Nuclear Operating Company
South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

April 16, 2007
NOC-AE-07002144
10CFR50.59

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

South Texas Project
Units 1 & 2
Docket Nos. STN 50-498, STN 50-499
10CFR50.59 Summary Report

Pursuant to the requirements of 10CFR50.59, the attached report contains a brief description and summary of the 10CFR50.59 evaluations of changes, tests and experiments conducted at the South Texas Project.

There are no commitments in this letter.

If there are any questions regarding this summary report, please contact J. R. Morris at (361) 972-8652 or me at (361) 972-7136.

Scott M. Head
Manager, Licensing

awh

Attachment: 2007 10CFR50.59 Evaluation Summaries

STI: 32143330

IE47

cc:
(paper copy)

Regional Administrator, Region IV
U. S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 289, Mail Code: MN116
Wadsworth, TX 77483

C. M. Canady
City of Austin
Electric Utility Department
721 Barton Springs Road
Austin, TX 78704

Richard A. Ratliff
Bureau of Radiation Control
Texas Department of State Health Services
1100 West 49th Street
Austin, TX 78756-3189

(electronic copy)

A. H. Gutterman, Esquire
Morgan, Lewis & Bockius LLP

Mohan C. Thadani
U. S. Nuclear Regulatory Commission

Steve Winn
Christine Jacobs
Eddy Daniels
Marty Ryan
NRG South Texas LP

Ed Alarcon
J. J. Nesrsta
R. K. Temple
Kevin Pollo
City Public Service

Jon C. Wood
Cox Smith Matthews

C. Kirksey
City of Austin

Summaries of the following 10CFR50.59 Evaluations are provided in this attachment:

1.	05-15364-10	Preheater Bypass Valve Test at Power
2.	04-12793-3	Change to Failure Modes & Effects Analysis for Auxiliary Feedwater
3.	05-4840-4	Change to Hot, Full-Power Feedwater Temperature

10CFR50.59 Evaluation Summaries

1. Preheater Bypass Valve Test at Power

Description: This evaluation supports a change to the Preheater Bypass Valve Operability Test procedure, to perform testing required by Technical Specification 4.6.3.1, Containment Isolation Valves. The valve operability testing will be performed for Containment isolation valve A2FW-FV-7192, Steam Generator 2D Preheater Outside Reactor Containment Bypass Valve. The test will stroke the valve during at-power operation.

Summary: All equipment impacted by the proposed test will stay within its design basis. All code requirements will be satisfied. In addition, the ensuing transient will not result in changing from a Condition I to a Condition II event. This test does not impact accident scenarios and the current dose analysis remains bounding.

2. Change to Failure Modes & Effects Analysis for Auxiliary Feedwater

Description: The change revises the STP UFSAR to reflect revised failure modes and effects analysis (FMEA) for the AFW system electrical components in the isolation valve cubicle (IVC). The change also revises accident analyses to credit faulted steam generator isolation using the safety-grade AFW outside containment isolation valves (OCIV) instead of the AFW regulating valves.

Summary: The proposed change replaces the faulted SG isolation function from the operator closing the AFW REG valves to the operator closing the AFW OCIVs. The AFW OCIVs are qualified to "harsh" environment conditions. The proposed change takes credit for the existing operator actions to isolate the faulted steam generator by closing the AFW OCIV or securing the pump. No new operator actions, procedure changes or equipment qualification will be performed. The required operator actions already exist in plant procedures No evaluation methodology is changed and no equipment or component needs to be requalified. No changes will be made to equipment important to safety or to operator actions. The proposed UFSAR change revises analysis to reflect plant conditions and does not change plant configuration. The change does not introduce the possibility of a malfunction of an SSC important to safety with a different result because no new failure modes are introduced. This change does not impact accident scenarios and the current dose analysis remains bounding.

3. Change to Hot, Full-Power Feedwater Temperature

Description: The proposed change revises the UFSAR to reflect the observed hot full-power feedwater temperature. This change revises the upper end of the hot full-power feedwater temperature range to 448 °F for all safety analyses described in the UFSAR.

Summary: There is no change made to the operating parameters. The results of the evaluation for higher HFP feedwater temperature show that all acceptance limits continue to be met, and the design basis limits for the fission product barriers are not exceeded.