

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components.
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I:

Part 20, Section 30.34 of Part 30; Section 40.41 of Part 40; Section 50.54 and 50.59 of Part 50; and Section 70.32 of Part 70. This renewed license is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect and is also subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 1850 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, which is attached hereto, as revised through Amendment No. 193 is hereby incorporated into this license. Nine Mile Point Nuclear Station, LLC shall operate the facility in accordance with the Technical Specifications.

- (3) The licensee shall submit an application for license amendment, including supporting analyses and evaluations by December 18, 1998. This amendment application shall contain the proposed methods for compliance with GDC 19 dose guidelines under accident conditions based upon system design and without reliance upon the use of potassium iodide.

LIMITING CONDITION FOR OPERATION

c. Scram Insertion Times

- (1) The average scram insertion time of all operable control rods, in the power operation condition, shall be no greater than:

<u>% Inserted From Fully Withdrawn</u>	<u>Average Scram Insertion Times (sec)</u>
5	0.375
20	0.90
50	2.00
90	5.00

- (2) Except as noted in 3.1.1.c(3), the maximum insertion scram time, in the power operation condition, shall be no greater than:

<u>% Inserted From Fully Withdrawn</u>	<u>Maximum Scram Insertion Times (sec)</u>
5	0.398
20	0.954
50	2.12
90	5.30

SURVEILLANCE REQUIREMENT

c. Scram Insertion Times

The maximum scram insertion time shall be demonstrated through measurement for*:

- (1) All control rods prior to thermal power exceeding 40% power with reactor pressure above 800 psig, after each major refueling outage or after a reactor shutdown that is greater than 120 days.
- (2) Specifically affected individual control rods following maintenance on or modification to the control rod or control rod drive system which could affect the scram insertion time of those specific control rods with reactor pressure above 800 psig.
- (3) At least 20 control rods, on a rotating basis, on a frequency of less than or equal to once per 180 days of cumulative power operation, with reactor pressure above 800 psig.

* For single control rod scram time tests, the control rod drive pumps shall be isolated from the accumulators.

LIMITING CONDITION FOR OPERATION

(3) Control rods with longer scram insertion time will be permitted provided that no other control rod in a nine-rod square array around this rod has a:

- (a) Scram insertion time greater than the maximum allowed,
- (b) Malfunctioned accumulator,
- (c) Valved out of service in a non-fully inserted position. —

d. Control Rod Accumulators

At all reactor operating pressures, a rod accumulator may be out of service provided that no other control rod in a nine-rod square array around this rod has a:

- (1) Malfunctioned accumulator,
- (2) Valved out of service in a non-fully inserted position,
- (3) Scram insertion greater than maximum permissible insertion time.

SURVEILLANCE REQUIREMENT

d. Control Rod Accumulators

Once a shift check the status of the accumulator pressure and level alarms in the control room.

LIMITING CONDITION FOR OPERATION

not returned to within the prescribed limits within two (2) hours, reactor power reductions shall be initiated at a rate not less than 10% per hour until APLHGR at all nodes is within the prescribed limits.

b. Linear Heat Generation Rate (LHGR)

During power operation, the Linear Heat Generation Rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the limiting value specified in the Core Operating Limits Report.

If at any time during power operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded at any location, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the LHGR at all locations is not returned to within the prescribed limits within two (2) hours, reactor power reductions shall be initiated at a rate not less than 10% per hour until LHGR at all locations is within the prescribed limits.

c. Minimum Critical Power Ratio (MCPR)

During power operation, the MCPR for all fuel at rated power and flow shall be within the limit provided in the Core Operating Limits Report.

If at any time during power operation it is determined by normal surveillance that the above limit is no longer met, action shall be initiated within 15 minutes to restore operation to within

SURVEILLANCE REQUIREMENT

b. Linear Heat Generation Rate (LHGR)

The LHGR as a function of core height shall be checked daily during reactor operation at $\geq 25\%$ rated thermal power.

c. Minimum Critical Power Ratio (MCPR)

- (1) MCPR shall be determined daily during reactor power operation at $>25\%$ rated thermal power.
- (2) MCPR operating limit shall be determined within 72 hours of completing scram time testing as required in Specification 4.1.1(c).