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REMARKS: PLEASE REFER TO THE ATTACHED MEMO FOR FILING INSTRUCTIONS

J R MORRIS
VICE PRESIDENT
CATAWBA NUCLEAR STATION

BY:

T K PASOUR CN01EP TKP/RBE

A053

A053

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments Manual. Please remove and replace the following pages:

REMOVE THESE PAGES	INSERT THESE PAGES
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TAB 16.5	
N/A	16.5-10-1 through 16.5-10-3 Revision 0

If you have any questions concerning the contents of this package update, contact Toni Pasour at (803)831-3566.

Randy Hart
Manager, Regulatory Compliance

Attachment

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

Consistent with Technical Specification 3.4.13, this COMMITMENT is applicable in MODES 1, 2, 3, and 4. However, per Reference 1, the NRC only requested the verification required by TR 16.5-10-1 to be performed in MODES 1, 2, and 3. In MODE 4, Technical Specification 3.4.13 still requires LEAKAGE to be determined every 72 hours. In addition, LEAKAGE is determined administratively every 24 hours for conservatism. Therefore, even though this SLC does not require TR 16.5-10-1 to be performed in MODE 4, RCS Unidentified LEAKAGE information will still be available in this MODE.

REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

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Randy Hart
Manager, Regulatory Compliance

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

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Manager, Regulatory Compliance

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1	Confirm LEAKAGE increase is not sustained.	72 hours
		OR		
		A.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours
B.	RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1	Confirm LEAKAGE increase is not sustained.	72 hours
		OR		
		B.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

Consistent with Technical Specification 3.4.13, this COMMITMENT is applicable in MODES 1, 2, 3, and 4. However, per Reference 1, the NRC only requested the verification required by TR 16.5-10-1 to be performed in MODES 1, 2, and 3. In MODE 4, Technical Specification 3.4.13 still requires LEAKAGE to be determined every 72 hours. In addition, LEAKAGE is determined administratively every 24 hours for conservatism. Therefore, even though this SLC does not require TR 16.5-10-1 to be performed in MODE 4, RCS Unidentified LEAKAGE information will still be available in this MODE.

REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments Manual. Please remove and replace the following pages:

REMOVE THESE PAGES	INSERT THESE PAGES
LIST OF EFFECTIVE SECTIONS	
Pages 1 through 4 Revision 30	Pages 1 through 4 Revision 31
TABLE OF CONTENTS	
Pages i through v Revision 8	Pages i through v Revision 9
TAB 16.5	
N/A	16.5-10-1 through 16.5-10-3 Revision 0

If you have any questions concerning the contents of this package update, contact Toni Pasour at (803)831-3566.

Randy Hart
Manager, Regulatory Compliance

Attachment

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

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REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
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803 831 3000

March 15, 2007

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Randy Hart
Manager, Regulatory Compliance

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

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1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

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Manager, Regulatory Compliance

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16.11-19	0	10/09/02
16.11-20	0	10/09/02
16.11-21	0	10/09/02
16.12-1	0	10/09/02
16.13-1	0	10/09/02
16.13-2	Deleted	
16.13-3	Deleted	
16.13-4	0	10/09/02

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

Consistent with Technical Specification 3.4.13, this COMMITMENT is applicable in MODES 1, 2, 3, and 4. However, per Reference 1, the NRC only requested the verification required by TR 16.5-10-1 to be performed in MODES 1, 2, and 3. In MODE 4, Technical Specification 3.4.13 still requires LEAKAGE to be determined every 72 hours. In addition, LEAKAGE is determined administratively every 24 hours for conservatism. Therefore, even though this SLC does not require TR 16.5-10-1 to be performed in MODE 4, RCS Unidentified LEAKAGE information will still be available in this MODE.

REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments Manual. Please remove and replace the following pages:

REMOVE THESE PAGES	INSERT THESE PAGES
LIST OF EFFECTIVE SECTIONS	
Pages 1 through 4 Revision 30	Pages 1 through 4 Revision 31
TABLE OF CONTENTS	
Pages i through v Revision 8	Pages i through v Revision 9
TAB 16.5	
N/A	16.5-10-1 through 16.5-10-3 Revision 0

If you have any questions concerning the contents of this package update, contact Toni Pasour at (803)831-3566.

Randy Hart
Manager, Regulatory Compliance

Attachment

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16.6-1	0	10/09/02
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16.11-17	0	10/09/02
16.11-18	0	10/09/02
16.11-19	0	10/09/02
16.11-20	0	10/09/02
16.11-21	0	10/09/02
16.12-1	0	10/09/02
16.13-1	0	10/09/02
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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1	Confirm LEAKAGE increase is not sustained.	72 hours
		OR		
		A.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours
B.	RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1	Confirm LEAKAGE increase is not sustained.	72 hours
		OR		
		B.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

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REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

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Randy Hart
Manager, Regulatory Compliance

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16.5-5	0	10/09/02
16.5-6	0	10/09/02
16.5-7	0	10/09/02
16.5-8	1	05/18/04
16.5-9	0	10/24/06
16.5-10	0	03/08/07
16.6-1	0	10/09/02
16.6-2	Deleted	
16.6-3	0	10/09/02
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16.6-5	0	10/09/02
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16.7-15	0	03/01/05
16.8-1	1	02/01/05
16.8-2	1	10/24/06
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16.9-21	0	10/09/02
16.9-22	0	10/09/02
16.9-23	2	05/25/05
16.9-24	2	10/24/06
16.9-25	0	04/03/06
16.10-1	0	10/09/02
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16.11-1	0	10/09/02
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16.11-10	0	10/09/02
16.11-11	1	03/20/03
16.11-12	0	10/09/02
16.11-13	0	10/09/02
16.11-14	0	10/09/02
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16.11-20	0	10/09/02
16.11-21	0	10/09/02
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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE ≥ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE ≥ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

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1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

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Randy Hart
Manager, Regulatory Compliance

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16.11-3	0	10/09/02
16.11-4	0	10/09/02
16.11-5	0	10/09/02

SECTION	REVISION NUMBER	REVISION DATE
16.11-6	0	10/09/02
16.11-7	1	02/27/03
16.11-8	0	10/09/02
16.11-9	0	10/09/02
16.11-10	0	10/09/02
16.11-11	1	03/20/03
16.11-12	0	10/09/02
16.11-13	0	10/09/02
16.11-14	0	10/09/02
16.11-15	0	10/09/02
16.11-16	0	10/09/02
16.11-17	0	10/09/02
16.11-18	0	10/09/02
16.11-19	0	10/09/02
16.11-20	0	10/09/02
16.11-21	0	10/09/02
16.12-1	0	10/09/02
16.13-1	0	10/09/02
16.13-2	Deleted	
16.13-3	Deleted	
16.13-4	0	10/09/02

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1 Confirm LEAKAGE increase is not sustained. OR A.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1 Confirm LEAKAGE increase is not sustained. OR B.2 Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours 72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

Consistent with Technical Specification 3.4.13, this COMMITMENT is applicable in MODES 1, 2, 3, and 4. However, per Reference 1, the NRC only requested the verification required by TR 16.5-10-1 to be performed in MODES 1, 2, and 3. In MODE 4, Technical Specification 3.4.13 still requires LEAKAGE to be determined every 72 hours. In addition, LEAKAGE is determined administratively every 24 hours for conservatism. Therefore, even though this SLC does not require TR 16.5-10-1 to be performed in MODE 4, RCS Unidentified LEAKAGE information will still be available in this MODE.

REFERENCES

1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.

DUKE ENERGY CORPORATION
Catawba Nuclear Station
4800 Concord Road
York, SC 29745

803 831 3000

March 15, 2007

Re: Catawba Nuclear Station
Selected Licensee Commitments Manual
Revision Date 03/08/07

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments Manual. Please remove and replace the following pages:

REMOVE THESE PAGES	INSERT THESE PAGES
LIST OF EFFECTIVE SECTIONS	
Pages 1 through 4 Revision 30	Pages 1 through 4 Revision 31
TABLE OF CONTENTS	
Pages i through v Revision 8	Pages i through v Revision 9
TAB 16.5	
N/A	16.5-10-1 through 16.5-10-3 Revision 0

If you have any questions concerning the contents of this package update, contact Toni Pasour at (803)831-3566.

Randy Hart
Manager, Regulatory Compliance

Attachment

LIST OF EFFECTIVE SECTIONS

SECTION	REVISION NUMBER	REVISION DATE
TABLE OF CONTENTS	9	03/08/07
16.1	0	10/09/02
16.2	1	10/17/02
16.3	0	10/09/02
16.5-1	1	10/24/06
16.5-2	Deleted	
16.5-3	1	02/20/04
16.5-4	0	10/09/02
16.5-5	0	10/09/02
16.5-6	0	10/09/02
16.5-7	0	10/09/02
16.5-8	1	05/18/04
16.5-9	0	10/24/06
16.5-10	0	03/08/07
16.6-1	0	10/09/02
16.6-2	Deleted	
16.6-3	0	10/09/02
16.6-4	0	10/09/02
16.6-5	0	10/09/02
16.7-1	0	10/09/02
16.7-2	1	08/20/04
16.7-3	0	10/09/02
16.7-4	0	10/09/02
16.7-5	1	01/02/03

SECTION	REVISION NUMBER	REVISION DATE
16.7-6	1	11/17/05
16.7-7	0	10/09/02
16.7-8	1	05/25/05
16.7-9	4	02/01/05
16.7-10	0	10/09/02
16.7-11	0	10/09/02
16.7-12	0	10/09/02
16.7-13	1	05/17/06
16.7-14	0	10/06/03
16.7-15	0	03/01/05
16.8-1	1	02/01/05
16.8-2	1	10/24/06
16.8-3	1	10/24/06
16.8-4	1	10/24/06
16.8-5	2	02/20/04
16.9-1	4	08/03/06
16.9-2	3	05/25/05
16.9-3	0	10/09/02
16.9-4	2	05/25/05
16.9-5	3	05/17/06
16.9-6	3	10/24/06
16.9-7	3	03/10/04
16.9-8	4	10/24/06
16.9-9	2	03/10/04

SECTION	REVISION NUMBER	REVISION DATE
16.9-10	4	10/24/06
16.9-11	2	03/10/04
16.9-12	1	03/10/04
16.9-13	0	10/09/02
16.9-14	1	09/25/06
16.9-15	0	10/09/02
16.9-16	0	10/09/02
16.9-17	0	10/09/02
16.9-18	0	10/09/02
16.9-19	1	01/02/03
16.9-20	0	10/09/02
16.9-21	0	10/09/02
16.9-22	0	10/09/02
16.9-23	2	05/25/05
16.9-24	2	10/24/06
16.9-25	0	04/03/06
16.10-1	0	10/09/02
16.10-2	1	10/24/06
16.10-3	0	08/21/03
16.11-1	0	10/09/02
16.11-2	0	10/09/02
16.11-3	0	10/09/02
16.11-4	0	10/09/02
16.11-5	0	10/09/02

SECTION	REVISION NUMBER	REVISION DATE
16.11-6	0	10/09/02
16.11-7	1	02/27/03
16.11-8	0	10/09/02
16.11-9	0	10/09/02
16.11-10	0	10/09/02
16.11-11	1	03/20/03
16.11-12	0	10/09/02
16.11-13	0	10/09/02
16.11-14	0	10/09/02
16.11-15	0	10/09/02
16.11-16	0	10/09/02
16.11-17	0	10/09/02
16.11-18	0	10/09/02
16.11-19	0	10/09/02
16.11-20	0	10/09/02
16.11-21	0	10/09/02
16.12-1	0	10/09/02
16.13-1	0	10/09/02
16.13-2	Deleted	
16.13-3	Deleted	
16.13-4	0	10/09/02

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16.5 REACTOR COOLANT SYSTEM

16.5-10 Reactor Coolant System (RCS) Unidentified LEAKAGE

COMMITMENT Action levels for RCS Unidentified LEAKAGE shall be implemented as stated in the REMEDIAL ACTIONS.

APPLICABILITY: ----------NOTE----------
This SLC is applicable to Unit 2 only and shall no longer apply upon mitigation of the pressurizer nozzle butt welds.

MODES 1, 2, 3, and 4.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. RCS Unidentified LEAKAGE $\geq$ 0.1 gpm above recent average value as determined by successive performances of TR 16.5-10-1.	A.1	Confirm LEAKAGE increase is not sustained.	72 hours
	OR		
	A.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours
B. RCS Unidentified LEAKAGE $\geq$ 0.25 gpm above baseline value as determined by successive performances of TR 16.5-10-1.	B.1	Confirm LEAKAGE increase is not sustained.	72 hours
	OR		
	B.2	Confirm LEAKAGE increase is from sources other than the pressurizer nozzle butt welds.	72 hours

(continued)

REMEDIAL ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5. AND C.3 Perform a bare metal visual inspection of the pressurizer nozzle butt welds.	6 hours 42 hours Prior to startup from MODE 5

TESTING REQUIREMENTS

TEST	FREQUENCY
TR 16.5-10-1 ----------NOTES---------- 1. Not required to be performed until 12 hours after establishment of steady state operation. 2. Only required to be performed in MODES 1, 2, and 3. ---------- Verify RCS Unidentified LEAKAGE within limits by performance of RCS water inventory balance.	--------NOTE-------- Only required to be performed during steady state operation ---------- 24 hours

BASES

In October of 2006, while performing inspections of its pressurizer Alloy 82/182 butt welds in accordance with MRP-139, "Primary System Piping Butt Weld Inspection and Evaluation Guideline", a pressurized water reactor licensee discovered several circumferential indications in its pressurizer surge, safety, and relief nozzles. Because of the potential importance of this issue, the NRC requested commitments from all pressurized water reactor licensees regarding the inspection and mitigation of these welds. This SLC implements the commitments made to the NRC via Reference 1. The welds on Unit 1's pressurizer spray, surge, safety, and relief nozzles were mitigated (overlayed) with Alloy 690 material during the End-of-Cycle 16 Refueling Outage; therefore, this SLC is only applicable to Unit 2. The corresponding Unit 2 welds will be overlayed during the End-of-Cycle 15 Refueling Outage;

BASES (continued)

therefore, upon completion of this activity, this SLC will no longer be applicable.

The recent average value of Condition A is determined on a rolling basis from data obtained over the previous 5 to 7 days of MODE 1 full power steady state operation. The baseline value of Condition B is determined from data obtained during the first 7 days of MODE 1 full power steady state operation after the most recent bare metal visual inspection.

The 24 hour Frequency of TR 16.5-10-1 for RCS Unidentified LEAKAGE is more restrictive than that required by Technical Specification 3.4.13, RCS Operational LEAKAGE. For the purposes of this SLC, the steady state provisions of this Technical Specification may be applied, since LEAKAGE results obtained during maneuvering or transient conditions are not useful.

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1. Letter from James R. Morris to NRC, Inspection and Mitigation of Alloy 82/182 Pressurizer Butt Welds, February 26, 2007.