Facility	Cooper		Date	e of E	xam:	Octob	oer 200	05										
Tier	Group					RO	K/A C	atego	ory Po	oints					SI	RO-On	ly Poin	ts
Contra Maria		К 1	К 2	K 3	К 4	К 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A	2	C	*	Total
1.	I - Martin	2	3	5				3	3			4	20		1	-	3	7
Emergency a Abnormal Pla	ant 2	1	1	1		N/A		1	1	N	/A	2	7		2			3
Evolutions	Tier Totals	3	4	6				4	4			6	27		5		4	10
2. Blant Sector	1	2	2	2	3	2	3	3	2	2	3	2	26		2		3	5
Plant Syster	2	2	1	1	1	1	1	1	1	1	1	l	12		l		2	3
	Tier Totals	4	3	3	4	3	4	4	3	3	4	3	38		3		5	8
3. Generic	Knowledge and Abilit	ies Cate	egoric	:S		1	2	2		3 3	4	۱ ۲	10	1	2	3	4	7
	outlines (i.e., than two).	except	for o	ne cat	egory	very a in Tie	applica er 3 of	able K f the S	ζ/A c SRO-	ategoi only c	ry are s outline	, the '	ed within 'Tier Total	each tie s" in ea	r of the ch K/A	e RO ar catego	ory shal	l not be less
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ES-401		В	WR I	Exam	inatio	n Out	line Form ES-401-1			
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR.	#	~
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	х						AK1.01 Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION : Natural Circulation (CFR: 41.8 to 41.10)	2.5	1	
295003 Partial or Complete Loss of AC / 6			х				AK3.05 Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER : Reactor SCRAM (CFR: 41.5 / 45.6)	3.7	2	
295004 Partial or Total Loss of DC Pwr / 6			x				AK3.03 Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER : Reactor SCRAM: Plant-Specific (CFR: 41.5 / 45.6)	3.1	3	
295005 Main Turbine Generator Trip / 3						x	2.4.49 Ability to perform without reference to procedures those actions that require immediate operation of system components and controls. (CFR: $41.10 / 43.2 / 45.6$)	4.0	4	
295006 SCRAM / 1					x		AA2.05 Ability to determine and/or interpret the following as they apply to SCRAM :Whether a reactor SCRAM has occurred. (CFR: 41.10 / 43.5 / 45.13)	4.6	5	
295016 Control Room Abandonment / 7			x				AK3.01 Knowledge of the reasons for the following responses as they apply to CONTROL ROOM ABANDONMENT : Reactor SCRAM (CFR: 41.5 / 45.6)	4.1	6	
295018 Partial or Total Loss of CCW / 8			х				AK3.04 Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER :Starting standby pump (CFR: 41.5 / 45.6)	3.3	7	
295019 Partial or Total Loss of Inst. Air / 8						x	2.4.4 Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures. (CFR: 41.10 / 43.2 / 45.6)4.3	4.0	8	
295021 Loss of Shutdown Cooling / 4				x			AA1.05 Ability to operate and/or monitor the following as they apply to LOSS OF SHUTDOWN COOLING :Reactor recirculation (CFR: 41.7 / 45.6)	3.1	9	
295023 Refueling Acc / 8		x					AK2.03 Knowledge of the interrelations between REFUELING ACCIDENTS and the following: Radiation monitoring equipment.	3.4	10	
295024 High Drywell Pressure / 5		x					EK2.13 Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following: Suppression pool spray: Plant-Specific (CFR: 41.7 / 45.8)	3.8	11	
295025 High Reactor Pressure / 3					x		EA2.04 Ability to determine and/or interpret the following as they apply to HIGH REACTOR PRESSURE:Suppression pool level (CFR: 41.10 / 43.5 / 45.13)	3.9	12	
295026 Suppression Pool High Water Temp. / 5			x				EK3.01 Knowledge of the reasons for the following responses as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:Emergency/normal depressurization. (CFR: 41.5 / 45.6)	3.8	13	

ES-401 BWR Examination Outline Form ES-401-1 Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO)(Continued)												
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#	r		
295027 High Containment Temperature / 5	See	NOT	TE 1		_							
295028 High Drywell Temperature / 5	x						EK1.01 Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL TEMPERATURE : Reactor water level measurement (CFR: 41.8 to 41.10)	3.5	14			
295030 Low Suppression Pool Wtr Lvl / 5					x		EA2.02Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL :Suppression pool temperature (CFR: 41.10 / 43.5 / 45.13)	3.9	15			
295030 Low Suppression Pool Wtr Lvl / 5				х			EA1.06 Ability to operate and/or monitor the following as they apply to LOW SUPPRESSION POOL WATER LEVEL: Condensate storage and transfer (make-up to the suppression pool): Plant-Specific (CFR: 41.7 / 45.6)	3.4	16			
295031 Reactor Low Water Level / 2						X	2.1.30 Ability to locate and operate components / including local controls. (CFR: 41.7 / 45.7)	3.9	17			
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1					-	х	2.4.31 Knowledge of annunciators alarms and indications / and use of the response instructions. (CFR: 41.10 / 45.3)	3.3	18			
295038 High Off-site Release Rate / 9				х			EA1.06 Ability to operate and/or monitor the following as they apply to HIGH OFF-SITE RELEASE RATE : Plant ventilation. (CFR: 41.7 / 45.6)	3.5	19			
600000 Plant Fire On Site / 8		х					AK2.04 Knowledge of the interrelations between PLANT FIRE ON SITE and the following:Breakers / relays / and disconnects	2.5	20			
K/A Category Totals:	2	3	5	3	3	4	Group Point Total		20			

ES-401 Emergency and Abn	Iorma	3WF al Pl	R Ex ant	amin Evoli	ation utions	Out s - Ti	line Form ES-401-1 ier 1/Group 1 (SRO)		
E/APE # / Name / Safety Function	K 1	К 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4									
295003 Partial or Complete Loss of AC / 6					х		AA2.04 Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER : System lineups (CFR: 41.10/43.5/45.13)	3.7	
295004 Partial or Total Loss of DC Pwr / 6						x	2.4.30 Knowledge of which events related to system operations/status should be reported to outside agencies. (CFR: 43.5 / 45.11)	3.6	
295005 Main Turbine Generator Trip / 3					х		AA2.06. Ability to determine and/or interpret the following as they apply to MAIN TURBINE GENERATOR TRIP : Feedwater temperature (CFR: 41.10 / 43.5 / 45.13)	2.7	
295006 SCRAM / 1									
295016 Control Room Abandonment / 7									
295018 Partial or Total Loss of CCW / 8									
295019 Partial or Total Loss of Inst. Air / 8					Х		AA2.02 Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR : Status of safety-related instrument air system loads (see AK2.1 - AK2.19) (CFR: 41.10 / 43.5 / 45.13)	3.7	
295021 Loss of Shutdown Cooling / 4									
295023 Refueling Acc / 8					х		AA2.05 Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS : Entry conditions of emergency plan (CFR: 41.10 / 43.5 / 45.13)	4.6	
295024 High Drywell Pressure / 5						x	2.1.32 Ability to explain and apply system limits and precautions. (CFR: 41.10 / 43.2 / 45.12)	3.8	
295025 High Reactor Pressure / 3									
295026 Suppression Pool High Water Temp. / 5									
295027 High Containment Temperature / 5	S	ee N	ote	1					
295028 High Drywell Temperature / 5						x	2.4.6 Knowledge symptom based EOP mitigation strategies. (CFR: 41.10 / 43.5 / 45.13)	4.0	
295030 Low Suppression Pool Wtr Lvl / 5									
295031 Reactor Low Water Level / 2									
295037 SCRAM Condition Present and Power Above APRM Downscale									
295038 High Off-site Release Rate / 9									
600000 Plant Fire On Site / 8									
K/A Category Totals:					4	3	Group Point Total		7

ES-401 Emergency as	ıd Ab	E	3WR al Plai	Exam 1t Evo	inatio olutior	n Out 1s - T	line Form ES-401-1 ier 1/Group 2 (RO)		
E/APE # / Name / Safety Function	к 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3						х	2.1.14 Knowledge of system status criteria which require the notification of plant personnel. (CFR: 43.5 / 45.12)	2.5	21
295007 High Reactor Pressure / 3									
295008 High Reactor Water Level / 2									
295009 Low Reactor Water Level / 2		x					AK2.03 Knowledge of the interrelations between LOW REACTOR WATER LEVEL and the following: Recirculation system. (CFR: 41.7 / 45.8)	3.1	22
295010 High Drywell Pressure / 5	X						AK1.01 Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL PRESSURE : Downcomer submergence: Mark-I&II (CFR: 41.8 to 41.10)	3.0	23
295011 High Containment Temp / 5	See	Note	1						
295012 High Drywell Temperature / 5									
295013 High Suppression Pool Temp. / 5									
295014 Inadvertent Reactivity Addition / 1			x				AK3.01 Knowledge of the reasons for the following responses as they apply to INADVERTENT REACTIVITY ADDITION: Reactor SCRAM (CFR: 41.5 / 45.6)	4.1	24
295015 Incomplete SCRAM / 1									
295017 High Off-site Release Rate / 9									
295020 Inadvertent Cont. Isolation / 5 & 7						х	2.4.50 Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.(CFR: 45.3)	3.3	25
295022 Loss of CRD Pumps / 1									
295029 High Suppression Pool Wtr Lvl / 5					х		EA2.03 Ability to determine and/or interpret the following as they apply to HIGH SUPPRESSION POOL WATER LEVEL :Drywell/containment water level (CFR: 41.10 / 43.5 / 45.13)	3.4	26
295032 High Secondary Containment Area Temperature / 5									
295033 High Secondary Containment Area Radiation Levels / 9									
295034 Secondary Containment Ventilation High Radiation / 9									

ES-401 Emergency at	line Form ES-401-1 ier 1/Group 2 (RO)(continued)								
E/APE # / Name / Safety Function	K 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295035 Secondary Containment High Differential Pressure / 5				x			EA1. Ability to operate and/or monitor the following as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE: SBGT/FRVS (CFR: 41.7 / 45.6)	3.8	27
295036 Secondary Containment High Sump/Area Water Level / 5									
500000 High CTMT Hydrogen Conc. / 5									
K/A Category Totals:	1	1	1	1	1	2	Group Point Total		7

ES-401 Emergency and Ab	E norma	3WR 1 Plat	Exam nt Evo	inatio olution	n Out 1s - Ti	tline ier 1/C	Form ES-401-1 Group 2 (SRO)		
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									
295007 High Reactor Pressure / 3									
295008 High Reactor Water Level / 2									
295009 Low Reactor Water Level / 2									
295010 High Drywell Pressure / 5									
295011-High Containment Temp / 5	See	Note	1						
295012 High Drywell Temperature / 5									
295013 High Suppression Pool Temp. / 5									
295014 Inadvertent Reactivity Addition / 1					x		AA2.05 Ability to determine and/or interpret the following as they apply to INADVERTENT REACTIVITY ADDITION : †Violation of safety limits (CFR: 41.10 / 43.5 / 45.13)	4.6	
295015 Incomplete SCRAM / 1						x	2.4.6 Knowledge symptom based EOP mitigation strategies. (CFR: 41.10 / 43.5 / 45.13)	4.0	
295017 High Off-site Release Rate / 9									
295020 Inadvertent Cont. Isolation / 5 & 7									
295022 Loss of CRD Pumps / 1									
295029 High Suppression Pool Wtr Lvl / 5									
295032 High Secondary Containment Area Temperature / 5									
295033 High Secondary Containment Area Radiation Levels / 9									
295034 Secondary Containment Ventilation High Radiation / 9					x		EA2.02 Ability to determine and/or interpret the following as they apply to SECONDARY CONTAINMENT VENTILATION HIGH RADIATION : Cause of high radiation levels (CFR: 41.10 / 43.5 / 45.13)	4.2	
295035 Secondary Containment High Differential Pressure / 5									
295036 Secondary Containment High Sump/Area Water Level / 5									
500000 High CTMT Hydrogen Conc. / 5									
K/A Category Totals:					2	1	Group Point Total		3

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System#/Name	К 1	К 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection Mode							x					A1.08Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) controls including: †Emergency generator loading (CFR: 41.5 / 45.5)	3.7	28
205000 Shutdown Cooling			х									K3.01 Knowledge of the effect that a loss or malfunction of the SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) will have on following: Reactor pressure (CFR: 41.7 / 45.4)	3.3	29
206000 HPCI											х	2.1.23 Ability to perform specific system and integrated plant procedures during different modes of plant operation. (CFR: 45.2 / 45.6)	3.9	30
206000 HPCI		x										K2.02 Knowledge of electrical power supplies to the following: System pumps: BWR-2,3,4 (CFR: 41.7)	2.8	31
2 07000 Isolation (Emergency)Condenser	See	Note	2											
209001 LPCS										х		A4.11 Ability to manually operate and/or monitor in the control room: System flow (CFR: 41.7 / 45.5 to 45.8)	3.7	32
209002 HPCS	See	Note	3					.	r		1			
211000 SLC									x			A3.01 Ability to monitor automatic operations of the STANDBY LIQUID CONTROL SYSTEM including: Pump discharge pressure: Plant-Specific (CFR: 41.7 / 45.7)	3.5	33
212000 RPS		-		х								K4.03 Knowledge of REACTOR PROTECTION SYSTEM design feature(s) and/or interlocks which provide for the following: The prevention of supplying power to a given RPS bus from multiple sources simultaneously (CFR: 41.7)	3.0	34
215003 IRM							x					A1. 04 Ability to predict and/or monitor changes in parameters associated with operating the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM controls including: Control rod block status (CFR: 41.5 / 45.5)	3.4	35
215003 IRM					x							K5.01 Knowledge of the operational implications of the following concepts as they apply to INTERMEDIATE RANGE MONITOR (IRM) SYSTEM : Detector operation (CFR: 41.5 / 45.3)	2.6	36
215004 Source Range Monitor										X		A4.05 Ability to manually operate and/or monitor in the control room: SRM back panel switches, meters, and indicating lights (CFR: 41.7 / 45.5 to 45.8)	3.1	37

ES-401				P	'lant S	BWF ysten	R Exa 1s - Ti	minat ier 2/0	ion O Group	utline 1 (R	0)(Ca	Form ES-4	01-1	
System#/Name	K 1	К 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
215005 APRM / LPRM				х								K4.02 Knowledge of AVERAGE POWER RANGE MONITOR/LOCAL POWER RANGE MONITOR SYSTEM design feature(s) and/or interlocks which provide for the following:Reactor SCRAM signals (CFR: 41.7)	4.1	38
217000 RCIC						x						K6.01 Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) : Electrical power (CFR: 41.7 / 45.7)	3.4	39
218000 ADS				х								K4.03 Knowledge of AUTOMATIC DEPRESSURIZATION SYSTEM design feature(s) and/or interlocks which provide for the following: ADS logic control (CFR: 41.7)	3.8	40
223002 PCIS/Nuclear SteamSupply Shutoff							Х					A1.01 Ability to predict and/or monitor changes in parameters associated with operating the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF controls including: System indicating lights and alarms (CFR: 41.5 / 45.5)	3.5	41
239002 SRVs	X											K1.07 Knowledge of the physical connections and/or cause effect relationships between RELIEF/SAFETY VALVES and the following: Suppression pool (CFR: 41.2 to 41.9 / 45.7 to 45.8)	3.6	42
239002 SRVs		x										K2.01 Knowledge of electrical power supplies to the following: SRV solenoids (CFR: 41.7)	2.8	43
259002 Reactor Water LevelControl						X						K6.03 Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR WATER LEVEL CONTROL SYSTEM : Main steam flow input (CFR: 41.7 / 45.7)	3.1	44
261000 SGTS						х						K6.01 Knowledge of the effect that a loss or malfunction of the following will have on the STANDBY GAS TREATMENT SYSTEM : A.C. electrical distribution (CFR: 41.7 / 45.7)	2.9	45
262001 AC Electrical Distribution											x	2.4.4 Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures. (CFR: 41.10 / 43.2 / 45.6)	4.0	46
262001 AC Electrical Distribution									X			A3.02 Ability to monitor automatic operations of the A.C. ELECTRICAL DISTRIBUTION including: Automatic bus transfer (CFR: 41.7 / 45.7)	3.2	47

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System#/Name	K 1	K 2	К 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
262002 UPS (AC/DC)								х				A2.01 Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of: (CFR: 41.5 / 45.6) Under voltage (2.6/2.8)	2.6	48
263000 DC ElectricalDistribution								х				A2.02 Ability to (a) predict the impacts of the following on the D.C. ELECTRICAL DISTRIBUTION ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of ventilation during charging (CFR: 41.5 / 45.6)	2.6	49
264000 EDGs					х							K5.06 Knowledge of the operational implications of the following concepts as they apply to EMERGENCY GENERATORS (DIESEL/JET) : Load sequencing (CFR: 41.5 / 45.3)	3.4	50
300000 Instrument Air	x											K1.04 Knowledge of the connections and / or cause effect relationships between INSTRUMENT AIR SYSTEM and the following: Cooling water to compressor (CFR: 41.2 to 41.9 / 45.7 to 45.8)	2.8	51
400000 Component CoolingWater			X									K3.01 Knowledge of the effect that a loss or malfunction of the CCWS will have on the following: (CFR: 41.7 / 45.6) Loads cooled by CCWS	2.9	52
400000 Component CoolingWater										x		A4.01 Ability to manually operate and / or monitor in the control room: (CFR: 41.7 / 45.5 to 45.8) A4.01 CCW indications and control	3.1	53
K/A Category Point Totals:	2	2	2	3	2	3	3	2	2	3	2			26

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Record of Rejected K/As

Tier/ Group	Randomly Selected K/A	Reason for Rejection
RO 1/1	295005 G2.1.23	Generic K/A without applicability to the evolution.
RO 1/1	295015 G2.1.27	Generic K/A without applicability to the evolution.
RO 1/1	295037 G2.4.30	Low RO importance (<2.5) with no related plant priority.
RO 2/1	215005 K4.03	Low RO importance (<2.5) with no related plant priority.
RO 2/1	217000 K6.02	Low RO importance (<2.5) with no related plant priority.
RO 2/1	400000 K5.01	Low RO importance (<2.5) with no related plant priority
SRO 2/1	203000 A2.15	Loop Select Logic not used at CNS.
SRO 2/1	206000 G2.1.28	No associated plant SRO task and no link to a 10 CFR 55.43 topic for 206000 G2.1.28.
SRO2/1	211000 G2.1.30	No associated plant SRO task and no link to a 10 CFR 55.43 topic for 211000 G2.1.30.
RO 2/2	233000 A3.01	Low RO importance (<2.5) with no related plant priority.
RO 2/2	241000 K3.01	Low RO importance (<2.5) with no related plant priority.
RO 2/2	286000 K1.02	Low RO importance (<2.5) with no related plant priority.
SRO 2/2	259001 G2.1.28	No associated plant SRO task and no link to a 10 CFR 55.43 topic for 259001 G2.1.28.
RO 3	2.1.4	Low RO importance (<2.5) with no related plant priority.
RO 3	2.1.13	Low RO importance (<2.5) with no related plant priority.
RO 3	2.2.3	Cooper Nuclear Station is a single unit facility.
RO 3	2.2.20	Low RO importance (<2.5) with no related plant priority.
RO 3	2.2.10	Low RO importance (<2.5) with no related plant priority.
RO 3	2.2.15	Low RO importance (<2.5) with no related plant priority.
RO 3	2.3.8	Low RO importance (<2.5) with no related plant priority.
RO 3	2.3.7	Low RO importance (<2.5) with no related plant priority.
SRO 3	2.2.4	Cooper Nuclear Station is a single unit facility.

RO1/1	295001.AK1.04	Cooper Nuclear Station procedures no longer require operator action based on cycle oscillations for loss of forced circulation flow. Randomly selected AK1.01 as replacement.
RO2/1	239002.K1.08	Could not develop a psychometrically valid question for this K/A. This relationship is intuitive for even non-licensed operators. Any question developed would likely be of difficulty 1 and not provide any level of discrimination between the competent and non-competent candidate. Randomly reselected K1.07 as its replacement.
RO2/1	262002.A2.02	Could not develop a psychometrically valid question that would include both parts of the K/A. Randomly reselected A2.01 as its replacement.
RO2/2	286000.A2.12	Could not develop a psychometrically valid question. Any question developed would likely be of difficulty 1 and not provide any discrimination. Randomly reselected A2.07 as its replacement.
SRO1/1	295004 G2.4.50	Could not develop a SRO Only question for operation of controls in the Alarm Response Manual as it applied to a loss of DC. Randomly Reselected 2.4.30 as its replacement.
SRO1/1	295005.AA2.01	Could not develop a SRO only question for a turbine trip and the ability to interpret turbine speed. Randomly reselected 295005.AA2.06 as its replacement.
RO2/2	268000.K1.06	Could not develop a psychometrically valid question with a difficulty of greater than 1. Randomly reselected 268000.K1.09 as its replacement.
RO2/2	272000.A4.04	Could not develop a psychometrically valid question with a difficulty of greater than 1. Randomly reselected 272000.A4.06 as its replacement.
SRO1/1	2.1.32	Could not develop a SRO only question for a procedure precaution and limitations for High Drywell Pressure. Randomly reselected 2.4.30 as its replacement.