



Morris Operation  
General Electric Company  
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March 19, 2007

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission, Region III  
2443 Warrenville Road  
Lisle, IL 60532-4352

REF: SNM-2500  
Encl: (1) Corrected 2006 Annual Effluent Release and Dose to Public Report

Subj: CORRECTION TO ANNUAL EFFLUENT RELEASE AND DOSE TO PUBLIC REPORT  
FOR 2006

Dear Regional Administrator:

A discrepancy was discovered in the identification of the well used to calculate Committed Effective Dose Equivalent to the public that could occur from groundwater. The well was identified, on page 1, as DM-4. The correct well, identified on page 2 of the report, is DM-5. Enclosure (1) corrects this discrepancy.

Sincerely,

Anthony E. McFadden  
Radiation Safety Officer

cc: ATTN: Document Control Desk  
Director, Spent Fuel Project Office  
Office of Nuclear Material Safety and Safeguards  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555-0001

J.E. Ellis (GEMO)

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REF: SNM-2500

Dear Regional Administrator:

In compliance with 10CFR72.44(d)(3) and SNM-2500 license condition 8.2.1, this report documents our estimate of quantities of principal radionuclides released to the environment by the GE Morris Operation in 2006. This report also provides an estimate of the maximum potential dose to the public resulting from GE Morris Operation effluents for 2006.

The only particulate radionuclide present on the stack monitor filters was Cs-137. Gaseous radionuclides evaluated were H-3 and Kr-85. The quantity of tritium released was calculated by multiplying basin water evaporative losses, by the average tritium levels in the fuel basins. The amount of Kr-85 released was calculated by multiplying the concentration found in samples taken directly over the basin water, by the airflow through the basin area.

COMPLY V1.6 (the EPA software program) was used to calculate the effective dose equivalent from the release of these radionuclides. The quantities released and the resultant maximum potential effective dose equivalents are shown in the following table.

Nuclide	Activity Discharged (Ci)
H-3	1.990 E-2
Kr-85	9.360 E-1
Cs-137	3.405 E-7

Effective Dose Equivalent      **2.5 E-7 mrem/year**

There are no liquid effluents from the site. Trace quantities of tritium were found in the site wells. Surface water tritium levels were below minimum detectable levels. Tritium values below detectable levels are conservatively reported as the minimum detectable level.

The maximum potential Committed Effective Dose Equivalent to the public that could occur from surface water was calculated to be **7.25 E-3 mrem** - based on a person consuming water all year from the Sanitary Lagoons (159 pCi/l H-3).

The maximum potential Committed Effective Dose Equivalent to the public that could occur from groundwater was calculated to be **7.05 E-3 mrem** - based on a person consuming water all year from the DM-5 well (154.5 pCi/l H-3).

Measurement of direct radiation at the GE Morris Operation owner control boundary is accomplished using TLDs prepared and processed by a contractor, and direct measurement

techniques. The calculated maximum potential Committed Effective Dose Equivalent to the public that could occur from direct radiation at the boundary of the owner controlled area was calculated to be **0.411 mrem** assuming the maximum time spent at the boundary is 24 hours per year.

The maximum potential radiation dose to the public for 2006 would result from the sum of the stack effluent releases, the dose from drinking surface water from the Sanitary Lagoons, ground water from monitoring well DM-5 and from direct radiation at the owner controlled boundary. The sum of these sources for 2005 is **0.425 mrem**.

Sincerely,



Anthony E. McFadden  
Radiation Safety Officer

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Director, Spent Fuel Project Office  
Office of Nuclear Material Safety and Safeguards  
U.S. Nuclear Regulatory Commission  
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Enclosure (1)