

May 11, 2007

L-PI-07-037 10 CFR 50.55a

U S Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Unit 1 Docket 50-282 License No. DPR-42

10 CFR 50.55a Request 1-RR-4-6 for the Prairie Island Nuclear Generating Plant Unit 1 Fourth Ten-year Interval Inservice Inspection Program

Reference:

1. Letter from NRR to NMC, "Prairie Island Nuclear Generating Plant, Units 1 and 2 – Issuance of Relief Request for the Risk-Informed Inservice Inspection Program (TAC NOS. MC5644 and MC5645)," ML053270079, dated January 5, 2006.

Pursuant to 10 CFR 50.55a(g)(5)(iii), Nuclear Management Company, LLC (NMC) requests Nuclear Regulatory Commission (NRC) approval of the enclosed request for the Fourth Ten-Year Inservice Inspection (ISI) Interval. As an alternate to existing American Society of Mechanical Engineers (ASME) Section XI requirements for piping weld selection and examination volumes, NMC has implemented the alternative methods specified in Code Case N-578 and EPRI TR-112657B-A for Risk-Informed ISI plans. Prairie Island Nuclear Generating Plant (PINGP) implemented this alternative through 10 CFR 50.55a request number 1-RR-4-5, which was authorized by the NRC on January 5, 2006 (reference 1). Full examination coverage could not be obtained during a piping weld examination completed during the 2006 Unit 1 refueling outage. The exam was limited based on piping design and limited accessibility. Relief for this examination is requested on the basis that compliance with the Code requirement is impractical. The details of the request, including the proposed alternative, are contained in enclosure 1.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

Michael D. Wadley

Site Vice President, Prairie Island Nuclear Generating Plant

Nuclear Management Company, LLC

Enclosures (4)

cc: Administrator, Region III, USNRC

Project Manager, Prairie Island, USNRC Resident Inspector, Prairie Island, USNRC Chief Boiler Inspector, State of Minnesota

Enclosure 1 Prairie Island Nuclear Generating Plant Unit 1, 4th Interval ISI 10 CFR 50.55a Request No. 1-RR-4-6, Rev. 0

Relief Request in Accordance with 10 CFR 50.55a(g)(5)(iii)

-- Inservice Inspection Impracticality--

1. ASME Code Component Affected

Code Class:

Class 1

Examination Category:

R-A

Item Number:

R1.16-5

Description:

Elements Subject to Intergranular Stress

Corrosion Cracking, Risk Category 5

Component Numbers:

Summary Number 300655-RI, Weld W-17.

Reducer to Valve Weld

2. Applicable Code Edition and Addenda

The applicable Code Edition and Addenda for the Prairie Island Unit 1 Fourth Ten-Year Inspection Interval is the 1998 Edition with 2000 Addenda.

3. Applicable Code Requirement

ASME Section XI (1998 Edition, 2000 addenda) Code requires full examination coverage of inservice inspection (ISI) components per Table IWB-2500-1. As an alternative to existing ASME Section XI requirements for piping weld selection and examination volumes, Prairie Island Nuclear Generating Plant (PINGP) has implemented the alternative methods as specified in Code Case N-578 and EPRI TR-112657B-A. Relief Request (RR) number 1-RR-4-5 "Risk Informed Examination of Class 1 and 2 Piping Welds (Code Case N-578 and EPRI TR-112657)," Revision 0, was submitted to the NRC on December 29, 2004 (reference 8.1) and supplemented by letter dated August 30, 2005 (reference 82) in a response to additional information. The NRC approved the use of 1-RR-4-5 on January 5, 2006 (reference 8.3).

From Code Case N-578, Table 1, "Examination Category R-A, Risk-Informed Piping Examinations," Item Number R1.16, "Elements Subject to Intergranular Stress Corrosion Cracking (IGSCC)" piping welds with risk category 5, medium are subject to be examined by volumetric examination method of essentially 100 percent of the examination volume identified in Figure IWB-2500-8(c). Per the Risk-Informed ISI (RI-ISI) Program, this volume must be extended to ½-inch beyond each side of the base material thickness transition or the counter bore. In addition, ultrasonic examinations for welds in piping must be performed using procedures, personnel, and equipment qualified to the requirements of Appendix VIII of the ASME Code, Section XI,

Enclosure 1 Prairie Island Nuclear Generating Plant Unit 1, 4th Interval ISI 10 CFR 50.55a Request No. 1-RR-4-6, Rev. 0

1995 Edition with the 1996 Addenda, "Performance Demonstration for Ultrasonic Examinations Systems."

4. Impracticality of Compliance

Relief is requested on the basis that compliance with the Code requirement is impractical due to limited accessibility for the exam due to piping design. The PINGP construction permit was issued in 1968. The facility was designed and constructed with limited accessibility due to component configurations and/or physical barriers for which 100 percent examination coverage is not achievable on some ISI components examined for the Fourth Ten-Year Interval.

The reducer to valve weld (W-17) is located in the safety injection system piping (see enclosure 2). The reducer and valve material are austenitic stainless steel. The reducer to valve weld (W-17) was subject to volumetric examination using personnel and procedures qualified in accordance with Appendix VIII, Supplement 2, during the 2006 Unit 1 refueling outage. The examination was conducted using 45 degree shear and 60 degree longitudinal transducers. The examination was limited to 50 percent in both the axial and circumferential directions from the reducer side of the weld due to the weld joint configuration connection to the valve. The credited volumetric examination of the Weld Required Volume (WRV) was limited to 50 percent and only a single-sided examination could be performed. It should be noted that the volumetric examination was performed through 92.8 percent of the Code WRV; however, the Performance Demonstration Initiative (PDI) Appendix VIII procedure used is not qualified for the detection of flaws on the far side of single sided access examinations on austenitic stainless steel piping welds. The techniques employed for the examination provide for a best effort examination. The following supporting documentation has been provided:

Enclosure 2, ISI Drawing ISI-24

Enclosure 3, Examination Report Number 2006U047

Enclosure 4, Examination Report Number 2006U055

Additionally, NMC looked at the remaining welds within this segment and risk group and determined no other welds would produce greater code coverage than that obtained on the weld examination performed on weld W-17. Due to the limited amount of welds within this segment and risk group, no other selection was possible.

5. Burden Caused by Compliance

To comply with the code-required examination of the weld, the component would have to be redesigned.

Enclosure 1 Prairie Island Nuclear Generating Plant Unit 1, 4th Interval ISI 10 CFR 50.55a Request No. 1-RR-4-6. Rev. 0

6. Proposed Alternative and Basis for Use

As an alternative to the code-required examination, NMC requests relief for a limited scan of 50 percent in lieu of the essentially 100 percent volumetric requirement for the reducer to valve weld (W-17). The exam was performed during the 2006 Unit 1 refueling outage and was described in Section 4 of this request. Additionally, the weld is included in the boundary examined by visual examination VT-2 during pressure testing. Pressure testing was completed June 2, 2006, with no evidence of leakage identified in the vicinity of the weld. The performance of the single-sided ultrasonic examination, in addition to the performance of the visual VT-2 examination, provides the best examination coverage possible within the limitations of the current design configuration.

In-service inspections at Prairie Island Unit 1 are completed to the greatest extent practical. When limitations to required inspections are encountered, procedure SWI NDE-LTS-1, "Limitations to NDE," is applied. The procedure requires a review of the procedures to obtain maximum coverage and documentation of the limitation. The procedure also examines whether an alternative method could be used to obtain better coverage as allowed by the Code. This procedure was used and the maximum inspection coverage was achieved. The limitation was noted on the ISI examination reports and was included in the 2006 ISI Outage Summary Report. In accordance with 10 CFR 50.55a(g)(5)(iv), NMC will continue to document these types of limitations.

Additionally, the previous Section XI (SXI) program did not select any of these risk category 5 reactor coolant system welds for inspection. The RI-ISI program selected one of these category 5 welds for inspection. Therefore, there was a small decrease in risk by implementing the RI-ISI inspection. If PINGP were not to take credit for this exam, there would be no change in risk between the previous SXI and the current RI-ISI programs for this risk category in the reactor coolant system and a zero impact on the change in risk from the SXI program.

Using the provisions described above as an alternative to the code requirement will continue to provide reasonable assurance of the structural integrity of this weld. Therefore, pursuant to 10 CFR 50.55a(g)(5)(iii), NMC requests relief from the ASME Section XI examination requirements for performing a volumetric examination of this reducer to valve weld.

7. <u>Duration of Proposed Alternative</u>

NMC requests approval of this proposed alternative for the duration of the PINGP Unit 1 Fourth Ten-Year Interval.

Enclosure 1 Prairie Island Nuclear Generating Plant Unit 1, 4th Interval ISI 10 CFR 50.55a Request No. 1-RR-4-6, Rev. 0

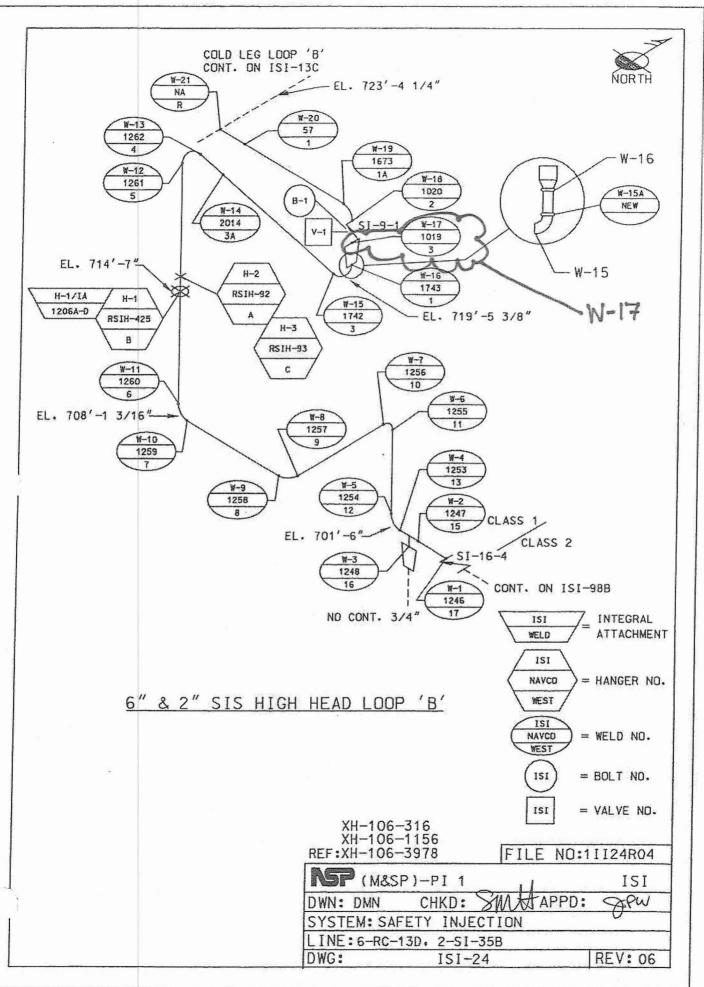
8. References

- 8.1 Letter from NMC to NRC Document Control Desk, "Relief Request to Implement Risk-Informed Inservice Inspection (ISI) Scheduling for the Fourth 10-Year Inspection interval for Prairie Islands Units 1 and 2," dated December 29, 2004 (ML043650201).
- 8.2 Letter from NMC to NRC Document Control Desk, "Response to Request for Additional Information Regarding the "Relief Request to Implement Risk-Informed Inservice Inspection (ISI) Scheduling for the Fourth 10-Year Inspection Interval for Prairie Island Units 1 and 2," dated August 30, 2005 (ML052430399).
- 8.3 Letter from NRR to NMC, "Prairie Island Nuclear Generating Plant, Units 1 and 2 Issuance of Relief Request for the Risk-Informed Inservice Inspection Program (TAC NOS. MC5644 and MC5645)," dated January 5, 2006 (ML053270079).

Enclosure 2

Prairie Island Nuclear Generating Plant Unit 1, 4th Interval 10 CFR 50.55a Relief Request No. 1-RR-4-6, Rev. 0

ISI Drawing ISI-24



Enclosure 3

Prairie Island Nuclear Generating Plant Unit 1, 4th Interval 10 CFR 50.55a Relief Request No. 1-RR-4-6, Rev. 0

Examination Report Number 2006U047

NMC

UT Calibration/Examination

- A	S	Site/Unit:	PI	/ PI1			Procedure:	SWI NDE-UT-16A			Outage I		No.: PI1RFO2006		06	
	Summary No.: 300			00655-RI Pro			Proce	edure Rev.:	re Rev.: 2			Report No.:		No.: '2	`2006U047	
	Woi	· · · · · · · · · · · · · · · · · · ·	ISI	ISI Wo			Order No.:		98273		_	Pa	ige: 1	of	4	
Code:	1998	B Edition,	2000 Addenda		Cat./Item: R-A/R1.16-			R1.16-5		Location:			Containmer	nt		
Drawing No.:			ISI-24			Descriptio	n: REDU	CER - VALVE								
System ID:	RC															
Component ID:	W-17								Size/L	ength: 1	.0" / 21.0	.0" Thickness/Diameter			: .612" / 6"	
Limitations:	See attached limitation record									Start	Time:	1600	Finis	n Time:	1610	
	Instrume	nt Setting	S		S	earch Unit		Cal.	I			Δvi	al Orientated 9	aarch Unit		
Serial No.:		040229	207	Serial No.	.:	01DHHI	3	Checks	Time	Date	Calibr		Axial Orientated Search		<u> </u>	
Manufacturer:		Paname	etrics	Manufact	urer:	KBA	4	Initial Cal.	1454	5/7/2006	Refle		Signal Amplitude %	Sweep Division	Sound	l Path
Model:	20110000000	Epocl		Size: _0.	375"/Rou	nd Shape:	Round		1600	5/7/2006	1.0" Notch		85%	8.0	1.4	0"
Delay:	0.000	Range: _	1.751"		1.5 MHz	Style: _	Comp		4040	5/7/2006	N/	Α				
M'ti Cal/Vel:	.1219	Pulser: _	Square/Max	Exam Ang	yle:4	5° # of Ele	ements: _	1 Inter. Cal. Final Cal.	1610 1732	5/7/2006						
Damping:	400 Ω	Reject:	0%	Mode: _		Shear										
Rep. Rate:	Auto	Freq.:	2.00	Measured	-	4:			Couplar							
Filter: Voltage:	.8-3.0 Mode: Fullwave Wedge Style:					MSWC	oc	Cal. Batch: #02243				Circumferential Orient		γ		
Ax. Gain (dB):	5.0	Circ. Gair	n (dB): 5.0		Soor	ch Unit Cable	r	Type: Mfg.:	Sonote		Calibr Refle		Signal Amplitude %	Sweep Division	Sound	l Path
10 Screen D	•		Sound Path	— Type:	Geard	RG-174					N/				+-	
Linearity Report		white	6LN004	Length:	6'	No. Conn.:	Exam Bato Type:	tch: #02243 Sonotrace 40								
,,	***************************************	ion Block		-	Scan Coverage			Mfg.:								
Cal. Block No.:		753		Upstream	Dowr	nstream S	can dB:	29.0					f/0!			
Thickness: 0.		Dia.:	Flat	CW Scan dB: 29.0			20.0	erence E		Gain	Re	ference/Simul Signal	Sweep	Τ.		
Cal. Blk. Temp.:	: 75 Temp	-	4304	Exam Sur		OI	_	— Seriai No.:	-	MT-032	dB		or Amplitude %		Sound	Path
Comp. Temp.:			4304	Manage P	Condition:	Grour	nd Flush	Туре:	SSRC	DMPAS	5.0	.7" SDI	H 20%	5.8	1.0	2"
Recordable Inc	dication(s):	Yes	□ No 🗸			Ultrasonic Inc	dication R	eport.)			N/A		-	 	<u> </u>	
Results:	NAD 🗸	IN		GEO 🗌					Co	mments: Si	ngle side	exam-Be	est effort exam	on far side	weld	
Percent Of Cove	Results: NAD IND GEO Comments: Single side exam-Best effort exam on far side weld volume. 50% coverage obtained. Reference Rpt# Percent Of Coverage Obtained > 90%: 50% Reviewed Previous Data: N/A 2006U55 for 60° exam data.															
Examiner	Level I	H		Signature	MI	/	Date I	Reviewer				Signa	ature			Date
Blechinger, To	odd P.		1	SIP. B	y	5/7	7/2006	Timm, Jeremy			6			5-	25-0	
Examiner Level N/A Signature Date						Date :	Site Review				Signa	ature	TIL.	- 0	Date	
N/A							\	()(1)						26-26	,	
Other	Level N	N/A		Signature			- 1	ANII Review		$\overline{}$		Sign	ature Out			Date
N/A								Daly, Gerald			(1)	unes).	hel	MA	W 292	006



Supplemental Report

Report No .:

2006U047

Page:

2 of 4

Summary No.: 300655-RI

Other: N/A

Examiner: Blechinger, Todd P.

TPB

Level: III

Reviewer: Timm, Jeremy

Date: 5-25-06 Date: 5-26-06

Examiner: N/A

Level: Level:

N/A N/A

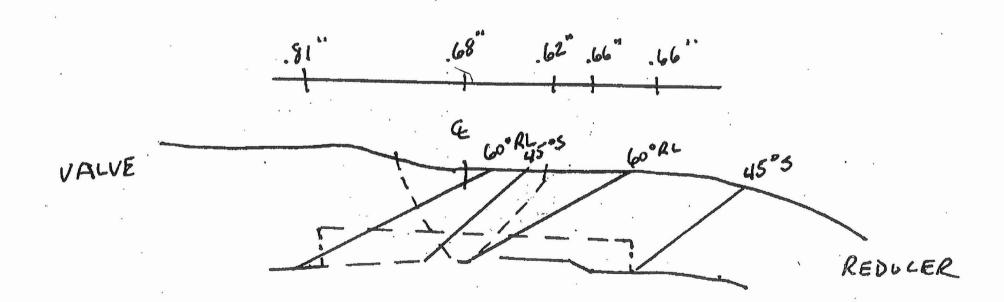
Site Review: Wren, Jerry P.

ANII Review: Daly, Gerald

Date: MAY 2 9 2006

Comments: W-17 profile and coverage plot taken @ 0°

Sketch or Photo: J:\ISIData\ISI\PI1_06 1R24 outage\Exam Supporting Documentation - Scans-Pics-etc\2006U047_055 300655-RI Supp1.bmp





Determination of Percent Coverage for UT Examinations - Pipe

Site/Unit:	e/Unit: PI / PI1		PI1	Procedo	ure: SWI NDE-UT-16A	Outage N	o.: PI1RF0	02006
Summary No.: 300655-RI		-RI	Procedure R	ev.: 2	Report No	o.: 2006U047		
Workscope:	orkscope: ISI			Work Order N	No.: 98273	 Pag	e: 3 o	f 4
_								
45 deg					*			
Scan 1	1	00.000	% Length X	100.000	% volume of length / 100 =	100.000	% total for S	can 1
Scan 2	$\overline{}$	0.000	—	0.000	% volume of length / 100 =	0.000	- % total for S	
							-	
Scan 3	1	00.000	% Length X	50.000	% volume of length / 100 =	50.000	% total for S	can 3
Scan 4	1	00.000	% Length X	50.000	% volume of length / 100 =	50.000	% total for S	can 4
	Add	totals an	nd divide by # sca	ns = 50.000	% total for 45 deg			
Other deg	<u> </u>		_ (to be used for	supplemental so	cans)			
The data to	be lis	sted belo	w is for coverage	that was not obta	ined with the 45 deg scans.			
Scan 1			% Length X	and the same of th	% volume of length / 100 =	4	_ % total for	Scan 1
Scan 2		VIII VIII VIII VIII VIII VIII VIII VII	% Length X		% volume of length / 100 =		_ % total for	Scan 2
Scan 3			% Length X		% volume of length / 100 =		% total for	Scan 3
Scan 4			% Length X		% volume of length / 100 =		— % total for ;	Scan 4
					<u>-</u>			
Percent co	omple	te cover	age					
Add totals	for ea	ch scan r	equired and divide	by # of scans to	determine;			
50.000	% T	otal for	complete exam					
				1				
Site Field S	Superv	visor:	Az	Mu	Date: 5	-26-06		



Limitation Record

Site/Unit:	PI	PI / PI1		Procedure:	SWI NDE-UT-16A	Outage No.:	PI1RFO2006				
Summary No.: 300655-RI			55-RI	Procedure Rev.:	2	Report No.:	2006U047				
Workscope:	Workscope: ISI		Work Order No.:	98273	Page:	4	of	4			

Description of Limitation:

No scans on valve due to configuration. Single side exam - Exam performed through 92.8% of code volume, however, procedure not qualified for detection on far side. Best effort exam on far side volume.

Sketch of Limitation: J:\ISIData\ISI\PI1_06 1R24 outage\Exam Supporting Documentation - Scans-Pics-etc\Todd 2006-05-10\DSC02253.JPG



Limitations removal requirements:

None

Radiation field: 1

150 mR / hr

Examiner	Level	III	8jgnature //	Date	Reviewer	Signature	Date
Blechinger,			X JUP. BY	5/7/2006	Timm, Jeremy		5-25-06
Examiner	Level	N/A	Signature (//	Date	Site Review	Signature	,
N/A			V		Wren, Jerry P.	1) en (. W_	5-26-56
Other	Level	N/A	Signature	Date	ANII Review	Signature	Date
N/A					Daly, Gerald	(1 years) 2. QD	MAY 2 9 2006
Additional - Li	mitation <	edit from S	Setup>				

Enclosure 4

Prairie Island Nuclear Generating Plant Unit 1, 4th Interval 10 CFR 50.55a Relief Request No. 1-RR-4-6, Rev. 0

Examination Report Number 2006U055

NMC

UT Calibration/Examination

and the second	Site/Unit: PI			/ PI1	/ PI1			Procedure: SWI NDE-UT-16				Outage f	No.: Pl	RFO2006	
Summary No.: 300			300655-RI	0655-RI Pro				2			Report No.:		2006U055		
	Wor	rkscope:		ISI		Work O	rder No.:		98273		_	Pa	ige: 1	of 2	
Code:	1998	Edition,	2000 Addenda		Cat./Item:	R-A/R1.	.16-5		Location:			Containmen	t		
Drawing No.:			ISI-24		Descript	tion: REDUCE	ER - VALVE	-							
System ID:	RC					-									
Component ID:	W-17							Size/L	ength: 1	.0" / 21.0		Thickness/Dia	meter:	.612" / 6"	
Limitations:	See attache	ed limitati	ion record						Start	Time:	1611	Finisl	n Time:	1617	
	Instrume	nt Setting	js		Search Unit		Cal.				Avia	Orientated S	oorob Unit		
Serial No.:		00XK	(65	Serial No.:	J104	13	Checks	Time	Date	Calibr				T	
Manufacturer:		Krautkr	ramer	Manufacturer	Mega	sonics	Initial Cal.	1530	5/7/2006	Refle		Signal Amplitude %	Sweep Division	Sound Path	
Model:		USN :	52L	Size: .25"x.	50"/Rect Shape:	Rectangle		1611	5/7/2006	1.0" N	lotch	80%	8.0	2.31"	
Delay:	0.000	Range:	.750"	Freq.: 2.25	5 MHz Style:	CGD	Inter. Cal.	1620	5/7/2006	N/	'A				
M'tl Cal/Vel:	.2289	Pulser:	Single	Exam Angle:	# of E	Elements: 2	Inter. Cal. Final Cal.	N/A 1730	5/7/2006						
Damping:	1000 Ω	Reject:	0%	Mode:	Ref. Lon									-	
Rep. Rate:	High	Freq.:	2-8 MHz			60°		Couplar		-				J	
Filter:	N/A	rrage cryic integral				gral	Cal. Batch:					ircumferential Orientated Search Unit			
Voltage: Ax. Gain (dB):	N/A 62.0	Circ Gai	in (dB): N/A		Sacrah Iluit Cal	.la	Type:	Sonotr		Calibr Refle		Signal Amplitude %	Sweep Division	Sound Path	
10 Screen D				Type:	Search Unit Cab		Mfg.:	Sonote		N/		mipikado 70	CITIOIOII		
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Linearity Repor	t No.:	200	6LN003					Sonotrace 40							
		ion Block			Scan Coverage		Mfg.:	Sonote	ch Inc.						
Cal. Block No.:					Upstream ☑ Downstream ☐ Scan dB: 68.0			erence E	Block		Reference/Simulator Block				
Thickness: 0		Dia.:		cw 🗀		Scan dB: N/	A Serial No.:	L	MT-032	Gain dB	Reflecto	Signal Amplitude %	Sweep Division	Sound Path	
Cal. Blk. Temp.				Exam Surface	Kinnell - Committee - Committe	OD	Туре:	SS RC	DMPAS	50.0	.7" SDH		5.5	1.59"	
Comp. Temp.:		. Tool: _	4304	Surface Cond	fition: Gro	und Flush	_								
Recordable Inc	dication(s):	Yes	No ✓	(If Yes, Ref. Att	ached Ultrasonic I	Indication Rep	ort.)								
Results:	NAD 🗸	11	ND 🗌	GEO				Co				06U047 for 45 to interrogate		lumo	
Percent Of Cov	erage Obtaine	ed > 90%:	0%	Reviewed Pr	evious Data:	N/A				exam po	eriorinea	to interrogate	iai side vo	nume.	
Examiner	Level I	11	(110	Signature		Date Re	viewer				Signal	ture		Date	
Blechinger, To	odd P.		AUP.	RUA		5/7/2006 Tin	nm, Jeremy				e Sta		5	-25-06	
Examiner	Level N	N/A		Signature		Date Site	e Review			17	Signa	ture		Date	
N/A				-			en, Jerry P.			ten	tw		5-21	0-06	
Other	Level N	N/A		Signature		1	III Review				Signa	ture		Date	
N/A						Da	ly, Gerald			11	unedt t.	Lif	MAY	2 9 2006	





Supplemental Report

Report No .:

2006U055

Page:

2 of 2

Summary No.: 300655-RI

Other: N/A

Examiner: Blechinger, Todd P.

TPB

Level:

Ш

Reviewer: Timm, Jeremy

Date: 5-25-06 Date: 5-26-06

Examiner: N/A

Level: Level: N/A

N/A

Site Review: Wren, Jerry P.

ANII Review: Daly, Gerald

Date: MAY 29 mis

Comments: W-17 profile and coverage plot taken @ 0°

Sketch or Photo: J:\ISIData\ISI\PI1_06 1R24 outage\Exam Supporting Documentation - Scans-Pics-etc\2006U047_055 300655-RI Supp1.bmp

