

April 19, 2007

Mr. William Levis  
Senior Vice President & Chief Nuclear Officer  
PSEG Nuclear LLC - N09  
Post Office Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2, ISSUANCE OF AMENDMENTS RE: ACCIDENT MONITORING INSTRUMENTATION AND SOURCE CHECK DEFINITION (TAC NOS. MD1654, MD1655, MD1656 AND MD1657)

Dear Mr. Levis:

The Commission has issued the enclosed Amendment Nos. 280 and 263 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 1, 2006, as supplemented by letters dated October 9, 2006, and February 21, 2007.

The amendments relocate the main steamline discharge radiation monitors (R46) from TS 3/4.3.3.1, "Radiation Monitoring Instrumentation" to TS 3/4.3.3.7, "Accident Monitoring Instrumentation." In addition, the amendments modify TS definition 1.31, "Source Check."

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/ra/

Richard B. Ennis, Senior Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. Amendment No. 280 to License No. DPR-70
2. Amendment No. 263 to License No. DPR-75
3. Safety Evaluation

cc w/encls: See next page

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Package Accession No.: ML070920306

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Unit 1 TS Accession No.: ML07109063

Unit 2 TS Accession No.: ML071090619

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PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 280  
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated May 1, 2006, as supplemented by letters dated October 9, 2006, and February 21, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/ra/*

Harold K. Chernoff, Chief  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating License  
and the Technical Specifications

Date of Issuance: April 19, 2007

ATTACHMENT TO LICENSE AMENDMENT NO. 280

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Replace the following page of Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Page 4

Insert

Page 4

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

1-6

3/4 3-36a

3/4 3-38a

3/4 3-55

3/4 3-57a

Insert

1-6

3/4 3-36a

3/4 3-38a

3/4 3-55

3/4 3-57a

PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 263  
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated May 1, 2006, as supplemented by letters dated October 9, 2006, and February 21, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. \_\_\_\_\_, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/ra/*

Harold K. Chernoff, Chief  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating License  
and the Technical Specifications

Date of Issuance: April 19, 2007



ATTACHMENT TO LICENSE AMENDMENT NO. 263

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Replace the following page of Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Page 4

Insert

Page 4

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

1-6

3/4 3-39a

3/4 3-41a

3/4 3-51a

3/4 3-52a

Insert

1-6

3/4 3-39a

3/4 3-41a

3/4 3-51a

3/4 3-52a

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 280 AND 263 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated May 1, 2006 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML061300615), as supplemented by letters dated October 9, 2006, and February 21, 2007 (ADAMS Accession Nos. ML062910181 and ML070600145, respectively), PSEG Nuclear, LLC (the licensee) submitted a request for changes to the Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2 Technical Specifications (TSs). The proposed amendment would relocate the main steamline discharge radiation monitors (R46) from TS 3/4.3.3.1, "Radiation Monitoring Instrumentation" to TS 3/4.3.3.7, "Accident Monitoring Instrumentation." In addition, the proposed amendment would modify TS definition 1.31, "Source Check."

The supplements dated October 9, 2006, and February 21, 2007, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on July 18, 2006 (71 FR 40753).

2.0 REGULATORY EVALUATION

In Section 50.36 of Title 10 of the *Code of Federal Regulations* (10 CFR), the NRC established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TSs.

In general, there are two classes of changes to TSs: (1) changes needed to reflect modifications to the design basis (TSs are derived from the design basis), and (2) changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time. In determining the acceptability of such changes, the NRC staff

interprets the requirements of 10 CFR 50.36, using as a model the accumulation of generically approved guidance in the improved Standard Technical Specifications (STSs). For this review, the NRC staff used NUREG-1431, Revision 3, "Standard Technical Specifications, Westinghouse Plants."

As described in the licensee's submittal dated May 1, 2006, the Salem TSs require 4 main steamline discharge radiation monitors (one R46 monitor for each of the 4 main steamlines). The R46 radiation monitors provide continuous monitoring of high-level, post-accident releases of radioactive noble gases via the safety-relief valves, atmospheric dump valves, and auxiliary feedpump turbine.

The licensee recently replaced the R46 monitors with monitors utilizing a different design. The previous R46 monitors used an off-line sampler through which a steam sample was passed and viewed by a Geiger-Muller tube detector. These detectors were fitted with a Cesium-137 check source which, when activated, positioned the source within view of the detector and caused an upscale deflection. These detectors have been replaced with ion chambers located in the mechanical penetration areas adjacent to the main steam lines. The ion chambers contain, internal to the detector geometry, a long-lived source of radioactivity that continuously causes ionization within the chamber, producing a continuous upscale reading. Additionally, the electronics include the capability for injecting a test signal into the detector electronics for upscale indication. This electronic test ensures continuity of the electronics. The electronics operate using a fail-safe methodology in which a detector or electronic malfunction will result in a downscale fail condition that is annunciated via a "fail" alarm indication. The proposed change will eliminate occupational radiation exposure that would be incurred by performing a monthly Source Check.

The licensee's application dated May 1, 2006, stated that the new monitors are designed to meet the requirements of NUREG-0737, "Clarification of TMI [Three Mile Island] Action Plan Requirements," Item II.F.1, "Additional Accident-Monitoring Instrumentation" and the intent of Regulatory Guide (RG) 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident."

### 3.0 TECHNICAL EVALUATION

#### 3.1 Relocation of Main Steamline Discharge Radiation Monitors from Radiation Monitoring Instrumentation TSs to Accident Monitoring Instrumentation TSs

The relocation of the main steamline discharge radiation monitors (R46) from TS 3/4.3.3.1, "Radiation Monitoring Instrumentation" to TS 3/4.3.3.7, "Accident Monitoring Instrumentation" would result in the following changes to the TSs:

- 1) Deletion of item 2.b.3, "Main Steamline Discharge (Safety Valves and Atmospheric Steam Dumps)" from TS Table 3.3-6, "Radiation Monitoring Instrumentation"
- 2) Deletion of item 2.b.3, "Main Steamline Discharge (Safety Valves and Atmospheric Steam Dumps)" from TS Table 4.3-3, "Radiation Monitoring Instrumentation Surveillance Requirements"

- 3) Addition of item 21, "Main Steamline Discharge (Safety Valves and Atmospheric Steam Dumps) Monitor" to TS Table 3.3-11, "Accident Monitoring Instrumentation"
- 4) Addition of item 21, "Main Steamline Discharge (Safety Valves and Atmospheric Steam Dumps) Monitor" to TS Table 4.3-11, "Surveillance Requirements for Accident Monitoring Instrumentation"

Due to the differences in the current requirements and format in TS 3/4.3.3.7 versus TS 3/4.3.3.1, the proposed TS relocations for the R46 monitors, as described above, would have the following effect:

- 1) The R46 monitors would only be required to be operable in Modes 1 through 3, as opposed to the currently specified Modes 1 through 4.
- 2) The monthly surveillance requirement for a Source Check of the R46 monitors would be eliminated.
- 3) The R46 monitors alarm setpoint and measurement range would no longer be shown in the Salem TSs.

The NRC staff reviewed the proposed changes against the Salem licensing basis, RG 1.97, and the STSs found in NUREG-1431, Revision 3, as described below.

#### *Change in R46 Monitor Operability Requirements*

For the proposed amendment, the R46 monitors would only be required to be operable in Modes 1 through 3, as opposed to the currently specified Modes 1 through 4.

For the STSs, the instrumentation related to RG 1.97 is contained in STS 3.3.3, "Post Accident Monitoring (PAM) Instrumentation." STS 3.3.3 specifies that the PAM instrumentation be operable in Modes 1, 2, and 3.

The licensee's supplement dated October 9, 2006, stated that the R46 monitor's alarm function is used in the emergency operating procedures (EOPs) to identify a Steam Generator Tube Rupture (SGTR) event EOP entry point and that the Salem EOPs, are applicable in Modes 1, 2, and 3. The licensee further stated that the R46 monitors (1) are required post-accident only; (2) have no normal (i.e., 10 CFR Part 20) effluent monitoring or primary-to-secondary leak detection functions; and (3) have no Mode 4 functional requirements. The SGTR accident analysis for Salem is discussed in Updated Final Safety Analysis Report (UFSAR) Section 15.4.4. The UFSAR states that the SGTR is assumed to take place at power. As shown in Salem TS Table 1.1, "Operational Modes," Mode 4, the plant is in hot shutdown (i.e., not at power).

The NRC staff finds that the proposed change (related to R46 monitor operability in Modes 1, 2, and 3) is consistent with the Salem licensing basis for an SGTR event, and, therefore, is acceptable. In addition, the change is consistent with the STS.

### *Elimination of Source Check for the R46 Monitors*

The proposed amendment will eliminate the current monthly surveillance requirement for a Source Check of the R46 monitors, however, the other surveillance requirements for the R46 monitors will be maintained. This includes a channel check each shift (i.e., at least once per 12 hours), a channel calibration every 18 months, and a channel functional test quarterly.

The STSs do not contain requirements for radiation monitoring instrumentation (similar to Salem TS 3/4.3.3.1) and do not contain a definition or any requirements for a Source Check. The surveillance requirements in STS 3.3.3 include a monthly channel check and a channel calibration every 18 months, or approximately at every refueling.

Per the current Salem TS 1.31 definition, the purpose of the Source Check is to qualitatively assess that the instrument channel would respond if the detector is exposed to a source of increased radiation. Currently, the R46 surveillance for a Source Check is performed monthly by exposing the detectors to an external radiation source. As described in Safety Evaluation (SE) Section 2.0, the new R46 monitors contain an internal radiation source in the detector that produces a continuous upscale reading. Failure of the detector will result in a downscale fail condition that is annunciated via a "fail" alarm indication.

The NRC staff finds that the internal radiation source and alarm function provide reasonable assurance that the R46 instrument channels are functioning properly. In addition, the proposed change is consistent with the requirements in 10 CFR Part 20 to keep radiation exposure as low as reasonably achievable (ALARA) since it will eliminate occupational radiation exposure that would be incurred by performing a monthly Source Check. Based on these considerations, the NRC staff finds that the elimination of the current monthly surveillance requirement for a Source Check of the R46 monitors is acceptable.

### *Deletion of Alarm Setpoint and Measurement Range from the TSs for R46 Monitors*

For the proposed amendment, the alarm setpoint and measurement range for the R46 monitors would no longer be shown in the Salem TSs due to differences in the format between TS Table 3.3-6, "Radiation Monitoring Instrumentation" and TS Table 3.3-11, "Accident Monitoring Instrumentation."

For the PAM instrumentation in STS Table 3.3.3-1, the table provides information regarding the required number of channels and the required actions. The table does not provide information regarding setpoints or the measurement range.

As discussed in the licensee's submittal dated October 9, 2006, the current alarm setpoint for the R46 monitors, shown in TS Table 3.3-6, is an administrative limit only. The TS value of 10 millirem per hour does not have a documented engineering basis. The alarm is set low enough to alert the operator that a significant post-accident release is occurring.

Following the proposed TS relocation, the R46 monitors will still be required (per surveillance requirement TS 4.3.3.7) to be demonstrated operable by performance of a channel check each shift, a channel calibration every 18 months, and a channel function test quarterly. The NRC staff finds that these surveillance requirements are adequate to determine operability of the R46 channels. The alarm setpoint does not provide useful information regarding whether the

R46 monitors are operable given that the setpoint is an administrative limit. In addition, any future changes to the setpoint would be adequately controlled by the requirements in 10 CFR 50.59. Based on the above considerations, the NRC staff concludes that is acceptable to delete R46 monitor alarm setpoint from the TSs.

The NRC staff finds that the measurement range is informational only, and similarly, does not provide information regarding whether the R46 monitors are operable. Any future changes to the range would be adequately controlled by the requirements in 10 CFR 50.59. As such, the NRC staff concludes that it is acceptable to delete this information from the TSs.

### 3.2 Revision of the Definition of Source Check

Currently, Salem TS Definition 1.31, "Source Check," reads as follows:

SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a source of increased radioactivity.

As described in the licensee's submittal dated February 21, 2007, the TS Definition 1.31 would be changed to read as follows:

SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to either (a) an external source of increased radioactivity, or (b) an internal source of radioactivity (keep-alive source), or (c) an equivalent electronic source check.

Following the proposed relocation of the R46 monitors, from TS 3/4.3.3.1 to TS 3/4.3.3.7, the proposed change to the definition of Source Check would affect the remaining radiation monitoring instrumentation in TS Table 4.3-3.

As discussed in the licensee's submittal dated October 9, 2006, currently, all the remaining radiation monitors in TS Table 4.3-3 are designed to have a qualitative source check assessment implemented via exposure to an external source. However, it is likely that in the future some, or all, of these monitors may be replaced with improved technology, making the exposure to an external source for the Source Check assessment obsolete.

The NRC staff finds that the use of an internal source of radioactivity or an equivalent electronic source check meet the intent of the current TS 1.31 definition and would provide reasonable assurance that the associated instrument channels are functioning properly. In addition, use of new technology, which would eliminate the need for performing a Source Check with an external source, is consistent with the requirements in 10 CFR Part 20 to keep radiation exposure as ALARA. Based on these considerations, the NRC staff finds the proposed change acceptable.

### 3.3 Technical Evaluation Conclusion

Based on the discussion in SE Sections 3.1 and 3.2, the NRC staff concludes that the proposed amendment is acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (71 FR 40753). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: S. Mazumdar  
R. Ennis

Date: April 19, 2007