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RG	DG	Title	Status
1.1		Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps	
1.6		Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems	
1.8		Qualification and Training of Personnel for Nuclear Power Plants	
1.21		Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants	
1.22		Periodic Testing of Protection System Actuation Functions	
1.27		Ultimate Heat Sink for Nuclear Power Plants	
1.40		Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants	
1.45	1173	Reactor Coolant Pressure Boundary Leakage Detection Systems	Publish DG for Comments Aug 07
1.47		Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems	
1.54		Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants	
1.59		Design Basis Floods for Nuclear Power Plants	
1.62		Manual Initiation of Protective Actions	
1.63		Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants	
1.69		Concrete Radiation Shields for Nuclear Power Plants	
1.70		Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)	
1.73		Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants	
1.77		Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors	
1.82		Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident	
1.84	1133	Design and Fabrication and Materials Code Case Acceptability, ASME Section III	Comment Period Closed Jan 07
1.89		Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants	
1.90		Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons	
1.93		Availability of Electric Power Sources	
1.100		Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power Plants	
1.102		Flood Protection for Nuclear Power Plants	
1.105	1141	Setpoints for Safety-Related Instrumentation	
1.107		Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures	
1.113		Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I	
1.114		Guidance to Operators at the Controls and to Senior Operators in the Control Room of a Nuclear Power Unit	

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RG	DG	Title	Status
1.121		Bases for Plugging Degraded PWR Steam Generator Tubes	
1.122		Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components	
1.125		Physical Models for Design and Operation of Hydraulic Structures and Systems for Nuclear Power Plants	
1.127		Inspection of Water-Control Structures Associated with Nuclear Power Plants	
1.131	1132	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants	Publish DG for Comments Jun 07
1.135		Normal Water Level and Discharge at Nuclear Power Plants	
1.137		Fuel-Oil Systems for Standby Diesel Generators	
1.147	1134	Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1	Comment Period Closed Jan 07
1.149		Nuclear Power Plant Simulation Facilities for Use in Operator Training and License Examinations	
1.151		Instrument Sensing Lines	
1.153		Criteria for Safety Systems	
1.155		Station Blackout	
1.156	1147	Environmental Qualification of Connection Assemblies for Nuclear Power Plants	
1.158		Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants	
1.163		Performance-Based Containment Leak-Test Program	
1.168		Verification, Validation, Reviews, and Audits for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.169		Configuration Management Plans for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.170		Software Test Documentation for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.171		Software Unit Testing for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.172		Software Requirements Specifications for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.173		Developing Software Life Cycle Processes for Digital Computer Software Used in Safety Systems of Nuclear Power Plants	
1.174		An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis	
1.175		An Approach for Plant-Specific, Risk-Informed Decisionmaking: Inservice Testing	
1.177		An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications	
1.178		An Approach for Plant-Specific Risk-Informed Decisionmaking for Inservice Inspection of Piping	
1.183		Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors	
1.192		Operation and Maintenance Code Case Acceptability, ASME OM Code	
1.193	1135	ASME Code Cases Not Approved for Use	Comment Period Closed Jul 06

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RG	DG	Title	Status
1.200		An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities	
	1136	Demonstrating the Feasibility and Reliability of Operator Manual Actions in Response to Fire	
	1138	(Proposed Appendix C to Regulatory Guide 1.200) NRC Staff Regulatory Position on ANS External Hazards PRA Standard	Comment Period Closed
	1148	Qualification of Safety-Related Battery Chargers and Inverters	Publish DG for Comments Aug 07
	1149	Qualification of Safety-Related Motor Control Centers	
	1150	Qualification of Safety-Related Motors for Nuclear Power Plants	
	1151	An Approach for Plant-Specific, Risk-Informed Decisionmaking for Digital Systems	
New		Response-Time Testing of Protection System Instrument Channels (IC 121-5)	
New		Guidance on Performing Seismic Margin Analysis for Advanced Reactors in the Context of SECY-93-087	
New		Guidance on Containment Performance and Containment Fragility Assessment for Advanced Reactors in the Context of SECY-93-087 and Input for PRA Review	
New		Proposed RG on performance of non-Appendix VIII UT examination"	
New		Proposed RG on inservice testing of pumps and valves for U.S. light-water reactors	
New		Preferred Power Supply	
New		Sizing Large Lead Storage Batteries	
New		Installation of Valve Regulated Batteries	
New		Maintenance and Testing of Valve Regulated Batteries	
New		Fatigue Management at Nuclear power Plants	
3.3		Quality Assurance Program Requirements for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants	
3.5	3024	Standard Format and Content of License Applications for Uranium Mills	
3.6		Content of Technical Specifications for Fuel Reprocessing Plants	
3.7		Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel Fabrication Plants	
3.8	3025	Preparation of Environmental Reports for Uranium Mills	
3.10		Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel Fabrication Plants	
3.12		General Design Guide for Ventilation Systems of Plutonium Processing and Fuel Fabrication Plants	
3.13		Guide for Acceptable Waste Storage Methods at UF6 Production Plants	
3.14		Seismic Design Classification for Plutonium Processing and Fuel Fabrication Plants	
3.16		General Fire Protection Guide for Plutonium Processing and Fuel Fabrication Plants	
3.17		Earthquake Instrumentation for Fuel Reprocessing Plants	
3.18		Confinement Barriers and Systems for Fuel Reprocessing Plants	

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RG	DG	Title	Status
3.25		Standard Format and Content of Safety Analysis Reports for Uranium Enrichment Facilities	
3.26		Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing Plants	
3.31		Emergency Water Supply Systems for Fuel Reprocessing Plants	
3.32		General Design Guide for Ventilation Systems for Fuel Reprocessing Plants	
3.38		General Fire Protection Guide for Fuel Reprocessing Plants	
3.39		Standard Format and Content of License Applications for Plutonium Processing and Fuel Fabrication Plants	
3.40		Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants	
3.42		Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR Parts 50 and 70	
3.46	3026	Standard Format and Content of License Applications, Including Environmental Reports, for In Situ Uranium Solution Mining	
3.52		Standard Format and Content for the Health and Safety Sections of License Renewal Applications for Uranium Processing and Fuel Fabrication	
3.53		Applicability of Existing Regulatory Guides to the Design and Operation of an Independent Spent Fuel Storage Installation	
3.55		Standard Format and Content for the Health and Safety Sections of License Renewal Applications for Uranium Hexafluoride Production	
3.67		Standard Format and Content for Emergency Plans for Fuel Cycle and Materials Facilities	
3.71		Nuclear Criticality Safety Standards for Fuels and Material Facilities	
4.1		Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants	
4.2		Preparation of Environmental Reports for Nuclear Power Stations	
4.4		Reporting Procedure for Mathematical Models Selected To Predict Heated Effluent Dispersion in Natural Water Bodies	
4.7		General Site Suitability Criteria for Nuclear Power Stations	
4.9		Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	
4.11		Terrestrial Environmental Studies for Nuclear Power Stations	
4.14	4011	Radiological Effluent and Environmental Monitoring at Uranium Mills	
4.15		Quality Assurance for Radiological Monitoring Programs (Normal Operations) -- Effluent Streams and the Environment	Comment Period Closed May 07
4.16		Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in Liquid and Gaseous Effluents from Nuclear Fuel Processing and Fabrication Plants and Uranium Hexafluoride Production Plants	
4.17		Standard Format and Content of Site Characterization Plans for High-Level-Waste Geologic Repositories	
4.20		Constraint on Releases of Airborne Radioactive Materials to the Environment for Licensees other than Power Reactors	

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RG	DG	Title	Status
	4012	Minimization of Contamination and Radioactive Waste Generation in Support of Decommissioning	Publish DG for Comments Aug 07
5.3		Statistical Terminology and Notation for Special Nuclear Materials Control and Accountability	
5.4		Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF ₄) and Uranium Hexafluoride (UF ₆)	
5.5		Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis of Nuclear-Grade Uranium Dioxide Powders and Pellets	
5.8		Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Drying and Fluidized Bed Operations	
5.11		Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste	
5.13		Conduct of Nuclear Material Physical Inventories	
5.15		Tamper-Indicating Seals for the Protection and Control of Special Nuclear Material	
5.17		Truck Identification Markings	
5.18		Limit of Error Concepts and Principles of Calculation in Nuclear Materials Control	
5.23		In Situ Assay of Plutonium Residual Holdup	
5.25		Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Equipment for Wet Process Operations	
5.26		Selection of Material Balance Areas and Item Control Areas	
5.27		Special Nuclear Material Doorway Monitors	
5.28		Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear Materials	
5.32		Communication with Transport Vehicles	
5.34		Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission Detection	
5.36		Recommended Practice for Dealing With Outlying Observations	
5.37		In Situ Assay of Enriched Uranium Residual Holdup	
5.39		General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic Distribution, and Impurity Determinations	
5.42		Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Equipment for Dry Process Operations	
5.48		Design Considerations - Systems for Measuring the Mass of Liquids	
5.49		Internal Transfer of Special Nuclear Material	
5.51		Management Review of Nuclear Material Control and Accounting Systems	
5.52		Standard Format and Content of a Licensee Physical Protection Plan for Strategic Special Nuclear Material at Fixed Sites (Other than Nuclear Power Plants)	
5.53		Qualification, Calibration, and Error Estimation Methods for Nondestructive Assay	
5.57		Shipping and Receiving Control of Strategic Special Nuclear Material	
5.58		Considerations for Establishing Traceability of Special Nuclear Material Accounting Measurements	

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RG	DG	Title	Status
5.67		Material Control and Accounting for Uranium Enrichment Facilities Authorized To Produce Special Nuclear Material of Low Strategic Significance	
	5021	Managing Safety / Security Interface	
8.2		Guide for Administrative Practices in Radiation Monitoring	
8.4		Direct-Reading and Indirect-Reading Pocket Dosimeters	
8.6		Standard Test Procedure for Geiger-Mueller Counters	
8.7		Instructions for Recording and Reporting Occupational Radiation Exposure Data	
8.8		Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be as Low as Is Reasonably Achievable	
8.10		Operating Philosophy for Maintaining Occupational Radiation Exposures as Low as Is Reasonably Achievable	
8.11		Applications of Bioassay for Uranium	
8.19		Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants - Design Stage Man-Rem Estimates	
8.22		Bioassay at Uranium Mills	
8.24		Health Physics Surveys During Enriched Uranium-235 Processing and Fuel Fabrication	
8.26		Applications of Bioassay for Fission and Activation Products	
8.27		Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants	
8.28		Audible-Alarm Dosimeters	
10.3		Guide for the Preparation of Applications for Special Nuclear Material Licenses of Less than Critical Mass Quantities	