

- (4) UE, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source of special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) UE, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

UE is authorized to operate the facility at reactor core power levels not in excess of 3565 megawatts thermal (100% power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan*

The Technical Specifications contained in Appendix A, as revised through Amendment No. 182 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Environmental Qualification (Section 3.11, SSER #3)**

Deleted per Amendment No. 169

* Amendments 133, 134, & 135 were effective as of April 30, 2000 however these amendments were implemented on April 1, 2000.

** The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

3.1 REACTIVITY CONTROL SYSTEMS

3.1.7 Rod Position Indication

LCO 3.1.7 The Digital Rod Position Indication (DRPI) System and the Demand Position Indication System shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTIONS

----- NOTE -----

Separate Condition entry is allowed for each inoperable rod position indicator and each demand position indicator.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One DRPI per group inoperable for one or more groups.	A.1 Verify the position of the rods with inoperable position indicators indirectly by using core power distribution measurement information.	Once per 8 hours
	OR A.2 Reduce THERMAL POWER to \leq 50% RTP.	8 hour

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>B. More than one DRPI per group inoperable for one or more groups.</p>	<p>B.1 Place the control rods under manual control.</p>	<p>Immediately</p>
	<p><u>AND</u></p>	
	<p>B.2 Monitor and record RCS T_{avg}.</p>	<p>Once per 1 hour</p>
	<p><u>AND</u></p>	
<p>C. One or more rods with inoperable DRPIs have been moved in excess of 24 steps in one direction since the last determination of the rod's position.</p>	<p>B.3 Verify the position of the rods with inoperable position indicators indirectly by using core power distribution measurement information.</p>	<p>Once per 8 hours</p>
	<p><u>AND</u></p>	
	<p>B.4 Restore inoperable position indicators to OPERABLE status such that a maximum of one DRPI per group is inoperable.</p>	<p>24 hours</p>
<p>C. One or more rods with inoperable DRPIs have been moved in excess of 24 steps in one direction since the last determination of the rod's position.</p>	<p>C.1 Verify the position of the rods with inoperable position indicators indirectly by using core power distribution measurement information.</p>	<p>4 hours</p>
	<p><u>OR</u></p>	
	<p>C.2 Reduce THERMAL POWER to $\leq 50\%$ RTP.</p>	<p>8 hours</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.2.1.2 ----- NOTE -----</p> <p>If F_Q^C(Z) measurements indicate maximum over z $\left[\frac{F_Q^C(Z)}{K(Z)} \right]$ has increased since the previous evaluation of F_Q^C(Z) :</p> <p>a. Increase F_Q^W(Z) by the appropriate factor specified in the COLR and reverify F_Q^W(Z) is within limits; or</p> <p>b. Repeat SR 3.2.1.2 once per 7 EFPD until two successive power distribution measurements indicate maximum over z $\left[\frac{F_Q^C(Z)}{K(Z)} \right]$ has not increased.</p> <hr/> <p>Verify F_Q^W(Z) is within limit</p>	<p>Once after each refueling prior to THERMAL POWER exceeding 75% RTP</p> <p><u>AND</u></p> <p>(continued)</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.2.4.1</p> <p>----- NOTE -----</p> <ol style="list-style-type: none"> 1. With input from one Power Range Neutron Flux channel inoperable and THERMAL POWER \leq 75% RTP, the remaining three power range channels can be used for calculating QPTR. 2. SR 3.2.4.2 may be performed in lieu of this Surveillance. <p>-----</p> <p>Verify QPTR is within limit by calculation.</p>	<p>7 days</p>
<p>SR 3.2.4.2</p> <p>----- NOTE -----</p> <p>Not required to be performed until 12 hours after input from one or more Power Range Neutron Flux channels are inoperable with THERMAL POWER $>$ 75% RTP.</p> <p>-----</p> <p>Verify QPTR is within limit using power distribution measurement information.</p>	<p>12 hours</p>

SURVEILLANCE REQUIREMENTS

----- NOTE -----
Refer to Table 3.3.1 1 to determine which SRs apply for each RTS Function.

SURVEILLANCE		FREQUENCY
SR 3.3.1.1	Perform CHANNEL CHECK.	12 hours
SR 3.3.1.2	<p>----- NOTE ----- Not required to be performed until 24 hours after THERMAL POWER is \geq 15% RTP.</p> <p>Compare results of calorimetric heat balance calculation to power range channel output. Adjust power range channel output if calorimetric heat balance calculation results exceed power range channel output by more than +2% RTP.</p>	24 hours
SR 3.3.1.3	<p>----- NOTE ----- Not required to be performed until 24 hours after THERMAL POWER is \geq 50% RTP.</p> <p>Compare results of incore power distribution measurements to Nuclear Instrumentation System (NIS) AFD. Adjust NIS channel if absolute difference is \geq 2%.</p>	31 effective full power days (EFPD)

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SURVEILLANCE		FREQUENCY
SR 3.3.1.4	<p>----- NOTE -----</p> <p>This Surveillance must be performed on the reactor trip bypass breaker for the local manual shunt trip only prior to placing the bypass breaker in service.</p> <p>-----</p> <p>Perform TADOT.</p>	62 days on a STAGGERED TEST BASIS
SR 3.3.1.5	Perform ACTUATION LOGIC TEST.	92 days on a STAGGERED TEST BASIS
SR 3.3.1.6	<p>----- NOTE -----</p> <p>Not required to be performed until 72 hours after achieving equilibrium conditions with THERMAL POWER \geq 75 % RTP.</p> <p>-----</p> <p>Calibrate excore channels to agree with incore power distribution measurements.</p>	92 EFPD
SR 3.3.1.7	<p>----- NOTE -----</p> <ol style="list-style-type: none"> 1. Not required to be performed for source range instrumentation prior to entering MODE 3 from MODE 2 until 4 hours after entry into MODE 3. 2. Source range instrumentation shall include verification that interlocks P-6 and P-10 are in their required state for existing unit conditions. <p>-----</p> <p>Perform COT.</p>	184 days

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