

June 15, 2007

Mrs. Mary G. Korsnick  
Vice President R.E. Ginna Nuclear Power Plant  
R.E. Ginna Nuclear Power Plant, LLC  
1503 Lake Road  
Ontario, NY 14519

SUBJECT: NRC RECEIPT OF R.E. GINNA NUCLEAR POWER PLANT RESPONSE TO  
GENERIC LETTER 2003-01 "CONTROL ROOM HABITABILITY" (TAC NO.  
MB9807)

Dear Mrs. Korsnick:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01 "Control Room Habitability" dated August 4, 2003 (Agencywide Documents and Access Management System (ADAMS) Accession No. ML032230027); June 30, 2004 (ADAMS Accession No. ML041890281); August 31, 2005 (ADAMS Accession No. ML052510419); and April 6, 2007 (ADAMS Accession No. ML071010544). This letter provides a status of your response and describes any actions that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control room meets the design bases (e.g. General Design Criterion (GDC) 1, 3, 4, 5, & 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2003-01, Item 2).

You reported the results of American Society for Testing Materials (ASTM) E741, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution, tracer gas tests for the R.E. Ginna control room which is a neutral pressure control room with isolation and recirculation in the emergency mode.

You determined that the most limiting unfiltered inleakage into the Control Room Envelope (CRE) was 4 (+/- 9) cfm for Control Room Emergency Air Treatment System (CREATS) Trains A & B; 15 (+/- 6) cfm for CREATS Train A; and, -1 (+/- 8) cfm for CREATS Train B. All these values are less than the value of 300 cfm assumed in the design basis radiological analyses for Control Room Habitability.

You also provided information that adequately supported a conclusion that the most limiting unfiltered inleakage into the CRE is incorporated into the hazardous chemical assessments, and that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your Technical Specifications (TS) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for control room habitability, and in light of the demonstrated inadequacy of a delta ( $\Delta$ ) P measurement to alone provide such verification (GL 2003-01, Item 1c). In your April 6, 2007, response you stated that until such time that a new TS surveillance requirement is approved, you would perform a self assessment of the integrity of the control room boundary in 2008, followed by another inleakage test using ASTM E741 methodology in 2011 which is consistent with Regulatory Guide 1.197 "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors." In your April 6, 2007, response, you also committed to submit a licence amendment request (LAR) to adopt TS Task Force Traveler 448 (TSTF-448) consistent with your existing CREATS design by August 27, 2007.

The information you provided also supported the fact that there are no compensatory measures in place to demonstrate control room habitability and that the Unit conforms to the GDC regarding control room habitability. Your commitment to submit an LAR based on TSTF-448 and the information discussed above is acceptable for the purposes of closing out your response to GL 2003-01.

If you have any questions regarding this correspondence, please contact me at 301-415-1364.

Sincerely,

*/RA/*

Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-244

cc: See next page

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