

May 24, 2007

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: James A. FitzPatrick Nuclear Power Plant

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON
MARCH 21, 2007, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION
AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING INFORMATION
PERTAINING TO THE JAMES A. FITZPATRICK NUCLEAR POWER PLANT,
LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., held a telephone conference call on March 21, 2007, to discuss and clarify the staff's requests for additional information concerning the James A. FitzPatrick Nuclear Power Plant, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's questions.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a discussion of the issues discussed with the applicant.

The applicant had an opportunity to comment on this summary.

/RA/

Tommy Le, Sr. Project Manager
License Renewal Branch B
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures:

1. List of Participants
2. List of Requests for Additional Information

cc w/encls: See next page

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ADAMS Accession No.: ML070960030

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TELEPHONE CONFERENCE CALL
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS
MARCH 21, 2007

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AFFILIATIONS

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Ngoc Le	NRC
James Medoff	NRC
Roy Mathew	NRC
Allison Black	NRC
Matthew Mitchell	NRC
Qi Gan Chi	NRC
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Alan Cox	ENO
Joe Pahacheck	ENO
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Chris Pimm	ENO
Steve Glover	ENO
Tom Moskalyk	ENO
Mike Stroud	ENO

REQUESTS FOR ADDITIONAL INFORMATION
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
LICENSE RENEWAL APPLICATION

MARCH 21, 2007

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., (Entergy) held a telephone conference call on March 21, 2007, to discuss and clarify the following questions concerning the James A. FitzPatrick Nuclear Power Plant (JAFNPP), license renewal application (LRA).

Discussion #1:

- A. Commitment #20 on metal fatigue: The NRC requested that Entergy perform an additional review of Commitment #20 on metal fatigue analyses to determine whether the actions of the commitment cover activities on performing new cumulative usage factors (CUF) calculations for Class 1 piping in the residual heat removal (RHR) and feedwater (FW) systems and on applying the appropriate F_{en} factors to the CUF values in order to establish the environmental CUF values for these NUREG/CR-6260 piping locations.

Additional Requested Actions to Address this Issue:

1. The NRC requested that Entergy perform an additional review of the existing wording of Commitment #20 to see if the existing wording in the commitment already addresses this issue. If the current commitment does address this issue, Entergy will clarify how the existing commitment addresses actions to perform new environmental CUF calculations for the Class 1 RHR and FW piping. If not, Entergy stated it will revise the commitment to propose additional actions to address this issue.
- B. Commitments #21 and #23 on management of cracking and loss of preload in core plate rim hold-down bolts: The NRC informed the applicant, that upon a peer review applicant's basis for managing cracking and loss of preload in the core plate rim hold-down bolts, Option 3 in Commitment #23 is not acceptable as a basis for managing these aging effects in the bolts for the period of extended operation (PEO).

Additional Requested Actions to Address this Issue:

1. Delete Commitment #21 on the LRA and use Commitment #23 as the basis for managing cracking and loss of preload in the core plate rim hold-down bolts. The basis is that the staff does not require two separate commitments on the core plate rim hold-down bolts.
2. Amend Commitment #23 as follows:
 - (A) Amend Option 2 of the Commitment to state: "Complete a plant-specific analysis to determine acceptance criteria for continued inspection of core plate rim hold down bolting in

accordance with boiling water reactor (BWR) VIP-25 and submit the inspection plan, along with the acceptance criteria and justification for the inspection plan, to the NRC two years prior to the period of extended operation for NRC review and approval.”

- (B) Delete Option 3 of the Commitment.
- (C) Amend the Commitment to add a paragraph after the options that states: “If Option 2 is selected, the analysis to determine acceptance criteria will address the information requested in request for additional information (RAI) 3.1.2-2A and 4.7.3.2-1.”
- (D) Amend the “Related LRA Section No. / Comments” entry for Commitment #23 to reflect the following references: A.2.2.7, A.2.1.7, B.17, 4.7.3.2, Audit Item 483, Audit Item 252, and RAI 4.7.3.2-1.

3. Amend the response to parts b and c of RAI 4.7.3.2-1 as follows:

- (A) Amend Option 2 of the response to the RAI to state: “Complete a plant-specific analysis to determine acceptance criteria for continued inspection of core plate rim hold down bolting in accordance with BWRVIP-25 and submit the inspection plan, along with the acceptance criteria and justification for the inspection plan, to the NRC two years prior to the period of extended operation for NRC review and approval.”
- (B) Delete Option 3 of the response to the RAI.

C. Clarification on aging management of top guide grid beam locations: The NRC informed the applicant, that upon further discussion with NRC management in the Division of Component Integrity, NRR, the position on inspections of top guide grid beam locations for the PEO is that the position in generic aging lessons learned (GALL) aging managing program (AMP) XI.M9, BWR Internals, is (i.e. five percent of the locations within six years of entering the PEO and an additional five percent of the locations within 12 years of entering the PEO) a sufficient basis for managing the effects of aging during the PEO if cracking has yet to be detected in the top guide grid beam locations. If prior examinations have been conducted and cracking has been detected in the grid beam locations, the staff position is that the cracking would put the top guide outside of the bounds of the recommendations in BWRVIP-26-A, and that the applicant would need to address the impact of cracking on the intended function of the top guide, as assessed within the scope of corrective action provisions in Renewal Applicant Action Item (1) in Section 4.1 of the staff’s safety evaluation on BWRVIP-26-A, dated December 7, 2000 (ADAMS Accession No. ML003776110).

Additional Requested Actions to Address this Issue:

1. The NRC informed the applicant that no additional commitments on top guide grid beam inspections are necessary at this time. No evidence of cracking has yet to be detected in the top guide grid beam locations to date. Entergy has therefore proposed to inspect the grid beam locations in the JAFNPP top guide in conformance with the position established for top guides on page XI M-4 of GALL AMP XI.M9, "BWR Internals." This is in conformance with GALL AMP XI.M9. However, the staff requested the Entergy Nuclear North's (ENN's) responsible BWRVIP-Engineer perform an additional review of ENN BWRVIP program and to clarify what ENN's administrative BWRVIP program process would require (this is an Entergy-established requirement not an NRC requirement) if cracking is detected in any of the top guide grid beam locations.

Discussion #2:

The discussion centered around information the applicant had previously submitted in response to our February, 13 2007, conference call. In their response they proposed deleting section 4.7.2 "Leak Before Break."⁴ NRC staff wondered how the issue of fatigue in the calculations in the updated final safety analysis report had been dispositioned such that a time-limited aging analysis (TLAA) no longer existed. The applicant believed the calculation should never have been called a TLAA because there is no mention of 40 years, only a number of cycles, which still bounds the period of license extension. NRC was concerned about how the NRC Office of General Counsel (OGC) would define a TLAA. The NRC action item was to talk with OGC and determine their characterization of the situation. If the calculation should be a TLAA, the applicant will not delete the section, but will provide information to close a review of the TLAA.

Note to Entergy Nuclear Operations, from N. Le, date May 24, 2007

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FitzPatrick Nuclear Power Plant

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