



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
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ATLANTA, GEORGIA 30303-8931

May 15, 2007

Tennessee Valley Authority
ATTN: Mr. Preston D. Swafford, Interim
Chief Nuclear Officer and
Senior Vice President
6A Lookout Place
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Chattanooga, TN 37402-2801

SUBJECT: BROWNS FERRY NUCLEAR PLANT, UNIT 1 - AUTHORIZATION TO
RESTART - DOCKET NO. 50-259

This letter documents the U.S. Nuclear Regulatory Commission's (NRC's) decision regarding the Tennessee Valley Authority's (TVA) plan to restart the Browns Ferry Nuclear Plant Unit 1 (BF1). Based on our extensive review of TVA's activities, completion of necessary revisions to the licensing and design bases, satisfactory inspection results, the recommendations of the NRC's Brown's Ferry Restart Oversight Panel and consultation with the Director of the Office of Nuclear Reactor Regulation (NRR) and the Executive Director for Operations (EDO), I authorize TVA to restart Browns Ferry Unit 1.

In a letter dated September 17, 1985, the NRC documented the understanding that TVA would not restart its reactors, including BF1, without NRC concurrence. TVA subsequently summarized its plans for recovery of BF1 in a letter dated December 13, 2002 (supplemented by letters dated February 28 and June 11, 2003), after the TVA Board of Directors approved the restart effort for Unit 1 in May 2002. In these letters, TVA described its proposed regulatory framework for the restart of BF1. The regulatory framework is based on the regulatory requirements, corrective actions, Special Programs, commitments, technical specification (TS) improvements, and TVA's internally identified deficiencies and concerns that were resolved prior to Units 2 and 3 restarts in 1991 and 1995, respectively. Prior to TVA's commencement of recovery activities, BF1 had been maintained in a defueled lay-up condition since 1985. In the Regulatory Framework letters, TVA identified 18 license amendments, 19 Generic Letters, 15 Bulletins, 11 Three Mile Island (TMI) action items, and 27 Special Programs for closeout. Some of these items required revisions to the licensing or design bases, while others needed confirmation of adequate implementation of programs or modifications to the plant in support of restart. In a letter dated August 14, 2003, the NRC staff provided an assessment accepting TVA's proposed regulatory framework.

To address the conduct of inspection activities and outline the NRC's criteria and process for determining the readiness for BF1 to restart, the NRC issued Inspection Manual Chapter (IMC) 2509, "Browns Ferry Unit 1 Restart Project Inspection Program" in September 2003. IMC 2509 provided for the establishment of the BF1 Restart Oversight Panel to monitor NRC activities. IMC 2509 also provided guidance for assessing the readiness of the unit to transition from IMC 2509 oversight to the Reactor Oversight Process (ROP).

The staff's review for BF1 restart included review and revision of the licensing and design basis; inspection of the plant hardware modifications, operations, and procedures; and the completion of an Operational Readiness Assessment Team (ORAT) inspection.

As part of its licensing efforts, the NRC staff conducted regulatory reviews of a number of licensing actions, including Special Programs, Unresolved Safety Issues, TMI Action Items, responses to Generic Communications, and restart commitments required for closeout. The NRC also reviewed and approved several revisions to the design and licensing basis needed to support restart. These items were identified in the regulatory framework letters. The NRC staff completed a series of licensing activities, including four onsite audits, 12 relief requests, 2 exemptions, and over 18 license amendments. As stated in the associated safety evaluations, the NRC staff found that TVA's activities associated with the approved licensing and design changes will be conducted in compliance with the Commission's rules and regulations, and will provide adequate protection to the health and safety of the public.

As part of its inspection program, the NRC staff reviewed TVA programs and plant activities related to the recovery of BF1. These activities included replacement, renovation, and removal of equipment and a review of plant programs, process and training of plant personnel. The NRC inspections of structural, electrical, mechanical and fire protection modifications have resulted in satisfactory findings. Onsite monitoring and review determined that activities involving replacement, renovation, and removal of equipment were satisfactorily conducted so as to maintain adequate nuclear and radiological safety. The NRC staff performed a review to verify the implementation of the licensee's design control programs, the installation and testing of modifications for Special Programs listed in the licensee's Nuclear Performance Plan, and the completion of required actions for outstanding generic communications issues.

In addition, the NRC conducted an Operational Readiness Assessment Team inspection on April 23-28, 2007 to assess management controls, implementation of site programs and personnel readiness to support safe restart and operation of Unit 1. The purpose of this inspection was to focus on the effectiveness of licensee management oversight, safety-significant activities, operator training and experience, corrective action programs, the maintenance program, operator response to annunciators and general plant conditions impacting safety, and the readiness to support three-unit operations. As described in a public meeting held on May 2, 2007 at the Browns Ferry site, the ORAT concluded that site programs, personnel and procedures were adequate for restart of Unit 1 and three unit power operations.

On May 1, 2007, the Director of NRR and I were briefed by the BF1 Restart Oversight Panel on the status of BF1 activities. Subsequently, we received the Panel's written recommendation by memorandum dated May 14, 2007. The Panel confirmed that all NRC open items had been addressed and verified the closure of the TVA open items as documented in the licensee's May 12, 2007 letter. In addition, the Panel communicated its efforts during the BF1 restart oversight to various stakeholders by holding public meetings of the Restart Oversight Panel, through information provided on the NRC's BF1 website, and through documents which can be found in the NRC's Public Document Room website and letters found in the NRC Public Document Room. On the basis of its review of the staff's licensing and inspection efforts and its review of the licensee's recovery efforts, the Panel recommended authorization for Unit 1 restart.

Again, based on consideration of the above factors and having consulted with the Director of the Office of NRR and the EDO, I authorize TVA to restart Browns Ferry Nuclear Plant Unit 1.

I understand that some additional work activities to ready Unit 1 for restart are ongoing. The NRC resident inspectors at the site will continue to observe these activities and the Commission will be kept informed of your progress. In addition, the NRC will conduct detailed inspection of your restart and power ascension activities.

Enclosed is the Inspection Plan and Schedule (IP&S) report which provides a record of the restart issues and documents closure of those issues. Consistent with IMC 2509, the Restart Oversight Panel reviewed TVA's plans for post-restart activities to ensure their acceptability and to plan for appropriate NRC oversight. Those items which are planned to be inspected after restart are indicated as post-restart items. The Panel determined that BF1 would receive NRC oversight using the Reactor Oversight Program (ROP) in four of seven cornerstones of safety. In separate correspondence, the NRC will provide a plan for transition to the ROP of the three remaining ROP Cornerstones. Additionally, an update to the IP&S report will be issued upon completion of post-restart issues.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/Victor M. McCree RA for/

William D. Travers
Regional Administrator

Docket No.: 50-259
License No.: DPR-33

Enclosure: Inspection Plan and Schedule Report

cc w/encl: (See page 4)

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Distribution w/encl: (See page 5)

Letter to Preston D. Swafford from William D. Travers dated May 15, 2007

SUBJECT: BROWNS FERRY NUCLEAR PLANT, UNIT 1 - AUTHORIZATION TO
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Browns Ferry Unit 1 Restart Inspection Plan and Schedule (IP&S) Report

Browns Ferry Units 1 (BF1), 2 (BF2), and 3 (BF3) were voluntarily shutdown by the Tennessee Valley Authority (TVA) in March 1985 for management and regulatory issues. TVA committed to obtain NRC approval prior to the restart of any of the Browns Ferry units. TVA submitted the Browns Ferry Nuclear Performance Plan, Volume 3, in August 1986, outlining the steps needed to recover the Browns Ferry units. NRC issued NUREG-1232, Volume 3, and its supplements, concluding that TVA's actions acceptably addressed Browns Ferry Nuclear Plant deficiencies. TVA obtained NRC permission to restart BF2 in 1991 and BF3 in 1995. In 2002, TVA commenced a 5-year program to recover BF1. Once BF1's startup program is completed, all three units will be similar in their respective plant configurations, and will have implemented corrective actions for outstanding regulatory and generic safety issues.

NRC Inspection Manual Chapter (MC) 2509, "Browns Ferry Unit 1 Restart Project Inspection Program," establishes the policy for the BF1 Restart Project inspection program covering BF1's renovation, and restart testing including power ascension. Inspection Procedures (IPs) or Technical Instructions (TIs), documented in Region II's inspection plan and schedule (IP&S), will provide inspection requirements for these main inspection areas that comprise the BF1 Restart Project Inspection Program.

A Restart Oversight Panel, consisting of participants from both Region II and NRR, has been established in accordance with MC 2509. This panel decides whether the required corrective actions and performance improvement changes have been implemented successfully by the licensee so that BF1 meets the relevant regulatory requirements and can be safely operated. The actual decision to permit the licensee restart to BF1 will be made by the Regional Administrator of Region II with the concurrence of the Director, Office of Nuclear Reactor Regulation (NRR) based on a written recommendation of the Restart Oversight Panel. The panel will also review the readiness of BF1 to be placed under the NRC's normal Reactor Oversight Process (ROP). The decision to permit BF1 to transition into the full oversight of the ROP will be made by the Regional Administrator of Region II with the concurrence of the Director, Office of NRR, based on a written recommendation from the Restart Oversight Panel.

In accordance with MC 2509, the NRC staff has developed IP&S which collects and provides inspection requirements for areas that comprise the BF1 Restart Project Inspection Program. This IP&S tracking tool identifies the appropriate inspection procedures or equivalent criteria to review the licensee's restart activities and to implement inspections. The tracking tool provides a means for tracking open items, allows for generating reports to support any interface with stakeholders, the Commission, and other agencies, and provides cross-references between the items tracked and applicable inspection reports. IP&S also allows for rescheduling of inspections to accommodate licensee schedule changes and additional inspections that may be identified by Region II or by the BF1 Restart Oversight Panel.

Issues or activities are considered closed in IP&S when the following three criteria have been met: 1) TVA has reviewed and closed the issue or activity in their quarterly update submitted to the NRC, 2) The issue or activity has been reviewed and/or an inspection has been completed by the NRC and an associated inspection report (IR) has been issued, and 3) The BF1 Restart Oversight Panel has been briefed or reviewed available documentation to satisfy the two aforementioned criteria to permit closure of the item.

The attached IP&S report includes open and closed items as discussed above. Documents associated with the NRC oversight of the BF1 Recovery can be found on the NRC's website at <http://www.nrc.gov/NRR/OVERSIGHT/ASSESS/bf1-recovery.html>.

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 1	Cable Ampacity	SP	CLOSED	37550	2004-007 2006-009	Mitigating Systems Initiating Events

Items numbered 1-108 constitute a basis for Restart as identified in NRC Regulatory Framework Letter dated August 14, 2003 and should be evaluated for closure prior to Restart.

BF1- 2	Cable Installation/ Cable Separation issues	SP	CLOSED	37550 51053 37551	2004-007 2004-009 2005-008 2005-009 2006-006 2006-007 2006-012 2006-009 2007-009	Mitigating Systems Initiating Events
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BF1- 3	Cable Splices	SP	CLOSED	51053 37550	2004-008 2005-008 2006-007	Mitigating Systems Initiating Events
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BF1- 4	Cable Tray Supports	SP	CLOSED	37551 62002	2003-010 2004-006 2005-008	Mitigating Systems Initiating Events
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BF1- 5	Component and Piece Part Qualification	SP	CLOSED	37550	2006-009	Mitigating Systems Initiating Events Barrier Integrity
BF1- 6	Conduit Supports	SP	CLOSED	37551	2004-006 2005-008	Mitigating Systems Initiating Events
BF1- 7	Configuration Management/ Design Baseline	SP	CLOSED	37551	2005-008	Mitigating Systems Initiating Events Barrier Integrity
BF1- 8	Containment Coatings	SP	CLOSED	37550 62002	2004-007 2004-008 2005-006 2006-009	Mitigating Systems Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 9	Control Rod Drive insert/withdrawal piping	SP	CLOSED	37551	2004-007 2004-008 2005-008 2005-009 2006-006	Mitigating Systems Initiating Events
BF1- 10	Design Calc Review	SP	CLOSED	37551	2005-008	Mitigating Systems Initiating Events Barrier Integrity
BF1- 11	Environmental Qualification	SP	CLOSED	37550	2003-010 2004-008 2005-008 2006-009	Mitigating Systems Initiating Events
BF1- 12	Fire Protection / 10CFR50, Appendix R	SP	CLOSED	37550 64704 37551	2004-009 2005-016 2006-012 2007-009	Mitigating Systems Initiating Events Barrier Integrity

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BF1- 13	Flexible Conduits	SP	CLOSED	37550	2004-007 2004-009 2006-006	Mitigating Systems Initiating Events
BF1- 14	Fuses	SP	CLOSED	37550 51053	2004-008 2005-008 2006-007	Mitigating Systems Initiating Events
BF1- 15	HVAC Duct Supports	SP	CLOSED	37550 50100	2006-008	Mitigating Systems
BF1- 16	Instrument Sensing Lines	SP	CLOSED	37550	2006-007 2006-009	Mitigating Systems Initiating Events

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 17	Instrument Tubing	SP	CLOSED	37550 37551	2005-008 2006-007 2006-008	Mitigating Systems Initiating Events
BF1- 18	Intergranular Stress Corrosion Cracking (IGSCC)	SP	CLOSED	37551 55050	2003-009 2003-010 2004-006 2004-007 2004-009 2005-008	 Barrier Integrity
BF1- 19	Large Bore Piping Supports	SP	CLOSED	50090	2003-009 2005-006 2005-008 2005-009 2006-006 2006-007 2006-009	Mitigating Systems Barrier Integrity
BF1- 20	Long Term Torus Integrity Program	SP	CLOSED	37550 37551 62002 50090	2003-009 2003-010 2004-007 2004-009 2004-011 2006-006 2006-007 2006-009	Mitigating Systems Barrier Integrity

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BF1- 21	Drywell Steel Platforms and Upper drywell platforms	SP	CLOSED	37550 37551 62002	2003-009 2004-007 2005-008	Mitigating Systems Barrier Integrity
BF1- 22	Misc Steel Frames	SP	CLOSED	37551	2003-009 2006-006	Mitigating Systems Barrier Integrity
BF1- 23	Moderate Energy Line Break (MELB)	SP	CLOSED	37550	2006-007	 Barrier Integrity
BF1- 24	Platform Thermal Growth	SP	CLOSED	37551	2003-009 2005-008 2006-006	Mitigating Systems Barrier Integrity

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BF1- 25	Probabilistic Safety Assessment (PSA)	SP	NOT REQUIRED	93804		Mitigating Systems
<p>Item #25, Probabilistic Safety Assessment (PSA) has been updated to "Not Required" based on document reviews, including NRC staff verification that commitments pertaining to PSA had been previously addressed. The Restart Oversight Panel determined that TVA resolved the PSA issue during Unit 3 restart with the submittal of the Browns Ferry Multi-Unit PSA and dependency matrices, as stated in TVA's Regulatory Framework letter dated December 13, 2002. Furthermore, NRC Regulatory Framework letter dated August 13, 2003 did not identify PSA as a special program.</p>						
BF1- 26	Q List	SP	CLOSED	37550	2006-007	Mitigating Systems
						Initiating Events
						Barrier Integrity
BF1- 27	Restart Test Program (RTP)	SP	CLOSED	37551 37550 35301 70315 70301	2005-006 2005-007 2005-008 2005-009 2006-006 2006-007 2006-008 2006-009 2007-009	Mitigating Systems
						Initiating Events
						Barrier Integrity
BF1- 28	Seismic Class II Over I / Spatial Systems Interactions and Water Spray	SP	CLOSED	37551	2005-008	Mitigating Systems
						Initiating Events
						Barrier Integrity

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BF1- 29	Small Bore Piping / Supports	SP	CLOSED	37550 37551	2005-008 2006-007 2006-008	Mitigating Systems Initiating Events
BF1- 30	Thermal Overloads	SP	CLOSED	37550 51053	2004-008 2005-008 2006-007	Mitigating Systems Initiating Events
BF1- 31	BU 79-02 Pipe Support Plate Designs Using Concrete Expansion Anchor Bolts	GEN	CLOSED	37550	2006-009	Mitigating Systems Initiating Events
BF1- 32	BU 79-12 Short Period Scrams at BWR Facilities	GEN	CLOSED	92701	2006-007	Mitigating Systems Initiating Events

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BF1- 33	BU 79-14 Seismic Analysis for As-Built Safety Related Piping Systems	GEN	CLOSED	37550 37551	2006-009	Mitigating Systems Barrier Integrity
BF1- 34	BU 80-06 Engineered Safety Feature Reset Controls	GEN	CLOSED	92701	2006-008	Mitigating Systems Barrier Integrity
BF1- 35	BU 84-02 Failure of GE type HFA Relays in use in class 1E Safety Systems	GEN	CLOSED	92701	2004-009	Mitigating Systems
BF1- 36	BU 86-02 Static "O" Ring Differential Pressure Switches	GEN	CLOSED	92701	2005-006	Mitigating Systems

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BF1- 37	BU 88-03 Inadequate Latch Engagement in HFA Type Relays Manufactured by GE	GEN	CLOSED	92701	2004-009	Mitigating Systems
BF1- 38	BU 88-04 Potential Safety Related Pump Loss	GEN	CLOSED	92701	2004-009	Mitigating Systems
BF1- 39	BU 88-07 Power Oscillation in Boiling Water Reactors	GEN	CLOSED	92701	2006-007	Initiating Events Barrier Integrity
BF1- 40	BU 88-10 Nonconforming Molded-Case Circuit Breakers	GEN	CLOSED	92701	2005-009	Mitigating Systems

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<i>BF1- 41</i>	BU 90-01 Loss of Fill Oil in Rosemount Transmitters	GEN	CLOSED	92701	2005-009	Mitigating Systems
<i>BF1- 42</i>	BU 93-02 Debris Plugging of Emergency Core Cooling Suction Strainers	GEN	CLOSED	92701	2006-006 2006-008	Barrier Integrity
<i>BF1- 43</i>	BU 93-03 Resolution of Issues Related to Reactor Vessel Water Level Instruments in BWR's	GEN	CLOSED	92701	2006-009	Mitigating Systems
<i>BF1- 44</i>	BU 95-02 Unexpected Clogging of a RHR Pump Strainer While Operating in Suppression Pool Cooling Mode	GEN	CLOSED	92701	2006-006	Mitigating Systems
						Barrier Integrity

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BF1- 45	BU 96-03 Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling Water Reactors	GEN	CLOSED	92701	2006-006 2006-009	Mitigating Systems Barrier Integrity
BF1- 46	GL 82-33 Instrumentation to Follow the Course of an Accident - R.G. 1.97	GEN	CLOSED	92701	2005-007	Mitigating Systems
BF1- 47	GL 83-08 Modification of Vacuum Breakers on Mark 1 Containments	GEN	CLOSED	92701	2005-006	 Barrier Integrity
BF1- 48	GL 83-28 Salem ATWS - Item 1.2 Trip Review - Item 4.5.2 Periodic On-Line Testing - Item 4.5.3 Intervals for On-Line Testing	GEN	CLOSED	92701	2006-007	Mitigating Systems Initiating Events

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BF1- 49	GL 87-02 Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors	GEN	CLOSED	92701	2004-006 2006-006 2007-009	Mitigating Systems Barrier Integrity
BF1- 50	GL 88-01 NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	GEN	CLOSED	92701	2005-008 2006-006 2006-007	Barrier Integrity
BF1- 51	GL 88-11 Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations	GEN	CLOSED	92701	2006-007	Barrier Integrity
BF1- 52	GL 88-14 Instrument Air Supply System Problems Affecting Safety Related Equipment	GEN	CLOSED	92701	2005-008	Mitigating Systems Initiating Events

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BF1- 53	GL 88-20 Individual Plant Examination and Supplement 4, Individual Plant Examination of External Events for Severe Accident Vulnerabilities	GEN	NOT REQUIRED	93804		Mitigating Systems Initiating Events Barrier Integrity
BF1- 54	GL 89-06 Safety Parameter Display System	GEN	CLOSED	92701	2006-007	Mitigating Systems Barrier Integrity
BF1- 55	GL 89-08 Erosion/Corrosion - Induced Pipe Wall Thinning	GEN	CLOSED	92701	2005-009	 Barrier Integrity
BF1- 56	GL 89-10 Safety-Related Motor-Operated Valve Testing and Surveillance	GEN	CLOSED	37550	2004-009 2006-009	Mitigating Systems

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 57	GL 89-13 Service Water System Problems Affecting Safety Related equipment	GEN	CLOSED	92701	2005-009 2006-007	Mitigating Systems Barrier Integrity
BF1- 58	GL 89-16 Installation of a Hardened Wetwell Vent	GEN	CLOSED	92701	2005-006	Barrier Integrity
BF1- 59	GL 92-01 Reactor Vessel Structural Integrity	GEN	CLOSED	55100 37550	2006-007	Barrier Integrity
BF1- 60	GL 92-04 Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs.	GEN	CLOSED	37550 37551	2006-009	Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 61	GL 94-02 Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in BWRs	GEN	CLOSED	92701	2006-007	Mitigating Systems Initiating Events Barrier Integrity
BF1- 62	GL 94-03 Intergranular Stress Corrosion Cracking of Core Shrouds in BWR's	GEN	CLOSED	92701	2005-008	 Barrier Integrity
BF1- 63	GL 95-07 Pressure Locking and Thermal Binding of Safety-Related and Power-Operated Gate Valves	GEN	CLOSED	37550 37551	2006-006	Mitigating Systems
BF1- 64	GL 96-01 Testing of Safety-Related Logic Circuits	GEN	CLOSED	37550	2006-009	Mitigating Systems

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 65	GL 96-05 Periodic Verification of Design Basis Capability of Motor Operated Valves	GEN	CLOSED	92701	2005-009	Mitigating Systems
BF1- 66	GL 96-06 Assurance of Equipment Operability and Containment Integrity During Design Basis Accident Conditions	GEN	CLOSED	92701	2005-008	Mitigating Systems Barrier Integrity
BF1- 67	GL 97-04 Assurance of Sufficient Net Positive Suction for Emergency Core Cooling and Containment Heat Removal Pumps	GEN	CLOSED	92701	2006-006 2006-009	Mitigating Systems Barrier Integrity
BF1- 68	GL 98-01 Readiness of Computer Systems at Nuclear Power Plants	GEN	CLOSED	92701	2005-006	Mitigating Systems

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 69	GL 98-04 Potential for Degradation of the ECCS and Containment Spray System After a LOCA because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	GEN	CLOSED	37550 37551	2006-006 2006-008	Mitigating Systems Barrier Integrity
BF1- 70	Control Room Design Review	TMI	CLOSED	92701	2006-007	Mitigating Systems Initiating Events
BF1- 71	Safety Parameter Display Console	TMI	CLOSED	92701	2006-007	Mitigating Systems Initiating Events Barrier Integrity
BF1- 72	Post-Accident Sampling System	TMI	CLOSED	92701	2006-006	 Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 73	Containment Isolation Dependability - Implement Diverse Isolation, containment purge valves	TMI	CLOSED	37550	2007-009	Mitigating Systems Barrier Integrity
BF1- 74	Accident - Monitoring - Containment High Range Radiation	TMI	CLOSED	92701	2006-006	Mitigating Systems Barrier Integrity
BF1- 75	Accident Monitoring - Containment Pressure	TMI	CLOSED	92701	2005-006	Mitigating Systems Barrier Integrity
BF1- 76	Accident - Monitoring - Containment Water Level	TMI	CLOSED	92701	2005-006	Mitigating Systems Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 77	(Generic Letter 84-23) Instrumentation for Detection of Inadequate Core Cooling	TMI	CLOSED	92701	2005-007	Mitigating Systems Barrier Integrity
BF1- 78	HPCI/RCIC Initiation Levels	TMI	CLOSED	92701	2005-007	Mitigating Systems
BF1- 79	ADS Actuation Modifications	TMI	CLOSED	92701	2005-007	Mitigating Systems Barrier Integrity
BF1- 80	Common reference level for BWR	TMI	CLOSED	92701	2005-008	Mitigating Systems

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 81	Qualification of ADS Accumulators	TMI	CLOSED	92701	2005-007	Mitigating Systems Barrier Integrity
BF1- 82	MPA-B011 Flood of equipment important to safety	MPA	CLOSED	92701	2006-007	Mitigating Systems Initiating Events
BF1- 83	MPA-B041 Fire Protection Final TS (including SER Supplement)	MPA	CLOSED	92701	2006-007	Mitigating Systems Initiating Events
BF1- 84	MPA-C002 BWR recirculation pump trip (ATWS)	MPA	CLOSED	92701	2006-007	Mitigating Systems Initiating Events

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
<i>BF1- 85</i>	TI 2500/20 - Inspection to determine compliance with ATWS Rule 10CFR 50.62	TI	CLOSED	92701	2006-007	Mitigating Systems Initiating Events
<i>BF1- 86</i>	TI 2515/111 EDSFI Followup	TI	CLOSED	92701 93811 TI 2515/111	2006-008	Mitigating Systems Initiating Events
<i>BF1- 87</i>	TI 2515/154 Spent Fuel Material Control and Accountability	TI	CLOSED	92701	2004-008	Initiating Events Barrier Integrity
<i>BF1- 88</i>	TI 2515/156 Offsite Power System Readiness	TI	CLOSED	92701	2004-008	Mitigating Systems Initiating Events

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 89	TI 2515/157 Mark I and II Containment Operations	TI	CLOSED	37550 37551		
						Barrier Integrity
BF1- 90	USI A-7 Mark 1 Long-Term Program, NUREG-0661, Supplement 1	MPA	CLOSED	92701	2006-007	
						Barrier Integrity
BF1- 91	USI A-9 ATWS	MPA	CLOSED	92701	2006-007	Mitigating Systems
						Initiating Events
BF1- 92	USI A-24 Qualification of Class 1E Safety-Related Equipment	MPA	CLOSED	92701	2006-007	Mitigating Systems

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 93	USI A-26 Reactor Vessel Pressure Transient Protection	MPA	CLOSED	92701	2006-007	Barrier Integrity
BF1- 94	USI A-44 Station Blackout	MPA	CLOSED	37550 37551	2006-009 2007-009	Mitigating Systems Initiating Events
BF1- 95	USI A-46 Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors	MPA	CLOSED	92701	2006-006 2007-009	Mitigating Systems
BF1- 96	USI A-75 MPA-B085, Required actions on Generic Implications of Salem ATWS Events, Section 1.2, Post Trip Review	MPA	CLOSED	92701	2006-007	Mitigating Systems Initiating Events

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 97	New Simulator qualification/ Fidelity and Operator Staffing-Review licensee's new simulator to support 3 unit operations. Review Licensed Operator Staffing Plans.	MISC	NOT REQUIRED	37550 37551		Mitigating Systems Initiating Events
BF1- 98	Lay-up Inspections	MISC	CLOSED	92050	2004-006	Mitigating Systems Barrier Integrity
BF1- 99	In-vessel inspections, core spray, UT- Full RPV internals/weld inspections complete 2005 for 10-yr ISI	RWA	CLOSED	55050 37550 37551 71111.08	2005-008 2005-009 2006-006 2006-007	Barrier Integrity
BF1- 100	Operational Readiness Assessment Team inspection (ORAT)	MISC	CLOSED	93806	2007-006	Mitigating Systems Initiating Events Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 101	Common Accident Signal	MISC	CLOSED	37551 71111.17	2005-006	Mitigating Systems Initiating Events
BF1- 102	Extended Power Uprate- Determine scope of needed inspection reviews to support licensing action	MISC	NOT REQUIRED	37550 37551		Mitigating Systems Initiating Events Barrier Integrity
BF1- 103	Enhanced Restart Monitoring-Restart Panel to determine duration. Proposed 24 hr coverage.	MISC	NOT REQUIRED	72518 72526		Mitigating Systems Initiating Events Barrier Integrity
BF1- 104	License Renewal Application-Determine scope of needed inspection reviews to support licensing action	LR	CLOSED	37550	2007-011	

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BF1- 105	Maintenance Rule-U1 to comply with scoping requirements of 50.65(b) for systems required to be operable or TS applicable.	MISC	CLOSED	37550	2006-009	Mitigating Systems Barrier Integrity
BF1- 106	License Amendment 234 Review - Improved Standard TS	MISC	CLOSED			
BF1- 107	Turbine Building Dynamic Seismic Analysis-Confirm seismic accelerations for Turbine Building	MISC	CLOSED	37550 37551	2007-009	Mitigating Systems
BF1- 108	GL 2003-01 Control Room Habitability	GEN	NOT REQUIRED	37550	2007-009	Mitigating Systems Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 109	RCS Pipe Replacement MODs	RWA	CLOSED	37550	2003-009	Mitigating Systems
				37551	2003-011	
				55050	2004-006	
				57050	2004-007	
				57060	2004-008	
				57090	2004-009	
				73753	2005-006	
				73755	2006-006	
				71111.08		
The following Items (109 and above) are being tracked for completeness only.						
BF1- 110	Common Accident Signal Logic Changes	RWA	CLOSED	71111.17	2004-007	Mitigating Systems
				37550	2004-008	
				37551	2004-009	
					2005-006	
					2005-007	
BF1- 111	ECCS Systems MODs	RWA	CLOSED	71111.17	2004-006	Mitigating Systems
				37550	2004-007	
				37551	2004-008	
					2004-009	
					2005-006	
					2005-007	
					2005-008	
					2005-009	
					2006-006	
					2006-007	
BF1- 112	RHRSW MODs	RWA	CLOSED	37550	2004-006	Mitigating Systems
				37551	2005-008	
				71111.17	2005-009	
					2006-007	

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BF1- 113	Cable Installation Activities	RWA	CLOSED	37550 37551	2003-011 2005-007 2005-009 2006-006 2006-007 2006-012 2006-009 2007-009	Mitigating Systems
BF1- 114	EQ MODs	RWA	CLOSED	35301 37550 37551	2004-009 2005-007 2006-006 2006-009	Mitigating Systems
BF1- 115	Cable Splices and Terminations	RWA	CLOSED	37550 37551	2006-007	Mitigating Systems
BF1- 116	CRDR MODs	RWA	CLOSED	71111.17 37550 37551	2004-006 2004-009 2005-006 2005-007 2005-008 2005-009 2006-006 2006-007	Mitigating Systems Initiating Events

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 117	Cable Tray and Conduit Modifications	RWA	CLOSED	62002 71111.17 37550 37551	2003-009 2003-010 2005-008	Mitigating Systems
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BF1- 118	Large Bore Pipe Support MODs	RWA	CLOSED	35301 37550 37551	2006-009	Mitigating Systems
						Barrier Integrity
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BF1- 119	Small Bore Pipe Support MODs	RWA	CLOSED	37551	2005-008 2006-008	Mitigating Systems
						Barrier Integrity
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BF1- 120	Instrument Sensing Lines MODs	RWA	CLOSED	35301 37550 37551	2006-008 2006-009	Mitigating Systems
						Barrier Integrity

<i>IP&S ID Number</i>	<i>Issue/Activity Description</i>	<i>Type of Issue/ Activity</i>	<i>Status</i>	<i>Inspection Procedure(s)</i>	<i>Inspection Report(s)</i>	<i>Cornerstone(s)</i>
BF1- 121	Fire Protection Changes	RWA	CLOSED	64704 37550 37551	2004-009 2005-016 2006-012 2007-009	Mitigating Systems Initiating Events Barrier Integrity
BF1- 122	Common Accident Signal Logic Acceptance Test	RT	CLOSED	37550 37551 92901	2006-008	Mitigating Systems Initiating Events
BF1- 123	Full ROP Transition Review	ROP	CLOSED	92701		Mitigating Systems Initiating Events Barrier Integrity
BF1- 124	Develop Enhanced ROP Baseline Inspection Plan	ROP	CLOSED	37550 37551		Mitigating Systems Initiating Events Barrier Integrity

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BF1- 125	Mitigating Systems Cornerstone Open Items Closeout Inspection	ROP	CLOSED	37550 37551	2007-009	Mitigating Systems
BF1- 126	Initiating Events Cornerstone Open Items Closeout Inspection	ROP	CLOSED	37550 37551	2007-009	Initiating Events
BF1- 127	Barrier Integrity Cornerstone Open Items Closeout Inspection	ROP	CLOSED	37550 37551	2007-009	Barrier Integrity
BF1- 128	Post SPOC II Review - SLC	SPOC	CLOSED	37551	2006-008	Mitigating Systems

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BF1- 129	Post SPOC II Review - 250 VDC	SPOC	CLOSED	37551	2006-008	Mitigating Systems
BF1- 130	Post SPOC II Review - CS	SPOC	CLOSED	37551	2006-009	Mitigating Systems
BF1- 131	Post SPOC II Review - RHR	SPOC	CLOSED	37551	2006-009	Mitigating Systems
BF1- 132	Post SPOC II Review - RPS	SPOC	CLOSED	37551	2006-009	Mitigating Systems

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BF1- 133	Post SPOC II Review - RBCCW	SPOC	CLOSED	37551	2006-009	Mitigating Systems
BF1- 134	Post SPOC II Review - CRD	SPOC	CLOSED	37551	2006-009	Mitigating Systems
BF1- 135	Post SPOC II Review - RCIC	SPOC	CLOSED	37551	2007-009	Mitigating Systems
BF1- 136	Post SPOC II Review - HPCI	SPOC	CLOSED	37551	2007-009	Mitigating Systems

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BF1- 137	Post SPOC II Review - Main Steam	SPOC	CLOSED	37551	2007-006	Mitigating Systems
BF1- 138	License Renewal Followup Inspection	LR	CLOSED	92701 37550 37551	2007-011	Mitigating Systems Barrier Integrity
BF1- 139	Core Spray Acceptance Test	RT	CLOSED	37551	2006-008	Mitigating Systems
BF1- 140	Control Rod Drive Acceptance Test	RT	CLOSED	37551	2006-009 2007-009	Mitigating Systems

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BF1- 141	250 V DC Distribution Acceptance Test	RT	CLOSED	37551	2006-008	Mitigating Systems
BF1- 142	Standby Liquid Control System Acceptance Test	RT	CLOSED	37551	2006-008	Mitigating Systems
BF1- 143	RHR Acceptance Test	RT	CLOSED	37551	2006-009	Mitigating Systems
BF1- 144	Reactor Core Isolation Cooling Acceptance Test	RT	CLOSED	37551	2006-009	Mitigating Systems

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BF1- 145	Reactor Protection Acceptance Test	RT	CLOSED	37551	2006-009	Mitigating Systems
BF1- 146	Reactor Building Closed Cooling Water Acceptance Test	RT	CLOSED	37551	2006-009	Mitigating Systems
BF1- 147	High Pressure Coolant Injection Acceptance Test	RT	CLOSED	37551	2007-009	Mitigating Systems
BF1- 148	Main Steam Acceptance Test	RT	CLOSED	37551	2007-006	Mitigating Systems

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BF1- 149	Diesel Generator Load Acceptance Test	RT	CLOSED	37551	2006-009	Mitigating Systems
BF1- 150	480 Volt Shutdown Board Load Shed Logic Acceptance Test	RT	CLOSED	37551	2006-008	Mitigating Systems
BF1- 151	ECCS Actuation Acceptance Test	RT	CLOSED	37551	2006-008 2006-009	Mitigating Systems
BF1- 152	Refueling and Outage Activities Inspection	MISC	CLOSED	71111.20	2006-009 2007-009	Mitigating Systems Initiating Events Barrier Integrity

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BF1- 153	Fire Protection Walkdown	MISC	CLOSED	71111.05	2006-004 2006-005 2006-009	Mitigating Systems Initiating Events Barrier Integrity
BF1- 154	Equipment Alignments		CLOSED	71111.04	2006-009	Mitigating Systems
BF1- 155	Power Ascension Testing	PAT	OPEN	71111.20		Mitigating Systems Initiating Events Barrier Integrity
BF1- 156	Self-Assessment	MISC	OPEN	35301		

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BF1- 157	GL 87-02 closeout - scope of inspections	GEN	CLOSED	37551	2007-009	
BF1- 158	URI, 06-08-02, Impact of Duct tape on Instrument tubing	URI	CLOSED		2006-009	
BF1- 159	IFI 95-55-01, Containment Air Monitors	IFI	CLOSED	92701	2006-009	
BF1- 160	Part 21 on OTEC instruments		CLOSED		2006-009	

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<i>BF1- 161</i>	Complete Area Turnover for all areas required for Fuel Load	AT	CLOSED		2006-009	Mitigating Systems Initiating Events Barrier Integrity
<i>BF1- 162</i>	Complete Area Turnover for all areas required for Mode 2	AT	CLOSED		2006-009 2007-009	Mitigating Systems Initiating Events Barrier Integrity
<i>BF1- 163</i>	URI 06-12-01, Feasibility of local operator actions to achieve safe shutdown	URI	OPEN			
<i>BF1- 164</i>	URI 06-09-03, Criteria was not adequately defined to ensure divisional separation for cables were maintained	URI	CLOSED		2007-009	

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LEGEND FOR TYPE FIELD	
<i>Type</i>	<i>Description</i>
<i>GEN</i>	<i>Generic Communication</i>
<i>IFI</i>	<i>Inspector Followup Item</i>
<i>LA</i>	<i>Licensing Action</i>
<i>LR</i>	<i>License Renewal</i>
<i>MISC</i>	<i>Miscellaneous</i>
<i>NCV</i>	<i>Noncited Violation</i>
<i>OI</i>	<i>Open Item</i>
<i>PAT</i>	<i>Power Ascension Testing</i>
<i>REG</i>	<i>Regulatory Issue</i>
<i>ROP</i>	<i>ROP Transition</i>
<i>RT</i>	<i>Restart Test</i>
<i>RWA</i>	<i>Renovation Work Activity</i>
<i>SP</i>	<i>Special Program</i>
<i>SPOC</i>	<i>System Pre-Operability Checklist</i>
<i>TMI</i>	<i>Three Mile Island</i>
<i>URI</i>	<i>Unresolved Item</i>
<i>VIO</i>	<i>Violation</i>
<i>AT</i>	<i>Area Turnover</i>
<i>MPA</i>	<i>Multi-Plant Action</i>
<i>TI</i>	<i>Technical Instruction</i>

NOTES

On March 5, 2007 the Panel added a new category in the issue status field. This category is the "Not Required" category which better clarifies and identifies those items which are not required to be completed prior to restart but will continue to be tracked for completeness. The change is only administrative in nature and does not impact NRC's assessment of Browns Ferry Unit 1 nor NRC's scheduling of resources. The items that were identified as "Not Required" and no longer require completion prior to Restart but will continue to be tracked in IP&S are the following: Item #25, Probabilistic Safety Assessment (PSA); Item #53, GL 88-20 Individual Plant Examination and Supplement 4, Individual Plant Examination of External Events for Severe Accident Vulnerabilities; Item #97, New Simulator qualification/ Fidelity and Operator Staffing-Review licensee's new simulator to support 3 unit operations; Item #102, Extended Power Uprate-Determine scope of needed inspection reviews to support licensing action; Item #103, Enhanced Restart Monitoring-Restart Panel to determine duration. Proposed 24 hr coverage; Item #108, GL 2003-01 Control Room Habitability