

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
1.3		Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Boiling Water Reactors	
1.4		Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Pressurized Water Reactors	
1.11		Instrument Lines Penetrating Primary Reactor Containment	
1.12		Nuclear Power Plant Instrumentation for Earthquakes	
1.16		Reporting of Operating Information -- Appendix A Technical Specifications	
1.25		Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors	
1.28		Quality Assurance Program Requirements (Design and Construction)	
1.30		Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment	
1.34		Control of Electroslag Weld Properties	
1.35		Inservice Inspection of UngROUTED Tendons in Prestressed Concrete Containments	
1.35.1		Determining Prestressing Forces for Inspection of Prestressed Concrete Containments	
1.39		Housekeeping Requirements for Water-Cooled Nuclear Power Plants	
1.43		Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components	
1.44		Control of the Use of Sensitized Stainless Steel	
1.50		Control of Preheat Temperature for Welding of Low-Alloy Steel	
1.65		Materials and Inspections for Reactor Vessel Closure Studs	
1.68.1		Preoperational and Initial Startup Testing of Feedwater and Condensate Systems for Boiling Water Reactor Power Plants	
1.68.2		Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants	
1.68.3		Preoperational Testing of Instrument and Control Air Systems	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
1.72		Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin	
1.78		Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release	
1.79		Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors	
1.96		Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants	
1.98		Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor	
1.99		Radiation Embrittlement of Reactor Vessel Materials	
1.101		Emergency Planning and Preparedness for Nuclear Power Reactors	
1.106		Thermal Overload Protection for Electric Motors on Motor-Operated Valves	
1.109		Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I	
1.110		Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors	
1.111		Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors	
1.115		Protection Against Low-Trajectory Turbine Missiles	
1.118		Periodic Testing of Electric Power and Protection Systems	
1.126		An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification	
1.133		Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors	
1.134		Medical Evaluation of Licensed Personnel at Nuclear Power Plants	
1.139		Guidance for Residual Heat Removal	
1.141		Containment Isolation Provisions for Fluid Systems	
1.143		Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants	
1.145		Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
1.148		Functional Specification for Active Valve Assemblies in Systems Important to Safety in Nuclear Power Plants	
1.154		Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors	
1.157		Best-Estimate Calculations of Emergency Core Cooling System Performance	
1.159		Assuring the Availability of Funds for Decommissioning Nuclear Reactors	
1.160		Monitoring the Effectiveness of Maintenance at Nuclear Power Plants	
1.166		Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions	
1.167		Restart of a Nuclear Power Plant Shut Down by a Seismic Event	
1.179		Standard Format and Content of License Termination Plans for Nuclear Power Reactors	
1.180		Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems	
1.181		Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)	
1.186		Guidance and Examples for Identifying 10 CFR 50.2 Design Bases	
1.187		Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments	
1.193		ASME Code Cases Not Approved for Use	
1.198		Procedures and Criteria for Assessing Seismic Soil Liquefaction at Nuclear Power Plant Sites	
1.199		Anchoring Components and Structural Supports in Concrete	
1.202		Standard Format and Content of Decommissioning Cost Estimates for Nuclear Power Reactors	
1.203		Transient and Accident Analysis Methods	
New		Regulatory Guide on High-Energy Line Leak-Before-Break Methodology	
New		Issuance of a new final regulatory guide (RG) for assessing seismic soil liquefaction at nuclear power plant sites.	
New		Proposed RG on spent fuel pool criticality.	
New		Proposed RG on implementation of voluntary 10 CFR 50.61 entry criteria.	
New		Installation of Transducers (IC 131-5)	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
2.1		Shield Test Program for Evaluation of Installed Biological Shielding in Research and Training Reactors	
2.2		Development of Technical Specifications for Experiments in Research Reactors	
2.3		Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in Research Reactors	
2.4		Review of Experiments for Research Reactors	
2.5		Quality Assurance Program Requirements for Research Reactors	
2.6		Emergency Planning for Research and Test Reactors	
3.11		Design, Construction, and Inspection of Embankment Retention Systems for Uranium Mills	
3.11.1		Operational Inspection and Surveillance of Embankment Retention Systems for Uranium Mill Tailings	
3.15		Standard Format and Content of License Applications for Storage Only of Unirradiated Power Reactor Fuel and Associated Radioactive Material	
3.19		Reporting of Operating Information for Fuel Reprocessing Plants	
3.20		Process Offgas Systems for Fuel Reprocessing Plants	
3.21		Quality Assurance Requirements for Protective Coatings Applied to Fuel Reprocessing and to Plutonium Processing and Fuel Fabrication Plants	
3.22		Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	
3.27		Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel Reprocessing Plants	
3.28		Welder Qualification for Welding in Areas of Limited Accessibility in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants	
3.29		Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for Use in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants	
3.30		Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel Reprocessing Plants	
3.37		Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic Stainless Steel Components of Fuel Reprocessing Plants	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
3.44		Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation (Water-Basin Type)	
3.48		Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation or Monitored Retrievable Storage Installation (Dry Storage)	
3.49		Design of an Independent Spent Fuel Storage Installation (Water-Basin Type)	
3.50		Standard Format and Content for a License Application To Store Spent Fuel and High-Level Radioactive Waste	
3.51		Calculational Models for Estimating Radiation Doses to Man from Airborne Radioactive Materials Resulting from Uranium Milling Operations	
3.54		Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation	
3.56		General Guidance for Designing, Testing, Operating, and Maintaining Emission Control Devices at Uranium Mills	
3.59		Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium Milling Operations	
3.60		Design of an Independent Spent Fuel Storage Installation (Dry Storage)	
3.61		Standard Format and Content for a Topical Safety Analysis Report for a Spent Fuel Dry Storage Cask	
3.62		Standard Format and Content for the Safety Analysis Report for Onsite Storage of Spent Fuel Storage Casks	
3.63		Onsite Meteorological Measurement Program for Uranium Recovery Facilities - Data Acquisition and Reporting	
3.64		Calculation of radon Flux Attenuation by Earthen Uranium Mill Tailings Covers	
3.69		Topical Guidelines for the Licensing Support Network	
3.72		Guidance for Implementation of 10 CFR 72.48, Changes, Tests, and Experiments	
3.73		Site Evaluations and Design Earthquake Ground Motion for Dry Cask Independent Spent Fuel Storage and Monitored Retrievable Storage Installations	
4.2S1		Supplement 1 to Regulatory Guide 4.2, Preparation of Supplemental Environmental Reports for Applications To Renew Nuclear Power Plant Operating Licenses	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
4.5		Measurements of Radionuclides in the Environment-- Sampling and Analysis of Plutonium in Soil	
4.6		Measurements of Radionuclides in the Environment-- Strontium-89 and Strontium-90 Analyses	
4.8		Environmental Technical Specifications for Nuclear Power Plants	
4.13		Performance, Testing, and Procedural Specifications for Thermoluminescence Dosimetry: Environmental Applications	
5.10		Selection and Usage of Pressure-Sensitive Seals on Containers for Onsite Storage of Special Nuclear Material	
6.3		Design, Construction, and Use of Radioisotopic Power Generators for Certain Land and Sea Applications	
6.6		Acceptance Sampling Procedures for Exempted and Generally Licensed Items Containing Byproduct Material	
6.7		Preparation of an Environmental Report To Support a Rule Making Petition Seeking an Exemption for a Radionuclide-Containing Product	
6.8		Identification Plaque for Irretrievable Well-Logging Sources	
7.1		Administrative Guide for Packaging and Transporting Radioactive Material	
7.2		Packaging and Transportation of Radioactively Contaminated Biological Materials	
7.3		Procedures for Picking Up and Receiving Packages of Radioactive Material	
7.4		Leakage Tests on Packages for Shipment of Radioactive Materials	
7.5		Administrative Guide for Obtaining Exemptions from Certain NRC Requirements over Radioactive Material Shipments	
7.6		Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels	
7.7		Administrative Guide for Verifying Compliance with Packaging Requirements for Shipments of Radioactive Materials	
7.8		Load Combinations for the Structural Analysis of Shipping Casks for Radioactive Material	
7.9		Standard Format and Content of Part 71 Applications for Approval of Packages for Radioactive Material	

Regulatory Guides Update Program - Phase 3

RG	DG	Title	Status
7.10		Establishing Quality Assurance Programs for Packaging Used in Transport of Radioactive Material	
7.11		Fracture Toughness Criteria of Base Material for Ferritic Steel Shipping Cask Containment Vessels with a Maximum Wall Thickness of 4 Inches (0.1 m)	
7.12		Fracture Toughness Criteria of Base Material for Ferritic Steel Shipping Cask Containment Vessels with a Wall Thickness Greater Than 4 Inches (0.1 m) But Not Exceeding 12 Inches (0.3 m)	
8.1		Radiation Symbol	
8.5		Criticality and Other Interior Evacuation Signals	
8.9		Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program	
8.13		Instruction Concerning Prenatal Radiation Exposure	
8.15		Acceptable Programs for Respiratory Protection	
8.18		Information Relevant to Ensuring that Occupational Radiation Exposures at Medical Institutions Will Be as Low as Is Reasonably Achievable	
8.20		Applications of Bioassay for I-125 and I-131	
8.21		Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and Manufacturing Plants	
8.23		Radiation Safety Surveys at Medical Institutions	
8.25		Air Sampling in the Workplace	
8.29		Instruction Concerning Risks from Occupational Radiation Exposure	
8.32		Criteria for Establishing a Tritium Bioassay Program	
8.33		Quality Management Program	
8.34		Monitoring Criteria and Methods To Calculate Occupational Radiation Doses	
8.35		Planned Special Exposures	
8.36		Radiation Dose to the Embryo/Fetus	
8.37		ALARA Levels for Effluents from Materials Facilities	
8.38		Control of Access to High and Very High Radiation Areas of Nuclear Plants	
8.39		Release of Patients Administered Radioactive Materials	
10.1		Compliance of Reporting Requirements for Persons Subject to NRC Regulations	
10.12		Preparation of Petitions for Rulemaking Under 10 CFR 2.802 and Preparation and Submission of Proposals for Regulatory Guidance Documents	