionizing radiation measuring instruments containing, for purposes of internal calibration or standardization, one or more sources of radioactive material; provided that:

1. Each source contains no more than one exempt quantity set forth in Schedule RHS 8–3;

2. Each instrument contains no more than 10 exempt quantities. For purposes of this subparagraph (k), an instrument's source(s) may contain either one type or different types of radionuclides and an individual exempt quantity may be composed of fractional parts of one or more of the exempt quantities in Schedule RHS 8–3, provided that the sum of such fractions shall not exceed unity; and

3. For purposes of this subparagraph (k), 0.05 microcuries of americium–241 is considered an exempt quantity under Schedule RHS 8–3.

(l) Spark gap irradiators containing not more than 1 microcurie of cobalt–60 per spark gap irradiator for use in electrically ignited fuel oil burners having a firing rate of at least 3 gallons per hour (11.4 liters per hour).

(3) Exempt quantities.

(a) Except as provided in (c) and (d) of this paragraph, any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns, or acquires radioactive material in individual quantities each of which does not exceed the applicable quantity set forth in Schedule RHS 8–3; however, these quantities shall not be administered in any form to human beings internally or externally for any purpose.

(b) Any person who possesses radioactive material received or acquired under the general license formerly provided in subparagraph RHS 7.203 A.2. is exempt from the requirements for a license set forth in this chapter to the extent that such person possesses, uses, transfers or owns such radioactive material. Such exemption does not apply for radium–226.

(c) This paragraph (3) does not authorize the production, packaging or repackaging of radioactive material for purposes of commercial distribution, or the incorporation of radioactive material into products intended for commercial distribution.

(d) No person may, for purposes of commercial distribution, transfer radioactive material in the individual quantities set forth in Schedule RHS 8–3, knowing or having reason to believe that such quantities of radioactive material will be transferred to persons exempt under this paragraph or equivalent regulations of the U.S. Nuclear Regulatory Commission, any Agreement State or Licensing State, except in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission pursuant to Section 32.18 of 10 CFR Part 32 or by the Department pursuant to 1200–2–10–.13(14), which license states that the radioactive material may be transferred by the licensee to persons exempt under this paragraph (3) or the equivalent regulations of the U.S. Nuclear Regulatory Commission, an Agreement State or Licensing State. [6]

Deleted: State. [2]

(4) Capsules containing carbon–14 urea for 'in vivo' diagnostic use for humans.

(a) Except as provided in subparagraphs (4)(b) and (c) below, any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns or acquires capsules containing 1 microcurie (37 kilobecquerels) carbon–14 urea (allowing for nominal variation that may occur during the manufacturing process) each, for 'in vivo' diagnostic use for humans.

(b) Any person who desires to use the capsules for research involving human subjects shall apply for and receive a specific license under Chapter 1200–2–10.

(c) Any person who desires to manufacture, prepare, process, produce, package, repackage, or transfer for commercial distribution such capsules shall apply for and receive a specific license pursuant to 10 CFR 32.21.

(d) Nothing in this section relieves persons from complying with applicable FDA, other Federal and State requirements governing receipt, administration and use of drugs.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed June 5, 1991; effective September 28, 1991. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–10–.05 RESERVED.

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[6] *Authority to transfer possession or control by the manufacturer, processor or producer of any equipment, device, commodity or other product containing byproduct material, whose subsequent possession, use, transfer and disposal by all other persons are exempted from regulatory requirements may be obtained only from the U.S. Nuclear Regulatory Commission, Washington, DC 20555.*

1200–2–10–.06 EXEMPTIONS: U.S. DEPARTMENT OF ENERGY AND U.S. NUCLEAR REGULATORY COMMISSION CONTRACTORS.

Any contractor or subcontractor of the U.S. Department of Energy (DOE) or the U.S. Nuclear Regulatory Commission (NRC) of the following categories operating within this State is exempt from these regulations to the extent that such contractor or subcontractor under his contract receives, possesses, uses, transfers or acquires sources of radiation:

(1) Prime contractors performing work for DOE at U.S. Government–owned or controlled sites including the transportation of sources of radiation to or from such sites and the performance of contract services during temporary interruption of such transportation;

(2) Prime contractors of DOE performing research in, or development, manufacture, storage, testing or transportation of atomic weapons or components thereof;

(3) Prime contractors of DOE using or operating nuclear reactors or other nuclear devices in the U.S. Government–owned vehicle or vessel; and

(4) Any other prime contractor or subcontractor of DOE or NRC when the State and NRC jointly determine (1) that, under the terms of the contract or subcontract, there is assurance that the work there under can be accomplished with protection of the public health and safety and (2) that, the exemption of such contractor or subcontractor is authorized by law.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.07 OTHER EXEMPTIONS.

(1) The following machines and equipment are exempt from these regulations:

(a) Domestic television receivers, providing the exposure rate at 5 centimeters from any outer surface is less than 0.5 milliroentgen per hour.

(b) Other electrical equipment that produces radiation incidental to its operation for other purposes, providing the dose rate to the whole body at the point of nearest approach to such equipment when any external shielding is removed does not exceed 0.5 rem per year. The production testing or factory servicing for such equipment shall not be exempt.

(c) Radiation producing machines while in transit or storage incident thereto.

(d) Radiation machines that are totally unusable except for salvage parts.

(2) Equipment described in paragraph (1) of this rule shall not be exempt if it is used or handled in such a manner that any individual might receive a dose of radiation in excess of the limits specified in these regulations.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.08 TYPES OF LICENSES.

~~Licenses for radioactive materials are of two types:~~

~~(1) General licenses provided in this chapter are effective without the filing of applications with the Division or the issuance of licensing documents to particular persons.~~

~~(2) Specific licenses are issued to named persons upon applications filed pursuant to this chapter.~~

(1) Licenses for radioactive materials are of two types:

(a) General licenses provided for in this chapter are effective without the filing of applications with the Division or the issuance of licensing documents to particular persons; however, the Division will require reporting of devices covered by the particular general license in accordance with 1200-2-10-.10(2)(c)13.

(b) Specific licenses are issued to named persons upon applications filed pursuant to this chapter.

(2) Reserved.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–10–.09 GENERAL LICENSES – SOURCE MATERIAL.

(1) A general license is hereby issued authorizing receipt, possession, use and transfer of not more than fifteen (15) pounds (6,803.89 grams) of source material at any one time:

(a) To commercial and industrial firms, research, educational and medical institutions and State and local government agencies, for research, development, educational, commercial, or operational purposes;

(b) Persons who receive, possess, use or transfer source material pursuant to the general license in this paragraph are prohibited from administering source material, or the radiation there from, either externally or internally, to human beings except as authorized by the Division in a specific license.

(c) Provided, that no such person shall, pursuant to this general license, receive more than a total of 150 pounds (68,038.90 grams) of source material in any one calendar year.

(d) Persons who receive, possess, use or transfer source material pursuant to the general license issued in (1) of this rule are exempt from the provisions of 1200–2–5 of these regulations to the extent that such receipt, possession, use or transfer is within the terms of such general license; provided, however, that this exemption shall not be deemed to apply to any such person who is also in possession of source material under a specific license issued pursuant to this chapter.

(2) A general license is hereby issued authorizing the receipt of title to source material without regard to quantity. The general license under this paragraph does not authorize any person to receive, possess, use or transfer source material.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.10 GENERAL LICENSES[7] – RADIOACTIVE MATERIAL OTHER THAN SOURCE MATERIAL.

(1) A general license is hereby issued to receive, acquire, own, possess, use and transfer radioactive material incorporated in a device or equipment that is listed in Schedule RHS 8–5 and has been manufactured pursuant to a specific license or equivalent licensing document, issued by the Division, the U.S. Nuclear Regulatory Commission, or any Agreement State and authorizing distribution under the general license of this paragraph or its equivalent.

~~(2) Certain measuring, gauging or controlling devices.~~

~~(a) A general license is hereby issued to commercial and industrial firms and research, educational and medical institutions, individuals in the conduct of their business, and State and local governmental agencies to acquire, receive, possess, use or transfer, in accordance with the provisions of (b), (c) and (d) of this paragraph, radioactive material contained in devices designed and manufactured for the purpose of detecting, measuring, gauging or controlling thickness, density, level, interface location, radiation leakage, or qualitative or quantitative chemical composition, or for producing light or an ionized atmosphere, when such devices are manufactured and labeled in accordance with the specifications contained in a specific license issued by the Division pursuant to 1200–2–10–.13(4) or in accordance with the specifications contained in a specific license issued by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State that authorizes distribution of devices to persons generally licensed by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State.~~

~~(b) Persons who own, receive, acquire, possess, use or transfer radioactive material in a device pursuant to the general license contained in (a) of this paragraph (2):~~

~~1. Shall assure that all labels affixed to the device at the time of receipt and bearing the statement that removal of the label is prohibited are maintained thereon and shall comply with all instructions and precautions provided by such labels;~~

~~2. Shall assure that the device is tested for leakage of radioactive material and proper operation of the on–off mechanism and indicator, if any, at no longer than six–month intervals or at such other intervals as are specified in the label; however,~~

~~(i) Devices containing only krypton need not be tested for leakage of radioactive material, and~~

~~(ii) Devices containing only tritium or not more than 100 microcuries of other beta and/or gamma emitting material or 10 microcuries of alpha emitting material and devices held in storage in the original shipping container prior to initial installation need not be tested for any purpose;~~

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[7] *Different general licenses are issued in this rule, each of which has its own specific conditions and requirements.*

3. Shall assure that the tests required by 2. of this subparagraph and other testing, installation, servicing, and removal from installation, involving the radioactive material, its shielding or containment, are performed.

(i) In accordance with the instructions provided by the labels, or

(ii) By a person holding an applicable specific license issued by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to perform such activities.

4. Shall maintain records showing compliance with the requirements of 2. and 3. of this subparagraph. The records shall show the results of tests. The records also shall show the dates of performance of, and the names of persons performing, testing, installation, servicing and removal from installation concerning the radioactive material, its shielding or containment;

5. Upon the occurrence of a failure of or damage to, or any indication of a possible failure of or damage to, the shielding of the radioactive material or the on–off mechanism or indicator, or upon the detection of 0.005 microcurie or more removable radioactive material, shall immediately suspend operation of the device until it has been repaired by the manufacturer or other person holding an applicable specific license issued by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to repair such devices, or disposed of by transfer to a person holding an applicable specific license to receive the radioactive material contained in the device and, within 30 days furnish to the Division a report containing a brief description of the event and the remedial action taken;

6. Shall not abandon the device containing radioactive material;

7. Except as provided in 8. of this subparagraph, shall transfer or dispose of the devices containing radioactive material only by transfer to a licensee of the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State, whose specific license authorizes him to receive the device and within 30 days after transfer of a device to a specific licensee shall furnish to the Division a report containing identification of the device by manufacturer's name and model number and the name and address of the person receiving the device. No report is required if the device is transferred to the specific licensee in order to obtain a replacement device;

8. Shall transfer the devices to another general licensee only:

(i) Where the device remains in use at a particular location. In such case the transferor shall give the transferee a copy of this paragraph (2) and any safety documents identified in the label of the device and within 30 days of the transfer, report to the Division the manufacturer's name and model number of the device transferred, the name and address of the transferee, and the name and/or position of an individual who may constitute a point of contact between the Division and the transferee; or

(ii) Where the device is held in storage in the original shipping container at its intended location of use prior to initial use by a general licensee;

9. Shall comply with the provisions of 1200–2–5–.140 and 1200–2–5–.141 for reporting radiation incidents, theft or loss of radioactive material.

~~(c) The general license provided in this paragraph is subject to the provisions of 1200–2–10–.16(1), (2) and (3), 1200–2–10–.23(1), (2) and (3), 1200–2–10–.26 through 1200–2–10–.28, and 1200–2–10–.30.~~

~~(d) The general license in 1200–2–10–.10(2)(a) does not authorize the manufacture of devices containing radioactive material.~~

(2) Certain detecting, measuring, gauging or controlling devices and certain devices for producing light or an ionized atmosphere.[8]

(a) A general license is hereby issued to commercial and industrial firms and research, educational and medical institutions, individuals in the conduct of their business and State or local government agencies to own, acquire, receive, possess, use or transfer, in accordance with the provisions of (b), (c) and (d) of this paragraph, radioactive material contained in devices designed and manufactured for the purpose of detecting, measuring, gauging or controlling thickness, density, level, interface location, radiation, leakage, or qualitative or quantitative chemical composition, or for producing light or an ionized atmosphere.

(b) 1. The general license in subparagraph (a) applies only to radioactive material contained in devices that have been manufactured or initially transferred and labeled in accordance with the specifications contained in:

(i) A specific license issued by the Division pursuant to 1200–2–10–.13(4) or

(ii) A specific license issued by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State that authorizes distribution of devices to persons generally licensed by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State.

2. The devices shall have been received from one of the above licensees or through a transfer made under part (2)(c)9.

(c) Persons who own, acquire, receive, possess, use or transfer radioactive material in a device pursuant to the general license contained in subparagraph (2)(a):

1. Shall assure that all labels affixed to the device at the time of receipt and bearing the statement that removal of the label is prohibited are maintained thereon and shall comply with all instructions and precautions provided by such labels;

2. Shall assure that the device is tested for leakage of radioactive material and proper operation of the on–off mechanism and indicator, if any, at no longer than six–month intervals or at such other intervals as are specified in the label; however,

(i) Devices containing only krypton need not be tested for leakage of radioactive material, and

(ii) Devices containing only tritium or not more than 100 microcuries of other beta and/or gamma emitting material or 10 microcuries of alpha

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[8] *Persons possessing radioactive material in devices under the general license in 1200–2–10–.10(2) before October 2, 1978, may continue to possess, use or transfer that material in accordance with the requirements in the 1972 edition of the regulations.*

emitting material and devices held in storage in the original shipping container prior to initial installation need not be tested for any purpose.

3. Shall assure that the tests required by part (2)(c)2. and other testing, installation, servicing and removal from installation involving the radioactive material, its shielding or containment, are performed:

   (i) In accordance with the instructions provided by the labels, or

   (ii) By a person holding an applicable specific license issued by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to perform such activities.

4. Shall maintain records showing compliance with the requirements of parts (2)(c)2. and (c)3. The records shall show the results of tests. The records also shall show the dates of performance of and the names of persons performing testing, installation, servicing and removal from installation of the radioactive material, its shielding or containment. The licensee shall retain these records as follows:

   (i) Each record of a test for leakage or radioactive material required by part (2)(c)2. shall be retained for three (3) years after the next required leak test is performed or until the sealed source is transferred or disposed of.

   (ii) Each record of a test of the on–off mechanism and indicator required by part (2)(c)2. shall be retained for three (3) years after the next required test of the on–off mechanism and indicator is performed or until the sealed source is transferred or disposed of.

   (iii) Each record that is required by part (2)(c)3. shall be retained for three (3) years from the date of the recorded event or until the sealed source is transferred or disposed of.

5. Shall immediately suspend operation of the device if there is a failure of or damage to, or any indication of a possible failure of or damage to, the shielding of the radioactive material or the on–off mechanism or indicator, or upon the detection of 0.005 microcurie (185 becquerel) or more removable radioactive material. The device may not be operated until it has been repaired by the manufacturer or other person holding an applicable specific license issued by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to repair such devices. The device and any radioactive material from the device may only be disposed of by transfer to a person holding an applicable specific license to receive the radioactive material contained in the device or as otherwise approved by the Division. The licensee shall within 30 days furnish to the Division at the address in Rule 1200–2–4–.07 a report containing a brief description of the event and the remedial action taken. In the case of detection of 0.005 microcurie or more removable radioactive material or failure of or damage to a source likely to result in contamination of the premises or the environs, the licensee shall within 30 days submit to the Division at the address in 1200–2–4–.07 a plan for ensuring that the premises and environs are acceptable for unrestricted use. Under these circumstances, the criteria set out in paragraph 1200–2–10–.36(2), "Radiological criteria for unrestricted use," may be applicable, as determined by the Division on a case–by–case basis;

6. Shall not abandon the device containing radioactive material;

7. Shall not export the device containing radioactive material except in accordance with 10 CFR 110;

8. Shall:

(i) Transfer or dispose of the device containing radioactive material only by export as provided by part (2)(c)7., by transfer to another general licensee as authorized in part (c)9. of this paragraph, or to a person authorized to receive the device by a specific license issued by the Division under this chapter or an equivalent license issued by the U.S. Nuclear Regulatory Commission or an Agreement State, or as otherwise approved under subpart (2)(c)8.(iii) below.

(ii) Shall within 30 days after the transfer of a device to a specific licensee or export furnish a report to the Division. The report shall contain:

(I) The identification of the device by manufacturer's (or initial transferor's) name, model number and serial number;

(II) The name, address and license number of the person receiving the device (license number not applicable if exported); and

(III) The date of the transfer.

(iii) Shall obtain written Division approval before transferring the device to any other specific licensee not specifically identified in subpart (2)(c)8.(i).

9. Shall transfer the device to another general licensee only if:

(i) The device remains in use at a particular location. In this case, the transferor shall give the transferee a copy of this paragraph (2) and any safety documents identified in the label of the device. Within 30 days of the transfer, the transferor shall report to the Division:

(I) The manufacturer's (or initial transferor's) name;

(II) The model number and the serial number of the device transferred;

(III) The transferee's name and mailing address for the location of use; and

(IV) The name, title and phone number of the responsible individual identified by the transferee in accordance with part (2)(c)12. to have knowledge of and authority to take actions to ensure compliance with the appropriate regulations and requirements; or

(ii) The device is held in storage by an intermediate person in the original shipping container at its intended location of use prior to initial use by a general licensee;

10. Shall comply with the provisions of 1200–2–5–.140 and 1200–2–5–.141 for reporting radiation incidents, theft or loss of radioactive material;

11. Shall respond to written requests from the Division to provide information relating to the general license within 30 calendar days of the date of the request, or other time specified in the request. If the general licensee cannot provide the requested information within the allotted time, it shall, within that same time period, request a longer period to supply the information by submitting a letter to the Division at the address in Rule 1200–2–4–.07 providing written justification as to why it cannot comply;

12. Shall appoint an individual responsible for having knowledge of the appropriate regulations and requirements and the authority for taking required actions to comply with appropriate regulations and requirements. The general licensee, through this individual, shall ensure the day–to–day compliance with appropriate regulations and requirements. This appointment does not relieve the general licensee of any of its responsibility in this regard;

13. Shall:

(i) Report these devices annually to the Division and shall pay the fee required by 1200-2-10-.31. Reporting shall be done by verifying, correcting and/or adding to the information provided in a request for a report received from the Division. The report information shall be submitted to the Division within 30 days of the date of the request or as otherwise indicated in the request.

(ii) In reporting devices, furnish the following information and any other information specifically requested by the Division:

(I) Name and mailing address of the general licensee;

(II) Information about each device: the manufacturer (or initial transferor), model number, serial number, the radioisotope and activity (as indicated on the label);

(III) Name, title and telephone number of the responsible person designated as a representative of the general licensee under part (c)12;

(IV) Address or location at which the device(s) are used and/or stored. For portable devices, the address of the primary place of storage. Each address for a location of use represents a separate general license;

(V) Certification by the responsible representative of the general licensee that the information concerning the device(s) has been verified through a physical inventory and checking of label information;

(VI) Certification by the responsible representative of the general licensee that they are aware of the requirements of the general license.

14. Shall be subject to the bankruptcy notification requirement in paragraph 1200–2–10–.16(7) if holding devices containing radioactive material that meet the following criteria, based on the activity indicated on the label:

(i) At least 10 mCi (370MBq) of cesium–137;

(ii) At least 0.1 mCi (3.7 MBq) of strontium–90;

(iii) At least 1 mCi (37 MBq) of cobalt–60; or

(iv) At least 1 mCi (37 MBq) of americium–241 or any other transuranic (i.e., element with atomic number greater than uranium (92))

15. Persons generally licensed by the U.S. Nuclear Regulatory Commission or an Agreement State with respect to devices meeting the criteria in part (2)(c)13 and (2)(c)14 of this paragraph are not subject to reporting requirements if the devices are used in areas subject to the Division's jurisdiction for a period less than 180 days in any calendar year. The Division will not request reporting information from such licensees.

16. Shall report changes to the mailing address for the location of use (including change in name of general licensee) to the Division, at the address in 1200–2–4–.07, within 30 days of the effective date of the change. For a portable device, a report of address change is only required for a change in the device's primary place of storage;

17. Shall not hold devices that are not in use for longer than two (2) years. If devices with shutters are not being used, the shutter shall be locked in the closed position. The testing required by part (c)2. need not be performed during the period of storage only. However, when devices are put back into service or transferred to another person and have not been tested within the required test interval, they shall be tested for leakage before use or transfer and the shutter tested before use. Devices kept in standby for future use are excluded from the two–year time limit if the general licensee performs quarterly physical inventories of these devices while they are in storage;

(d) The general license provided in this paragraph is subject to the provisions of 1200–2–10–.16(1), (2) and (3), 1200–2–10–.23(1), (2) and (3), 1200–2–10–.26 through 1200–2–10–.28 and 1200–2–10–.30.

(e) The general license in 1200–2–10–.10(2)(a) does not authorize the manufacture or import of devices containing radioactive material.

(3) Luminous safety devices for aircraft.

(a) A general license is hereby issued to own, receive, acquire, possess and use tritium or promethium–147 contained in luminous safety devices for use in aircraft, provided:

1. Each device contains not more than ten (10) curies of tritium or 300 millicuries of promethium–147; and

2. Each device has been manufactured, assembled or imported in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission, or each device has been manufactured or assembled in accordance with the specifications contained in a specific license or equivalent licensing document issued by the Division or an Agreement State to the manufacturer or assembler of such device pursuant to licensing requirements equivalent to those in Section 32.53 of 10 CFR Part 32.

(b) Persons who own, receive, acquire, possess or use luminous safety devices pursuant to the general license in (a) of this paragraph (3) are exempt from the requirements of Chapter 1200–2–5, except that they shall comply with the provisions of 1200–2–5–.140 and 1200–2–5–.141.

(c) This general license does not authorize the manufacture, assembly or repair of luminous safety devices containing tritium or promethium–147.

(d) This general license does not authorize the ownership, receipt, acquisition, possession or use of promethium–147 contained in instrument dials.

(e) The general license provided in this paragraph is subject to the provisions of 1200–2–10–.16 through 1200–2–10–.30, as applicable.

(4) Calibration and reference sources.

(a) A general license is hereby issued to those persons listed below to own, receive, acquire, possess, use and transfer, in accordance with the provisions of (d) and (e) of this paragraph (4), americium–241 in the form of calibration or reference sources:

1. Any person who holds a specific license issued by the Division that authorizes the receipt, possession, use and transfer of radioactive materials; and

2. Any person who holds a specific license issued by the U.S. Nuclear Regulatory Commission that authorizes the receipt, possessions, use and transfer of special nuclear material.

(b) A general license is hereby issued to receive, possess, use and transfer plutonium in the form of calibration or reference sources in accordance with the provisions of (d) and (e) of this paragraph (4) to any person who holds a specific license issued by the Division that authorizes him to receive, possess, use and transfer radioactive material.

(c) A general license is hereby issued to own, receive, possess, use and transfer radium–226 in the form of calibration or reference sources in accordance with the provisions of (d) and (e) of this paragraph to any person who holds a specific license issued by the Division that authorizes him to receive, possess, use, and transfer radioactive material.

(d) The general licenses in (a), (b) and (c) of this paragraph apply only to calibration or reference sources that have been manufactured in accordance with the specifications contained n a specific license issued to the manufacturer or importer of the sources by the U.S. Nuclear Regulatory Commission pursuant to Section 32.57 of 10 CFR, Part 32 or Section 70.39 of 10 CFR, Part 70 or that have been manufactured in accordance with the specifications contained in a specific license issued to the manufacturer by the Division or any Agreement State or Licensing State pursuant to licensing requirements equivalent to those contained in Section 32.57 of 10 CFR, Part 32 or Section 70.39 of 10 CFR, Part 70.

(e) The general licenses provided in (a), (b) and (c) of this paragraph are subject to the provisions of 1200–2–10–.16, 1200–2–10–.22, 1200–2–10–.23, 1200–2–10–.26, 1200–2–10–.27, 1200–2–10–.28, 1200–2–10–.30, and Chapters 1200–2–4 and 1200–2–5 of these regulations. In addition, persons who own, receive, acquire, possess, use and transfer one or more calibration or reference sources pursuant to these general licenses:

1. Shall not possess at any one time, at any one location of storage or use, more than 5 microcuries of americium–241, 5 microcuries of plutonium or 5 microcuries of radium–226 in such sources;

2. Shall not receive, possess, use or transfer such source unless the source, or the storage container, bears a label that includes one of the following statements, as appropriate, or a similar statement that contains the information called for in one of the following statements, as appropriate:

   (i) The receipt, possession, use and transfer of this source, Model_____, Serial No._____, are subject to a general license and the regulations of the U.S. Nuclear Regulatory Commission or of a state with which the Commission has entered into an agreement for the exercise of regulatory authority. Do not remove this label.

   CAUTION – RADIOACTIVE MATERIAL – THIS SOURCE CONTAINS (AMERICIUM–241) (PLUTONIUM)[9]. DO NOT TOUCH RADIOACTIVE PORTION OF THIS SOURCE

   _____
   (name of manufacturer or importer)

   (ii) The receipt, possession, use and transfer of this source, Model_____, Serial No._____, are subject to a general license and the regulations of a Licensing State. Do not remove this label.

   CAUTION – RADIOACTIVE MATERIAL – THIS SOURCE CONTAINS RADIUM–226. DO NOT TOUCH RADIOACTIVE PORTION OF THIS SOURCE

   _____
   (name of manufacturer or importer)

3. Shall not transfer, abandon or dispose of such source except by transfer to a person authorized by a license from the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to receive the source;

4. Shall store such source, except when the source is being used, in a closed container adequately designed and constructed to contain americium–241, plutonium or radium–226 that might otherwise escape during storage; and

5. Shall not use such source for any purpose other than the calibration of radiation detectors or the standardization of other sources.

(f) These general licenses do not authorize the manufacture of calibration or reference sources containing americium–241, plutonium or radium–226.

(5) Ownership of radioactive material. A general license is hereby issued to own radioactive material without regard to quantity. Notwithstanding any other provisions of this chapter, this general license does not authorize the manufacture, production, transfer, receipt, possession or use of radioactive material.

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[9] *Showing only the name of the appropriate material.*

(6) Ice detection devices.

(a) A general license is hereby issued to own, receive, acquire, possess, use and transfer strontium–90 contained in ice detection devices, provided each device contains not more than fifty microcuries of strontium–90 and each device has been manufactured or imported in accordance with a specific license issued by the U.S. Nuclear Regulatory Commission or each device has been manufactured in accordance with the specifications contained in a specific license or equivalent licensing document issued by the Division or any Agreement State to the manufacturer of such device pursuant to licensing requirements equivalent to those in Section 32.61 of 10 CFR, Part 32 of the regulations of the U.S. Nuclear Regulatory Commission.

(b) Persons who own, receive, acquire, possess, use or transfer strontium–90 contained in ice detection devices pursuant to the general license in (a) of this paragraph (6):

1. Shall, upon occurrence of visually observable damage, such as a bend or crack or discoloration from overheating to the device, discontinue use of the device until it has been inspected, tested for leakage and repaired by a person holding a specific license or equivalent licensing document from the U.S. Nuclear Regulatory Commission or an Agreement State to manufacture or service such devices; or shall dispose of the device pursuant to the provisions of these regulations;

2. Shall assure that all labels affixed to the device at the time of receipt, and that bear a statement that prohibits removal of the labels, are maintained thereon;

3. Are exempt from the requirements of Chapter 1200–2–5 of these regulations except that such persons shall comply with the provisions of 1200–2–5–.120(1) and 1200–2–5–.140 and 1200–2–5–.141.

(c) This general license does not authorize the manufacture, assembly, disassembly or repair of strontium–90 sources in ice detection devices.

(d) The general license provided in this paragraph is subject to the provisions of 1200–2–10–.16, 1200–2–10–.22, 1200–2–10–.23, 1200–2–10–.26, 1200–2–10–.27, 1200–2–10–.28, 1200–2–10–.30.

(7) Radioactive material for certain in vitro clinical or laboratory testing.

(a) A general license is hereby issued to any physician, veterinarian in the practice of veterinary medicine, clinical laboratory or hospital to receive, acquire, possess, transfer or use, for any of the following stated tests, in accordance with the provisions of (b), (c), (d), (e) and (f) of this paragraph (7), the following radioactive materials in prepackaged units for use in in vitro clinical or laboratory tests not involving internal or external administration of radioactive material, or the radiation there from, to human beings or animals:

1. Iodine–125, in units not exceeding 10 microcuries each.

2. Iodine–131, in units not exceeding 10 microcuries each.

3. Carbon–14, in units not exceeding 10 microcuries each.

4. Hydrogen–3 (tritium), in units not exceeding 50 microcuries each.

5. Iron–59, in units not exceeding 20 microcuries each.

6. Cobalt–57, in units not exceeding 10 microcuries each.

7. Selenium–75, in units not exceeding 10 microcuries each.

8. Mock Iodine–125 reference or calibration sources, in units not exceeding 0.05 microcurie of iodine–129 and 0.005 microcurie of americium–241 each.

(b) No person shall receive, acquire, possess, use or transfer radioactive material pursuant to the general license established by (a) of this paragraph (7) until he has filed an application for and received from the Division a copy of Form RHS 8–5I with number assigned. The general licensee shall furnish on the application the following information and such other information as may be required by that form:

1. Name and address of the licensee;

2. The location of use; and

3. A statement that the licensee has appropriate radiation measuring instruments to carry out in vitro clinical or laboratory tests with radioactive materials as authorized under this general license and that such tests will be performed only by personnel competent in the use of such instruments and in the handling of the radioactive materials.

(c) A person who receives, acquires, possesses or uses radioactive material pursuant to this general license shall comply with the following:

1. The general licensee shall not possess at any one time, pursuant to this general license, at any one location of storage or use, a total amount of iodine–125, iodine–131, cobalt–57, selenium–75 and/or iron–59 in excess of 200 microcuries.

2. The general licensee shall store the radioactive material, until used, in the original shipping container or in a container providing equivalent radiation protection.

3. The general licensee shall use the radioactive material only for the uses authorized by (a) of this paragraph (7).

4. The general licensee shall not transfer the radioactive material except by transfer to a person authorized to receive it by a license pursuant to this Chapter 1200–2–10, from the U.S. Nuclear Regulatory Commission, or an Agreement State or Licensing State nor transfer the radioactive material in any manner other than in the unopened, labeled shipping container as received from the supplier.

5. The general licensee shall dispose of the Mock Iodine 125 reference or calibration sources described in (a) of this paragraph (7) as required by 1200–2–5–.120.

(d) The general licensee shall not receive, acquire, possess or use radioactive material pursuant to (a) of this paragraph (7):

1. Except as prepackaged units that are labeled in accordance with the provisions of a specific license issued by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or Licensing State that authorizes manufacture and distribution of iodine–125, iodine–131, carbon–14, hydrogen–3 (tritium), selenium–75, cobalt–57, iron–59, or Mock Iodine–125 to persons generally licensed; and

2. Unless one of the following statements, as appropriate, or a substantially similar statement that contains the information called for in one of the following statements, appears on a label affixed to each prepackaged unit or appears in a leaflet or brochure that accompanies the package:

   (i) This radioactive material may be received, acquired, possessed and used only by physicians, veterinarians in the practice of veterinary medicine, clinical laboratories or hospitals and only for in vitro clinical or laboratory tests not involving internal or external administration of the material, or the radiation there from, to human beings or animals. Its receipt, acquisition, possession, use and transfer are subject to the regulations and a general license of the U.S. Nuclear Regulatory Commission or of a State with which the Commission has entered into an agreement for the exercise of regulatory authority.

   _____
   (name of manufacturer)

   (ii) This radioactive material may be received, acquired, possessed, and used only by physicians, veterinarians in the practice of veterinary medicine, clinical laboratories or hospitals and only for *in vitro* clinical or laboratory tests not involving internal or external administration of the material, or the radiation there from, to human beings or animals. Its receipt, acquisition, possession, use and transfer are subject to the regulations and a general license of a Licensing State.

   _____
   (name of manufacturer)

(e) Licensees possessing or using radioactive materials under this general license shall report in writing to the Director, Division of Radiological Health, at the address in Rule 1200–2–4–.07, any changes in the information furnished in the application submitted under subparagraph 1200-2-10-.10(7)(b). The report shall be furnished within 30 days after the effective date of such change.

(f) Any person using radioactive material pursuant to this general license is exempt from the requirements of Chapter 1200–2–5 with respect to radioactive materials covered by this general license, except that such person using the Mock Iodine 125 described in part (a)8. shall comply with the provisions of 1200–2–5–.120, 1200–2–5–.140, and 1200–2–5–.141.

(8) Reserved.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006. Amendment filed March 12, 2007; effective May 26, 2007.*

Deleted: Capsules containing carbon–14 urea for 'in vivo' diagnostic use for humans.¶
(a) . Except as provided in subparagraphs (8)(b) and (c) below, any person is exempt from these regulations to the extent that such person receives, possesses, uses, transfers, owns or acquires capsules containing 1 microcurie (37 kilobecquerels) carbon–14 urea (allowing for nominal variation that may occur during the manufacturing process) each, for 'in vivo' diagnostic use for humans.¶
(b) . Any person who desires to use the capsules for research involving human subjects shall apply for and receive a specific license under Chapter 1200–2–10.¶
(c) . Any person who desires to manufacture, prepare, process, produce, package, repackage, or transfer for commercial distribution such capsules shall apply for and receive a specific license pursuant to 10 CFR 32.21.¶
(d) . Nothing in this section relieves persons from complying with applicable FDA, other Federal and State requirements governing receipt, administration and use of drugs.

1200–2–10–.11 FILING OF APPLICATION FOR SPECIFIC LICENSES.

(1) Application for specific licenses shall be filed in duplicate on a form prescribed by the Division.

(2) The Division may at any time after the filing of the original application, and before the expiration of the license, require further statements in order to enable the Division to determine whether the application should be granted or denied or whether a license should be modified or revoked.

(3) Each application shall be signed by the applicant or licensee or a person duly authorized to act for and on his behalf.

(4) An application for a license may include a request for a license authorizing one or more activities.

(5) In his application, the applicant may incorporate by reference information contained in previous applications, statements or reports filed with the Division provided such references are specific.

(6) Applications and documents submitted to the Division may be made available for public inspection except that the Division may withhold any document or part thereof from public inspection if disclosure of its contents involves proprietary information.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.12 GENERAL REQUIREMENTS FOR THE ISSUANCE OF SPECIFIC LICENSES.

A license application will be approved if the Division determines that:

(1) The applicant has properly trained a sufficient number of personnel to use the material in question for the purpose required in accordance with these regulations in such a manner as to protect the public health and safety or property;

(2) The applicant's proposed equipment, facilities and procedures are in good repair and working order and designed to protect the public health and safety or property;

(3) The applicant satisfies all applicable requirements of these regulations;

(4) The applicant or an existing licensee in any of the classes specified in (a) of this paragraph and not otherwise specifically exempted by (m) of this paragraph has provided financial assurance as herein specified. (See (6) of this rule for definitions of terms used in this paragraph.)

(a) Classes for financial assurance:

1. Major processors

2. Waste handlers

3. Ore refineries

4. Former USAEC or USNRC licensed facilities

5. Other persons with or applicants for a specific license as determined by the Commissioner.

(b) The financial assurance shall be filed with and maintained by the Director, Division of Radiological Health (hereafter referred to as Director), in a dollar amount determined by the Commissioner as necessary to provide for the protection of public health and safety in the event of abandonment, insolvency or other inability of the licensee to perform to the satisfaction of the Commissioner. The Commissioner shall consider the following in making his determination of the financial assurance requirements for each individual applicant or licensee:

1. The probable extent of contamination through the use or possession of radioactive material at the facility or site and the probable cost of removal of such contamination to a level in conformance with prevailing national standards or guidelines. This consideration shall encompass all probable contaminating event associated with the licensee's methods or modes of operation;

2. The amount of possible off–site property damage caused by operation of the facility or site;

3. The cost of removal and disposal of sources of radiation that are or would be generated, stored, processed or otherwise present at the licensed facility or site; and

4. The costs involved in reclaiming the property on which the facility or site is located. For purposes of this part, "reclaiming" shall mean return of the property to a condition or state such that the property no longer presents a public health or safety hazard or threat to the environment.

(c) Each applicant or licensee of each facility to which it is applicable must file and maintain with the Director financial assurance for reclaiming the facility in accordance with the requirements of this subparagraph.

1. The applicant or licensee must choose from the financial assurance mechanisms as specified in (d) of this paragraph. (NOTE: See also (e), (f) and (g) of this paragraph.)

2. The applicant or licensee must file and maintain financial assurance in an amount at least equal to the current reclaiming cost estimate.

(i) Whenever the reclaiming cost estimate increases to an amount greater than the amount of financial assurance currently filed with the Director, the licensee must, within 60 days after the increase, file additional financial assurance at least equal to this increase.

(ii) Whenever the current reclaiming cost estimate decreases, and upon the written request of the licensee, the Commissioner shall, provided the decrease is validated, reduce the amount of financial assurance required for the facility to the amount of the current reclaiming cost estimate. Upon such occurrence, the Director shall, as appropriate considering the financial assurance mechanism(s) on file, either cause to be released to the licensee cash or collateral equal to this reduction or allow the licensee

to substitute for the mechanism(s) on file a new mechanism(s) in the reduced amount.

3. An applicant for a license must file the financial assurance instruments(s) before the license can be issued.

4. The financial assurance must be maintained by the applicant or licensee until the Commissioner releases the licensee from the requirements of this subparagraph, as specified in this part, or until the Commissioner orders forfeiture of the financial assurance as provided in 5. of this subparagraph.

5. The Commissioner may order that any financial assurance filed by a licensee pursuant to this subparagraph be forfeited to the State if the Commissioner determines that the licensee has failed to perform reclaiming in a manner deemed acceptable by the Commissioner to assure health and safety from radiation hazards and other license requirements when required to do so. Any such forfeiture action shall follow the procedures provided in (h) of this paragraph.

(d) Mechanisms of financial assurance.

1. Surety Bond – An applicant or licensee may satisfy the requirements of (c) of this paragraph by obtaining and filing a surety bond that conforms to the requirements of this part.

(i) The surety company issuing the bond must be licensed to do business as a surety in Tennessee.

(ii) The wording of the surety bond must be identical to the wording specified in (j)1. of this paragraph.

(iii) The bond must guarantee that:

(I) Funds will be available to perform reclaiming in a manner deemed acceptable by the Commissioner to assure health and safety from radiation hazards and other requirements of the license for the facility whenever required to do so.

(II) The licensee will provide alternate financial assurance as specified in this paragraph and obtain the Director's written approval of the assurance provided within 90 days of receipt by both the licensee and the Director of a notice of cancellation of the bond from the surety.

(iv) Under the terms of the bond, the surety will become liable on the bond obligation when the licensee fails to perform as guaranteed by the bond. Following a determination by the Commissioner that the licensee has failed to so perform, under the terms of the bond the surety will perform reclamation to the satisfaction of the State as guaranteed by the bond or will forfeit the amount of the penal sum, as provided in (c)5. of this paragraph.

(v) The penal sum of the bond must be in an amount at least adequate to provide the necessary financial assurance.

(vi) Under the terms of the bond, the surety may cancel the bond by sending notice of cancellation by certified mail to the licensee and to the Director. Cancellation may not occur, however, during the 180 days beginning on the date of receipt of the notice of cancellation by both the licensee and the Director, as evidenced by the return receipts.

(vii) The surety will not be liable for deficiencies in the performance of reclaiming after the Commissioner releases the licensee from the financial assurance requirements as provided in (c)4. of this paragraph.

2. Personal Bond Supported by a Letter of Credit – An applicant or licensee may satisfy the requirements of (c) of this paragraph by filing his personal performance guarantee accompanied by collateral in the form of an irrevocable standby letter of credit. He must guarantee funds to perform reclaiming in accordance with acceptable practice for protection of health and safety and other requirements of the license for the facility. The irrevocable standby letter of credit supporting this guarantee must conform to the following requirements:

(i) The institution issuing the letter of credit must be an entity that has the authority to issue letter of credit and whose letter–of–credit operations are regulated and examined by a Federal or State agency.

(ii) The wording of the letter of credit must be identical to the wording specified in (j)2. of this paragraph.

(iii) The letter of credit must be accompanied by a letter from the licensee referring to the letter of credit by number, issuing institution and date and providing the following information: The radioactive material license number, name and address of the facility and the amount of funds assured for reclaiming of the facility by the letter of credit. (NOTE: This letter from the licensee may also contain his personal performance guarantee.)

(iv) The letter of credit must be irrevocable and issued for a period of at least one (1) year. The letter of credit must provide that the expiration date will be automatically extended for a period of at least one (1) year unless, at least 180 days before the current expiration date, the issuing institution notifies both the licensee and the Director by certified mail of a decision not to extend the expiration date. Under the terms of the letter of credit, the 180 days will begin on the date when both the licensee and the Director have received the notice, as evidenced by the return receipts.

(v) The letter of credit must be issued in an amount at least adequate to provide the necessary financial assurance.

(vi) The Commissioner may draw on the letter of credit upon forfeiture as provided in (c)5. of this paragraph. The Commissioner will also draw on the letter of credit if the licensee does not establish alternate financial assurance as specified in this paragraph and obtain written approval of such alternate assurance from the Director within 90 days after receipt by both the licensee and the Director of a notice from the issuing institution that it has decided not to extend the letter of credit beyond the current expiration date. The Director may delay the drawing if the issuing institution grants an extension of the term of the credit. During the last 30 days of any such extension the Commissioner will draw on the letter of credit if the licensee has failed to provided alternate financial

assurance as specified in this paragraph and obtain written approval of such assurance from the Director.

3. Personal Bond Supported by Insurance – An applicant or licensee may satisfy the requirements of (c) of this paragraph by filing his personal performance guarantee accompanied by collateral in the form of an insurance policy. He must guarantee funds sufficient to perform reclaiming in a manner deemed acceptable by the Commissioner for protection of health and safety and other requirements of the license for the facility. The insurance policy supporting this guarantee must conform to the following requirements:

(i) The insurer must be licensed to transact the business of insurance or eligible to provide insurance as an excess or surplus lines insurer in the State of Tennessee.

(ii) The insurance policy must be accompanied by a certificate of insurance whose wording is identical to the wording specified in (j)3. of this paragraph.

(iii) The insurance policy must be for a face amount at least adequate to provide the necessary financial assurance. The term "face amount" means the total amount the insurer is obligated to pay under the policy. Actual payments by the insurer will not change the face amount, although the insurer's future liability will be lowered by the amount of the payments.

(iv) The insurance policy must guarantee that funds will be available for reclaiming the facility whenever reclaiming is necessary.

(v) Upon forfeiture of financial assurance as provided in (c)5. of this paragraph, the Commissioner will direct the insurer to pay the full-face amount to the State.

(vi) The licensee must maintain the policy in full force and effect until the Commissioner releases the financial assurance mechanism as provided in this paragraph. Failure to pay the premium, without substitution of alternate financial assurance as specified in this paragraph, will constitute a significant violation of these regulations, warranting such remedy as the Commissioner deems necessary. Such violation will be deemed to begin upon receipt by the Director of a notice of future cancellation, termination or failure to renew due to nonpayment of the premium, rather than upon the date of expiration.

(vii) The policy must provide that the insurer may not cancel, terminate or fail to renew the policy except for failure to pay the premium. The automatic renewal of the policy must, at a minimum, provide the insured with the option of renewal at the face amount of the expiring policy. If there is a failure to pay the premium, the insurer may elect to cancel, terminate or fail to renew the policy by sending notice by certified mail to the licensee and the Director. Cancellation, termination or failure to renew may not occur, however, during the 180 days beginning with the date of receipt of the notice by both the Director and the licensee, as evidenced by the return receipts. Cancellation, termination or failure to renew may not occur and the policy will remain in full force and effect in the event that on or before the date of expiration:

(I) The Commissioner deems the facility abandoned;

(II) The license is terminated or revoked or renewal is denied;

(III) Closure is ordered by the Commissioner or a court of competent jurisdiction;

(IV) The licensee is named as debtor in a voluntary or involuntary proceeding under Title 11 (Bankruptcy), U.S. Code; or

(V) The premium due is paid.

(viii) Commencing on the date that liability to make payments pursuant to the policy accrues, the insurer will thereafter annually increase the face amount of the policy. Such increase must be equivalent to the face amount of the policy, less any payments made, multiplied by an amount equivalent to 85 percent (85%) of the most recent investment rate or of the equivalent coupon–issue yield announced by the U.S. Treasury for 26–week Treasury securities.

4. Personal Bond Supported by Securities – An applicant or licensee may satisfy the requirements of (c) of this paragraph by filing his personal performance guarantee accompanied by collateral in the form of securities. He must guarantee sufficient funds to perform reclaiming in accordance with acceptable practices for protection of health and safety and other requirements of the license for the facility. The securities supporting this guarantee must be fully registered as to principal and interest in such manner as to identify the State and the Division as holder of such collateral and to identify that person filing such collateral. These securities must have a current market value at least adequate to provide the necessary financial assurance and must be included among the following types:

(i) Negotiable certificates of deposit assigned irrevocably to the State.

(I) Such certificates of deposit must be automatically renewable and must be assigned to the State in writing and recorded as such in the records of the financial institution issuing such certificate.

(II) Such certificates of deposit must also include a statement signed by an officer of the issuing financial institution that waives all rights of lien that the institution has or might have against the certificate.

(ii) Negotiable United States Treasury securities assigned irrevocably to the State.

(iii) Negotiable general obligation municipal or corporate bonds that have at least an "A" rating by Moody's and/or Standard & Poor's rating services and that are assigned irrevocably to the State.

5. Personal Bond Supported by Cash – An applicant or licensee may satisfy the requirements of (c) of this paragraph by filing his personal performance guarantee accompanied by cash in an amount at least adequate to provide the necessary financial assurance.

6. Financial Test and Corporate Guarantee.

(i) An applicant or licensee may satisfy the requirements of (c) of this paragraph by demonstrating that he passes a financial test as specified in this part. To pass this test the licensee must meet the criteria of either item (I) or (II) of this subpart as follows:

(I) The applicant or licensee must have:

I. Two of the following three ratios: a ratio of total liabilities to net worth less than 2.0, a ratio of the sum of net income plus depreciation, depletion and amortization to total liabilities greater than 0.1, and a ratio of current assets to current liabilities greater than 1.5;

II. Net working capital and tangible net worth each at least six (6) times the current reclaiming cost estimate;

III. Tangible net worth of at least $10 million; and

IV. Assets in the United States amounting to at least 90 percent (90%) of this total assets or at least six (6) times the current reclaiming cost estimate.

(II) The applicant or licensee must have:

I. A current rating for his most recent bond issuance of AAA, AA, A or BBB as issued by Standard & Poor's, or Aaa, Aa, A or Baa as issued by Moody's;

II. Tangible net worth at least six (6) times the current reclaiming cost estimate;

III. Tangible net worth of at least $10 million; and

IV. Assets located in the United States amounting to at least 90 percent (90%) of his total assets or at least six (6) times the current reclaiming cost estimate.

(ii) The phrase "current reclaiming cost estimates" as used in (i) of this part refers to the cost estimates required to be shown in paragraphs 1–4 of the letter from the applicant's or licensee's chief financial officer.

(iii) To demonstrate that he meets this test, the applicant or licensee must submit the following items to the Director:

(I) A letter signed by the applicant's or licensee's chief financial officer and worded as specified in (j)4. of this paragraph;

(II) A copy of the independent certified public accountant's report on examination of the applicant's or licensee's financial statements for the latest completed fiscal year; and

(III) A special report from the applicant's or licensee's independent certified public accountant to the applicant or licensee stating that:

I. He has compared the data that the letter from the chief financial officer specifies as having been derived from the independently audited, year–end financial statements for the latest fiscal year with the amounts in such financial statements; and

II. In connection with that procedure, no matters came to his attention that caused him to believe that the specified data should be adjusted.

(iv) After the initial submission of items specified in (iii) of this part, the licensee must send updated information to the Director within 90 days after the close of each succeeding fiscal year. This information must consist of all three items specified in (iii) of this part.

(v) If the licensee no longer meets the requirements of (i) of this part, he must send notice to the Director of intent to establish alternate financial assurance as specified in this paragraph. The notice must be sent by certified mail within 90 days after the end of the fiscal year for which the year–end financial data show that the licensee no longer meets the requirements. The licensee must provide the alternate financial assurance within 120 days after the end of such fiscal year.

(vi) The Director may, based on a reasonable belief that the licensee may no longer meet the requirements of (i) of this part, require reports of financial condition at any time from the licensee in addition to those specified in (iii) of this part. If the Director finds, based on such reports or other information, that the licensee no longer meets the requirements of (i) of this part, the licensee must provide alternate financial assurance as specified in this paragraph within 30 days after notification of such a finding.

(vii) The Director may disallow use of this test based on qualifications in the opinion expressed by the independent certified public accountant in his report on examination of the applicant's or licensee's financial statements. An adverse opinion or a disclaimer of opinion will be cause for disallowance. The Director will evaluate other qualifications on an individual basis. The applicant or licensee must provide alternate financial assurance as specified in this paragraph within 30 days after notification of the disallowance.

(viii) An applicant or licensee may meet the requirements of (c) of this paragraph by obtaining a written guarantee, hereafter referred to as "corporate guarantee." The guarantor must be the parent corporation of the licensee. The guarantor must meet the requirements for applicants or licensees in (i) through (vii) of this part and must comply with the terms of the corporate guarantee. The wording of the corporate guarantee must be identical to the wording specified in (j)5. of this paragraph. The corporate guarantee must accompany the items sent to the Director as specified in (iii) of this part. The terms of the corporate guarantee must provide that:

(I) If the licensee fails to perform reclaiming of a facility covered by a corporate guarantee for reclaiming in accordance with acceptable practices to protect health and safety and other license requirements whenever required to do so, the guarantor will do so or forfeit to the State monies in an amount equal to the current reclaiming cost estimate for the facility, as provided in (c)5. of this paragraph.

(II) The corporate guarantee will remain in force unless the guarantor sends notice of cancellation by certified mail to the licensee and to the Director. Cancellation may not occur, however, during the 180 days beginning on the date of receipt of the notice of cancellation by both the licensee and the Director as evidenced by the return receipts.

(III) If the licensee fails to provide alternate financial assurance as specified in this paragraph and obtain the written approval of such alternate assurance from the Director within 90 days after receipt by both the licensee and the Director of a notice of cancellation of the corporate guarantee from the guarantor, the guarantor will provide such alternative financial assurance in the name of the licensee.

(e) Use of Multiple Financial Mechanisms – In meeting the requirements of (c) of this paragraph, an applicant or licensee may utilize more than one financial assurance mechanism per facility. These mechanisms are limited to personal bonds supported by letters of credit, insurance, securities or cash. The mechanisms must be as specified in (d) of this paragraph, except that it is the combination of mechanisms, rather than the single mechanism, that must provide financial assurance for the necessary amount.

(f) Use of a Financial Mechanism for Multiple Facilities – An applicant or licensee may use a financial assurance mechanism specified in (d) of this paragraph to meet the requirements of (c) of this paragraph for more than one facility he owns or operates in Tennessee. If so, the mechanism submitted to the Director must include a list showing, for each facility, the license number, name, address and amount of funds for reclaiming care assured by the mechanism. The amount of funds available through the mechanism must be no less than the sum of funds that would be available if a separate mechanism had been filed and maintained for each facility.

(g) Substituting Alternate Financial Assurance – In meeting the requirements of (c) of this paragraph, a licensee may substitute alternate financial assurance meeting the requirements of this paragraph for the financial assurance already filed with the Director for the facility. However, the existing financial assurance shall not be released by the Commissioner until the substitute financial assurance has been received and approved by him.

(h) Procedures for Forfeiture of Financial Assurance.

1. Upon the determination of abandonment, insolvency or other inability of the licensee to perform to the satisfaction of the Commissioner, a notice of non–compliance shall be served upon the licensee. Such notice shall be hand–delivered or forwarded by certified mail. The notice of non–compliance shall specify in what respects the licensee has failed to perform as required.

2. If the Commissioner determines that the licensee has failed to perform as specified in the notice of non–compliance, or as specified in any subsequent compliance agreement that may have been reached by the licensee and the Commissioner, the Director shall cause a notice of show cause meeting to be served upon the licensee. Such notice shall be signed by the Director and either hand–delivered or forwarded by certified mail to the licensee. The notice of show cause meeting shall establish the date, time and location of a meeting scheduled to provide the licensee with the opportunity to show cause why the Director should not pursue forfeiture of the financial assurance filed to guarantee such performance.

3. If no mutual compliance agreement is reached at the show cause meeting, or, upon the Commissioner's determination that the licensee has failed to perform as specified in such agreement that was reached, the Director shall request the Commissioner to order forfeiture of the financial assurance filed to guarantee such performance.

4. The Commissioner shall order forfeiture of the financial assurance upon his validation of the Director's determinations and upon his determination that the procedures of this subparagraph have been followed. The Commissioner may, however, at his discretion, provide opportunity for the licensee to be heard before himself before issuing such order. Upon issuance, a copy of the order shall be hand–delivered or forwarded by certified mail to the licensee. Any such order issued by the Commissioner shall become effective 30 days after the receipt by the licensee.

5. If necessary, upon the effective date of the order of forfeiture, the Commissioner shall give notice to the State Attorney General who shall collect the forfeiture.

6. All funds from forfeited financial assurances shall be deposited in the State's radiation reclamation trust fund account for use by the Commissioner as set forth in Section 68–202–405 of the Act.

(i) Incapacity of Applicants or Licensees, Guarantors, or Financial Institutions.

1. An applicant or licensee must notify the Director by certified mail of the commencement of a voluntary or involuntary proceeding under Title 11 (Bankruptcy), U.S. Code, naming the applicant or licensee as debtor, within ten (10) days after commencement of the proceeding. A guarantor of a corporate guarantee as specified in (d)6. of this paragraph must make such a notification if he is named as debtor, as required under the terms of the corporate guarantee.

2. An applicant or licensee who fulfills the requirements of this paragraph by obtaining a surety bond, letter of credit or insurance policy will be deemed to be without the required financial assurance in the event of bankruptcy of the issuing institution or a suspension or revocation of the authority of the institution issuing the surety bond, letter of credit or insurance policy to issue such instruments. The applicant or licensee must establish other financial assurance within 30 days after such an event.

(j) Wording of the Instruments.

1. A surety bond guaranteeing funds for reclaiming as specified in (d)1. of this paragraph, must be worded as follows except that the instructions in parentheses are to be replaced with the relevant information and the parentheses deleted:

SURETY BOND

Date bond executed: _____

Effective date: _____

Principal: (legal name and business address of applicant or licensee)

Type of organization: (insert "individual," "joint venture," "partnership" or "corporation")

State of incorporation: _____

Surety(ies): (Name(s) and business address(es))

License number, name, address and reclaiming cost for each facility guaranteed by this bond (list amounts separately):

$_____

Total penal sum of bond: $_____

Surety's bond number: _____

KNOW ALL PERSONS BY THESE PRESENTS, that we, the Principal and Surety(ies) hereto are firmly bound to the Tennessee Department of Environment and Conservation (hereinafter called Department), in the above penal sum for the payment of which we bind ourselves, our heirs, executors, administrators, successors and assigns jointly and severally; provided that, where the Surety(ies) are corporations acting as co–sureties, we, the Sureties, bind ourselves in such sum "jointly and severally" only for the purpose of allowing a joint action or actions against any or all of us, and for all other purposes each Surety binds itself, jointly and severally with the Principal, for the payment of such sum only as is set forth opposite the name of such Surety, but if no limit of liability is indicated, the limit of liability shall be the full amount of the penal sum.

WHEREAS said Principal is required, under the Tennessee Radiological Health Act, as amended, to have a license in order to receive, possess, store and use radioactive material at the facility identified above, and

WHEREAS said Principal is required to provide financial assurance for reclaiming as a condition of the license;

NOW, THEREFORE, the conditions of this obligation are such that if the Principal shall faithfully perform reclaiming, whenever required to do so, of each facility for which this bond guarantees funds for reclaiming, to the satisfaction of the Commissioner, Tennessee Department of Environment and Conservation in accordance with acceptable practices for protection of health and safety pursuant to all applicable laws, statutes, rules and regulations, as such laws, statutes, rules and regulations may be amended,

OR, if the Principal shall provide alternate financial assurance as specified in 1200–2–10–.12(4), and obtain the written approval of such assurance from the

Director, Division of Radiological Health (hereinafter called Director), within 90 days after the date notice of cancellation is received by both the Principal and the Director from the Surety(ies), then this obligation shall be null and void, otherwise it is to remain in full force and effect.

The Surety(ies) shall become liable on this bond obligation only when the Principal has failed to fulfill the conditions described above.

Upon notification by the Director that the Principal has been found in violation of the reclaiming requirements of the Division, for a facility for which this bond guarantees funds for performance of reclaiming, the Surety(ies) shall forfeit the reclaiming cost amount guaranteed for the facility to the Department as directed by the Director.

Upon notification by the Director that the Principal has filed to provide alternate financial assurance as specified in 1200–2–10–.12(4), and obtain written approval of such assurance from the Director during the 30 days following receipt by both the Principal and the Director of a notice of cancellation of the bond, the Surety(ies) shall forfeit funds in the amount guaranteed for the facility(ies) to the Department as directed by the Director.

The Surety(ies) hereby waive(s) notification of amendments to licenses, applicable laws, statutes, rules and regulations and agree(s) that no such amendment shall in any way alleviate its (their) obligation on this bond.

The liability of the Surety(ies) shall not be discharged by any payment or succession of payments hereunder, unless and until such payment or payments shall amount in the aggregate to the penal sum of the bond, but in no event shall the obligation of the Surety(ies) hereunder exceed the amount of said penal sum.

The Surety(ies) may cancel the bond by sending notice of cancellation by certified mail to the Principal and to the Director, provided, however, that cancellation shall not occur during the 180 days beginning on the date of receipt of the notice of cancellation by both the Principal and the Director, as evidenced by the return receipts.

The Principal may terminate this bond by sending written notice to the Surety(ies), provided, however, that no such notice shall become effective until the Surety(ies) receive(s) written authorization for termination of the bond by the Director.

IN WITNESS WHEREOF, the Principal and Surety(ies) have executed this SURETY BOND and have affixed their seals on the date set forth above.

The persons whose signatures appear below hereby certify that they are authorized to execute this surety bond on behalf of the Principal and Surety(ies) and that the wording of this surety bond is identical to the wording specified in 1200–2–10–.12(4)(j)1. as such regulation was constituted on the date this bond was executed.

PRINCIPAL:

(Signature(s))

(Name(s))

(Title(s))

(Corporate seal)

CORPORATE SURETY(IES):

(Name and address)

State of incorporation: _____

Liability limit: $_____

(Signature(s))

(Name(s) and title(s))

Corporate seal:

(For every co–surety, provide signature(s), corporate seal and other information in the same manner as for Surety above.)

Bond premium: $_____

2. A letter of credit, as specified in (d)2. of this paragraph, must be worded as follows except that instructions in parentheses are to be replaced with the relevant information and the parentheses deleted:

IRREVOCABLE STANDBY LETTER OF CREDIT

Director
Division of Radiological Health
Tennessee Department of Environment and Conservation

Dear Sir or Madam:

We hereby establish our Irrevocable Standby Letter of Credit No. _____ in your favor, at the request and for the account of (applicant's or licensee's name and address) up to the aggregate amount of (in words) U.S. dollars $ _____, available upon presentation of:

1) your sight draft, bearing reference to this letter of credit No. _____, and

2) your signed statement reading as follows: "I certify that the amount of the draft is payable pursuant to regulations issued under authority of the Tennessee Radiological Health Act, as amended."

This letter of credit is effective as of (date) and shall expire on (date at least one (1) year later), but such expiration date shall be automatically extended for a period of (at least one (1) year) on (date) and on each successive expiration date, unless, at least 180 days before the current expiration date, we notify both you and (applicant's or licensee's name) by certified mail that we have decided not to extend this letter of credit beyond the current expiration date. In the event

you are so notified, any unused portion of the credit shall be available upon presentation of your sight draft for 180 days after the date of receipt by both you and (licensee's name), as shown on the signed return receipts.

Whenever this letter of credit is drawn on under and in compliance with the terms of this credit, we shall duly honor such draft upon presentation to us, and we shall forfeit the amount of the draft to the State of Tennessee in accordance with your instructions.

We certify that the wording of this letter of credit is identical to the wording specified in 1200–2–10–.12(4)(j)2 as such regulation was constituted on the date shown immediately below.

(Signature(s) and title(s) of official(s) of issuing institution) (Date)

This credit is subject to (insert "the most recent edition of the Uniform Customs and Practice for Documentary Credits, published by the International Chamber of Commerce," or "the Uniform Commercial Code").

3. A Certificate of insurance, as specified in (d)3. of this paragraph must be worded as follows except that instructions in parentheses are to be replaced with the relevant information and the parentheses deleted:

CERTIFICATE OF INSURANCE FOR RECLAIMING

Name and Address of Insurer
(herein called the "Insurer"): _____

Name and Address of Insured
(herein called the "Insured") : _____

Facilities Covered: (List for each facility: The license number, name, address and the amount of insurance for reclaiming (these amounts for all facilities covered must total the face amount shown below))

Face Amount: $ _____

Policy Number: _____

Effective Date: _____

The Insurer hereby certifies that it has issued to the Insured the policy of insurance identified above to provide financial assurance for reclaiming the facilities identified above. The Insurer further warrants that such policy conforms in all respects with the requirements of 1200–2–10–.12(4)(j)3., as applicable and as such regulations were constituted on the date shown immediately below. It is agreed that any provision of the policy inconsistent with such regulation is hereby amended to eliminate such inconsistency.

Whenever requested by the Director, Division of Radiological Health, Tennessee Department of Environment and Conservation, the Insurer agrees to furnish to the Director, Division of Radiological Health a duplicate original of the policy listed above, including all endorsements thereon.

I hereby certify that the wording of this certificate is identical to the wording specified in 1200–2–10–.12(4)(j)3 as such regulation was constituted on the date shown immediately below.

(Authorized signature for Insurer)

(Name of person signing)

(Title of person signing)

Signature of witness or notary: _____

(Date)

4. A letter from the chief financial officer, as specified in (d)6. of this paragraph must be worded as follows except that instructions in parentheses are to be replaced with the relevant information and the parentheses deleted:

LETTER FROM CHIEF FINANCIAL OFFICER

(Address to Director, Division of Radiological Health)

I am the chief financial officer of (name and address of firm). This letter is in support of this firm's use of the financial test to demonstrate financial assurance, as specified in 1200–2–10–.12(4).

(Fill out the following four paragraphs regarding facilities and associated cost estimates. If your firm has no facilities that belong in a particular paragraph, write "None" in the space indicated. For each facility, include its license number, name, address and current reclaiming cost estimates.)

1. This firm is the licensee at the following facility for which financial assurance for reclaiming is demonstrated through the financial test specified in 1200–2–10–.12(4). The current reclaiming cost estimate covered by the test is: $ _____.

2. This firm guarantees, through the corporate guarantee specified in 1200–2–10–.12(4), the reclaiming of the following facility owned or operated by a subsidiary of this firm. The current cost estimates for reclaiming so guaranteed is: $ _____.

3. In other states, this firm, as licensee or guarantor, is demonstrating financial assurance for reclaiming of the following facilities through the use of a test equivalent or substantially equivalent to the financial test specified in 1200–2–10–.12(4). The current reclaiming cost estimates covered by such a test are shown for each facility: $ _____ _____.

4. This firm is the licensee of the following facilities receiving, possessing, using or storing radioactive material for which financial assurance for reclaiming is <u>not</u> demonstrated either to the Division, another State, or the U.S. Nuclear Regulatory Commission through the financial test or any other financial assurance mechanisms specified in 1200–2–10–.12(4) or equivalent or substantially equivalent mechanisms. The current

reclaiming cost estimates not covered by such financial assurance are shown for each facility: $ _____.

This firm (insert "is required" or "is not required") to file a Form 10K with the Securities and Exchange Commission (SEC) for the latest fiscal year.

The fiscal year of this firm ends on (month, day). The figures for the following items marked with an asterisk are derived from this firm's independently audited, year-end financial statement for the latest completed fiscal year, ending (date).

(Fill in Alternative I if the criteria of (d)6.(i)(I) of this paragraph are used. Fill in Alternative II if the criteria of (d)6.(i)(II) of this paragraph are used).

ALTERNATIVE I

1.	Sum of current reclaiming cost estimates (total of all cost estimates shown in the four paragraphs above)	$ _____
*2.	Total liabilities (if any portion of the reclaiming cost estimate is included in total liabilities, you may deduct the amount of that portion from this line and add that amount to lines 3 and 4)	$ _____
*3.	Tangible net worth	$ _____
*4.	Net worth	$ _____
*5	Current assets	$ _____
*6	Current liabilities	$ _____
*7	Net working capital (line 5 minus line 6)	$ _____
*8.	The sum of net income plus depreciation, depletion, and amortization	$ _____
*9.	Total assets in U.S. (required only if less than 90% of firm's assets are located in the U.S.)	$ _____

		YES	NO
10.	Is line 3 at least $10 million?	_____	_____
11.	Is line 3 at least 6 times line 1?	_____	_____
12.	Is line 7 at least 6 times line 1?	_____	_____
*13.	Are at least 90% of firm's assets located in the U.S.? If not, complete line 14	_____	_____

14.	Is line 9 at least 6 times line 1?	_____	_____
15.	Is line 2 divided by line 4 less than 2.0?	_____	_____
16.	Is line 8 divided by line 2 greater than 0.1?	_____	_____
17.	Is line 5 divided by line 6 greater than 1.5?	_____	_____

ALTERNATIVE II

1.	Sum of current reclaiming cost estimates (total of all cost estimates shown in the four paragraphs above)	$ _____
2.	Current bond rating of most recent issuance of this firm and name of rating service	$ _____
3.	Date of issuance of bond	$ _____
4.	Date of maturity of bond	$ _____
*5.	Tangible net worth	$ _____
*6	Total assets in U.S. (required only if less than 90% of firm's assets are located in the U.S.)	$ _____

		YES	NO
7.	Is line 5 at least $10 million?	_____	_____
8.	Is line 5 at least 6 times line 1?	_____	_____
9.	Are at least 90% of firm's assets located in the U.S.? If not, complete line 10.	_____	_____
10.	Is line 6 at least 6 times line 1?	_____	_____

I hereby certify that the wording of this letter is identical to the wording specified in 1200–2–10–.12(4)(j)4 as such regulations were in effect on the date shown immediately below.

(Signature)

(Name)

(Title)

(Date)

5. A corporate guarantee, as specified in (d)6. of this paragraph, must be worded as follows except that instructions in parentheses are to be replaced with the relevant information and the parentheses deleted:

CORPORATE GUARANTEE FOR RECLAIMING

Guarantee made this (date) by (name of guaranteeing entity), a business corporation organized under the laws of the State of (insert name of State), herein referred to as guarantor, to the Tennessee Department of Environment and Conservation (Department), obligee, on behalf of our subsidiary (applicant or licensee) of (business address).

Recitals

1. Guarantor meets or exceeds the financial test criteria and agrees to comply with the reporting requirements for guarantors as specified in 1200–2–10–.12(4)(d)6.

2. (Applicant or licensee) owns or operates and is licensed by the Department to receive, possess, store and use radioactive material at the facility covered by this guarantee: (List for the facility: license number, name and address).

3. For value received from (licensee), guarantor guarantees to the Department that in the event that (licensee) fails to perform reclaiming of the above facility in a manner deemed acceptable by the Commissioner to assure health and safety from radiation hazards and other license requirements, the guarantor shall do so or forfeit to the State of Tennessee, as specified in 1200–2–10–.12(4) monies in an amount equal to the current reclaiming cost estimates as specified in 1200–2–10–.12(4).

4. Guarantor agrees that if, at the end of any fiscal year before termination of this guarantee, the guarantor fails to meet the financial test criteria, guarantor shall send within 30 days, by certified mail, notice to the Director of the Department's Division of Radiological Health (Division Director) and to (licensee) that he intends to provide alternate financial assurance as specified in 1200–2–10–.12(4), in the name of (licensee). Within 90 days after the end of such fiscal year, the guarantor shall establish such financial assurance unless (licensee) has done so.

5. The guarantor agrees to notify the Division Director, by certified mail, of a voluntary or involuntary proceeding under Title 11 (Bankruptcy), U.S. Code, naming guarantor as debtor, within ten (10) days after commencement of this proceeding.

6. Guarantor agrees that within 30 days after being notified by the Division Director of a determination that guarantor no longer meets the financial test criteria or that he is disallowed from continuing as a guarantor for reclaiming he shall establish alternate financial assurance as specified in 1200–2–10–.12(4) in the name of (licensee) unless (licensee) has done so.

7. Guarantor agrees to remain bound under this guarantee notwithstanding any or all of the following: amendment or modification of the license, the extension or reduction of the time of performance of reclaiming or any other modification or alteration of an obligation of the licensee pursuant to these regulations.

8. Guarantor agrees to remain bound under this guarantee for so long as

(licensee) must comply with the applicable financial assurance requirements of 1200-2-10-.12(4) for the above-listed facility, except that guarantor may cancel this guarantee by sending notice by certified mail to the Division Director and to (licensee), such cancellation to become effective no earlier than 180 days after receipt of such notice by both the Department and (licensee), as evidenced by the return receipts.

9. Guarantor agrees that if (licensee) fails to provide alternate financial assurance as specified in 1200-2-10-.12(4), and obtain written approval of such assurance from the Division Director within 30 days after a notice of cancellation by the guarantor is received by the Division Director from guarantor, guarantor shall provide such alternate financial assurance in the name of (licensee).

10. Guarantor expressly waives notice of acceptance of this guarantee by the Department or by (licensee). Guarantor also expressly waives notice of amendments or modification of the facility license.

I hereby certify that the wording of this guarantee is identical to the wording specified in 1200-2-10-.12(4)(j)5 as such regulations were in effect on the date first above written.

Effective Date: _____

(Name of guarantor)

(Authorized signature for guarantor)

(Name of person signing)

(Title of person signing)

Signature of witness or notary: _____

(k) Persons licensed at the time these financial assurance regulations become effective and upon notice by the Department must, within a period of 90 days following such notice, provide the required financial assurance.

(l) The Department may reevaluate, at any time, the adequacy of existing financial assurance and may require their adjustment by either increasing or decreasing the amount of financial assurance required so that adequate funds will be available.

(m) Except that the following persons are exempt from the requirements of this paragraph:

1. State and local government agencies.

2. Educational institutions accredited by the Southern Association of Colleges and Schools.

3. Licensees of the State Licensing Board for the Healing Arts and those medical facilities possessing or utilizing radioactive materials for medical purposes when supervised by such licensees.

4. Veterinarians possessing or utilizing radioactive materials in their veterinary practice.

5. Persons possessing or utilizing radioactive materials for in vitro medical purposes.

6. Persons possessing or utilizing only generally licensed quantities of radioactive materials.

(5) The applicant or an existing licensee, for whom financial assurance is required and where it is intended that the licensed facility will eventually cease to operate while containing, storing or possessing radioactive sources on the premises and will require continuing and perpetual care or surveillance over the facility to protect public health, safety or welfare, shall deposit sums to a Perpetual Care Trust Fund maintained by the Commissioner in the name of the State.

(a) The Commissioner shall consider the following in making his determination of the Perpetual Care Trust Fund deposits for each individual applicant or licensee.

1. The nature of the licensed radioactive material; including its radiotoxicity, half–life, chemical and physical form and containment;

2. Size and type of facility to be decommissioned; and

3. The anticipated cost to the State of perpetual care and surveillance.

(b) The Department may reevaluate at any time the adequacy of a licensee's contributions to the existing Perpetual Care Trust Fund and may adjust by increasing or decreasing the rate of contribution or the specified amount required of a licensee so that the fund may be adequate in amount to meet perpetual care requirements of that licensee.

(6) Definitions of terms used in paragraph (4) of this rule

(a) 'Current reclaiming cost estimate' means the most recent of the estimates prepared in accordance with (c)1., 2. and 3. of paragraph (4) of this rule.

(b) 'Director' means the Director of the Division of Radiological Health of the Department of Environment and Conservation.

(c) 'Parent corporation' means a corporation that directly owns at least 50 percent (50%) of the voting stock of the corporation that is the facility owner or operator; the latter corporation is deemed a "subsidiary" of the parent corporation.

(d) The following terms are used in the specifications for the financial tests for financial assurance for reclaiming. The definitions are intended to assist in the understanding of these regulations and are not intended to limit the meanings of terms in a way that conflicts with generally accepted accounting practices.

1. 'Assets' means all existing and all probable future economic benefits obtained or controlled by a particular entity.

2. 'Current assets' means cash or other assets or resources commonly identified as those that are reasonably expected to be realized in cash or sold or consumed during the normal operating cycle of the business.

3. 'Current liabilities' means obligations whose liquidation is reasonably expected to require the use of existing resources properly classifiable as current assets or the creation of other current liabilities.

4. 'Independently audited' refers to an audit performed by an independent certified public accountant in accordance with generally accepted auditing standards.

5. 'Liabilities' means probable future sacrifices of economic benefits arising from present obligations to transfer assets or provide services to other entities in the future as a result of past transactions or events.

6. 'Net working capital' means current assets minus current liabilities.

7. 'Net worth' means total assets minus total liabilities and is equivalent to owner's equity.

8. 'Tangible net worth' means the tangible assets that remain after deducting liabilities; such assets would not include intangibles such as goodwill and rights to patents of royalties.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed October 22, 1987; effective December 6, 1987.*

1200–2–10–.13 SPECIAL REQUIREMENTS FOR ISSUANCE OF SPECIFIC LICENSES.

(1) Human use of radioactive materials in institutions. In addition to the requirements set forth in 1200–2–10–.12, a specific license for human use of radioactive material in institutions will be issued only if:

(a) The applicant has appointed a radiation safety committee to oversee the use of licensed material throughout the institution and to review the institution's radiation safety program. Membership of the committee must include at least the following: an authorized user for each type of use permitted by the license, a representative of the nursing staff, a representative of the institution's management, and the Radiation Safety Officer;

(b) The applicant possesses facilities for the clinical care of patients;

(c) The physician designated on the application as the individual user has experience in the handling and administration of radioisotopes and, where applicable, the clinical management of radioactive patients as outlined in 1200–2–10–.33; and

(d) If the application is for a license to use unspecified quantities or multiple types of radioactive material, the applicant's staff has experience in e use of a variety of radioactive materials for a variety of human uses.

(2) Human use of radioactive materials by individual physicians.

(a) In addition to the requirements set forth in 1200–2–10–.12, a specific license for the human use of radioactive materials will be issued to an individual physician or group of physicians only if:

1. The application is for use in the applicant's practice in an office outside a medical institution;

2. The applicant has access to a hospital possessing facilities to hospitalize and monitor the applicant's radioactive patients whenever it is clinically indicated; and

3. The applicant has experience in the handling and administration of radioisotopes, and, where applicable, the clinical management of radioactive patients, as outlined in 1200–2–10–.33.

(b) The Department will not approve an application by an individual physician or group of physicians for a specific license to receive, possess, or use radioactive material on the premises of a medical institution unless:

1. The use of radioactive material is limited to:

(i) The administration of radiopharmaceuticals for diagnostic or therapeutic purposes;

(ii) The performance of diagnostic studies on patients to whom a radiopharmaceutical has been administered;

(iii) The performance of in vitro diagnostic studies; or

(iv) The calibration and quality control checks of radioactive assay instrumentation, radiation safety instrumentation and diagnostic instrumentation.

2. The physician brings the radioactive material with him and removes the radioactive material when he departs. (The institution cannot receive, possess, or store radioactive material other than the amount of material remaining n the patient); and

3. The medical institution does not hold a radioactive material license under 1200–2–10–.13(1).

(3) Human use of sealed sources. In addition to the requirements set forth in 1200–2–10–.12, a specific license for human use of sealed sources will be issued only if the applicant, or if the application is made by an institution, the individual user (1) has training as outlined in 1200–2–10–.33 in the therapeutic use of the sealed source considered (teletherapy unit, beta applicator, etc.) and (2) is a physician.

(4) Multiple quantities of types of radioactive material for use in research and development. In addition to the requirements set forth in 1200–2–10–.12, a specific license for multiple quantities or types of radioactive material for use in research and development will be issued only if:

(a) The applicant has established an isotope committee (composed of such persons as a radiological safety officer, a representative of the business office, and one or more persons trained or experienced in the safe use of radioactive materials) that will review and approve, in advance of purchase of radioisotopes, proposals for use; and

(b) The applicant has appointed a radiological safety officer who will advise and assist on radiological safety problems.

~~(5) Manufacture and distribution of devices to persons generally licensed under 1200-2-10-.10(2). In addition to the requirements set forth in 1200-2-10-.12, a specific license to distribute certain devices of the types enumerated in 1200-2-10-.10(2) to persons generally licensed under 1200-2-10-.10(2) will be issued only if:~~

~~(a) The applicant submits information relating to the design, manufacture, prototype testing, quality control, labels, proposed uses, installation, servicing, leak testing, operating and safety instructions, and potential hazards of the device to provide assurance that:~~

~~1. The device can be safely operated by persons not having training in radiological protection;~~

~~2. Under ordinary conditions of handling, storage and use of the device, the radioactive material contained in the device will not be released or inadvertently removed from the device, and no person will receive in any period of one calendar quarter a dose in excess of ten percent (10%) of the limits specified in 1200-2-5-.50; and~~

~~3. Under accident conditions (such as fire and explosion) associated with handling, storage and use of the device, it is unlikely that any person would receive an external radiation dose or dose commitment in excess of the dose to the appropriate organ as specified in Table RHS-7-1;~~

~~TABLE RHS-7-1 TABLE OF ORGAN DOSES~~

~~Part of Body~~	~~rem~~
~~Whole body; head and trunk; active blood forming organs; gonads; or lens of eye~~	~~15~~
~~Hands and forearms; feet and ankles; localized areas of skin averaged over areas no larger than 1 square centimeter~~	~~200~~
~~Other organs~~	~~50~~

~~(b) Each device bears a durable, legible clearly visible label or labels approved by the Division that contain in a clearly identified and separate statement:~~

~~1. Instructions and precautions for safe installation, operation and servicing of the device (documents such as operating and service manual may be identified in he label and used to provide this information);~~

~~2. The requirements, or lack of requirement, for leak testing, or for testing any on-off mechanism and indicator, including the maximum time interval for such testing, and the identification of the radioactive material by isotope, quantity of radioactivity and date of determination of the quantity; and~~

~~3. The information called for in one of the following statements in the same or similar form:~~

~~(i) The receipt, possession, use, and transfer of this device Model _____, Serial No. _____, are subject to a general license or the equivalent and the regulations of the U.S. Nuclear Regulatory~~

~~Commission or of a State with which the NRC has entered into an agreement for the exercise of regulatory authority. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.~~

~~CAUTION – RADIOACTIVE MATERIAL~~

~~_____~~
~~(Name of manufacturer or distributor)~~

~~(ii) The receipt, possession, use and transfer of this device Model _____, Serial No. _____ are subject to a general license or the equivalent and the regulations of a Licensing State. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.~~

Deleted: _____,

~~CAUTION – RADIOACTIVE MATERIAL~~

~~_____~~
~~(Name of manufacturer or distributor)~~

~~(c) In the event the applicant desires that the device be tested at intervals longer than six (6) months, either for proper operation of the on–off mechanism and indicator, if any, or for leakage of radioactive material, or for both, he shall include in his application information to demonstrate that such longer interval is justified by performance characteristics of the device or similar devices, and by design features that have a significant bearing on the probability or consequences of leakage of radioactive material or failure of the on–off mechanism indicator. In determining the acceptable interval for the test for leakage of radioactive material, the Division will consider information on particulars that include, but are not necessarily limited to:~~

~~1. Primary containment (source capsule);~~

~~2. Protection of primary containment;~~

~~3. Method of sealing containment;~~

~~4. Containment construction materials;~~

~~5. Form of contained radioactive material;~~

~~6. Maximum temperature withstood during prototype tests;~~

~~7. Maximum pressure withstood during prototype tests;~~

~~8. Maximum quantity of contained radioactive material;~~

~~9. Radiotoxicity of contained radioactive material; and~~

~~10. Operating experience with identical devices or similarly designed and constructed devices;~~

~~(d) In the event the applicant desires that the general licensee under 1200–2–10–.10(2) or under equivalent regulations of the U.S. Nuclear Regulatory Commission, an~~

~~Agreement State or a Licensing State, be authorized to install the device, collect the sample to be analyzed by a specific licensee for leakage of radioactive material, service the device, test the on–off mechanism and indicator, or remove the device from installation, he shall include in his application written instructions to be followed by the general licensee, estimated calendar quarter doses associated with such activities and the basis for such estimates. The submitted information shall demonstrate that performance of such activity or activities by an individual untrained in radiological protection, in addition to other handling, storage and use of devices under the general license, will not cause that individual to receive a calendar quarter dose in excess of ten percent (10%) of the limits specified in 1200–2–5–.50:~~

~~(e) Each person licensed under 1200–2–10–.13(5) shall:~~

~~1. Furnish a copy of the general license contained in 1200–2–10–.10(2) to each person to whom he directly or through an intermediate person transfers radioactive material in a device for use pursuant to the general license contained in 1200–2–10–.10(2).~~

~~2. Furnish a copy of the general license contained in the U.S. Nuclear Regulatory Commission's, Agreement State's, or Licensing State's regulation equivalent to 1200–2–10–.10(2), or alternatively, furnish a copy of the general license contained in 1200–2–10–.10(2), to each person to whom he directly or through an intermediate person transfers radioactive material in a device for use pursuant to the general license of the U.S. Nuclear Regulatory Commission, the Agreement State or the Licensing State. If a copy of the general license in 1200–2–10–.10(2) is furnished to such person, it shall be accompanied by a note explaining that use of the device is regulated by the U.S. Nuclear Regulatory Commission, Agreement State or Licensing State under requirements substantially the same as those in 1200–2–10–.10(2); and~~

~~(f) Each person licensed under 1200–2–10–.13(5) to distribute devices to generally licensed persons shall:~~

~~1. Report to the Division, at its office located at the address in Rule 1200–2–4–.07, all transfers of such devices to persons for use under the general license in 1200–2–10–.10(2).~~

~~2. Report to the U.S. Nuclear Regulatory Commission all transfers of such devices to persons for use under the U.S. Nuclear Regulatory Commission general license in Section 31.5 of 10 CFR Part 31.~~

~~3. Report to the responsible Agreement or Licensing State agency all transfers of devices manufactured and distributed pursuant to 1200–2–10–.13(5) for use under a general license in that state's regulations equivalent to 12010–.10(2).~~

~~4. Reports required by 1., 2., and 3. of this subparagraph (f) shall identify each general licensee by name and address, an individual by name and/or position who may constitute a point of contact between the Division and the general licensee, the type and model number of device transferred, and the quantity and type of radioactive material contained in the device. If one or more intermediate persons will temporarily possess the device at the intended place of use prior to its possession by the user, the report shall include identification of each intermediate person by name, address, contact and relationship to the intended user. If no transfers have been made to persons generally licensed under 1200–~~

~~2–10–.10(2) during the reporting period the report shall so indicate. The report shall cover each calendar quarter and shall be filed within 30 days thereafter.~~

~~5. Keep records showing the name, address, and a point of contact for each general licensee to whom he directly or through an intermediate person transfers radioactive material in devices for use pursuant to the general license provided in 1200–2–10–.10(2) or equivalent regulations of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State. The record shall show the date of each transfer, the isotope and quantity of radioactivity in each device transferred, the identity of any intermediate person, and compliance with the report requirements of this subparagraph. The records required by this part 5. shall be maintained for a period of five (5) years from the date of the recorded event.~~

(5) Manufacture, distribution or initial distribution of devices to persons generally licensed under 1200–2–10–.10(2). In addition to the requirements set forth in 1200–2–10–.12, a specific license to distribute certain devices of the types enumerated in 1200–2–10–.10(2) to persons generally licensed under 1200–2–10–.10(2) or equivalent regulations of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State will be issued only if:

(a) The applicant submits sufficient information relating to the design, manufacture, prototype testing, quality control, labels, proposed uses, installation, servicing, leak testing, operating and safety instructions, and potential hazards of the device to provide assurance that:

1. The device can be safely operated by persons not having training in radiological protection;

2. Under ordinary conditions of handling, storage and use of the device, the radioactive material contained in the device will not be released or inadvertently removed from the device, and no person will receive in one year a dose in excess of ten percent (10%) of the limits specified in 1200–2–5–.50; and

3. Under accident conditions (such as fire and explosion) associated with handling, storage and use of the device, it is unlikely that any person would receive an external radiation dose or dose commitment in excess of the dose to the appropriate organ as specified in Table RHS 7–1;

TABLE RHS 7–1 TABLE OF ORGAN DOSES

Part of Body	rem	mSv / Sv
Whole body: head and trunk; active blood forming organs; gonads; or lens of eye	15	150 mSv
Hands and forearms; feet and ankles; localized areas of skin averaged over areas no larger than 1 square centimeter	200	2 Sv
Other organs	50	500 mSv

(b) Each device bears a durable, legible clearly visible label or labels approved by the Division that contain(s) in a clearly identified and separate statement:

1. Instructions and precautions for safe installation, operation and servicing of the device (documents such as operating and service manuals may be identified in the label and used to provide this information);

2. The requirements, or lack of requirement, for leak testing, or for testing any on–off mechanism and indicator, including the maximum time interval for such testing, and the identification of the radioactive material by isotope, quantity of radioactivity and date of determination of the quantity; and

3. The information called for in one of the following statements in the same or similar form:

(i) The receipt, possession, use, and transfer of this device, Model _____,[10] Serial No. _____,[10] are subject to a general license or the equivalent and the regulations of the U.S. Nuclear Regulatory Commission or of a State with which the NRC has entered into an agreement for the exercise of regulatory authority. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.

CAUTION – RADIOACTIVE MATERIAL

_____

(Name of manufacturer or distributor)

(ii) The receipt, possession, use and transfer of this device Model _____,[10] Serial No. _____,[10] are subject to a general license or the equivalent and the regulations of a Licensing State. This label shall be maintained on the device in a legible condition. Removal of this label is prohibited.

CAUTION – RADIOACTIVE MATERIAL

_____

(Name of manufacturer or distributor)

(c) Each device having a separable source housing that provides the primary shielding for the source also bears, on the source housing, a durable label containing the device model number and serial number, the isotope and quantity, the words "CAUTION – RADIOACTIVE MATERIAL," and, if practicable, the radiation symbol described in 1200–2–5–.110 and the name of the manufacturer or initial distributor.

(d) Each device meeting the criteria of 10 CFR 31.5(c)(13)(i) bears a permanent (e.g., embossed, etched, stamped, or engraved) label affixed to the source housing if separable, or the device if the source housing is not separable, that includes the words, "CAUTION – RADIOACTIVE MATERIAL," and, if practicable, the radiation symbol described in 1200–2–5–.110.

[10] *If specified elsewhere in labeling affixed to the device, the model, serial number and manufacturer or distributor may be omitted from this label.*

(e) In the event the applicant desires that the device be tested at intervals longer than six (6) months, either for proper operation of the on–off mechanism and indicator, if any, or for leakage of radioactive material, or for both, he shall include in his application information to demonstrate that such longer interval is justified by performance characteristics of the device or similar devices, and by design features that have a significant bearing on the probability or consequences of leakage of radioactive material or failure of the on–off mechanism indicator. In determining the acceptable interval for the test for leakage of radioactive material, the Division will consider information that includes, but is not limited to:

1. Primary containment (source capsule);

2. Protection of primary containment;

3. Method of sealing containment;

4. Containment construction materials;

5. Form of contained radioactive material;

6. Maximum temperature withstood during prototype tests;

7. Maximum pressure withstood during prototype tests;

8. Maximum quantity of contained radioactive material;

9. Radiotoxicity of contained radioactive material; and

10. Operating experience with identical devices or similarly designed and constructed devices;

(f) In the event the applicant desires that the general licensee under 1200–2–10–.10(2) or under equivalent regulations of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State, be authorized to install the device, collect the sample to be analyzed by a specific licensee for leakage of radioactive material, service the device, test the on–off mechanism and indicator, or remove the device from installation, he shall include in his application written instructions to be followed by the general licensee, estimated calendar quarter doses associated with such activity or activities and the basis for such estimates. The submitted information shall demonstrate that performance of such activity or activities by an individual untrained in radiological protection, in addition to other handling, storage and use of devices under the general license, will not cause that individual to receive in one year a dose in excess of ten percent (10%) of the limits specified in 1200–2–5–.50;

(g) Before radioactive material may be transferred in a device for use under a general license, each person licensed under 1200–2–10–.13(5) shall furnish the following information to each person to whom he directly or through an intermediate person transfers radioactive material in a device. In the case of a transfer through an intermediate person, the information shall be provided to the intended user and to the intermediate person prior to initial transfer to the intermediate person.

For use under the general license contained in 1200–2–10–.10(2)	For use under equivalent regulations of the U.S. Nuclear Regulatory Commission or an Agreement State or a Licensing State

1.	A copy of the general license contained in 1200-2-10-.10(2).	A copy of the general license contained in the U.S. Nuclear Regulatory Commission's, Agreement State's, or Licensing State's regulations equivalent to 1200-2-10-.10(2). Alternatively, he may furnish a copy of the general license contained in 1200-2-10-.10(2). If a copy of the general license in 1200-2-10-.10(2) is furnished to such a person, it shall be accompanied by a note explaining that the use of the device is regulated by the U.S. Nuclear Regulatory Commission, Agreement State or Licensing State under requirements substantially the same as those in 1200-2-10-.10(2):
	If parts 1200-2-10-.10(2)(c)2. through 4. or 1200-2-10-.10(2)(c)13. do not apply to the particular device, those parts may be omitted:	If paragraphs (c)(2) through (4) or (c)(13), or sections of the Agreement State or Licensing State regulations equivalent to these paragraphs, do not apply to the particular device, these paragraphs may be omitted.
2.	A copy of 1200-2-10-.26, 1200-2-5-.140 and 1200-2-5-.141	A copy of 10 CFR §§31.2, 30.51, 20.2201, and 20.2202 or the Agreement State or Licensing State regulations equivalent to these NRC regulations
3.	A list of services that may only be performed by a specific licensee;	
4.	Information on acceptable disposal options including estimated costs of disposal;	
5.	A statement that regulatory agencies may issue citations and civil penalties for improper disposal;	
6.	The name or title, address, and phone number of the person at the appropriate regulatory agency from whom additional information may be obtained;	

(h) An alternative approach to informing customers may be proposed by the licensee for approval by the Division.

(i) Each device that is transferred after [insert date 1 year after effective date of this rule] shall meet the labeling requirements in subparagraphs 1200-2-10-.13(5)(b), (c) and (d).

(j) Each person licensed under 1200-2-10-.13(5) to distribute devices to generally licensed persons shall:

1. Report to the Division, at its offices located at the address in Rule 1200-2-4-.07, all transfers of such devices to persons for use under the general license in 1200-2-10-.10(2).

2. Report to the U.S. Nuclear Regulatory Commission all transfers of such devices to persons for use under the U.S. Nuclear Regulatory Commission general license in Section 31.5 of 10 CFR Part 31.

3. Report to the responsible Agreement or Licensing State agency all transfers of devices manufactured and distributed pursuant to 1200–2–10–.13(5) for use under a general license in that state's regulations equivalent to 1200-2-10–.10(2).

4. Reports required by parts (5)(j)1., 2., and 3. shall identify:

(i) Each general licensee by name and mailing address for the location of use: if there is no mailing address for the location of use, an alternate address for the general licensee shall be submitted along with information on the actual location of use;

(ii) The name, title and phone number of the person identified by the general licensee as having knowledge of and authority to take required actions to ensure compliance with the appropriate regulations and requirements;

(iii) The date of transfer;

(iv) The type, model number and serial number of the device transferred; and

(v) The quantity and type of radioactive material contained in the device.

5. If one or more intermediate persons will temporarily possess the device at the intended place of use prior to its possession by the user, the report shall include the same information for both the intended user and each intermediate person, and clearly designate the intermediate person(s).

6. For devices received from a general licensee, the report shall include the identity of the general licensee by name and address, the type, model number and serial number of the device received, the date of receipt, and, in the case of devices not initially transferred by the reporting licensee, the name of the manufacturer or initial transferor.

7. If the licensee makes changes to a device possessed by a general licensee, such that the label must be changed to update required information, the report shall identify the general licensee, the device, and the changes to information on the device label.

8. The report shall cover each calendar quarter, shall be filed within 30 days of the end of the calendar quarter, and shall clearly indicate the period covered by the report.

9. The report shall clearly identify the specific licensee submitting the report and include the license number of the specific licensee.

10. If no transfers have been made to or from persons generally licensed under 1200–2–10–.10(2) during the reporting period, the report shall so indicate.

11. Keep records showing the name, address of use, and responsible individual for each general licensee to whom he directly or through an intermediate person

transfers radioactive material in devices for use pursuant to the general license provided in 1200–2–10–.10(2) or equivalent regulations of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State. The records shall show the date of each transfer, the model number, serial number and the isotope and quantity of radioactivity in each device transferred, the identity of any intermediate person(s), and compliance with the report requirements of this subparagraph. The records required by this part (5)(j)11. shall be maintained for a period of three (3) years from the date of the recorded event.

(6) The use of sealed sources in industrial radiography. In addition to the requirements set forth in 1200–2–10–.12, a specific license for use of sealed sources in industrial radiography will be issued only if:

(a) The applicant will have a program for training radiographers and radiographer's assistants and submits to the Division for approval a schedule or description of such program that specifies the:

1. Initial training:

(i) This initial training will consist of a complete training program as outlined in 1200–2–8–.07; or

(ii) Resumes of prior training and experience of individuals that show fulfillment of the requirements of 1200–2–8–.07(7)(a) and (b) and the program for the initial training of such individuals in the licensee's or registrant's specific industrial radiography program as outlined in 1200–2–8–.07(7)(c), (d) and (e);

2. Periodic training;

3. On–the–job training;

4. Means to be used by the applicant to determine the radiographer's knowledge and understanding of and ability to comply with Division regulations and licensing requirements and the operating and emergency procedures of the applicant; and

5. Means to be used by the applicant to determine the radiographer's assistant's knowledge and understanding of and ability to comply with the operating and emergency procedures of the applicant;

(b) The applicant has established and submits to the Division for approval written operating and emergency procedures as described in 1200–2–8–.05(2);

(c) The applicant will have an internal inspection system to assure that Division regulations, license provisions, and the applicant's operating and emergency procedures are followed by radiographers and radiographer's assistants; the inspection system shall include the performance of internal inspections at intervals not to exceed three (3) months and the retention of records of such inspections for inspection by the Division.

(d) The applicant submits to the Division a description of his overall organizational structure pertaining to the radiography program, including specified delegations of authority and responsibility for operation of the program; and

(e) The applicant who desires to conduct his own leak tests must establish procedures to be followed in testing sealed sources for possible leakage and contamination and submit to the Division for approval a description of such procedures including:

1. Instrumentation to be used;

2. Method of performing tests, *e.g.*, points on equipment to be smeared and method of taking smear; and

3. Pertinent experience of the person who will perform the test.

(7) Multiple quantities or types of radioactive material for use in processing. In addition to the requirements set forth in 1200–2–10–.12, a specific license for multiple quantities or types of radioactive material for use in processing for distribution to other authorized persons will be issued only if [11]:

(a) The applicant's staff has experience in the use of radioisotopes for processing and distribution; and

(b) The applicant has appointed a radiological safety officer who will advise and assist on radiological safety problems.

(8) Introduction of radioactive material into products in exempt concentrations. In addition to the requirements set forth in 1200–2–10–.12, a specific license authorizing the introduction of radioactive material into a product or material owned by or in the possession of the licensee or another to be transferred to persons exempt under 1200–2–10–.04(1)(a) will be issued only if:

(a) The applicant submits a description of the product or material into which the radioactive material will be introduced, intended use of the radioactive material and the product or material into which it is introduced, method of introduction, initial concentration of the radioactive material in the product or material, control methods to assure that no more than the specified concentration is introduced into the product or material, estimated time interval between introduction and transfer of the product or material and estimated concentration of the radioactive material in the product or material at the time of transfer; and

(b) The applicant provides assurance that the concentrations of radioactive material at the time of transfer will not exceed the concentrations in Schedule RHS 8–4, that reconcentration of the radioactive material in concentrations exceeding those in Schedule RHS 8–4 is not likely, that lower concentrations cannot be used, and that the product or material is not likely to be incorporated in any food, beverage, cosmetic, drug, or other commodity or product designed for ingestion or inhalation by, or application to, a human being. Each person licensed under this paragraph (8), shall file an annual report with the Division that shall identify the type and quantity of each product or material into which radioactive material has been introduced during the reporting period, name and address of the person who owns or possesses the product or material into which radioactive material has been introduced at the time of introduction, the type and quantity of radioactive material introduced into each product or material, and the initial concentrations of radioactive material in the product or

[11] *Authority to transfer possession or control by the manufacturer, processor, or producer of any equipment, device, commodity, or other product containing by product material whose subsequent possession, use, transfer, and disposal by all other persons are exempted from regulatory requirements may be obtained only from the U.S. Nuclear Regulatory Commission, Washington, DC 20555.*

material at the tie of transfer of the radioactive material by the licensee. If no transfers of radioactive material have been made pursuant to this paragraph (8) during the reporting period, the report shall so indicate. The report shall be submitted within 30 days after the end of each calendar year.

(9) Radioactive material in luminous safety devices for use in aircraft. In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture, assemble, repair, or distribute to persons generally licensed under 1200–2–10–.10(3) luminous safety devices containing radioactive materials for use in aircraft will be issued only if the requirements of Sections 32.53, 32.54, 32.55,32.56 and 32.101 of 10 CFR Part 32 or their equivalent are met.

(10) Manufacture, preparation or transfer for commercial distribution of radiopharmaceuticals containing radioactive material for medical use under group licenses.

(a) In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture and distribute radiopharmaceuticals containing radioactive material for use by persons licensed pursuant to 1200–2–10–.14 for uses listed in Group I, Group II, Group IV, or Group V or 1200–2–10–.14(6) will be issued only if:

1. The requested site for manufacture and/or distribution of radiopharmaceuticals is located within Tennessee;

2. The applicant submits evidence that the applicant is at least one of the following:

(i) Registered or licensed with the U.S. Food and Drug Administration (FDA) as a drug manufacturer;

(ii) Registered or licensed with a State agency as a drug manufacturer; or

(iii) Licensed as a pharmacy by the Tennessee Board of Pharmacy.

3. The applicant submits information on the radionuclide; chemical and physical form; packaging including maximum activity per vial, syringe, generator or other container of the radioactive drug;, and shielding provided by the packaging of the radioactive material for safe handling and storage of radiopharmaceuticals by group licensees; and

4. The applicant satisfies the following labeling requirements:

(i) A label is affixed to each transport radiation shield, whether it is constructed of lead, glass, plastic or other material, of a radioactive drug to be transferred for commercial distribution. The label shall include the radiation symbol and the words "CAUTION, RADIOACTIVE MATERIAL" or "DANGER, RADIOACTIVE MATERIAL"; the name of the radioactive drug or its abbreviation; and the quantity of radioactivity at a specified date and time. For radioactive drugs with a half–life greater than 100 days, the time may be omitted.

(ii) A label is affixed to each syringe, vial or other container used to hold a radioactive drug to be transferred for commercial distribution. The label shall include the radiation symbol and the words "CAUTION, RADIOACTIVE MATERIAL" or "DANGER, RADIOACTIVE MATERIAL" and an identifier that ensures that the syringe, vial or other

container can be correlated with the information on the transport radiation shield label.

(b) A licensee described above by subpart (a)2.(iii):

1. May prepare radioactive drugs for medical use, as defined in subparagraph 1200–2–4–.04(o), provided that the radioactive drug is prepared by either an authorized nuclear pharmacist, as specified below in parts (b)2. and (b)3. of this paragraph, or an individual under the supervision of an authorized nuclear pharmacist.

2. May allow a pharmacist to work as an authorized nuclear pharmacist if:

(i) This individual qualifies as an authorized nuclear pharmacist as defined in subparagraph 1200–2–4–.04(1)(lll),

(ii) This individual meets the requirements specified in part 1200–2–10–.35(1)(a)2. and the licensee has received an approved license amendment identifying this individual as an authorized nuclear pharmacist, or

(iii) This individual is designated as an authorized nuclear pharmacist in accordance with part 4. of this subparagraph.

3. The actions authorized above in parts 1. and 2. are permitted in spite of more restrictive language in license conditions.

4. May designate a pharmacist (as defined in subparagraph 1200–2–4–.04(1)(lll)) as an authorized nuclear pharmacist if the individual is identified as of {April 18, 2002}, as an 'authorized user' on a nuclear pharmacy license issued by the Division under this chapter.

5. Shall provide to the Division a copy of each individual's certification by the Board of Pharmaceutical Specialties, the Division, U.S. Nuclear Regulatory Commission or other Agreement State license and a copy of the state pharmacy license or registration, no later than 30 days after the date that the licensee allows, pursuant to subparts 2.(i) and 2.(iii) of this subparagraph, the individual to work as an authorized nuclear pharmacist.

(c) A licensee shall possess and use instrumentation to measure the radioactivity of radioactive drugs. The licensee shall have procedures for use of the instrumentation. The licensee shall measure by direct amount of radioactivity in dosages of alpha–, beta–, or photon–emitting radioactive drugs before transfer for commercial distribution. In addition, the licensee shall:

1. Perform tests before initial use, periodically and following repair, on each instrument for accuracy, linearity and geometry dependence, as appropriate for the use of the instrument; and make adjustments when necessary; and

2. Check each instrument for constancy and proper operation at the beginning of each day of use.

(d) Nothing in this rule relieves the licensee from complying with applicable FDA, other Federal and State requirements governing radioactive drugs.

(11) Manufacture and distribution of generators or reagent kits for preparation of radiopharmaceuticals containing radioactive material. In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture and distribute generators or reagent kits containing radioactive material for preparation of radiopharmaceuticals by persons licensed pursuant to 1200–2–10–.14 for uses listed in Group III of 1200–2–10–.14(6) will be issued only if [12]:

(a) The requested site for manufacture and/or distribution of generators or reagent kits is located within this State;

(b) The applicant submits evidence that:

1. The generator or reagent kit is to be manufactured, labeled and packaged in accordance with the Federal Food, Drug and Cosmetic Act or the Public Health Service Act, such as a new drug application (NDA) approved by the United States Food and Drug Administration (FDA), a biologic product license issued by FDA or a "Notice of Claimed Investigational Exemption for a New Drug" (IND) accepted by FDA; or

2. The manufacture and distribution of the generator or reagent kit are not subject to the Federal Food, Drug and Cosmetic Act and the Public Health Service Act;

(c) The applicant submits information on the radionuclide, chemical and physical form, packaging including maximum activity per package, and shielding provided by the packaging of the radioactive material contained in the generator or reagent kit;

(d) The label affixed to the generator or reagent kit contains information on the radionuclide, quantity and date of assay;

(e) The label affixed to the generator or reagent kit, or the leaflet or brochure that accompanies the generator or reagent kit, contains:

1. Information, from a radiation safety standpoint, on the procedures to be followed and the equipment and shielding to be used in eluting the generator or processing radioactive material with the reagent kit; and

2. A statement that this generator or reagent kit, as appropriate, is approved for use by persons licensed by the Division pursuant to 1200–2–10–.14, Group III, or under equivalent licenses of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State; and

(f) The labels, leaflets or brochures required by (d) and (e) of this paragraph are in addition to the labeling required by the FDA and they may be separate from or, with the approval of FDA, may be combined with the labeling required by FDA.

(12) Manufacture and distribution of sources or devices containing radioactive material for medical uses. In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture and distribute sources or devices containing radioactive material to persons

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[12] *Although the Department does not regulate the manufacture and distribution of reagent kits that do not contain radioactive material, it does regulate the use of such reagent kits for the preparation of radiopharmaceuticals containing radioactive material as part of its licensing and regulation of the users of radioactive material. Any manufacturer of reagent kits that do not contain radioactive material who desires to have reagent kits approved by the Department for use by persons licensed pursuant to 1200–2–10–.14 for Group III may submit the pertinent information specified in this paragraph (10).*

licensed pursuant to 1200–2–10–.14 for use as a calibration or reference source or for uses listed in Group VI of 1200–2–10–.14(6) will be issued only if:

(a) The requested site for manufacture and/or distribution of sources and devices is located within this State;

(b) The applicant submits information regarding each type of source or device pertinent to an evaluation of its radiation safety, including:

1. The radioactive material contained, its chemical and physical form and amount;

2. Details of design and construction of the source or device;

3. Procedures for, and results of, prototype tests to demonstrate that the source or device will maintain its integrity under stresses likely to be encountered;

4. For devices containing radioactive material, the radiation profile of a prototype device;

5. Details of quality control procedures to assure that production sources and devices meet the standards of the design and prototype tests;

6. Procedures and standards for calibrating sources and devices;

7. Legend and methods for labeling sources and devices as to their radioactive content; and

8. Instructions for handling and storing the source or device for radiation safety; these instructions are to be included on a durable label attached to the source or device or attached to a permanent storage container for the source or device; provided that instructions that are too lengthy for such label may be summarized on the label and printed in detail on a brochure that is referenced on the label;

(c) The label affixed to the source or device, or to the permanent storage container for the source or device, contains information on the radionuclide, quantity, and date of assay, and a statement that the (name of source or device) is licensed by the Division for distribution to persons licensed pursuant to 1200–2–10–.14, Group VI, or under equivalent licenses of the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State; provided that such labeling for sources that do not require long term storage (*e.g.*, gold–198 seeds) may be on a leaflet or brochure that accompanies the source; and

(d) In the event the applicant desires that the source or device be required to be tested for leakage of radioactive material at intervals longer than six (6) months, the applicant shall include in his application information to demonstrate that such longer interval is justified by performance characteristics of the source or device or similar sources or devices and by design features that have a significant bearing on the probability or consequences of leakage of radioactive material from the source. In determining the acceptable interval for test of leakage of radioactive material, the Division will consider information that includes, but is not limited to:

1. Primary containment (source capsule);

2. Protection of primary containment;

3. Method of sealing containment;

4. Containment construction materials;

5. Form of contained radioactive material;

6. Maximum temperature withstood during prototype tests;

7. Maximum pressure withstood during prototype tests;

8. Maximum quantity of contained radioactive material;

9. Radiotoxicity of contained radioactive material; and

10. Operating experience with identical sources or devices or similarly designed and constructed devices;

(13) Manufacture and distribution of radioactive material for certain in vitro clinical or laboratory testing under general license. In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture or distribute radioactive material for use under the general license of 1200–2–10–.10(7) will be issued only if:

(a) The radioactive material is to be prepared for distribution in prepackaged units of:

1. Iodine–125 in units not exceeding 10 microcuries each.

2. Iodine–131 in units not exceeding 10 microcuries each.

3. Carbon–14 in units not exceeding 10 microcuries each.

4. Hydrogen–3 (tritium) in units not exceeding 50 microcuries each.

5. Iron–59 in units not exceeding 20 microcuries each.

6. Cobalt–57 in units not exceeding 10 microcuries each.

7. Selenium–75 in units not exceeding 10 microcuries each.

8. Mock Iodine–125 in units not exceeding 0.05 microcurie of iodine–129 and 0.005 microcurie of americium–241 each.

(b) Each prepackaged unit bears a durable, clearly visible label:

1. Identifying the radioactive contents as to chemical form and radionuclide, and indicating that the amount of radioactivity does not exceed 10 microcuries of iodine–131, iodine–125, cobalt–57, selenium–75, or carbon–14; 50 microcuries of hydrogen–3 (tritium); 20 microcuries of iron–59; or Mock Iodine–125 in units not exceeding 0.05 microcurie of iodine–129 and 0.005 microcurie of americium–241 each; and

2. Displaying the radiation caution symbol and the words, "Caution, Radioactive Material" and "Not for Internal or External Use in Humans or Animals."

(c) One of the following statements, as appropriate, or a substantially similar statement that contains the information called for in one of the following statements, appears on a label affixed to each prepackaged unit or appears in a leaflet or brochure that accompanies the package:

1. This radioactive material may be received, acquired, possessed and used only by physicians, veterinarians in the practice of veterinary medicine, clinical laboratories or hospitals and only for in vitro clinical or laboratory tests not involving internal or external administration of the material, or the radiation there from, to human beings or animals. Its receipt, acquisition, possession, use and transfer are subject to the regulations and a general license of the U.S. Nuclear Regulatory Commission or of a state with which the Commission has entered into an agreement for the exercise of regulatory authority.

_____
(Name of Manufacturer)

2. This radioactive material may be received, acquired, possessed and used only by physicians, veterinarians in the practice of veterinary medicine, clinical laboratories or hospitals and only for in vitro clinical or laboratory tests not involving internal or external administration of the material, or the radiation there from, to human beings or animals. Its receipt, acquisition, possession, use and transfer are subject to the regulations and a general license of a Licensing State.

_____
(Name of Manufacturer)

(d) The label affixed to the unit, or the leaflet or brochure that accompanies the package, contains adequate information as to the precautions to be observed in handling and storing such radioactive material. In the case of the Mock Iodine–125 reference or calibration source, the information accompanying the source must also contain directions to the licensee regarding the waste disposal requirements set out in 1200–2–5–.120.

Deleted: Iodine–12

(14) Distribution of radioactive material in exempt quantities [13].

(a) An application for a specific license to distribute NARM to persons exempt from these regulations pursuant to 1200–2–10–.04(3) will be approved if:

1. The radioactive material is not contained in any food, beverage, cosmetic, drug, or other commodity designed for ingestion or inhalation by, or application to, a human being;

2. The radioactive material is in the form of processed chemical elements, compounds, or mixtures, tissue samples, bioassay samples, counting standards, plated or encapsulated sources, or similar substances, identified as radioactive and to be used for its radioactive properties, but is not incorporated into any

[13] *See footnote 2 of this chapter.*

manufactured or assembled commodity, product, or device intended for commercial distribution; and

3. The applicant submits copies of prototype labels and brochures and the Division approves such labels and brochures.

(b) The license issued under 1200–2–10–.13(14)(a) is subject to the following conditions:

1. No more than ten (10) exempt quantities shall be sold or transferred in any single transaction. However, an exempt quantity may be composed of fractional parts of one or more of the exempt quantity provided the sum of the fractions shall not exceed unity.

2. Each exempt quantity shall be separately and individually packaged. No more that 10 such packaged exempt quantities shall be contained in any outer package for transfer to persons exempt pursuant to 1200–2–10–.04(3). The outer package shall be such that the dose rate at the external surface of the package does not exceed 0.5 millirem per hour.

3. The immediate container of each quantity or separately packaged fractional quantity of radioactive material shall bear a durable, legible label that:

(i) Identifies the radionuclide and the quantity of radioactivity; and

(ii) Bears the words "Radioactive Material."

4. In addition to the labeling information required by 1200–2–10–.13(14)(b)3., the label affixed to the immediate container, or an accompanying brochure, shall:

(i) State that the contents are exempt from Licensing State requirements;

(ii) Bear the words "Radioactive Material – Not for Human Use – Incorporation into Foods, Beverages, Cosmetics, Drugs, or Medicinals, or into Products Manufactured for Commercial Distribution is Prohibited – Exempt Quantities Should Not Be Combined"; and

(iii) Set forth appropriate additional radiation safety precautions and instructions relating to the handling, use, storage and disposal of the radioactive material.

(c) Each person licensed under 1200–2–10–.13(14) shall maintain records identifying, by name and address, each person to whom radioactive material is transferred for use under 1200–2–10–.04(3) or the equivalent regulations of a Licensing State, and stating the kinds and quantities of radioactive material transferred. An annual summary report stating the total quantity of each radionuclide transferred under the specific license shall be filed with the Division. Each report shall cover the year ending June 30, and shall be filed within 30 days thereafter. If no transfers of radioactive material have been made pursuant to 1200–2–10–.13(14) during the reporting period, the report shall so indicate.

(15) Incorporation of naturally occurring and accelerator–produced radioactive material into gas and aerosol detectors. An application for a specific license authorizing the incorporation of NARM into gas and aerosol detectors to be distributed to persons exempt under 1200–2–10–.04(2)(i) will be approved if the application satisfies requirements equivalent to those

contained in Section 32.26 of 10 CFR Part 32. The maximum quantity of radium–226 in each device shall not exceed 0.1 microcurie.

(16) Manufacture of calibration sources containing americium–241, plutonium or radium–226 for distribution to persons generally licensed under 1200–2–10–.10(4). In addition to the requirements set forth in 1200–2–10–.12, a specific license to manufacture calibration and reference sources containing americium–241, plutonium or radium–226 to persons generally licensed under 1200–2–10–.10(4) will be issued only if the requirements of Sections 32.57, 32.58, 32.59, and 32.102 of 10 CFR Part 32 and Section 70.39 of 10 CFR Part 70 or their equivalent are met.

(17) Emergency preparedness.

(a) Emergency preparedness for possession of radioactive material other than uranium and plutonium.

1. In addition to the requirements set forth in 1200–2–10–.12, all specific licenses issued, or for which an initial application or an application to amend is submitted, to possess radioactive materials in unsealed form, on foils or plated sources, or sealed in glass in excess of the quantities in Table RHS 7–2 must contain either:

(i) An evaluation showing that the maximum dose to a persón offsite due to a release of radioactive materials would not exceed 1 rem effective dose equivalent or 5 rems to the thyroid; or

(ii) An emergency plan for responding to a release of radioactive material.

2. One or more of the following factors may be used to support an evaluation submitted under (a)1.(i) of this paragraph:

(i) The radioactive material is physically separated so that only a portion could be involved in an accident;

(ii) All or part of the radioactive material is not subject to release during an accident because of the way it is stored or packaged;

(iii) The release fraction in the respirable size range would be lower than the release fraction shown in Table RHS 7–2 due to the chemical or physical form of the material;

(iv) The solubility of the radioactive material would reduce the dose received;

(v) Facility design or engineered safety features in the facility would cause the release fraction to be lower than shown in Table RHS 7–2;

(vi) Operating restrictions or procedures would prevent a release fraction as large as that shown in Table RHS 7–2; or

(vii) Other factors appropriate for the specific facility.

Table RHS 7-2 Quantities of radioactive materials requiring consideration of the need for an emergency plan for responding to a release.

Deleted:

Radioactive material [a]	Release fraction	Quantity (curies)
Actinium-228	0.001	4,000
Americium-241	0.001	2
Americium-242	0.001	2
Americium-243	0.001	2
Antimony-124	0.01	4,000
Antimony-126	0.01	6,000
Barium-133	0.01	10,000
Barium-140	0.01	30,000
Bismuth-207	0.01	5,000
Bismuth-210	0.01	600
Cadmium-109	0.01	1,000
Cadmium-113	0.01	80
Calcium-45	0.01	20,000
Californium-252	0.001	9 (20 mg)
Carbon-14	0.01	50,000
	Non CO	
Cerium-141	0.01	10,000
Cerium-144	0.01	300
Cesium-134	0.01	2,000
Cesium-137	0.01	3,000
Chlorine-36	0.5	100
Chromium-51	0.01	300,000
Cobalt-60	0.001	5,000
Copper-64	0.01	200,000
Curium-242	0.001	60
Curium-243	0.001	3
Curium-244	0.001	4
Curium-245	0.001	2
Europium-152	0.01	500
Europium-154	0.01	400
Europium-155	0.01	3,000
Germanium-68	0.01	2,000
Gadolinium-153	0.01	5,000
Gold-198	0.01	30,000
Hafnium-172	0.01	400
Hafnium-181	0.01	7,000
Holmium-166m	0.01	100
Hydrogen-3	0.5	20,000
Iodine-125	0.5	10
Iodine-131	0.5	10
Indium-114m	0.01	1,000
Iridium-192	0.001	40,000
Iron-55	0.01	40,000
Iron-59	0.01	7,000
Krypton-85	1.0	6,000,000
Lead-210	0.01	8
Manganese-56	0.01	60,000
Mercury-203	0.01	10,000
Molybdenum-99	0.01	30,000
Neptunium-237	0.001	2
Nickel-63	0.01	20,000
Niobium-94	0.01	300
Phosphorus-32	0.5	100
Phosphorus-33	0.5	1,000
Polonium-210	0.01	10
Potassium-42	0.01	9,000
Promethium-145	0.01	4,000
Promethium-147	0.01	4,000
Ruthenium-106	0.01	200
Samarium-151	0.01	4,000
Scandium-46	0.01	3,000
Selenium-75	0.01	10,000
Silver-110m	0.01	1,000
Sodium-22	0.01	9,000
Sodium-24	0.01	10,000
Strontium-89	0.01	3,000
Strontium-90	0.01	90
Sulfur-35	0.5	900
Technetium-99	0.01	10,000
Technetium-99m	0.01	400,000
Tellurium-127m	0.01	5,000
Tellurium-129m	0.01	5,000
Terbium-160	0.01	4,000
Thulium-170	0.01	4,000
Tin-113	0.01	10,000
Tin-123	0.01	3,000
Tin-126	0.01	1,000
Titanium-44	0.01	100
Vanadium-48	0.01	7,000
Xenon-133	1.0	900,000
Yttrium-91	0.01	2,000
Zinc-65	0.01	5,000
Zirconium-93	0.01	400
Zirconium-95	0.01	5,000
Any other beta-gamma emitter	0.01	10,000
Mixed fission products	0.01	1,000
Mixed corrosion products	0.01	10,000
Contaminated equipment beta-gamma	0.001	10,000
Irradiated material, any form other than solid noncombustible	0.01	1,000
Irradiated material, solid noncombustible	0.001	10,000
Mixed radioactive waste, beta-gamma	0.01	1,000
Packaged mixed waste, beta-gamma [b]	0.001	10,000
Any other alpha emitter	0.001	2
Contaminated equipment, alpha	0.0001	20
Packaged waste, alpha [b]	0.0001	20
Combinations of radioactive materials listed above [a]		

[a] For combinations of radioactive materials, consideration of the need for an emergency plan is required if the sum of the ratios of the quantity of each radioactive material authorized to the quantity listed for that material in Table RHS 7-2 exceeds one.

[b] Waste packaged in Type B containers does not require an emergency plan.

(b) Emergency preparedness for possession of uranium hexafluoride.

1. In addition to the requirements set forth in 1200–2–10–.12, all specific licenses to possess uranium hexafluoride in excess of 50 kilograms in a single container or 1000 kilograms total must contain either:

   (i) An evaluation showing that the maximum intake of uranium by a member of the public due to a release would not exceed 2 milligrams; or

   (ii) An emergency plan for responding to the radiological hazards of an accidental release of source material and to any associated chemical hazards directly incident thereto.

2. One or more of the following factors may be used to support an evaluation submitted under (b)1.(i) of this paragraph:

   (i) All or part of the radioactive material is not subject to release during an accident because of the way it is stored or packaged;

   (ii) Facility design or engineered safety features in the facility would reduce the amount of the release; or

   (iii) Other factors appropriate for the specific facility.

(c) Emergency preparedness for possession of plutonium.

1. In addition to the requirements set forth in 1200–2–10–.12, all specific licenses to possess plutonium in excess of 2 curies in unsealed form or on foils or plated sources must contain either:

   (i) An evaluation showing that the maximum dose to a member of the public offsite due to a release of plutonium would not exceed 1 rem effective dose equivalent, or

   (ii) An emergency plan for responding to the radiological hazards of an accidental release of special nuclear material and to any associated chemical hazards directly incident thereto.

2. One or more of the following factors may be used to support an evaluation submitted under (c)1.(i) of this paragraph:

   (i) The plutonium is physically separated so that only a portion could be involved in an accident;

   (ii) All or part of the plutonium is not subject to release during an accident because of the way it is stored or packaged;

   (iii) In the case of fires or explosions, the release fraction would be lower than 0.001 due to the chemical or physical form of the material;

   (iv) The solubility of the material released would reduce the dose received;

   (v) The facility design or engineered safety features in the facility would cause the release fraction to be lower than 0.001;

(vi) Operating restrictions or procedures would prevent a release large enough to cause a member of the public offsite to receive a dose exceeding 1 rem effective dose equivalent; or

(vii) Other factors appropriate for the specific facility.

(d) An emergency plan for responding to a release of radioactive material submitted under (a)1.(ii), (b)1.(ii) or (c)1.(ii) of this paragraph must include the following information:

1. Facility description. A brief description of the licensee's facility and area near the site.

2. Types of accidents. An identification of each type of accident for which protective actions may be needed.

3. Classification of accidents. A classification system for classifying accidents as alerts or site area emergencies.

4. Detection of accidents. Identification of the means of detecting each type of radioactive materials accident in a timely manner.

5. Mitigation of consequences. A brief description of the means and equipment for mitigating the consequences of each type of accident, including those provided to protect workers onsite, and a description of the program for maintaining the equipment.

6. Assessment of releases. A brief description of the methods and equipment to assess releases of radioactive material.

7. Responsibilities. A brief description of the responsibilities of licensee personnel should an accident occur, including identification of personnel responsible for promptly notifying offsite response organizations and the Division of Radiological Health; also responsibilities for developing, maintaining and updating the plan.

8. Notification and coordination. A commitment to and a brief description of the means to promptly notify offsite response organizations and request offsite assistance, including medical assistance for the treatment of contaminated, injured onsite workers when appropriate. A control point must be established. The notification and coordination must be planned so that unavailability of some personnel, parts of the facility, and some equipment will not prevent the notification and coordination. The licensee shall also commit to notify the Division of Radiological Health immediately after notification of the offsite response organizations and not later than one hour after the licensee declares an emergency [14].

Deleted: emergency. [10]

9. Information to be communicated. A brief description of the types of information on facility status, radioactive releases and recommended protective

[14] *These reporting requirements do not supersede or release licensees of complying with the requirements under the Emergency Planning and Community Right–to–Know Act of 1986, Title III, Public Law 99–499 or other state or federal reporting requirements.*

actions, if necessary, to be given to offsite response organizations and to the Division of Radiological Health.

10. Training. A brief description of the frequency, performance objectives and plan for the training that the licensee will provide workers on how to respond to an emergency including any special instructions and orientation tours the licensee would offer to fire, police, medical and other emergency personnel. The training shall familiarize personnel with site–specific emergency procedures. Also, the training shall thoroughly prepare site personnel for their responsibilities in the event of accident scenarios postulated as most probable for the specific site, including the use of team training for such scenarios.

11. Safe shutdown. A brief description of the means of restoring the facility to a safe condition after an accident.

12. Exercises. Provisions for conducting quarterly communications checks with offsite response organizations and biennial onsite exercises to test response to simulated emergencies. Quarterly communications checks with offsite response organizations must include the check and update of all necessary telephone numbers. The licensee shall invite offsite response organizations to participate in the biennial exercises. Participation of offsite response organizations in biennial exercises although recommended is not required. Exercises must use accident scenarios postulated as most probable for the specific site and the scenarios shall not be known to most exercise participants. The licensee shall critique each exercise using individuals not having direct implementation responsibility for the plan. Critiques of exercises must evaluate the appropriateness of the plan, emergency procedures, facilities, equipment, training of personnel and overall effectiveness of the response. Deficiencies found by the critiques must be corrected.

13. Hazardous chemicals. A certification that the applicant has met its responsibilities under the Emergency Planning and Community Right–to–Know Act of 1986, Title III, Public Law 99–499, if applicable to the applicant's activities at the proposed place of the use of the source material.

(e) The licensee shall allow the offsite response organizations expected to respond in case of an accident 60 days to comment on the licensee's emergency plan before submitting it to the Division of Radiological Health. The licensee shall provide any comments received within the 60 days to the Division of Radiological Health with the emergency plan.

(f) Licensees required to submit emergency plans by this paragraph shall follow the emergency plan approved by the Division of Radiological Health. The licensee may change the plan without Division of Radiological Health approval if the changes do not decrease the effectiveness of the plan. The licensee shall furnish the change to the Division of Radiological Health and to affected offsite response organizations within six months after the change is made. Proposed changes that decrease the effectiveness of the approved emergency plan may not be implemented without application to and prior approval by the Division of Radiological Health.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed March 31, 1992; effective May 15, 1992. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–10–.14 SPECIFIC LICENSES FOR CERTAIN GROUPS OF MEDICAL USES OF RADIOACTIVE MATERIAL.

(1) Subject to provisions of (2), (3), (4), and (5) of this rule, an application for a specific license pursuant to 1200–2–10–.13(1), (2), or (3) for any medical use or uses of radioactive material specified in one or more of Groups I to VI, inclusive, of (6) of this rule will be approved for all of the uses within the group or groups that include the use or uses specified in the application if:

(a) The applicant satisfies the requirements of 1200–2–10–.13(1), (2), or (3);

(b) The applicant, or the physician designated in the application as the individual user, has clinical experience as outlined in Rule 1200–2–10–.33 in the types of uses included in the group or groups;

(c) The applicant or the physician designated in the application as the individual user and all other personnel who will be involved in the preparation and use of the radioactive material have training and experience in the handling of radioactive material in the uses included in the group or groups;

(d) The applicant will have radiation detection and measuring instrumentation for conducting the procedures involved in the uses included in the group or groups; and

(e) The applicant has radiation safety operating procedures for handling and disposal of the radioactive material involved in the uses included in the group or groups.

(2) Any licensee who is authorized to use radioactive material pursuant to one or more groups in (1) and (6) of this rule is subject to the following conditions:

(a) For Groups I, II, IV and V no licensee shall receive, posses or use radioactive material except as a radiopharmaceutical manufactured in the form to be administered to the patient, labeled, packaged and distributed in accordance with:

1. A specific license issued to the manufacturer by the Division pursuant to 1200–2–10–.13(10); or

2. A specific license issued to the manufacturer by the U.S. Nuclear Regulatory Commission pursuant to §32.72 of 10 CFR Part 32, an Agreement State or a Licensing State pursuant to equivalent licensing requirements;

(b) For Group III

1. No licensee shall receive, possess or use generators or reagent kits containing radioactive material or shall use reagent kits that do not contain radioactive material to prepare radiopharmaceuticals containing radioactive material, except:

(i) Reagent kits not containing radioactive material that are approved by the Division, U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State for use by persons licensed pursuant to this rule for Group III or equivalent regulations;

(ii) Generators or reagent kits containing radioactive material that are manufactured, labeled, packaged and distributed in accordance with a specific license issued by the Division pursuant to 1200–2–10–.13(11), a specific license issued by the U.S. Nuclear Regulatory Commission pursuant to §32.73 of 10 CFR Part 32, or a specific license issued by an Agreement State or a Licensing State pursuant to equivalent regulation.

2. Any licensee who uses generators or reagent kits shall:

(i) Elute the generator or process radioactive material with the reagent kit in accordance with instructions approved by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State and furnished by the manufacturer on the label attached to or in the leaflet or brochure that accompanies the generator or reagent kit;

(ii) Before administration to patients, cause each elution or extraction of technetium–99m from a molybdenum–99/technetium–99m generator to be tested to determine either the total molybdenum–99 activity or the concentration of molybdenum–99. This testing shall be conducted according to written procedures and by personnel who have been specifically trained to perform the test;

(iii) Prohibit the administration to patients of technetium–99m containing more than one (1) microcurie of molybdenum–99 per millicurie of technetium–99m, or more than five (5) microcuries of molybdenum–99 per administered dose, at the time of administration; and

(iv) Maintain records of the molybdenum–99 test conducted on each elution for inspection by the Division.

(c) For Group VI

1. No licensee shall receive, possess or use radioactive material except as contained in a sealed source or device that has been manufactured, labeled, packaged and distributed in accordance with:

(i) A specific license issued by the Division pursuant to 1200–2–10–.13(12);

(ii) A specific license issued to the manufacturer by the U.S. Nuclear Regulatory Commission pursuant to §32.74 of 10 CFR Part 32, or a specific license issued to the manufacturer by an Agreement State or Licensing State pursuant to equivalent regulation.

2. Any licensee who possesses and uses sources or devices containing radioactive material shall:

(i) Cause each sealed source or device containing more than one hundred (100) microcuries of radioactive material with a half–life greater than 30 days, except iridium–192 seeds encased in nylon ribbon, to be tested for contamination and/or leakage at intervals not to exceed six (6) months or at such other intervals as are approved by the Division, the U.S. Nuclear Regulatory Commission, any Agreement State or Licensing State and described by the manufacturer on the label attached to the source, device or permanent container thereof, or in a leaflet or brochure that accompanies the source or device. Each source or device shall be so

tested prior to its first use unless the supplier furnished a certificate that the source or device has been so tested within six (6) months prior to the transfer;

(ii) Assure that the test required by 1200–2–10–.14(2)(c)2(i) shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample or, in the case of radium, the escape of radon at the rate of 0.001 microcurie per 24 hours. The test sample shall be taken from the source or from the surfaces of the device in which the source is mounted or stored on which one might expect contamination to accumulate. Records of leak test results shall be kept in units of microcuries and maintained for inspection by the Division;

(iii) If the test required by 1200–2–10–.14(2)(c)2.(i), reveals the presence of 0.005 microcurie or more of removable contamination or, in the case of radium, the escape of radon at the rate of 0.001 microcurie or more per 24 hours, immediately withdraw the source from use and cause it to be decontaminated and repaired or to be disposed of in accordance with Division regulations. A report shall be filed with the Division, at the address in Rule 1200–2–4–.07, within five (5) days of the test ; the report shall describe the equipment involved, the test results, and the corrective action taken;

(iv) Follow the radiation safety and handling instructions approved by the Division, the U.S. Nuclear Regulatory Commission, or another Agreement State and furnished by the manufacturer on the label attached to the source, device or permanent container thereof, or in a leaflet or brochure that accompanies the source or device, and maintain such instruction in a legible and available form;

(v) Maintain a written accountability of the issue from storage and return to storage of all sealed sources. This record shall include but is not limited to the following information: dates, number of sealed sources, location of use, quantity of radioactive material in each sealed source and signature of individual(s) involved in each removal from and each return to storage;

(vi) Conduct a physical inventory at least quarterly to account for all sealed sources and devices received and possessed. Records of the inventories shall be maintained for inspection by the Division and shall include the quantities and kinds of radioactive material, location of sources and devices, and the date of inventory;

(vii) Assure that sealed sources or standard medical applicator cells containing cobalt–60 as wire are not opened while in the licensee's possession unless specifically authorized by a license issued by this Division;

(viii) Release of individuals treated with temporary implants.

(I) Immediately after removing the last temporary implant source from an individual, the licensee shall make a radiation survey of the patient or the human research subject with a radiation detection survey instrument to confirm that all sources have been removed. The licensee shall not release from confinement for medical care an individual treated by temporary implant until all sources have been removed.

(II) A licensee shall retain a record of surveys for three (3) years. Each record shall include the date of the survey, the name of the individual, the dose rate from the individual expressed as millirem per hour and measured at 1–meter from the individual, the survey instrument used, and the initials of the individual who made the survey; and——

(ix) Comply with the provisions of 1200–2–7–.03(3) and (4).

(d) For Groups I, II, and III any licensee using radioactive material for clinical procedures other than those specified in the product labeling (package insert) shall comply with the product labeling regarding:

1. Chemical and physical form;

2. Route of administration; and

3. Dosage range.

(e) For groups IV, V and VI. Release of individuals containing radiopharmaceuticals or permanent implants.

1. The licensee may authorize the release from its control of any individual who has been administered radiopharmaceuticals or permanent implants containing radioactive material if the total effective dose equivalent to any other individual from exposure to the released individual is not likely to exceed 5 millisieverts (0.5 rem).

2. The licensee shall provide the released individual with instructions, including written instructions, on actions recommended to maintain doses to other individuals ALARA if the total effective dose equivalent to any other individual is likely to exceed 1 millisievert (0.1 rem). If the dose to a breast–feeding infant or child could exceed 1 millisievert (0.1 rem) assuming there were no interruption of breast–feeding, the instructions shall also include:

(i) Guidance on the interruption or discontinuation of breast–feeding and

(ii) Information on the consequences of failure to follow the guidance.

3. The licensee shall maintain a record of the basis for authorizing the release of an individual, for three (3) years after the date of release, if the total effective dose equivalent is calculated by:

(i) Using the retained activity rather than the activity administered,

(ii) Using an occupancy factor less than 0.25 at 1–meter,

(iii) Using the biological or effective half–life, or

(iv) Considering the shielding by tissue.

4. The licensee shall maintain a record, for three (3) years after the date of release, that instructions were provided to a breast–feeding woman if the radiation dose

to the infant or child from continued breast–feeding could result in a total effective dose equivalent exceeding 5 millisieverts (0.5 rem).

(3) Any licensee who is licensed pursuant to 1200–2–10–.14(1) for one or more of the medical use groups in this rule is authorized, subject to the provisions of paragraphs (3) and (4), to receive, possess and use for calibration and reference standards:

(a) Any radioactive material listed in Group I, Group II or Group III of (6) of this rule with a half–life not longer than 100 days in amounts not to exceed 15 millicuries total;

(b) Any radioactive material listed in Group I, Group II or Group III of (6) of this rule with a half–life greater than 100 days in amounts not to exceed 200 microcuries total;

(c) Technetium–99m in amounts not to exceed 30 millicuries;

(d) Any radioactive material, in amounts not to exceed 3 millicuries per source, contained in calibration or reference sources that have been manufactured, labeled, packaged and distributed in accordance with:

1. A specific license issued by the Division pursuant to 1200–2–10–.13(12);

2. A specific license issued by the U.S. Nuclear Regulatory Commission pursuant to §32.74 of 10 CFR Part 32; or

3. A specific license issued to the manufacturer by an Agreement State or Licensing State pursuant to equivalent regulations.

(4) Any licensee who possesses sealed sources as calibration or reference sources pursuant to 1200–2–10–.14(3) shall:

(a) Cause each sealed source containing radioactive material, other than hydrogen–3, with a half–life greater than thirty days in any form other than gas to be tested for leakage and/or contamination at intervals not to exceed six months. In the absence of a certificate from a transferor indicating that a test has been made within six (6) months prior to the transfer, the sealed source shall not be used until tested, provided, however, that no leak tests are required when:

1. The source contains 100 microcuries or less of beta and/or gamma emitting material or 10 microcuries or less of alpha emitting material; or

2. The sealed source is stored and is not being used; such sources shall, however, be tested for leakage prior to any use or transfer unless they have been leak tested within six (6) months prior to the date of use or transfer;

(b) Assure that the leak test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. The test sample shall be taken from the sealed source or from the surface of the device in which the sealed source is mounted or stored on which contamination might be expected to accumulate. Records of leak test results shall be kept in units of microcuries and maintained for inspection by the Division;

(c) If the leak test reveals the presence of 0.005 microcurie or more of removable contamination, immediately withdraw the sealed source from use and cause it to be decontaminated and repaired or to be disposed of in accordance with Division

regulations. A report shall be filed within five (5) days of the test with the Division describing the equipment involved, the test results and the corrective action taken.

(5) Any licensee or registrant who possesses and uses calibration and reference sources pursuant to 1200–2–10–.14(3)(d) shall:

(a) Follow the radiation safety and handling instructions approved by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State and furnished by the manufacturer on the label attached to the source, or permanent container thereof, or in the leaflet or brochure that accompanies the source, and maintain such instruction in a legible and available form; and

(b) Conduct a quarterly physical inventory to account for all sources received and possessed. Records of the inventories shall be maintained for inspection by the Division and shall include the quantities and kinds of radioactive material, location of sources, and the date of the inventory.

(6) Groups of medical uses of radioactive material.

(a) Group I. Use of prepared radiopharmaceuticals for certain diagnostic studies involving measurements of uptake, dilution and excretion. This group does not include uses involving imaging and tumor localization.

1. Iodine–123 as sodium iodide;

2. Iodine–125 as sodium iodide, iodinated human serum albumin, oleic acid or sodium iothalamate;

3. Iodine–131 as sodium iodide, iodinated human serum albumin, labeled rose bengal, triolein, or sodium iodohippurate;

4. Cobalt–57 as labeled cyanocobalamin;

5. Cobalt–58 as labeled cyanocobalamin;

6. Cobalt–60 as labeled cyanocobalamin;

7. Chromium–51 as sodium chromate or labeled human serum albumin;

8. Potassium–42 as chloride;

9. Sodium–24 as chloride;

10. Iron–59 as citrate;

11. Technetium–99m as pertechnetate; and

12. Any radioactive material in a radiopharmaceutical and for a diagnostic use involving measurements of uptake, dilution or excretion for which a "Notice of Claimed Investigational Exemption for a New Drug" (IND) has been accepted by the U.S. Food and Drug Administration (FDA).

(b) Group II. Use of prepared radiopharmaceuticals for diagnostic imaging and localization studies.

1. Iodine–131 as sodium iodide, iodinated human serum albumin, macro–aggregated iodinated human serum albumin, colloidal (micro–aggregated) iodinated human serum albumin, rose bengal or sodium iodohippurate;

2. Iodine–125 as sodium iodide or fibrinogen;

3. Iodine–123 as sodium iodide;

4. Chromium–51 as human serum albumin;

5. Fluorine–18 in solution;

6. Gallium–67 as citrate;

7. Gold–198 in colloidal form;

8. Mercury–197 as chlormerodrin;

9. Mercury–203 as chlormerodrin;

10. Selenium–75 as selenomethionine;

11. Strontium–85 as nitrate;

12. Strontium–87m as chloride;

13. Technetium–99m as pertechnetate, sulfur colloid or macro–aggregated human serum albumin;

14. Thallium–201 as chloride;

15. Ytterbium–169 as pentatate sodium;

16. Indium–113m as chloride;

17. Any radiopharmaceutical prepared from a reagent kit listed in (c)3. of this paragraph; and

18. Any radioactive material in a radiopharmaceutical and for a diagnostic use involving imaging or localizing for which a "Notice of Claimed Investigational Exemption for a New Drug" (IND) has been accepted by the U.S. Food and Drug Administration (FDA).

(c) Group III. Use of generators and reagent kits for the preparation and use of radiopharmaceuticals for certain diagnostic studies.

1. Molybdenum–99/technetium–99m generators for the elution of technetium–99m as pertechnetate;

2. Technetium–99m as pertechnetate for use with reagent kits for preparation and use of radiopharmaceuticals containing technetium–99m as provided in (c)3. and (c)5. of this subparagraph;

3. Reagent kits for preparation of technetium–99m labeled:

(i) Sulfur colloid;

(ii) Pentatate sodium;

(iii) Etidronate sodium;

(iv) Human serum albumin;

(v) Human serum albumin microspheres;

(vi) Polyphosphates;

(vii) Macroaggregated human serum albumin;

(viii) Medronate sodium;

(ix) Stannous pyrophosphate;

(x) Gluceptate sodium;

(xi) Oxidronate sodium;

(xii) Disofenin;

(xiii) Succimer.

4. Tin–113/indium–113m generators for the elution of indium–113m as chloride; and

5. Any generator or reagent kit for preparation and diagnostic use of a radiopharmaceutical for which generator or reagent kit a "Notice of Claimed Investigational Exemption for a New Drug" (IND) has been accepted by the U.S. Food and Drug Administration (FDA).

(d) Group IV. Use of prepared radiopharmaceuticals for certain therapeutic uses that do not normally require hospitalization for purposes of radiation safety:

1. Iodine–131 as iodide for treatment of hyperthyroidism and cardiac dysfunction;

2. Phosphorus–32 as soluble phosphate for treatment of polycythemia vera, leukemia and bone metastases;

3. Phosphorus–32 as colloidal chromic phosphate for intracavitary treatment of malignant effusions;

4. Any therapeutic material in a radiopharmaceutical for a therapeutic use not normally requiring hospitalization for purposes of radiation safety for which a "Notice of Claimed Investigational Exemption for a New Drug" (IND) has been accepted by the U.S. Food and Drug Administration (FDA).

(e) Group V. Use of prepared radiopharmaceuticals for certain therapeutic uses that normally require hospitalization for purposes of radiation safety:

1. Gold–198 as colloid for intracavitary treatment of malignant effusions;

2. Iodine–131 as iodide for treatment of thyroid carcinoma;

3. Any radioactive material in a radiopharmaceutical and for a therapeutic use normally requiring hospitalization for radiation safety reasons for which a "Notice of Claimed Investigational Exemption for a New Drug" (IND) has been accepted by the U.S. Food and Drug Administration (FDA).

(f) Group VI. Use of sealed sources and devices containing radioactive material for certain medical uses:

1. Americium–241 as a sealed source in a device for bone mineral analysis;

2. Cesium–137 encased in needles and applicator cells for topical, interstitial, and intracavitary treatment of cancer;

3. Cobalt–60 encased in needles and applicator cells for topical, interstitial, and intracavitary treatment of cancer;

4. Gold–198 as seeds for interstitial treatment of cancer;

5. Iodine–125 as a sealed source in a device for bone mineral analysis;

6. Iridium–192 as seeds encased in a nylon ribbon for interstitial treatment of cancer;

7. Strontium–90 sealed in an applicator for treatment of superficial eye condition;

8. Radon–222 as seeds for interstitial treatment of cancer;

9. Radium–226 encased in needles, applicator cells, and plaques for topical, interstitial and intracavitary treatment of cancer; and

10. Iodine–125 as seeds for interstitial treatment of cancer.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–10–.15 ISSUANCE OF SPECIFIC LICENSES.

(1) Upon a determination that an applicant meets the requirements of the Act and the regulations of the Division will issue a specific license authorizing the proposed activity in such form and containing such conditions and limitations as it deems appropriate or necessary.

(2) The Division may incorporate in any license at the time of issuance, or thereafter by amendment, such additional requirements and conditions with respect to the licensee's receipt, possession, use and transfer of radioactive material subject to this chapter as it deems appropriate or necessary in order to:

(a) Protect the public health and safety or property;

(b) Require such reports and the keeping of such records and provide for such inspection of activities under the license as may be necessary to evaluate activities conducted under the license; and

(c) Prevent loss or theft of material subject to this chapter.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.16 SPECIFIC TERMS AND CONDITIONS OF LICENSES.

(1) Each license issued pursuant to this chapter shall be subject to all provisions of the Act, now or hereafter in effect, and to all rules, regulations, and orders of the Division.

(2) Neither the license nor any right under the license shall be assigned or otherwise transferred in violation of the provisions of the Act.

(3) Each person licensed by the Division pursuant to this chapter shall confine his use and possession of the material licensed to the locations and purposes authorized in the license.

(4) Each licensee authorized under 1200–2–10–.13(5) to distribute certain devices to generally licensed persons shall:

(a) Report to the Division within 30 days after the end of each calendar quarter all transfers of such devices to persons generally licensed under 1200–2–10–.10(2) or, if no transfers have been made during the reporting period, the report shall so indicate. For all transfers the report shall identify each general licensee by name and address, an individual by name and/or position who may constitute a point of contact between the Division and the general licensee, the type and model number of device transferred and the quantity and type of radioactive material contained in the device; and

(b) Furnish to each general licensee in this State to whom he transfers such device a copy of the general license contained in 1200–2–10–.10(2).

(5) Each specific licensee shall notify the Division in writing when the licensee decides to permanently discontinue all activities involving radioactive materials authorized under the license.

(6) Each licensee preparing technetium–99m radiopharmaceuticals from molybdenum–99/technetium–99m generator shall test the generator eluates for molybdenum–99 breakthrough in accordance with 1200–2–10–.14(2)(b)2.

(7) ~~Each licensee shall~~ Each specific licensee and each general licensee meeting the criteria of part 1200-2-10-.10(2)(b)14 shall:

(a) Provide the Division written notification, at the address in Rule 1200–2–4–.07, immediately following the filing of a voluntary or involuntary petition for bankruptcy under any Chapter of Title 11 (Bankruptcy) of the United States Code (U.S.C.):

1. By or against the licensee;

2. By or against an entity (as that term is defined in 11 U.S.C. 101(14)) controlling the licensee or listing the license or licensee as property of the estate; or

3. By or against an affiliate (as that term is defined in 11 U.S.C. 101(2)) of the licensee;

(b) Include in the notification required in (7)(a) of this rule the bankruptcy court in which the petition for bankruptcy was filed; and

(c) Include in the notification required in (7)(a) of this rule the date of the filing of the petition.

(8) When temporary job–sites are authorized on a specific license, radioactive material may be used at temporary job–sites, in areas not under exclusive federal jurisdiction, throughout the State of Tennessee.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed May 5, 1988; effective August 29, 1988. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006. Amendment filed March 12, 2007; effective May 26, 2007.*

~~**1200–2–10–.17 EXPIRATION OF LICENSES.**~~

~~Except as provided in 1200–2–10–.18(2), each specific license shall expire at the end of the day, in the month and year stated therein.~~

1200–2–10–.17 Expiration and Termination of Licenses and Decommissioning of Sites and Separate Buildings or Outdoor Areas.

(1) Expiration of specific licenses. Except as provided in 1200–2–10–.17(2), each specific license shall expire at the end of the day, in the month and year stated therein.

(2) Termination of specific licenses:

(a) Specific licenses shall continue in effect, beyond the expiration date if necessary, with respect to possession of radioactive material until the Division notifies the licensee in writing that the license is terminated. During this time, the licensee shall:

1. Limit actions involving radioactive material to those related to decommissioning; and

2. Continue to control entry to restricted areas until they are suitable for release in accordance with Division requirements.

(b) Specific licenses, including expired licenses, will be terminated by written notice to the licensee when the Division determines that:

1. The licensee has properly disposed of radioactive material;

2. The licensee has made reasonable effort to eliminate residual radioactive contamination, if present; and

3. The premises are suitable for release in accordance with Division requirements. The licensee may demonstrate suitability for release by:

(i) Performance of the radiation survey described in 1200-2-10-.17(3)(d)2, or

(ii) Submission of other information that the Division determines is acceptable;

4. The licensee has complied with any requests for information from the Division; and

5. The licensee has submitted a written request for license termination to the Division.

(3) Decommissioning of sites or separate buildings or outdoor areas:

(a) Each specific licensee shall notify the Division in writing, at the address in 1200-2-4-.07, within 60 days of any of the following occurrences:

1. The license has expired pursuant to 1200-2-10-.17(2);

2. The licensee has decided to permanently cease principal activities, as defined in this rule:

(i) At the entire site, or

(ii) In any separate building or outdoor area that contains residual radioactivity such that the building or outdoor area is unsuitable for release in accordance with Division requirements;

3. No principal activities under the license have been conducted for 24 months; or

4. No principal activities have been conducted for 24 months in any separate building or outdoor area that contains residual radioactivity such that the building or outdoor area is unsuitable for release in accordance with Division requirements.

(b) Each specific licensee:

1. If not required by 1200-2-10-.17(3)(g) to submit a decommissioning plan, shall begin decommissioning its site or any separate building or outdoor area that contains residual radioactivity within 60 days of any occurrence listed in 1200-2-10-.17(3)(a).

2. If required by 1200-2-10-.17(3)(g) to submit a decommissioning plan, shall:

(i) Submit a decommissioning plan within 12 months of notification of any occurrence listed in 1200-2-10-.17(3)(a), and

(ii) Begin decommissioning upon Division approval of that plan.

(c) Coincident with the notification required by 1200-2-10-.17(3)(a), the specific licensee shall maintain in effect all financial assurances that were established, pursuant to 1200-2-10-.12(3) in conjunction with a license issuance or renewal, or that are required by this rule.

1. The Division will determine if the licensee shall increase, or may decrease, the amount of the financial assurance to cover the detailed cost estimate for decommissioning established pursuant to 1200-2-10-.17(3)(i)5.

2. The licensee may with Division approval reduce the amount of the financial assurance as decommissioning proceeds and radiological contamination is reduced at the site.

(d) As the final steps in decommissioning, specific licensees shall:

1. Certify the disposition of all licensed material, including accumulated wastes; and

2. Demonstrate that the premises are suitable for release in accordance with Division requirements.

(i) The licensee shall:

(I) Conduct a radiation survey of the premises where the licensed activities were carried out and submit a report of the results of this survey, or

(II) Submit other information that the Division determines is acceptable.

(ii) The licensee shall, as appropriate:

(I) Report levels of gamma radiation in units of microroentgens (millisieverts) per hour at 1 meter from surfaces, and

(II) Report levels of radioactivity, including alpha and beta, in units of:

I. Disintegrations per minute or microcuries (megabecquerels) per 100 square centimeters — removable and fixed — for surfaces,

II. Microcuries (megabecquerels) per milliliter for water, and

III. Picocuries (becquerels) per gram for solids such as soils or concrete, and

(III) Specify the survey instrument(s) used and certify that each instrument was properly calibrated and tested at the time of the survey.

(e) Except as provided in 1200–2–10–.17(3)(k)(3), specific licensees shall complete decommissioning of the site or separate building or outdoor area so that the site, building or outdoor area is suitable for release in accordance with Division requirements as soon as practicable but no later than 24 months following the initiation of decommissioning.

(f) Except as provided in 1200–2–10–.17(3)(k)(3), when decommissioning involves the entire site, the specific licensee shall request license termination as soon as practicable but no later than 24 months following the initiation of decommissioning.

(g) A specific licensee shall submit a decommissioning plan if:

1. Required to do so by license condition; or

2. The Division determines that the procedures and activities necessary to carry out decommissioning of the site or separate building or outdoor area have not been previously approved by the Division and that these procedures could increase potential health and safety impacts to workers or to the public. Some examples are procedures:

(i) That would involve techniques not applied routinely during cleanup or maintenance operations;

(ii) In which workers would be entering areas not normally occupied where surface contamination and radiation levels are significantly higher than routinely encountered during operation;

(iii) That could result in significantly greater airborne concentrations of radioactive materials than are present during operation; or

(iv) That could result in significantly greater releases of radioactive material to the environment than those associated with operation.

(h) Specific licensees shall not carry out procedures with potential health and safety impacts before Division approval of the decommissioning plan.

(i) The proposed decommissioning plan for the site or separate building or outdoor area shall include:

1. A description of the conditions of the site or separate building or outdoor area sufficient to evaluate the acceptability of the plan;

2. A description of planned decommissioning activities;

3. A description of methods used to ensure protection of workers and the environment against radiation hazards during decommissioning;

4. A description of the planned final radiation survey;

5. A detailed cost estimate for decommissioning, comparison of that estimate with present funds set aside for financial assurance, and a plan for assuring the availability of adequate funds for completion of decommissioning; and

6. For decommissioning plans calling for completion of decommissioning later than 24 months after plan approval, the plan shall include a justification for the delay based on the criteria in 1200–2–10–.17(3)(k)(3).

(j) The Division will approve the proposed decommissioning plan if the information in the plan demonstrates that the licensee:

1. Will complete decommissioning as soon as practicable; and

2. Will adequately protect the health and safety of workers and the public.

(k) Requests for extensions:

1. A licensee may request a delay in initiating decommissioning.

(i) The Division may grant this delay, if the Division determines that this delay is not detrimental to the public health and safety and is otherwise in the public interest.

(ii) The request for a delay shall be submitted no later than 30 days before notification pursuant to 1200–2–10–.17(3)(a).

(iii) The schedule for decommissioning set forth in 1200–2–10–.17(3)(b) shall not start until the Division has made a determination on the request.

2. A licensee may request an alternative schedule for the submittal of a decommissioning plan. The Division may approve the alternative schedule, if the Division determines that the alternative schedule is necessary to the effective conduct of decommissioning

operations and presents no undue risk from radiation to the public health and safety and is otherwise in the public interest.

3. A licensee may request an alternative schedule for the completion of decommissioning of the site or separate building or outdoor area, and license termination if appropriate. The Division may approve the alternative schedule for completion of decommissioning, if the Division determines that it is warranted by consideration of the following:

(i) Whether it is technically feasible to complete decommissioning within the allotted 24–month period;

(ii) Whether sufficient waste disposal capacity is available to allow completion of decommissioning within the allotted 24–month period;

(iii) Whether allowing short–lived radionuclides to decay will achieve a significant volume reduction in wastes requiring disposal;

(iv) Whether allowing short–lived radionuclides to decay will achieve a significant reduction in radiation exposure to workers;

(v) Other site–specific factors that the Division may determine are beyond the control of the licensee.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–10–.18 RENEWAL OF LICENSE.

(1) Applications for renewal of specific licenses shall be filed in accordance with 1200–2–10–.11.

(2) In any case in which a licensee, not less than thirty (30) days prior to expiration of his existing license, has filed an application in proper form for renewal or for a new license authorizing the same activities, such existing license shall not expire until the application has been finally determined by the Department.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.19 AMENDMENT OF LICENSES AT REQUEST OF LICENSEE.

Applications for amendment of a license shall be filed in accordance with 1200–2–10–.11 and shall specify the respects in which the licensee desires his license to be amended and the grounds for such amendment.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.20 DIVISION ACTION ON APPLICATION TO RENEW OR AMEND.

In considering an application by a licensee to renew or amend his license, the Division will apply the criteria set forth in 1200–2–10–.12 and 1200–2–10–.13, as applicable.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.21 INALIENABILITY OF LICENSES.

No license issued or granted under this chapter and no right to possess or utilize radioactive material granted by any license issued pursuant to this chapter shall be transferred, assigned, or in any manner disposed of, either voluntarily or involuntarily, directly or indirectly, through transfer of control of any license to any person unless the Division shall, after securing full information, find that the transfer is in accordance with the provisions of the Act, and shall give its consent in writing.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.22 TRANSFER OF MATERIAL.

(1) No licensee shall transfer radioactive material except as authorized pursuant to this rule.

(2) Any licensee may transfer radioactive material:

(a) To the Division provided such transfer is accepted by the Division in writing;

(b) To the U.S. Department of Energy;

(c) To any person exempt from the regulations in this chapter to the extent permitted under such exemption;

(d) To any person authorized to receive such material under terms of a general license or its equivalent, or a specific license or equivalent licensing document, issued by the Division, the U.S. Nuclear Regulatory Commission, any Agreement State or a Licensing State; or

(e) As otherwise authorized by the Division in writing.

(3) Before transferring sources of radiation to a specific licensee of the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State, or to a general licensee who is required to register with or report to the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State prior to receipt of the source of radiation, the transferor of the source of radiation shall verify that the transferee's authorization is for the receipt of the type, form, and quantity of the source of radiation to be transferred.

(4) The following methods for the verification required in 1200–2–10–.22(3) are acceptable:

(a) The transferor may have in his possession, and read, a current copy of the transferee's specific license or registration certificate;

(b) The transferor may have in his possession a written certification by the transferee that he is authorized by license or registration certificate to receive the type, form and quantity of the source of radiation to be transferred, specifying the license or registration certificate number, issuing agency, and expiration date;

(c) For emergency shipments the transferor may accept oral certification containing all of the information specified in 1200–2–10–.22(4)(b) provided that written certification is forwarded to the transferor within ten (10) days following the oral communication;

(d) The transferor may obtain other information compiled by a reporting service from official records of the Division, the U.S. Nuclear Regulatory Commission or the licensing agency of any state as to the identity of licensees and the scope and expiration dates of licenses and registrations; or

(e) When none of the methods of verification described in 1200–2–10–.22(4)(a) through (d) are readily available or when a transferor desires to verify that information received by one of such methods is correct or up–to–date, the transferor may obtain and record confirmation from the Division, the U.S. Nuclear Regulatory Commission, or the licensing agency of any state that the transferee is authorized to receive the source of radiation.

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed March 12, 2007; effective May 26, 2007.*

1200–2–10–.23 MODIFICATION, REVOCATION, AND TERMINATION OF LICENSES.

(1) The terms and conditions of all licenses may be subject to amendment, revision, or modification or the license may be suspended or revoked by reason of amendments to the Act, or by reason of rules or regulations issued by the Department.

(2) Any license may be revoked, suspended, or modified, in whole or in part, for any material false statement in the application or in any statement of fact required under provisions of the Act, or because of conditions revealed by such application or statement of fact or any report, record, or inspection or other means that would warrant the Department to refuse to grant a license on an original application, or for violation of, or failure to observe any of the terms and conditions of the Act, or of the license, or of any rule or regulation of the Department. This action will be taken pursuant to Tennessee Code Annotated Chapter 23.

(3) The Department may terminate a specific license upon request submitted by the licensee to the Department in writing.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.24 REGISTRATION.

(1) The owner or person having possession of any radiation machine or accelerator, except those specifically exempted in 1200–2–10–.07, shall register such sources within ten (10) days after acquisition of such machine. The owner or possessor of any accelerator shall substitute an application for certified registration required in Chapter 1200–2–9. The application for certified registration must be received by the Department within ten (10) days after

acquisition of the accelerator; however, an accelerator may not be energized until registered pursuant to Chapter 1200–2–9. In addition, every person who provides inspections as provided for in 1200–2–10–.27(4) and every person who assembles, installs, or services radiation machines shall register with the Division of Radiological Health, Tennessee Department of Environment and Conservation. Registration under this rule shall be on Department Form RHS 8–4, Form RHS 8–4a or Form RHS 8–4b, as appropriate, as furnished by the Department and may be obtained from the Division of Radiological Health, L&C Annex, 3rd Floor, 401 Church Street, Nashville, Tennessee 37243–1532. A registration fee in accordance with the Classification and Fee Schedule in 1200–2–10–.24(3) shall be due upon receipt of an invoice from the Division of Radiological Health following the submittal of the completed registration form. The check for the fee shall be made payable to "Treasurer, State of Tennessee."

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(2) An annual registration fee will be due the first working day following January 1 of each year as long as the radiation machine or service is subject to registration. Each registrant shall submit the annual fee payable to, "Treasurer, State of Tennessee," in the appropriate dollar amount in accordance with the Classification and Fee Schedule in 1200–2–10–.24(3) to the Division of Radiological Health. Payment shall be accompanied by a copy of the fee invoice properly completed. The invoice for the annual fee will be dated January 17 and will require payment by March 17 of the indicated year. At the time of the annual payment a registrant of only Class II radiation machines may request specific times or list restricted hours during normal work hours for inspections pursuant to 1200–2–10–.27 by personnel of the Division of Radiological Health, Tennessee Department of Environment and Conservation.

(3) For purposes of inspections and payment of fees, the Classification and Fee Schedule shall be as follows:

(a) Radiation Machines

CLASS I	
Dental Radiation Machines: All diagnostic equipment used exclusively for dental diagnostic procedures.	$ 65.00 per tube
CLASS II	
Priority Two Medical Radiation Machines: All medical diagnostic x–ray equipment, not in Class III, used exclusively for medical or veterinary diagnostic procedures.	$ 150.00 per tube
CLASS III	
Priority One Medical Radiation Machines: All diagnostic x–ray equipment used in radiologists' offices, orthopedic surgeons' offices or hospitals exclusively for medical diagnostic procedures.	$ 200.00 per tube
CLASS IV	
Therapy Medical Radiation Machines: All x–ray equipment with energies less than 0.9 MeV used for the purpose of medical or veterinary radiation therapy.	$ 300.00 per tube

	CLASS V	
	Priority Two Industrial and Educational Radiation Machines: Closed–beam analytical radiation machines, gauges or industrial radiation machines used in shielded room or cabinet radiography.	$ 600.00 per tube
	CLASS VI	
	Priority One Industrial and Educational Radiation Machines: All x–ray machines used for industrial radiography and all open–beam analytical x–ray machines and all radiation machines not specifically included in Class I, II, III, IV, V or VII.	$ 900.00 per tube
	CLASS VII	
	Accelerators: All devices defined as accelerators as per "State Regulations for Protection Against Radiation."	$ 2,000.00 annual fee, plus an initial fee of $ 375.00 per maximum nominal rated MeV for initial certified registration review (initial review fee not to exceed $ 150,000.00)
(b)	A person providing inspection services as permitted by paragraph 1200–2–10–.27(4), except as provided by subparagraph 1200–2–10–.24(3)(f), shall pay an annual registration fee of six hundred dollars.	$ 600.00
(c)	A person providing assembly/installation/servicing, except as provided by subparagraph 1200–2–10–.24(3)(f), shall pay an annual registration fee of six hundred dollars.	$ 600.00

(d) A registrant may qualify to pay a registration fee equal to eighteen percent (18%) of that listed in this paragraph (3), subject to the following conditions:

1. All tubes subject to registration are inspected in accordance with subparagraph 1200–2–10–.27(3), (4) and (5).

   (i) For purposes of the eighteen percent (18%) fee, the first inspection performed on an x–ray tube on or after January 1, 2003, will establish a baseline date for that tube.

      (I) Each subsequent inspection of a tube shall be performed within 30 days of the appropriate anniversary of the baseline date, according to the schedule set out in 1200–2–10–.27(3)(a).

      (II) An inspection performed more than 30 days before or after the appropriate baseline date shall establish a new baseline date for that tube.

Deleted: Priority Two Industrial and Educational Radiation Machines:¶ Closed–beam analytical radiation machines, gauges or industrial radiation machines used in shielded room or cabinet radiography. ... [1]

(III) An inspection performed more than 30 days after a baseline date shall not qualify the registrant for the eighteen percent (18%) fee.

(IV) An inspection performed more than 30 days before a baseline date and meeting all other requirements found in paragraphs 1200–2–10–.27(3), (4) and (5) shall qualify the registrant for the eighteen percent (18%) fee.

(ii) Reserved.

2. Each newly acquired tube subject to registration is inspected within six (6) months of ownership or possession.

3. An individual who satisfies the requirements in paragraph 1200–2–10–.27(4) performs all inspections.

4. The registrant submits to the Division, at L&C Annex, 3rd Floor, 401 Church Street, Nashville, TN 37243–1532:

(i) Copies of the appropriate State evaluation forms within 60 days after the inspection.

(ii) Copies of applicable service reports to document correction of any deficiencies noted within 60 days after the inspection.

(iii) A signed "X–Ray Inspection Notification and Certification of Compliance" form within 60 days of the inspection.

5. Inspections found by the Division to be unsatisfactory under this subparagraph or under paragraph 1200–2–10–.27(4) or (5) shall not qualify for the 18 percent (18%) fee for the next appropriate fee year.

(i) The registrant shall correct and re–submit the report(s) and documentation of an inspection found to be unsatisfactory within 30 days of the date of notification by the Division. Failure to correct and re–submit the report(s) and documentation of an unsatisfactory inspection will result in enforcement action.

(ii) The 30–day correction period shall not establish a new baseline. It shall not:

(I) Qualify an existing tube for reduced fee for the following calendar year, or

(II) Qualify a newly acquired tube for reduced fee for the current calendar year.

(e) Reserved.

(f) A person providing inspection services, as permitted by paragraph 1200–2–10–.27(4), or a person providing assembly/installation/servicing, who is a staff member of the facility registered pursuant to Tennessee Code Annotated (T.C.A.) §68–202–101 et seq. and these regulations, and who performs such inspection services or

assembly/installation/servicing only for that registrant, shall not be subject to subparagraphs (b) and (c) above.

(4) Any failure to pay an invoiced amount by the date specified on the invoice, unless qualified by 1200–2–10–.24(3)(d) above, shall be deemed to constitute a violation of Tennessee Code Annotated §68–203–101 et seq.

(5) Whenever there is a change in information such as address, ownership, possessor or location of use from that declared on the last previous registration, the completion of a new Form RHS 8–4 shall be required within 10 days of the change.

(6) Each registrant, or his estate, who permanently discontinues the use of or transfers all of his radiation machines at an installation shall notify the Division in writing within sixty (60) days of such action. In the event of a transfer, the notification shall include the name and address of the transferee.

(7) No person shall state or imply that any activity under such a registration has been approved by the Division.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed January 26, 1993; effective March 12, 1993. Amendment filed October 1, 2001; effective December 15, 2001. Amendment filed October 17, 2001; effective December 31, 2001. September 4, 2003; effective November 18, 2003. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–10–.25 REPORTS.

(1) Any person who sells, leases, transfers, assembles, reassembles, or lends radiation machines, except those exempted from registration by 1200–2–10–.07 shall report to the Division, within thirty (30) days after the end of each calendar quarter, the name and address of persons to whom they have transferred such items and the date of transfer. Persons routinely engaged in the sale, transfer, leasing, lending, assembling, or reassembling of x–ray equipment shall report each calendar quarter, including a report for calendar quarters in which no radiation machine transfer occurs. Such reports shall be held proprietary by the Division.

(2) Each out–of–state person who brings radiation machines into the State, except those exempted in 1200–2–10–.07, for any temporary use shall:

(a) Notify the Division in writing at least three (3) days prior to engaging in such use. Such notification shall indicate the location, period and type of proposed use within the State. If, for a specific case, the 3–day period would impose an undue hardship, he may, upon application to the Division obtain permission to proceed sooner;

(b) Register the radiation machines with this Division on Form RHS 8–4 prior to entry into the State; and

(c) Comply with all applicable regulations of the Division including the payment of the fee for the Class, as appropriate, contained in 1200–2–10–.24(3).

Authority: *T.C.A. §4–5–201 et seq., 68–202–201 et seq. and 68–203–202.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed January 26, 1993; effective March 12, 1993.*

1200–2–10–.26 RECORDS.

Each licensee and registrant shall keep records showing the receipt, transfer and disposal of all sources of radiation.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.27 INSPECTIONS.

(1) Each licensee or registrant shall afford the Department at all reasonable times opportunity to inspect sources of radiation, premises, facilities and activities subject to these regulations and records maintained pursuant to these regulations.

(a) Department inspectors may consult privately with workers concerning matters of occupational radiation protection and other matters related to applicable provisions of the regulations, license and certified registration to the extent the inspectors deem necessary for the conduct of an effective and thorough inspection.

1. During the course of an inspection, any worker may bring privately to the attention of the inspectors, either orally or in writing, any past or present condition that he has reason to believe may have contributed to or caused any violation of the Act, these regulations, or license or certified registration condition, or any unnecessary exposure to radiation or radioactive material under the licensee's or registrant's control. Any such notice in writing shall comply with (2) of this rule.

2. The licensee or registrant or licensee's or registrant's representative may accompany Division inspectors during other phases of an inspection.

3. The provision of 1200–2–10–.27(1)(a)1. shall not be interpreted as authorization to disregard instructions pursuant to 1200–2–4–.12.

(b) If at the time of inspection, an individual has been authorized by the workers to represent them during inspections by the Division, the licensee or registrant shall notify the inspectors of such authorization and shall give the workers' representative an opportunity to accompany the inspectors during the inspection of physical working conditions.

1. Different representative of licensees or registrants and workers may accompany the inspectors during different phases of an inspection if there is no resulting interference with the conduct of the inspection. However, only one worker's representative at a time may accompany the inspectors.

2. Any workers' representative shall be an employee of the licensee or registrant and should be a worker as defined in 1200–2–4–.04(1)(rrr) and shall have received instructions as specified in 1200–2–4–.12.

Comment [BAD1]: must double check this

3. In addition to the licensee's or registrant's representative and with the approval of the licensee or registrant and the workers' representative, an individual who is not routinely engaged in work under control of the licensee or registrant, for example, a consultant to the licensee or registrant or to the workers'

representative, shall be afforded the opportunity to accompany Division inspectors during the inspection of physical working conditions.

4. The workers' representative for any area containing proprietary information shall be an individual previously authorized by the licensee or registrant to enter that area.

5. Notwithstanding the other provisions of this rule, Division inspectors are authorized to refuse to permit accompaniment by any individual who deliberately interferes with a fair and orderly inspection.

(2) Requests by Workers for Inspection.

(a) Any worker or representative of workers who believes that a violation of the Act, these regulations, conditions of a certified registration, or license conditions exists or has occurred in activities subject to these regulations with regard to radiological working conditions in which the worker is engaged, may request an inspection by registering a complaint of the alleged violation with the Commissioner, Tennessee Department of Environment and conservation; Director, Division of Radiological Health; or Division inspectors.

1. Any such complaint shall be in writing, shall set forth the specific grounds for the complaint and shall be signed by the worker or representative of workers.

2. A copy of the complaint shall be provided the licensee or registrant by the Division no later than at the time of inspection except that, upon request of the worker registering such complaint, his name and the name of individuals referred to therein shall not appear in such a copy or on any record published, released or made available by the Division except for good cause shown.

(b) If, upon receipt of such complaint, the Division determines that the complaint meets the requirements set forth in 1200–2–10–.27(2)(a) and that there are reasonable grounds to believe that the alleged violation exists or has occurred, an inspection will be made as soon as practicable, to determine if such alleged violation exists or has occurred. Inspections pursuant to this paragraph need not be limited to matters referred to in the request for an inspection.

(c) If it is determined that there are no reasonable grounds to believe that a violation exists or has occurred, the complainant shall be notified by the Division in writing.

(d) No licensee or registrant shall discharge or in any manner discriminate against any worker because such worker has filed any complaint or instituted or caused to be instituted any proceeding under these regulations or has testified or is about to testify in any such proceeding or because of the exercise by such worker on behalf of himself or others of any option afforded by these regulations.

(3) Inspections of radiation machines are to be conducted:

(a) According to Class as follows:

CLASS I – once every four (4) years

CLASSES II and V – once every two (2) years

CLASSES III, IV, VI and VII – annually

(b) By personnel of the Division of Radiological Health, Tennessee Department of Environment and Conservation, or

(c) As provided in 1200–2–10–.27(4), and

(d) According to the same criteria and to the satisfaction of the Division and provided the appropriate Division forms are completed and submitted along with any documentation required by subparagraph 1200–2–10–.24(3)(e), and

(e) By the Division of Radiological Health on a selected number of those facilities providing an inspection report as permitted by 1200–2–10–.27(4).

(4) The Division will accept, as inspections for a reduced registration fee as provided for in subparagraph 1200–2–10–.24(3)(d), inspections by individuals, other than employees of the Division:

(a) Whose inspections are satisfactory to the Division;

(b) Who are registered with the Division;

(c) Who are staff inspectors, or who have paid an annual registration fee to the Division; and

(d) Who meet one set of the following criteria:

Formal Education or Certification	Plus	Experience
1. Bachelor's degree in a physical science or mathematics		Four years of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed
2. Bachelor's degree in a physical science or a biological science with a physical science minor and one year of graduate work in health physics		Three years of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed
3. Master's degree in health physics or radiological health		Two years of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed
4. Doctor's degree in health physics or radiological health		One year of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed
5. Certification by the American Board of Health Physics or by the American Board of Radiology or be a Fellow, Canadian College of Physicists in Medicine		One year of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed

Formal Education or Certification	Plus	Experience
6. Two (2) notarized letters of reference from persons registered to provide inspections for reduction in fees and meeting any of the above sets of criteria certifying to the individual's capabilities to perform the necessary inspections		Five years of applied health physics experience in a program with radiation safety problems similar to those in the program to be surveyed

(5) Inspections satisfactory to the Division. The following constitute a proper inspection and must occur:

(a) The inspection of an x–ray facility subject to registration under "State Regulations for Protection Against Radiation" shall identify the compliance status of the facility and each piece of equipment subject to registration with respect to requirements in Chapters 1200–2–4, 5, 6, 8, 9 and 10.

(b) The qualified individual performing the inspection shall record the results of the inspection on evaluation forms provided by the Division, one form for each facility plus an appropriate form, or forms, for each piece of equipment. The evaluation forms shall describe the compliance status of the facility and equipment as it exists at the time of the inspection. The Division will accept computer–generated forms if these contain the same questions as Division forms contain.

(c) The qualified individual shall provide the evaluation and certification of compliance forms to the registrant promptly.

(d) The registrant shall submit evaluation and certification of compliance forms to the Division as set out in 1200–2–10–.24(3)(d).

(e) A registrant whose inspection reveals an item of non–compliance shall correct the item promptly following notification by the qualified individual. The registrant shall provide appropriate documentation of the correction to the Division as set out in 1200–2–10–.24(3).

(f) If as a result of inadvertent error or excusable neglect a tube(s) is not inspected, the Commissioner or the Commissioner's designee may grant the 18 percent (18%) fee for all other tubes provided they were timely inspected by a qualified individual.

(g) For a tube that is inoperable at the time of inspection, the qualified individual shall submit a form indicating the tube was inoperable. The tube shall be inspected within 60 days of its becoming functional.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed January 26, 1993; effective March 12, 1993. Amendment filed October 17, 2001; effective December 31, 2001. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed November 17, 2005; effective January 31, 2006.*

Deleted: *Amendment filed _____; effective _____.*

1200–2–10–.28 TESTS.

Each licensee and registrant shall perform, upon instruction from the Division, or shall permit the Division to perform, such tests as the Division may require including, but not limited to, tests of:

(1) Sources of radiation;

(2) Facilities wherein sources of radiation are used or stored;

(3) Radiation detection and monitoring instruments; and

(4) Other equipment and devices used in connection with utilization or storage of licensed or registered sources of radiation.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.29 RECIPROCAL RECOGNITION OF LICENSES.

(1) Subject to these regulations, any individual in another state who holds a specific license from the U.S. Nuclear Regulatory Commission, an Agreement State or Licensing State, and issued by the agency having primary jurisdiction, where the licensee maintains an office for directing the licensed activities and at which radiation safety records are normally maintained, may possess or use the licensed radioactive material to conduct the activities authorized by such license within this State for a period not in excess of one hundred eighty (180) days in any period of twelve (12) consecutive months and will be considered, without obtaining a specific licensing document from this Division, a licensee of this State provided that:

(a) The out–of–state licensing document does not limit the activity authorized by such document to specified installations or locations;

(b) The out–of–state licensee notifies the Division in writing at least three (3) days prior to each entry into this State to engage in such activity. Such notification shall indicate the location, period, type of proposed possession, use and supervisor within this State, and shall be accompanied by a copy of the pertinent licensing document or shall indicate in the notification that such licensing document has previously been submitted to this Division. If for a specific case, the three (3) day period would impose an undue hardship, the Division may authorize such person to proceed sooner upon notification by telephone of intent to conduct the proposed activity provided that the licensee shall file in writing the information required in this paragraph within three (3) days of the telephone notification;

(c) The out–of–state licensee shall not transfer or dispose of radioactive material possessed or used under the provisions of this rule except by transfer to a person:

1. Specifically licensed by the Division, the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State to receive such material; or

2. Exempt from the requirements for a license for such material under 1200–2–10–.04(1)(a);

(d) The out–of–state licensee complies with all applicable regulations of the Division and with all the terms and conditions of his licensing document, except any such terms and conditions that may be inconsistent with applicable regulations of the Division; and

(e) The Division may require the out–of–state licensee to supply such other information as the Division may request.

(2) Notwithstanding the provision of paragraph (1) above, any person who holds a specific license issued by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State authorizing the holder to manufacture, install, or service a device described in 1200–2–10–.10(2)(a) within the areas subject to the jurisdiction of the licensing body is hereby granted a general license to install and service such device in this State provided that:

(a) The device has been manufactured, labeled, installed and serviced in accordance with applicable provisions of the specific license issued to such person by the U.S. Nuclear Regulatory Commission, an Agreement State or a Licensing State; and

(b) Such person shall assure that any labels required to be affixed to the device under regulations of the authority that licensed manufacture of the device bear a statement that "Removal of this label is prohibited."

(3) The Division may withdraw, limit, or qualify its acceptance of any specific license or equivalent licensing document issued by another agency, or any product distributed pursuant to such licensing document, upon determining that such action is necessary in order to protect the public health and safety or property.

Authority: *T.C.A. §68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986.*

1200–2–10–.30 PACKAGING AND TRANSPORTATION OF RADIOACTIVE MATERIAL.

(1) Except as authorized in a general license or a specific license issued by the Division, or as exempted in this rule, no licensee may:

(a) Deliver licensed material to a carrier for transport; or

(b) Transport licensed material.–

(2) An application by physicians as defined in 1200–2–4–.04(1)(nn) for an amendment to a specific license may be submitted to the Division to request specific conditions to their license to transport radioactive material in the course of their practice of medicine.

(3) A licensee who, under a general or specific license, transports licensed material outside its site of authorized use or on public highways, or who delivers licensed material to a carrier for transport, shall comply with the requirements of this rule and with the applicable requirements of the U.S. DOT regulations in 49 CFR Parts 170 through 189 appropriate to the mode of transport.

(a) The licensee shall particularly note U.S. DOT regulations in the following areas:

1. Packaging: 49 CFR Part 173, Subparts A and B and I;.

2. Marking and labeling: 49 CFR 172, Subpart D, 172.400 through 172.407, 172.436 through 172.440 and Subpart E;

3. Placarding: 49 CFR Part 172, Subpart F, especially 172.500 through 172.519, 172.556 and Appendices B and C;

4. Accident reporting: 49 CFR Part 171, 171.15 and 171.16;

5. Shipping papers and emergency information: 49 CFR Part 172, Subparts C and G;

6. Hazardous material employee training: 49 CFR Part 172, Subpart H; and

7. Hazardous material shipper/carrier registration: 49 CFR Part 107, Subpart G.

(b) The licensee shall also note U.S. DOT regulations pertaining to the following modes of transportation:

1. Rail: 49 CFR Part 174, Subparts A through D and K;

2. Air: 49 CFR Part 175;

3. Vessel: 49 CFR Part 176, Subparts A through F and M; and

4. Public highway: 49 CFR Part 177 and Parts 390 through 397.

(4) If U.S. DOT regulations are not applicable to a shipment of licensed material, the licensee shall conform to the standards and requirements of the U.S. DOT specified above in subparagraph (3)(a) to the same extent as if the shipment or transportation were subject to U.S. DOT regulations. A request for modification, waiver or exemption from those requirements, and any notification referred to in those requirements, shall be filed with, or made to, the Director of the Division of Radiological Health at the address given in Rule 1200–2–4–.07.

(5) Exemption for low–level materials.

(a) A licensee is exempt from all requirements of this rule with respect to shipment or carriage of a package containing radioactive material having a specific activity not greater than 0.002 μCi/g (70 Bq/g).

(b) A licensee is exempt from all requirements of this rule other than paragraphs 1200–2–10–.30(3), (4) and (10), with respect to shipment or carriage of the following packages, provided the packages contain no fissile material or the fissile material exemption standards of 10 CFR 71.53 are satisfied:

1. A package containing no more than a Type A quantity of radioactive material;

2. A package in which the only radioactive material is low specific activity (LSA) material or surface contaminated objects (SCO), provided the external radiation level at 3 meters from the unshielded material or objects does not exceed 10 mSv/h (1 rem/h); or

3. A package transported within locations within the United States that contains only americium or plutonium in special form with an aggregate radioactivity not to exceed 20 curies.

(c) A licensee is exempt from all requirements of this rule other than paragraphs 1200–2–10–.30(3) and (4) and (10), with respect to shipment or carriage of low–specific–activity (LSA) material in group LSA–I, or surface contaminated objects (SCO's) in group SCO–I.

(6) General license: U.S. NRC–approved package.

(a) A general license is hereby issued to any licensee of the Division to transport, or to deliver to a carrier for transport, licensed material in a package for which a license, certificate of compliance or other approval has been issued by the U.S. Nuclear Regulatory Commission.

(b) This general license applies only to a licensee who:

1. Has a copy of the certificate of compliance, or other approval of the package, and has the drawings and other documents referenced in the approval relating to the use and maintenance of the packaging and to the actions to be taken before shipment;

2. Complies with the terms and conditions of the license, certificate, or other approval, as applicable, and the applicable requirements of Subparts A, G and H of 10 CFR 71;

3. Submits in writing to the Director, Division of Radiological Health, at the address given in Rule 1200–2–4–.07, before the licensee's first use of the package, the licensee's name and license number and the package identification number specified in the package approval; and

4. Has submitted to the Division and received Division approval for a quality assurance program that satisfies the provisions found in Subpart H of 10 CFR 71.

(d) This general license applies only when the package approval authorizes use of the package under this general license.

(e) For a Type B or fissile material package, the design of which was approved by U.S. NRC before April 1, 1996, the general license is subject to the additional restrictions below in paragraph (7).

(7) Previously approved package.

(a) A Type B package previously approved by U.S. NRC but not designated as B(U) or B(M) in the identification number of the U.S. NRC Certificate of Compliance, may be used under the general license above in paragraph (5) with the following additional conditions:

1. Fabrication of the packaging was satisfactorily completed by August 31, 1986, as demonstrated by application of its model number in accordance with Sec. 71.85(c);

2. A package used for a shipment to a location outside the United States is subject to multilateral approval, as defined in U.S. DOT regulations at 49 CFR 173.403; and

3. A serial number that uniquely identifies each packaging that conforms to the approved design is assigned to, and legibly and durably marked on, the outside of each packaging.

(b) A Type B(U) package, a Type B(M) package, a low specific activity (LSA) material package or a fissile material package, previously approved by the U.S. NRC but without the designation '–85' in the identification number of the U.S. NRC Certificate of Compliance, may be used under the general license above in paragraph (5) with the following additional conditions:

1. Fabrication of the package was satisfactorily completed by April 1, 1999 as demonstrated by application of its model number in accordance with 10 CFR 71.85(c);

2. A package used for a shipment to a location outside the United States is subject to multilateral approval as defined in U.S. DOT regulations at 49 CFR 173.403; and

3. A serial number that uniquely identifies each packaging that conforms to the approved design is assigned to and legibly and durably marked on the outside of each packaging.

(8) General license: U.S. DOT specification container.

(a) A general license is issued to any licensee of the Division to transport, or to deliver to a carrier for transport, licensed material in a specification container for fissile material or for a Type B quantity of radioactive material as specified in U.S. DOT regulations at 49 CFR Parts 173 and 178.

(b) This general license applies only to a licensee who:

1. Has a copy of the specification;

2. Complies with the terms and conditions of the specification and the applicable requirements of this rule; and

3. Has submitted to the Division and received Division approval for a quality assurance program that satisfies the provisions found in Subpart H of 10 CFR 71.

(c) This general license is subject to the limitation that the specification container may not be used for a shipment to a location outside the United States, except by multilateral approval, as defined in U.S. DOT regulations at 49 CFR 173.403.

(9) General license: Use of foreign approved package.

(a) A general license is issued to any licensee of the Division to transport, or to deliver to a carrier for transport, licensed material in a package the design of which has been approved in a foreign national competent authority certificate that has been revalidated by U.S. DOT as meeting the applicable requirements of 49 CFR 171.12.

(b) This general license applies only to a licensee who:

1. Has a copy of the applicable certificate, the revalidation and the drawings and other documents referenced in the certificate, relating to the use and maintenance of the packaging and to the actions to be taken before shipment;

2. Complies with the terms and conditions of the certificate and revalidation and with the applicable requirements of this rule; and

3. Has submitted to the Division and received Division approval for a quality assurance program that satisfies the provisions found in Subpart H of 10 CFR 71.

(c) This general license applies only to shipments made to or from locations outside the United States.

(10) Preliminary determinations.

(a) Before the first use of any packaging for the shipment of licensed material:

1. The licensee shall ascertain that there are no cracks, pinholes, uncontrolled voids, or other defects that could significantly reduce the effectiveness of the packaging or impact compliance with the standards specified in 10 CFR 71.

2. Where the maximum normal operating pressure will exceed 35 kPa (5 lbf/in2) gauge, the licensee shall test the containment system at an internal pressure at least 50 percent (50%) higher than the maximum normal operating pressure, to verify the capability of that system to maintain its structural integrity at that pressure; and

3. The licensee shall conspicuously and durably mark the packaging with its model number, serial number, gross weight and a package identification number assigned by the U.S. Nuclear Regulatory Commission (U.S. NRC). Before applying the model number, the licensee shall determine that the packaging has been fabricated in accordance with the design approved by the U.S. NRC.

(b) Reserved.

(11) Routine determinations.

(a) Before each shipment of licensed material, the licensee shall ensure that the package with its contents satisfies the applicable requirements of this rule and of the license. The licensee shall determine that:

1. The package is proper for the contents to be shipped in accordance with 49 CFR 173.401–435;

2. The package is in unimpaired physical condition except for superficial defects such as marks or dents;

3. Each closure device of the packaging, including any required gasket, is properly installed, secured and free of defects;

4. Any system for containing liquid is adequately sealed and has adequate space or other specified provision for expansion of the liquid in accordance with 10 CFR 71, Subpart F;

5. Any pressure relief device is operable and set in accordance with written procedures;

6. The package has been loaded and closed in accordance with written procedures;

7. For fissile material, any moderator or neutron absorber, if required, is present and in proper condition;

8. Any structural part of the package that could be used to lift or tie down the package during transport is rendered inoperable for that purpose, unless it satisfies the design requirements of 10 CFR 71.45;

9. The level of non–fixed (removable) radioactive contamination on the external surfaces of each package offered for shipment is as low as reasonably achievable and within the limits specified in U.S. DOT regulations in 49 CFR 173.443;

10. External radiation levels around the package and around the vehicle, if applicable, will not exceed the limits specified in 10 CFR 71.47 at any time during transportation; and

11. Accessible package surface temperatures will not exceed the limits specified in 10 CFR 71.43(g) at any time during transportation.

(b) Reserved.

(12) Air transport of plutonium.

(a) Notwithstanding the provisions of any general licenses and notwithstanding any exemptions stated directly in this rule or included indirectly by citation of 49 CFR Chapter I, as may be applicable, the licensee shall assure that plutonium in any form, whether for import, export or domestic shipment, is not transported by air or delivered to a carrier for air transport unless:

1. The plutonium is contained in a medical device designed for individual human application; or

2. The plutonium is contained in a material in which the specific activity is not greater than 0.002 μCi/g (70 Bq/g) of material and in which the radioactivity is essentially uniformly distributed; or

3. The plutonium is shipped in a single package containing no more than an A2 quantity of plutonium in any isotope or form and is shipped in accordance with paragraphs 1200–2–10–.30(3) and (4); or

4. The plutonium is shipped in a package specifically authorized for the shipment of plutonium by air in the Certificate of Compliance for that package issued by the U.S. Nuclear Regulatory Commission.

(b) Nothing in subparagraph (a) of this paragraph is to be interpreted as removing or diminishing the requirements of 10 CFR 73.24.

(c) For a shipment of plutonium by air that is subject to part (a)4. above, the licensee shall, through special arrangement with the carrier, require compliance with 49 CFR 175.704, U.S. Department of Transportation regulations applicable to the air transport of plutonium.

(13) Opening instructions. Before delivery of a package to a carrier for transport, the licensee shall ensure that any special instructions needed to safely open the package have been sent to, or otherwise made available to, the consignee for the consignee's use in accordance with subparagraphs 1200-2-5-.115(5)(a) and (b).

(14) Records.

(a) Each licensee shall maintain, for a period of three (3) years after shipment, a record of each shipment of licensed material not exempt under paragraph 1200-2-10-.30(9), showing where applicable:

1. Identification of the packaging by model number and serial number;

2. Verification that there are no significant defects in the packaging, as shipped;

3. Volume and identification of coolant;

4. Type and quantity of licensed material in each package and the total quantity of each shipment;

5. For each item of irradiated fissile material:

(i) Identification by model number and serial number;

(ii) Irradiation and decay history to the extent appropriate to demonstrate that its nuclear and thermal characteristics comply with license conditions; and

(iii) Any abnormal or unusual condition relevant to radiation safety;

6. Date of the shipment;

7. For fissile packages and for Type B packages, any special controls exercised;

8. Name and address of the transferee;

9. Address to which the shipment was made; and

10. Results of the determinations required by paragraph 1200-2-10-.30(11) and by the conditions of the package approval.

(b) The licensee shall make available to the Division for inspection, upon reasonable notice, all records required by this rule. Records are only valid if stamped, initialed, or signed and dated by authorized personnel or otherwise authenticated.

(15) The licensee shall maintain sufficient written records to furnish evidence of the quality of packaging. The records to be maintained include results of the determinations required by paragraph 1200–2–10–.30(11); design, fabrication and assembly records; results of reviews, inspections, tests and audits; results of monitoring work performance and materials analyses; and results of maintenance, modification and repair activities. Inspection, test and audit records shall identify the inspector or data recorder, the type of observation, the results, the acceptability and the action taken in connection with any deficiencies noted. The records shall be retained for three (3) years after the life of the packaging to which they apply.

(16) Inspection and tests. In addition to the requirements in paragraph 1200–2–10–.27(1) and Rule 1200–2–10–.28, the licensee shall notify the Director, Division of Radiological Health, at the address given in Rule 1200–2–4–.07, at least 45 days before fabrication of a package to be used for the shipment of licensed material having a decay heat load in excess of 5 kW or with a maximum normal operating pressure in excess of 103 kPa (15 lbf/in^2) gauge.

(17) Reports. The licensee shall report to the Director, Division of Radiological Health, within 30 days:

(a) Any instance in which there is significant reduction in the effectiveness of any approved Type B, or fissile, packaging during use;

(b) Details of any defects with safety significance in Type B, or fissile, packaging after first use, with the means employed to repair the defects and prevent their recurrence; or

(c) Instances in which the conditions of approval in the certificate of compliance were not observed in making a shipment.

(18) Advance notification of shipment of irradiated reactor fuel and nuclear waste.

(a) As specified in subparagraphs (b), (c) and (d) below, each licensee shall provide advance notification to the governor of Tennessee, or the governor's designee, and to the Director, Division of Radiological Health, of the shipment of licensed material through or across the boundary of the State, before the transport, or delivery to a carrier for transport, of licensed material outside the confines of the licensee's plant or other place of use or storage.

(b) Advance notification is required under this section for shipments of irradiated reactor fuel in quantities less than that subject to advance notification requirements of 10 CFR 73.37(f). Advance notification is also required under this section for shipment of licensed material, other than irradiated fuel, meeting the following three conditions:

1. The licensed material is required by 10 CFR 71 to be in Type B packaging for transportation;

2. The licensed material is being transported to or across the State boundary en route to a disposal facility or to a collection point for transport to a disposal facility; and

3. The quantity of licensed material in a single package exceeds the least of the following:

(i) 3000 times the A_1 value of the radionuclides as specified in appendix A, Table A–1 for special form radioactive material;

(ii) 3000 times the A_2 value of the radionuclides as specified in appendix A, Table A–1 for normal form radioactive material; or

(iii) 1000 TBq (27,000 Ci).

(c) Procedures for submitting advance notification.

1. The notification shall be made in writing to the office of each appropriate governor or governor's designee and to the Director, Division of Radiological Health.

2. A notification delivered by mail shall be postmarked at least seven (7) days before the beginning of the seven (7) day period during which departure of the shipment is estimated to occur.

3. A notification delivered by messenger shall reach the office of the governor, or of the governor's designee, and of the Director, Division of Radiological Health, at least four (4) days before the beginning of the seven (7) day period during which departure of the shipment is estimated to occur.

(i) A list of the names and mailing addresses of the governors' designees receiving advance notification of transportation of nuclear waste was published in the Federal Register on June 30, 1995 (60 FR 34306).

(ii) The list will be published annually in the Federal Register on or about June 30 to reflect any changes in information.

(iii) A list of the names and mailing addresses of the governors' designees is available on request from the Director, Office of State Programs, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

(iv) The licensee shall retain a copy of the notification as a record for three (3) years.

(d) Information to be furnished in advance notification of shipment. Each advance notification of shipment of irradiated reactor fuel or nuclear waste shall contain the following information:

1. The name, address and telephone number of the shipper, carrier and receiver of the irradiated reactor fuel or nuclear waste shipment;

2. A description of the irradiated reactor fuel or nuclear waste contained in the shipment, as specified in the regulations of U.S. DOT in 49 CFR 172.202 and 172.203(d);

3. The point of origin of the shipment and the seven (7) day period during which departure of the shipment is estimated to occur;

4. The seven (7) day period during which arrival of the shipment at the State's boundaries is estimated to occur;

5. The destination of the shipment and the seven (7) day period during which arrival of the shipment is estimated to occur; and

6. A point of contact, with a telephone number, for current shipment information.

(e) Revision notice. A licensee who finds that schedule information previously furnished to the governor, or governor's designee, and to the Director, Division of Radiological Health, in accordance with this section, will not be met, shall telephone a responsible individual in the office of the governor of the State, or of the governor's designee, and of the Division of Radiological Health and inform those individuals of the extent of the delay beyond the schedule originally reported. The licensee shall maintain a record of the name of the individual contacted for three (3) years.

(f) Cancellation notice.

1. Each licensee who cancels an irradiated reactor fuel or nuclear waste shipment for which advance notification has been sent shall send a cancellation notice to the governor of each State, or to the governor's designee, previously notified, and to the Director, Division of Radiological Health.

2. The licensee shall state in the notice that it is a cancellation and identify the advance notification that is being canceled. The licensee shall retain a copy of the notice as a record for three (3) years.

Authority: *T.C.A. §68–202–206 and 68–202–101 et seq.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed January 8, 1990; effective May 1, 1990. Amendment filed July 18, 2002; effective October 1, 2002. Amendment filed _____; effective _____.*

1200–2–10–.31 FEES FOR LICENSES.

(1) A fee shall be assessed and collected on the application for and annual maintenance of licenses regarding radioactive materials, as follows:

(a) Application filing fees from applicants for licenses to use or possess radioactive materials or any other activity authorized under this chapter that requires a license from the Department.

(b) Annual maintenance fees from licensees or persons required to possess a license under this chapter, including reciprocal activity under 1200–2–10–.29.

(2) The application filing fees shall be the same amount as the annual maintenance fees set forth in (6) through (19) of this rule. A radioactive material license application will not be considered for completeness unless the application–filing fee has been paid in full. Within 15 days of receipt of an application, an invoice for the fee will be prepared and mailed to the applicant. The application–filing fee is not refundable, except as specified in Public Chapter 417, Acts of 1991. Applicants for licenses greater than Category 8 shall pay the application fee annually until the license is issued or denied. (An application–filing fee shall be required when a licensee applies for a license modification to change to a higher numbered category, in which case the application fee will be the amount of the proposed new Category. The application–filing fee shall serve as full payment of fees for the balance of the calendar year in which the license is issued.)

(3) If a license authorizes activities under more than one Category, the application and annual maintenance fee shall be the cumulative total for each applicable category under which the license is issued.

(4) The annual maintenance fees, based on the categories in (6) through (19) of this rule shall be payable to the Division of Radiological Health by check made payable to "Treasurer, State of Tennessee" by February 17 of each year, as indicated on the annual invoice, until the license is terminated in accordance with these regulations.

(a) Provided that the licensee has demonstrated to the satisfaction of the Division that all of the requirements concerning disposal of radioactive material and the decontamination of facilities are met, the termination of the license is administratively accomplished by using one of the following:

1. As requested by the licensee;

2. By the Department for cause; or

3. In accordance with these regulations.

(b) The failure to acquire radioactive material or the disposal of radioactive material without notifying the Department and requesting termination in writing does not constitute termination of the license.

(5) Complete Applications

(a) For the purpose of determining whether or not the Division has acted in the time frame established to process applications set forth in (5)(e), the evaluation period shall not begin until a complete application has been filed in the Division of Radiological Health Nashville office. All items on the application form shall be completed in sufficient detail to allow the Division to determine that the applicant's equipment, facilities and radiation protection program are adequate to protect health and minimize danger to life and property.

(b) The Division shall denote the date that all applications for radioactive material license are received in its Nashville office.

(c) Upon receipt of an application, the Division must examine it to ensure that it is complete and advise the applicant in writing of its findings via certified mail. Sixty (60) days will be allowed for the initial and each subsequent review per (c)(3) of this rule.

1. If an application is determined to be incomplete, the Division must notify the applicant in writing via certified mail of the finding with a brief explanation of the deficiencies. The application–filing fee shall be retained by the Division.

2. After receiving notice from the Division that the application was incomplete, the applicant shall have one hundred eighty (180) calendar days to correct the deficiencies. If properly corrected, the application will be processed and no additional application fee is required, except for the possibility of those above Category 8. If the deficiencies are not corrected within the 180–day correction period, the fee will be forfeited in its entirety to the Division with no further action taken on the application by the Division. If the applicant re–applies, a new application fee must be paid in full.

3. Upon receipt of a corrected application revised pursuant to part 1. or 2. of this subparagraph (c), the Division shall re–evaluate the application and notify the applicant of its finding as to whether or not the deficiencies in the application

have been completed. The same procedure to notify an applicant as to whether or not the application is complete will follow the requirements specified by this subparagraph, with the exception being that the 180–day correction period begins from the receipt of the initial application – not receipt of the revised application.

4. Any person possessing licensable quantities of unlicensed radioactive material during the review of an application for a license for the radioactive material, shall be in violation of 1200–2–10–.02.

(d) Revisions to an application, to reflect changes in radioactive material or its use, will be accepted by the Division during the application–processing period. However, notwithstanding (5)(e) of this rule, the deadline for evaluation as to issuance of a license will restart upon each and every revision.

(e) The Division shall make a decision to issue or deny a request for a new radioactive material license, except Category 12, and notify the applicant of that decision in no more than 365 days after receipt of a complete application, unless the Division has requested technical assistance in the review of the application from the Nuclear Regulatory Commission.

(6) CATEGORY GL ——— $ 150.00

Any person possessing radioactive material, under the terms of any general license issued under these regulations, in a form or device on which a test for leakage of radioactive material is required.

(7) CATEGORY 1 ——— $ 300.00

A specific license for source material used exclusively for shielding radiation.

(8) CATEGORY 2 ——— $ 600.00

(a) Reserved.

(b) The application, use or possession of radioactive material as chromatography sources or gauges not requiring assignment to another category.

(c) The application, use or possession of radioactive material for in vitro use only, total quantity not to exceed 200 microcuries.

(d) Any person who packages or containerizes, loads transport vehicles or ships radioactive materials to a licensed disposal/processing facility in Tennessee.

In addition to application and annual maintenance fees, there is also levied a fee of one and one–half cent per pound ($0.015/lb) on all items contaminated or potentially contaminated with radioactive material or on low–level radioactive waste received at a processing, storage, disposal or refurbishing facility in Tennessee.

Not withstanding the requirements of this paragraph 1200–2–10–.31(8) and Rule 1200–2–10–.32, licensees with multiple sites within the state will be levied only one fee if items are moved directly from one site to another.

The operator of the disposal/processing facility shall collect the fee of one and one–half cent per pound ($0.015/lb). For each calendar month, he shall remit the total of fees collected for the month to the Division of Radiological Health by the 25th day of the following month.

(e) The application, use or possession of radioactive material for the calibration for hire of radiation detection, monitoring and measuring instruments.

(f) The performance for hire of leak tests on sealed sources of radioactive material.

(9) CATEGORY 3 ——— $ 900.00

(a) The application, use or possession of radioactive material, unless specific to a higher numbered category, by an academic institution, but does not include licenses authorizing all radioisotopes with atomic number 3 through 83.

(b) The possession and use of radioactive material for civil defense activities.

(c) The application, use or possession of radioactive material by a medical institution or physicians for use in radiopharmaceuticals for the diagnosis or therapy of humans.

(d) Reserved.

(e) Reserved.

(f) Reserved.

(g) The application, use or possession of radioactive material for demonstration or training purposes.

(h) The application, use or possession of radioactive material for in vitro use only, total quantity exceeding 200 microcuries.

(i) The use of sealed sources for soil and/or construction materials testing at temporary job–sites by licensees with licensed authorization for no more than two (2) devices.

(j) The use of radioactive material as chromatography sources at temporary job–sites by licensees with licensed authorization for no more than two (2) devices.

(k) The use of gauging and measuring devices at temporary job–sites by licensees with licensed authorization for no more than two (2) devices.

(10) CATEGORY 4 ——— $ 1,500.00

(a) The application, use or possession of radioactive material by a medical institution or physicians for interstitial, intracavitary or superficial treatment of humans using sealed sources, seeds or wires.

(b) The application, use or possession of radioactive material in sealed sources for irradiation of materials in which the source is not removed from its shield (self–contained irradiators).

(c) The application, use or possession of radioactive material for analytical testing purposes.

(11) CATEGORY 5 ——— $ 2,100.00

(a) The use of radioactive material in research and development, manufacturing, testing, processing and assembling of products. This group includes the use of source material in the manufacture of items such as mantles, alloys, gases, liquids, metals, ceramics, glass or photographic products.

(b) The use of radioactive material in a process that incorporates that material into a product in exempt concentrations.

(c) The possession and use of radioactive material in curie quantities in a number of sources in gauges and gauging applications that require frequent changes and therefore frequent review of the program to ensure that the hazard potential does not exceed the scope of the radiation safety program.

(d) The use of a single radioactive material in the fabrication of sealed sources or ampoules.

(e) The receipt of prepackaged radioactive material waste from other persons by a nuclear waste handler for storage for less than three (3) months before transfer only to persons licensed to receive or dispose of the material.

(f) The use of sealed sources for soil and/or construction materials testing at temporary job–sites by licensees with licensed authorization for more than two (2) devices.

(g) The use of radioactive material as chromatography sources at temporary job–sites by licensees with licensed authorization for more than two (2) devices.

(h) The use of gauging and measuring devices at temporary job–sites by licensees with licensed authorization for more than two (2) devices.

(i) The application, use or possession of radioactive material by a medical institution or physicians for the treatment of humans with sealed sources contained in teletherapy devices.

(j) The application, use or possession of radioactive material by a veterinarian for the treatment of animals using sealed sources, seeds or wires.

(12) CATEGORY 6 ——— $ 6,000.00

(a) The application, use or possession of radioactive material including source and/or special nuclear material in unsealed form in less than multi–curie quantities for use in the fabrication of sealed sources without regard to amount of contained radioactivity.

(b) The manufacture of devices and/or sources that require in–depth review before approval by the Division. Each device and/or source reviewed shall be subject to this fee.

(c) The preparation, use or distribution of radiopharmaceuticals to locations other than the licensee's address for use in medical diagnosis or therapy.

(d) The use of radiography (the examination of the structure of materials by nondestructive methods using radioactive material) on the licensee's premises in a permanent shielded facility or temporary job–sites.

(e) The possession and use of radioactive material by academic and medical institutions under a license authorizing all radioisotopes with atomic numbers 3 through 83.

(f) Reserved.

(g) The application of radioactive material to soil, water, air, plants and animals, if the application involves an actual or potential release in or to unrestricted areas.

(h) The possession, use and distribution of radioactive material at one or more satellite facilities, or the possession and use of radioactive material at one or more satellite facilities, by medical institutions.

(i) The application, use or possession of radioactive material by a medical institution or physicians for research using humans and/or animals.

(13) CATEGORY 7 ——— $ 4,000.00

(a) Reserved.

(b) Reserved.

(c) The application, use or possession of radioactive material for well logging, well surveys or tracer studies.

(14) CATEGORY 8 ——— $ 11,250.00

(a) The receipt of radioactive material waste from other persons by a nuclear waste handler, for the purpose of packaging or repackaging the material prior to transfer only to persons licensed to receive or dispose of the material.

(b) The commercial collection, laundering or dry cleaning of wearing apparel that is contaminated with radioactive material.

(15) CATEGORY 9 ——— $ 15,000.00

(a) The possession of radioactive material or equipment contaminated or potentially contaminated with radioactive material as a result of operations involving the recovery of an element, compound or mixture from ores not subject to licensure because of the radioactive material content of the ore.

(b) Facilities that possess radioactive material as a result of operations (not directly involving radioactive decontamination activities) involving recovery of materials or other manufacturing processes (not directly manufacturing radioactive items or products).

(16) CATEGORY 10 ——— $ 22,500.00

(a) Facilities storing radioactive material, contaminated equipment and/or potentially contaminated equipment for transfer to authorized recipients as a service to the nuclear industry.

(b) Possession and refurbishment of contaminated equipment and/or potentially contaminated equipment that has been used at nuclear power plants.

(17) CATEGORY 11 ——— $ 30,000.00

(a) The collection, transfer, sorting and/or brokerage of radioactive material as sealed source, residue, product or as material in or on equipment; and/or

The decontamination of products and/or equipment containing radioactive material and/or contaminated with radioactive material; and/or

The possession, storage and incineration of radioactive material or items contaminated with radioactive materials.

(b) On site possession and storage of radioactive material and/or equipment contaminated with radioactive material as a result of operations involving the recovery of an element, compound or mixture from ores subject to licensure because of the radioactive material content of the ore or concentration of the radioactive material during the processing of the ore.

(c) Facilities involved in the manufacture of product lines containing radioactive material in the manufactured product.

(d) Possession of radioactive material for processing. This material may exist in ores, concentrates, compounds or metals.

(e) The possession of multi–curie quantities of unsealed radioactive material either as waste or for further processing and/or conversion into specific marketable products.

(f) Operations involving the fabrication of sealed sources or manufacture of compounds for distribution to other specific or general licensees.

(g) The possession and use of radioactive material in a sealed source for irradiation of materials in which the source is exposed for irradiation purposes (non self–contained irradiators).

(18) CATEGORY 12 ——— $ 375,000.00

(a) The application for and/or operation of a low–level radioactive waste disposal facility.

(b) The maximum length of reviewing time (the period of time when there are no outstanding unanswered questions) after receipt of a new application and the appropriate fee for a Category 12 specific license and the issuance of a license is 60 months.

(19) CATEGORY 13 ——— At least $ 150.00 not greater than $ 375,000.00

The application, use or possession of radioactive material for uses or procedures not specifically included in any other category.

The fee shall be determined on a case–by–case basis.

The determination shall be based on an analysis of the hazard, the scope of the difficulty encountered in the review process and the specifics of the activity pursuant to the categories established above

Authority: *T.C.A. §4–5–201 et seq., 68–202–201 et seq. and 68–203–101 et seq.* ***Administrative History:*** *Original rule filed September 3, 1991; effective October 18, 1991. Amendment filed March 31, 1992; effective May 15, 1992. Amendment filed October 1, 2001; effective December 15, 2001.*

1200–2–10–.32 LICENSING OF SHIPPERS OF RADIOACTIVE MATERIAL INTO OR WITHIN TENNESSEE.

(1) This rule applies to any shipper who transports or offers for transport into or within Tennessee on public waterways, roadways, railways or other transportation facilities upon which United States Department of Transportation (USDOT) regulations are applicable, any radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities for packaging, repackaging, processing, refurbishing, storage pending disposal or disposal.

(2) All persons subject to the provisions of this rule shall comply with all applicable provisions of the USDOT Regulations (49 CFR) of October 1, 1990, as amended, the U.S. Nuclear Regulatory Commission (NRC) Regulations (10 CFR) of November 30, 1988, as amended, and any disposal/processing facility radioactive material license requirements with special emphasis regarding the packaging, transportation, disposal, storage pending disposal or delivery of radioactive material.

(3) Definitions used in this rule.

(a) 'Carrier' means any person who transports radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities.

(b) 'Disposal' means isolation of radioactive waste from the biosphere.

(c) 'Disposal/Processing Facility' means any facility located within Tennessee that accepts radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities for packaging, repackaging, processing, refurbishing, storage pending disposal or disposal.

(d) (Reserved)

(e) (Reserved)

(f) 'License for delivery' means an authorization issued by the Division to any shipper of radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities to transport such radioactive material or offer such material for transport to a disposal/processing facility.

(g) 'Shipper' means any person, whether a resident of Tennessee or a non–resident:

1. Who transfers radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities to a carrier for transport;

2. Who transports his own radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities;

3. Who transports radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities he has packaged, repackaged, processed or stored pending disposal for another person;

4. Who transfers radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities to another person if such materials are transported into or within the state.

(h) 'Transport' means the movement of radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities into or within the State of Tennessee on waterways, roadways, railways or other transportation facilities upon which USDOT regulations are applicable.

(4) Licensing for Delivery.

(a) Before any shipper transports or causes to be transported radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities to a disposal/processing facility within the State for subsequent processing, he shall obtain a license for delivery of such materials from the Division. An application for a license for delivery shall be submitted on Division Form RHS–30 together with any necessary fee to the Division at the address in Rule 1200–2–4–.07. The check for payment of the fee is to be made payable to "Treasurer: State of Tennessee."

(b) Before a license for delivery shall be issued, the shipper must deposit and maintain with the Division an acceptable form of financial assurance in the amount of Five Hundred Thousand Dollars ($500,000.00); or, provide to the Division satisfactory evidence of liability insurance.

1. For purposes of this paragraph, liability insurance shall mean coverage of Five Hundred Thousand Dollars ($500,000.00) per occurrence and one Million Dollars ($1,000,000.00) aggregate, or as otherwise provided by State law.

2. Any insurance carried pursuant to Section 2210 of Title 42 of the United States Code and U.S. NRC Regulations (10 CFR Part 140) of November 30, 1988, as amended shall be sufficient to meet the requirements of 1200-2-10-.32(4)(b).

3. Liability insurance shall be specific to the packaging, transportation, disposal, storage and delivery of radioactive waste.

4. Shippers maintaining liability insurance for the purpose of this paragraph may provide to the Division a certificate of insurance from their insurer indicating the policy number, limits of liability, policy date and specific coverage for packaging, transportation, disposal, storage pending disposal and delivery of radioactive materials.

5. A cash or corporate surety bond previously posted will be returned to the shipper upon notification to the Division in writing of his intention to cease shipments of radioactive waste into or within the State. Such bond will be returned after the last such shipment is accepted safely at its destination.

(c) Each license for delivery application shall include a certification to the Division that the shipper will comply fully with all applicable State and Federal laws, administrative rules and regulations, licenses, or license conditions of the disposal/processing facility regarding the packaging, transportation, storage pending disposal, disposal and delivery of radioactive materials.

(d) Each license for delivery application shall include a certification that the shipper will hold the State of Tennessee harmless for all claims, actions or proceedings in law or equity arising out of radiological injury or damage to persons or property occurring during the transportation of its radioactive waste into or within the State including all costs of defending the same; provided, however, that nothing contained herein shall be construed as a waiver of the State's sovereign immunity; and, further provided that agencies of the State of Tennessee shall not be subject to the requirement of (4)(b) of this rule.

(5) Disposal/processing facility operator.

(a) Owners and operators of disposal/processing facilities shall permanently record, and report to the Division within twenty-four (24) hours after discovery, all conditions in violation of the requirement of this rule discovered as a result of inspections required by any license under which the facility is operated. In addition, owners and operators of disposal/processing facilities shall record all violations of these regulations and license conditions and maintain the record for inspection by the Division.

(b) Prior to the receipt of radioactive waste and/or items contaminated or potentially contaminated with licensable quantities of radioactive material or from licensable activities at a disposal/processing facility in Tennessee, the owners and operators of such facility shall notify each shipper of any special requirements, if any, in effect regarding the packaging, transportation, storage pending disposal, disposal or delivery of such wastes at that facility.

(c) No owner or operator of a disposal/processing facility located within this State shall accept radioactive waste and/or items contaminated or potentially contaminated with

licensable quantities of radioactive material or from licensable activities for packaging, repackaging, processing, refurbishing, storage pending disposal or disposal unless the shipper of such waste has a valid license for delivery issued pursuant to this rule.

(d) The owner or operator of a disposal/processing facility shall, along with the remittance of the fee collected pursuant to 1200–2–10–.31(8)(d), submit a listing containing the name and address of each shipper and the volume and poundage from each shipper for the calendar month.

(6) Penalties. All shippers shall be subject to fees and Civil Penalties as authorized and specified in Tennessee Code Annotated 68–202–212 and other pertinent regulations of the Division.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–201 et seq., and Acts of 1991, Public Chapter 417.* ***Administrative History:*** *Original rule filed September 3, 1991; effective October 18, 1991. Amendment filed November 17, 2005; effective January 31, 2006.*

1200–2–10–.33 ACCEPTABLE TRAINING AND EXPERIENCE FOR MEDICAL USES OF RADIOACTIVE MATERIAL.

(1) General Training. To qualify as adequately trained to use or directly supervise the use of radioactive material listed in Groups I, II and/or III, Rule 1200–2–10–.14, a physician should have:

(a) Training in basic radioisotope handling techniques consisting of lectures, laboratory sessions, discussion groups or supervised experience in a nuclear medicine laboratory in the following areas: (200 hours)

Deleted: areas: . (200

1. Radiation physics and instrumentation (approx. 100 hours)
2. Radiation Protection (approx. 30 hours)
3. Mathematics pertaining to the use and measurement of radioactivity (approx. 20 hours)
4. Radiation biology (approx. 20 hours)
5. Radiopharmaceutical chemistry (approx. 30 hours)

(b) Experience with the types and quantities of radioactive material for which the application is being made, or equivalent (500 hours). For authorization for Group III (generators and reagent kits), this experience should include personal participation in five procedures to elute Tc–99m, including testing of eluate, and five procedures to prepare radiopharmaceuticals from Group III reagent kits.

(c) Supervised clinical training in an institutional nuclear medicine program (500 hours). The clinical training should cover all appropriate types of diagnostic procedures and include:

1. Supervised examination of patients to determine the suitability for radioisotope diagnosis and recommendation on dosage to be prescribed.
2. Collaboration in calibration of the dose and the actual administration of the dose to the patient, including calculation of the radiation dose, related measurement and plotting data.

3. Follow–up of patients when required.

4. Study and discussion with preceptor of case histories to establish most appropriate diagnostic procedures, limitation, contraindication, etc.

(d) The requirements specified in 1200–2–10–.33(1)(a), (b) and (c) may be satisfied concurrently in a three month training program IF all three areas are integrated into the program.

(e) In lieu of the requirements in 1200–2–10–.33(1)(a), (b), and (c), certification by the American Board of Nuclear Medicine or the American Board of Radiology in Diagnostic Radiology with Special Competence in Nuclear Radiology will be accepted as evidence that a physician has had adequate training and experience to use Groups I, II, and III.

(2) Training Requirements for Specific Diagnosis Procedures. For applicant who wishes to be authorized for only one or two specific diagnostic procedures the physician named to use or directly supervise the use of radioactive material should have training in basic radioisotope handling techniques and clinical procedures commensurate with the procedures and quantities of radioactive material being requested.

(3) Training Requirements for Therapy Procedures involving Radiopharmaceuticals. To qualify as adequately trained to use or directly supervise the use of radioactive material listed in Groups IV and/or, V, Rule 1200–2–10–.14, a physician should have:

(a) Training in basic radioisotope handling techniques applicable to the uses of unsealed sources for therapy procedures, including: (80 hours)

1. Radiation physics and instrumentation (approx. 25 hours)

2. Radiation Protection (approx. 25 hours)

3. Mathematics pertaining to the use and measurement of radioactivity (approx. 10 hours)

4. Radiation biology (approx. 20 hours)

(These requirements are in lieu of, not in addition to, those specified in subparagraph 1200–2–10–.33(1)(a), above.)

(b) Clinical training in specific therapy procedures;

1. For Group IV

(i) Iodine–131 for treatment of hyperthyroidism and/or cardiac conditions: Clinical experience in the diagnosis of thyroid function and active participation in the treatment of ten patients.

(ii) Phosphorus–32 for treatment of polycythemia vera, leukemia and/or bone metastases: Active participation in the treatment of three patients with any combination of these three conditions.

(iii) Colloidal phosphorus–32 intracavitary treatment: Active participation in the treatment of three patients.

2. For Group V

(i) Iodine 131 for treatment of thyroid carcinoma: Clinical experience in diagnosis of thyroid function, personal participation in the treatment of ten patients with hyperthyroidism and/or cardiac dysfunction, and active participation in the treatment of three patients with thyroid carcinoma.

(ii) Colloidal gold 198 for intracavitary treatment: Active participation in the treatment of three patients.

(4) Training Requirements for Therapy Procedures Involving Sealed Sources. To qualify as adequately trained to use or directly supervise the use of radioactive material listed in Group VI, Rule 1200–2–10–.14 or other sealed sources in therapy procedures, a physician should have:

(a) Training in basic radioisotope handling techniques consisting of lectures, laboratory sessions, discussion groups or supervised experience in the following areas: (200 hours)

1.	Radiation physics and instrumentation	(approx. 110 hours)
2.	Radiation protection	(approx. 40 hours)
3.	Mathematics pertaining to the use and measurement of radioactivity	(approx. 25 hours)
4.	Radiation biology	(approx. 25 hours)

(b) Experience with the types and quantities of radioactive material for which the application is being made, or equivalent (500 hours). This experience should include:

1. Review of initial source calibration and periodic spot–check measurements of teletherapy units,

2. Calibration of ion chambers and survey meters,

3. Preparation of treatment plans and treatment times,

4. Knowledge of appropriate radiation safety, quality control, and emergency procedures for handling and using sealed sources, and

5. Initial source calibration of sealed sources other than teletherapy sources that are used for treatment purposes.

(c) Clinical training shall include active practice in therapeutic radiology with a minimum of three (3) years experience of which at least one (1) year should have been spent in a formal training program accredited by the Residency Review Committee of Radiology and the Liaison Committee on Graduate Medical Education. This training must include therapeutic treatment of patients of both sexes, all ages, various organs, etc., using sealed sources.

(d) In lieu of the requirements in 1200–2–10–.33(4)(a), (b) and (c), certification by the American Board of Radiology in Radiology or Therapeutic Radiology will be accepted as evidence that a physician has had adequate training and experience to use Group VI.

(5) Training for Physicians Wishing to Use Strontium 90 Ophthalmic Eye Applicators Only. To qualify as adequately trained to use or supervise the use of a Strontium 90 eye applicator only, a physician should submit:

(a) Evidence of certification by the American Board of Radiology in radiology or therapeutic radiology, or

(b) Evidence of:

1. Active practice in therapeutic radiology or ophthalmology, and –

2. Training in basic radioisotope handling techniques, including (24 hours)

Deleted: radioisotope

(i) Radiation physics and instrumentation (6 hours)

(ii) Radiation protection (6 hours)

(iii) Mathematics pertaining to the use and measurement of radioactivity (4 hours)

Deleted:

Deleted:

(iv) Radiation biology (8 hours)

3. Evidence of active participation in the treatment of five patients (to be submitted on Preceptor Statement). "Active participation" should include supervised examination of patients, collaboration and calculations concerning the dose to be used, administration of the dose to the patient, and follow–up and study of patient case histories.

(6) For each physician named in Item 4 of Form RHS 8–5 complete Supplement A of Form RHS 8–5A and Items 8 and 9 of Form RHS 8–5 (Preceptor statement and the statement of training and experience in basic radioisotope handling techniques). For each subject covered in basic training, state where the training was obtained, the dates, total number of hours and type of training (e.g., lectures, laboratory sessions).

SCHEDULE RHS 8–3

EXEMPT QUANTITIES

Radioactive Material	Micro–curies
Antimony–122 (Sb 122)	100
Antimony–124 (Sb 124)	10
Antimony–125 (Sb 125)	10
Arsenic–73 (As 73)	100
Arsenic–74 (As 74)	10
Arsenic–76 (As 76)	10
Arsenic–77 (As 77)	100
Barium–131 (Ba 131)	10
Barium–133 (Ba 133)	10
Barium–140 (Ba 140)	10
Bismuth–210 (Bi 210)	1
Bromine–82 (Br 82)	10
Cadmium–109 (Cd 109)	10
Cadmium–115m (Cd 115m)	10
Cadmium–115 (Cd 115)	100
Calcium–45 (Ca 45)	10
Calcium–47 (Ca 47)	10
Carbon–14 (C 14)	100
Cerium–141 (Ce 141)	100
Cerium–143 (Ce 143)	100
Cerium–144 (Ce 144)	1
Cesium–129 (Cs 129)	100
Cesium–131 (Cs 131)	1,000
Cesium–134m (Cs 134m)	100
Cesium–134 (Cs 134)	1
Cesium–135 (Cs 135)	10
Cesium–136 (Cs 136)	10
Cesium–137 (Cs 137)	10
Chlorine–36 (Cl 36)	10
Chlorine–38 (Cl 38)	10
Chromium–51 (Cr 51)	1,000
Cobalt–57 (Co 57)	100
Cobalt–58m (Co 58m)	10
Cobalt–58 (Co 58)	10
Cobalt–60 (Co 60)	1
Copper–64 (Cu 64)	100
Dysprosium–165 (Dy 165)	10
Dysprosium–166 (Dy 166)	100
Erbium–169 (Er 169)	100
Erbium–171 (Er 171)	100
Europium–152 (Eu 152)9.2 h	100
Europium–152 (Eu 152)13 yr	1
Europium–154 (Eu 154)	1
Europium–155 (Eu 155)	10
Fluorine–18 (F 18)	1,000
Gadolinium–153 (Gd 153)	10
Gadolinium–159 (Gd 159)	100
Gallium–67 (Ga 67)	100
Gallium–72 (Ga 72)	10
Germanium–68 (Ge 68)	10
Germanium–71 (Ge 71)	100
Gold–195 (Au 195)	10
Gold–198 (Au 198)	100
Gold–199 (Au 199)	100
Hafnium–181 (Hf 181)	10
Holmium–166 (Ho 166)	100
Hydrogen–3 (H 3)	1,000
Indium–111 (In 111)	100
Indium–113m (In 113m)	100
Indium–114m (In 114m)	10
Indium–115m (In 115m)	100
Indium–115 (In 115)	10
Iodine–123 (I 123)	100
Iodine–125 (I 125)	1
Iodine–126 (I 126)	1
Iodine–129 (I 129)	0.1
Iodine–131 (I 131)	1
Iodine–132 (I 132)	10
Iodine–133 (I 133)	1
Iodine–134 (I 134)	10
Iodine–135 (I 135)	10
Iridium–192 (Ir 192)	10
Iridium–194 (Ir 194)	100
Iron–52 (Fe 52)	10
Iron–55 (Fe 55)	100
Iron–59 (Fe 59)	10
Krypton–85 (Kr 85)	100
Krypton–87 (Kr 87)	10
Lanthanum–140 (La 140)	10
Lutetium–177 (Lu 177)	100
Manganese–52 (Mn 52)	10
Manganese–54 (Mn 54)	10
Manganese–56 (Mn 56)	10
Mercury–197m (Hg 197m)	100
Mercury–197 (Hg 197)	100
Mercury–203 (Hg 203)	10
Molybdenum–99 (Mo 99)	100
Neodymium–147(Nd 147)	100
Neodymium–149 (Nd 149)	100
Nickel–59 (Ni 59)	100
Nickel–63 (Ni 63)	10
Nickel–65 (Ni 65)	100
Niobium–93m (Nb 93m)	10
Niobium–95 (Nb 95)	10
Niobium–97 (Nb 97)	10
Osmium–185 (Os 185)	10
Osmium–191m (Os 191m)	100
Osmium–191 (Os 191)	100

Radioactive Material	Micro-curies
Osmium-193 (Os 193)	100
Palladium-103 (Pd 103)	100
Palladium-109 (Pd 109)	100
Phosphorus-32 (P 32)	10
Platinum-191 (Pt 191)	100
Platinum-193m (Pt 193m)	100
Platinum-193 (Pt 193)	100
Platinum-197m (Pt 197m)	100
Platinum-197 (Pt 197)	100
Polonium-210 (Po 210)	0.1
Potassium-42 (K 42)	10
Potassium-43 (K 43)	10
Praseodymium-142 (Pr 142)	100
Praseodymium-143 (Pr 143)	100
Praseodymium-147 (Pr 147)	100
Promethium-147 (Pm 147)	10
Promethium-149 (Pm 149)	10
Rhenium-186 (Re 186)	100
Rhenium-188 (Re 188)	100
Rhodium-103m (Rh 103m)	100
Rhodium-105 (Rh 105)	100
Rubidium-81 (Rb 81)	10
Rubidium-86 (Rb 86)	10
Rubidium-87 (Rb 87)	10
Ruthenium-97 (Ru 97)	100
Ruthenium-103 (Ru 103)	10
Ruthenium-105 (Ru 105)	10
Ruthenium-106 (Ru 106)	1
Samarium-151 (Sm 151)	10
Samarium-153 (Sm 153)	100
Scandium-46 (Sc 46)	10
Scandium-47 (Sc 47)	100
Scandium-48 (Sc 48)	10
Selenium-75 (Se 75)	10
Silicon-31 (Si 31)	100
Silver-105 (Ag 105)	10
Silver-110m (Ag 110m)	1
Silver-111 (Ag 111)	100
Sodium-22 (Na 22)	10
Sodium-24 (Na 24)	10
Strontium-85 (Sr 85)	10
Strontium-89 (Sr 89)	1
Strontium-90 (Sr 90)	0.1
Strontium-91 (Sr 91)	10
Strontium-92 (Sr 92)	10
Sulfur-35 (S 35)	100
Tantalum-182 (Ta 182)	10
Technetium-96 (Tc 96)	10
Technetium-97m (Tc 97m)	100
Technetium-97 (Tc 97)	100
Technetium-99m (Tc 99m)	100
Technetium-99 (Tc 99)	10
Tellurium-125m (Te 125m)	10
Tellurium-127m (Te 127m)	10
Tellurium-127 (Te 127)	100
Tellurium-129m (Te 129m)	10
Tellurium-129 (Te 129)	100
Tellurium-131m (Te 131m)	10
Tellurium-132 (Te 132)	10
Terbium-160 (Tb 160)	10
Thallium-200 (Tl 200)	100
Thallium-201 (Tl 201)	100
Thallium-202 (Tl 202)	100
Thallium-204 (Tl 204)	10
Thulium-170 (Tm 170)	10
Thulium-171 (Tm 171)	10
Tin-113 (Sn 113)	10
Tin-125 (Sn 125)	10
Tungsten-181 (W 181)	10
Tungsten-185 (W 185)	10
Tungsten-187 (W 187)	100
Vanadium-48 (V 48)	10
Xenon-131m (Xe 131m)	1,000
Xenon-133 (Xe 133)	100
Xenon-135 (Xe 135)	100
Ytterbium-175 (Yb 175)	100
Yttrium-87 (Y 87)	10
Yttrium-88 (Y 88)	10
Yttrium-90 (Y 90)	10
Yttrium-91 (Y 91)	10
Yttrium-92 (Y 92)	100
Yttrium-93 (Y 93)	100
Zinc-65 (Zn 65)	10
Zinc-69m (Zn 69m)	100
Zinc-69 (Zn 69)	1,000
Zirconium-93 (Zr 93)	10
Zirconium-95 (Zr 95)	10
Zirconium-97 (Zr 97)	10
Any radioactive material not listed above other than alpha-emitting radioactive material	0.1
Any alpha emitting radioactive material not listed above other than transuranic radioactive material	0.01

SCHEDULE RHS 8–4

EXEMPT CONCENTRATIONS

Element (atomic number)	Isotope	Column I Gas Concentration μCi/ml [a]	Column II and Solid Concentration μCi/ml [b]
Antimony (51)	Sb–122		3 (E–4)
	Sb–124		2 (E–4)
	Sb–125		1 (E–3)
Argon (18)	Ar–37	1 (E–3)	
	Ar–41	4 (E–7)	
Arsenic (33)	As–73		5 (E–3)
	As–74		5 (E–4)
	As–76		2 (E–4)
	As–77		8 (E–4)
Barium (56)	Ba–131		2 (E–3)
	Ba–140		3 (E–4)
Beryllium (4)	Be–7		2 (E–2)
Bismuth (83)	Bi–206		4 (E–4)
Bromine (35)	Br –82	4 (E–7)	3 (E–3)
Cadmium (48)	Cd–109		2 (E–3)
	Cd–115m		3 (E–4)
	Cd–115		3 (E–4)
Calcium (20)	Ca–45		9 (E–5)
	Ca–47		5 (E–4)
Carbon (6)	C–14	1 (E–6)	8 (E–3)
Cerium (58)	Ce–141		9 (E–4)
	Ce–143		4 (E–4)
	Ce–144		1 (E–4)
Cesium (55)	Cs–131		2 (E–2)
	Cs–134m		6 (E–2)
	Cs–134		9 (E–5)
Chlorine (17)	Cl–38	9 (E–7)	4 (E–3)
Chromium (24)	Cr–51		2 (E–2)
Cobalt (27)	Co–57		5 (E–3)
	Co–58		1 (E–3)
	Co–60		5 (E–4)
Copper (29)	Cu–64		3 (E–3)
Dysprosium (66)	Dy–165		4 (E–3)
	Dy–166		4 (E–4)
Erbium (68)	Er–169		9 (E–4)
	Er–171		1 (E–3)
Europium (63)	Eu–152 (Tr = 9.2h)		6 (E–4)
	Eu–155		2 (E–3)
Fluorine(9)	F–18	2 (E–6)	8 (E–3)
Gadolinium (64)	Gd–153		2 (E–3)
	Gd–159		8 (E–4)
Gallium (31)	Ga–72		4 (E–4)
Germanium (32)	Ge–71		2 (E–2)
Gold (79)	Au–196		2 (E–3)
	Au–198		5 (E–4)
	Au–199		2 (E–3)

Element (atomic number)	Isotope	Column I Gas Concentration μCi/ml [a]	Column II and Solid Concentration μCi/ml [b]
Hafnium (72)	Hf–181		7 (E–4)
Hydrogen (1)	H–3	5 (E–6)	3 (E–2)
Indium (49)	In–113m		1 (E–2)
	In–114m		2 (E–4)
Iodine (53)	I–126	3 (E–9)	2 (E–5)
	I–131	3 (E–9)	2 (E–5)
	I–132	8 (E–8)	6 (E–4)
	I–133	1 (E–8)	7 (E–5)
	I–134	2 (E–7)	1 (E–3)
Iridium (77)	Ir–190		2 (E–3)
	Ir–192		4 (E–4)
	Ir–194		3 (E–4)
Iron (26)	Fe–55		8 (E–3)
	Fe–59		6 (E–4)
Krypton (36)	Kr–85m	1 (E–6)	
	Kr–85	3 (E–6)	
Lanthanum (57)	La–140		2 (E–4)
Lead (82)	Pb–203		4 (E–3)
Lutetium (71)	Lu–177		1 (E–3)
Manganese (25)	Mn–52		3 (E–4)
	Mn–54		1 (E–3)
	Mn–56		1 (E–3)
Mercury (80)	Hg–197m		2 (E–3)
	Hg–197		3 (E–3)
	Hg–203		2 (E–4)
Molybdenum (42)	Mo–99		2 (E–3)
Neodymium (60)	Nd–147		6 (E–4)
	Nd–149		3 (E–3)
Nickel (28)	Ni–65		1 (E–3)
Niobium (41)	Nb–95		1 (E–3)
	Nb–97		9 (E–3)
Osmium (76)	Os–185		7 (E–4)
	Os–191m		3 (E–2)
	Os–191		2 (E–3)
	Os–193		6 (E–4)
Palladium (46)	Pd–103		3 (E–3)
	Pd–109		9 (E–4)
Phosphorus (15)	P–32		2 (E–4)
Platinum (78)	Pt–191		1 (E–3)
	Pt–193m		1 (E–2)
	Pt–197m		1 (E–2)
	Pt–197		1 (E–3)
Polonium (84)	Po–210		7 (E–6)
Potassium (19)	K–42		3 (E–3)
Praseodymium (59)	Pr–142		3 (E–4)
	Pr–143		5 (E–4)
Promethium (61)	Pm–147		2 (E–3)
	Pm–149		4 (E–4)
Radium (88)	Ra–226		1 (E–7)
	Ra–228		3 (E–7)
Rhenium (75)	Re–183		6 (E–3)

Element (atomic number)	Isotope	Column I Gas Concentration μCi/ml [a]	Column II and Solid Concentration μCi/ml [b]
	Re–186		9 (E–4)
	Re–188		6 (E–4)
Rhodium (45)	Rh–103m		1 (E–1)
	Rh–105		1 (E–3)
Rubidium (37)	Rb–86		7 (E–4)
Ruthenium (44)	Ru–97		4 (E–3)
	Ru–103		8 (E–4)
	Ru–105		1 (E–3)
	Ru–106		1 (E–4)
Samarium (62)	Sm–153		8 (E–4)
Scandium (21)	Sc–46		4 (E–4)
	Sc–47		9 (E–4)
	Sc–48		3 (E–4)
Selenium (34)	Se–75		3 (E–3)
Silicon (14)	Si–31		9 (E–3)
Silver (47)	Ag–105		1 (E–3)
	Ag–110m		3 (E–4)
	Ag–111		4 (E–4)
Sodium (11)	Na–24		2 (E–3)
Strontium (38)	Sr–85		1 (E–3)
	Sr–89		1 (E–4)
	Sr–91		7 (E–4)
	Sr–92		7 (E–4)
Sulfur (16)	S–35	9 (E–8)	6 (E–4)
Tantalum (73)	Ta–182		4 (E–4)
Technetium (43)	Tc–96m		1 (E–1)
	Tc–96		1 (E–3)
Tellurium (52)	Te–125m		2 (E–3)
	Te–127m		6 (E–4)
	Te–127		3 (E–3)
	Te–129m		3 (E–4)
	Te–131m		6 (E–4)
	Te–132		3 (E–4)
Terbium (65)	Tb–160		4 (E–4)
Thallium (81)	Tl–200		4 (E–3)
	Tl–201		3 (E–3)
	Tl–202		1 (E–3)
	Tl–204		1 (E–3)
Thulium (69)	Tm–170		5 (E–4)
	Tm–171		5 (E–3)
Tin (50)	Sn–113		9 (E–4)
	Sn–125		2 (E–4)
Tungsten (74)	W–181		4 (E–3)
	W–187		7 (E–4)
Vanadium (23)	V–48		3 (E–4)
Xenon (54)	Xe–131m	4 (E–6)	
	Xe–133	3 (E–6)	
	Xe–135	1 (E–6)	
Ytterbium (70)	Yb–175		1 (E–3)
Yttrium (39)	Y–90		2 (E–4)
	Y–91m		3 (E–2)

Element (atomic number)	Isotope	Column I Gas Concentration μCi/ml [a]	Column II and Solid Concentration μCi/ml [b]
	Y–91		3 (E–4)
	Y–92		6 (E–4)
	Y–93		3 (E–4)
Zinc (30)	Zn–65		1 (E–3)
	Zn–69m		7 (E–4)
	Zn–69		2 (E–2)
Zirconium (40)	Zr–95		6 (E–4)
	Zr–97		2 (E–4)
Beta and/or gamma emitting radioactive material not listed above with half–life less than 3 years.		1 (E–10)	1 (E–6)

[a] Values are given in Column I only for those materials normally used as gases.
[b] μCi/gm for solids.

NOTE 1: Many radioisotopes disintegrate into isotopes that are also radioactive. In expressing the concentrations in Schedule RHS 8–4 the activity stated is that of the parent isotope and takes into account the daughters.

NOTE 2: For purposes of 1200–2–10–.04 where there is involved a combination of isotopes, the limit for the combination should be derived as follows: Determine for each isotope in the product the ratio between the concentration present in the product and the exempt concentration established in Schedule RHS 8–4 for the specific isotope when not in combination. The sum of such ratios may not exceed "1" (i.e., unity).

EXAMPLE:

$$\frac{\text{Concentration of Isotope A in Product}}{\text{Exempt concentration of Isotope A}} + \frac{\text{Concentration of Isotope B in Product}}{\text{Exempt concentration of Isotope B}} \leq 1$$

SCHEDULE RHS 8–5

GENERAL LICENSING OF CERTAIN NAMED DEVICES

The following devices and equipment incorporating radioactive material, when manufactured, tested, and labeled by the manufacturer in accordance with the specification contained in a specific license or equivalent licensing document issued by the Division, the U.S. Nuclear Regulatory Commission or any Agreement State are placed under a general license pursuant to 1200–2–10–.10(1):

(1) Static elimination device. Devices designed for use as static eliminators that contain, as a sealed source or sources, radioactive material consisting of a total of not more than 500 microcuries of polonium 210 per device.

(2) Ion generating tube. Devices designed for ionization of air that contain, as a sealed source or sources, radioactive material consisting of a total of not more than 50 millicuries of hydrogen 3 (tritium) per device.

Authority: *T.C.A. §68–202–101 et seq. and 68–202–206.* ***Administrative History:*** *Original rule certified June 7, 1974. Amendment filed August 15, 1978; effective October 2, 1978. Amendment filed April 3, 1986; effective May 31, 1986. Amendment filed June 5, 1991; effective September 28, 1991.*

1200–2–10–.34 SUPPLEMENTAL FEES FOR CALENDAR YEAR 2001.

(1) Purpose. Adequate funds are required to facilitate the proper administration of The Radiological Health Service Act and The Medical Radiation Inspection Safety Act. Failure to properly administer these acts threatens the health and safety of the citizens of the state. Operating revenue for the administration of these acts is collected on a calendar year basis. Projected revenue needs of the Division in 2001 cannot be met by current registration and licensing fees. Rulemaking to increase 2001 fees cannot be completed prior to the first assessment date, January 1, 2001. Therefore, one time supplemental fees are hereby established to provide the Division with additional revenue during Calendar Year 2001. Division invoices will establish due dates for payment of these supplemental fees, except that after the effective date of this rule the operator of a disposal/processing facility shall begin to collect and submit the base fee ($0.01/lb) required by 1200–2–10–.31(8)(d) and the supplemental fee ($0.005/lb) together.

(2) Supplemental Fees Schedules.

(a) In addition to the fees established in paragraph (3) of Rule 1200–2–10–.24 Registration, persons subject to registration anytime during Calendar Year 2001 shall pay a supplemental fee to be determined according to Schedule I of this paragraph:

SCHEDULE I

Class I Equipment	$ 10.00 per tube
Class II Equipment	$ 40.00 per tube
Class III Equipment	$ 20.00 per tube
Class IV Equipment	$ 50.00 per tube
Class V Equipment	$ 200.00 per tube
Class VI Equipment	$ 300.00 per tube
Class VII Equipment	$ 500.00 per tube

SCHEDULE I

plus, for each accelerator initial review, a supplemental fee of $ 125.00 per maximum nominal rated MeV (total supplemental initial review fee not to exceed $ 50,000.00)	
A person providing inspection services under paragraph 1200–2–10–.27(4), except as provided by subparagraph 1200–2–10–.24(3)(f)————	$ 200.00

(b) In addition to the fees established in paragraphs (6) through (19) of Rule 1200–2–10–.31 Fees for Licenses, persons subject to licensure anytime during Calendar Year 2001 shall pay a supplemental fee to be determined according to Schedule II of this paragraph:

SCHEDULE II

Category GL	————	$ 50.00
Category 1	————	$ 100.00
Category 2	————	$ 200.00
Category 2d	In addition to the supplemental fee for Category 2, the operator of a disposal/processing facility shall collect and remit a supplemental fee, on items contaminated or potentially contaminated with radioactive material or on low–level radioactive waste received, of————	$0.005/lb
	Not withstanding the requirements of paragraph 1200–2–10–.31(8) and Rule 1200–2–10–.32, licensees with multiple sites within the state will be levied the supplemental fee only once on items moved directly from one site to another.	
Category 3	————	$ 300.00
Category 4	————	$ 500.00
Category 5	————	$ 700.00
Category 6	————	$ 2,000.00
Category 7	————	$ 1,000.00
Category 8	————	$ 3,750.00
Category 9	————	$ 5,000.00
Category 10	————	$ 7,500.00
Category 11	————	$ 10,000.00
Category 12	————	$ 125,000.00
Category 13	————	At least $ 50.00 not greater than $ 125,000.00
	The Category 13 supplemental fee shall be determined on a case–by–case basis. The determination shall be based on an analysis of the hazard, the scope of the difficulty encountered in the review process and the specifics of the activity, following the categories established in paragraphs (6) through (19) of Rule 1200–2–10–.31.	

Authority: *T.C.A. §§4–5–201 et seq., 68–202–201 et seq. and 68–203–201 et seq.* ***Administrative History:*** *Original rule filed April 11, 2001; effective June 25, 2001.*

1200–2–10–.35 TRAINING FOR AN AUTHORIZED NUCLEAR PHARMACIST.

(1) Training for an authorized nuclear pharmacist.

(a) Except as provided below in subparagraph (b), a licensee shall require the authorized nuclear pharmacist to be a pharmacist who:

1. Has current board certification as a nuclear pharmacist by the Board of Pharmaceutical Specialties, or

2. Has completed 700 hours in a structured educational program consisting of both:

(i) Didactic training in the following areas:

(I) Radiation physics and instrumentation;

(II) Radiation protection;

(III) Mathematics pertaining to the use and measurement of radioactivity;

(IV) Chemistry of radioactive material for medical use; and

(V) Radiation biology; and

(ii) Supervised experience in a nuclear pharmacy involving the following:

(I) Shipping, receiving and performing related radiation surveys;

(II) Using and performing checks for proper operation of dose calibrators, survey meters and, if appropriate, instruments used to measure alpha– or beta–emitting radionuclides;

(III) Calculating, assaying and safely preparing dosages for individuals;

(IV) Using administrative controls to avoid mistakes in the administration of radioactive material;

(V) Using procedures to prevent or minimize contamination and using proper decontamination procedures; and

3. Has obtained written certification, signed by a preceptor authorized nuclear pharmacist, that the above training has been satisfactorily completed and that the individual has achieved a level of competency sufficient to operate independently a nuclear pharmacy.

(b) Training for experienced nuclear pharmacists. A licensee may apply for and must receive a license amendment identifying an experienced nuclear pharmacist as an authorized nuclear pharmacist before it allows this individual to work as an authorized nuclear pharmacist. A pharmacist who has completed a structured educational program as specified in part 1200–2–10–.35(1)(a)2. before April 18, 2002, and who is working in a nuclear pharmacy would qualify as an experienced nuclear pharmacist. An experienced nuclear pharmacist need not comply with the requirements on

preceptor statement (see part 1200–2–10–.35(1)(a)3. to qualify as an authorized nuclear pharmacist.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–101 and 206* ***Administrative History:*** *Original rule filed July 18, 2002; effective October 1, 2002.*

1200–2–10–.36 RADIOLOGICAL CRITERIA FOR LICENSE TERMINATION.

(1) General provisions and scope.

(a) The criteria in this rule apply to the decommissioning of facilities licensed under Chapter 1200–2–10 and Chapters 1200–2–7, 1200–2–8, 1200–2–9, 1200–2–11 and 1200–2–12. For low–level waste disposal facilities (Chapter 1200–2–11), the criteria apply only to ancillary surface facilities that support radioactive waste disposal activities.

(b) Reserved.

(c) After a site has been decommissioned and the license terminated in accordance with the criteria in this rule, the Division will require additional cleanup if, based on new information, it determines that the criteria of this rule were not met and residual radioactivity remaining at the site could result in a significant threat to public health and safety.

(d) When calculating TEDE to the average member of the critical group the licensee shall determine the peak annual TEDE dose expected within the first 1,000 years after decommissioning.

(2) Radiological criteria for unrestricted use. A site will be considered acceptable for unrestricted use if:

(a) The residual radioactivity that is distinguishable from background radiation results in a TEDE to an average member of the critical group that does not exceed 25 mrem (0.25 mSv) per year, including that from groundwater sources of drinking water, and

(b) The residual radioactivity has been reduced to levels that are as low as reasonably achievable (ALARA). Determination of the levels that are ALARA must take into account consideration of any detriments, such as deaths from transportation accidents, potentially expected to result from decontamination and waste disposal.

(3) Criteria for license termination under restricted conditions. A site will be considered acceptable for license termination under restricted conditions if:

(a) A licensee can demonstrate that further reductions in residual radioactivity necessary to comply with the provisions of paragraph 1200–2–10–.36(2):

1. Would result in net public or environmental harm or

2. Were not being made because the residual levels associated with restricted conditions are ALARA. Determination of the levels that are ALARA must take into account consideration of any detriments, such as traffic accidents, expected to potentially result from decontamination and waste disposal;

(b) The licensee has made provisions for legally enforceable institutional controls that provide reasonable assurance that the TEDE from residual radioactivity distinguishable from background to the average member of the critical group will not exceed 25 mrem (0.25 mSv) per year;

(c) The licensee has provided sufficient financial assurance to enable an independent third party, including a governmental custodian of a site, to assume and carry out responsibilities for any necessary control and maintenance of the site. Acceptable financial assurance mechanisms are specified in paragraph 1200–2–10–.12(4); and

(d) Residual radioactivity at the site has been reduced so that if the institutional controls were no longer in effect, there is reasonable assurance that the TEDE from residual radioactivity distinguishable from background to the average member of the critical group is ALARA and would not exceed either:

1. 100 mrem (1 mSv) per year; or

2. 500 mrem (5 mSv) per year provided the licensee:

(i) Demonstrates that further reductions in residual radioactivity necessary to comply with the 100 mrem/y (1 mSv/y) value of part 1. above:

(I) Are not technically achievable,

(II) Would be prohibitively expensive or

(III) Would result in net public or environmental harm;

(ii) Makes provisions for durable institutional controls;

(iii) Provides sufficient financial assurance to enable a responsible government entity or independent third party, including a governmental custodian of a site, both to carry out periodic rechecks of the site and to assume and carry out responsibilities for any necessary control and maintenance of those controls. Periodic rechecks shall be carried out no less frequently than every five (5) years to assure that the institutional controls remain in place as necessary to meet the criteria of subparagraph 1200–2–10–.36(3)(b). Acceptable financial assurance mechanisms are those in subparagraph 1200–2–10–.12(4)(d).

(4) Alternate criteria for license termination.

(a) The Division may terminate a license using alternate criteria greater than the dose criterion of paragraph 1200–2–10–.36(2) and subparagraph 1200–2–10–.36(3)(b), if the licensee:

1. Provides assurance that public health and safety would continue to be protected, and that it is unlikely that the dose from all man–made sources combined, other than medical, would be more than the 1 mSv/y (100 mrem/y) limit of Rules 1200–2–5–.60 and 1200–2–5–.61, by submitting an analysis of possible sources of exposure;

2. Has employed to the extent practicable restrictions on site use according to the provisions of paragraph 1200–2–10–.36(3) in minimizing exposures at the site; and

(i) Reduces doses to ALARA levels, taking into consideration any detriments such as traffic accidents expected to potentially result from decontamination and waste disposal.

(ii) Reserved.

(b) The use of alternate criteria to terminate a license requires the approval of the Division. The Division will consider staff recommendations to address any comments provided by the Environmental Protection Agency and any public comments submitted under paragraph (5) below.

(5) Public notification and public participation. Whenever the Division deems such notice to be in the public interest, the Division may:

(a) Notify and solicit comments from:

1. Local governments and other State government agencies in the vicinity of the site that could be affected by the decommissioning; and

2. The Environmental Protection Agency for cases where the licensee proposes to release a site under paragraph 1200–2–10–.36(4).

(b) Publish a notice in the Tennessee Administrative Register, and in another appropriate forum that is readily accessible to individuals near the site, and solicit comments from affected parties. Another appropriate forum may include local newspapers and letters to State or local organizations.

Authority: *T.C.A. §§4–5–201 et seq. and 68–202–101 et seq.* ***Administrative History:*** *Original rule filed July 18, 2002; effective October 1, 2002.*

1200–2–10–.37 SCHEDULE 10–6: DETERMINATION OF A_1 AND A_2.

(1) Values of A_1 and A_2 for individual radionuclides, which are the bases for many activity limits elsewhere in these regulations, are given in Table A–1. The curie (Ci) values specified are obtained by converting from the terabecquerel (TBq) figure. The curie values are expressed to three significant figures to assure that the difference in the TBq and Ci quantities is 1/10 of one percent (0.1 %) or less. Where values of A_1 or A_2 are unlimited, it is for radiation control purposes only. For nuclear criticality safety, some materials are subject to controls placed on fissile material.

(2) For individual radionuclides whose identities are known but that are not listed in Table A–1, the determination of the values of A_1 and A_2 requires Division approval, except that the values of A_1 and A_2 in Table A–2 may be used without obtaining Division approval.

(3) In the calculations of A_1 and A_2 for a radionuclide not in Table A–1, a single radioactive decay chain, in which radionuclides are present in their naturally occurring proportions, and in which no daughter nuclide has a half–life either longer than ten (10) days or longer than that of the parent nuclide, shall be considered as a single radionuclide. The activity to be taken into account, and the A_1 or A_2 value to be applied, shall be those corresponding to the parent nuclide of that chain. In the case of radioactive decay chains in which any daughter

nuclide has a half–life either longer than ten (10) days or greater than that of the parent nuclide, the parent and those daughter nuclides shall be considered as mixtures of different nuclides.

(4) For mixtures of radionuclides whose identities and respective activities are known, the following conditions apply:

(a) For special form radioactive material, the maximum quantity transported in a Type A package:

$$\sum_{I} \frac{B(i)}{A_1(i)} \text{ less than or equal to } 1$$

(b) For normal form radioactive material, the maximum quantity transported in a Type A package:

$$\sum_{I} \frac{B(i)}{A_2(i)} \text{ less than or equal to } 1$$

Where B(i) is the activity of radionuclide I and $A_1(i)$ and $A_2(i)$ are the A_1 and A_2 values for radionuclide I, respectively.

Alternatively, an A_1 value for mixtures of special form material may be determined as follows:

$$A_1 \text{ for mixture} = \frac{1}{\sum_{I} \frac{f(i)}{A_1(i)}}$$

Where f(i) is the fraction of activity of nuclide I in the mixture and $A_1(i)$ is the appropriate A_1 value for nuclide I.

An A_1 value for mixtures of normal form material may be determined as follows:

$$A_2 \text{ for mixture} = \frac{1}{\sum_{I} \frac{f(i)}{A_2(i)}}$$

Where f(i) is the fraction of activity of nuclide I in the mixture and $A_2(i)$ is the appropriate A_2 value for nuclide I.

(5) When the identity of each radionuclide is known, but the individual activities of some of the radionuclides are not known, the radionuclides may be grouped. The lowest A_1 or A_2 value, as appropriate, for the radionuclides in each group may be used in applying the formulas in paragraph (4). Groups may be based on the total alpha activity and the total beta/gamma activity when these are known, using the lowest A_1 or A_2 values for the alpha emitters and beta/gamma emitters.

Table A–1: A_1 and A_2 Values for Radionuclides

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	Specific Activity (Ci/g)
Ac–225	Actinium (89)	0.6	16.2	1 (E–2)	0.270	2.1 (E+3)	5.8 (E+4)
Ac–227		40	1080	2 (E–5)	5.41 (E–4)	2.7	7.2 (E+1)
Ac–228		0.6	16.2	0.4	10.8	8.4 (E+4)	2.2 (E+6)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	Specific Activity (Ci/g)
Ag–105	Silver (47)	2	54.1	2	54.1	1.1 (E+3)	3.0 (E+4)
Ag–108m		0.6	16.2	0.6	16.2	9.7 (E–1)	2.6 (E+1)
Ag–110m		0.4	10.8	0.4	10.8	1.8 (E+2)	4.7 (E+3)
Ag–111		0.6	16.2	0.5	13.5	5.8 (E+3)	1.6 (E+5)
Al–26	Aluminum (13)	0.4	10.8	0.4	10.8	7.0 (E–4)	1.9 (E–2)
Am–241	Americium (95)	2	54.1	2 (E–4)	5.41 (E–3)	1.3 (E–1)	3.4
Am–242m		2	54.1	2 (E–4)	5.41 (E–3)	3.6 (E–1)	1.0 (E+1)
Am–243		2	54.1	2 (E–4)	5.41 (E–3)	7.4 (E–3)	2.0 (E–1)
Ar–37	Argon (18)	40	1080	40	1080	3.7 (E+3)	9.9 (E+4)
Ar–39		20	541	20	541	1.3	3.4 (E+1)
Ar–41		0.6	16.2	0.6	16.2	1.5 (E+6)	4.2 (E+7)
Ar–42		0.2	5.41	0.2	5.41	9.6	2.6 (E+2)
As–72	Arsenic (33)	0.2	5.41	0.2	5.41	6.2 (E+4)	1.7 (E+6)
As–73		40	1080	40	1080	8.2 (E+2)	2.2 (E+4)
As–74		1	27.0	0.5	13.5	3.7 (E+3)	9.9 (E+4)
As–76		0.2	5.41	0.2	5.41	5.8 (E+4)	1.6 (E+6)
As–77		20	541	0.5	13.5	3.9 (E+4)	1.0 (E+6)
At–211	Astatine (85)	30	811	2	54.1	7.6 (E+4)	2.1 (E+6)
Au–193	Gold (79)	6	162	6	162	3.4 (E+4)	9.2 (E+5)
Au–194		1	27.0	1	27.0	1.5 (E+4)	4.1 (E+5)
Au–195		10	270	10	270	1.4 (E+2)	3.7 (E+3)
Au–196		2	54.1	2	54.1	4.0 (E+3)	1.1 (E+5)
Au–198		3	81.1	0.5	13.5	9.0 (E+3)	2.4 (E+5)
Au–199		10	270	0.9	24.3	7.7 (E+3)	2.1 (E+5)
Ba–131	Barium (56)	2	54.1	2	54.1	3.1 (E+3)	8.4 (E+4)
Ba–133m		10	270	0.9	24.3	2.2 (E+4)	6.1 (E+5)
Ba–133		3	81.1	3	81.1	9.4	2.6 (E+2)
Ba–140		0.4	10.8	0.4	10.8	2.7 (E+3)	7.3 (E+4)
Be–7	Beryllium (4)	20	541	20	541	1.3 (E+4)	3.5 (E+5)
Be–10		20	541	0.5	13.5	8.3 (E–4)	2.2 (E–2)
Bi–205	Bismuth (83)	0.6	16.2	0.6	16.2	1.5 (E–3)	4.2 (E+4)
Bi–206		0.3	8.11	0.3	8.11	3.8 (E+3)	1.0 (E+5)
Bi–207		0.7	18.9	0.7	18.9	1.9	5.2 (E+1)
Bi–210m		0.3	8.11	3 (E–2)	0.811	2.1 (E–5)	5.7 (E–4)
Bi–210		0.6	16.2	0.5	13.5	4.6 (E+3)	1.2 (E+5)
Bi–212		0.3	8.11	0.3	8.11	5.4 (E+5)	1.5 (E+7)
Bk–247	Berkelium (97)	2	54.1	2 (E–4)	5.41 (E–3)	3.8 (E–2)	1.0
Bk–249		40	1080	8 (E–2)	2.16	6.1 (E+1)	1.6 (E+3)
Br–76	Bromine (35)	0.3	8.11	0.3	8.11	9.4 (E+4)	2.5 (E+6)
Br–77		3	81.1	3	81.1	2.6 (E+4)	7.1 (E+5)
Br–82		0.4	10.8	0.4	10.8	4.0 (E+4)	1.1 (E+6)
C–11	Carbon (6)	1	27	0.5	13.5	3.1 (E+7)	8.4 (E+8)
C–14		40	1080	2	54.1	1.6 (E–1)	4.5
Ca–41	Calcium (20)	40	1080	40	1080	3.1 (E–3)	8.5 (E–2)
Ca–45		40	1080	0.9	24.3	6.6 (E+2)	1.8 (E+4)
Ca–47		0.9	24.3	0.5	13.5	2.3 (E+4)	6.1 (E+5)
Cd–109	Cadmium (48)	40	1080	1	27.0	9.6 (E+1)	2.6 (E+3)
Cd–113m		20	541	9 (E–2)	2.43	8.3	2.2 (E+2)
Cd–115m		0.3	8.11	0.3	8.11	9.4 (E+2)	2.5 (E+4)
Cd–115		4	108	0.5	13.5	1.9 (E+4)	5.1 (E+5)
Ce–139	Cerium (58)	6	162	6	162	2.5 (E+2)	6.8 (E+3)
Ce–141		10	270	0.5	13.5	1.1 (E+3)	2.8 (E+4)
Ce–143		0.6	16.2	0.5	13.5	2.5 (E+4)	6.6 (E+5)
Ce–144		0.2	5.41	0.2	5.41	1.2 (E+2)	3.2 (E+3)
Cf–248	Californium (98)	30	811	3 (E–3)	8.11 (E–2)	5.8 (E+1)	1.6 (E+3)
Cf–249		2	54.1	2 (E–4)	5.41 (E–3)	1.5 (E–1)	4.1
Cf–250		5	135	5 (E–4)	1.35 (E–2)	4.0	1.1 (E+2)
Cf–251		2	54.1	2 (E–4)	5.41 (E–3)	5.9 (E–2)	1.6
Cf–252		0.1	2.70	1 (E–3)	2.70 (E–2)	2.0 (E+1)	5.4 (E+2)
Cf–253		40	1080	6 (E–2)	1.62	1.1 (E+3)	2.9 (E+4)
Cf–254		3 (E–3)	8.11 (E–2)	6 (E–4)	1.62 (E–2)	3.1 (E+2)	8.5 (E+3)
Cl–36	Chlorine (17)	20	541	0.5	13.5	1.2 (E–3)	3.3 (E–2)
Cl–38		0.2	5.41	0.2	5.41	4.9 (E+6)	1.3 (E+8)
Cm–240	Curium (96)	40	1080	2 (E–2)	0.541	7.5 (E+2)	2.0 (E+4)
Cm–241		2	54.1	0.9	24.3	6.1 (E+2)	1.7 (E+4)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	(Ci/g)
Cm–242		40	1080	1 (E–2)	0.270	1.2 (E+2)	3.3 (E+3)
Cm–243		3	81.1	3 (E–4)	8.11 (E–3)	1.9	5.2 (E+1)
Cm–244		4	108	4 (E–4)	1.08 (E–2)	3.0	8.1 (E+1)
Cm–245		2	54.1	2 (E–4)	5.41 (E–3)	6.4 (E–3)	1.7 (E–1)
Cm–246		2	54.1	2 (E–4)	5.41 (E–3)	1.1 (E–2)	3.1 (E–1)
Cm–247		2	54.1	2 (E–4)	5.41 (E–3)	3.4 (E–6)	9.3 (E–5)
Cm–248		4 (E–2)	1.08	5 (E–5)	1.35 (E–3)	1.6 (E–4)	4.2 (E–3)
Co–55	Cobalt (27)	0.5	13.5	0.5	13.5	1.1 (E+5)	3.1 (E+6)
Co–56		0.3	8.11	0.3	8.11	1.1 (E+3)	3.0 (E+4)
Co–57		8	216	8	216	3.1 (E+2)	8.4 (E+3)
Co–58m		40	1080	40	1080	2.2 (E+5)	5.9 (E+6)
Co–58		1	27.0	1	27.0	1.2 (E+3)	3.2 (E+4)
Co–60		0.4	10.8	0.4	10.8	4.2 (E+1)	1.1 (E+3)
Cr–51	Chromium (24)	30	811	30	811	3.4 (E+3)	9.2 (E+4)
Cs–129	Cesium (55)	4	108	4	108	2.8 (E+4)	7.6 (E+5)
Cs–131		40	1080	40	1080	3.8 (E+3)	1.0 (E+5)
Cs–132		1	27.0	1	27.0	5.7 (E+3)	1.5 (E+5)
Cs–134m		40	1080	9	243	3.0 (E+5)	8.0 (E+6)
Cs–134		0.6	16.2	0.5	13.5	4.8 (E+1)	1.3 (E+3)
Cs–135		40	1080	0.9	24.3	4.3 (E–5)	1.2 (E–3)
Cs–136		0.5	13.5	0.5	13.5	2.7 (E+3)	7.3 (E+4)
Cs–137		2	54.1	0.5	13.5	3.2	8.7 (E+1)
Cu–64	Copper (29)	5	135	0.9	24.3	1.4 (E+5)	3.9 (E+6)
Cu–67		9	243	0.9	24.3	2.8 (E+4)	7.6 (E+5)
Dy–159	Dysprosium (66)	20	541	20	541	2.1 (E+2)	5.7 (E+3)
Dy–165		0.6	16.2	0.5	13.5	3.0 (E+5)	8.2 (E+6)
Dy–166		0.3	8.11	0.3	8.11	8.6 (E+3)	2.3 (E+5)
Er–169	Erbium (68)	40	1080	0.9	24.3	3.1 (E+3)	8.3 (E+4)
Er–171		0.6	16.2	0.5	13.5	9.0 (E+4)	2.4 (E+6)
Es–253	Einsteinium (99) [a]	200	5400	2 (E–2)	5.41 (E–1)	.....	.....
Es–254		30	811	3 (E–3)	8.11 (E–2)	.....	.....
Es–254m		0.6	16.2	0.4	10.8	.....	.....
Es–255							
Eu–147	Europium (63)	2	54.1	2	54.1	1.4 (E+3)	3.7 (E+4)
Eu–148		0.5	13.5	0.5	13.5	6.0 (E+2)	1.6 (E+4)
Eu–149		20	541	20	541	3.5 (E+2)	9.4 (E+3)
Eu–150		0.7	18.9	0.7	18.9	6.1 (E+4)	1.6 (E+6)
Eu–152m		0.6	16.2	0.5	13.5	8.2 (E+4)	2.2 (E+6)
Eu–152		0.9	24.3	0.9	24.3	6.5	1.8 (E+2)
Eu–154		0.8	21.6	0.5	13.5	9.8	2.6 (E+2)
Eu–155		20	541	2	54.1	1.8 (E+1)	4.9 (E+2)
Eu–156		0.6	16.2	0.5	13.5	2.0 (E+3)	5.5 (E+4)
F–18	Fluorine (9)	1	27.0	0.5	13.5	3.5 (E+6)	9.5 (E+7)
Fe–52	Iron (26)	0.2	5.41	0.2	5.41	2.7 (E+5)	7.3 (E+6)
Fe–55		40	1080	40	1080	8.8 (E+1)	2.4 (E+3)
Fe–59		0.8	21.6	0.8	21.6	1.8 (E+3)	5.0 (E+4)
Fe–60		40	1080	0.2	5.41	7.4 (E–4)	2.0 (E–2)
Fm–255	Fermium (100) [b]	40	1080	0.8	21.6		
Fm–257		10	270	8 (E–3)	2.16 (E–1)		
Ga–67	Gallium (31)	6	162	6	162	2.2 (E+4)	6.0 (E+5)
Ga–68		0.3	8.11	0.3	8.11	1.5 (E+6)	4.1 (E+7)
Ga–72		0.4	10.8	0.4	10.8	1.1 (E+5)	3.1 (E+6)
Gd–146	Gadolinium (64)	0.4	10.8	0.4	10.8	6.9 (E+2)	1.9 (E+4)
Gd–148		3	81.1	3 (E–4)	8.11 (E–3)	1.2	3.2 (E+1)
Gd–153		10	270	5	135	1.3 (E+2)	3.5 (E+3)
Gd–159		4	108	0.5	13.5	3.9 (E+4)	1.1 (E+6)
Ge–68	Germanium (32)	0.3	8.11	0.3	8.11	2.6 (E+2)	7.1 (E+3)
Ge–71		40	1080	40	1080	5.8 (E+3)	1.6 (E+5)
Ge–77		0.3	8.11	0.3	8.11	1.3 (E+5)	3.6 (E+6)
H–3	Hydrogen (1)		See T–Tritium				
Hf–172	Hafnium (72)	0.5	13.5	0.3	8.11	4.1 (E+1)	1.1 (E+3)
Hf–175		3	81.1	3	81.1	3.9 (E+2)	1.1 (E+4)
Hf–181		2	54.1	0.9	24.3	6.3 (E+2)	1.7 (E+4)
Hf–182		4	108	3 (E–2)	0.811	8.1 (E–6)	2.2 (E–4)
Hg–194	Mercury (80)	1	27.0	1	27.0	1.3 (E–1)	3.5

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	Specific Activity (Ci/g)
Hg–195m		5	135	5	135	1.5 (E+4)	4.0 (E+5)
Hg–197m		10	270	0.9	24.3	2.5 (E+4)	6.7 (E+5)
Hg–197		10	270	10	270	9.2 (E+3)	2.5 (E+5)
Hg–203		4	108	0.9	24.3	5.1 (E+2)	1.4 (E+4)
Ho–163	Holmium (67)	40	1080	40	1080	2.7	7.6 (E+1)
Ho–166m		0.6	16.2	0.3	8.11	6.6 (E–2)	1.8
Ho–166		0.3	8.11	0.3	8.11	2.6 (E+4)	7.0 (E+5)
I–123	Iodine (53)	6	162	6	162	7.1 (E+4)	1.9 (E+6)
I–124		0.9	24.3	0.9	24.3	9.3 (E+3)	2.5 (E+5)
I–125		20	541	2	54.1	6.4 (E+2)	1.7 (E+4)
I–126		2	54.1	0.9	24.3	2.9 (E+3)	8.0 (E+4)
I–129		Unlimited	Unlimited	Unlimited	Unlimited	6.5 (E–6)	1.8 (E–4)
I–131		3	81.1	0.5	13.5	4.6 (E+3)	1.2 (E+5)
I–132		0.4	10.8	0.4	10.8	3.8 (E+5)	1.0 (E+7)
I–133		0.6	16.2	0.5	13.5	4.2 (E+4)	1.1 (E+6)
I–134		0.3	8.11	0.3	8.11	9.9 (E+5)	2.7 (E+7)
I–135		0.6	16.2	0.5	13.5	1.3 (E+5)	3.5 (E+6)
In–111	Indium (49)	2	54.1	2	54.1	1.5 (E+4)	4.2 (E+5)
In–113m		4	108	4	108	6.2 (E+5)	1.7 (E+7)
In–114m		0.3	8.11	0.3	8.11	8.6 (E+2)	2.3 (E+4)
In–115m		6	162	0.9	24.3	2.2 (E+5)	6.1 (E+6)
Ir–189	Iridium (77)	10	270	10	270	1.9 (E+3)	5.2 (E+4)
Ir–190		0.7	18.9	0.7	18.9	2.3 (E+3)	6.2 (E+4)
Ir–192		1	27.0	0.5	13.5	3.4 (E+2)	9.2 (E+3)
Ir–193m		10	270	10	270	2.4 (E+3)	6.4 (E+4)
Ir–194		0.2	5.41	0.2	5.41	3.1 (E+4)	8.4 (E+5)
K–40	Potassium (19)	0.6	16.2	0.6	16.2	2.4 (E–7)	6.4 (E–6)
K–42		0.2	5.41	0.2	5.41	2.2 (E+5)	6.0 (E+6)
K–43		1.0	27.0	0.5	13.5	1.2 (E+5)	3.3 (E+6)
Kr–81	Krypton (36)	40	1080	40	1080	7.8 (E–4)	2.1 (E–2)
Kr–85m		6	162	6	162	3.0 (E+5)	8.2 (E+6)
Kr–85		20	541	10	270	1.5 (E+1)	3.9 (E+2)
Kr–87		0.2	5.41	0.2	5.41	1.0 (E+6)	2.8 (E+7)
La–137	Lanthanum (57)	40	1080	2	54.1	1.6 (E–3)	4.4 (E–2)
La–140		0.4	10.8	0.4	10.8	2.1 (E+4)	5.6 (E+5)
Lu–172	Lutetium (71)	0.5	13.5	0.5	13.5	4.2 (E+3)	1.1 (E+5)
Lu–173		8	216	8	216	5.6 (E+1)	1.5 (E+3)
Lu–174m		20	541	8	216	2.0 (E+2)	5.3 (E+3)
Lu–174		8	216	4	108	2.3 (E+1)	6.2 (E+2)
Lu–177		30	811	0.9	24.3	4.1 (E+3)	1.1 (E+5)
MFP	For mixed fission products, use formula for mixtures or Table A–2						
Mg–28	Magnesium (12)	0.2	5.41	0.2	5.41	2.0 (E+5)	5.4 (E+6)
Mn–52	Manganese (25)	0.3	8.11	0.3	8.11	1.6 (E+4)	4.4 (E+5)
Mn–53		Unlimited	Unlimited	Unlimited	Unlimited	6.8 (E–5)	1.8 (E–3)
Mn–54		1	27.0	1	27.0	2.9 (E+2)	7.7 (E+3)
Mn–56		0.2	5.41	0.2	5.41	8.0 (E+5)	2.2 (E+7)
Mo–93	Molybdenum (42)	40	1080	7	189	4.1 (E–2)	1.1
Mo–99		0.6	16.2	0.5	13.5 [c]	1.8 (E+4)	4.8 (E+5)
N–13	Nitrogen (7)	0.6	16.2	0.5	13.5	5.4 (E+7)	1.5 (E+9)
Na–22	Sodium (11)	0.5	13.5	0.5	13.5	2.3 (E+2)	6.3 (E+3)
Na–24		0.2	5.41	0.2	5.41	3.2 (E+5)	8.7 (E+6)
Nb–92m	Niobium (41)	0.7	18.9	0.7	18.9	5.2 (E+3)	1.4 (E+5)
Nb–93m		40	1080	6	162	8.8	2.4 (E+2)
Nb–94		0.6	16.2	0.6	16.2	6.9 (E–3)	1.9 (E–1)
Nb–95		1	27.0	1	27.0	1.5 (E+3)	3.9 (E+4)
Nb–97		0.6	16.2	0.5	13.5	9.9 (E+5)	2.7 (E+7)
Nd–147	Neodymium (60)	4	108	0.5	13.5	3.0 (E+3)	8.1 (E+4)
Nd–149		0.6	16.2	0.5	13.5	4.5 (E+5)	1.2 (E+7)
Ni–59	Nickel (28)	40	1080	40	1080	3.0 (E–3)	8.0 (E–2)
Ni–63		40	1080	30	811	2.1	5.7 (E+1)
Ni–65		0.3	8.11	0.3	8.11	7.1 (E+5)	1.9 (E+7)
Np–235	Neptunium (93)	40	1080	40	1080	5.2 (E+1)	1.4 (E+3)
Np–236		7	189	1 (E–3)	2.70 (E–2)	4.7 (E–4)	1.3 (E–2)
Np–237		2	54.1	2 (E–4)	5.41 (E–3)	2.6 (E–5)	7.1 (E–4)
Np–239		6	162	0.5	13.5	8.6 (E+3)	2.3 (E+5)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	Specific Activity (Ci/g)
Os–185	Osmium (76)	1	27.0	1	27.0	2.8 (E+2)	7.5 (E+3)
Os–191m		40	1080	40	1080	4.6 (E+4)	1.3 (E+6)
Os–191		10	270	0.9	24.3	1.6 (E+3)	4.4 (E+4)
Os–193		0.6	16.2	0.5	13.5	2.0 (E+4)	5.3 (E+5)
Os–194		0.2	5.41	0.2	5.41	1.1 (E+1)	3.1 (E+2)
P–32	Phosphorus (15)	0.3	8.11	0.3	8.11	1.1 (E+4)	2.9 (E+5)
P–33		40	1080	0.9	24.3	5.8 (E+3)	1.6 (E+5)
Pa–230	Protactinium (91)	2	54.1	0.1	2.70	1.2 (E+3)	3.3 (E+4)
Pa–231		0.6	16.2	6 (E–5)	1.62 (E–3)	1.7 E–3)	4.7 (E–2)
Pa–233		5	135	0.9	24.3	7.7 (E+2)	2.1 (E+4)
Pb–201	Lead (82)	1	27.0	1	27.0	6.2 (E+4)	1.7 (E+6)
Pb–202		40	1080	2	54.1	1.2 (E–4)	3.4 (E–3)
Pb–203		3	81.1	3	81.1	1.1 (E+4)	3.0 (E+5)
Pb–205		Unlimited	Unlimited	Unlimited	Unlimited	4.5 (E–6)	1.2 (E–4)
Pb–210		0.6	16.2	9 (E–3)	0.243	2.8	7.6 (E+1)
Pb–212		0.3	8.11	0.3	8.11	5.1 (E+4)	1.4 (E+6)
Pd–103	Palladium (46)	40	1080	40	1080	2.8 (E+3)	7.5 (E+4)
Pd–107		Unlimited	Unlimited	Unlimited	Unlimited	1.9 (E–5)	5.1 (E–4)
Pd–109		0.6	16.2	0.5	13.5	7.9 (E+4)	2.1 (E+6)
Pm–143	Promethium (61)	3	81.1	3	81.1	1.3 (E+2)	3.4 (E+3)
Pm–144		0.6	16.2	0.6	16.2	9.2 (E+1)	2.5 (E+3)
Pm–145		30	811	7	189	5.2	1.4 (E+2)
Pm–147		40	1080	0.9	24.3	3.4 (E+1)	9.3 (E+2)
Pm–148m		0.5	13.5	0.5	13.5	7.9 (E+2)	2.1 (E+4)
Pm–149		0.6	16.2	0.5	13.5	1.5 (E+4)	4.0 (E+5)
Pm–151		3	81.1	0.5	13.5	2.7 (E+4)	7.3 (E+5)
Po–208	Polonium (84)	40	1080	2 (E–2)	0.541	2.2 (E+1)	5.9 (E+2)
Po–209		40	1080	2 (E–2)	0.541	6.2 (E–1)	1.7 (E+1)
Po–210		40	1080	2 (E–2)	0.541	1.7 (E+2)	4.5 (E+3)
Pr–142	Praseodymium(59)	0.2	5.41	0.2	5.41	4.3 (E+4)	1.2 (E+6)
Pr–143		4	108	0.5	13.5	2.5 (E+3)	6.7 (E+4)
Pt–188	Platinum (78)	0.6	16.2	0.6	16.2	2.5 (E+3)	6.8 (E+4)
Pt–191		3	81.1	3	81.1	8.7 (E+3)	2.4 (E+5)
Pt–193m		40	1080	9	243	5.8 (E+3)	1.6 (E+5)
Pt–193		40	1080	40	1080	1.4	3.7 (E+1)
Pt–195m		10	270	2	54.1	6.2 (E+3)	1.7 (E+5)
Pt–197m		10	270	0.9	24.3	3.7 (E+5)	1.0 (E+7)
Pt–197		20	541	0.5	13.5	3.2 (E+4)	8.7 (E+5)
Pu–236	Plutonium (94)	7	189	7 (E–4)	1.89 (E–2)	2.0 (E+1)	5.3 (E+2)
Pu–237		20	541	20	541	4.5 (E+2)	1.2 (E+4)
Pu–238		2	54.1	2 (E–4)	5.41 (E–3)	6.3 (E–1)	1.7 (E+1)
Pu–239		2	54.1	2 (E–4)	5.41 (E–3)	2.3 (E–3)	6.2 (E–2)
Pu–240		2	54.1	2 (E–4)	5.41 (E–3)	8.4 (E–3)	2.3 (E–1)
Pu–241		40	1080	1 (E–2)	0.270	3.8	1.0 (E+2)
Pu–242		2	54.1	2 (E–4)	5.41 (E–3)	1.5 (E–4)	3.9 (E–3)
Pu–244		0.3	8.11	2 (E–4)	5.41 (E–3)	6.7 (E–7)	1.8 (E–5)
Ra–223	Radium (88)	0.6	16.2	3 (E–2)	0.811	1.9 (E+3)	5.1 (E+4)
Ra–224		0.3	8.11	6 (E–2)	1.62	5.9 (E+3)	1.6 (E+5)
Ra–225		0.6	16.2	2 (E–2)	0.541	1.5 (E+3)	3.9 (E+4)
Ra–226		0.3	8.11	2 (E–2)	0.541	3.7 (E–2)	1.0
Ra–228		0.6	16.2	4 (E–2)	1.08	1.0 (E+1)	2.7 (E+2)
Rb–81	Rubidium (37)	2	54.1	0.9	24.3	3.1 (E+5)	8.4 (E+6)
Rb–83		2	54.1	2	54.1	6.8 (E+2)	1.8 (E+4)
Rb–84		1	27.0	0.9	24.3	1.8 (E+3)	4.7 (E+4)
Rb–86		0.3	8.11	0.3	8.11	3.0 (E+3)	8.1 (E+4)
Rb–87		Unlimited	Unlimited	Unlimited	Unlimited	3.2 (E–9)	8.6 (E–8)
Rb (natural)		Unlimited	Unlimited	Unlimited	Unlimited	6.7 (E+6)	1.8 (E+8)
Re–183	Rhenium (75)	5	135	5	135	3.8 (E+2)	1.0 (E+4)
Re–184m		3	81.1	3	81.1	1.6 (E+2)	4.3 (E+3)
Re–184		1	27.0	1	27.0	6.9 (E+2)	1.9 (E+4)
Re–186		4	108	0.5	13.5	6.9 (E+3)	1.9 (E+5)
Re–187		Unlimited	Unlimited	Unlimited	Unlimited	1.4 (E–9)	3.8 (E–8)
Re–188		0.2	5.41	0.2	5.41	3.6 (E+4)	9.8 (E+5)
Re–189		4	108	0.5	13.5	2.5 (E+4)	6.8 (E+5)
Re (natural)		Unlimited	Unlimited	Unlimited	Unlimited		2.4 (E–8)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	(Ci/g)
Rh–99	Rhodium (45)	2	54.1	2	54.1	3.0 (E+3)	8.2 (E+4)
Rh–101		4	108	4	108	4.1 (E+1)	1.1 (E+3)
Rh–102m		2	54.1	0.9	24.3	2.3 (E+2)	6.2 (E+3)
Rh–102		0.5	13.5	0.5	13.5	4.5 (E+1)	1.2 (E+3)
Rh–103m		40	1080	40	1080	1.2 (E+6)	3.3 (E+7)
Rh–105		10	270	0.9	24.3	3.1 (E+4)	8.4 (E+5)
Rn–222	Radon (86)	0.2	5.41	4 (E–3)	0.108	5.7 (E+3)	1.5 (E+5)
Ru–97	Ruthenium (44)	4	108	4	108	1.7 (E+4)	4.6 (E+5)
Ru–103		2	54.1	0.9	24.3	1.2 (E+3)	3.2 (E+4)
Ru–105		0.6	16.2	0.5	13.5	2.5 (E+5)	6.7 (E+6)
Ru–106		0.2	5.41	0.2	5.41	1.2 (E+2)	3.3 (E+3)
S–35	Sulfur (16)	40	1080	2	54.1	1.6 (E+3)	4.3 (E+4)
Sb–122	Antimony (51)	0.3	8.11	0.3	8.11	1.5 (E+4)	4.0 (E+5)
Sb–124		0.6	16.2	0.5	13.5	6.5 (E+2)	1.7 (E+4)
Sb–125		2	54.1	0.9	24.3	3.9 (E+1)	1.0 (E+3)
Sb–126		0.4	10.8	0.4	10.8	3.1 (E+3)	8.4 (E+4)
Sc–44	Scandium (21)	0.5	13.5	0.5	13.5	6.7 (E+5)	1.8 (E+7)
Sc–46		0.5	13.5	0.5	13.5	1.3 (E+3)	3.4 (E+4)
Sc–47		9	243	0.9	24.3	3.1 (E+4)	8.3 (E+5)
Sc–48		0.3	8.11	0.3	8.11	5.5 (E+4)	1.5 (E+6)
Se–75	Selenium (34)	3	81.1	3	81.1	5.4 (E+2)	1.5 (E+4)
Se–79		40	1080	2	54.1	2.6 (E–3)	7.0 (E–2)
Si–31	Silicon (14)	0.6	16.2	0.5	13.5	1.4 (E+6)	3.9 (E+7)
Si–32		40	1080	0.2	5.41	3.9	1.1 (E+2)
Sm–145	Samarium (62)	20	541	20	541	9.8 (E+1)	2.6 (E+3)
Sm–147		Unlimited	Unlimited	Unlimited	Unlimited	8.5 (E–1)	2.3 (E–8)
Sm–151		40	1080	4	108	9.7 (E–1)	2.6 (E+1)
Sm–153		4	108	0.5	13.5	1.6 (E+4)	4.4 (E+5)
Sn–113	Tin (50)	4	108	4	108	3.7 (E+2)	1.0 (E+4)
Sn–117m		6	162	2	54.1	3.0 (E+3)	8.2 (E+4)
Sn–119m		40	1080	40	1080	1.4 (E+2)	3.7 (E+3)
Sn–121m		40	1080	0.9	24.3	2.0	5.4 (E+1)
Sn–123		0.6	16.2	0.5	13.5	3.0 (E+2)	8.2 (E+3)
Sn–125		0.2	5.41	0.2	5.41	4.0 (E+3)	1.1 (E+5)
Sn–126		0.3	8.11	0.3	8.11	1.0 (E–3)	2.8 (E–2)
Sr–82	Strontium (38)	0.2	5.41	0.2	5.41	2.3 (E+3)	6.2 (E+4)
Sr–85m		5	135	5	135	1.2 (E+6)	3.3 (E+7)
Sr–85		2	54.1	2	54.1	8.8 (E+2)	2.4 (E+4)
Sr–87m		3	81.1	3	81.1	4.8 (E+5)	1.3 (E+7)
Sr–89		0.6	16.2	0.5	13.5	1.1 (E+3)	2.9 (E+4)
Sr–90		0.2	5.41	0.1	2.70	5.1	1.4 (E+2)
Sr–91		0.3	8.11	0.3	8.11	1.3 (E+5)	3.6 (E+6)
Sr–92		0.8	21.6	0.5	13.5	4.7 (E+5)	1.3 (E+7)
T	Tritium (1)	40	1080	40	1080	3.6 (E+2)	9.7 (E+3)
Ta–178	Tantalum (73)	1	27.0	1	27.0	4.2 (E+6)	1.1 (E+8)
Ta–179		30	811	30	811	4.1 (E+1)	1.1 (E+3)
Ta–182		0.8	21.6	0.5	13.5	2.3 (E+2)	6.2 (E+3)
Tb–157	Terbium (65)	40	1080	10	270	5.6 (E–1)	1.5 (E+1)
Tb–158		1	27.0	0.7	18.9	5.6 (E–1)	1.5 (E+1)
Tb–160		0.9	24.3	0.5	13.5	4.2 (E+2)	1.1 (E+4)
Tc–95m	Technetium (43)	2	54.1	2	54.1	8.3 (E+2)	2.2 (E+4)
Tc–96m		0.4	10.8	0.4	10.8	1.4 (E+6)	3.8 (E+7)
Tc–96		0.4	10.8	0.4	10.8	1.2 (E+4)	3.2 (E+5)
Tc–97m		40	1080	40	1080	5.6 (E+2)	1.5 (E+4)
Tc–97		Unlimited	Unlimited	Unlimited	Unlimited	5.2 (E–5)	1.4 (E–3)
Tc–98		0.7	18.9	0.7	18.9	3.2 (E–5)	8.7 (E–4)
Tc–99m		8	216	8	216	1.9 (E+5)	5.3 (E+6)
Tc–99		40	1080	0.9	24.3	6.3 (E–4)	1.7 (E–2)
Te–118	Tellurium (52)	0.2	5.41	0.2	5.41	6.8 (E+3)	1.8 (E+5)
Te–121m		5	135	5	135	2.6 (E+2)	7.0 (E+3)
Te–121		2	54.1	2	54.1	2.4 (E+3)	6.4 (E+4)
Te–123m		7	189	7	189	3.3 (E+2)	8.9 (E+3)
Te–125m		30	811	9	243	6.7 (E+2)	1.8 (E+4)
Te–127m		20	541	0.5	13.5	3.5 (E+2)	9.4 (E+3)
Te–127		20	541	0.5	13.5	9.8 (E+4)	2.6 (E+6)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	Specific Activity (Ci/g)
Te–129m		0.6	16.2	0.5	13.5	1.1 (E+3)	3.0 (E+4)
Te–129		0.6	16.2	0.5	13.5	7.7 (E+5)	2.1 (E+7)
Te–131m		0.7	18.9	0.5	13.5	3.0 (E+4)	8.0 (E+5)
Te–132		0.4	10.8	0.4	10.8	1.1 (E+4)	3.0 (E+5)
Th–227	Thorium (90)	9	243	1 (E–2)	0.270	1.1 (E+3)	3.1 (E+4)
Th–228		0.3	8.11	4 (E–4)	1.08 (E–2)	3.0 (E+1)	8.2 (E+2)
Th–229		0.3	8.11	3 (E–5)	8.11 (E–4)	7.9 (E–3)	2.1 (E–1)
Th–230		2	54.1	2 (E–4)	5.41 (E–3)	7.6 (E–4)	2.1 (E–2)
Th–231		40	1080	0.9	24.3	2.0 (E+4)	5.3 (E+5)
Th–232		Unlimited	Unlimited	Unlimited	Unlimited	4.0 (E–9)	1.1 (E–7)
Th–234		0.2	5.41	0.2	5.41	8.6 (E+2)	2.3 (E+4)
Th (natural)		Unlimited	Unlimited	Unlimited	Unlimited	8.1 (E–9)	2.2 (E–7)
Ti–44	Titanium (22)	0.5	13.5	0.2	5.41	6.4	1.7 (E+2)
Tl–200	Thallium (**81.1)	0.8	21.6	0.8	21.6	2.2 (E+4)	6.0 (E+5)
Tl–201		10	270	10	270	7.9 (E+3)	2.1 (E+5)
Tl–202		2	54.1	2	54.1	2.0 (E+3)	5.3 (E+4)
Tl–204		4	108	0.5	13.5	1.7 (E+1)	4.6 (E+2)
Tm–167	Thulium (69)	7	189	7	189	3.1 (E+3)	8.5 (E+4)
Tm–168		0.8	21.6	0.8	21.6	3.1 (E+2)	8.3 (E+3)
Tm–170		4	108	0.5	13.5	2.2 (E+2)	6.0 (E+3)
Tm–171		40	1080	10	270	4.0 (E+1)	1.1 (E+3)
U–230	Uranium (92)	40	1080	1 (E–2)	0.270	1.0 (E+3)	2.7 (E+4)
U–232		3	81.1	3 (E–4)	8.11 (E–3)	8.3 (E–1)	2.2 (E+1)
U–233		10	270	1 (E–3)	2.70 (E–2)	3.6 (E–4)	9.7 (E–3)
U–234		10	270	1 (E–3)	2.70 (E–2)	2.3 (E–4)	6.2 (E–3)
U–235		Unlimited	Unlimited	Unlimited	Unlimited	8.0 (E–8)	2.2 (E–6)
U–236		10	270	1 (E–3)	2.70 (E–2)	2.4 (E–6)	6.5 (E–5)
U–238		Unlimited	Unlimited	Unlimited	Unlimited	1.2 (E–8)	3.4 (E–7)
U (natural)		Unlimited	Unlimited	Unlimited	Unlimited	2.6 (E–8)	7.1 (E–7)
U (enriched 5% or less)		Unlimited	Unlimited	Unlimited	Unlimited		(See Table A–3)
U (enriched more than 5%)		10	270	1 (E–3)	2 70 (E–2)		(See Table A–3)
U (depleted)		Unlimited	Unlimited	Unlimited	Unlimited		(See Table A–3)
V–48	Vanadium (23)	0.3	8.11	0.3	8.11	6.3 (E+3)	1.7 (E+5)
V–49		40	1080	40	1080	3.0 (E+2)	8.1 (E+3)
W–178	Tungsten (74)	1	27.0	1	27.0	1.3 (E+3)	3.4 (E+4)
W–181		30	811	30	811	2.2 (E+2)	6.0 (E+3)
W–185		40	1080	0.9	24.3	3.5 (E+2)	9.4 (E+3)
W–187		2	54.1	0.5	13.5	2.6 (E+4)	7.0 (E+5)
W–188		0.2	5.41	0.2	5.41	3.7 (E+2)	1.0 (E+4)
Xe–122	Xenon (54)	0.2	5.41	0.2	5.41	4.8 (E+4)	1.3 (E+6)
Xe–123		0.2	5.41	0.2	5.41	4.4 (E+5)	1.2 (E+7)
Xe–127		4	108	4	108	1.0 (E+3)	2.8 (E+4)
Xe–131m		40	1080	40	1080	3.1 (E+3)	8.4 (E+4)
Xe–133		20	541	20	541	6.9 (E+3)	1.9 (E+5)
Xe–135		4	108	4	108	9.5 (E+4)	2.6 (E+6)
Y–87	Yttrium (39)	2	54.1	2	54.1	1.7 (E+4)	4.5 (E+5)
Y–88		0.4	10.8	0.4	10.8	5.2 (E+2)	1.4 (E+4)
Y–90		0.2	5.41	0.2	5.41	2.0 (E+4)	5.4 (E+5)
Y–91m		2	54.1	2	54.1	1.5 (E+6)	4.2 (E+7)
Y–91		0.3	8.11	0.3	8.11	9.1 (E+2)	2.5 (E+4)
Y–92		0.2	5.41	0.2	5.41	3.6 (E+5)	9.6 (E+6)
Y–93		0.2	5.41	0.2	5.41	1.2 (E+5)	3.3 (E+6)
Yb–169	Ytterbium (70)	3	81.1	3	81.1	8.9 (E+2)	2.4 (E+4)
Yb–175		30	811	0.9	24.3	6.6 (E+3)	1.8 (E+5)
Zn–65	Zinc (30)	2	54.1	2	54.1	3.0 (E+2)	8.2 (E+3)
Zn–69m		2	54.1	0.5	13.5	1.2 (E+5)	3.3 (E+6)
Zn–69		4	108	0.5	13.5	1.8 (E+6)	4.9 (E+7)
Zr–88	Zirconium (40)	3	81.1	3	81.1	6.6 (E+2)	1.8 (E+4)
Zr–93		40	1080	0.2	5.41	9.3 (E–5)	2.5 (E–3)
Zr–95		1	27.0	0.9	24.3	7.9 (E+2)	2.1 (E+4)
Zr–97		0.3	8.11	0.3	8.11	7.1 (E+4)	1.9 (E+6)

Symbol of radio–nuclide	Element and atomic number	A_1 (TBq)	A_1 (Ci)	A_2 (TBq)	A_2 (Ci)	Specific Activity (TBq/g)	(Ci/g)

[a] International shipments of einsteinium require multilateral approval of A_1 and A_2 values.

[b] International shipments of fermium require multilateral approval of A_1 and A_2 values.

[c] 20 Ci for Mo–99 for domestic use.

Table A–2: General Values for A_1 and A_2

Contents	A_1		A_2	
	(TBq)	(Ci)	(TBq)	(Ci)
Only beta– or gamma–emitting nuclides are known to be present.	0.2	5	0.02	0.5
Alpha–emitting nuclides are known to be present, or no relevant data are available	0.10	2.70	2 (E–5)	5.41 (E–4)

Table A–3: Activity–mass Relationships for Uranium

Uranium Enrichment [a] wt % U–235 present	Specific Activity	
	TBq/g	Ci/g
0.45	1.8 (E–8)	5.0 (E–7)
0.72	2.6 (E–8)	7.1 (E–7)
1.0	2.8 (E–8)	7.6 (E–7)
1.5	3.7 (E–8)	1.0 (E–6)
5.0	1.0 (E–7)	2.7 (E–6)
10.0	1.8 (E–7)	4.8 (E–6)
20.0	3.7 (E–7)	1.0 (E–5)
35.0	7.4 (E–7)	2.0 (E–5)
50.0	9.3 (E–7)	2.5 (E–5)
90.0	2.2 (E–6)	5.8 (E–5)
93.0	2.6 (E–6)	7.0 (E–5)
95.0	3.4 (E–6)	9.1 (E–5)

[a] The figures for uranium include representative values for the activity of the uranium–234 that is concentrated during the enrichment process.

***Authority**: T.C.A. §§4–5–201 et seq. and 68–202–101 et seq.* ***Administrative History:** Original rule filed July 18, 2002; effective October 1, 2002.*

Priority Two Industrial and Educational Radiation Machines: Closed–beam analytical radiation machines, gauges or industrial radiation machines used in shielded room or cabinet radiography.	$ 600.00 per tube

March 25, 2004

Mr. Lawrence E. Nanney, Director
Division of Radiological Health
Department of Environment and Conservation
L & C Anne, Third Floor
401 Church Street
Nashville, TN 37243-1532

Dear Mr. Nanney:

We have reviewed the proposed revisions to the Tennessee "State Regulations for Protection Against Radiation" received by our office on February 19, 2004. Our review and comments are in response to the State's request for Nuclear Regulatory Commission (NRC) evaluation of the changes to Tennessee's radiation control rules that incorporated the amendments identified on the States Regulation Status Data Sheet (SRS). These regulations were reviewed by comparison to the equivalent NRC rules in 10 CFR Parts 20, 30, 31, and 32 and the requirements of the 3 amendments identified in the enclosed SRS. We discussed our review of the regulations with Barbara Davis on March 25, 2004.

As a result of our review, we have two comments and two editorial suggestions. Under our current procedure, a finding that a State regulation meets the compatibility and health and safety categories of the equivalent NRC regulation may only be made based on a review of the final State regulation. Please provide a final amended version of your rules showing the location of any changes made in response to our comments. If there are any comments which Tennessee believes are in error, the State should identify the section of their regulations that meet the designated compatibility category. Please note that we have limited our review to regulations required for compatibility and/or health and safety, but have determined that if these regulations are adopted, incorporating our comments and without significant change, they would meet the compatibility and health and safety categories established in the Office of State and Tribal Programs (STP) Procedure SA-200.

The SRS Data Sheet summarizes our knowledge of the status of other Tennessee regulations, as indicated. Please let us know if you note any inaccuracies, or have any comments on the information contained in the SRS Data Sheet. This letter, including the SRS Data Sheet, is posted on the STP website: http://www.hrsd.ornl.gov/nrc/rulemaking.htm.

If you have any questions regarding the comments, the compatibility and health and safety categories, or any of the NRC regulations used in the review, please contact me, or Dr. Stephen Salomon of my staff at (301) 415-2368 or SNS@NRC.GOV.

Sincerely,

\RA By K.N. Schneider for
Josephine Piccone, Deputy Director
Office of State and Tribal Programs

Enclosures:
As stated

March 25, 2004

Mr. Lawrence E. Nanney, Director
Division of Radiological Health
Department of Environment and Conservation
L & C Anne, Third Floor
401 Church Street
Nashville, TN 37243-1532

Dear Mr. Nanney:

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The SRS Data Sheet summarizes our knowledge of the status of other Tennessee regulations, as indicated. Please let us know if you note any inaccuracies, or have any comments on the information contained in the SRS Data Sheet. This letter, including the SRS Data Sheet, is posted on the STP website: http://www.hrsd.ornl.gov/nrc/rulemaking.htm.

If you have any questions regarding the comments, the compatibility and health and safety categories, or any of the NRC regulations used in the review, please contact me, or Dr. Stephen Salomon of my staff at (301) 415-2368 or SNS@NRC.GOV.

Sincerely,

\RA By K.N. Schneider for
Josephine Piccone, Deputy Director
Office of State and Tribal Programs

Enclosures: As stated
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COMPATIBILITY COMMENTS ON TENNESSEE FINAL REGULATIONS

	STATE SECTION	NRC SECTION	RATS ID	CATEGORY	SUBJECT and COMMENTS
1	1200-2-5-.32(64)	20.1003	2002-1	A	**Definition: "Shallow-dose equivalent (H_s)"** The phase, "averaged over an area of 1 square centimeter," was not deleted. The deletion in the previous definition was one of the purposes of the new regulation. The State needs to delete this phrase in the definition to meet compatibility.
2	None	31.5(c)(13)	2001-1	B	**Certain detecting, measuring, gauging, or controlling devices and certain devices for producing light or an ionized atmosphere** The State omitted paragraph 31.5(c)(13)(F)(iv) concerning 180 day reciprocity. The State needs to include this provision to meet compatibility.

Editorial Suggestions

1. In 1200–2-10-.10(2)(c)13.(iii), paragraph (iii) in the State regulation is mislabeled. It should read (ii).

2. In 1200-2-10-.16(7) on p. 69, The reference to (b) should be (c) to read:1200-2-10-.10(2)(c)14.

STATE REGULATION STATUS

State: TENNESSEE

[Three Amendments reviewed are identified by a ★ at the beginning of each equivalent NRC regulation.]

Tracking Ticket Number: 4-37

Date: MARCH 25, 2004

NRC Chronology Identification	FR Notice (State Due Date)	RATS ID	Proposed (P) / Final (F)[1] Rule / ML #[5]	NRC Review / Y, N[2] / Date / ML #[5]	Final State Regulation[1] (Effective Date)
Standards for Protection Against Radiation-Part 20	56 FR 23360; 56 FR 61352; 57 FR 38588; 57 FR 57877; 58 FR 67657; 59 FR 41641; 60 FR 20183; (1/1/94)	1991-3	F	N 12/18/97	
Safety Requirements for Radiographic Equipment-Part 34	55 FR 843; (1/10/94)	1991-1			
ASNT Certification of Radiographers-Part 34	56 FR 11504; (none)	1991-2			Not required[3]
Notification of Incidents-Parts 20, 30, 31, 34, 39, 40, 70	56 FR 64980; (10/15/94)	1991-4	F	N 12/2/96	12/28/96
Quality Management Program and Misadministrations-Part 35	56 FR 34104; (1/27/95)	1992-1			
Eliminating the Recordkeeping Requirements for Departures from Manufacturer's Instructions-Parts 30,35	57 FR 45566; (none)	1992-2			Not required[3]
Licensing and Radiation Safety Requirements for Irradiators-Part 36	58 FR 7715; (7/1/96)	1993-2			Not applicable SECY-95-112[4]
Definition of Land Disposal and Waste Site QA Program-Part 61	58 FR 33886; (7/22/96)	1993-3	F	N 12/2/96	12/28/96
Decommissioning Recordkeeping and License Termination: Documentation Additions [Restricted areas and spill sites]-Parts 30, 40	58 FR 39628; (10/25/96)	1993-1			1997-Current LBR adequate per MRB
Self-Guarantee as an Additional Financial Mechanism-Parts 30, 40, 70	58 FR 68726; 59 FR 1618 (none)	1994-1	F	N 12/2/96	Not required[3] Alternate rule 1987
Uranium Mill Tailings Regulations: Conforming NRC Requirements to EPA Standards-Part 40	59 FR 28220; (7/1/97)	1994-2			Not applicable to TN
★Timeliness in Decommissioning Material Facilities-Parts 30, 40, 70	59 FR 36026; (8/15/97)	1994-3	P 2/13/04 ML040540516	N 3/25/04 ML040850212	

NRC Chronology Identification	FR Notice (State Due Date)	RATS ID	Proposed (P) / Final (F)[1] Rule / ML #[5]	NRC Review / Y, N[2] / Date / ML #[5]	Final State Regulation[1] (Effective Date)
Preparation, Transfer for Commercial Distribution, and Use of Byproduct Material for Medical Use-Parts 30, 32, 35	59 FR 61767; 59 FR 65243 60 FR 322; (1/1/98)	1995-1	F ML020320229	N 2/5/02 ML020360223	2/4/02
Frequency of Medical Examinations for Use of Respiratory Protection Equipment-Part 20	60 FR 7900; (3/13/98)	1995-2	F ML020320229	N 2/5/02 ML020360223	2/4/02
Low-Level Waste Shipment Manifest Information and Reporting-Parts 20, 61	60 FR 15649; 60 FR 25983 (3/1/98)	1995-3	F ML020320229	N 2/5/02 ML020360223	2/4/02
Performance Requirements for Radiography Equipment-Part 34	60 FR 28323; (6/30/98)	1995-4	F ML020320229	N 2/5/02 ML020360223	2/4/02
Radiation Protection Requirements: Amended Definitions and Criteria-Parts 19, 20	60 FR 36038; (8/14/98)	1995-5	F ML020320229	N 2/5/02 ML020360223	2/4/02
Clarification of Decommissioning Funding Requirements-Parts 30, 40, 70	60 FR 38235; (11/24/98)	1995-6			
Medical Administration of Radiation and Radioactive Materials-Parts 20, 35	60 FR 48623; (10/20/98)	1995-7	F ML020320229	N 2/5/02 ML020360223	2/4/02
10 CFR Part 71: Compatibility with the International Atomic Energy Agency-Part 71	60 FR 50248; 61 FR 28724 (4/1/99)	1996-1	F ML020320229	Y 2/5/02 ML020360223	2/4/02
One Time Extension of Certain Byproduct, Source and Special Nuclear Materials Licenses-Parts 30, 40, 70	61 FR 1109; (none)	1996-2			Not required[3]
Termination or Transfer of Licensed Activities: Recordkeeping Requirements-Parts 20, 30, 40, 61, 70	61 FR 24669; (6/17/99)	1996-3			
Resolution of Dual Regulation of Airborne Effluents of Radioactive Materials; Clean Air Act-Part 20	61 FR 65120; (1/9/00)	1997-1	F ML020320229	N 2/5/02 ML020360223	2/4/02
Fissile Material Shipments and Exemptions-Part 71	62 FR 5907; (none)	1997-4			Not required[3]
Recognition of Agreement State Licenses in Areas Under Exclusive Federal Jurisdiction Within an Agreement State-Part 150	62 FR 1662; (2/27/00)	1997-2	F ML020320229	N 2/5/02 ML020360223	2/4/02

NRC Chronology Identification	FR Notice (State Due Date)	RATS ID	Proposed (P) / Final (F)[1] Rule / ML #[5]	NRC Review / Y, N[2] / Date / ML #[5]	Final State Regulation[1] (Effective Date)
Criteria for the Release of Individuals Administered Radioactive Material-Parts 20, 35	62 FR 4120; (5/29/00)	1997-3	F ML020320229	N 2/5/02 ML020360223	2/4/02
Licenses for Industrial Radiography and Radiation Safety Requirements for Industrial Radiography Operations-Parts 30, 34, 71, 150	62 FR 28947; (6/27/00)	1997-5	F ML020320229	N 2/5/02 ML020360223	2/4/02
Radiological Criteria for License Termination-Parts 20, 30, 40, 70	62 FR 39057; (8/20/00)	1997-6			
Exempt Distribution of a Radioactive Drug Containing One Microcurie of Carbon-14 Urea-Part 30	62 FR 63634; (1/02/01)	1997-7	F ML020320229	N 2/5/02 ML020360223	2/4/02
Deliberate Misconduct by Unlicensed Persons-Parts 30, 40, 61, 70, 71, 150	63 FR 1890; 63 FR 13773 (2/12/01)	1998-1	F ML020320229	N 2/5/02 ML020360223	2/4/02
Self-Guarantee of Decommissioning Funding by Nonprofit and Non-Bond-Issuing Licensees-Parts 30, 40, 70	63 FR 29535; (none)	1998-2			Not required[3]
License Term for Medical Use Licenses-Part 35	63 FR 31604; (none)	1998-3			Not required[3]
Licenses for Industrial Radiography and Radiation Safety Requirements for Industrial Radiographic Operations-Part 34	63 FR 37059; (7/9/01)	1998-4	F ML020320229	Y 2/5/02 ML020360223	2/4/02
Minor Corrections, Clarifying Changes, and a Minor Policy Change-Parts 20, 35, 36	63 FR 39477; 63 FR 45393 (10/26/01)	1998-5	P 10/17/03 ML033010588	Y 12/01/03 ML033360370	
Transfer for Disposal and Manifests: Minor Technical Conforming Amendment-Part 20	63 FR 50127; (11/20/01)	1998-6	P 10/17/03 ML033010588	N 12/01/03 ML033360370	
Radiological Criteria for License Termination of Uranium Recovery Facilities-Part 40	64 FR 17506; (6/11/02)	1999-1			Not applicable to TN
Requirements for Those Who Possess Certain Industrial Devices Containing Byproduct Material to Provide Requested Information-Part 31	64 FR 42269; (none)	1999-2			Not required[3]

NRC Chronology Identification	FR Notice (State Due Date)	RATS ID	Proposed (P) / Final (F)[1] Rule / ML #[5]	NRC Review / Y, N[2] / Date / ML #[5]	Final State Regulation[1] (Effective Date)
Respiratory Protection and Controls to Restrict Internal Exposure-Part 20	64 FR 54543; 64 FR 55524 (2/2/03)	1999-3	P 10/17/03 ML033010588	Y 12/01/03 ML033360370	
Energy Compensation Sources for Well Logging and Other Regulatory Clarifications-Part 39	65 FR 20337; (5/17/03)	2000-1	P 10/17/03 ML033010588	N 12/01/03 ML033360370	
New Dosimetry Technology-Parts 34, 36, 39	65 FR 63750; (1/8/04)	2000-2	P 10/17/03 ML033010588	N 12/01/03 ML033360370	
★Requirements for Certain Generally Licensed Industrial Devices Containing Byproduct Material - Parts 30, 31, 32	65 FR 79162; (2/16/04)	2001-1	P 2/13/04 ML040540516	Y 03/25/04 ML040850212	
★Revision of the Skin Dose Limit - Part 20	67 FR 16298; (4/5/05)	2002-1	P 2/13/04 ML040540516	Y 03/25/04 ML040850212	
Medical Use of Byproduct Material - Parts 20, 32, and 35	67 FR 20249; (4/24/05)	2002-2			
Financial Assurance for Materials Licensees – Parts 30, 40, 70	68 FR 57327 12/3/06	2003-1			

1. Or other generic Legally Binding Requirements.

2. (Y/N) Y means "Yes," there are comments in the review letter that the State needs to address. N means "No," there are no comments in the review letter.

3. Not required means these regulations are not required for purposes of compatibility.

4. A State need not adopt a specific regulation if the State has no licensees that would be subject to that regulation. See: "Final Policy Statement on Adequacy and Compatibility of Agreement State Programs," III.1. Time Frame for Adoption of Compatible State Regulations, p. 6, SECY-95-112, May 3, 1995.

5. ADAMS ML Number

August 31, 2006

Mr. Lawrence E. Nanney, Director
Division of Radiological Health
Department of Environment and Conservation
L & C Annex, Third Floor
401 Church Street
Nashville, TN 37243-1532

Dear Mr. Nanney:

By letter dated March 25, 2004, we sent you the results of our review of the proposed changes to the Tennessee regulations which were submitted to the NRC by letter dated February 13, 2004. These regulations were reviewed by comparison to the equivalent Nuclear Regulatory Commission (NRC) rules in 10 CFR Parts 30, 31, and 32, and the requirements of the General License (GL) amendment identified in the enclosed State Regulation Status (SRS) Data Sheet. The review we conducted omitted a specific review of Sections 10 CFR Parts 31.5 and 31.6. The text of these sections was not revised by the GL amendment, but the compatibility designations were changed from Compatibility Categories C and D respectively, to Category B.

As noted in All Agreement States letter STP-05-072, dated September 28, 2005, NRC staff will continue to review Agreement State GL Device proposed and final rules but will hold in abeyance any determination on those rules and compatibility of those rules, if the Agreement State has the essential elements of the NRC's rule and is more restrictive than the NRC's rule. For Agreement States without a GL Device rule or a GL Device rule less restrictive than the NRC rules, the staff will factor this determination into results of the NRC's review of the State's proposed and final rules and the compatibility findings during Integrated Materials Performance Evaluation Program (IMPEP) reviews.

We discussed our review of the GL Amendment with Beth Murphy on August 21, 2006. Although we had only one comment for the 2001-1 amendment stated in the March 25, 2004 letter, as a result of our re-review against the Compatibility Category B designation, we have three comments that have been identified in the enclosure. Comment 1 indicates that the State's regulation is more restrictive than the NRC's regulations, and there is no corrective action required for this comment at this point. However, Comments 2 and 3 indicate that the State's regulations are less restrictive than the NRC's regulations, and they need to be addressed in the State's future rulemaking process. We request that when a final version of Tennessee regulations is adopted and published in response to our comments, that a copy of the "as published" regulations be provided to us for review as requested in STP Procedure SA-201, Review of State Regulatory Requirements.

Please note that we have limited our review to regulations required for compatibility and/or health and safety. The SRS Data Sheet summarizes our knowledge of the status of other Tennessee regulations, as indicated. Please let us know if you note any inaccuracies, or have any comments on the information contained in the SRS Data Sheet. This letter, including the SRS Data Sheet, is posted on the STP website: http://www.hsrd.ornl.gov/nrc/rulemaking.htm.

If you have any questions regarding the comments, the compatibility and health and safety categories, or any of the NRC regulations used in the review, please contact me, or Ms. Monica Orendi at (301) 415-3938 or by e-mail at MLO1@nrc.gov.

Sincerely,

Dennis K. Rathbun, Deputy Director
Office of State and Tribal Programs

Enclosures:
As stated

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Sincerely,

\RA By KNSchneider For

Dennis K. Rathbun, Deputy Director
Office of State and Tribal Programs

Enclosures:
As stated

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COMMENTS ON PROPOSED TENNESSEE REGULATIONS AGAINST COMPATIBILITY AND HEALTH AND SAFETY CATEGORIES

	STATE SECTION	NRC SECTION	RATS ID	CATEGORY	SUBJECT and COMMENTS
1	1200-2-10-.10(2)(c)13	31.5(c)(13)(i) and (iv)	2001-1	B	**Certain detecting, measuring, gauging, or controlling devices and certain devices for producing light or an ionized atmosphere** Tennessee's regulation requires registration of all the GL devices, while 10 CFR 31.5(c)(13)(i) only require registration of certain devices. Also, Tennessee's regulation does not have registration exemption of devices that are used for less than 180 days from registration, while 10 CFR 31.5(c)(13)(iv) does. Tennessee's rules have the essential elements of the NRC, but are more restrictive than the NRC's GL rule. As noted in the September 28, 2005 All Agreement States Letter STP-05-072, the determination on this provision will be held in abeyance until such time that the NRC completes its review and response to the Organization of Agreement State petition and State of Florida's request on compatibility changes for the GL rule.

STATE SECTION		NRC SECTION	RATS ID	CATEGORY	SUBJECT and COMMENTS
2	1200-2-10-.10(2)(c)13	31.5(c)(13)(ii) and (iii)	2001-1	B	**Certain detecting, measuring, gauging, or controlling devices and certain devices for producing light or an ionized atmosphere** Tennessee has omitted the annually registration requirement of these devices stated in 10 CFR 31.5(c)(13)(ii). The reference to 10 CFR 31.5(c)(12) in 1200-2-10-.10(2)(c)13.(iii)(III) should be under part (c)(12), instead of (b)(12). Tennessee needs to add the requirements of 10 CFR 31.5(c)(13)(ii) to 1200-2-10-.10(2)(c)13 and correct the reference in 1200-2-10-.10(2)(c)13.(iii)(III) to meet the Compatibility Category B designation assigned to 10 CFR 31.5(c)(13).
3	1200-2-10-.10(2)(e)	31.5(d)	2001-1	B	**Certain detecting, measuring, gauging, or controlling devices and certain devices for producing light or an ionized atmosphere** Tennessee has omitted the phrase "or import" after the phrase "authorize the manufacture" in 1200-2-10-.10(2)(e). Tennessee needs to add the phrase to meet the Compatibility Category B designation assigned to 10 CFR 31.5(d).

STATE REGULATION STATUS

State: TENNESSEE
[1 Amendment reviewed identified by a ★ at the beginning of the equivalent NRC requirement.]

Tracking Ticket Number: 6-16
Date: August 31, 2006

NRC Chronology Identification	FR Notice (State Due Date)	RATS ID	Proposed (P) / Final (F)[1] Rule / License Condition (LC) ML #[5]	NRC Review / Y, N[2] / Date / ML #[5]	Final State Regulation[1] (Effective Date)
Safety Requirements for Radiographic Equipment-Part 34	55 FR 843; (1/10/94)	1991-1			Superceded by 97-5
ASNT Certification of Radiographers-Part 34	56 FR 11504; (none)	1991-2			Not required[3]
Standards for Protection Against Radiation-Part 20	56 FR 23360; 56 FR 61352; 57 FR 38588; 57 FR 57877; 58 FR 67657; 59 FR 41641; 60 FR 20183; (1/1/94)	1991-3	F F ML062130090	N 12/8/97 N 08/ /06 ML	
Notification of Incidents-Parts 20, 30, 31, 34, 39, 40, 70	56 FR 64980; (10/15/94)	1991-4	F	N 12/2/96	12/28/96
Quality Management Program and Misadministrations-Part 35	56 FR 34104; (1/27/95)	1992-1			Superceded by 2002-2
Eliminating the Recordkeeping Requirements for Departures from Manufacturer's Instructions-Parts 30,35	57 FR 45566; (none)	1992-2			Not required[3]
Licensing and Radiation Safety Requirements for Irradiators-Part 36	58 FR 7715; (7/1/96)	1993-2			Not applicable SECY-95-112[4]
Definition of Land Disposal and Waste Site QA Program-Part 61	58 FR 33886; (7/22/96)	1993-3	F	N 12/2/96	12/28/96
Decommissioning Recordkeeping and License Termination: Documentation Additions [Restricted areas and spill sites]-Parts 30, 40	58 FR 39628; (10/25/96)	1993-1			1997-Current LBR adequate per MRB
Self-Guarantee as an Additional Financial Mechanism-Parts 30, 40, 70	58 FR 68726; 59 FR 1618 (none)	1994-1			Not required[3]
Uranium Mill Tailings Regulations: Conforming NRC Requirements to EPA Standards-Part 40	59 FR 28220; (7/1/97)	1994-2			Not applicable to TN

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Timeliness in Decommissioning Material Facilities-Parts 30, 40, 70	59 FR 36026; (8/15/97)	1994-3	P 2/13/04 ML040540516	N 3/25/04 ML040850212	
Preparation, Transfer for Commercial Distribution, and Use of Byproduct Material for Medical Use-Parts 30, 32, 35	59 FR 61767; 59 FR 65243 60 FR 322; (1/1/98)	1995-1	F ML020320229	N 2/1/02 ML020360223	10/02/02
Frequency of Medical Examinations for Use of Respiratory Protection Equipment-Part 20	60 FR 7900; (3/13/98)	1995-2	F ML020320229	N 2/1/02 ML020360223	10/02/02
Low-Level Waste Shipment Manifest Information and Reporting-Parts 20, 61	60 FR 15649; 60 FR 25983 (3/1/98)	1995-3	F ML020320229	N 2/1/02 ML020360223	10/02/02
Performance Requirements for Radiography Equipment-Part 34	60 FR 28323; (6/30/98)	1995-4	F ML020320229	N 2/1/02 ML020360223	10/02/02
Radiation Protection Requirements: Amended Definitions and Criteria-Parts 19, 20	60 FR 36038; (8/14/98)	1995-5	F ML020320229	N 2/1/02 ML020360223	10/02/02
Clarification of Decommissioning Funding Requirements-Parts 30, 40, 70	60 FR 38235; (11/24/98)	1995-6			Not required[3]
Medical Administration of Radiation and Radioactive Materials-Parts 20, 35	60 FR 48623; (10/20/98)	1995-7	F ML040980538	N 5/06/04 ML041270512	10/02/02
10 CFR Part 71: Compatibility with the International Atomic Energy Agency-Part 71	60 FR 50248; 61 FR 28724 (4/1/99)	1996-1	F ML020320229	N 2/1/02 ML020360223	10/02/02
One Time Extension of Certain Byproduct, Source and Special Nuclear Materials Licenses-Parts 30, 40, 70	61 FR 1109; (none)	1996-2			Not required[3]
Termination or Transfer of Licensed Activities: Recordkeeping Requirements-Parts 20, 30, 40, 61, 70	61 FR 24669; (6/17/99)	1996-3			
Resolution of Dual Regulation of Airborne Effluents of Radioactive Materials; Clean Air Act-Part 20	61 FR 65120; (1/9/00)	1997-1	F ML020320229	N 2/1/02 ML020360223	10/02/02

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Recognition of Agreement State Licenses in Areas Under Exclusive Federal Jurisdiction Within an Agreement State-Part 150	62 FR 1662; (2/27/00)	1997-2	F ML020320229	N 2/1/02 ML020360223	10/02/02
Criteria for the Release of Individuals Administered Radioactive Material-Parts 20, 35	62 FR 4120; (5/29/00)	1997-3	F ML020320229	N 2/1/02 ML020360223	10/02/02
Fissile Material Shipments and Exemptions-Part 71	62 FR 5907; (none)	1997-4			Not required[3]
Licenses for Industrial Radiography and Radiation Safety Requirements for Industrial Radiography Operations-Parts 30, 34, 71, 150	62 FR 28947; (6/27/00)	1997-5	F ML020320229	N 2/1/02 ML020360223	10/02/02
Radiological Criteria for License Termination-Parts 20, 30, 40, 70	62 FR 39057; (8/20/00)	1997-6	F ML040980538	Y 5/06/04 ML041270512	10/02/02
Exempt Distribution of a Radioactive Drug Containing One Microcurie of Carbon-14 Urea-Part 30	62 FR 63634; (1/02/01)	1997-7	F ML020320229	N 2/1/02 ML020360223	10/02/02
Deliberate Misconduct by Unlicensed Persons-Parts 30, 40, 61, 70, 71, 150	63 FR 1890; 63 FR 13773 (2/12/01)	1998-1	F ML020320229	N 2/1/02 ML020360223	10/02/02
Self-Guarantee of Decommissioning Funding by Nonprofit and Non-Bond-Issuing Licensees- Parts 30, 40, 70	63 FR 29535; (none)	1998-2			Not required[3]
License Term for Medical Use Licenses-Part 35	63 FR 31604; (none)	1998-3			Not required[3]
Licenses for Industrial Radiography and Radiation Safety Requirements for Industrial Radiographic Operations-Part 34	63 FR 37059; (7/9/01)	1998-4	F ML040980538	N 5/06/04 ML041270512	10/02/02
Minor Corrections, Clarifying Changes, and a Minor Policy Change-Parts 20, 35, 36	63 FR 39477; 63 FR 45393 (10/26/01)	1998-5	F ML062130094	N 8/ /06 ML	
Transfer for Disposal and Manifests: Minor Technical Conforming Amendment-Part 20	63 FR 50127; (11/20/01)	1998-6	F ML062130094	N 8/ /06 ML	

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Radiological Criteria for License Termination of Uranium Recovery Facilities-Part 40	64 FR 17506; (6/11/02)	1999-1			Not applicable to TN
Requirements for Those Who Possess Certain Industrial Devices Containing Byproduct Material to Provide Requested Information-Part 31	64 FR 42269; (none)	1999-2			Not required[3]
Respiratory Protection and Controls to Restrict Internal Exposure-Part 20	64 FR 54543; 64 FR 55524 (2/2/03)	1999-3	F ML062130094	N 8/ /06 ML	
Energy Compensation Sources for Well Logging and Other Regulatory Clarifications-Part 39	65 FR 20337; (5/17/03)	2000-1	F ML062130094	N 8/ /06 ML	
New Dosimetry Technology-Parts 34, 36, 39	65 FR 63750; (1/8/04)	2000-2	F ML062130094	N 8/ /06 ML	
★Requirements for Certain Generally Licensed Industrial Devices Containing Byproduct Material - Parts 30, 31, 32	65 FR 79162; (2/16/04)	2001-1	P[7] 2/13/04 ML040540516	Y 8/31/06 ML062410102	
Revision of the Skin Dose Limit - Part 20	67 FR 16298; (4/5/05)	2002-1	P 2/13/04 ML040540516	Y 03/25/04 ML040850212	
Medical Use of Byproduct Material - Parts 20, 32, and 35	67 FR 20249; (4/24/05)	2002-2			
Financial Assurance for Materials Licensees – Parts 30, 40, 70	68 FR 57327 12/3/06	2003-1			
Compatibility With IAEA Transportation Safety Standards and Other Transportation Safety Amendments – Part 71	69 FR 3697; (10/01/07)	2004-1			
Security Requirements for Portable Gauges Containing Byprodcut Material - Part 30	70 FR 2001; (7/11/08)	2005-1			

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Medical Use of Byproduct Material - Recognition of Speciality Boards - Part 35	70 FR 16336; 71 FR 1926 (4/29/08)	2005-2			
Increased Controls for Risk-Significant Radioactive Sources (NRC Order EA-05-090)[6]	70 FR 72128; (12/1/05)	2005-3	LC ML052590043	N 9/19/2005 ML052630474	
Minor Amendments-Parts 20, 30, 32, 35, 40 and 70	71 FR 15005 (3/27/09)	2006-1			

1. Or other generic Legally Binding Requirements.

2. (Y/N) Y means "Yes," there are comments in the review letter that the State needs to address. N means "No," there are no comments in the review letter.

3. Not required means these regulations are not required for purposes of compatibility.

4. A State need not adopt a specific regulation if the State has no licensees that would be subject to that regulation. See: "Final Policy Statement on Adequacy and Compatibility of Agreement State Programs," III.1. Time Frame for Adoption of Compatible State Regulations, p. 6, SECY-95-112, May 3, 1995.

5. ADAMS ML Number.

6. By letter dated September 2, 2005, from Paul H. Lohaus, Director, Office of State and Tribal Programs, Agreement States were given 90 days to issue legally binding requirements satisfying the requirements of NRC Order EA-05-090.

7. A re-review was completed on this item due to the change in compatibility category of 31.5 and 31.6. More information can be found in the All Agreement States letter STP 05-072