June 13, 2007

Mr. Timothy J. O'Connor
Vice President Nine Mile Point
Nine Mile Point Nuclear Station, LLC
P.O. Box 63
Lycoming, NY 13093

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT NOS. 1 AND 2, REPORT OF CHANGES OR ERRORS DISCOVERED IN THE CURRENT ACCEPTABLE EMERGENCY CORE COOLING SYSTEM EVALUATION MODELS (TAC NOS. MD4029 AND MD4030)

Dear Mr. O'Connor:

By letter dated January 11, 2007 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML070170468), Nine Mile Point Nuclear Station, LLC submitted a 30-day report in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR) 50.46(a)(3)(ii) for a calculated peak cladding temperature (PCT) change of more than 50 degrees for Nine Mile Point Nuclear Station, Unit No. 2 (NMP2). The report resulted from your December 13, 2006, evaluation of a notification by the General Electric Company (GE) of a newly discovered sensitivity to the assumed axial power shape for small break loss-of-coolant accident (LOCA) cases for Boiling-Water Reactor/3 - 6 plants. For NMP2, the evaluation estimated the impact on calculated PCT for a top-peaked power shape to be 90 ^{0}F for the limiting analysis. Because the current PCT calculated for the NMP2 GE14 fuel is 1370 ^{0}F, the new PCT was estimated to be 1460 ^{0}F (1370 ^{0}F plus 90 ^{0}F). The PCT for Nine Mile Point Nuclear Station, Unit No. 1 (NMP1) was determined to be unaffected by the new sensitivity. The report concluded that, because of the large margin between the new PCT of 1460 ^{0}F and the 10 CFR 50.46 acceptance criteria of 2200 ^{0}F, no reanalysis or other actions are planned. The report also noted that no other changes or errors to the emergency core cooling system evaluation models used for NMP1 or NMP2 were identified during 2006.

The Nuclear Regulatory Commission staff has reviewed the information provided in your report and has determined that, because there were no other changes or errors found in the evaluation models and because the new evaluated PCT has a margin of 740 ^{0}F below the 10 CFR 50.46 acceptance criteria limit of 2200 ^{0}F for the limiting analysis, no further reanalysis or other actions are required.

If you have any questions, please contact me at 301-415-1547.

Sincerely,

/RA/

Marshall J. David, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-220 and 50-410

cc: See next page

If you have any questions, please contact me at 301-415-1547.

Sincerely,

/RA/

Marshall J. David, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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T. O'Connor

Nine Mile Point Nuclear Station

cc:

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