



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
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May 8, 2007

Mr. James Shetler, Assistant General Manager
Energy Supply
Sacramento Municipal Utility District
6201 'S' Street
P.O. Box 15830
Sacramento, California 95852

SUBJECT: NRC INSPECTION REPORT 050-00312/07-002

Dear Mr. Shetler:

An NRC inspection was conducted on April 23 through 26, 2007, at your Rancho Seco Nuclear Generating Station. At the conclusion of the site visit, an exit briefing was conducted with the acting Plant Manager and other members of your staff. The enclosed report presents the scope and results of the inspection.

The inspection was an examination of activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. Areas inspected include self-assessment, auditing and corrective action; maintenance and surveillance; decommissioning performance and status review; and occupational radiation exposure. No violations were identified; therefore, no response to this letter is required.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/Adams.html>. To the extent possible, your response should not include any personal privacy, proprietary, or safeguards information so that it can be made available to the public without redaction.

Should you have any questions concerning this inspection, please contact Mr. Emilio Garcia, Health Physicist, at (530) 756-3910 or the undersigned at (817) 860-8191.

Sincerely,

/RA RJEvans for/

D. Blair Spitzberg, Ph.D., Chief
Fuel Cycle and Decommissioning Branch

Docket No.: 050-00312
License No.: DPR-54

Enclosure:
NRC Inspection Report
050-00312/07-002

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SUNSI Review Complete: EMG ADAMS: Yes No Initials: EMG
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U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket No.: 050-00312
License No.: DPR-54
Report No.: 050-00312/07-002
Licensee: Sacramento Municipal Utility District
Facility: Rancho Seco Nuclear Generating Station
Location: 14440 Twin Cities Road
Herald, California
Dates: April 23 through 26, 2007
Inspector: Emilio M. Garcia, Health Physicist
Approved By: D. Blair Spitzberg, Ph.D., Chief
Fuel Cycle and Decommissioning Branch
Attachments: Supplemental Information
Partial List of Documents Reviewed

EXECUTIVE SUMMARY

Rancho Seco Nuclear Generating Station
NRC Inspection Reports 050-00312/07-002

This inspection was a routine, announced inspection of decommissioning activities being conducted at the Rancho Seco Nuclear Generating Station. Areas inspected include self-assessment, auditing and corrective action; maintenance and surveillance; decommissioning performance and status review; and occupational radiation exposure.

Self-assessment, Auditing and Corrective Action

- The licensee was effectively maintaining its Corrective Action Program to control the identification, evaluation, and resolution of problems (Section 1.1).
- Audits conducted in calendar year 2006 and scheduled for calendar year 2007 addressed all facility activities required to be audited. All auditors met qualification requirements (Section 1.2).

Maintenance and Surveillances

- The licensee continued to maintain its liquid effluent monitor. Checks and calibrations of the liquid effluent monitor had been conducted at the required intervals (Section 2).

Decommissioning Performance and Status Review

- The licensee continued to dismantle and remove contaminated components and to remediate contaminated surfaces in a safe manner (Section 3).

Occupational Radiation Exposure

- A licensee audit of the occupational radiation exposure program was conducted in accordance with quality assurance program requirements. The licensee also had an effective program for identifying and correcting deficiencies or weaknesses related to the control of radiation and radioactive materials (Section 4.1).
- Changes made to the occupational radiation protection program did not decrease its effectiveness (Section 4.2).
- The licensee maintained an effective program for monitoring occupational radiation exposures. Occupational exposures for calendar year 2006 were below regulatory limits (Section 4.3).
- The licensee continued to maintain and implement an ALARA program including assigned responsibilities, procedures, training, planning, dose estimates and dose goals (Section 4.4).

Report Details

Summary of Facility Status

The Rancho Seco Nuclear Generating Station was permanently shut down in June 1989. All spent reactor fuel has since been moved to an onsite Independent Spent Fuel Storage Installation (ISFSI). At the time of this inspection, the licensee was conducting decommissioning under the provisions of the incremental decommissioning option of Rancho Seco's Post Shutdown Decommissioning Activities Report dated March 20, 1997.

Decommissioning activities included the auxiliary building, reactor building, spent fuel building and exterior areas. All major components in the auxiliary, reactor, and fuel handling building had been removed, packaged and shipped for disposal. In the reactor building, the removal of the remaining concrete and steel were underway.

1 Self-assessment, Auditing, and Corrective Action (IP 40801)

1.1 Identification, Evaluation, and Resolution of Problems

a. Inspection Scope

The inspector reviewed the licensee's administrative procedures that control the identification, evaluation, and resolution of problems.

b. Observations and Findings

The licensee's program for the resolution of non-conformances, material or programmatic deficiencies, and conditions adverse to quality or safety remained as described in Section 1.2 of Inspection Report 050-00312/2006-002. There were no changes to the procedures of the Rancho Seco Corrective Action Program since this program area was last inspected in June 2006.

During calendar year 2006, 31 Potential Deviation from Quality (PDQ) reports were initiated and 8 were determined to be a Deviation from Quality (DQ). As of April 26, 2007, eight PDQs had been initiated during the calendar year with five determined to be DQs. No Corrective Action Requests or Stop Work Orders had been initiated in 2006 or 2007. The inspector reviewed the list of PDQs and DQs that remained open. There were 14 open items, including 4 opened in 2007. The oldest item was from calendar year 2005. The inspector concluded that the licensee was appropriately addressing the timely resolution of these open items.

The inspector reviewed agendas and minutes of Commitment Management Review Group (CMRG) meetings and noted that the CMRG met as least once a month as required by procedure Rancho Seco Administrative Procedure RSAP-0260, Commitment Management Review Group & Compliance Management Tracking Systems. Group membership was as described in this procedure. The records indicate that the CMRG was conducting initial reviews and characterizations of new PDQs. The CMRG was also assigning tasks, establishing priorities and reviewing proposed

resolutions for PDQs and other identified problems. The inspector concluded that the licensee was effectively maintaining its Rancho Seco Corrective Action Program to control the identification, evaluation, and resolution of problems.

c. Conclusion

The licensee was effectively maintaining its Corrective Action Program to control the identification, evaluation, and resolution of problems.

1.2 Quality Assurance Audit Organization, Staffing, and Qualifications

a. Inspection Scope

The inspector reviewed the conduct of Quality Assurance (QA) audits and surveillances and the status of the QA audit organization, staffing, and qualifications.

b. Observations and Findings

The licensee's QA audit organization and staffing had changed since the last inspection, conducted in June 2006. Two of the lead auditors had left the organization and had been replaced by one full time auditor. Additional auditors were brought in for special audits as needed. Records indicate that all audits conducted in 2006 and 2007 were led by auditors with current lead auditor certifications.

The inspector reviewed records of audits conducted since the last inspection in June 2006. The licensee had conducted 11 audits in calendar year 2006 and 2 audits in calendar year 2007. Two additional audits were in progress during the inspection. A total of 15 audits were scheduled to be performed in calendar year 2007. Audits conducted in calendar years 2006-2007, and those planned for the remainder of 2007, addressed all 34 facility activities listed in Rancho Seco Quality Manual (RSQM), Section XVIII.

In calendar year 2006, the licensee conducted 27 surveillances. As of April 23, 2007, the licensee had conducted 13 surveillances during calendar year 2007. The inspector selected audit reports 06-A-008, 06-A-009, 07-A-001 and 07-A-002 and surveillance reports 06-S-012, 06-S-017, 06-S-019, 06-S-26, 06-S-27, 07-S-004, 07-S-006, and 07-S-011 for review. The inspector confirmed that the audits and surveillances were conducted in accordance with the RSQM. The individuals that conducted the audits and surveillances were independent of the areas being audited. The audits used approved checklists. The team personnel were qualified and were authorized to perform the audits or surveillances in the areas audited. The audits and surveillances were conducted in a timely manner.

c. Conclusion

Audits conducted in calendar year 2006 and scheduled for calendar year 2007 addressed all facility activities required to be audited. All auditors met qualification requirements.

2 Maintenance and Surveillance (IP 62801)

2.1 Inspection Scope

The inspector reviewed the status of required surveillances and testing. The inspector interviewed the Maintenance Superintendent and reviewed selected records.

2.2 Observations and Findings

With the relocation of the spent fuel to the ISFSI, the licensee no longer had any safety-related structures, Systems or Components (SSC) as defined in 10 CFR 50.65(b)(1), nor any non-safety-related SSC as defined in 10 CFR 50.65(b)(2). The licensee previously reviewed its Maintenance Rule procedure and concluded that it was no longer required to be implemented at the site.

The inspector reviewed the maintenance of liquid effluent radiation monitor, R-15017A. The licensee performed quarterly tests and annual calibrations of this monitor. Records maintained by the licensee indicate that the licensee had performed these surveillances at the required intervals.

The inspector reviewed a printout of the Rancho Seco Computerized Surveillance Schedule dated April 24, 2007. This document listed the status of surveillances and routine tests. Twelve routine tests or surveillances were beyond their due dates but none were beyond their grace period. The licensee did not expect that any of these surveillances or tests would exceed their grace period.

2.3 Conclusion

The licensee continued to maintain its liquid effluent monitor. Checks and calibrations of the liquid effluent monitor had been conducted at the required intervals.

3 Decommissioning Performance and Status Review (IP 71801)

3.1 Inspection Scope

The inspector interviewed cognizant personnel, reviewed selected documents and toured portions of the site to observe work activities including housekeeping, safety practices, fire loading and radiological controls.

3.2 Observations and Findings

The inspector conducted tours of the reactor, auxiliary, fuel handling, and turbine buildings to observe dismantling and decommissioning activities in progress. The work was being conducted in a safe and orderly manner. The inspector conducted independent radiological surveys using a Ludlum Model 2401-EC survey meter (NRC No. 21176G, calibration due date August 4, 2007). Radiological controls, including postings and barriers, were in place. Good housekeeping and fire protection practices were noted in areas toured.

The reactor vessel segmentation project had been completed. Twenty-one sections of the reactor vessel had been cut, packaged and shipped to a low level radioactive waste disposal site.

The reactor building concrete and steel removal project was in progress. As described in Inspection Report 050-00312/2007-01, this project involved the removal of 34,541,000 pounds of contaminated concrete and reinforcing steel from the reactor building. The reactor building concrete and steel removal project was expected to be completed by March 27, 2008.

The licensee estimated that the final waste water basin release will occur by June 4, 2007. The licensee has begun the dismantlement of its waste water system. Removal of waste water basin concrete was scheduled to take place between June to August, 2007. The licensee plans to remove between 1.5 to 2.0 million pounds of concrete from the basins. The completion of this project was scheduled for late Fall of 2007.

The licensee plans to complete decommissioning by June 2008.

3.3 Conclusion

The licensee continued to dismantle and remove contaminated components and to remediate contaminated surfaces in a safe manner.

4 **Occupational Radiation Exposure (IP 83750)**

4.1 Audits and Surveillances

a. Inspection Scope

The inspector reviewed the recently conducted radiation safety audit report to verify implementation of the commitments made in Section XVIII, Audits, of the RSQM, as it relates to occupational radiation safety. The inspector reviewed the qualification records for the individuals involved in the audit. The inspector also reviewed Radiation Protection Occurrence (RPO) reports for 2006 and 2007.

b. Observations and Findings

The inspector reviewed audit Report 06-A-008, Radiological Safety and Control and ALARA Program. The audit was conducted November 22 through December 21, 2006, and the report was issued on January 3, 2007. The inspector confirmed that the audit was conducted according to the commitments in the RSQM. The individual that conducted the audit was independent of the function being audited. The audit included the use of an approved checklist. The lead auditor was qualified and authorized to perform the audit in the areas audited. The other two auditors had current qualification records each with over 30 years of experience in the areas audited as well as professional certifications. The audit did not identify any item that constituted a PDQ, but the auditors identified five recommendations. The audit was conducted in a timely manner.

The Radiation Protection Occurrence (RPO) Reports were used by the licensee to document the identification, immediate actions, investigation, and corrective actions of any radiological deficiency or violation noted by any plant worker. The RPO Report does not replace the PDQ process. In some cases, the RPO can be the basis for initiating a PDQ. Procedure RP.305.36, "Radiological Protection Occurrence Report," was the procedure controlling this process.

The inspector reviewed the RPO log and noted that there had been three RPOs initiated in 2006 and one in 2007. These RPOs had been processed in accordance with the requirements of procedure RP.305.36. For each of these RPOs, the licensee had investigated the occurrence and taken corrective actions.

c. Conclusions

A licensee audit of the occupational radiation exposure program was conducted in accordance with QA program requirements. The licensee also had an effective program for identifying and correcting deficiencies or weaknesses related to the control of radiation and radioactive materials.

4.2 Changes

a. Inspection Scope

The inspector reviewed major changes made since the last inspection in the areas of organization, personnel, facilities, equipment, programs and procedures to determine if these changes negatively affected occupational radiation protection.

b. Observations and Findings

There had been no organizational changes since this area was last inspected in June 2006. The licensee was developing a number of procedures for implementing various milestones as the decommissioning progressed.

The number of Radiation Work Permits (RWP) had decreased from 12 in 2006 to 8 in 2007, as of April 24, 2007. This change in the number of RWPs was due to completion of special projects and changes in radiological conditions due to remediation activities. At the time of the inspection, seven RWPs remained active.

The licensee had increased by one the number of contract technicians employed to reflect the work being performed. The site had received additional funding for contract technicians to cover work through June 1, 2009, if needed. The licensee purchased and placed into service two new instruments to support radioactive waste surveys. The licensee was in the process of generating calibration procedures for these instruments. Until the new procedures are implemented, calibrations would be performed as per the manufacturer's manual as permitted by procedure RP.311, Radiation Protection Instrument Control Program.

Radiation protection procedures revised in 2006 or 2007 that were reviewed by the inspector are listed below:

Number	Title	Rev	Effective
RP.305	Radiation Protection Plan	10	06/28/2006
RP.305.08A	Routine and Radiation Work Permit Surveys	6	01/29/2007
RP.305.09B	Personnel Contamination Monitoring	6	06/20/2006
RP.305.09D	Personnel and Clothing Decontamination and Reports	7	06/20/2006
RP.305.16	Radioactive Materials Receipts	5	10/17/2006
RP.311.VI.03	Counting Equipment Voltage Plateau	1	08/15/2006

The inspector concluded that these changes would not have a negative effect on occupational radiation protection.

c. Conclusions

Changes made to the occupational radiation protection program did not decrease its effectiveness.

4.3 External and Internal Exposure Control and Other Radiation Protection Inspection Areas

a. Inspection Scope

The licensee's personnel radiation monitoring program was inspected for compliance with applicable requirements and commitments.

b. Observations and Findings

The licensee was continuing to use Optically Stimulated Luminescent (OSL) dosimeters for evaluating beta/gamma external doses, and neutron dosimeters for neutron dose. The dosimeters were provided by a vendor which was accredited under the National Voluntary Laboratory Accreditation Program (NVLAP) for the type of dosimeters used. In addition, the licensee used Electronic Dosimeter (EDs) for controlling the day-to-day personnel exposures. The licensee continued to use a vendor-supplied computerized dose tracking system for reading the EDs and for automatically assigning the estimated dose to the individual.

The Radiological Health Supervisor stated that during calendar year 2006 no individual had received an internal exposure that required a committed effective dose equivalent be assigned.

During calendar years 2006 and 2007, no individual had been classified as a declared pregnant worker, and no planned special exposures had been conducted.

The annual report for 2006 radiation exposures required by 10 CFR 20.2206(b) was timely submitted on April 23, 2007. This report indicated that the Total Effective Dose Equivalent (TEDE) received by occupationally exposed individuals was below the regulatory limit of 5 rem. The highest reported TEDE was 1.920 rem during calendar year 2006. The lens of the eye, skin of the whole body, internal, and skin of the maximally exposed extremity doses were all below applicable limits.

The Regulatory Guide 1.16 Annual Exposure Report for calendar year 2006 was submitted on March 15, 2007. The report was submitted in a timely manner.

c. Conclusions

The licensee maintained an effective program for monitoring occupational radiation exposures. Occupational exposures for calendar year 2006 were below regulatory limits.

4.4 Maintaining Occupational Exposures ALARA

a. Inspection Scope

The inspector discussed the licensee's program for maintaining occupational radiation exposures As Low As Reasonably Achievable (ALARA) with the Supervising Radiation Engineering Specialist, and the inspector reviewed the licensee's applicable organization, procedures, goals and objectives, worker awareness and involvement.

b. Observations and Findings

Procedure RP 0305.04, Radiation Work Permits, requires dose estimates for all RWPs. The procedure also requires that an ALARA job planning meeting be conducted for all RWPs that have a dose estimate of greater than 1.0 person-rem, unless all exposures were expected from multiple low-dose entries and the radiation protection supervision had concluded that an ALARA job planning meeting would not significantly reduce radiation exposures.

The inspector reviewed the list of RWPs for calendar years 2006 and 2007 and noted that dose estimates had been prepared for each RWP. The licensee decided not to conduct ALARA job planning meetings for any of the 2007 RWPs because all exposures were expected to be from multiple low-dose entries, and the radiation protection supervisor had concluded that an ALARA job planning meeting would not significantly reduce radiation exposures. The ALARA estimate for calendar year 2007 was 20.3 Person-Rem, with a 17.2 Person-Rem goal.

The inspector reviewed a memorandum dated February 7, 2007, from the Supervising Radiation Engineering Specialist to the CMRG. This memorandum reported the 2006 personnel radiation exposures and compared them to the ALARA estimates and goals. The ALARA estimate for 2006 was 24.5 Person-Rem with a goal of 20.8 Person-Rem.

The actual exposure was 30.1 Person-Rem. The actual exposure was 37.2% greater than estimated. This higher exposure was due to required clean up work not previously expected, more maintenance on equipment than anticipated and more hours required for compacting radioactive waste. These results were presented to the ALARA Committee during their February 14, 2007, meeting. The ALARA committee accepted the report and concluded that the additional exposures received were due to the unique nature of the equipment required for the segmentation projects, and that the lessons learned from these projects were not applicable to the reactor building concrete and steel removal project that was in progress.

Both the Site Access (CAT I) and Radiological Controlled Area Access (CAT II) Training included modules on ALARA. During tours of the radiologically controlled areas, the inspector observed ALARA postings and observed individuals using good ALARA practices.

c. Conclusions

The licensee continued to maintain and implement the ALARA program including assigned responsibilities, procedures, training, planning, dose estimates and dose goals.

5 Exit Meeting Summary

The inspector presented the inspection results to the acting plant manager and other members of licensee staff at the exit meeting on April 26, 2007. The licensee did not identify as proprietary any information provided to, or reviewed by, the inspector.

ATTACHMENT 1

PARTIAL LIST OF PERSONS CONTACTED

Sacramento Municipal Utility District

M. Bua, Radiation Protection/Chemistry Superintendent
J. Field, Engineering Superintendent
W. Hawley, Dismantlement Superintendent - Operations, Acting Plant Manager
G. Howard, Project Manager
R. Jones, Supervising Quality Engineer
D. Koontz, ISFSI Supervisor
S. Nicolls, Radiological Health Supervisor
S. Porterfield, Supervising Radiation Engineering Specialist
S. Redeker, Manager Plant Closure and Decommissioning
G. Roberts, Maintenance Superintendent
E. Ronningen, Dismantlement Superintendent - Radiological

INSPECTION PROCEDURES USED

IP 40801	Self-assessment, Auditing and Corrective Action
IP 62801	Maintenance and Surveillance
IP 71801	Decommissioning Performance and Status Review
IP 83750	Occupational Radiation Exposure

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS

ALARA	As Low As Reasonably Achievable
CAT	Category as in CAT I training
CFR	Code of Federal Regulations
CMRG	Commitment Management Review Group
DQ	Deviation from Quality
ED	Electronic Dosimeter
IP	Inspection Procedure
ISFSI	Independent Spent Fuel Storage Installation
NVLAP	National Voluntary Laboratory Accreditation Program
OSL	Optically Stimulated Luminescent Dosimeter
PDQ	Potential Deviation from Quality
RP	Radiation Protection
RPO	Radiation Protection Occurrences
RSAP	Rancho Seco Administrative Procedure
RSQM	Rancho Seco Quality Manual
RWP	Radiation Work Permit
SSC	Structures, Systems or Components
TEDE	Total Effective Dose Equivalent

ATTACHMENT 2

PARTIAL LIST OF DOCUMENTS REVIEWED

Audits and Surveillances

- 2006 Surveillance Log.
- 2007 Surveillance Log as of April 23, 2007.
- 2007 Quality Audit Schedule, NQA 07-009 dated February 26, 2007.
- 2006 Quality Audit Log.
- 2007 Quality Audit Log as of April 23, 2007.
- Auditor/Surveillance Qualification records for Leon Brown, Mike Murdock, and John Newey.
- Lead Auditor Certification records for Robert E. Jones, Michael L. Braum, Glenn Howard, Richard Mannheimer and Wayne Thomas.
- Rancho Seco Audit Report No. 06-A-008, Radiological Safety and Control and ALARA Program, conducted November 22 through December 21, 2006, and the report was issued January 3, 2007.
- Rancho Seco Audit Report No. 06-A-009, Control and Accountability of Special Nuclear Materials, conducted November 20 through December 20, 2006, and the report was issued on January 17, 2007.
- Rancho Seco Audit Report No. 07-A-001, Off-Site Calculation Manual (ODCM), conducted February 26 through March 13, 2007, and the report was issued on March 14, 2007.
- Rancho Seco Audit Report No. 07-A-002, Procurement Document Control and Document Control, conducted March 22 through April 9, 2007, and the report was issued on April 16, 2007.
- Surveillance Report 06-S-012, Objective: Verify that surveys in the upper parking lot area were performed in accordance with the reference instructions and procedures. Surveillance period May 17, 2006. Report date May 30, 2006.
- Surveillance Report 06-S-017, Objective: Tour the rail car facility at the Energy Solutions burial site. Surveillance period July 20, 2006. Report date August 3, 2006.
- Surveillance Report 06-S-019, Objective: Tour the IMPACT Services facility at Oak Ridge, TN. Surveillance period July 26, 2006. Report date August 3, 2006.
- Surveillance Report 06-S-026, Objective: Verify that the radioactive waste shipment meets DOT and SMUD requirements prior to departure from Rancho Seco. Surveillance period November 13, 2006. Report date November 11, 2006.

- Surveillance Report 06-S-027, Objective: Verify that information is being communicated from the Security Operations Supervisor to the security officers in the field. Surveillance period November 29 and 30, 2006. Report date December 7, 2006.
- Surveillance Report 07-S-004, Objective: Verify that the annual training requirement for a behavior observation program as outlined in RSAP-1003, Access Screening Requirements, is being complied with. Surveillance period January 2 through 30, 2007. Report date February 1, 2007.
- Surveillance Report 07-S-006, Objective: Verify the implementation of 10 CFR Part 50 License Conditions 2.B.1, 2.B.3 and 2.B.4. Surveillance period February 12, 2007. Report date February 12, 2007.
- Surveillance Report 07-S-011, Objective: Verify that the radioactive waste shipment meets DOT and SMUD requirements prior to departure from Rancho Seco. Surveillance period March 29, 2007. Report date April 2, 2007.

Data Sheets

- Potential Deviation from Quality Log, 2006 and 2007 through April 23, 2007.
- 2006 RWPs list as of September 26, 2006.
- 2007 RWPs list as of April 9, 2007.

Meeting Minutes

- NQA 07-018, CMRG Meeting Held on April 4, 2007.
- NQA 07-013, CMRG Meeting Held on March 14, 2007.
- NQA 07-008, CMRG Meeting Held on February 14, 2007.
- NQA 07-001, CMRG Meeting Held on January 10, 2007.
- NQA 06-050, CMRG Meeting Held on December 20, 2006.
- NQA 06-048, CMRG Meeting Held on December 14, 2006.
- NQA 06-043, CMRG Meeting Held on November 8, 2006.
- NQA 06-034, CMRG Meeting Held on August 23, 2006.

Procedures

- Rancho Seco Administrative Procedure RSAP-0260, Commitment Management Review Group & Compliance Management Tracking System, Revision 13, effective January 9, 2006.
- Radiation Control Manual Procedure RP.305, Radiation Protection Plan, Revision 10, effective June 28, 2006.
- Radiation Control Manual Procedure RP.305.04, Radiation Work Permits, Revision 10, effective June 12, 2003.
- Radiation Control Manual Procedure RP.305.08A, Routine and Radiation Work Permit Surveys, Revision 6, effective January 29, 2007.
- Radiation Control Manual Procedure RP.305.09B, Personnel Contamination Monitoring, Revision 6, effective June 20, 2006.
- Radiation Control Manual Procedure RP.305.09D, Personnel and Clothing Decontamination and Reports, Revision 7, effective June 20, 2006.
- Radiation Control Manual Procedure RP.305.16, Radioactive Materials Receipts, Revision 5, effective October 17, 2006.
- Radiation Control Manual Procedure RP.311.VI.03, Counting Equipment Voltage Plateau, Revision 1, effective August 15, 2006.

Memorandums

- RPC 07-006, Annual ALARA Report for 2006, dated February 7, 2007.
- RPC 07-007, 2007 Radiation Exposure Estimate and ALARA Goals, dated February 7, 2007.

Reports

- Regulatory Guide 1.16 Annual Exposure Report (for Calendar Year 2006), NQA 07-014, March 15, 2007.
- 10 CFR 20.2206 Annual Report of Individual Monitoring (for Calendar Year 2006), NQA 07-020, April 23, 2007.