

March 15, 2007

Mr. J. V. Parrish
Chief Executive Officer
Energy Northwest
P.O. Box 968 (Mail Drop 1023)
Richland, WA 99352-0968

SUBJECT: COLUMBIA GENERATING STATION - CLOSEOUT OF NRC GENERIC
LETTER 2003-01, "CONTROL ROOM HABITABILITY" (TAC NO. MB9789)

Dear Mr. Parrish:

On July 12, 2003, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 2003-01, "Control Room Habitability" (Agencywide Documents Access and Management System (ADAMS) Accession No. ML031620248), to all holders of operating licenses for nuclear power reactors except those who had permanently ceased operation and had certified that fuel had been removed from the reactor vessel.

GL 2003-01 requested that each licensee confirm that its facility control room meets its design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, and 19; draft GDC; or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in the design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into the hazardous chemical assessments (GL 2003-01, Item 1b); and (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and corrective actions needed to retire these compensatory measures (GL 2003-01, Item 2).

By letters dated August 11, 2003 (ADAMS Accession No. ML032260330), and March 2, 2007 (ADAMS Accession No. ML070720555), Energy Northwest (the licensee), responded to GL 2003-01 for the Columbia Generating Station (CGS).

In your response, you reported the results of tracer gas tests conducted in accordance with American Society for Testing Materials (ASTM) E741, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution," for the CGS Control Room Envelope (CRE), which is pressurized for accident mitigation. You reported that you conducted a test in September 2000 and a subsequent test in the fall of 2003.

Your August 11, 2003, letter discussed the tracer gas test conducted in 2000. You reported that the most limiting unfiltered inleakage into the CRE was more than the value of 10.55 cubic feet per minute (cfm) assumed in the design-basis radiological analyses for CRH. You further stated that a compensatory measure of administering potassium iodide (KI) to control room

operators to reduce the control room thyroid dose to below the 30-roentgen equivalent man (rem) limit in the event of a design-basis accident had been implemented.

Your license amendment request (LAR) dated September 30, 2004 (ADAMS Accession No. ML042930316), as supplemented, to adopt the Alternative Source Term (AST) methodology discussed the tracer gas test conducted in 2003. In addition, this LAR also included revised inleakage assumptions of 50 cfm for one train in service and 75 cfm for both trains in service. You reported that the most limiting unfiltered inleakage into the CRE was 21 standard cubic feet per minute (scfm) for one train in service and 53 scfm for both trains in service, both of which are less than the values of 50 cfm for one train in service and 75 cfm for both trains in service, assumed in your LAR to adopt the AST.

You also provided information that adequately supported a conclusion that the most limiting unfiltered inleakage into the CRE is incorporated into the hazardous chemical assessments, and that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your technical specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design-basis analysis for CRH, and in light of the demonstrated inadequacy of a delta (Δ) P measurement to alone provide such verification (GL 2003-01, Item 1c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TS to reference an acceptable surveillance methodology. In your March 2, 2007, letter you stated that you would submit an LAR to revise your TSs utilizing the guidance provided in Technical Specification Task Force Traveler 448 (TSTF-448) by July 31, 2007.

Based on your original inleakage assumption of 10.55 cfm and your 2000 tracer gas test result, you reported implementation of a compensatory measure of administering KI, as previously discussed. Your LAR to adopt the AST methodology, which will retire this compensatory measure, was approved and issued as Amendment No. 199, dated November 27, 2006 (ADAMS Accession No. ML062610440). In your March 2, 2007, letter, you stated that you will retire this compensatory measure upon implementation of Amendment No. 199, which is scheduled to occur no later than March 27, 2007.

Your March 2, 2007, letter stated that CGS is required to meet the GDCs regarding CRH. Based on the information provided, your commitment to submit an LAR based on TSTF-448, and the above discussion, the NRC staff finds your response to be acceptable for the purpose of closing GL 2003-01 for CGS.

J. V. Parrish

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If you have any questions regarding this response, please contact me at (301) 415-2296.

Sincerely,

/RA/

Carl F. Lyon, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-397

cc: See next page

J. V. Parrish

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Columbia Generating Station

cc:

Mr. W. Scott Oxenford (Mail Drop PE04)
Vice President, Technical Services
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Mr. Albert E. Mouncer (Mail Drop PE01)
Vice President, Corporate Services/
General Counsel/CFO
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Chairman
Energy Facility Site Evaluation Council
P.O. Box 43172
Olympia, WA 98504-3172

Mr. Douglas W. Coleman (Mail Drop PE20)
Manager, Regulatory Programs
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Mr. Gregory V. Cullen (Mail Drop PE20)
Supervisor, Licensing
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-4005

Chairman
Benton County Board of Commissioners
P.O. Box 190
Prosser, WA 99350-0190

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 69
Richland, WA 99352-0069

Mr. Dale K. Atkinson (Mail Drop PE08)
Vice President, Nuclear Generation
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Mr. William A. Horin, Esq.
Winston & Strawn
1700 K Street, N.W.
Washington, DC 20006-3817

Mr. Matt Steuerwalt
Executive Policy Division
Office of the Governor
P.O. Box 43113
Olympia, WA 98504-3113

Ms. Lynn Albin
Washington State Department of Health
P.O. Box 7827
Olympia, WA 98504-7827

Technical Services Branch Chief
FEMA Region X
130 228th Street, S.W.
Bothell, WA 98201-9796

Ms. Cheryl M. Whitcomb (Mail Drop PE03)
Vice President, Organizational
Performance & Staffing/CKO
Energy Northwest
P.O. Box 968
Richland, WA 99352-0968

Assistant Director
Nuclear Safety and Energy Siting Division
Oregon Department of Energy
625 Marion Street, NE
Salem, OR 97301-3742

Special Hazards Program Manager
Washington Emergency Management Div.
127 W. Clark Street
Pasco, WA 99301

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